

Program EVALPLOT
(Version 2021-1)

by

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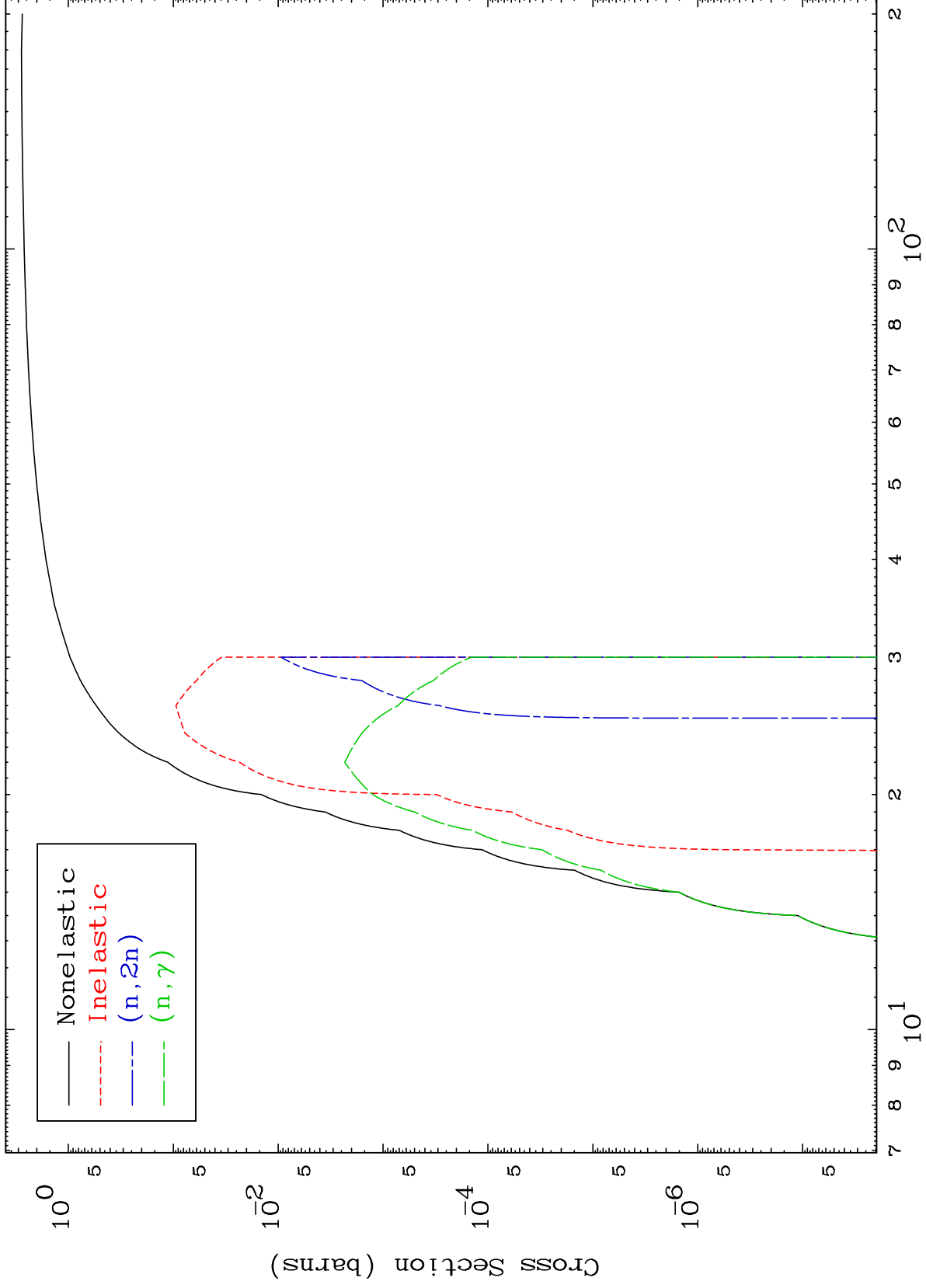
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8610

0 Kelvin Cross Sections

86-Rn-206



1

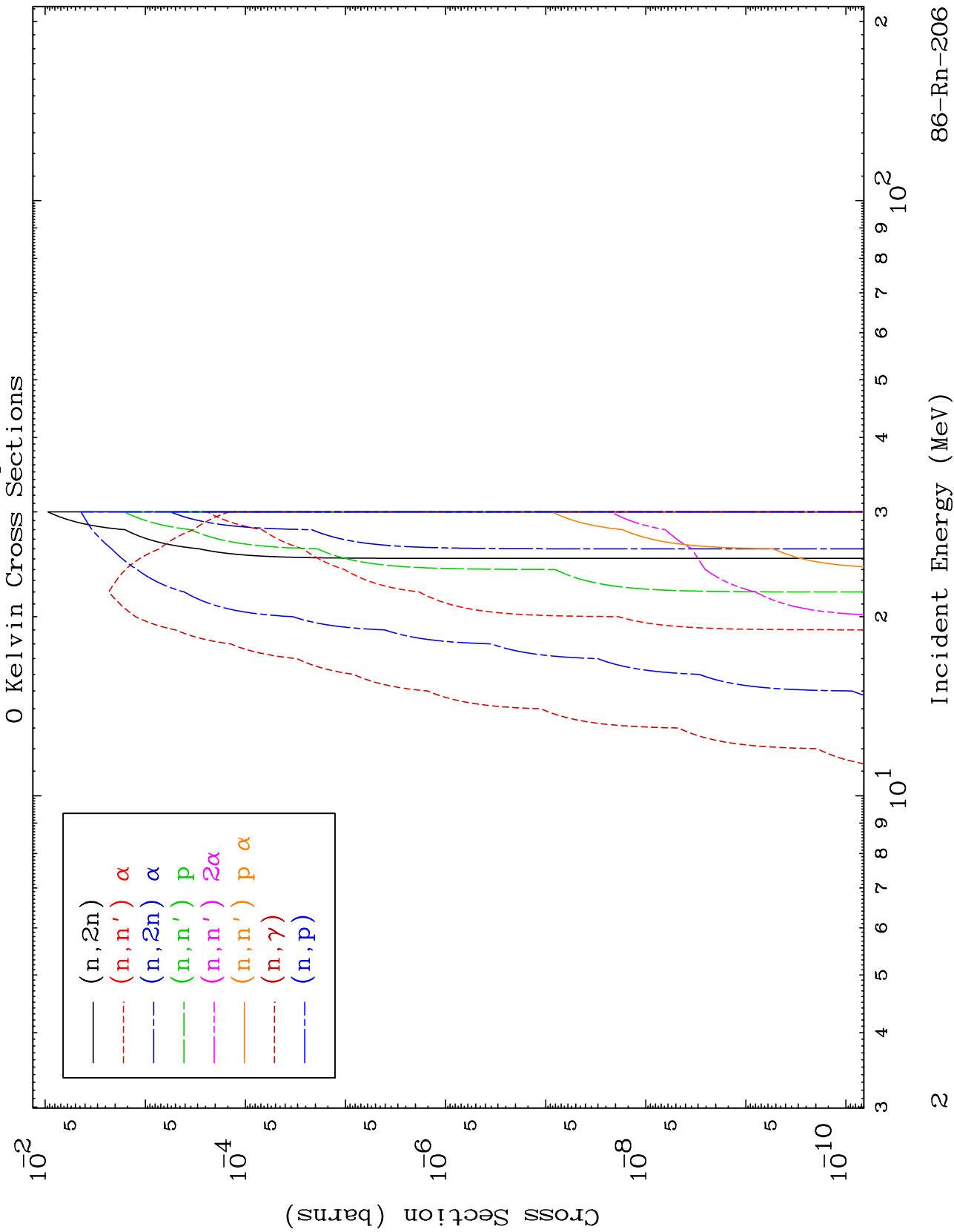
Incident Energy (MeV)

86-Rn-206

MAT 8610

α Neutron Absorption

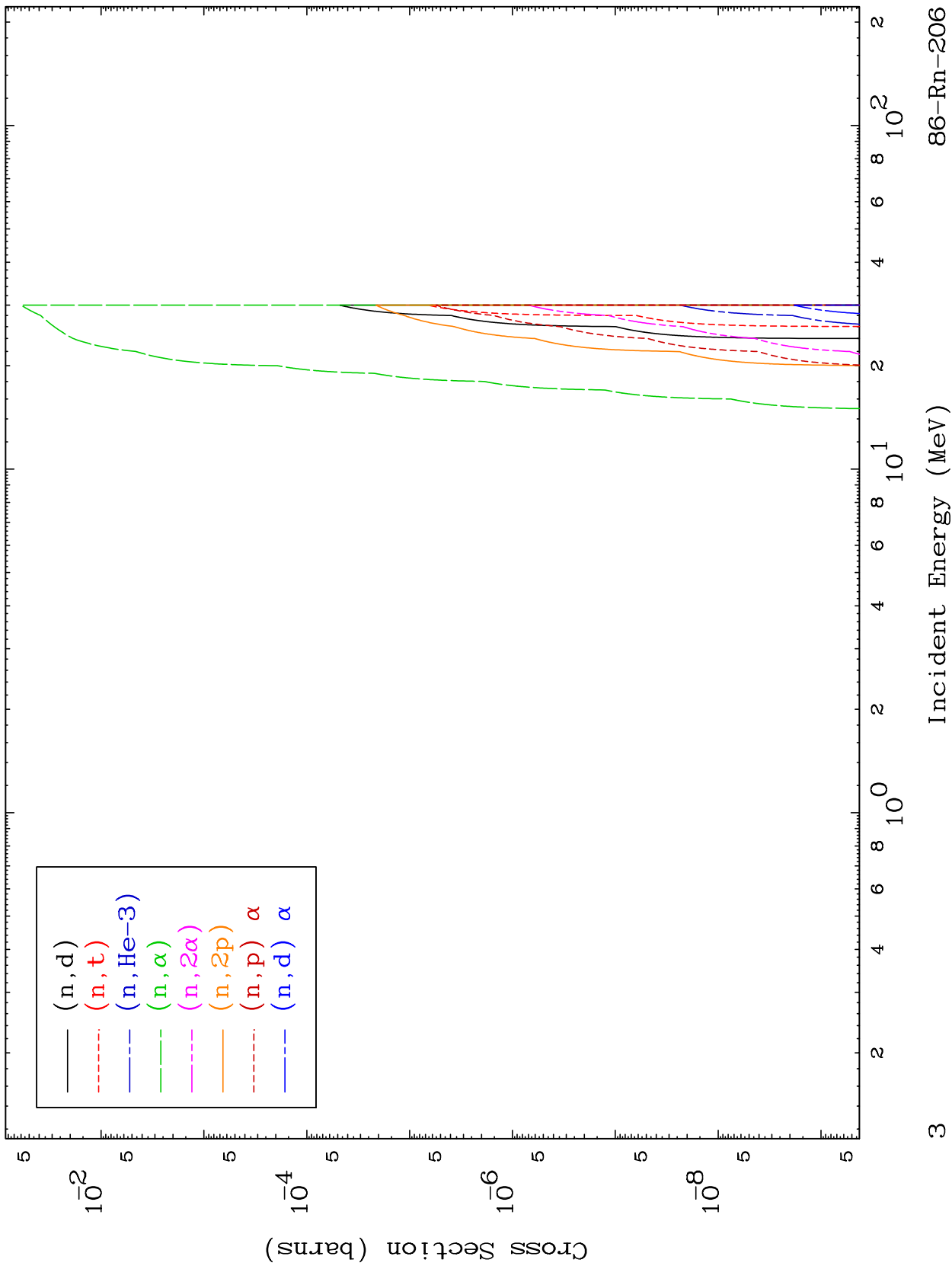
86-Rn-206

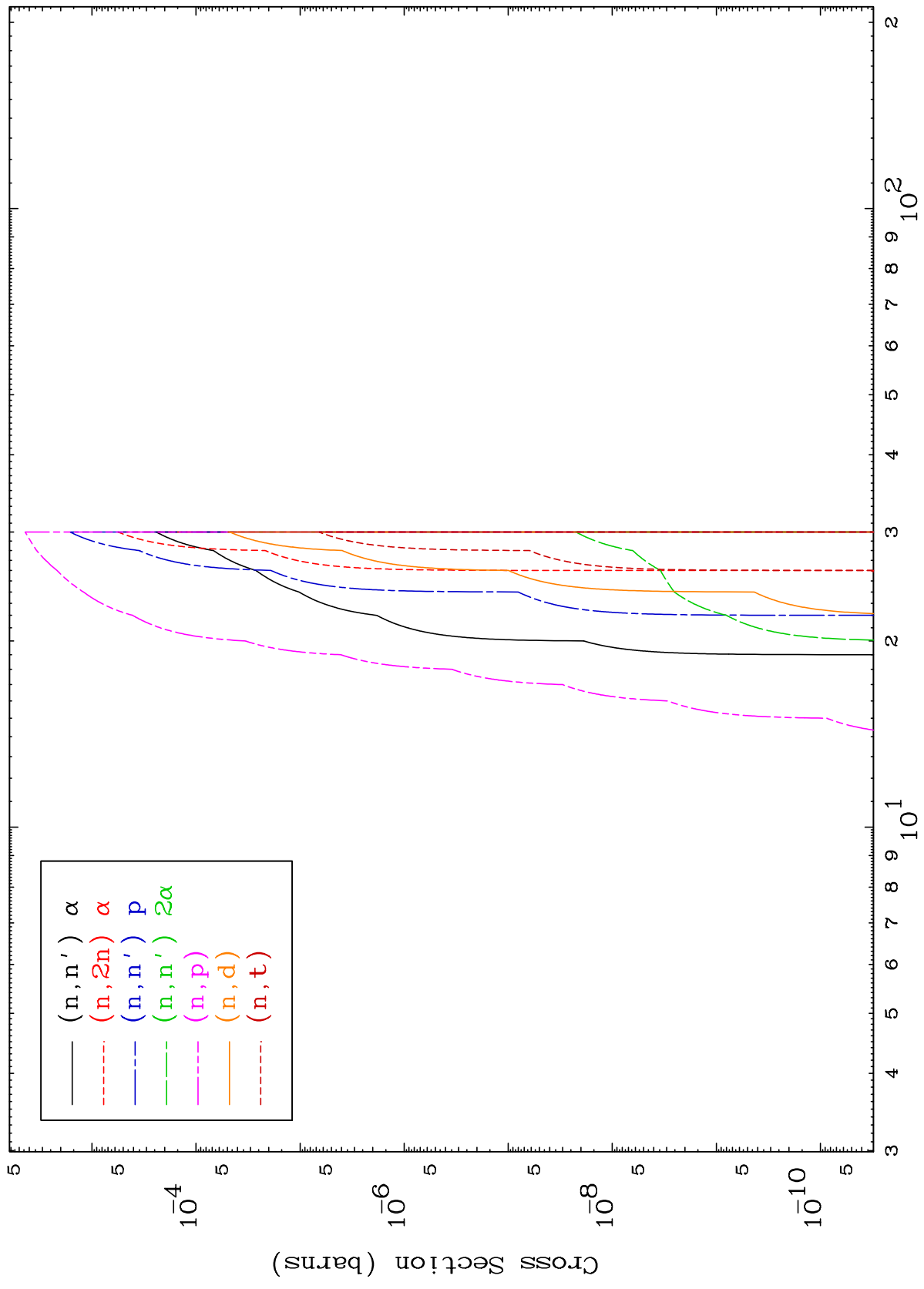


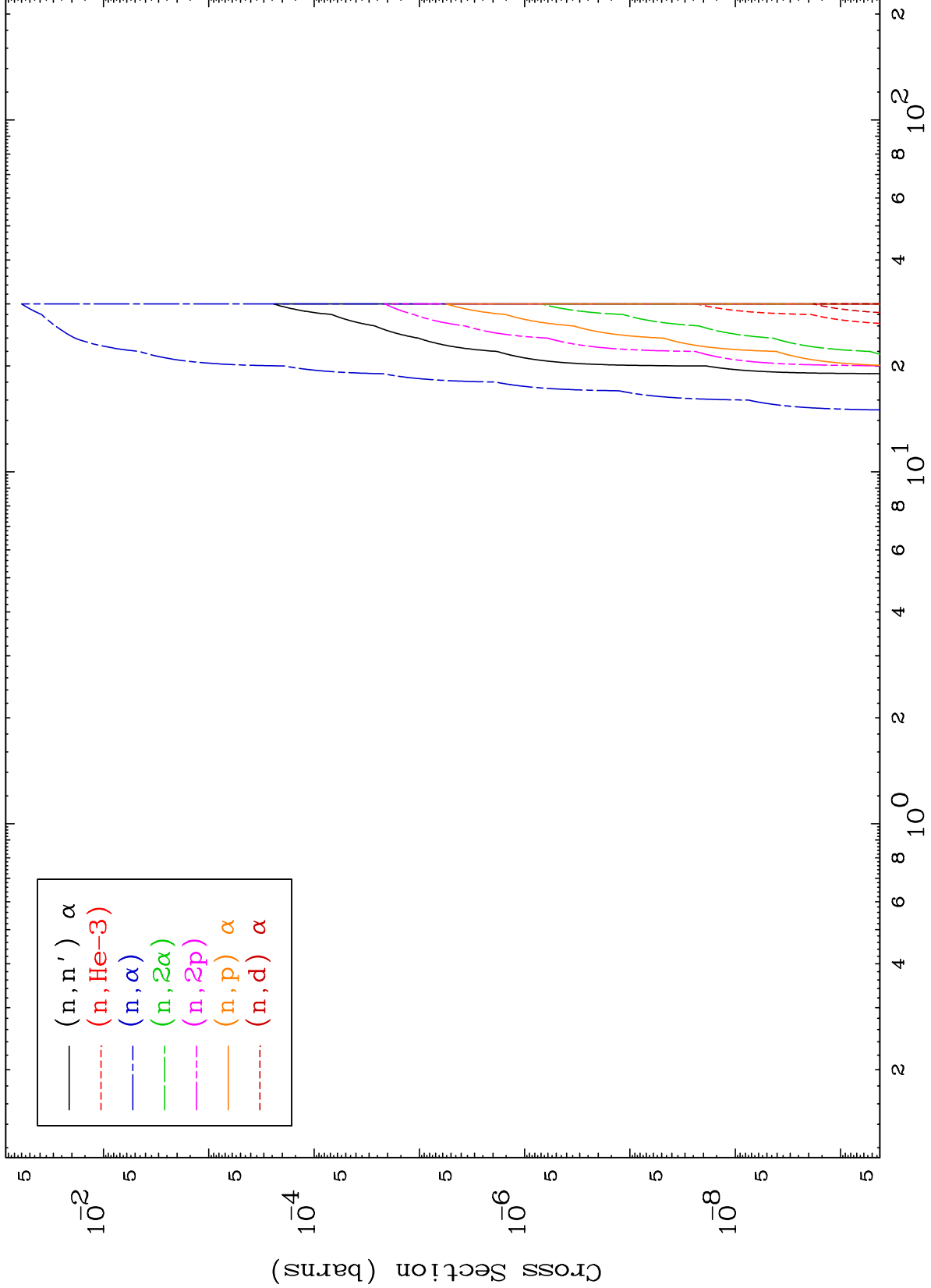
MAT 8610

α Neutron Absorption
0 Kelvin Cross Sections

86-Rn-206



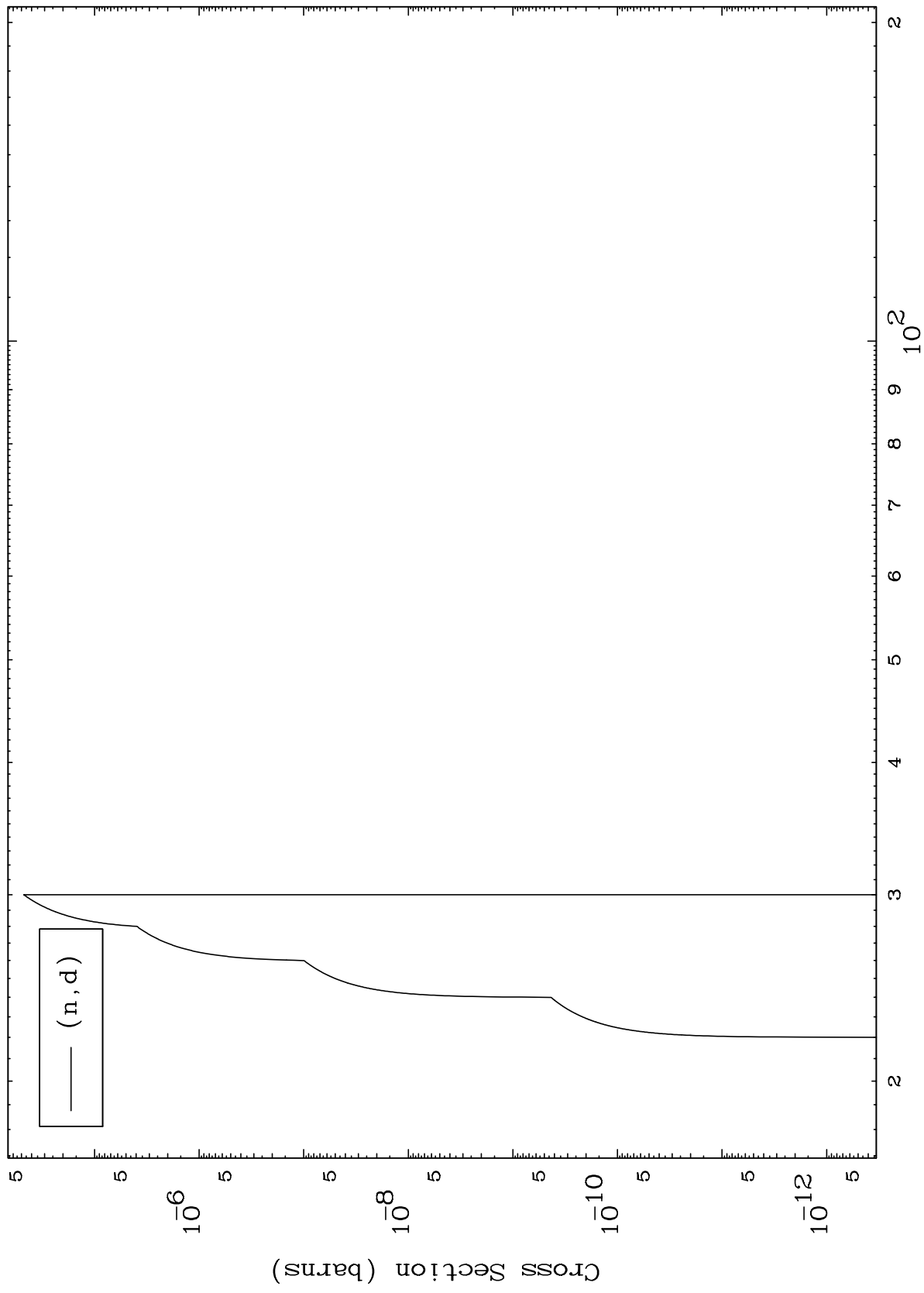




MAT 8610

(α, d) Levels
0 Kelvin Cross Sections

86-Rn-206



7

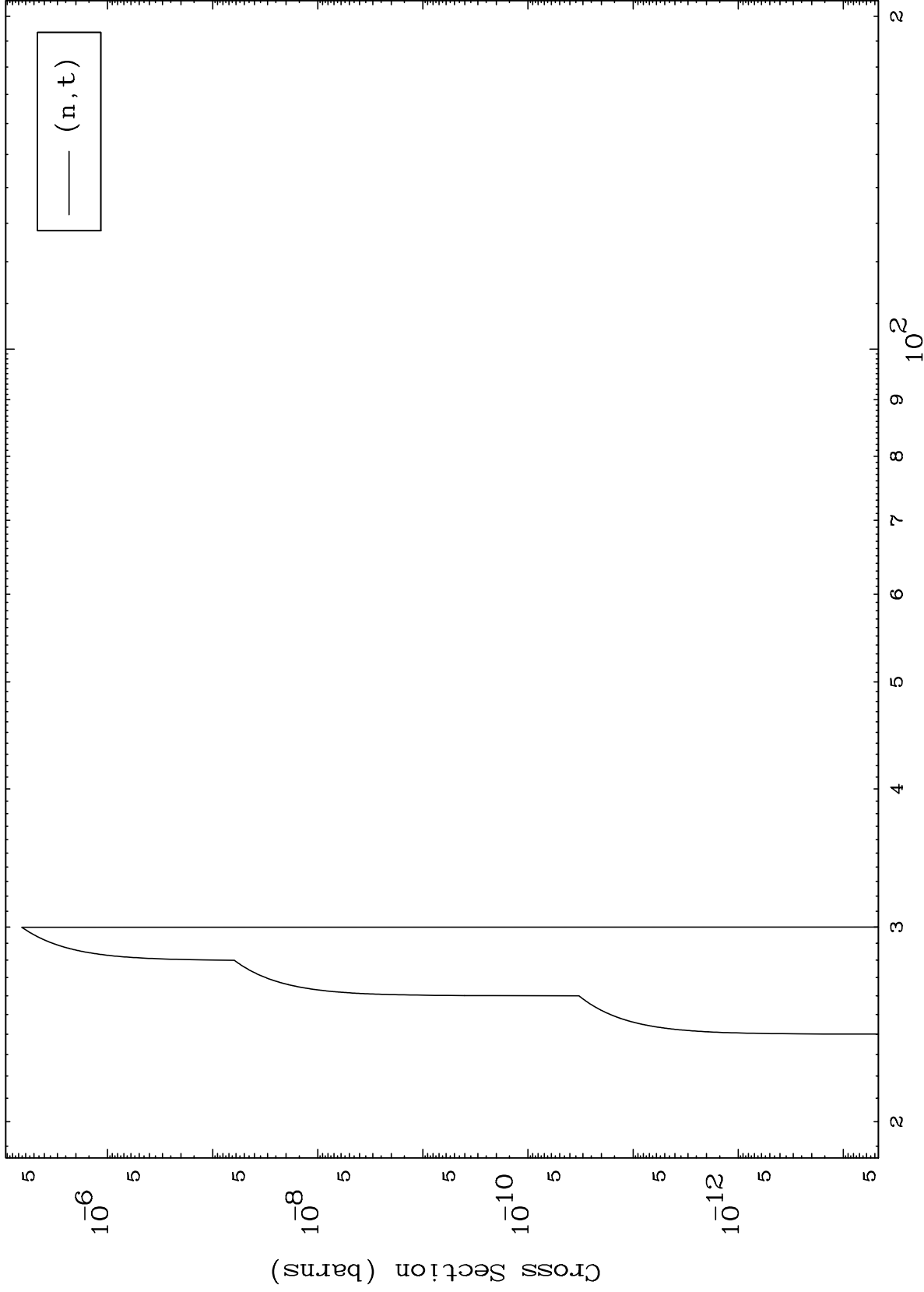
Incident Energy (MeV)

86-Rn-206

MAT 8610

(α, t) Levels
0 Kelvin Cross Sections

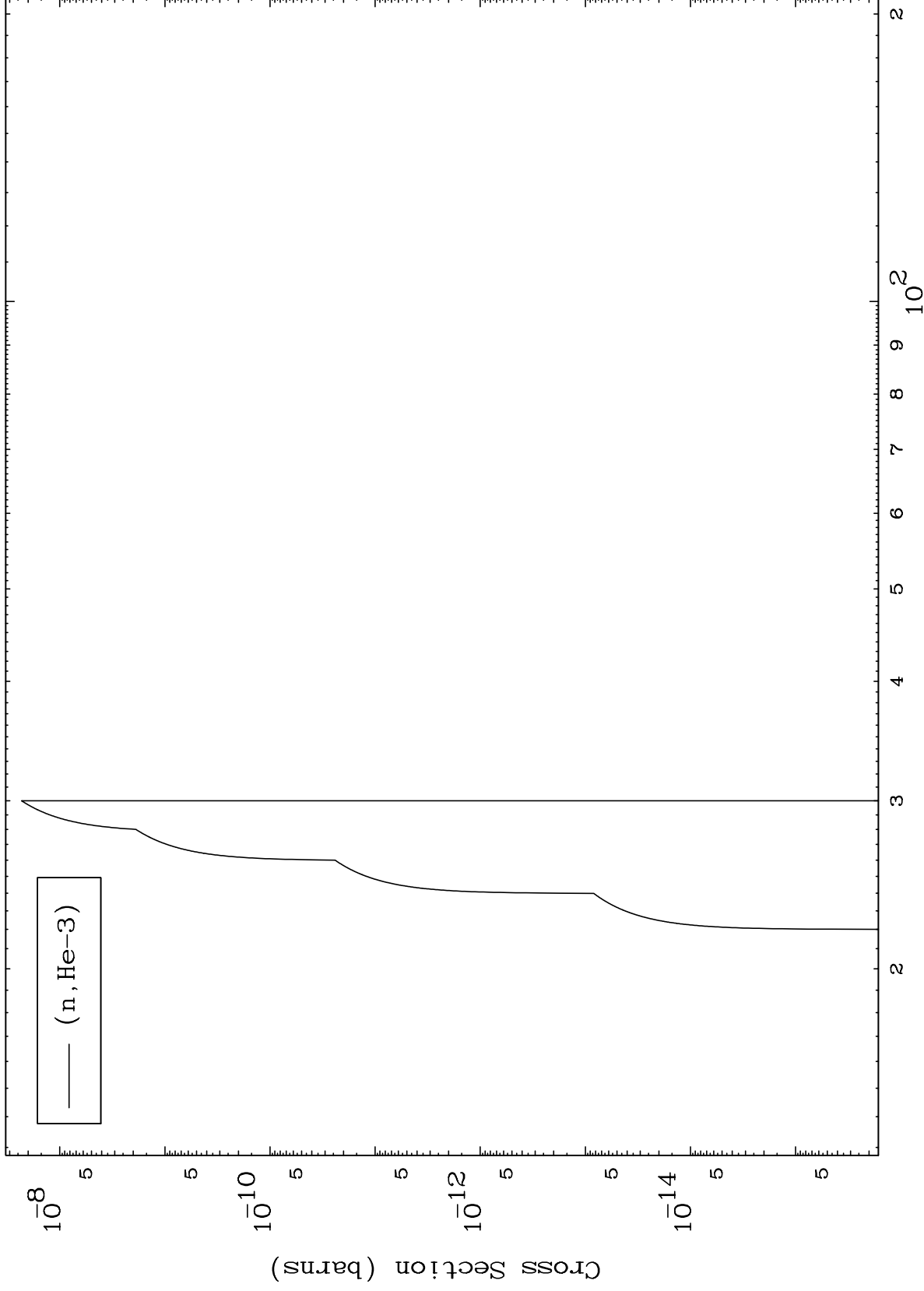
86-Rn-206



8

Incident Energy (MeV)

86-Rn-206

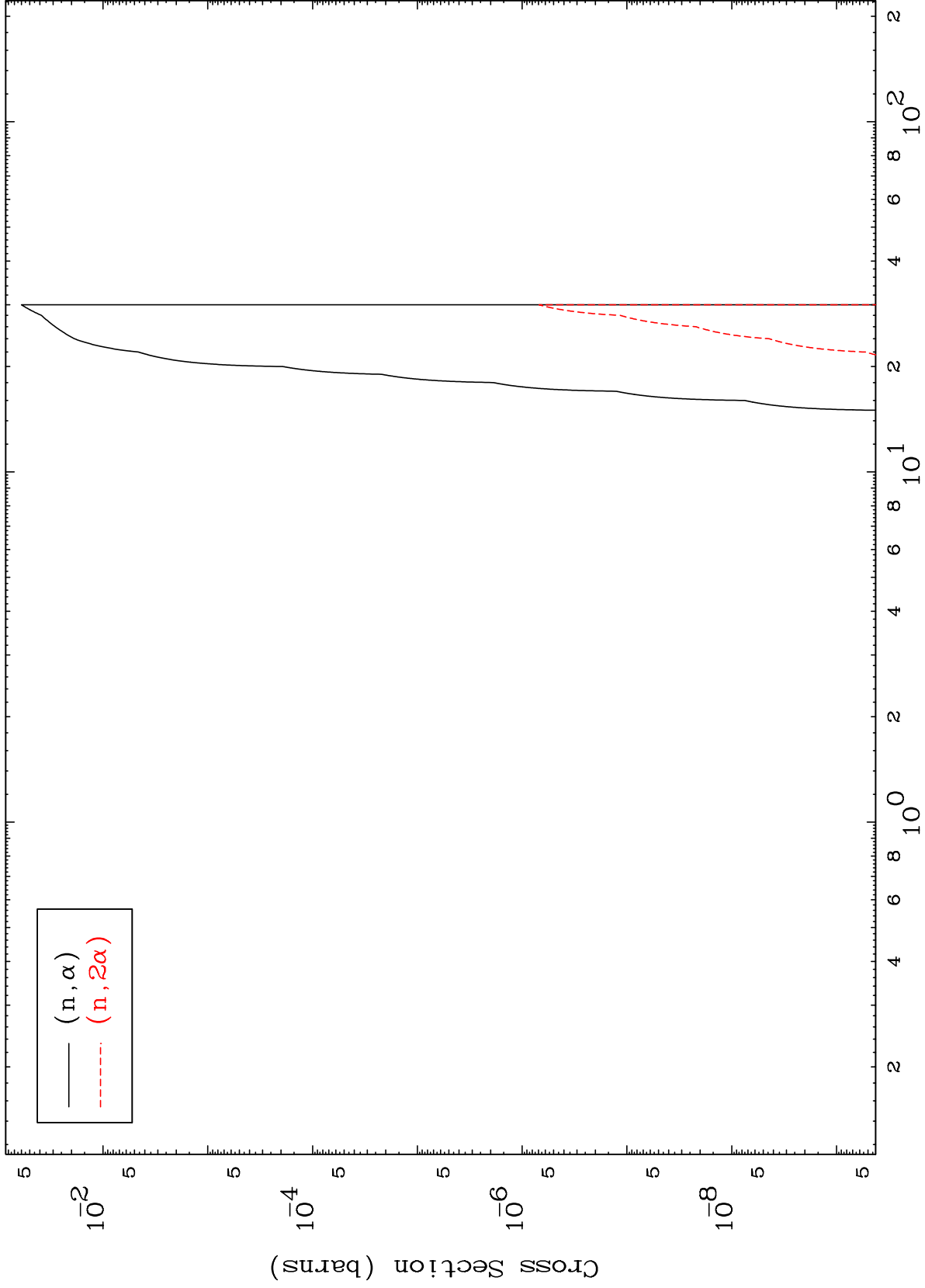


MAT 8610

(α, α) Levels

86-Rn-206

0 Kelvin Cross Sections



10

Incident Energy (MeV)

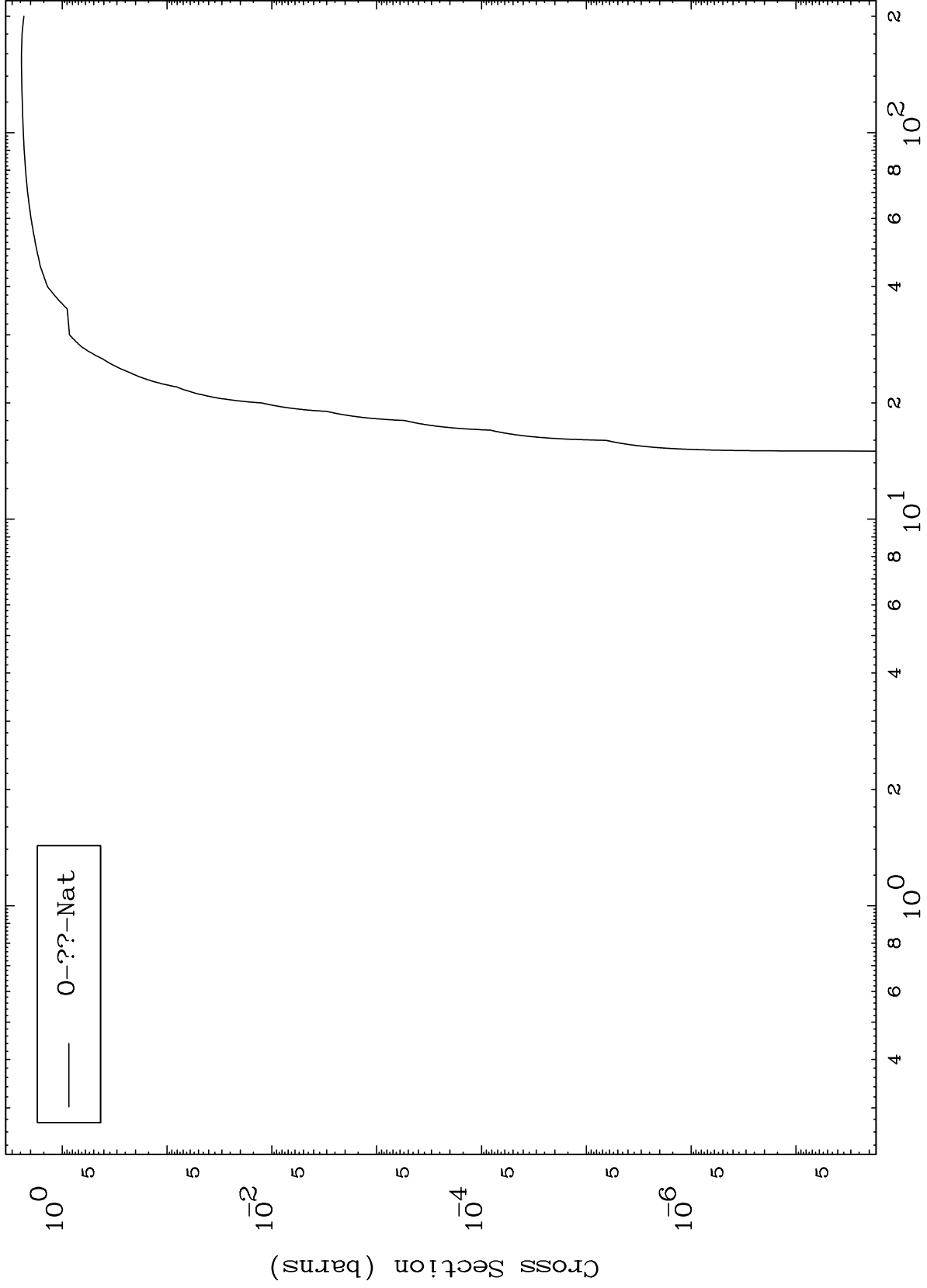
86-Rn-206

MAT 8610

Fission

86-Rn-206

Radionuclide Production Cross Section

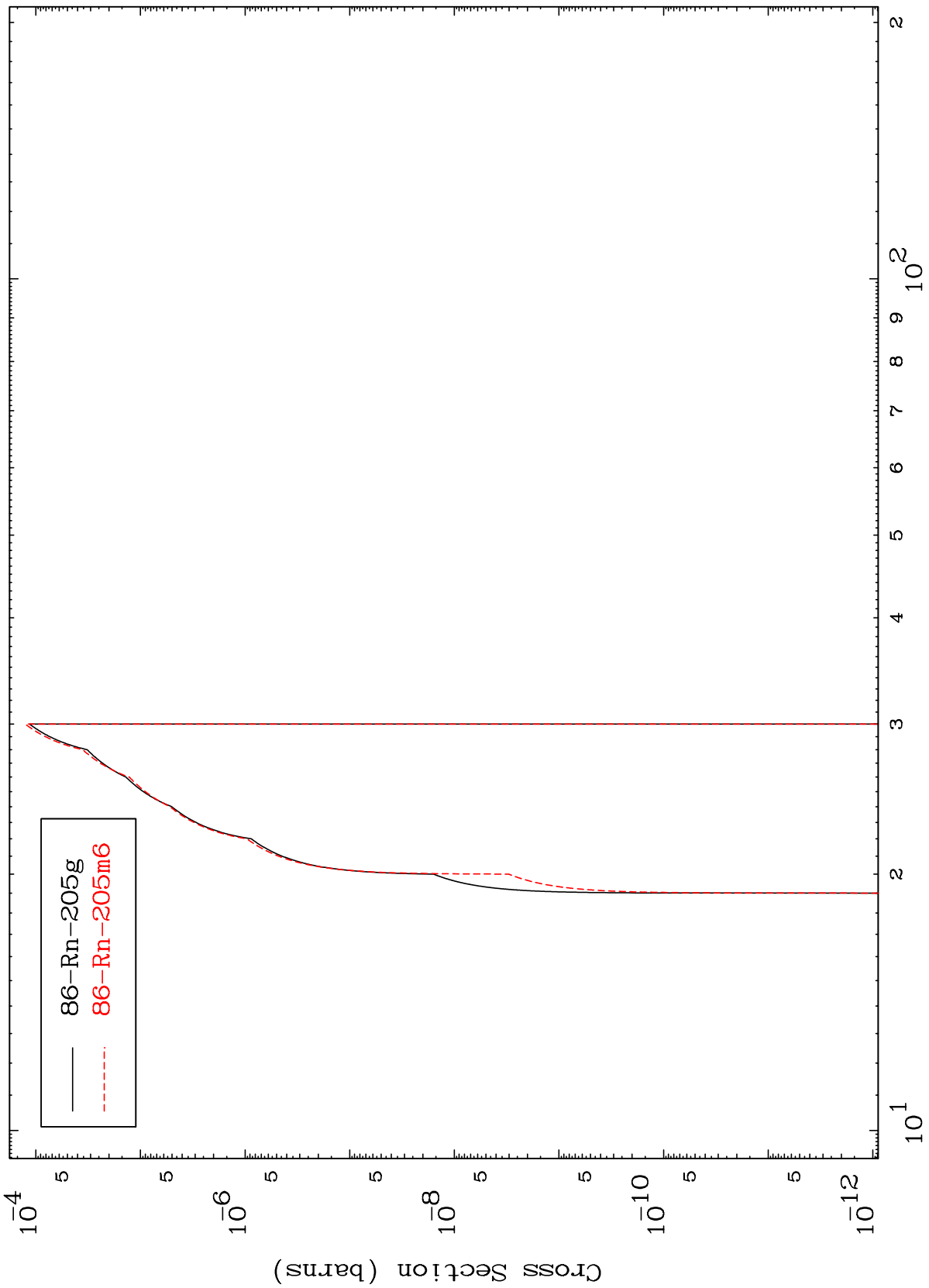


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(n,n') α

86-Rn-206

Radionuclide Production Cross Section



Incident Energy (MeV)

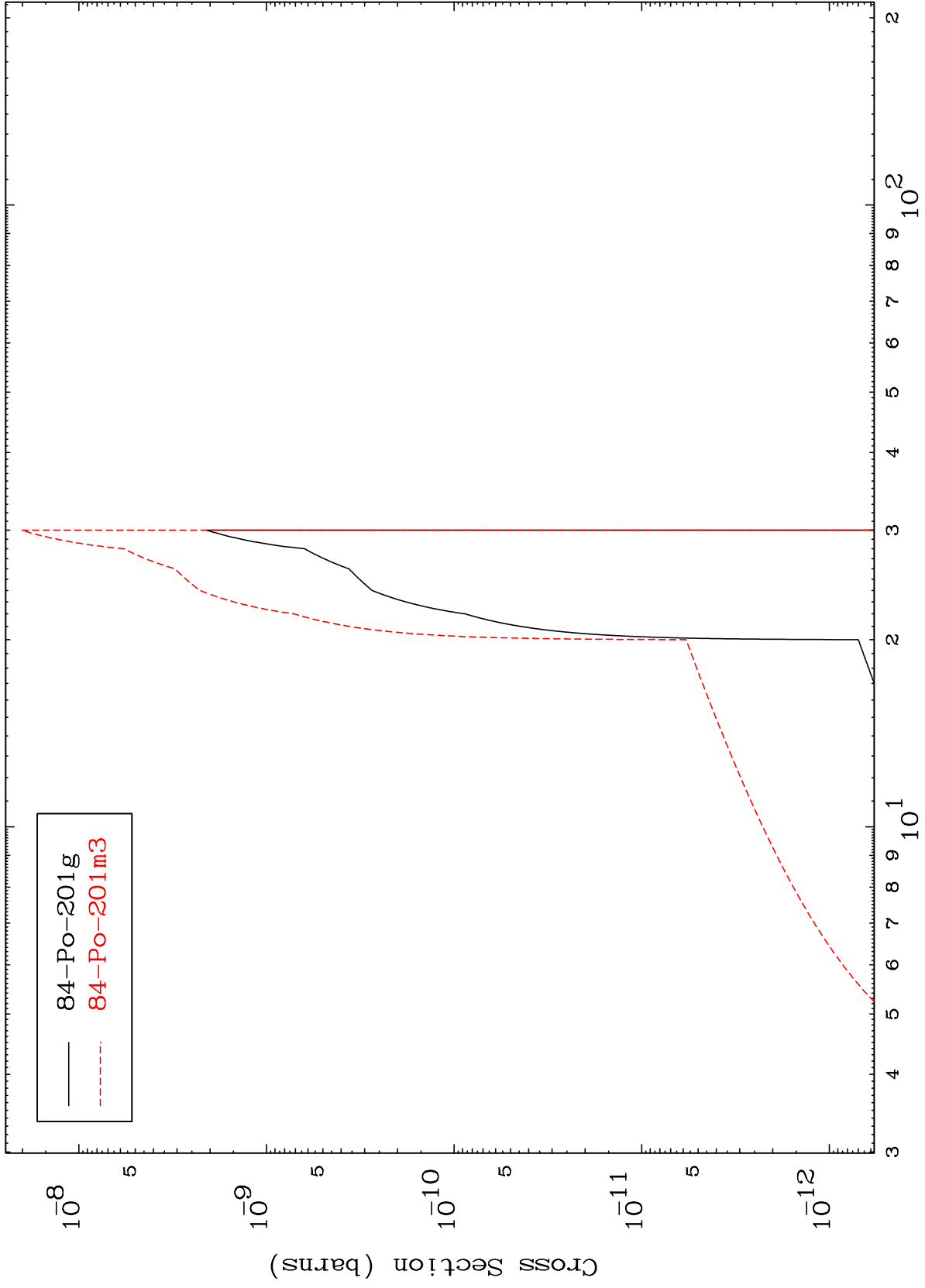
86-Rn-206

MAT 8610

(n,n') 2α

86-Rn-206

Radionuclide Production Cross Section



13

Incident Energy (MeV)

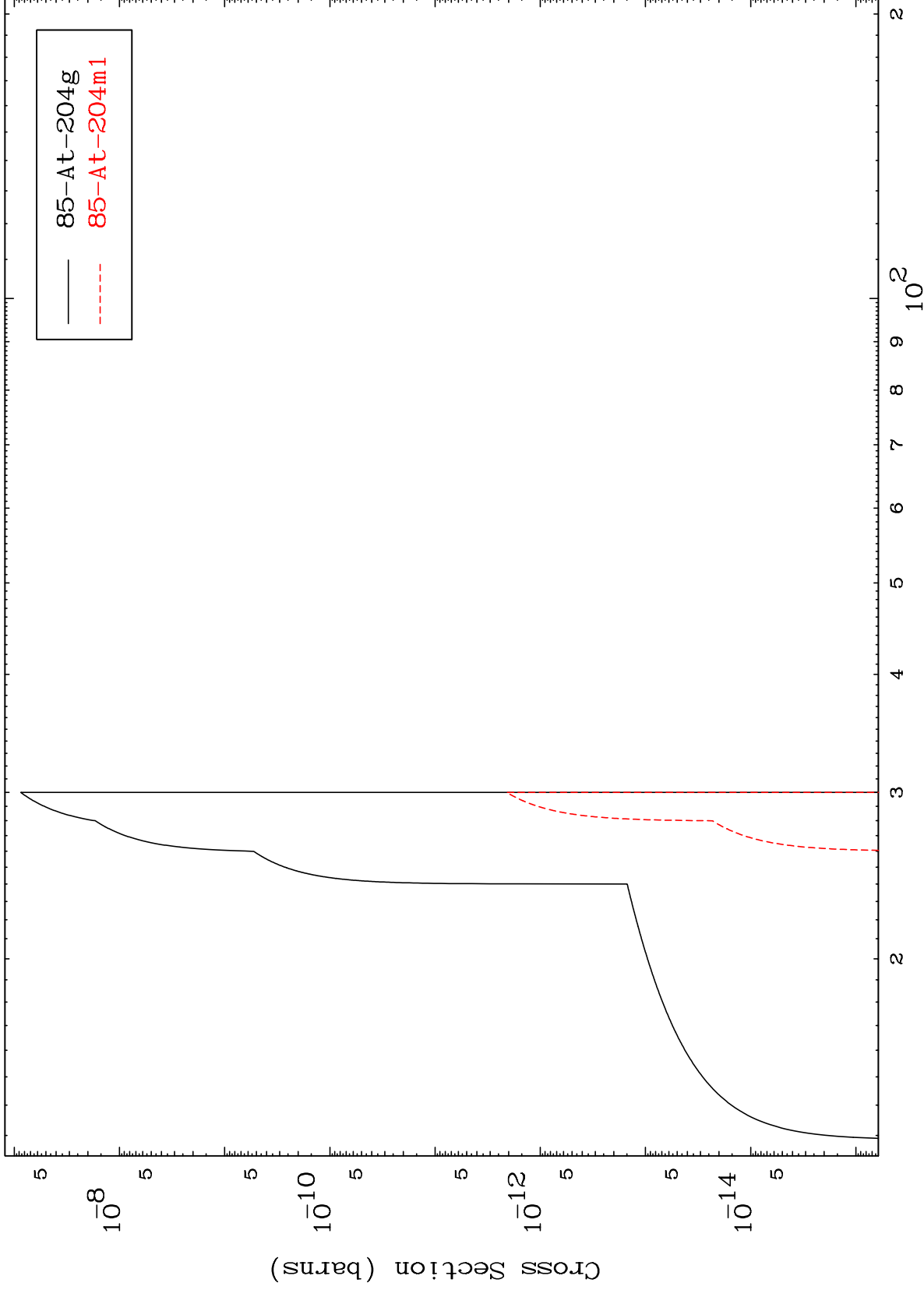
86-Rn-206

MAT 8610

(n,n') p α

86-Rn-206

Radionuclide Production Cross Section



14

Incident Energy (MeV)

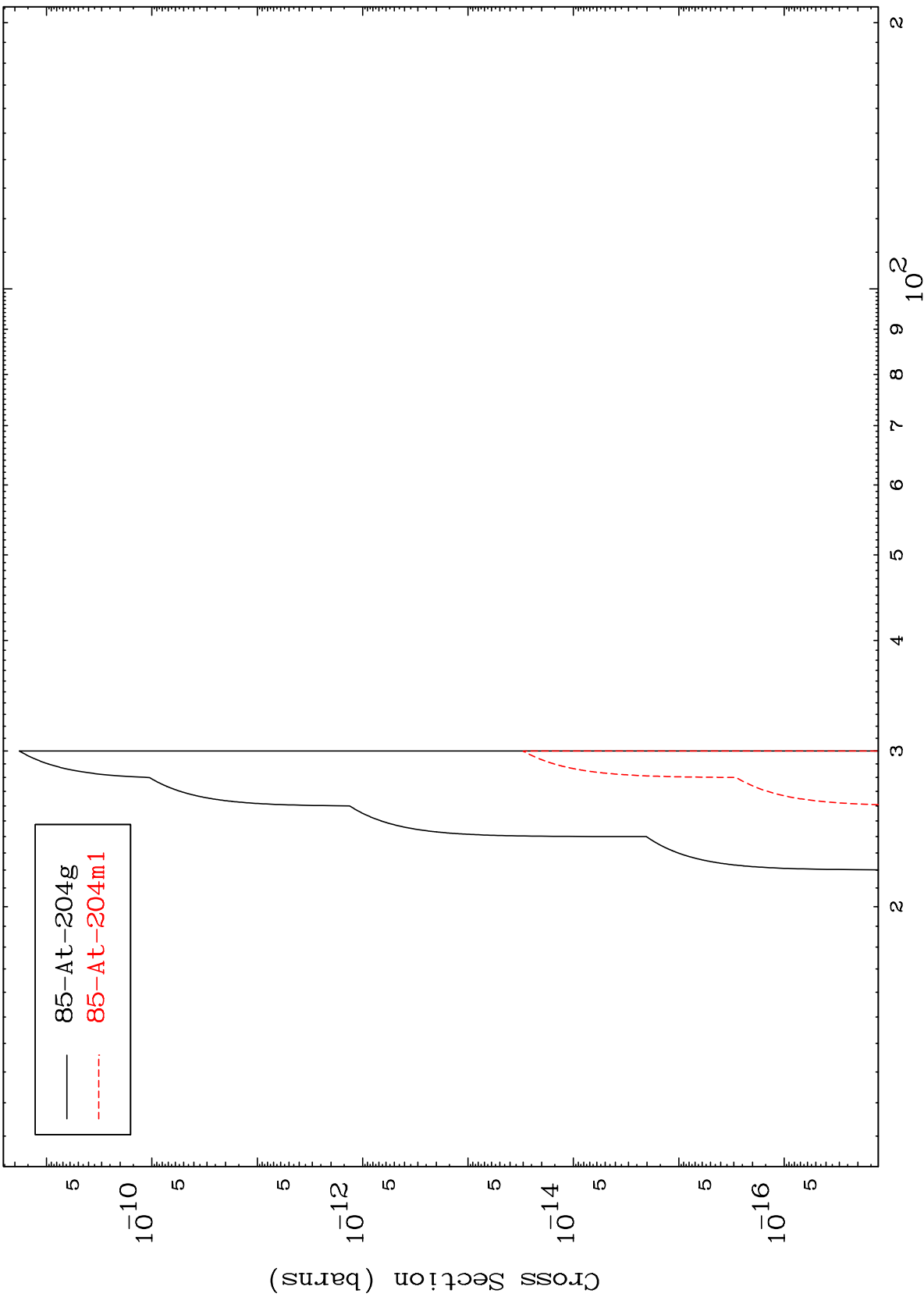
86-Rn-206

MAT 8610

(n,d) α

86-Rn-206

Radionuclide Production Cross Section



15

Incident Energy (MeV)

86-Rn-206