

Program EVALPLOT  
(Version 2021-1)

by

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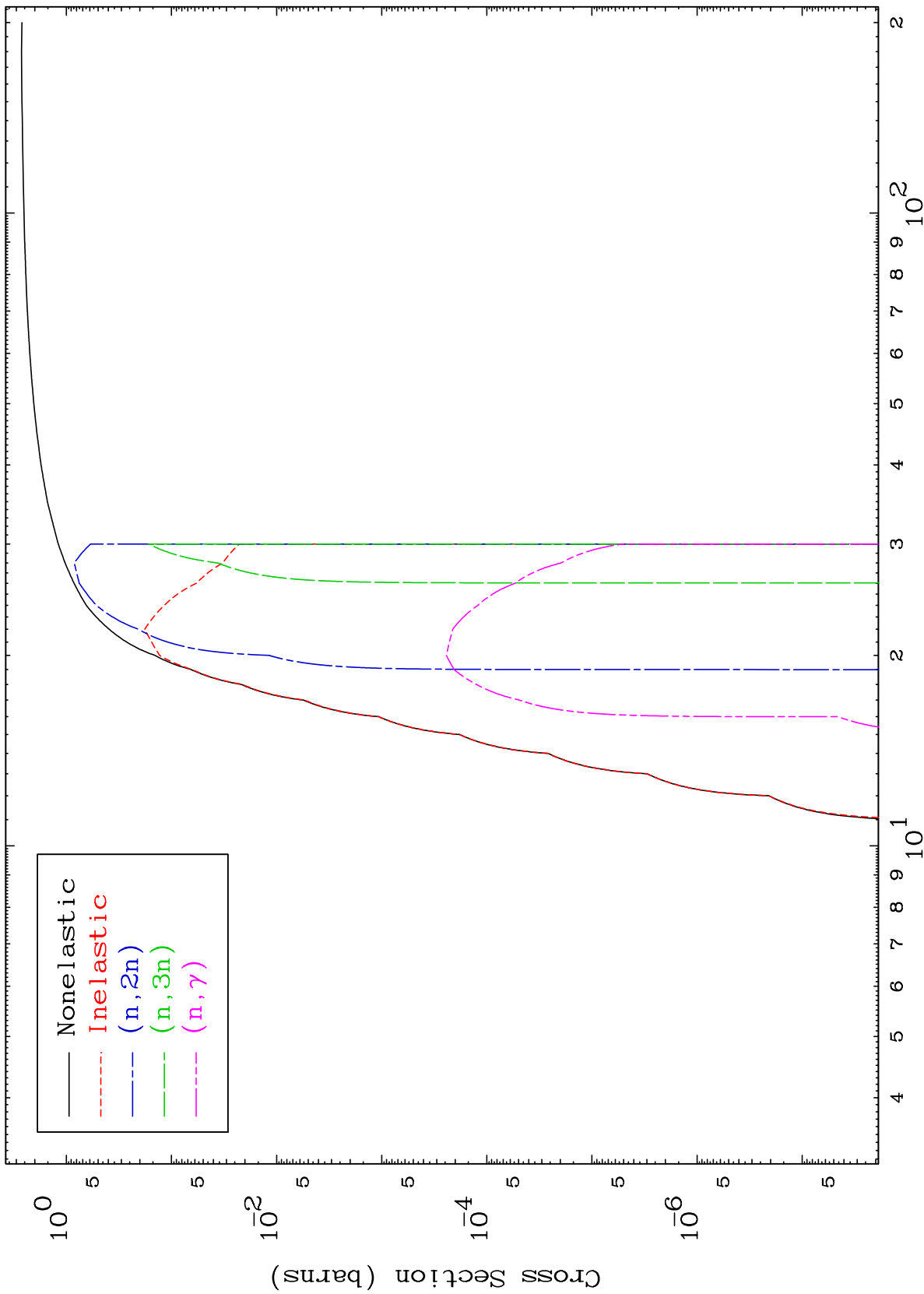
Press Mouse Button to Start

MAT 7523

0 Kelvin

$\alpha$  Major Cross Sections

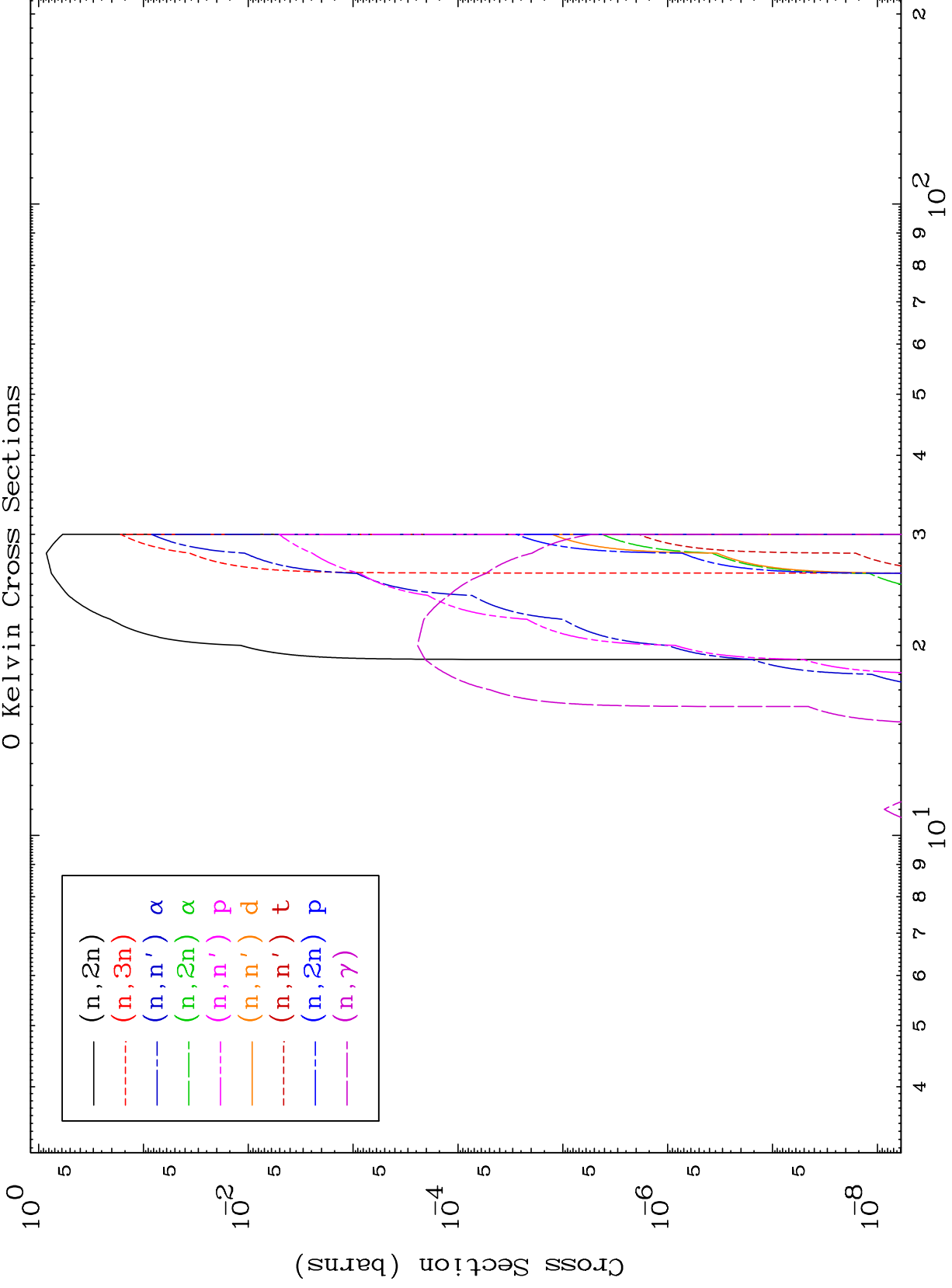
<sup>75</sup>Re-184m



MAT 7523

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

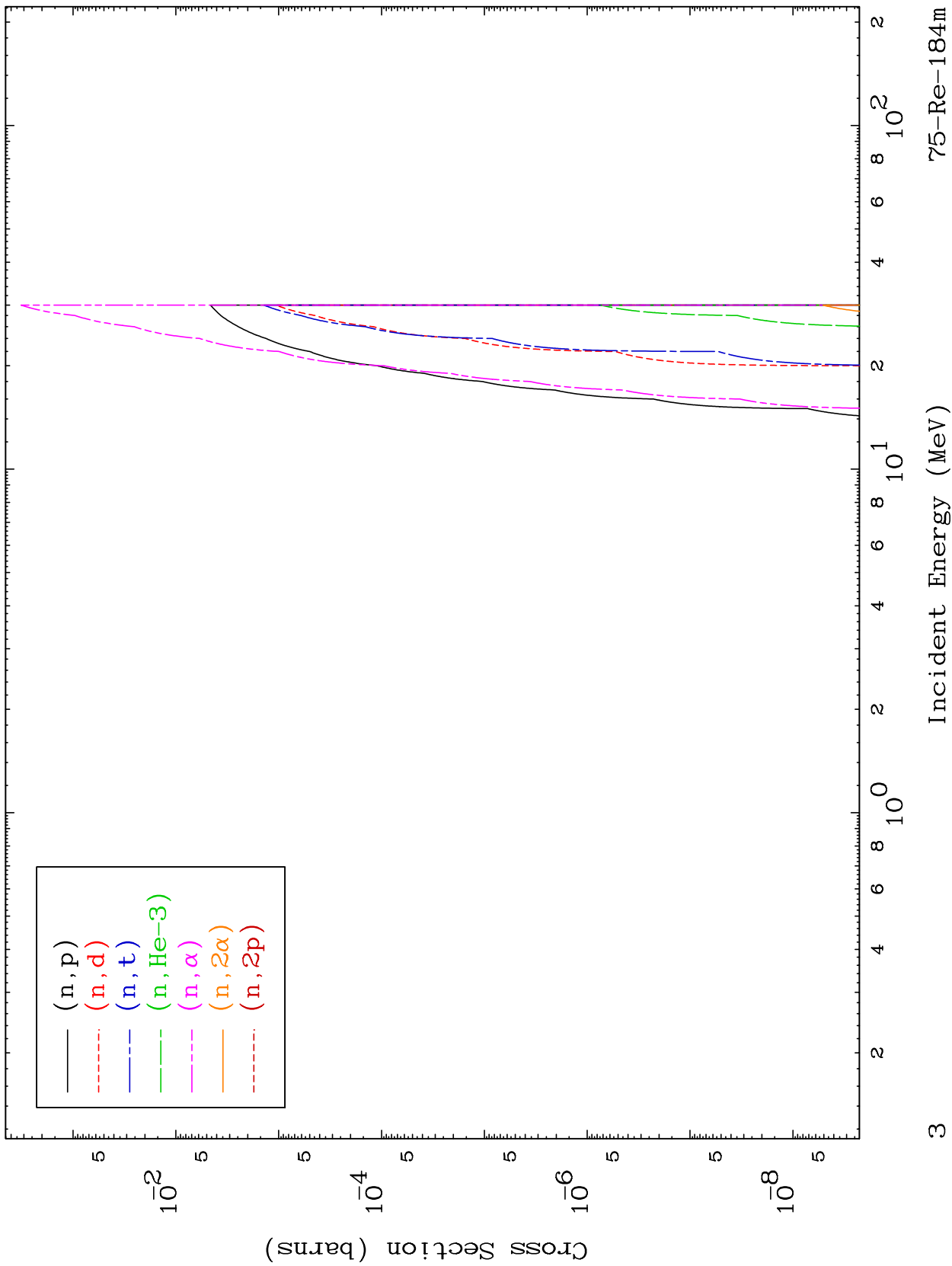
75-Re-184m



MAT 7523

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

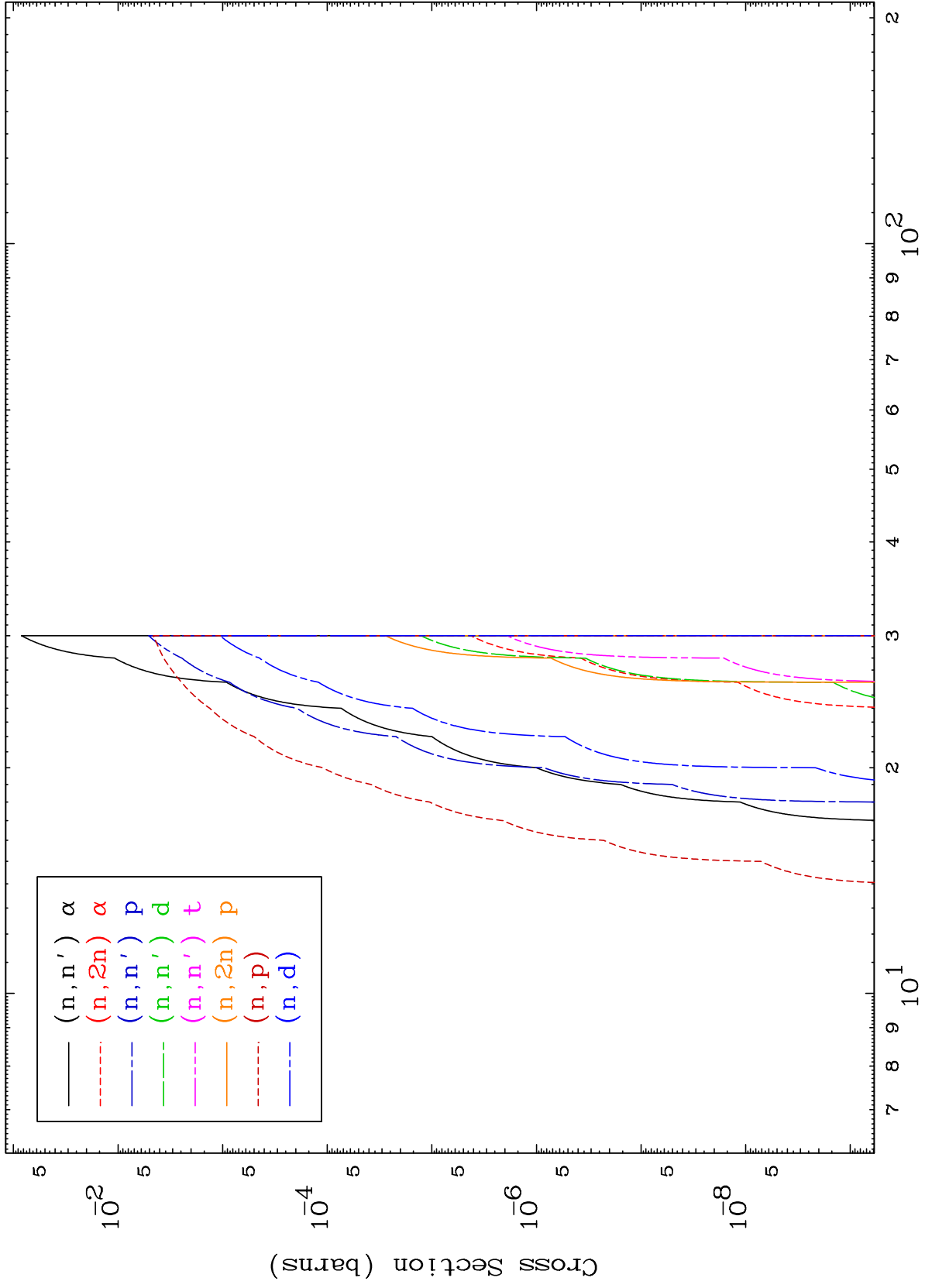
<sup>75</sup>Re-184m

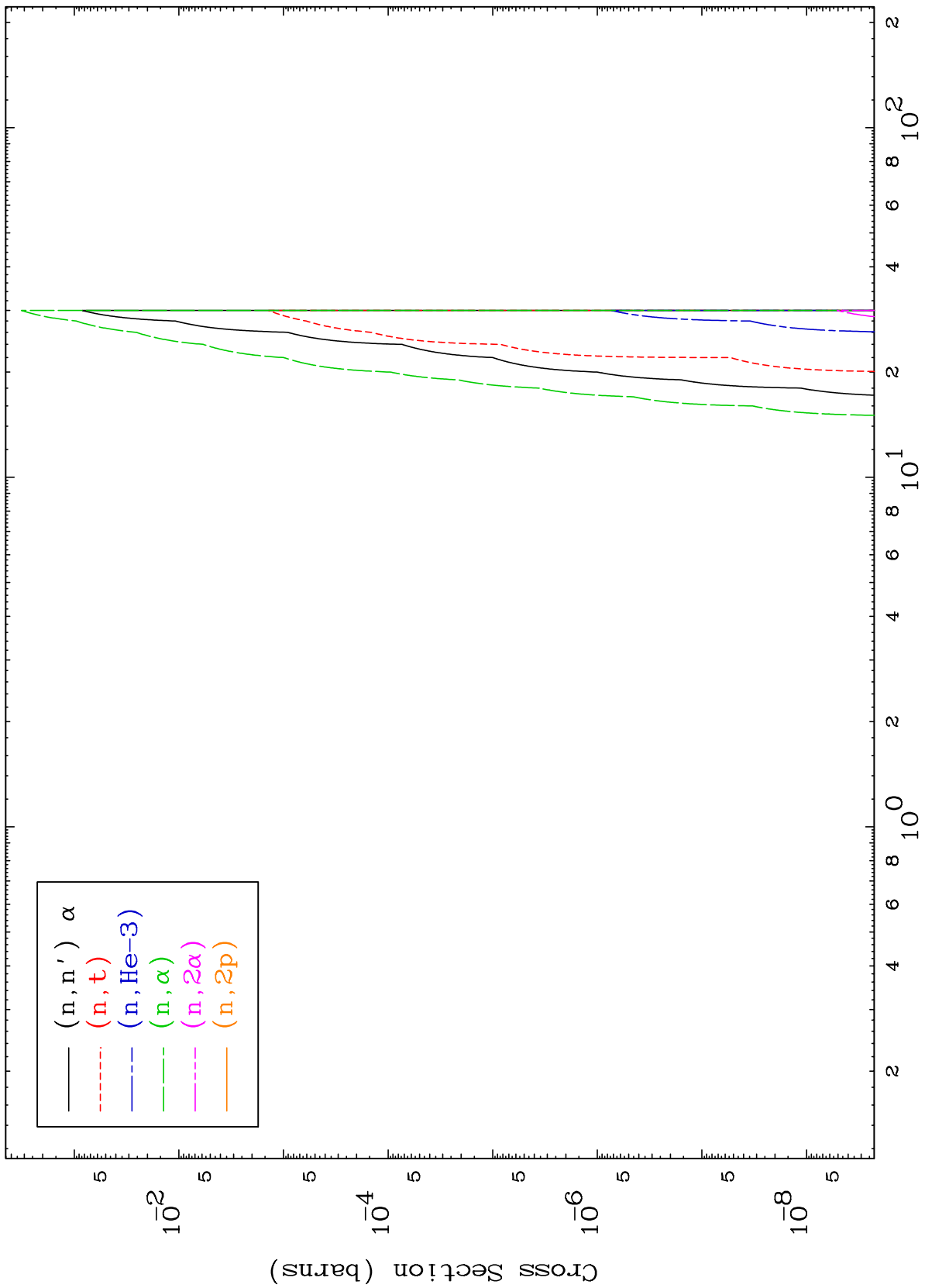


MAT 7523

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

75-Re-184m



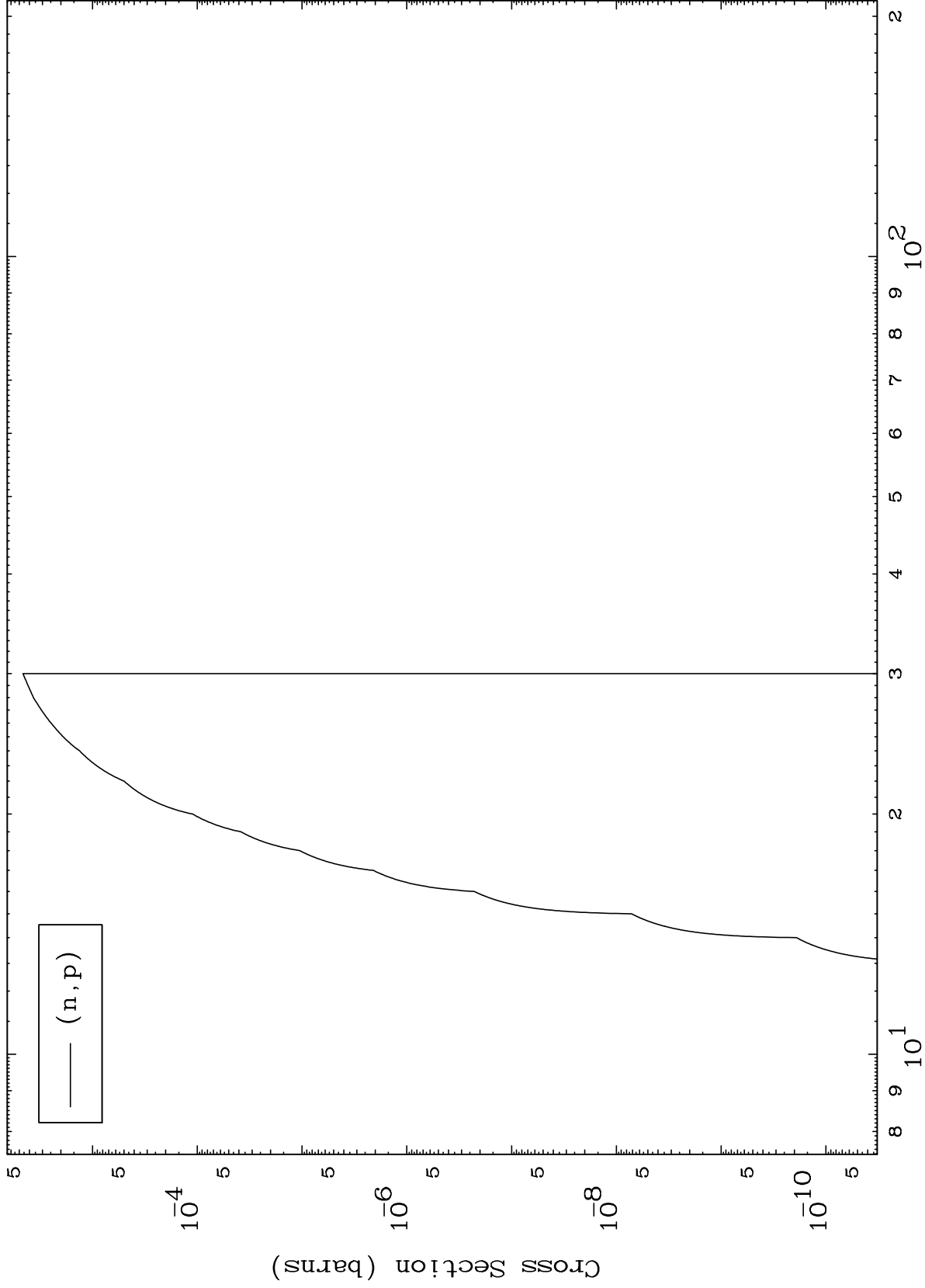


MAT 7523

( $\alpha, p$ ) Levels

75-Re-184m

0 Kelvin Cross Sections



Incident Energy (MeV)

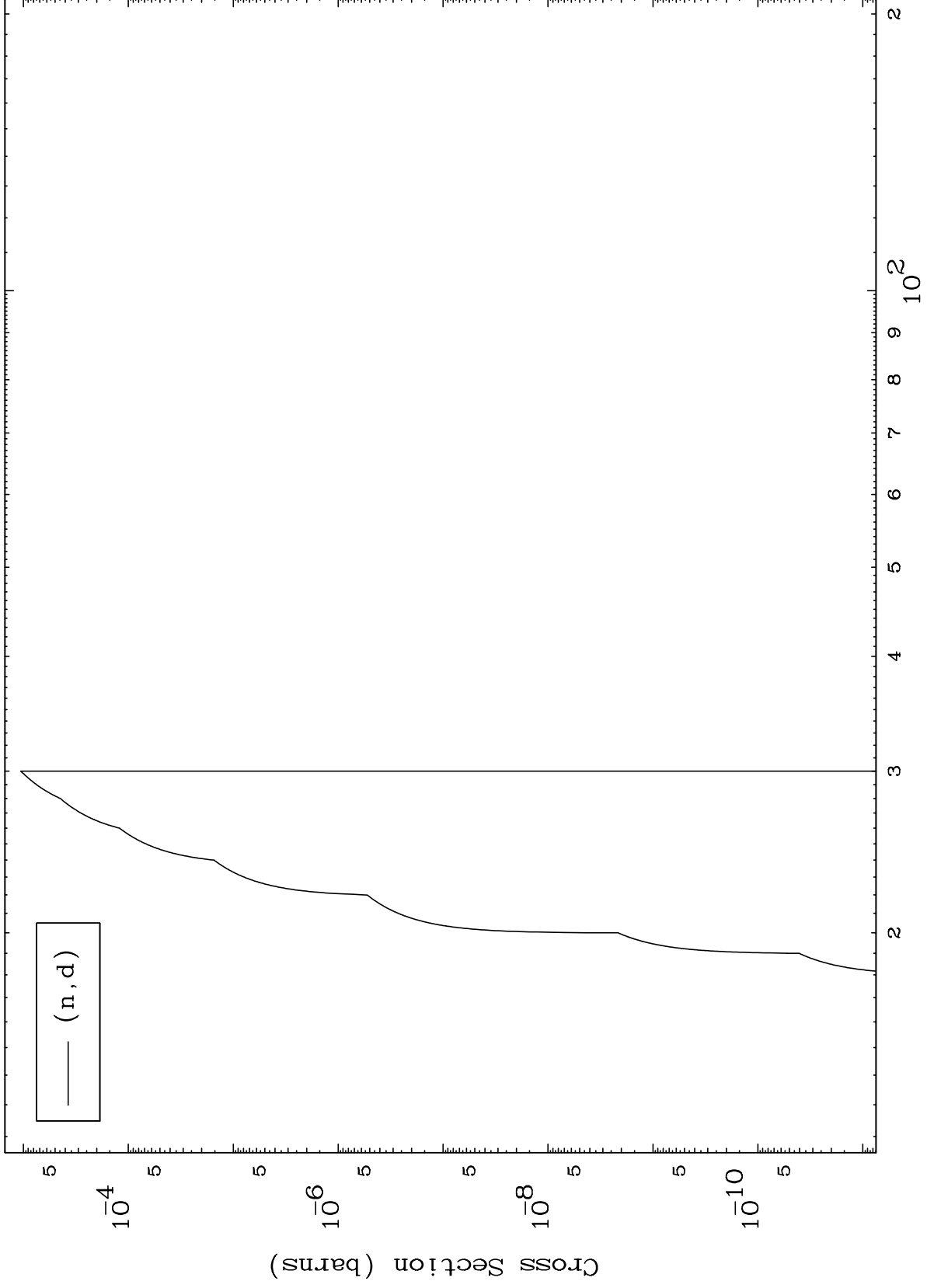
75-Re-184m

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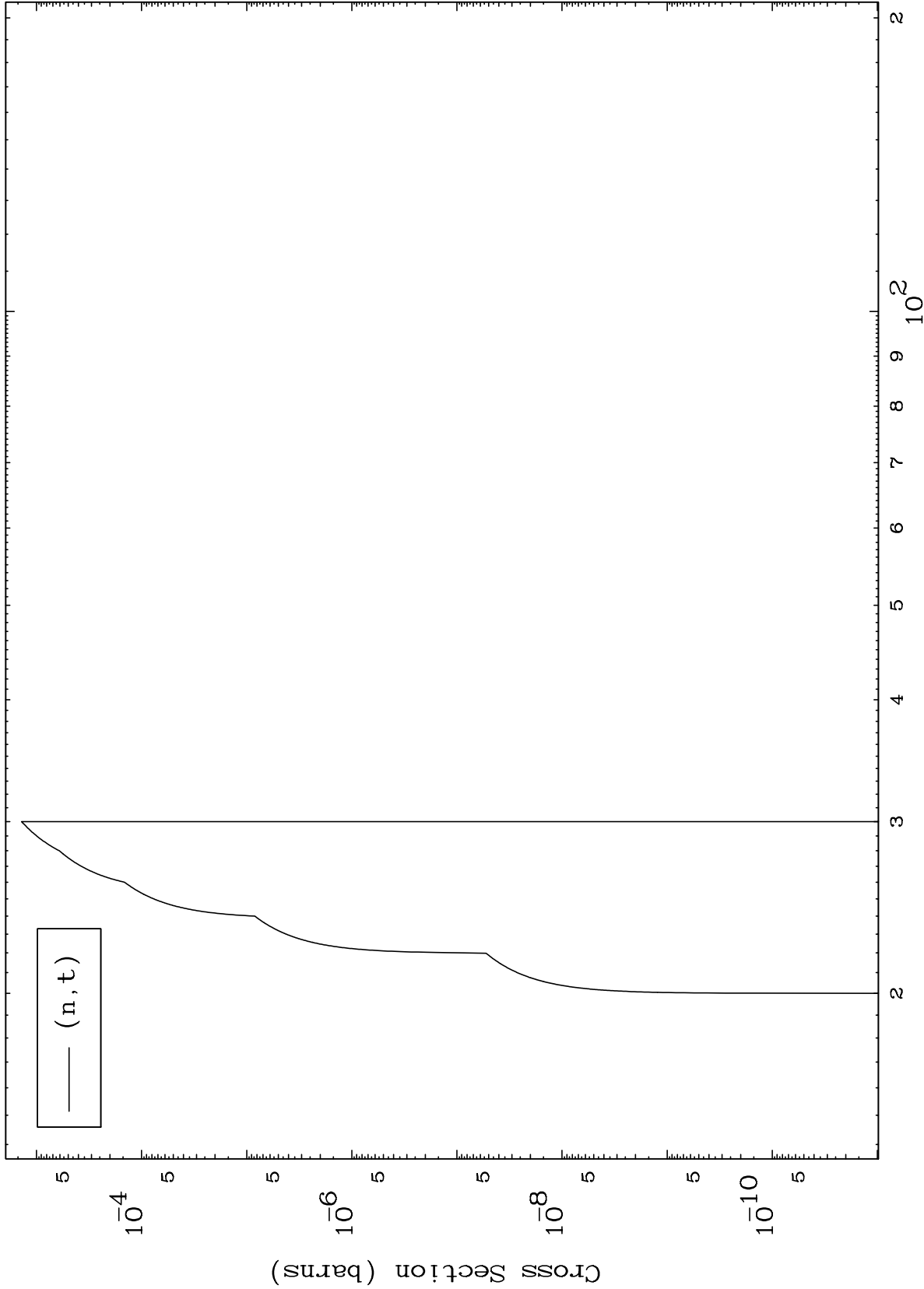
MAT 7523

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

<sup>75</sup>Re-184m



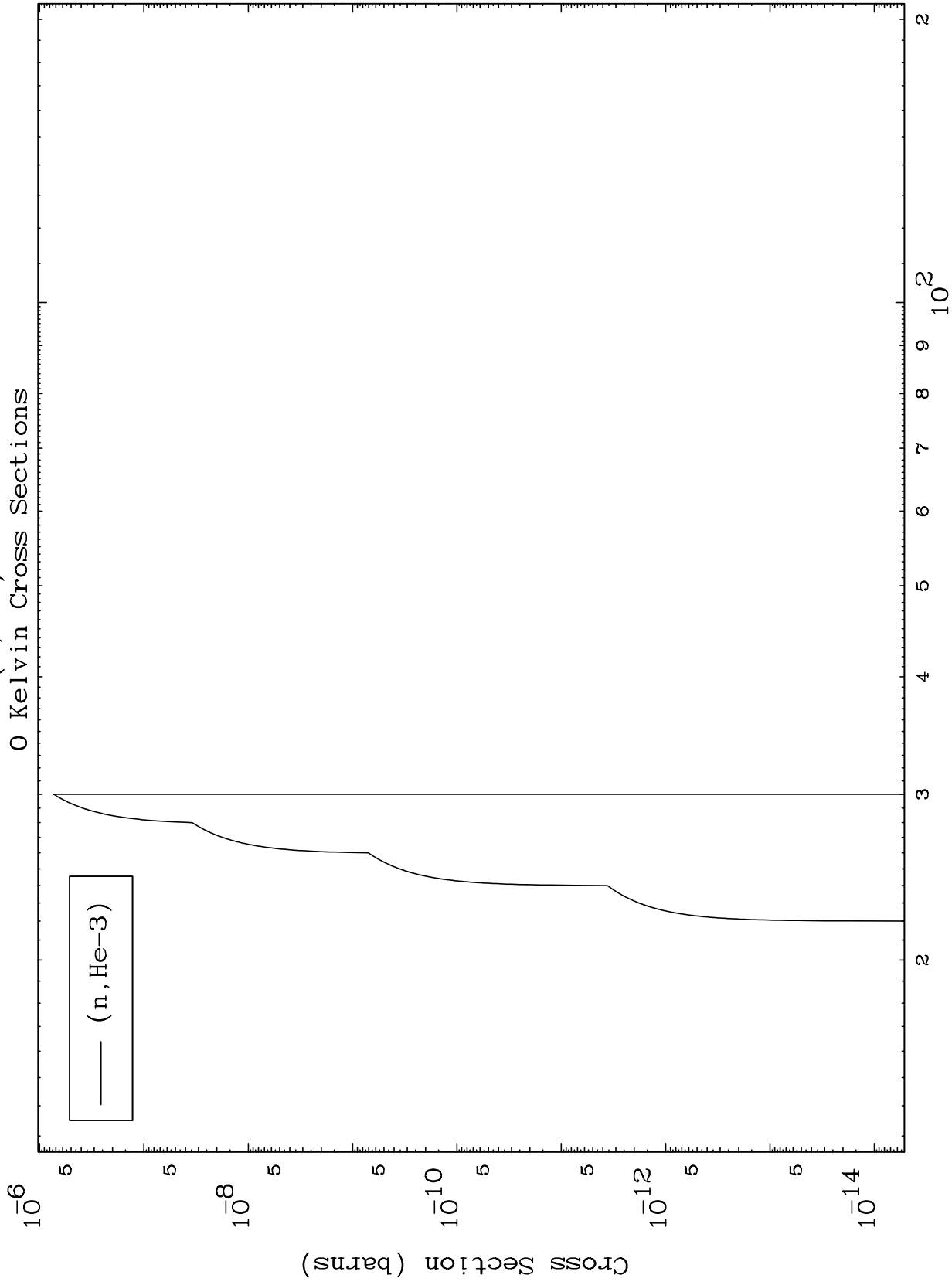




MAT 7523

( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections

75-Re-184m

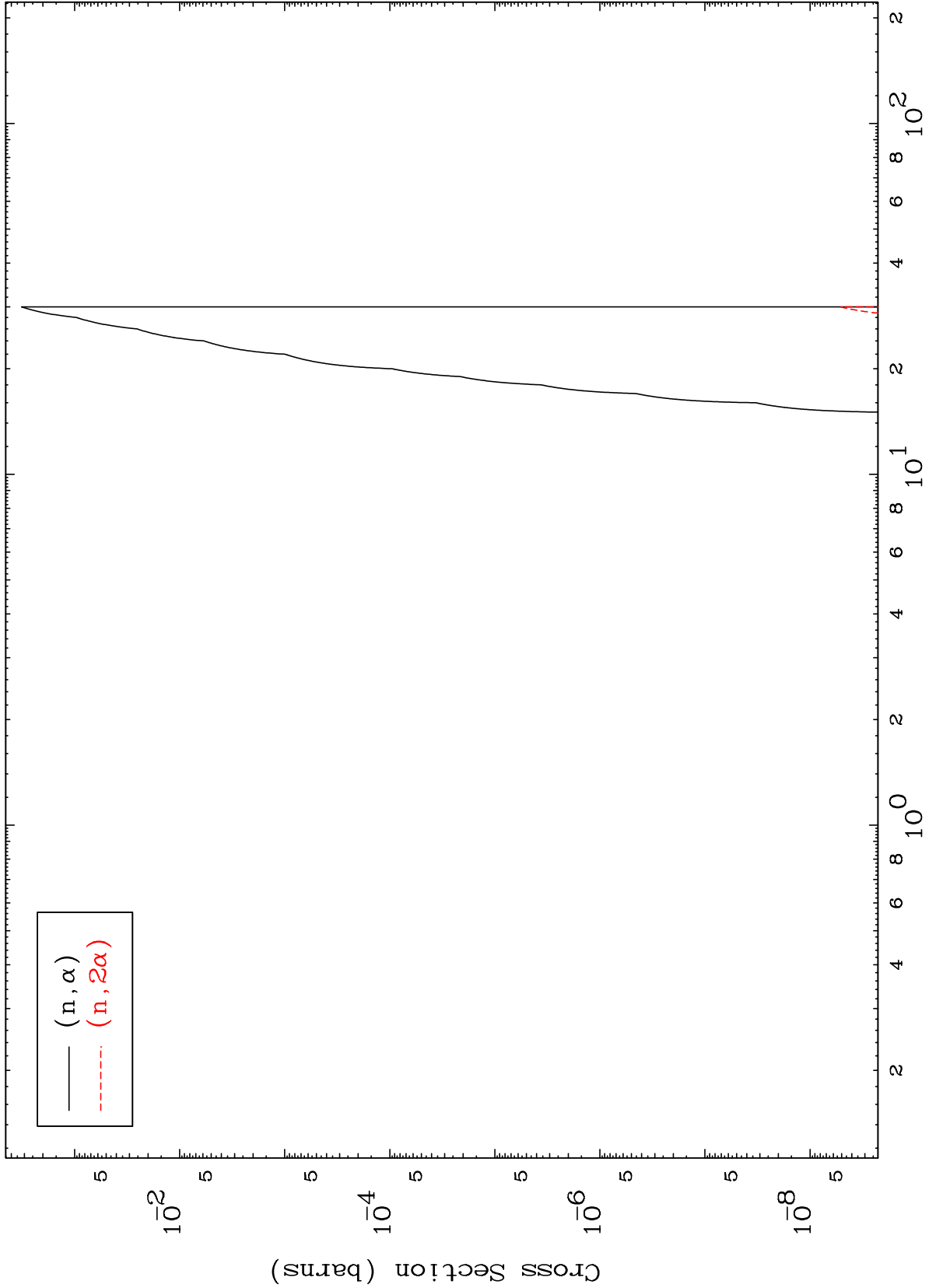


MAT 7523

( $\alpha, \alpha$ ) Levels

75-Re-184m

0 Kelvin Cross Sections



10

Incident Energy (MeV)

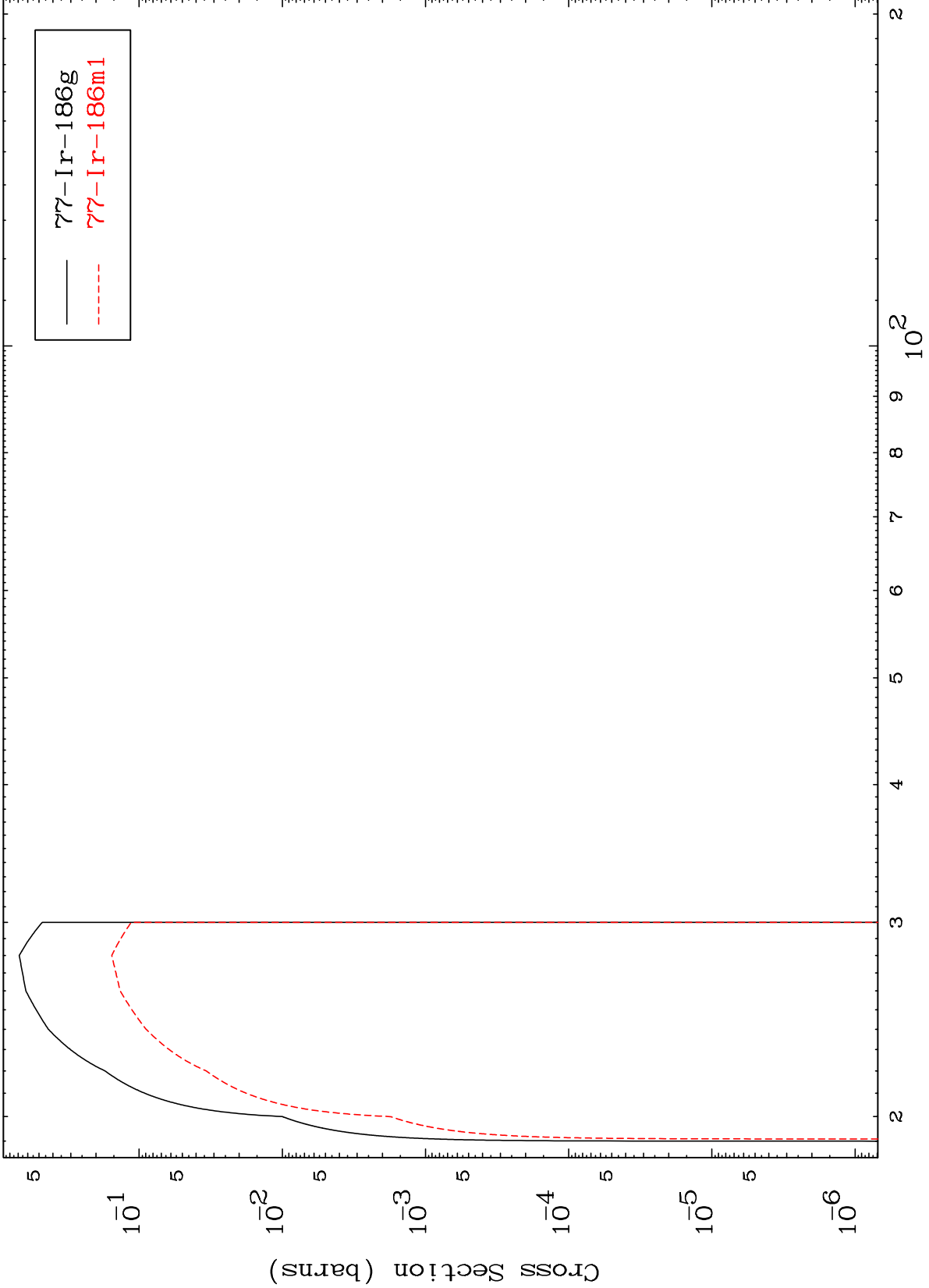
75-Re-184m

MAT 7523

(n,2n)

75-Re-184m

Radionuclide Production Cross Section

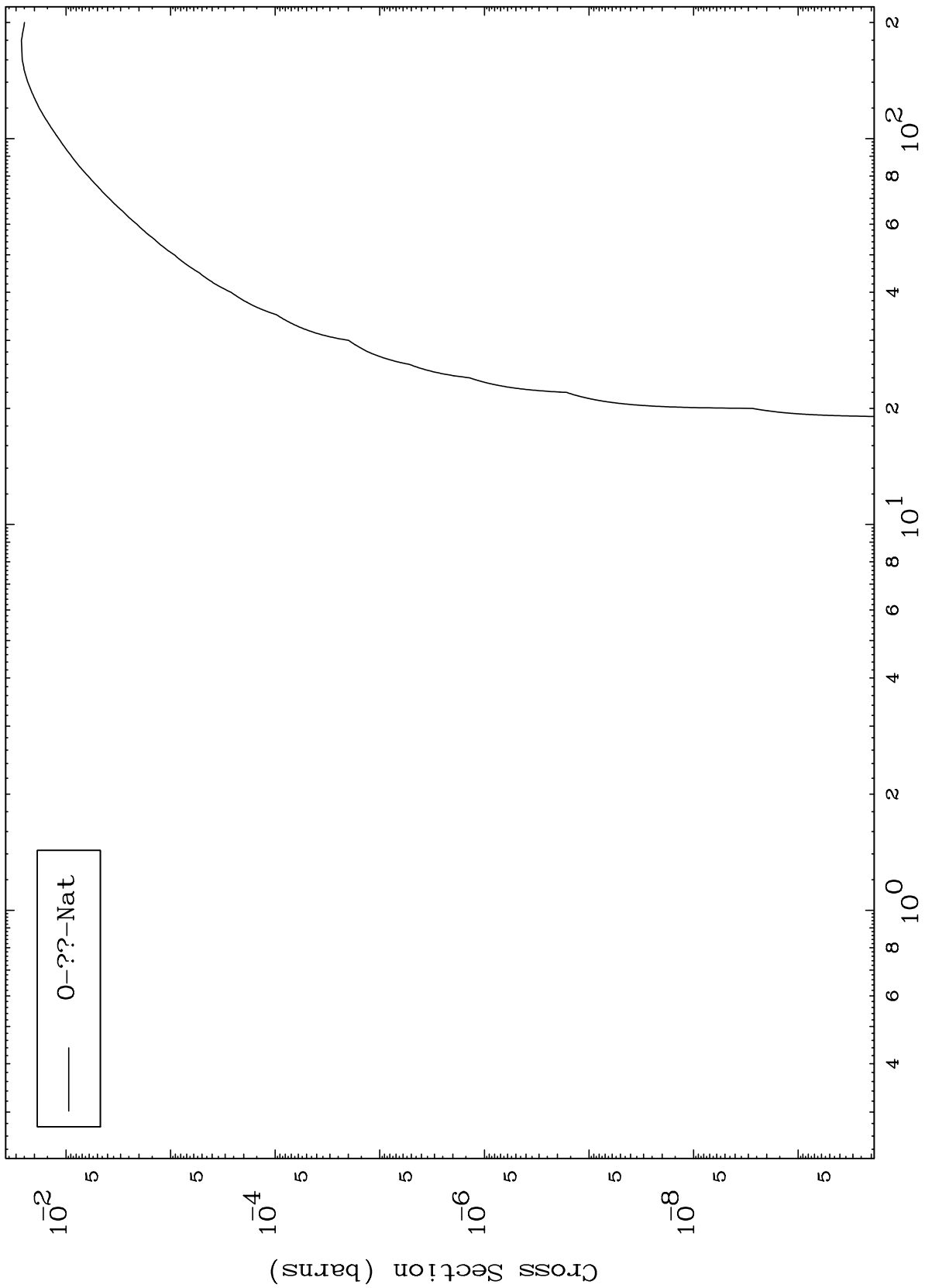


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Fission

75-Re-184m

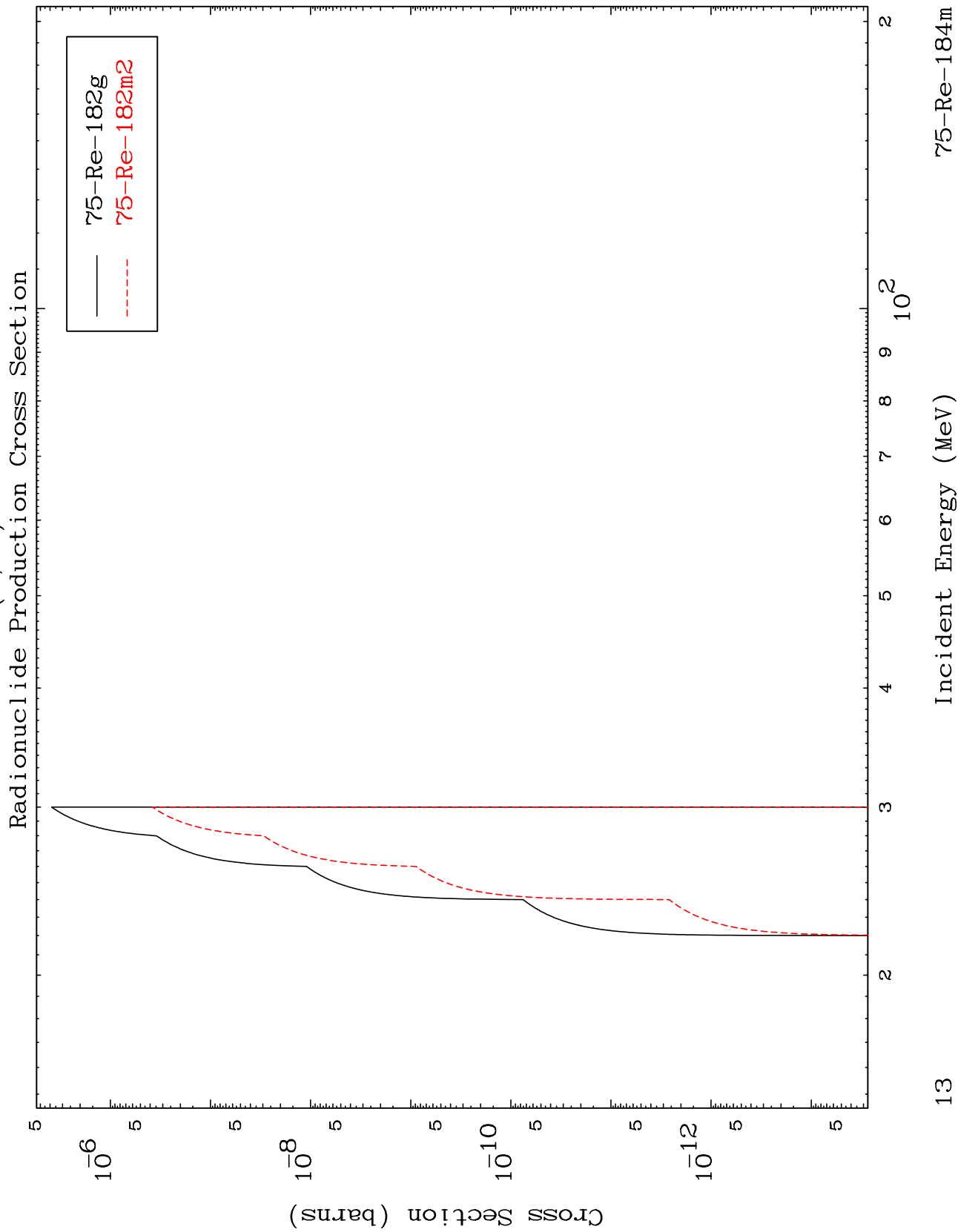
Radionuclide Production Cross Section



MAT 7523

$(n,2n) \alpha$

<sup>75</sup>Re-184m

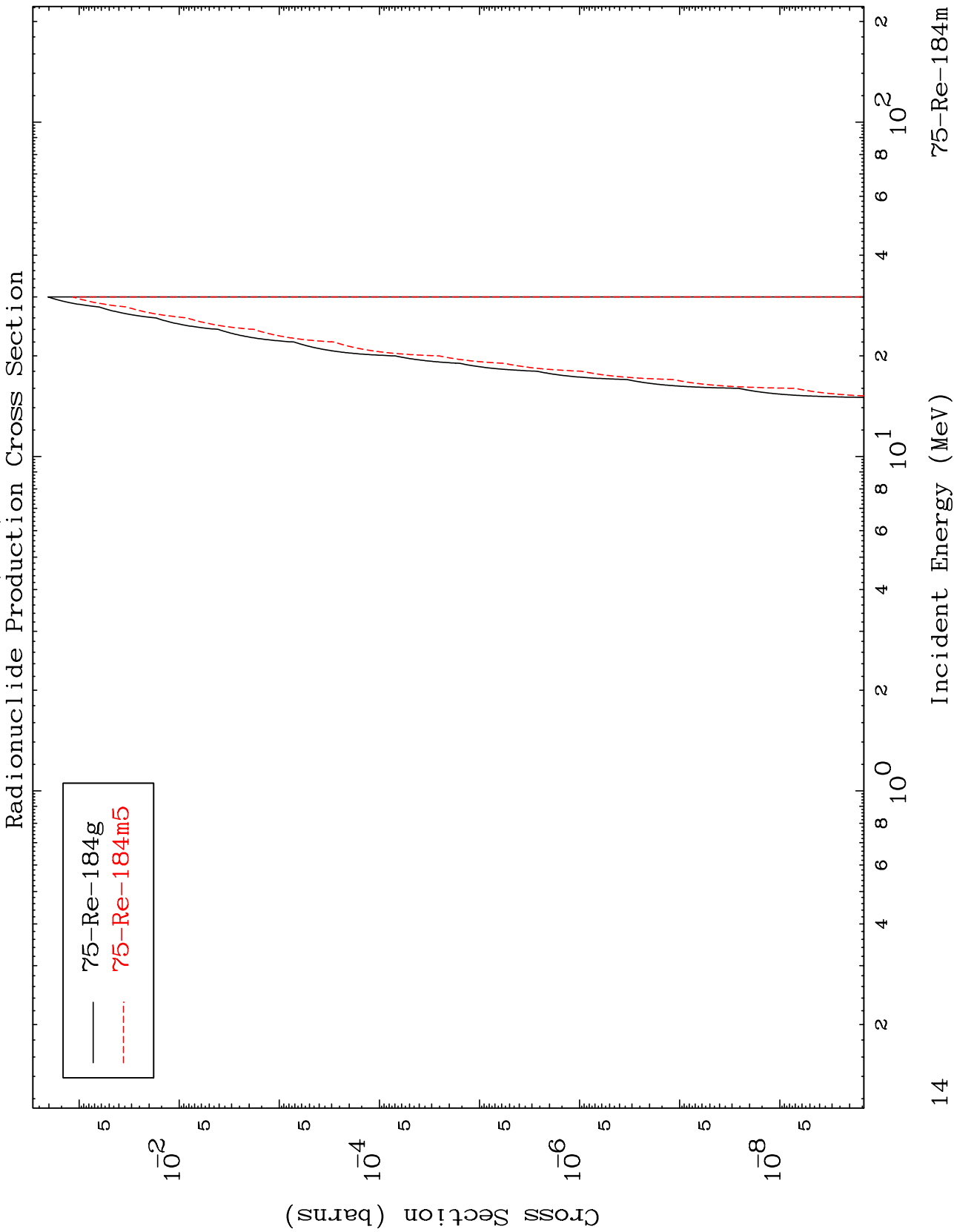


13

<sup>75</sup>Re-184m

MAT 7523

<sup>75</sup>Re-184m

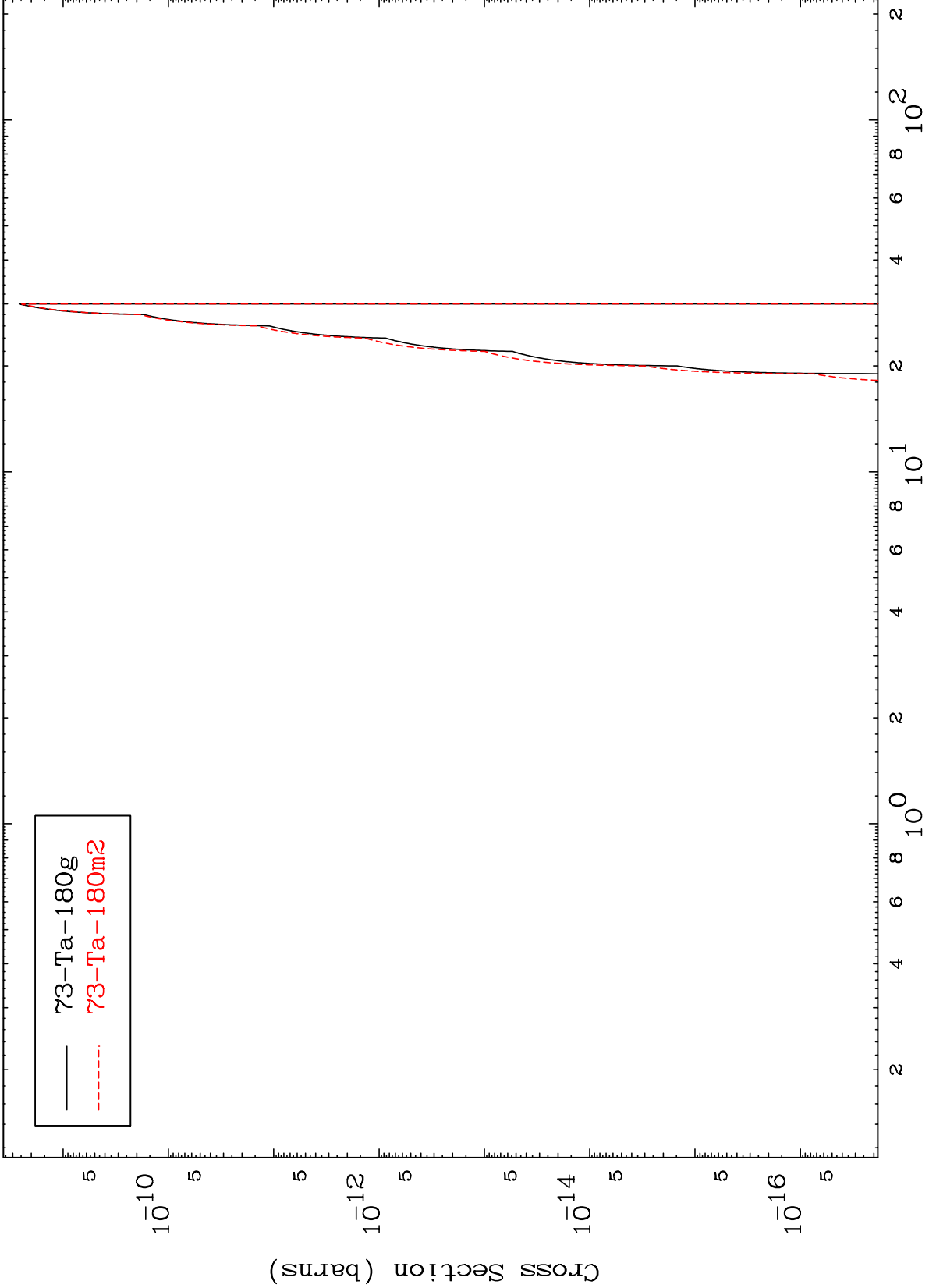


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(n,2α)

75-Re-184m

Radionuclide Production Cross Section



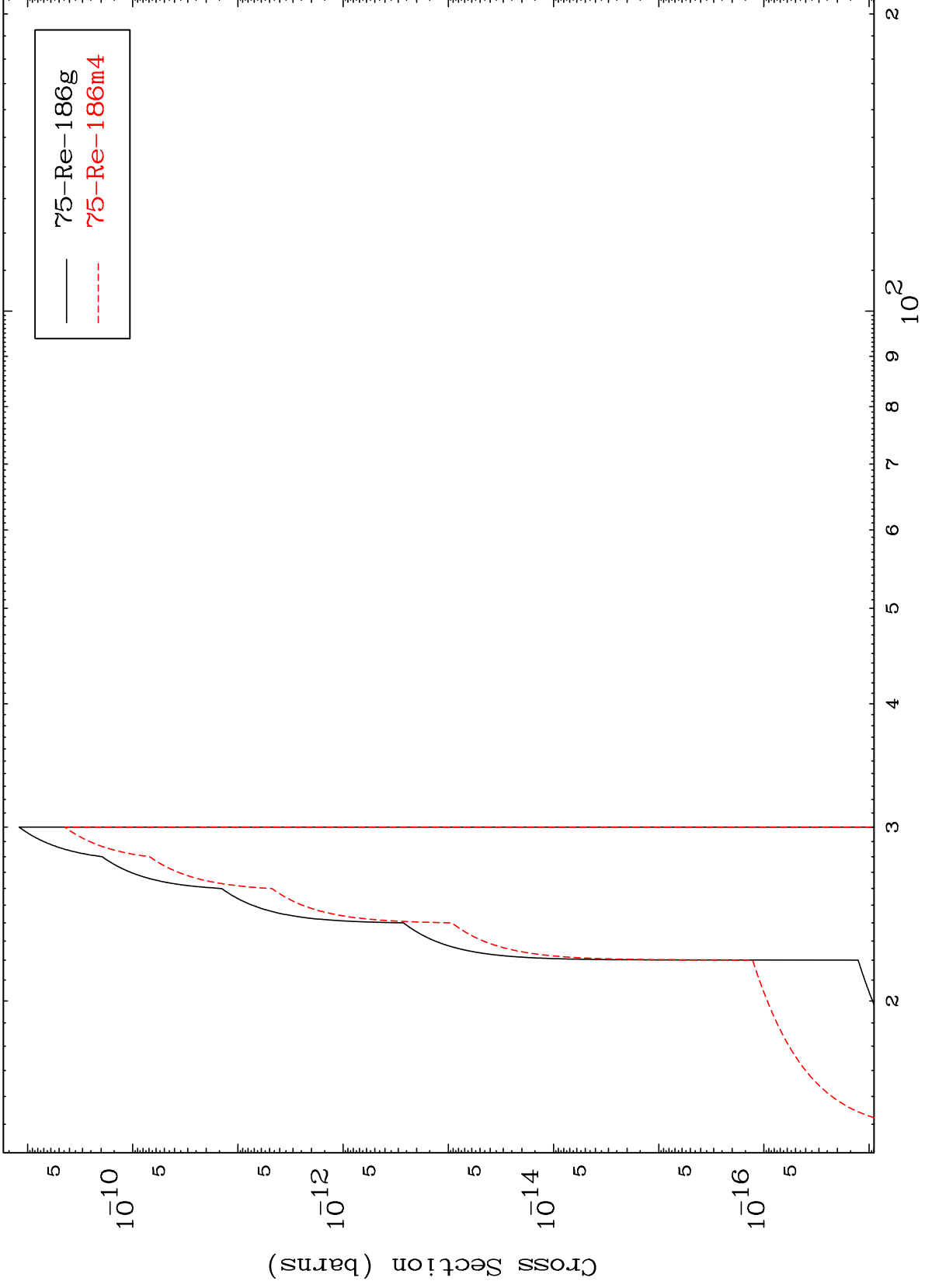
73-Ta-180g  
73-Ta-180m2



MAT 7523

75-Re-184m

(n,2p)  
Radionuclide Production Cross Section



16

Incident Energy (MeV)

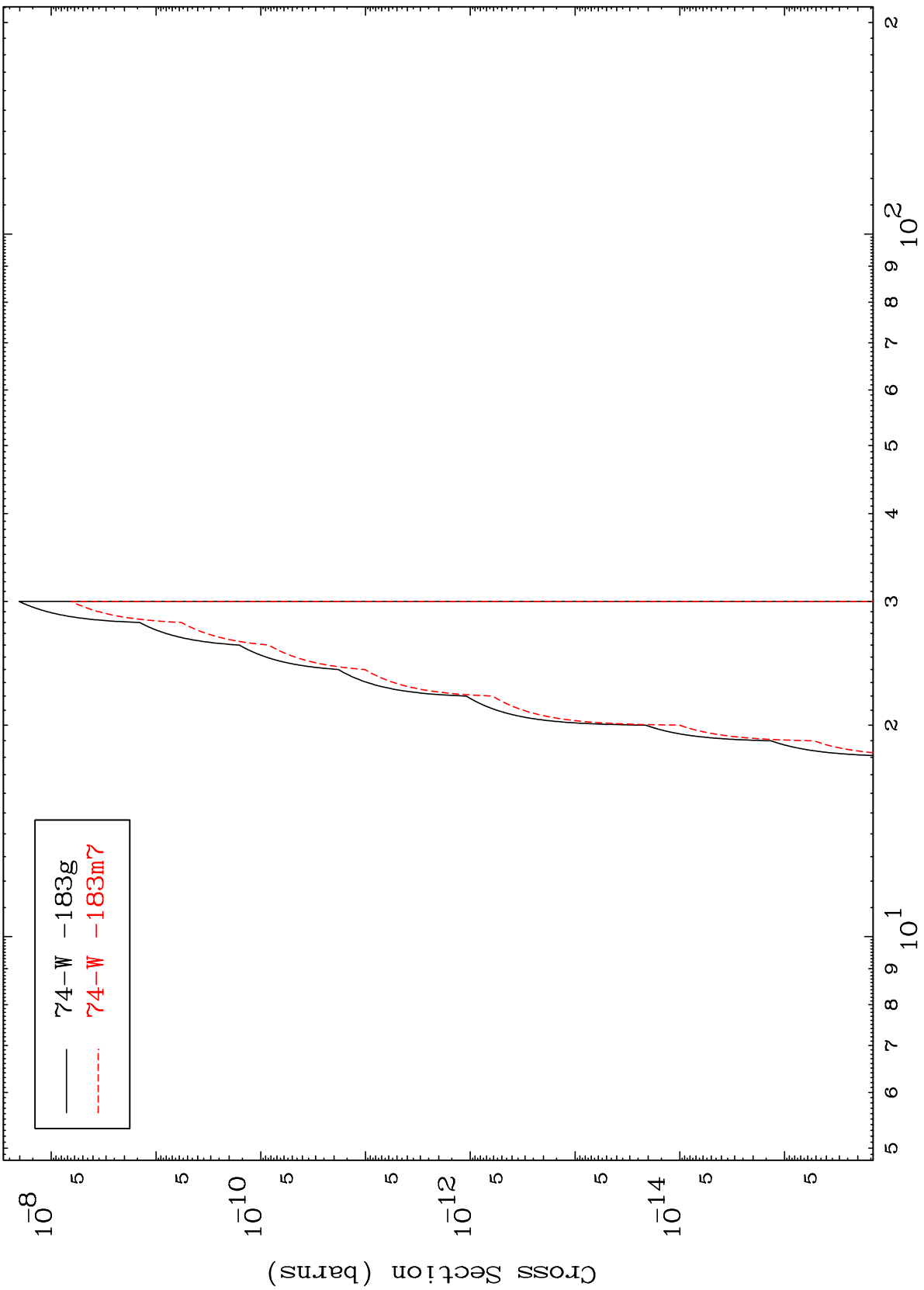
75-Re-184m

MAT 7523

(n,p)  $\alpha$

<sup>75</sup>Re-184m

Radionuclide Production Cross Section



74-W -183g  
74-W -183m7

17

Incident Energy (MeV)

<sup>75</sup>Re-184m