

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

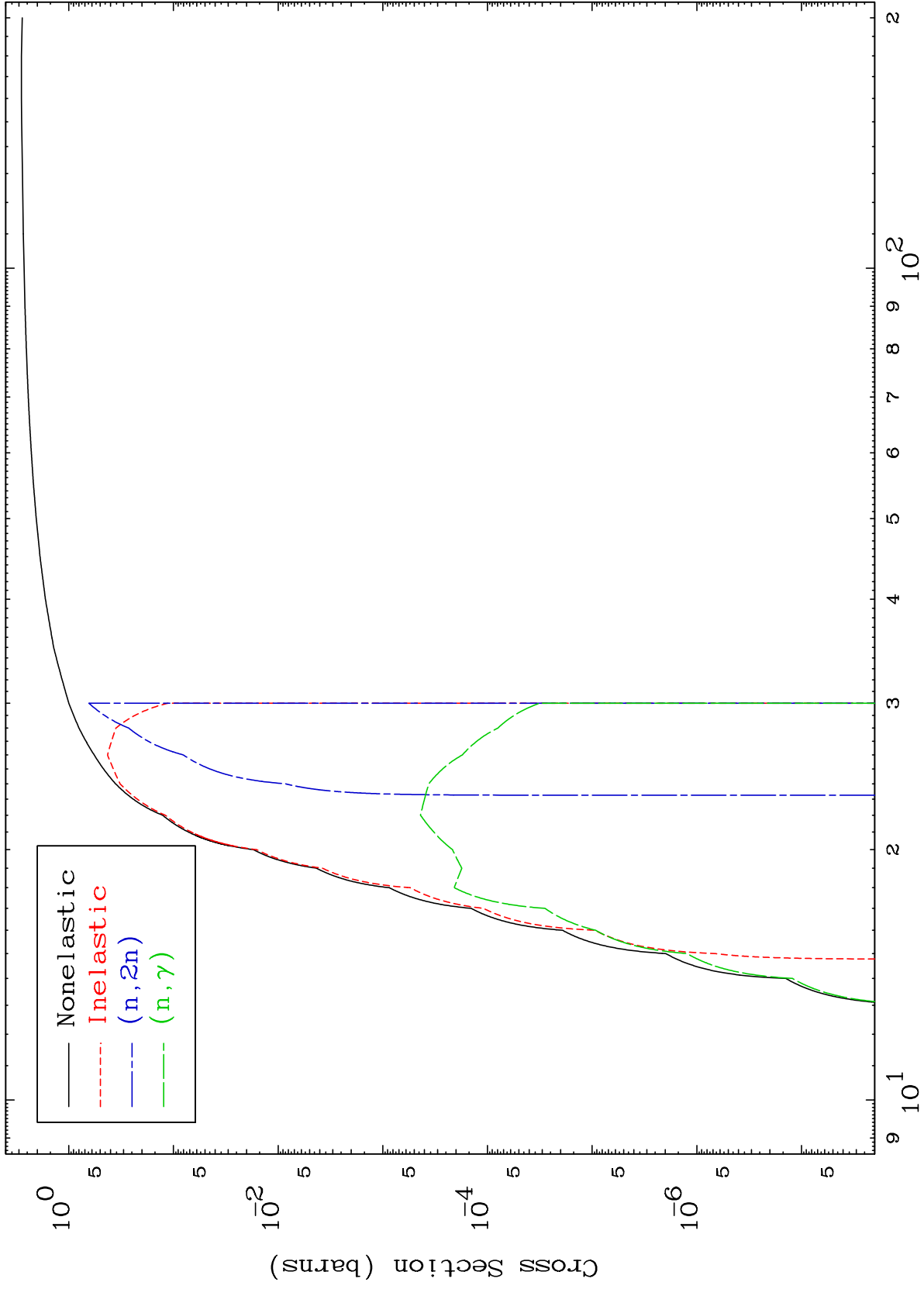
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8625

0 Kelvin Cross Sections

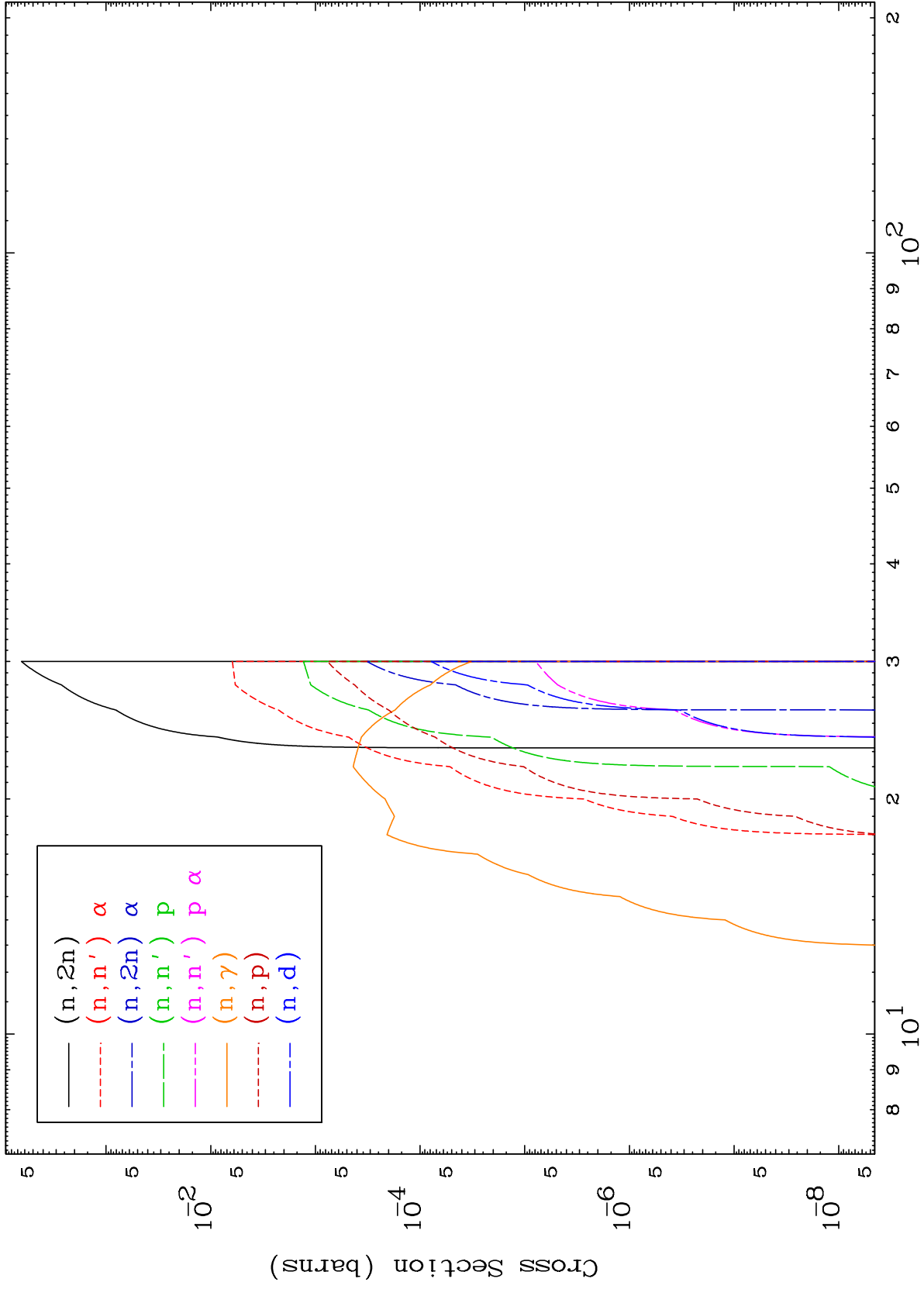
86-Rn-211



86-Rn-211

Incident Energy (MeV)

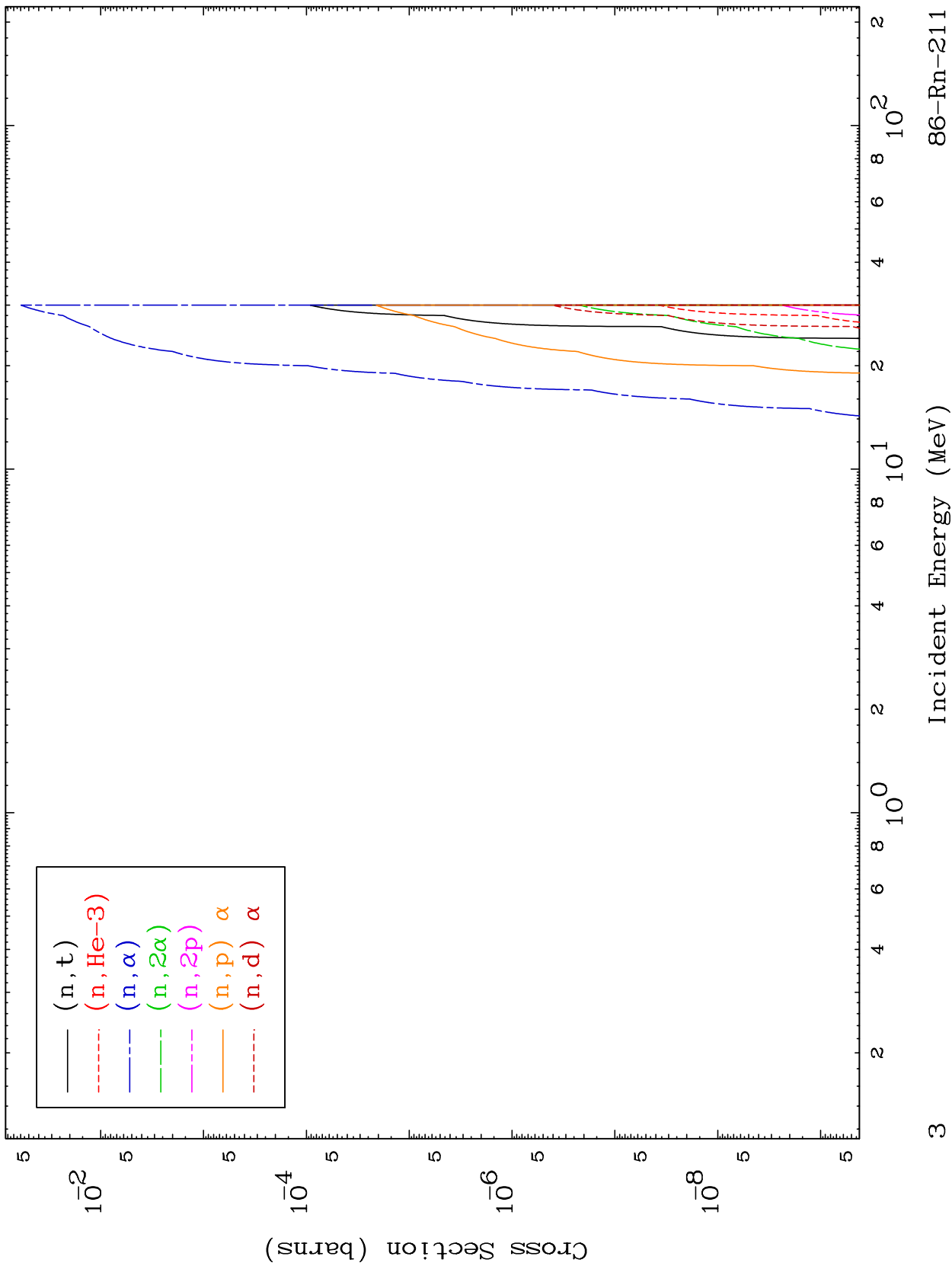
1

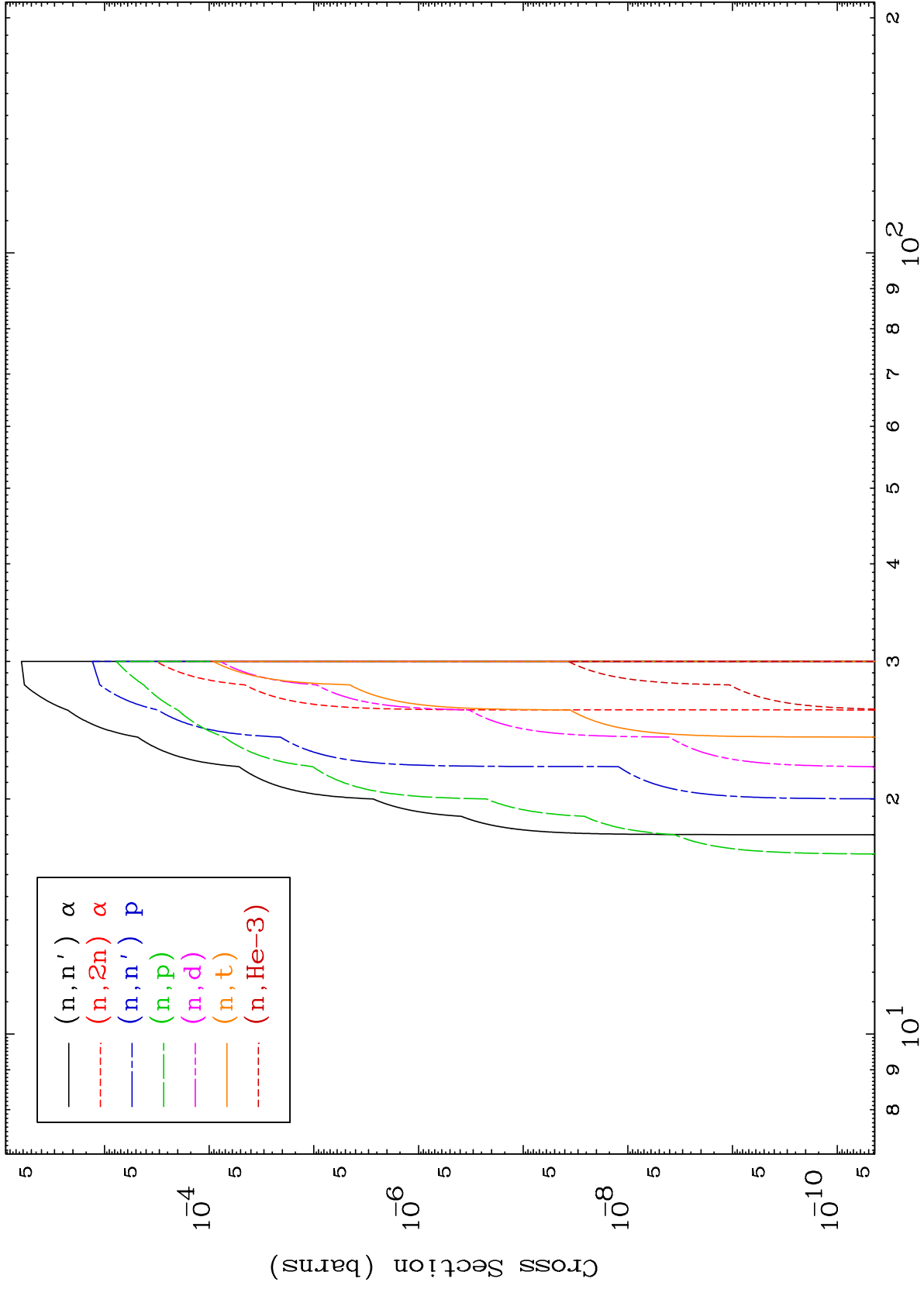


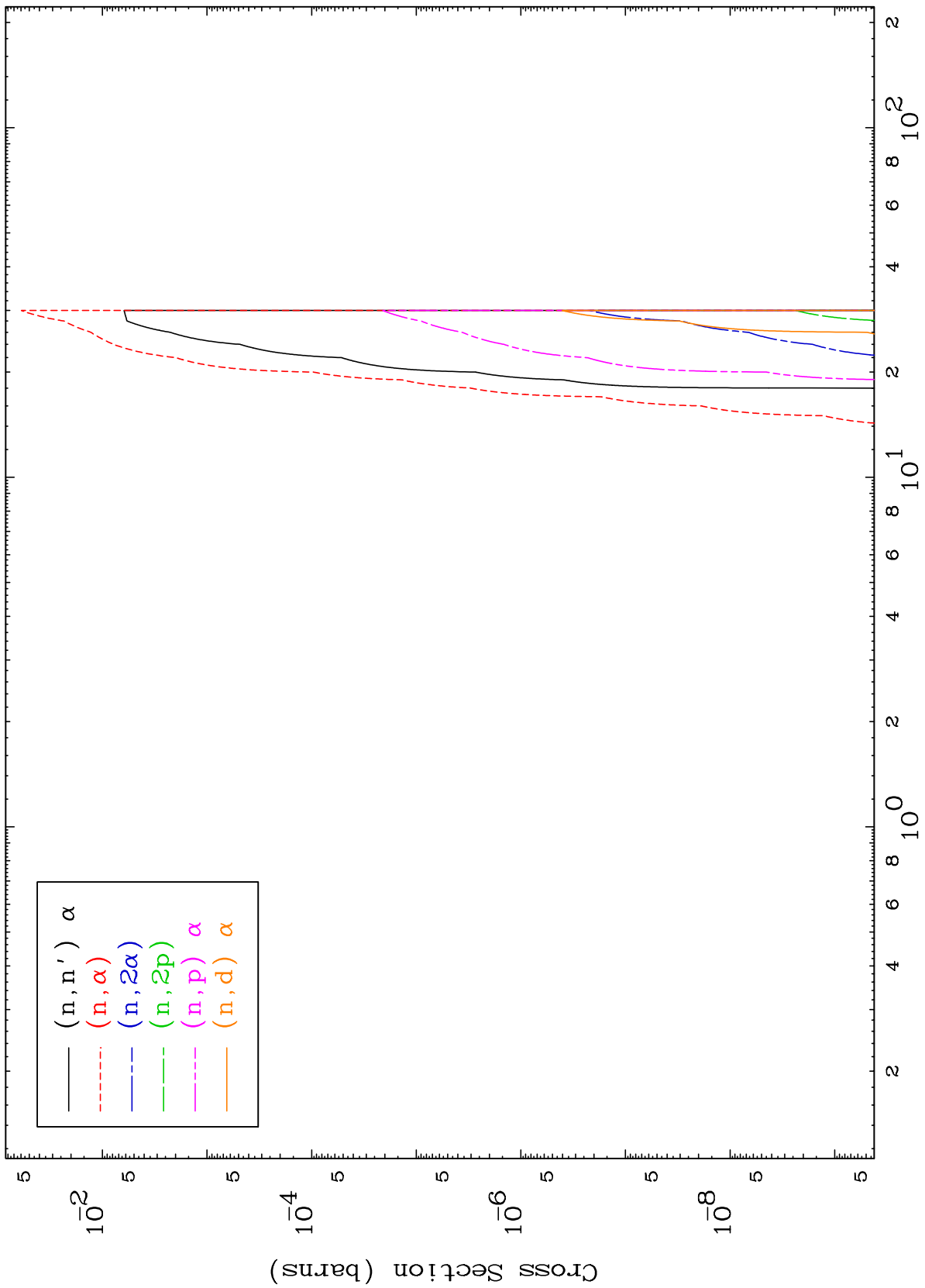
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$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

86-Rn-211



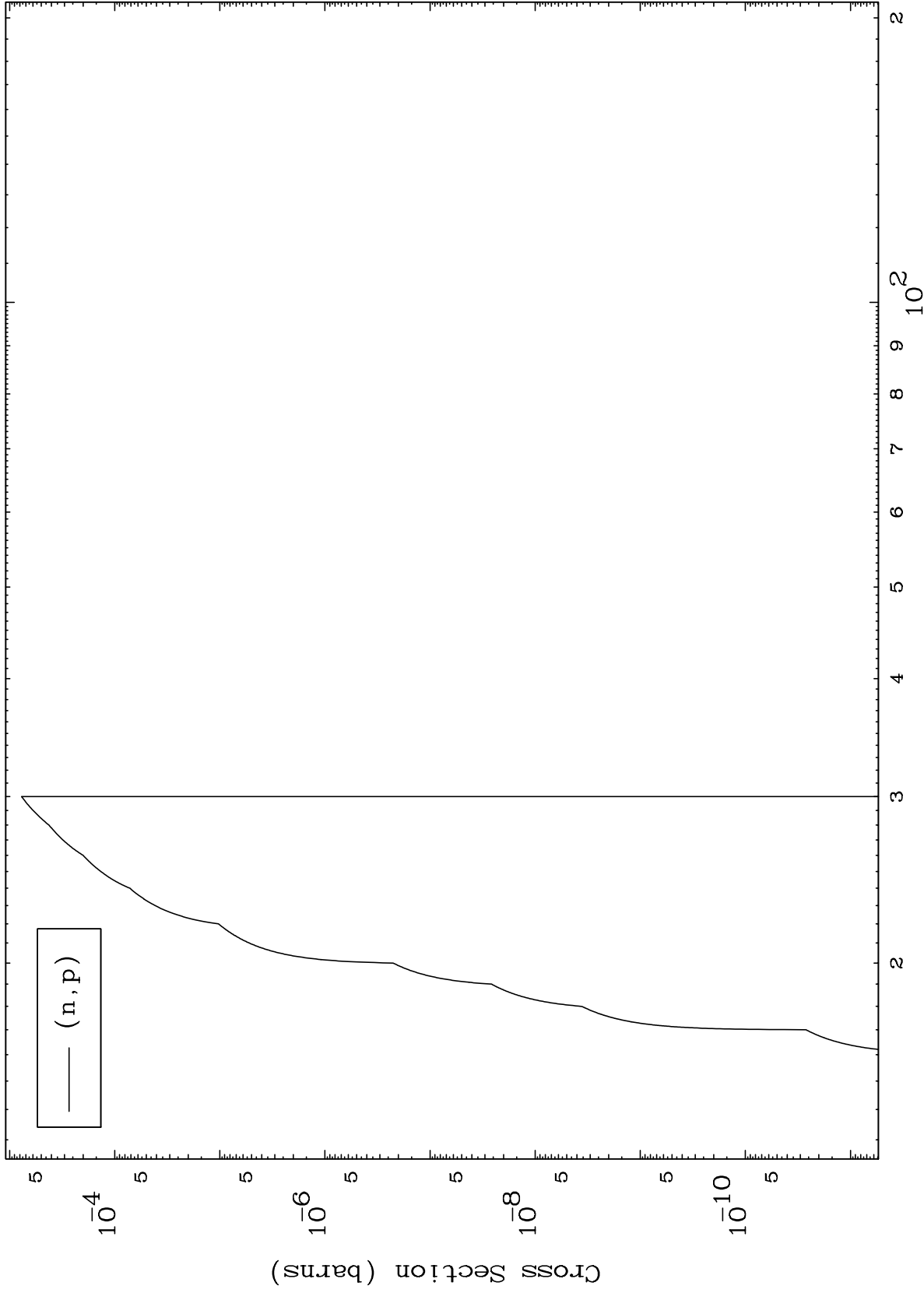




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86-Rn-211

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections



86-Rn-211

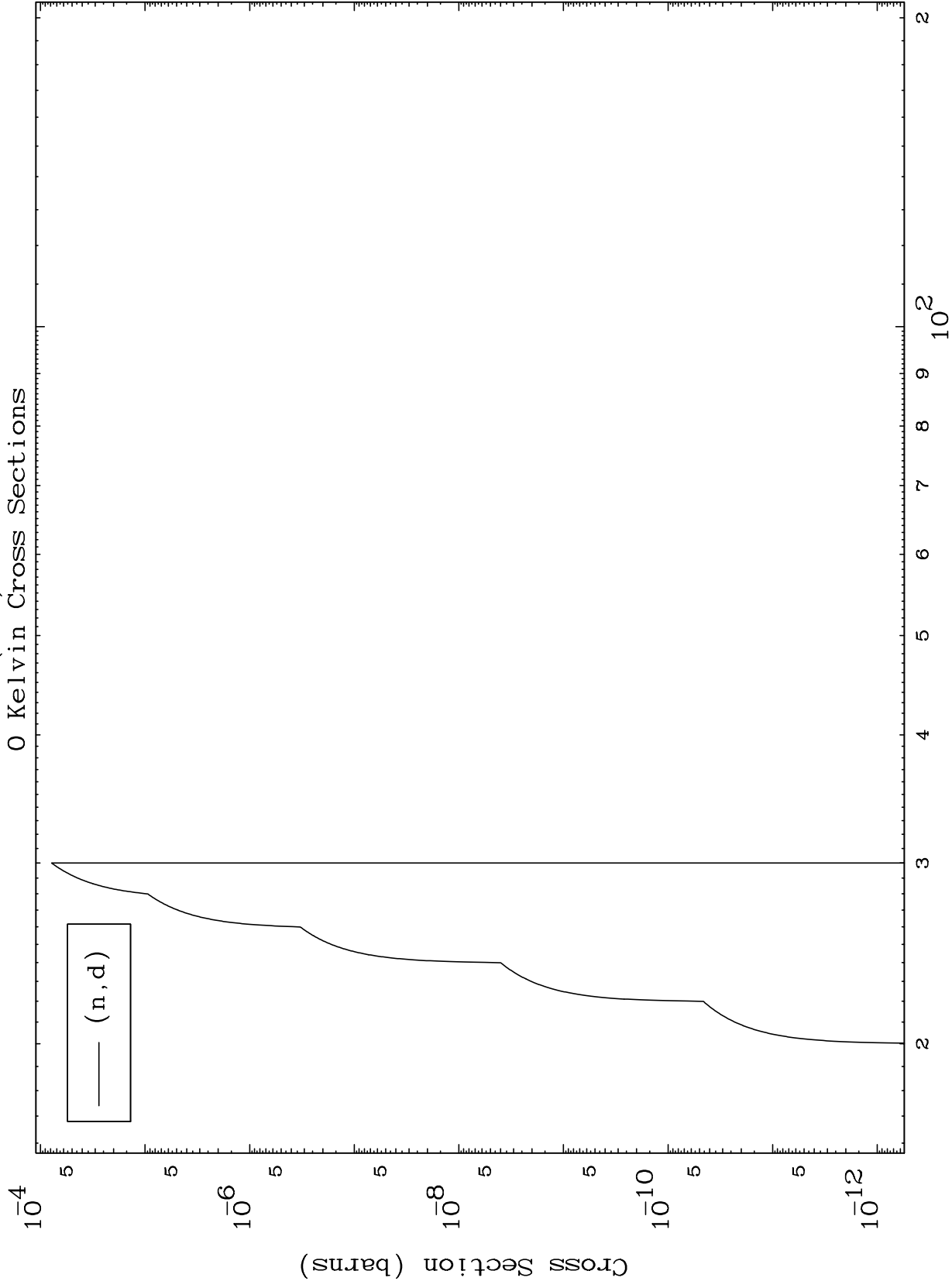
Incident Energy (MeV)

6

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86-Rn-211

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections



86-Rn-211

Incident Energy (MeV)

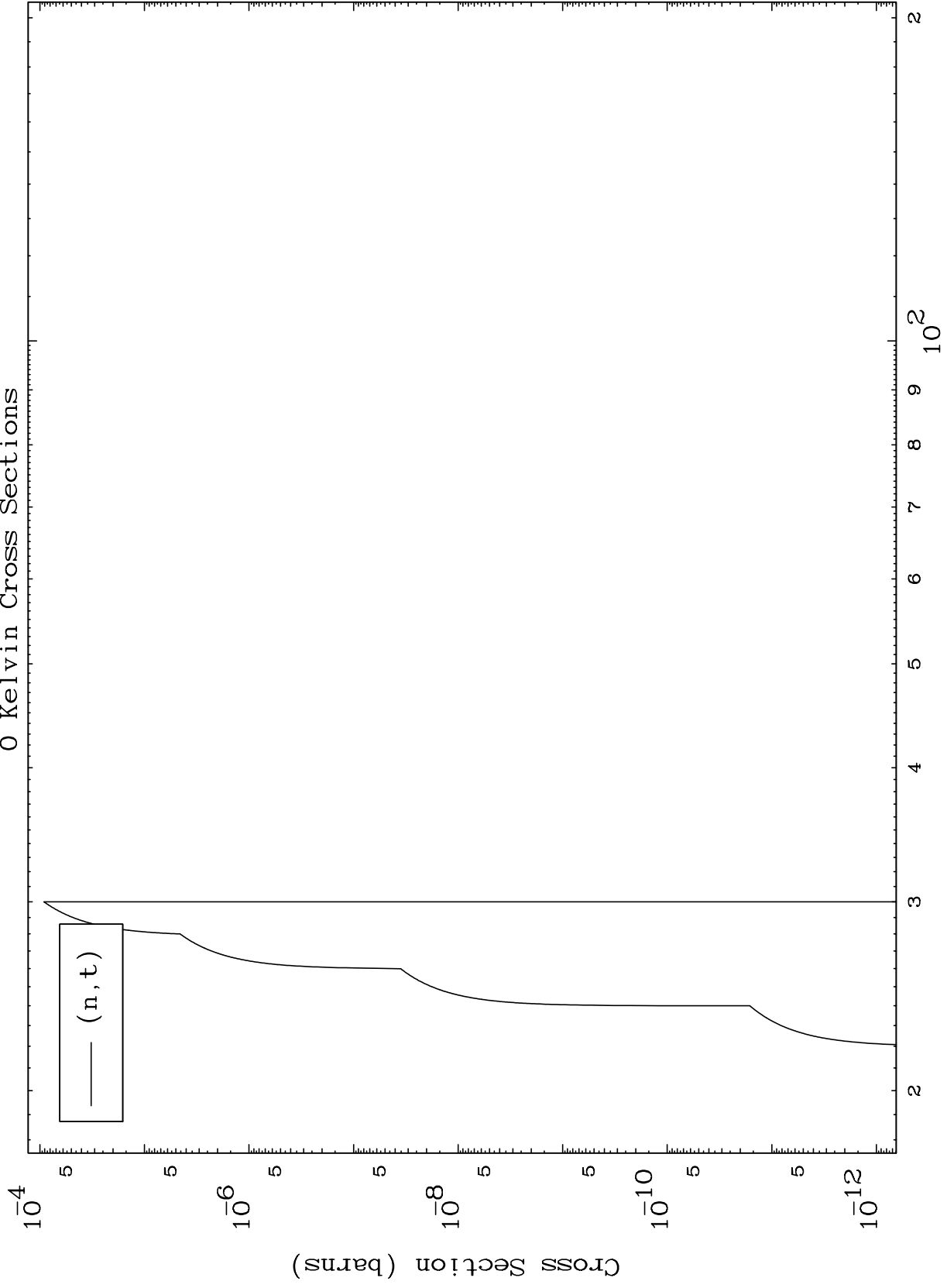
7



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( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

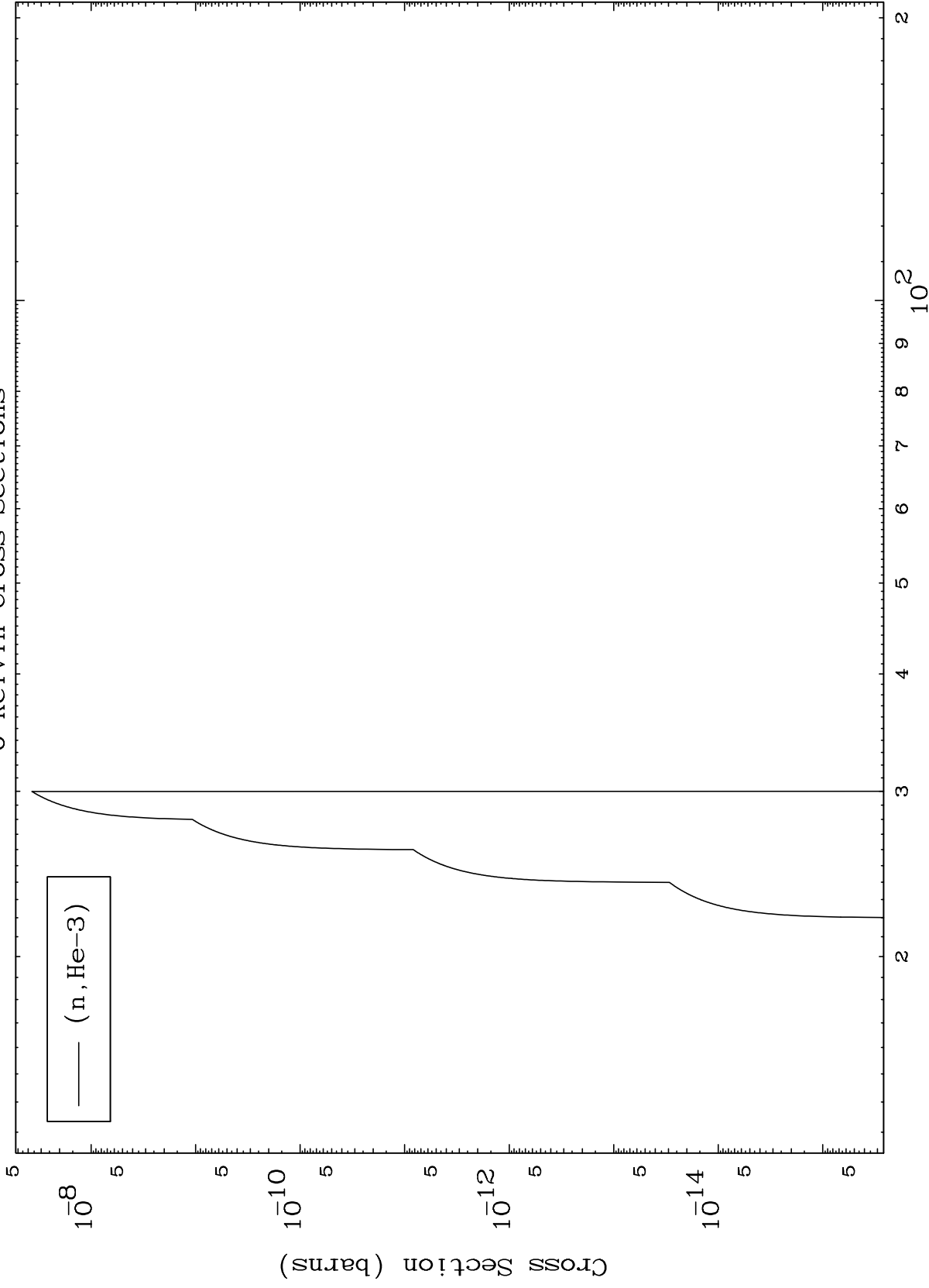
86-Rn-211



8

Incident Energy (MeV)

86-Rn-211

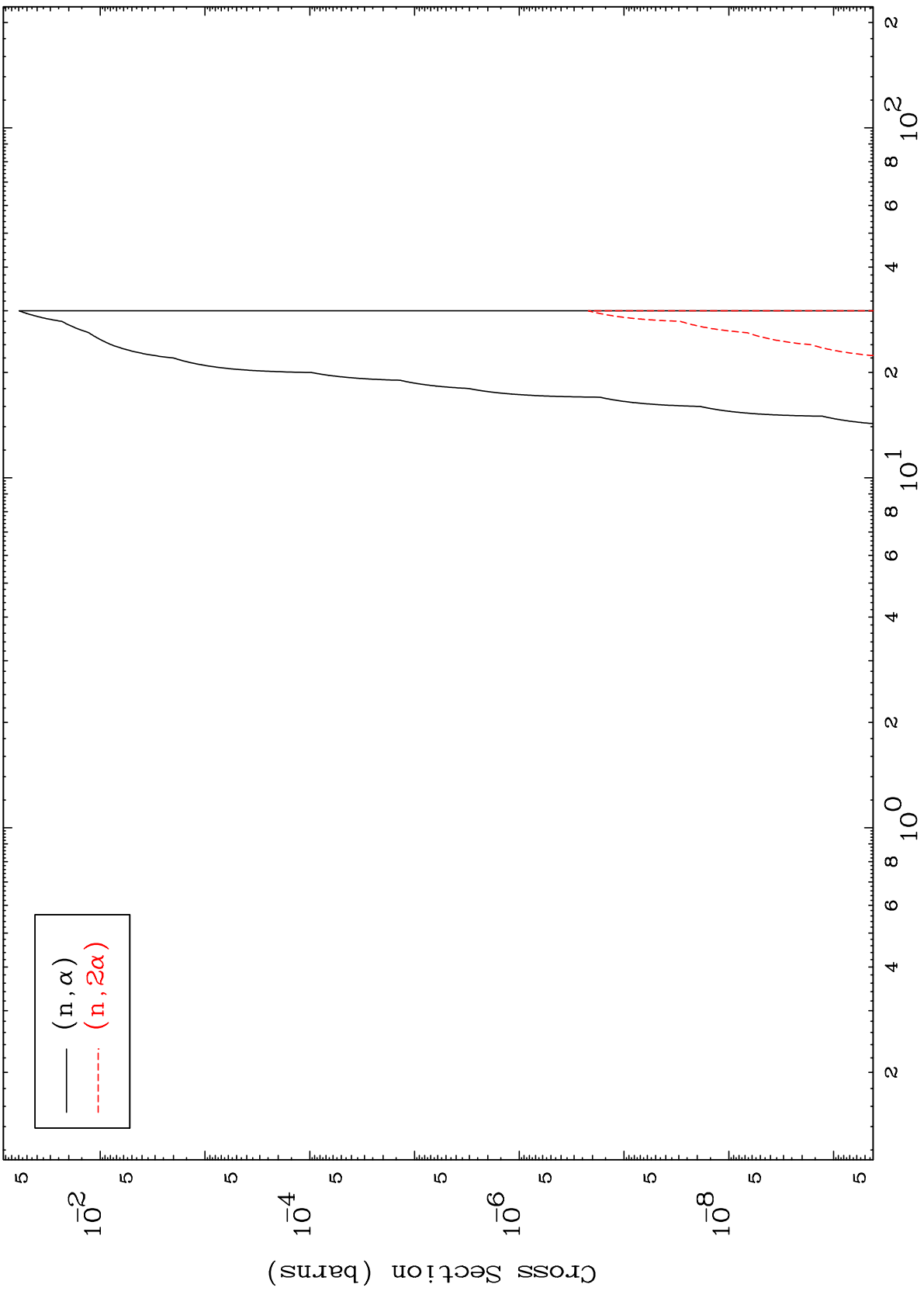


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( $\alpha, \alpha$ ) Levels

86-Rn-211

0 Kelvin Cross Sections



— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

10

Incident Energy (MeV)

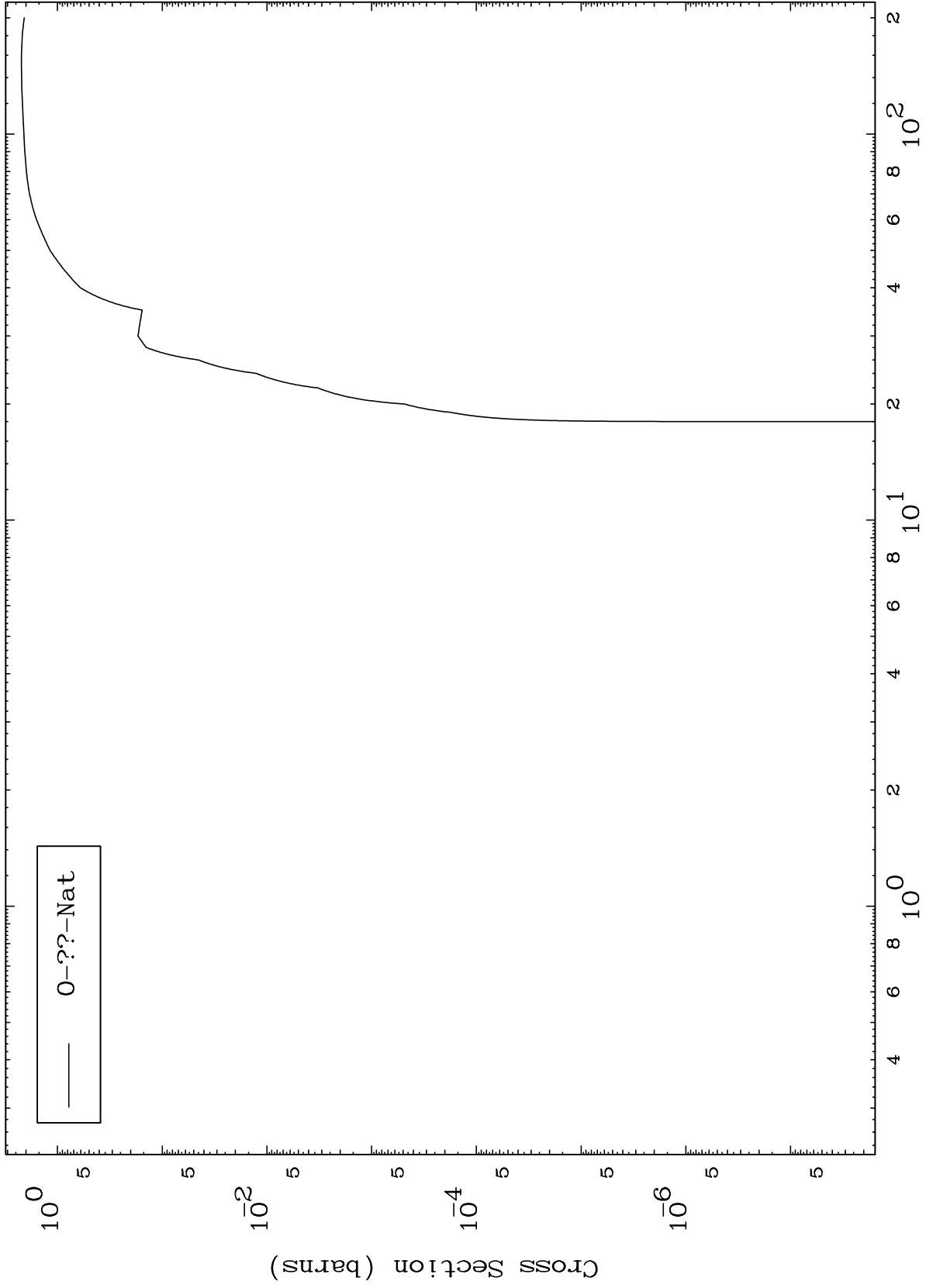
86-Rn-211

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Fission

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Radionuclide Production Cross Section

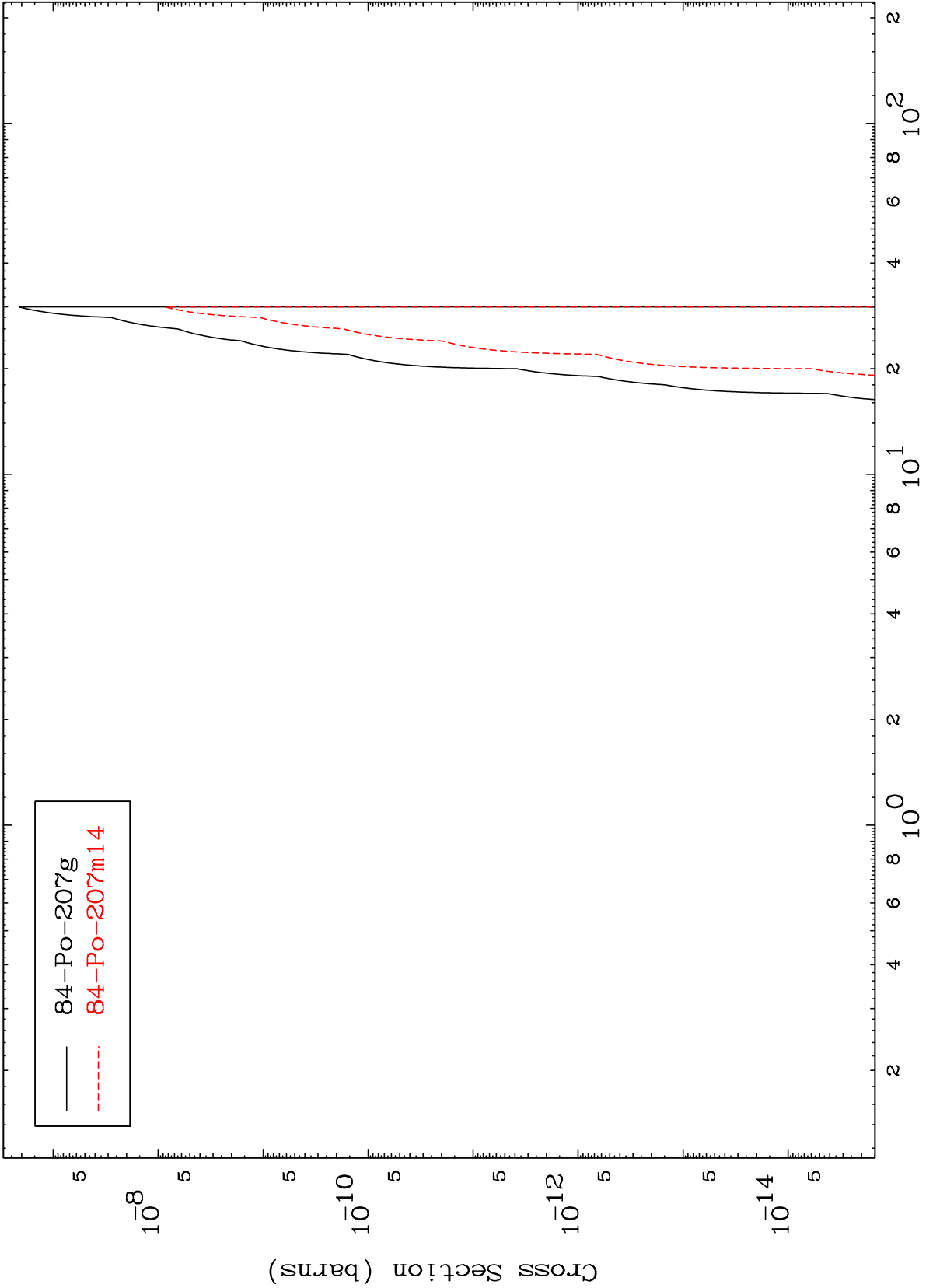


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(n,2α)

86-Rn-211

Radionuclide Production Cross Section



12

Incident Energy (MeV)

86-Rn-211