

Program EVALPLOT  
(Version 2021-1)

by

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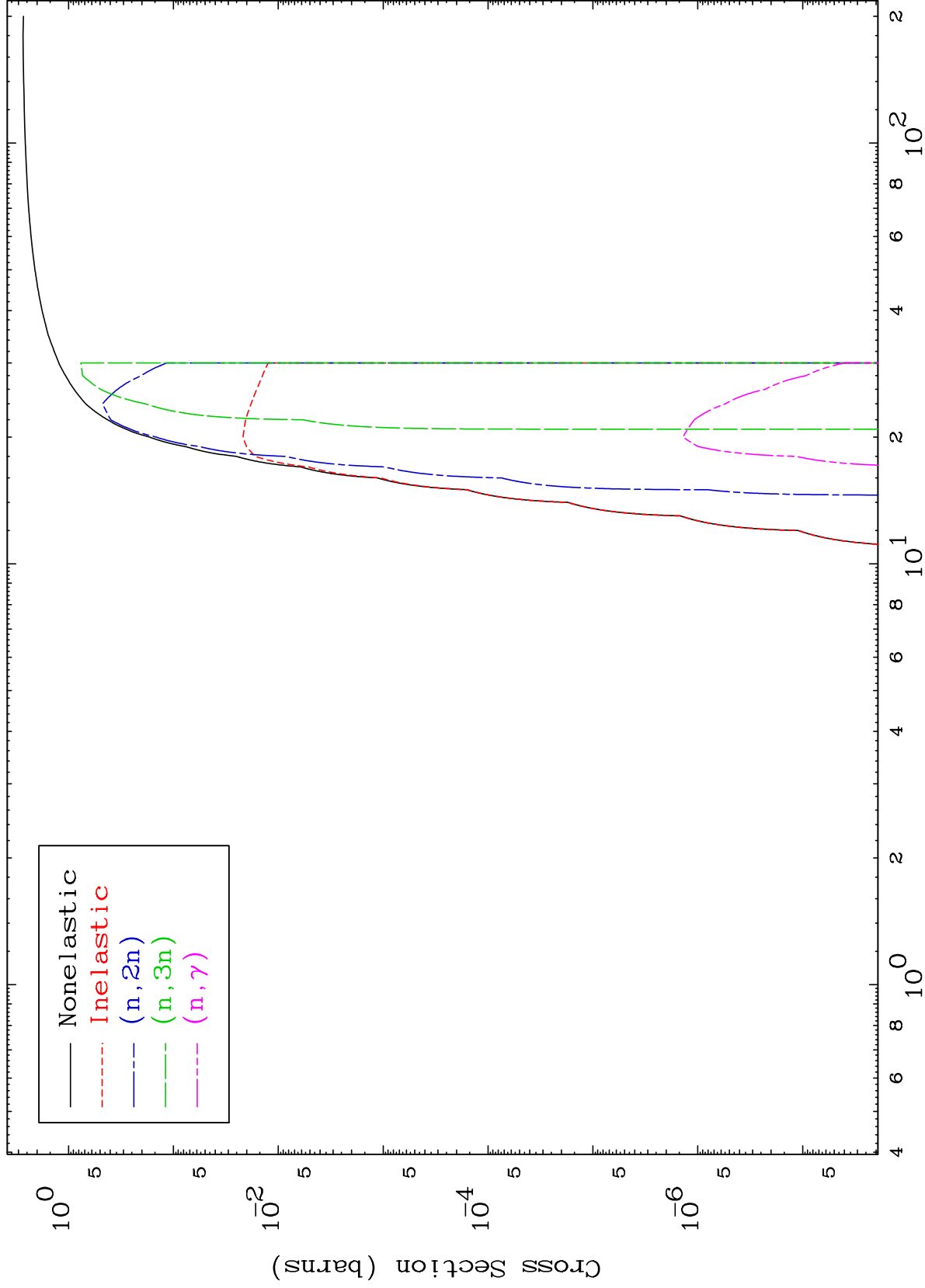
Press Mouse Button to Start

MAT 7541

$\alpha$  Major

75-Re-190m

0 Kelvin Cross Sections



Incident Energy (MeV)

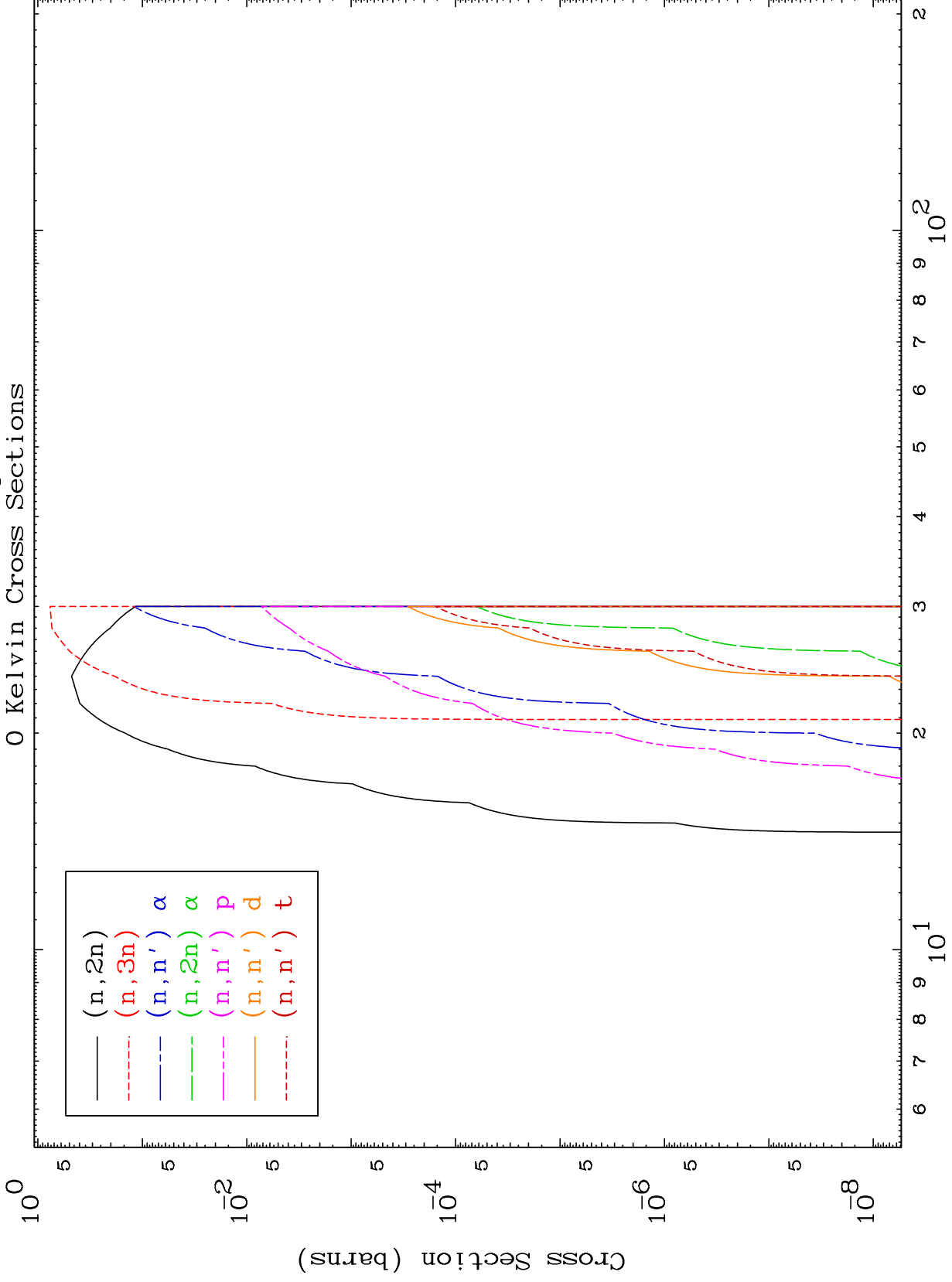
75-Re-190m

1

MAT 7541

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

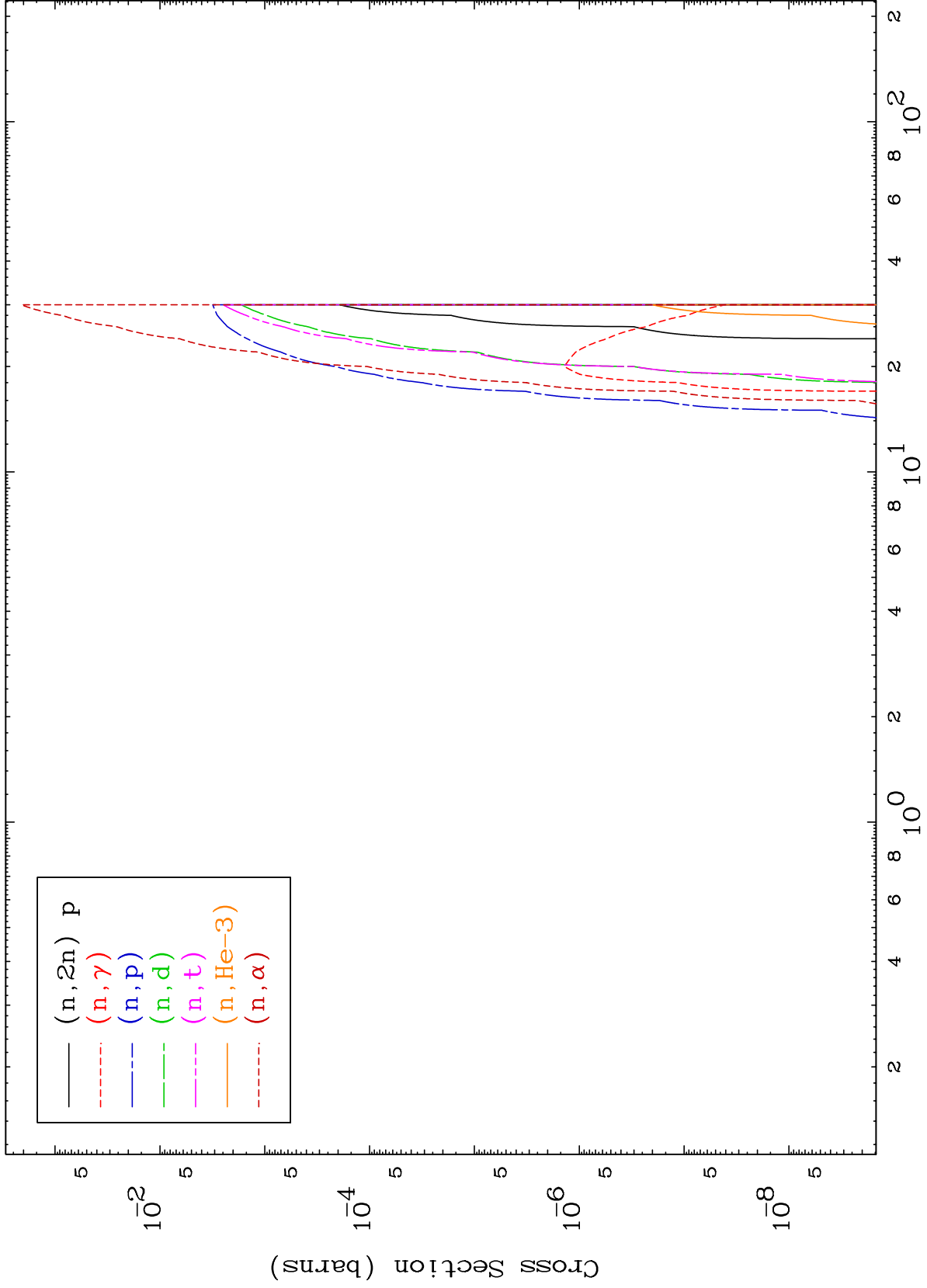
75-Re-190m



MAT 7541

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

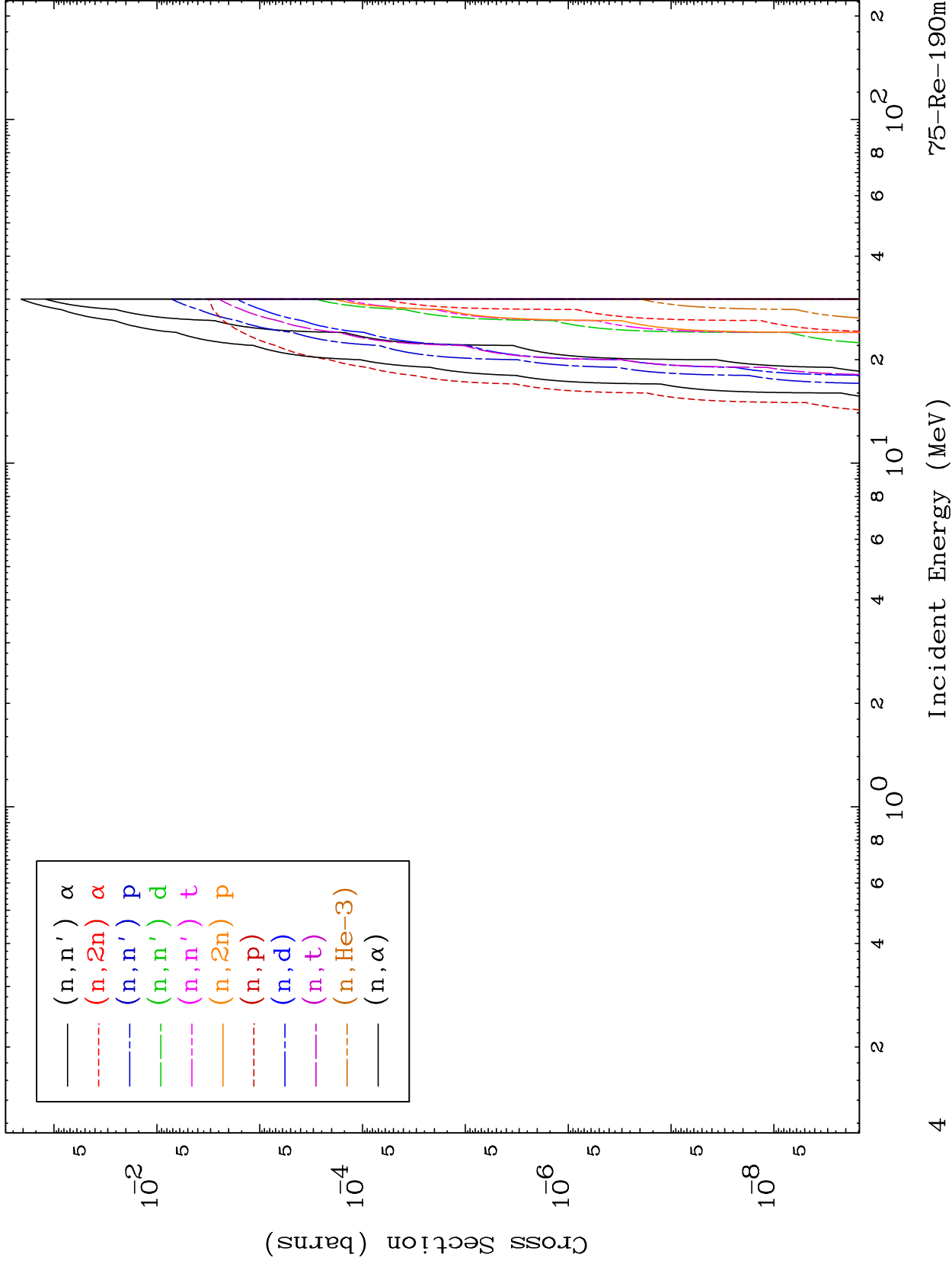
75-Re-190m



MAT 7541

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

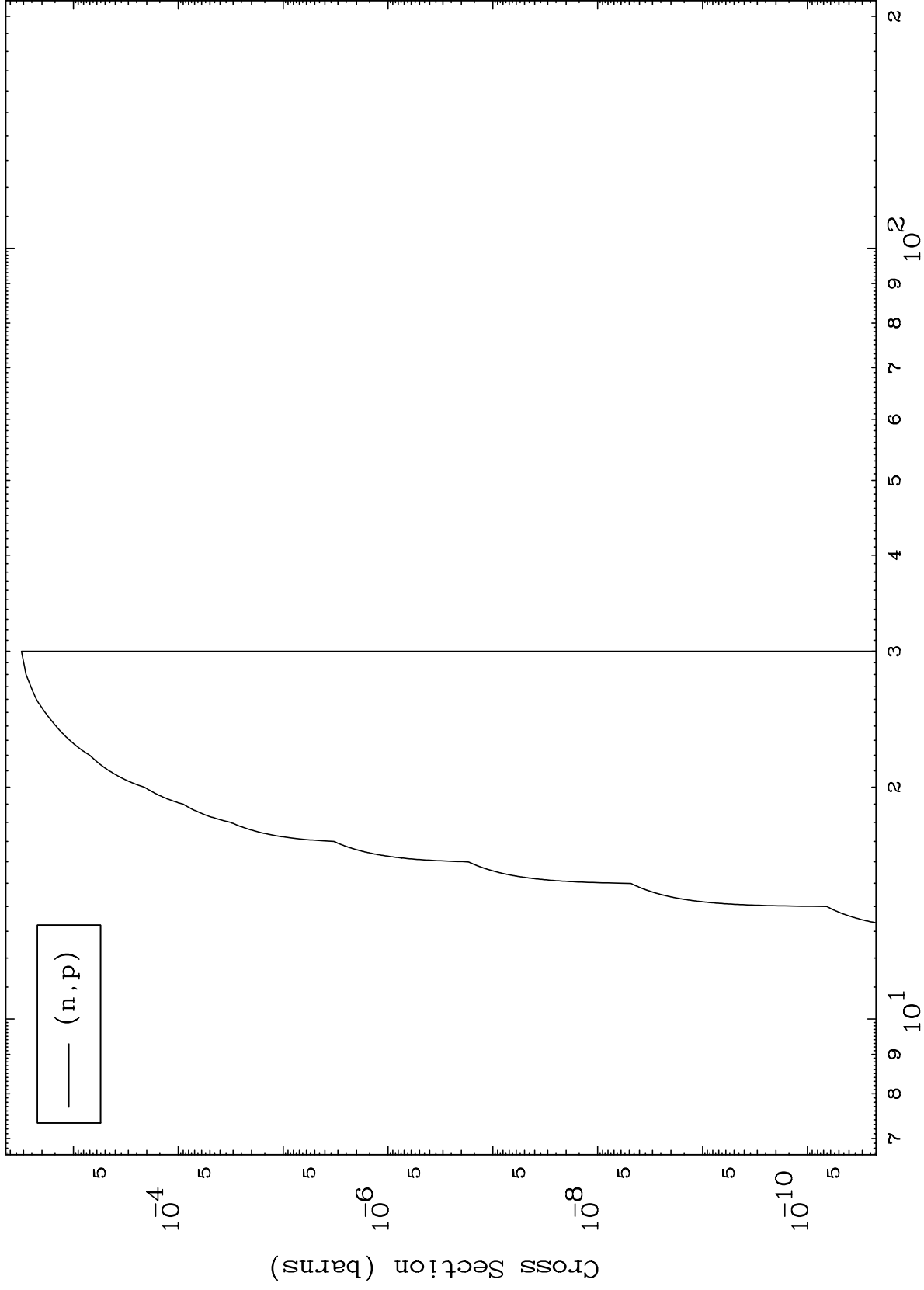
75-Re-190m



MAT 7541

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

75-Re-190m



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Incident Energy (MeV)

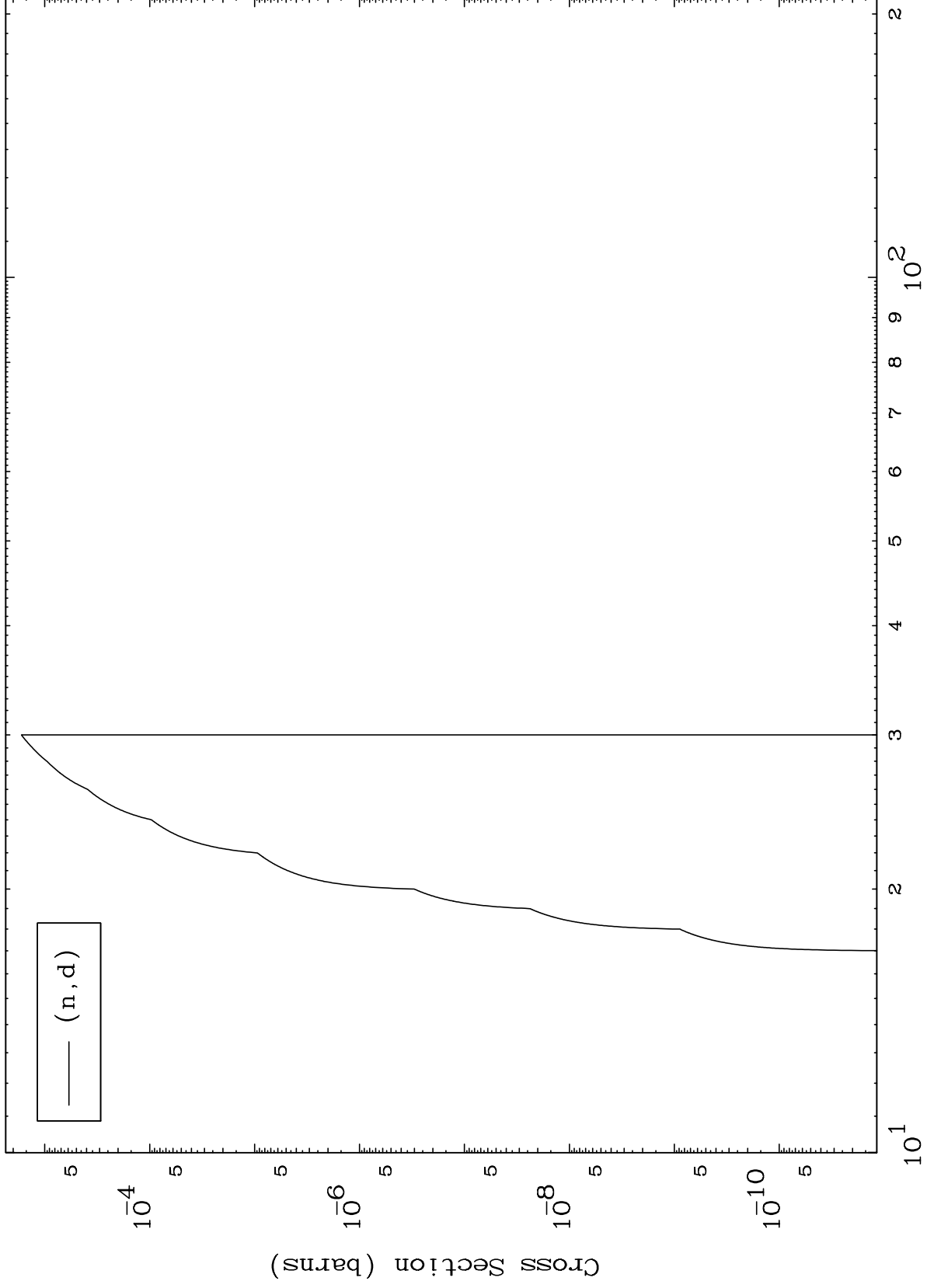
75-Re-190m

MAT 7541

( $\alpha, d$ ) Levels

<sup>75</sup>Re-190m

0 Kelvin Cross Sections



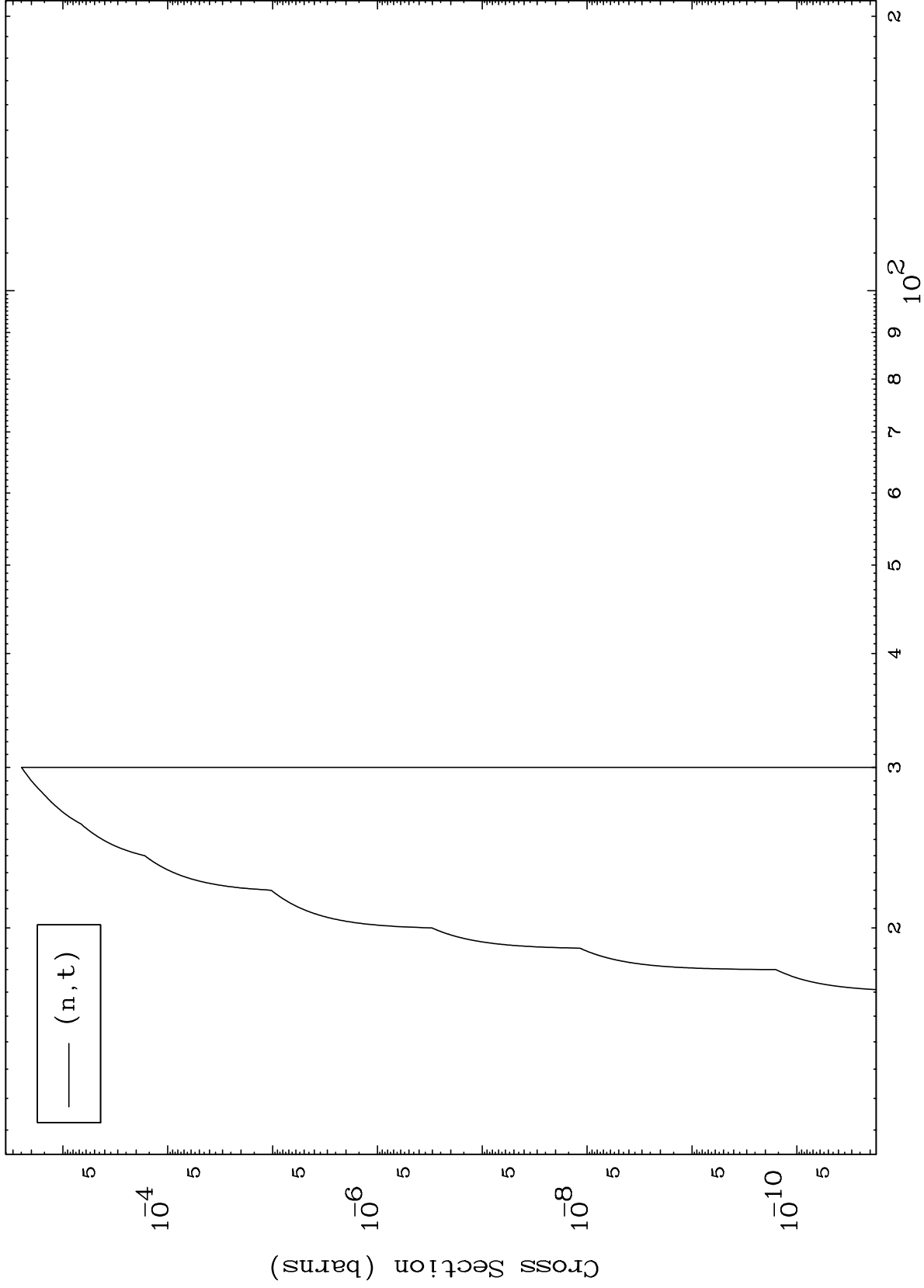
Incident Energy (MeV)

<sup>75</sup>Re-190m

MAT 7541

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

75-Re-190m



7

Incident Energy (MeV)

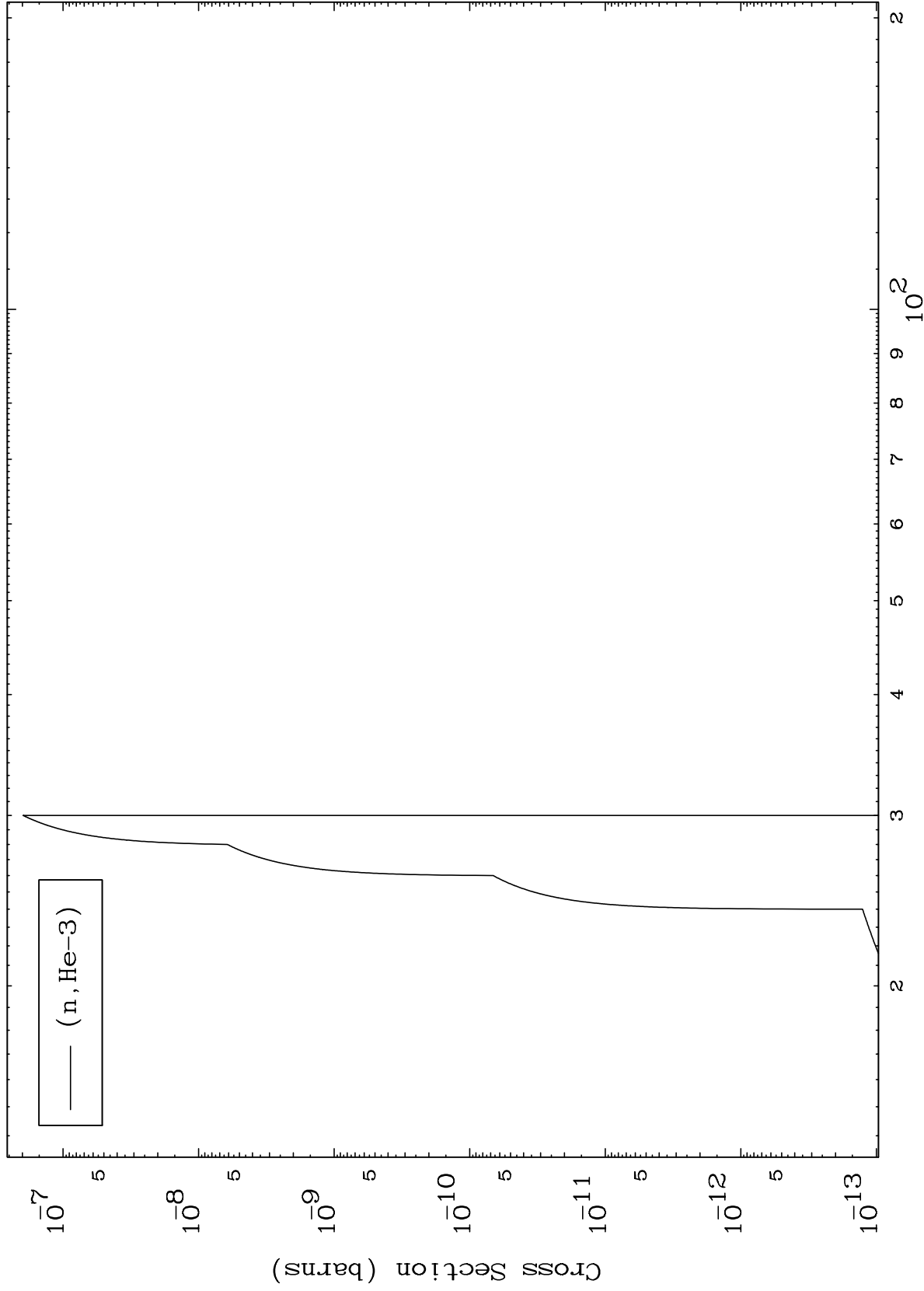
75-Re-190m



MAT 7541

( $\alpha$ , He3) Levels  
0 Kelvin Cross Sections

75-Re-190m

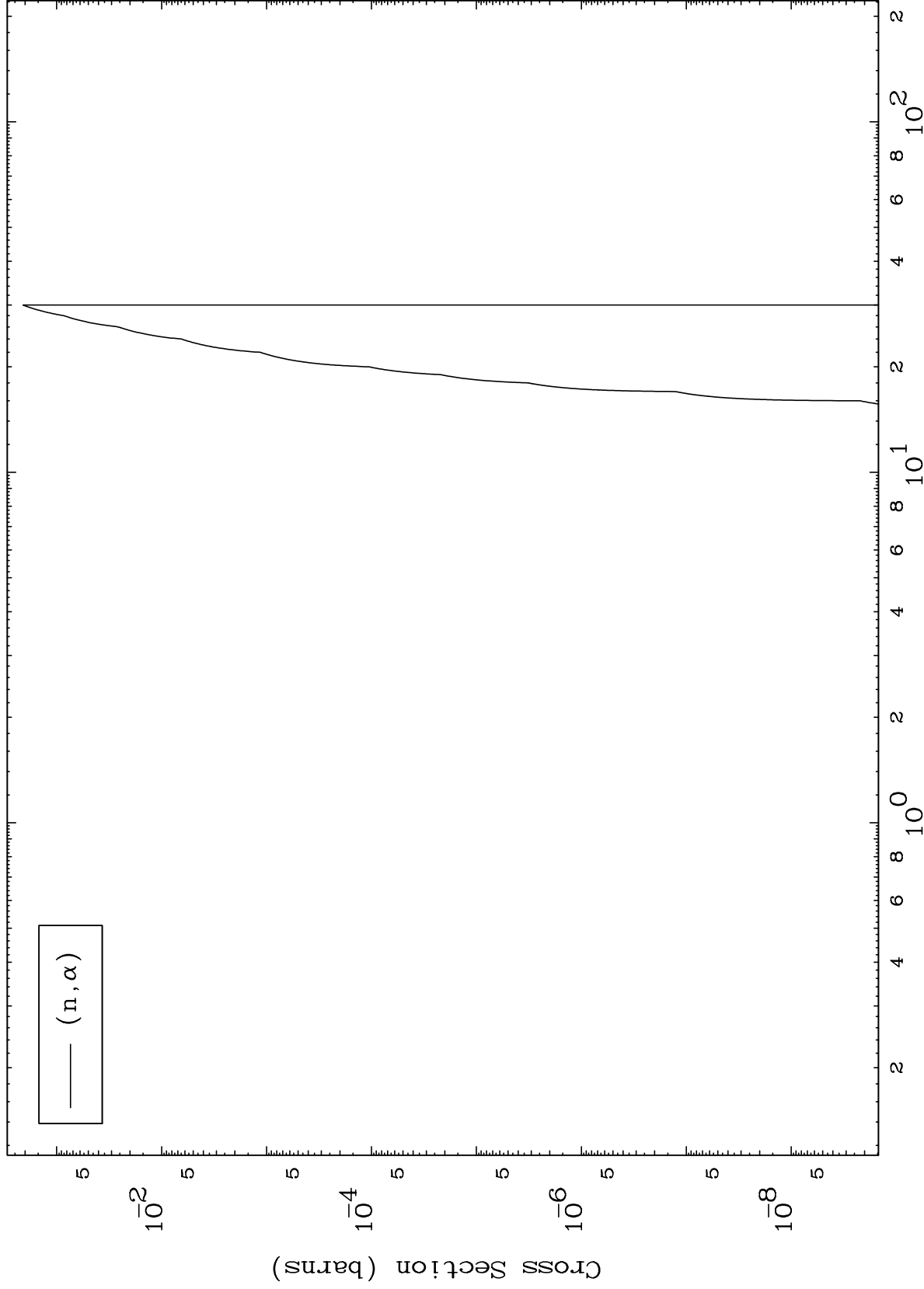


MAT 7541

( $\alpha, \alpha$ ) Levels

75-Re-190m

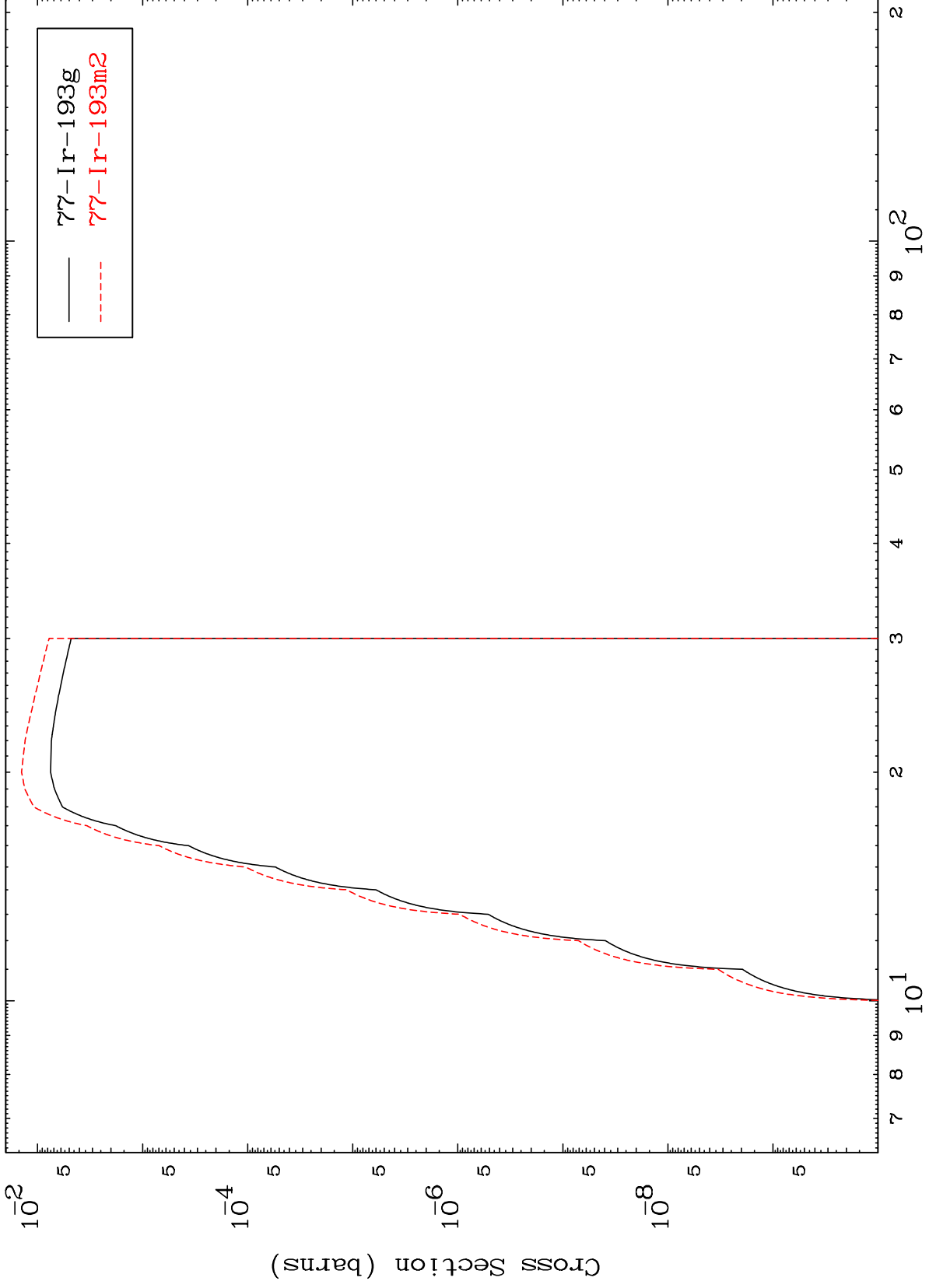
0 Kelvin Cross Sections



MAT 7541

75-Re-190m

Inelastic  
Radionuclide Production Cross Section



10

Incident Energy (MeV)

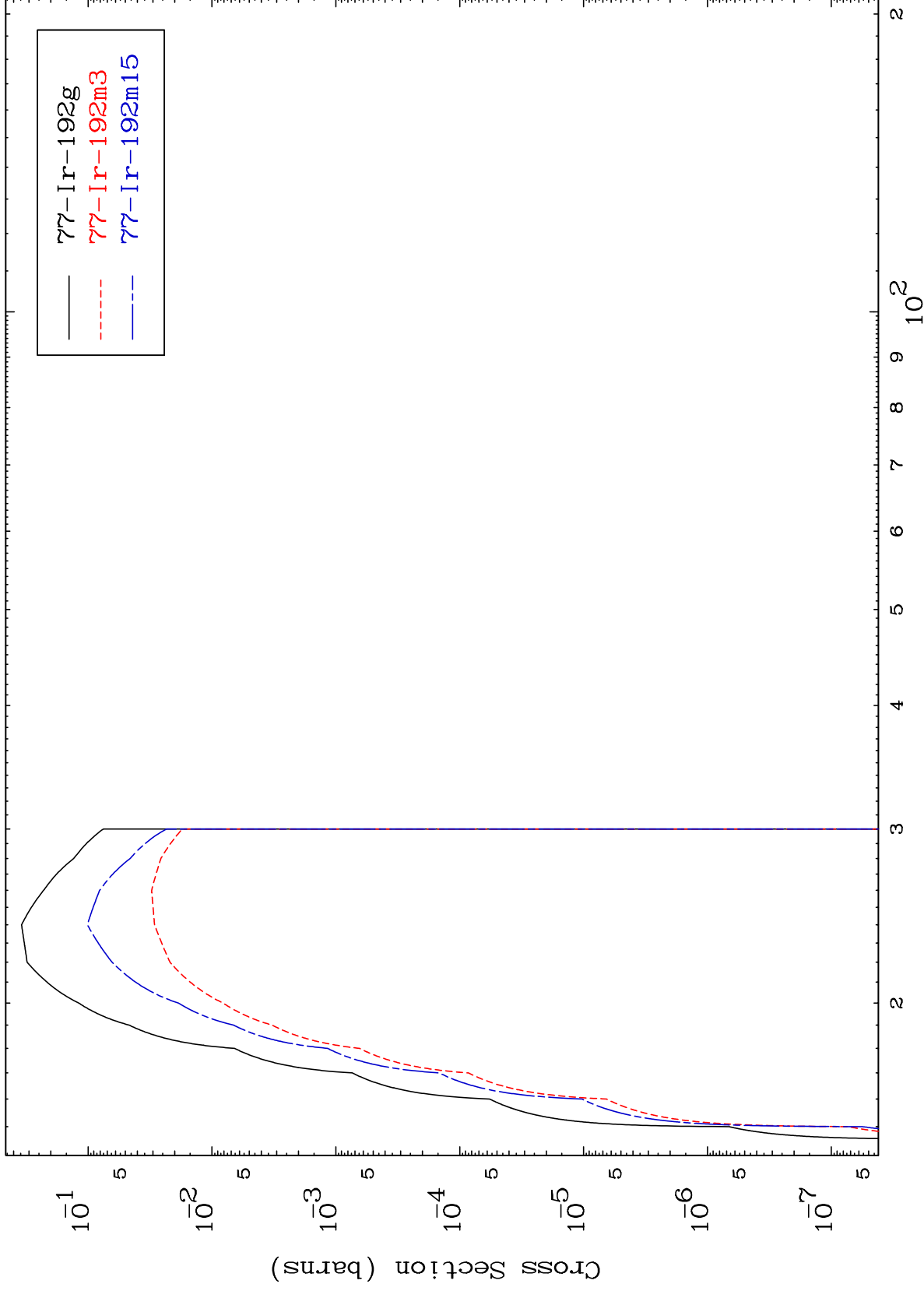
75-Re-190m

MAT 7541

(n,2n)

75-Re-190m

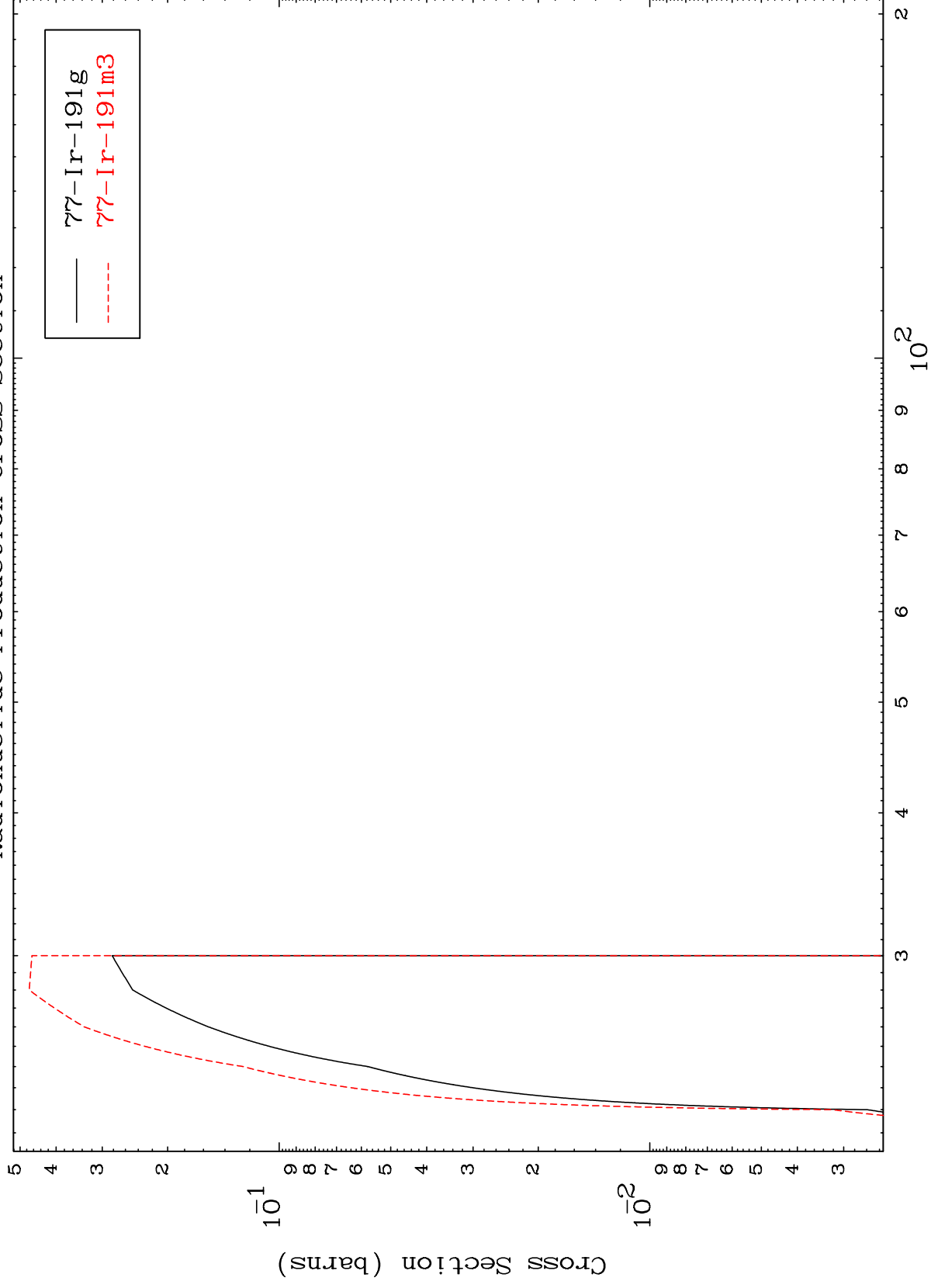
Radionuclide Production Cross Section



MAT 7541

75-Re-190m

(n,3n)  
Radionuclide Production Cross Section



12

Incident Energy (MeV)

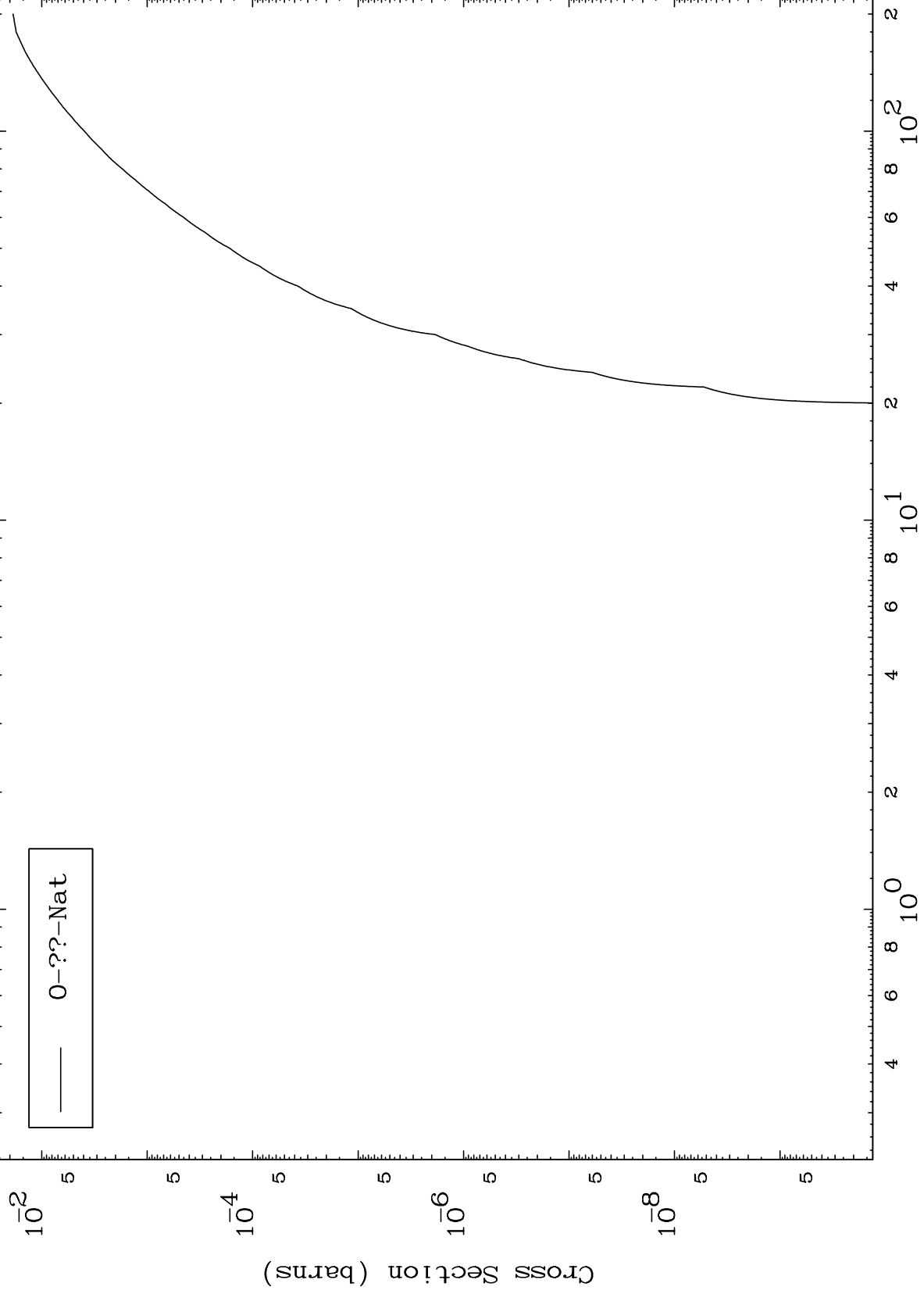
75-Re-190m

MAT 7541

Fission

75-Re-190m

Radionuclide Production Cross Section

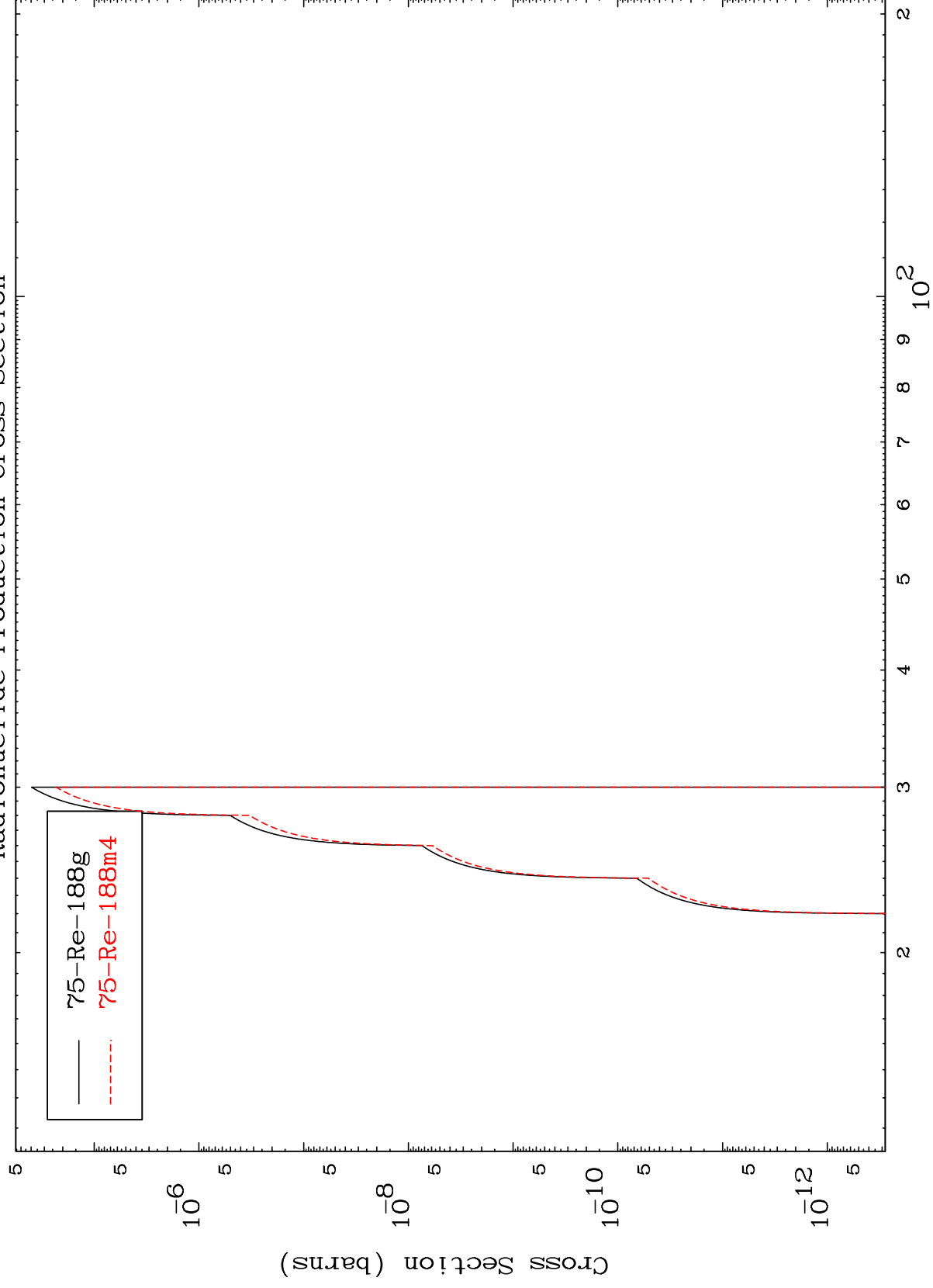


MAT 7541

75-Re-190m

(n,2n)  $\alpha$

Radionuclide Production Cross Section



14

Incident Energy (MeV)

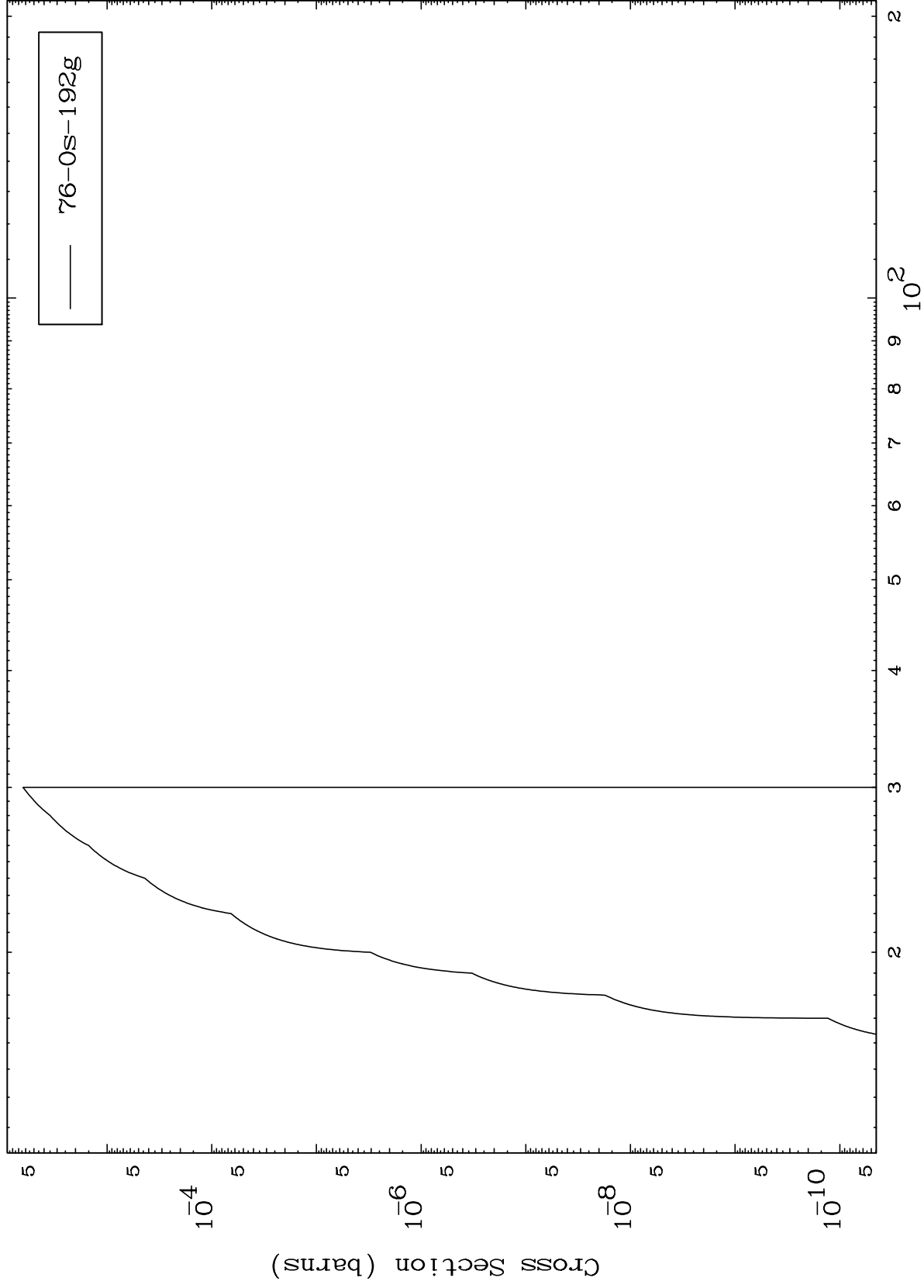
75-Re-190m

MAT 7541

(n,n') p

75-Re-190m

Radionuclide Production Cross Section



15

Incident Energy (MeV)

75-Re-190m

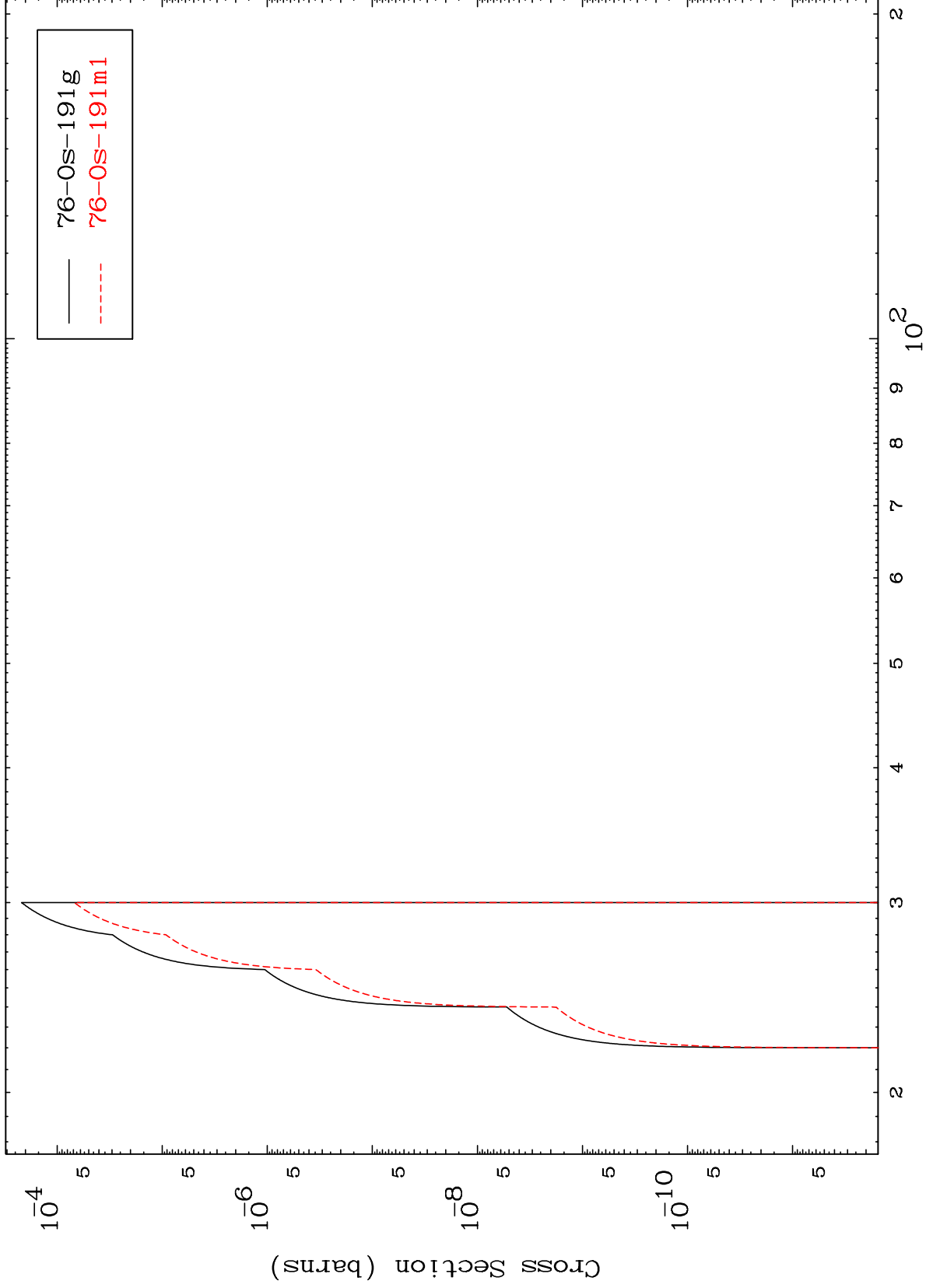


MAT 7541

(n,n') d

75-Re-190m

Radionuclide Production Cross Section



16

Incident Energy (MeV)

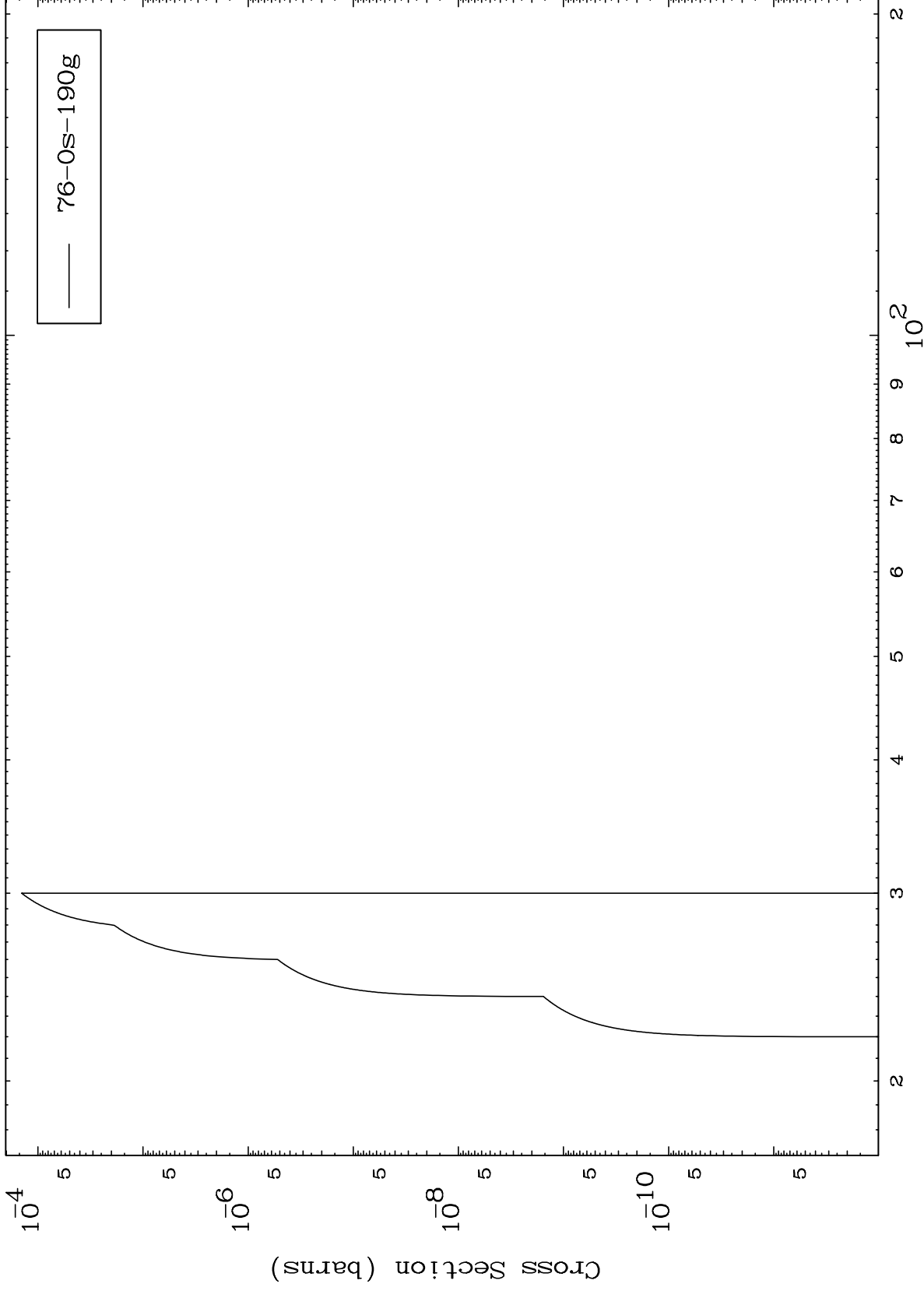
75-Re-190m

MAT 7541

(n,n') t

75-Re-190m

Radionuclide Production Cross Section



17

Incident Energy (MeV)

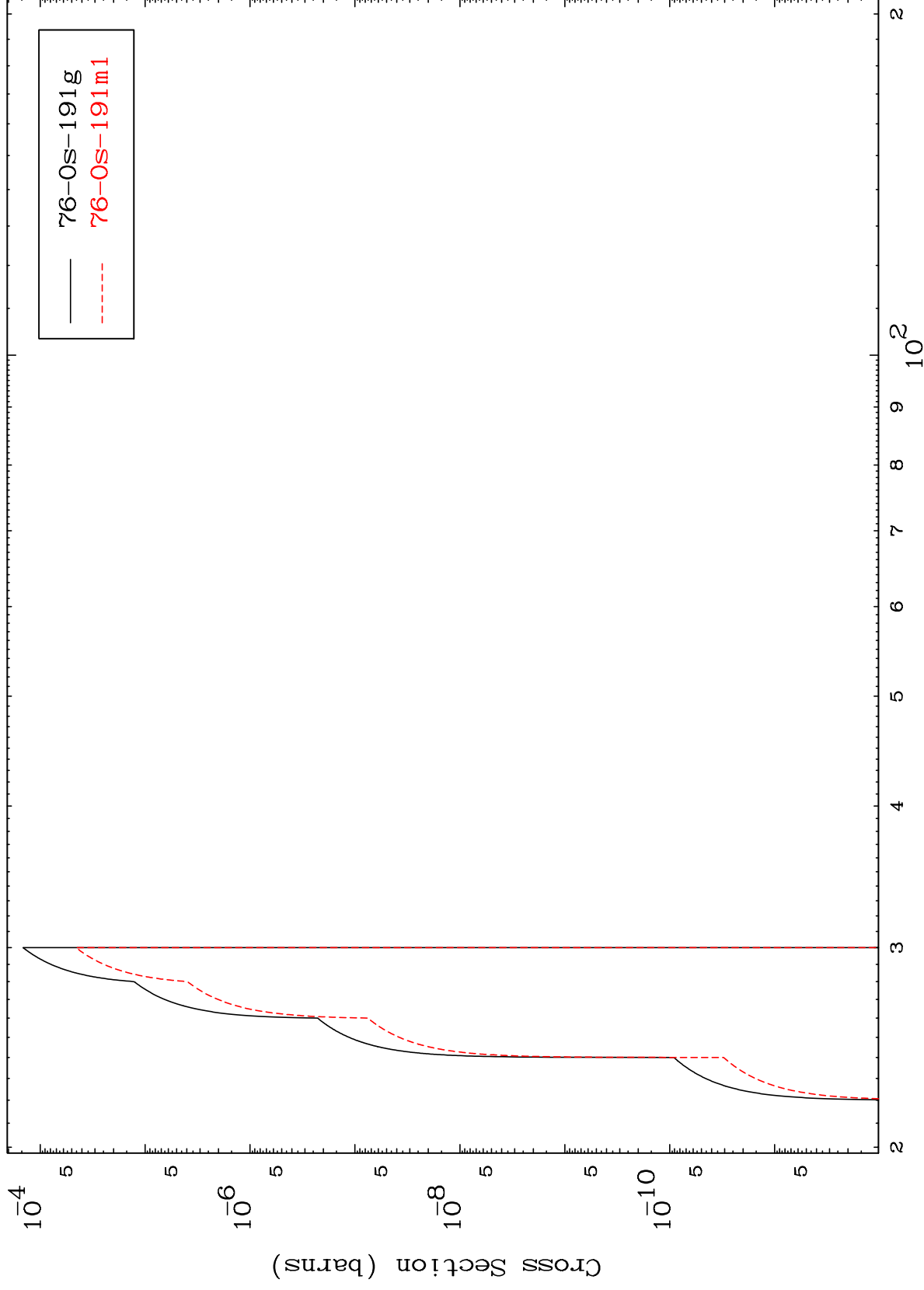
75-Re-190m

MAT 7541

(n,2n) p

75-Re-190m

Radionuclide Production Cross Section



18

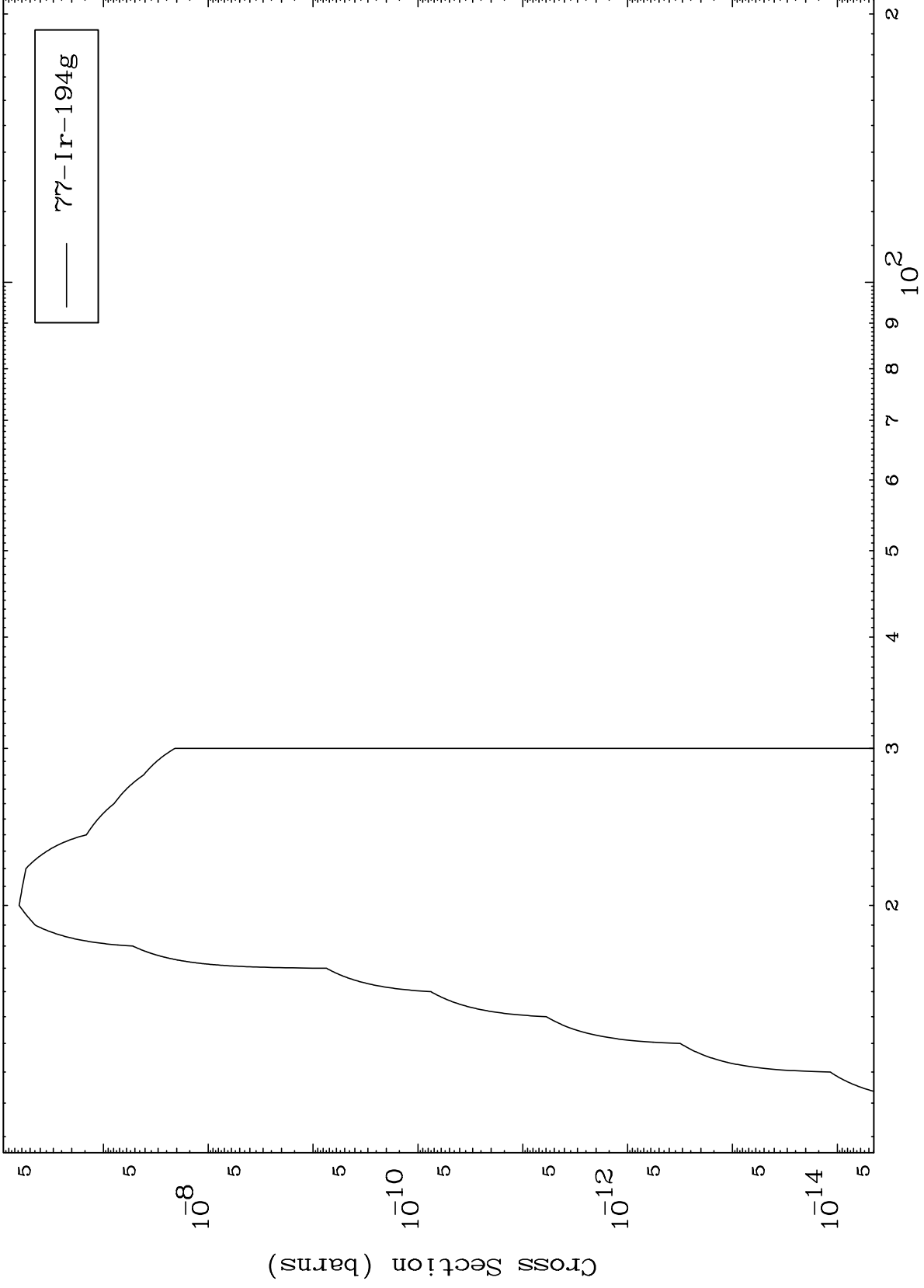
Incident Energy (MeV)

75-Re-190m

MAT 7541

75-Re-190m

(n,  $\gamma$ )  
Radionuclide Production Cross Section



19

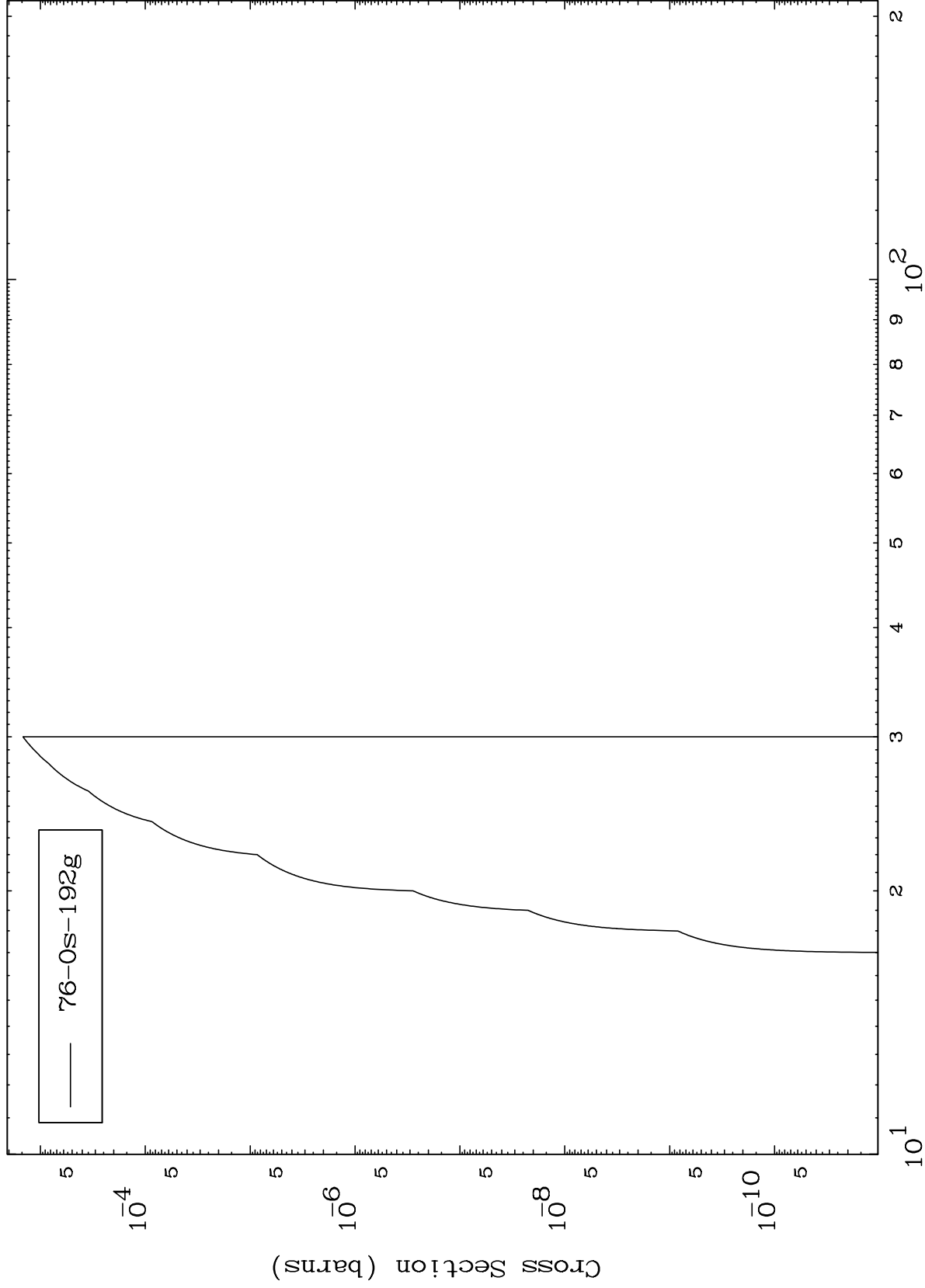
Incident Energy (MeV)

75-Re-190m

MAT 7541

75-Re-190m

(n,d)  
Radionuclide Production Cross Section



75-Re-190m

Incident Energy (MeV)

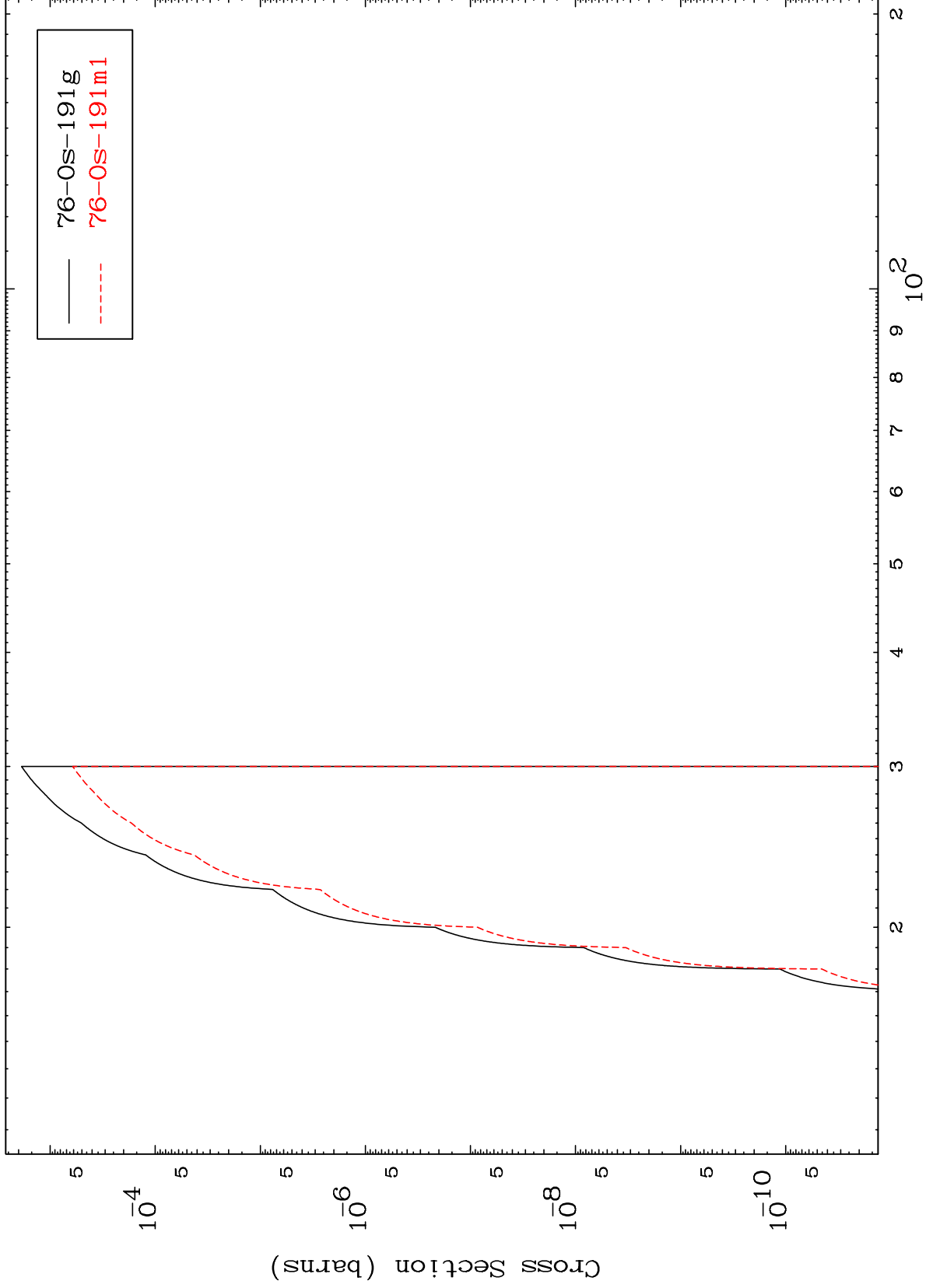
20

MAT 7541

(n, t)

75-Re-190m

Radionuclide Production Cross Section



21

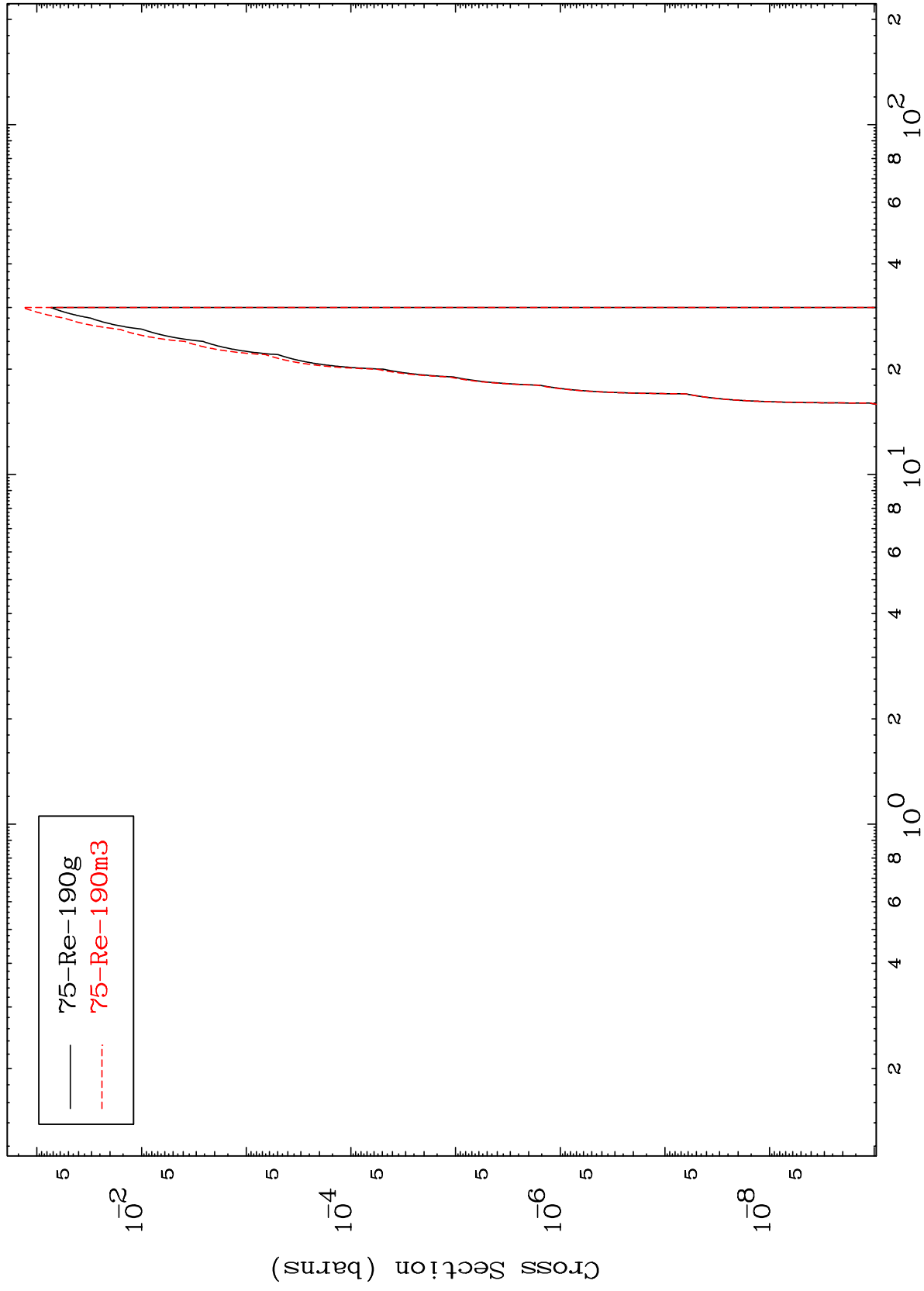
Incident Energy (MeV)

75-Re-190m

MAT 7541

<sup>75</sup>Re-190m

Radionuclide Production Cross Section



22

<sup>75</sup>Re-190m

Incident Energy (MeV)