

Program EVALPLOT
(Version 2021-1)

by

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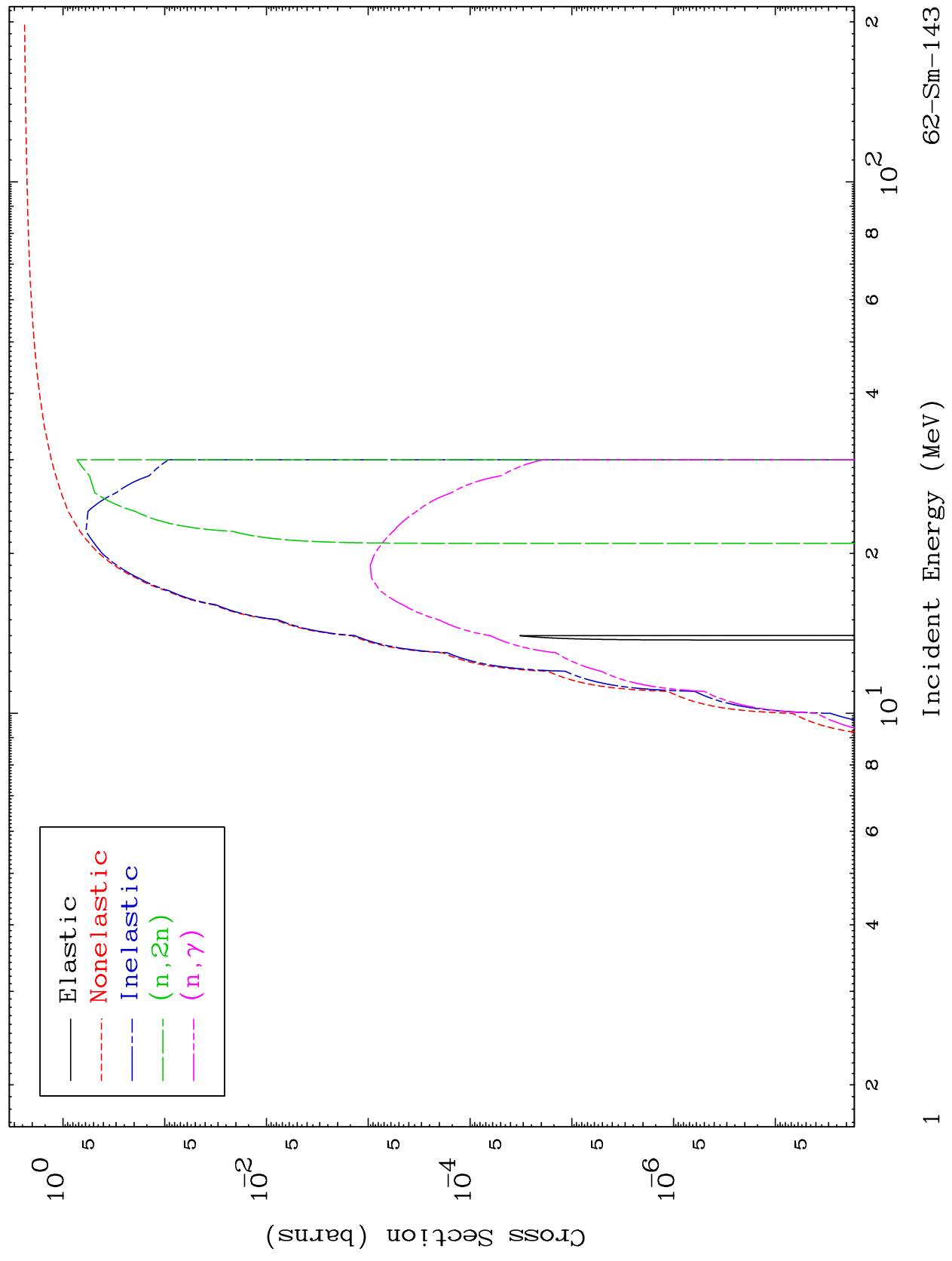
Press Mouse Button to Start

MAT 6222

0 Kelvin

Major Cross Sections

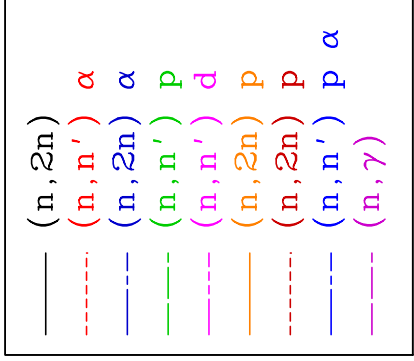
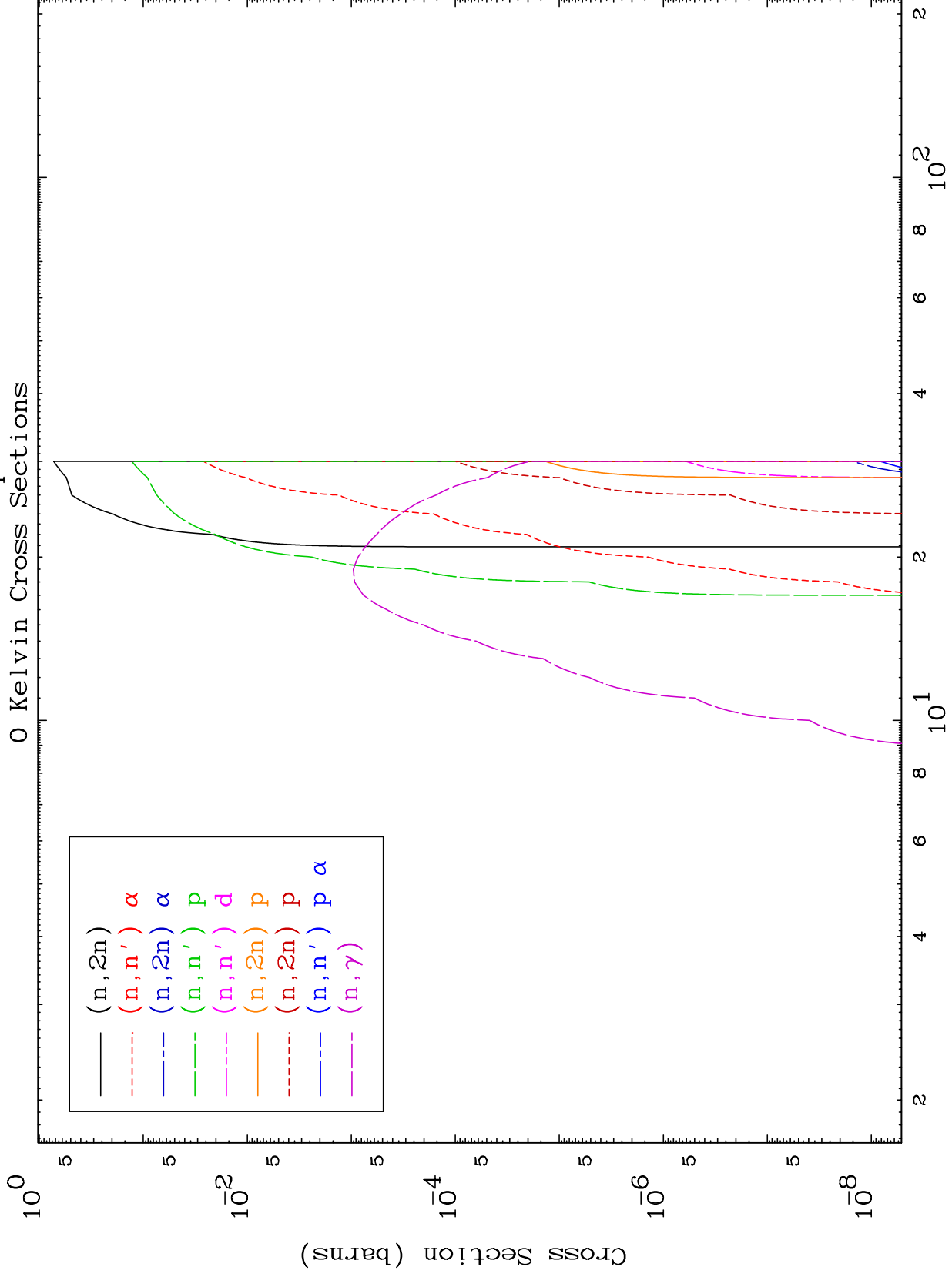
$^{62}\text{Sm-143}$



MAT 6222

α Neutron Absorption
0 Kelvin Cross Sections

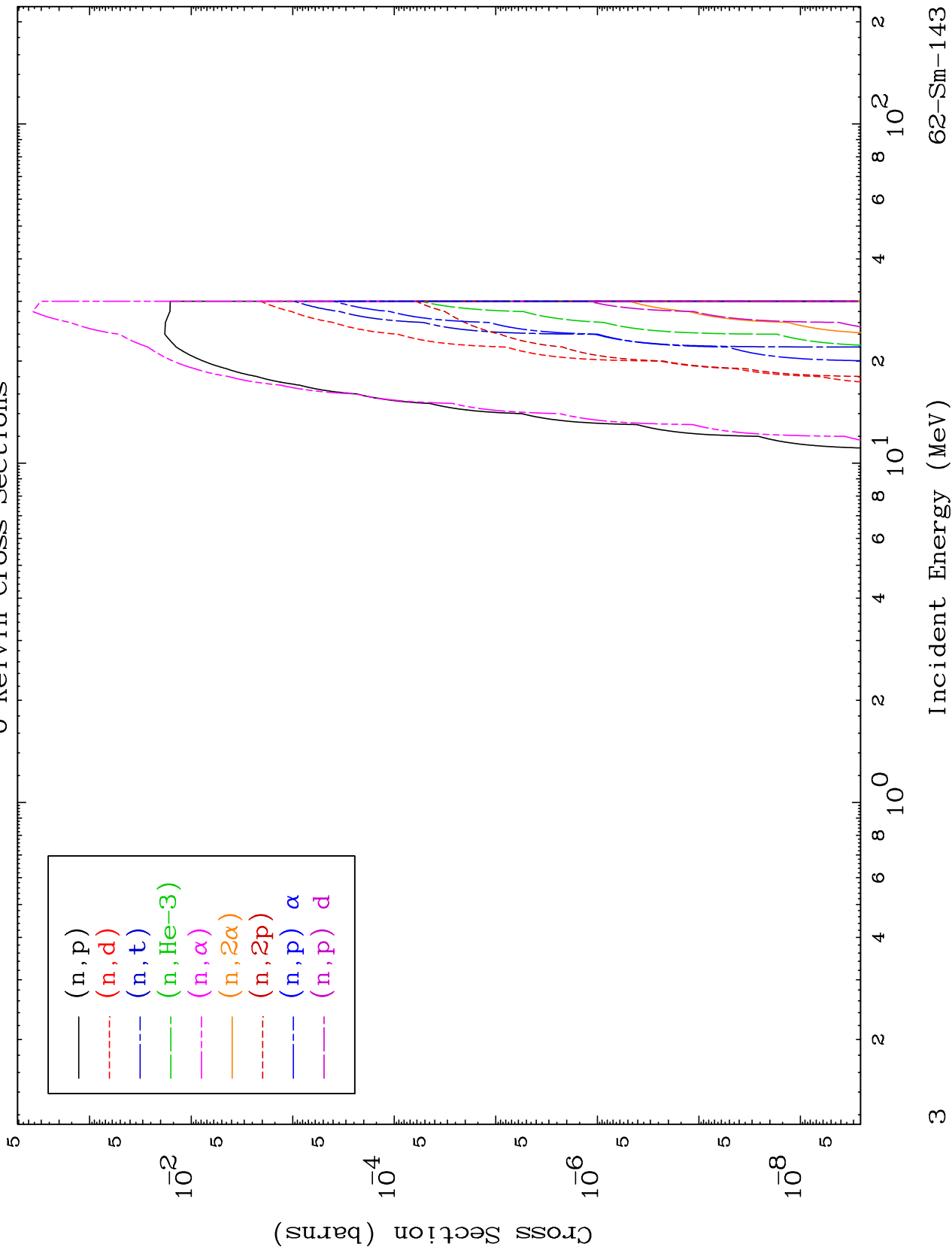
62-Sm-143

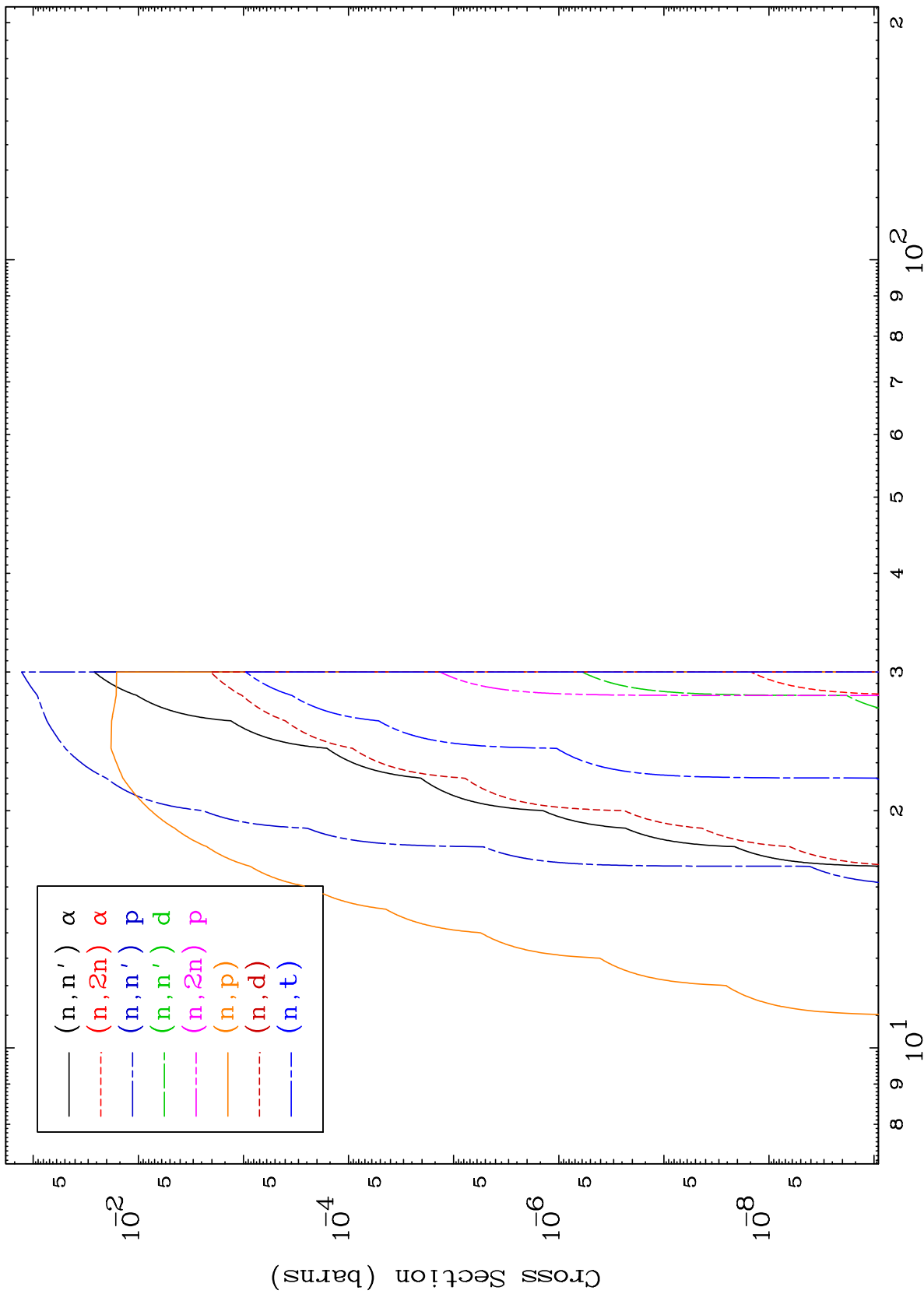


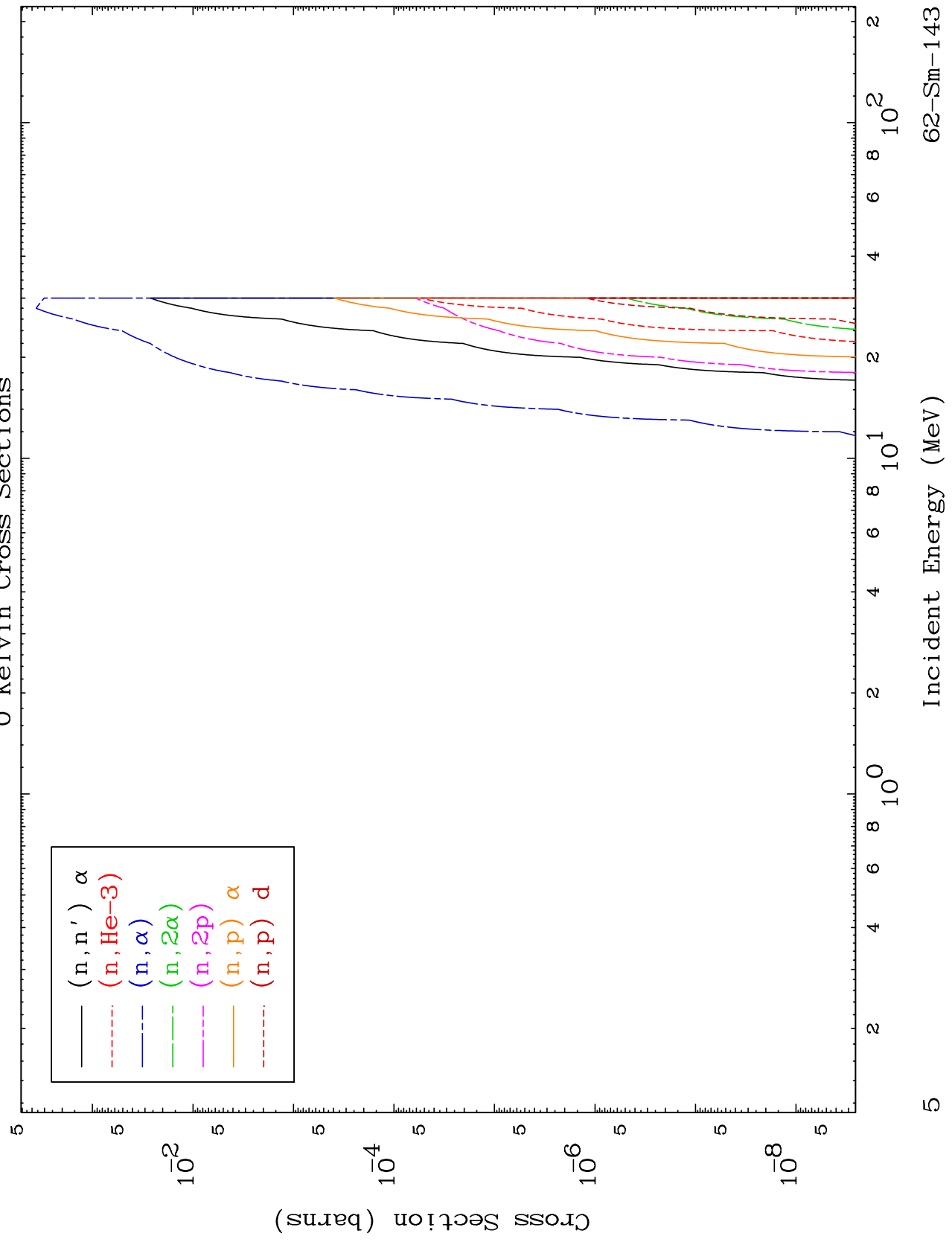
2

Incident Energy (MeV)

62-Sm-143



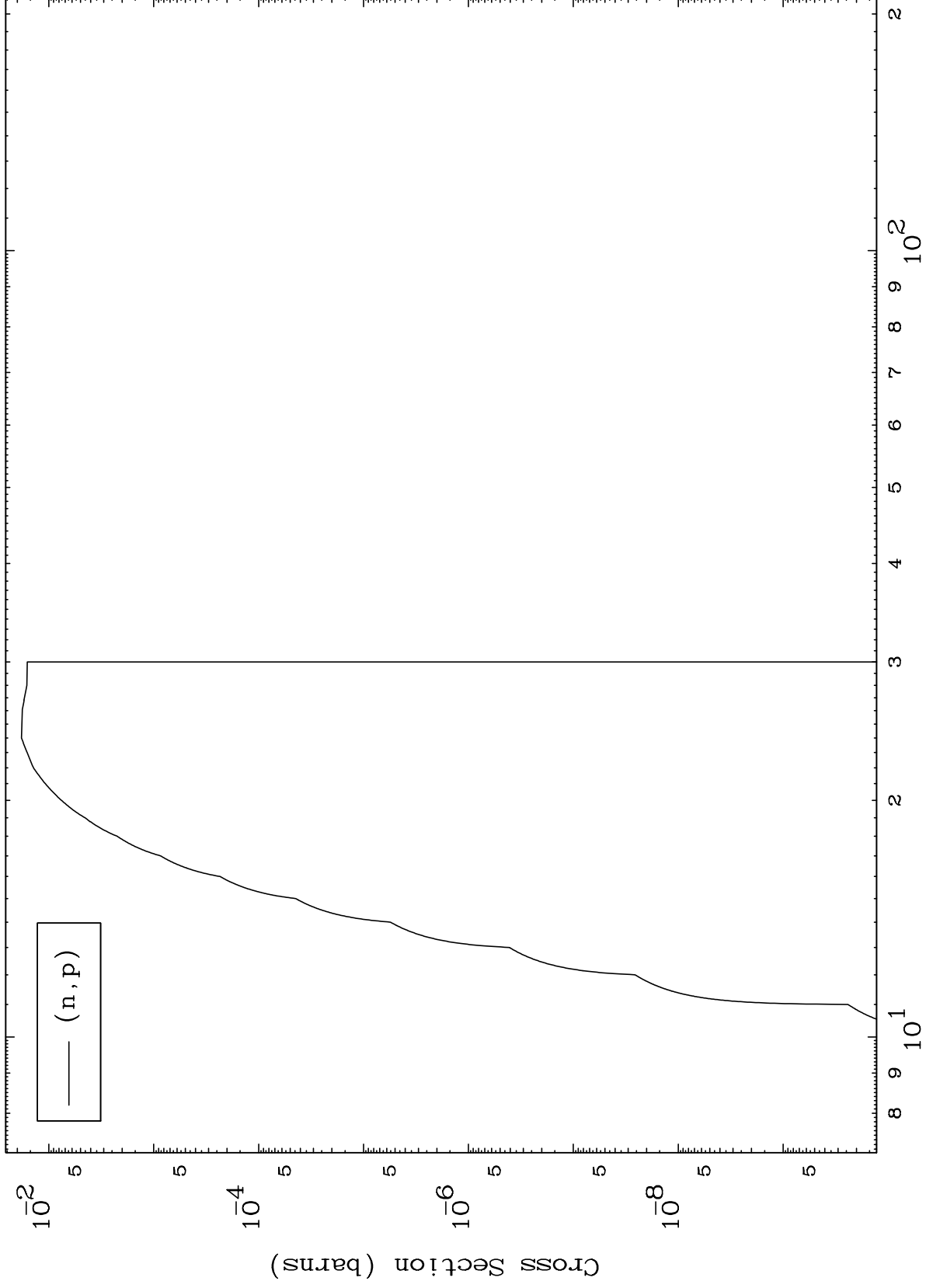




MAT 6222

62-Sm-143

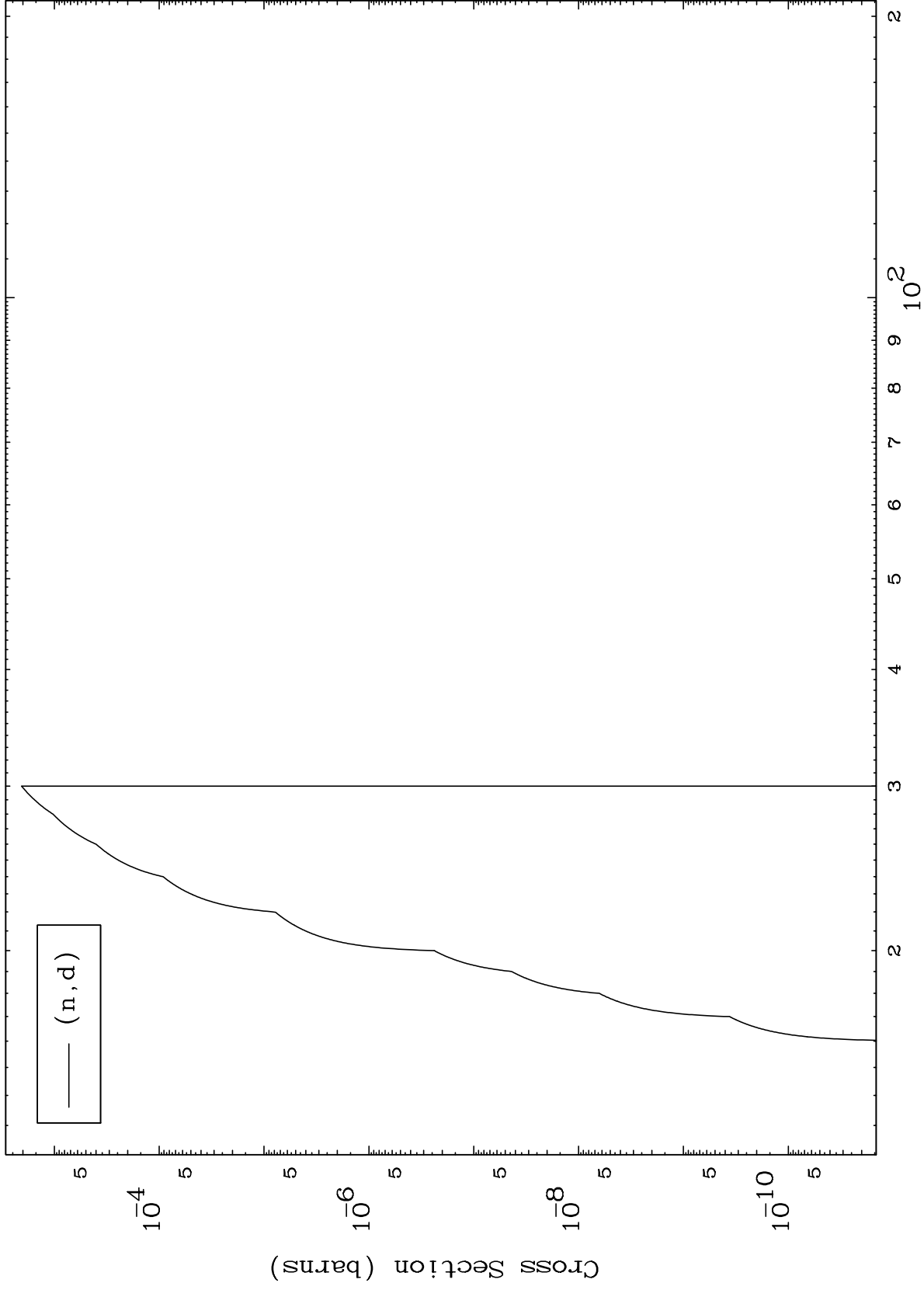
(α, p) Levels
0 Kelvin Cross Sections

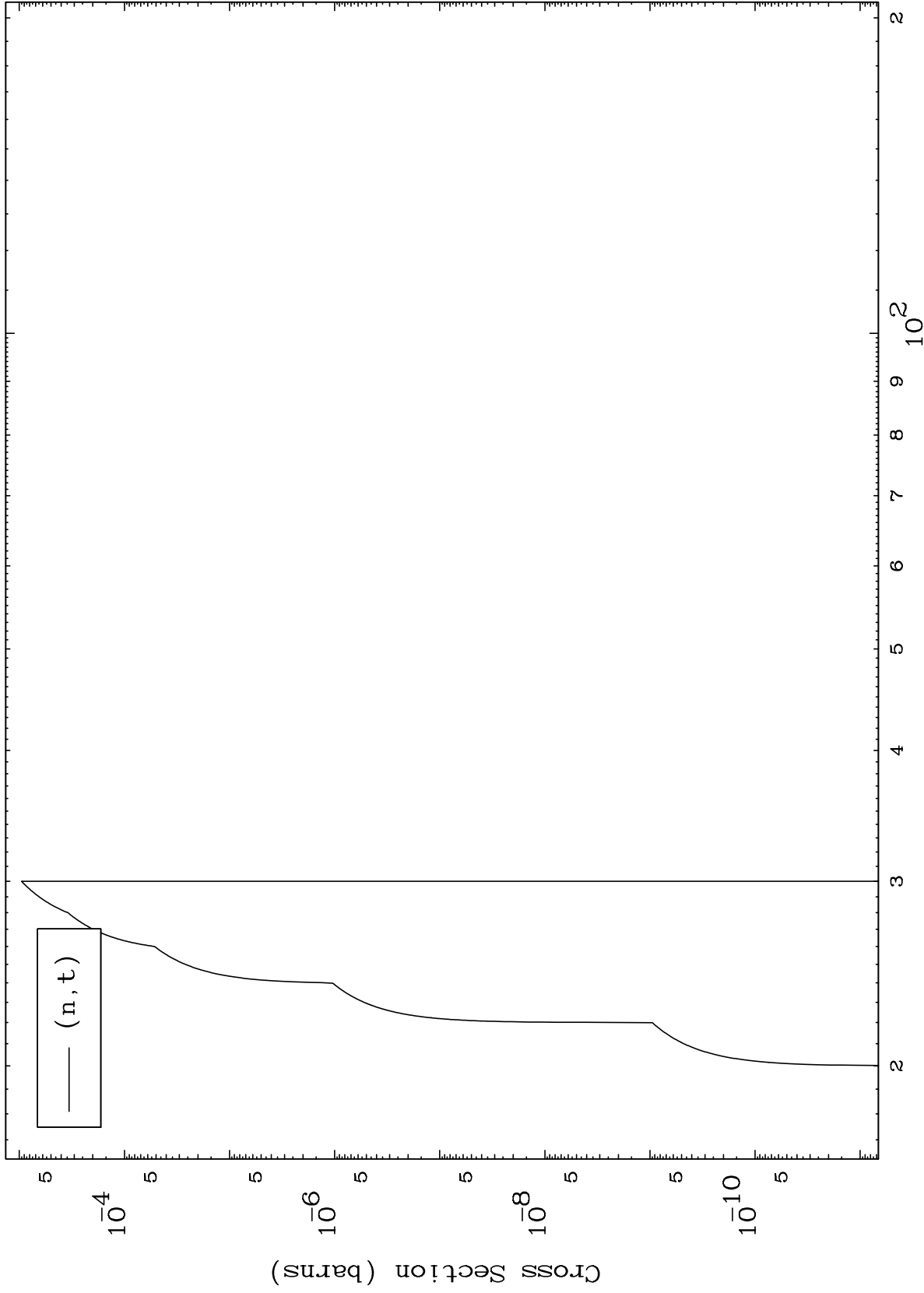


62-Sm-143

Incident Energy (MeV)

6

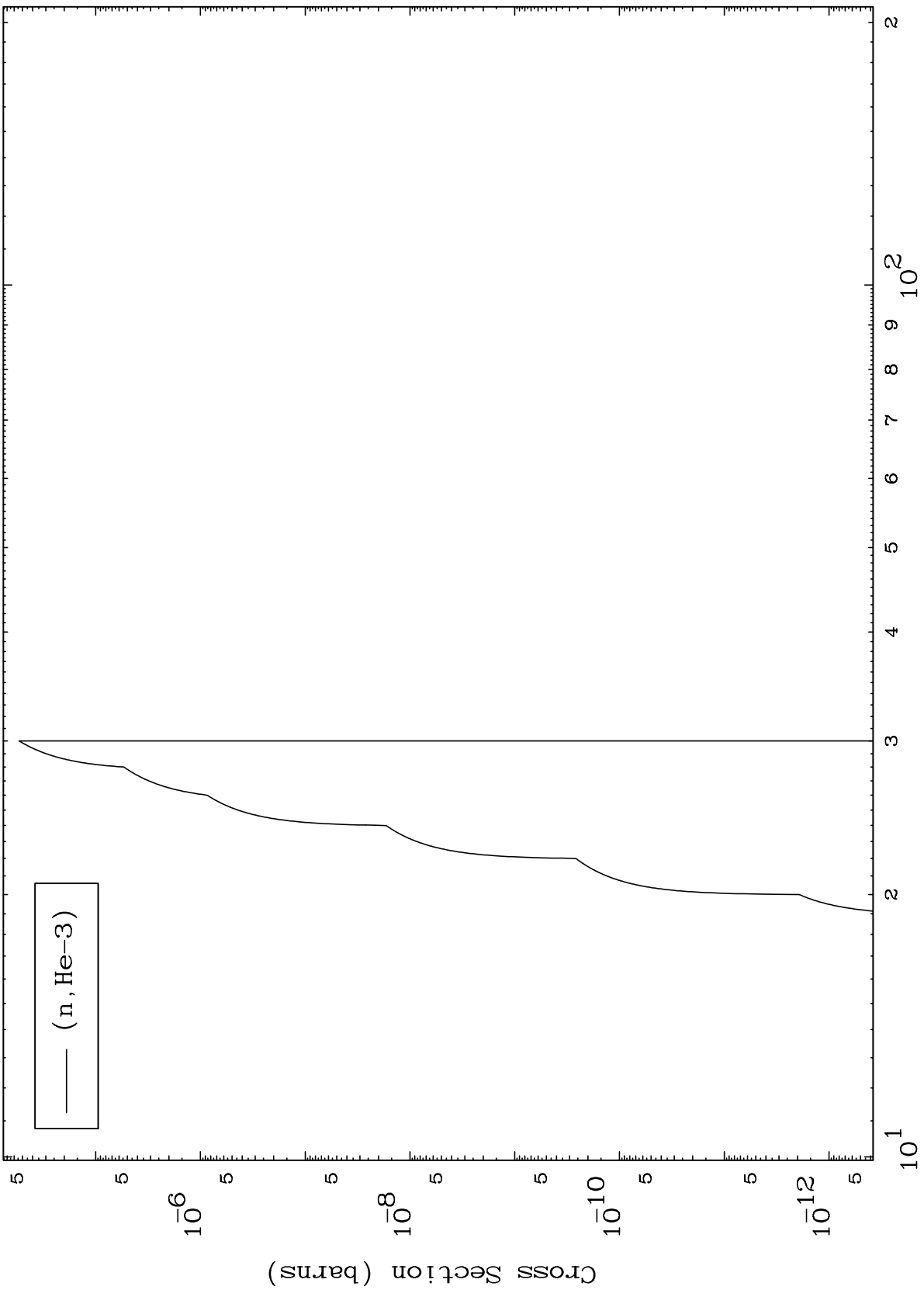




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62-Sm-143

($\alpha, \text{He3}$) Levels
0 Kelvin Cross Sections



62-Sm-143

Incident Energy (MeV)

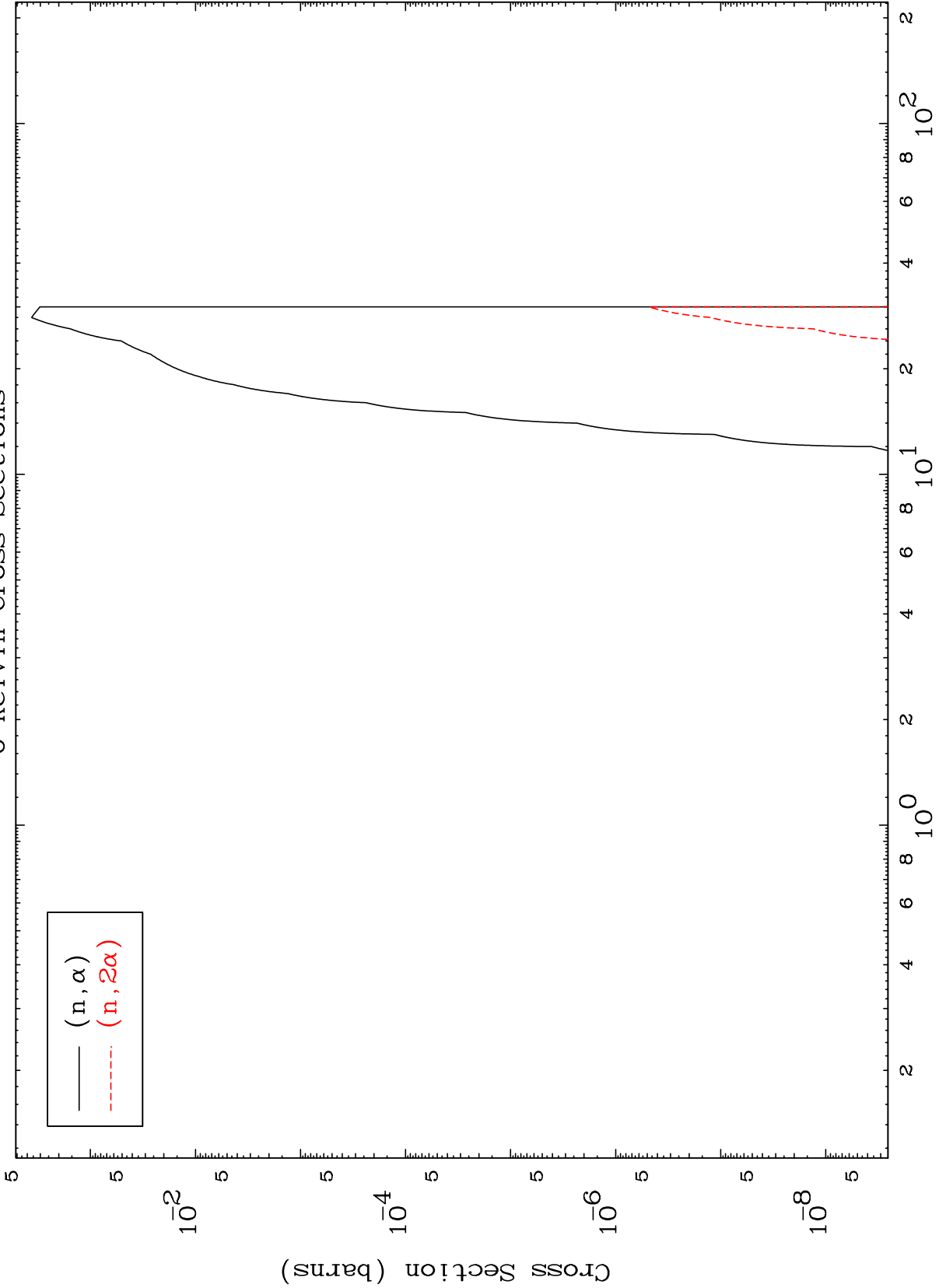
9

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(α, α) Levels

62-Sm-143

0 Kelvin Cross Sections



— (n, α)
- - - ($n, 2\alpha$)

10

Incident Energy (MeV)

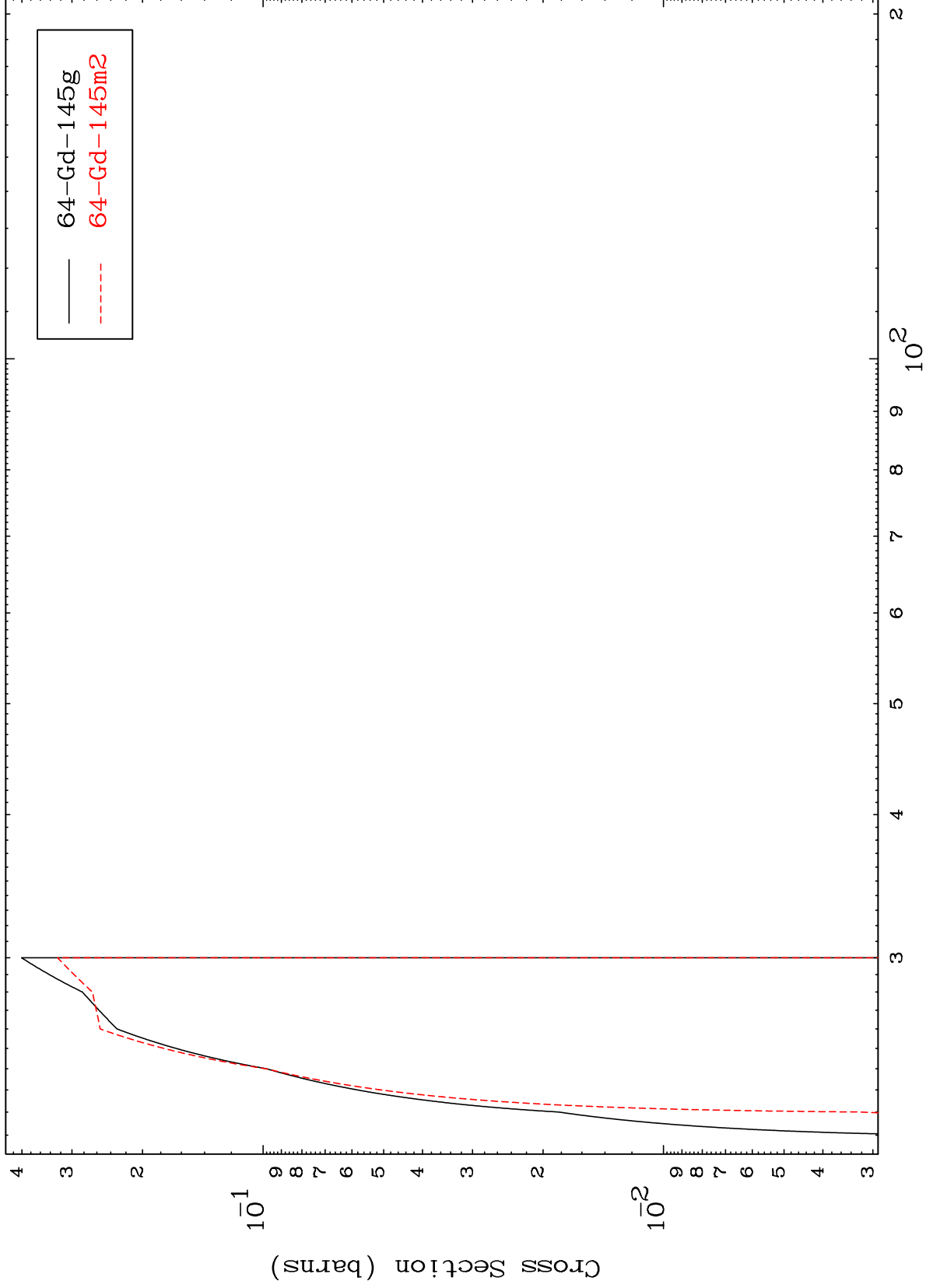
62-Sm-143

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(n,2n)

62-Sm-143

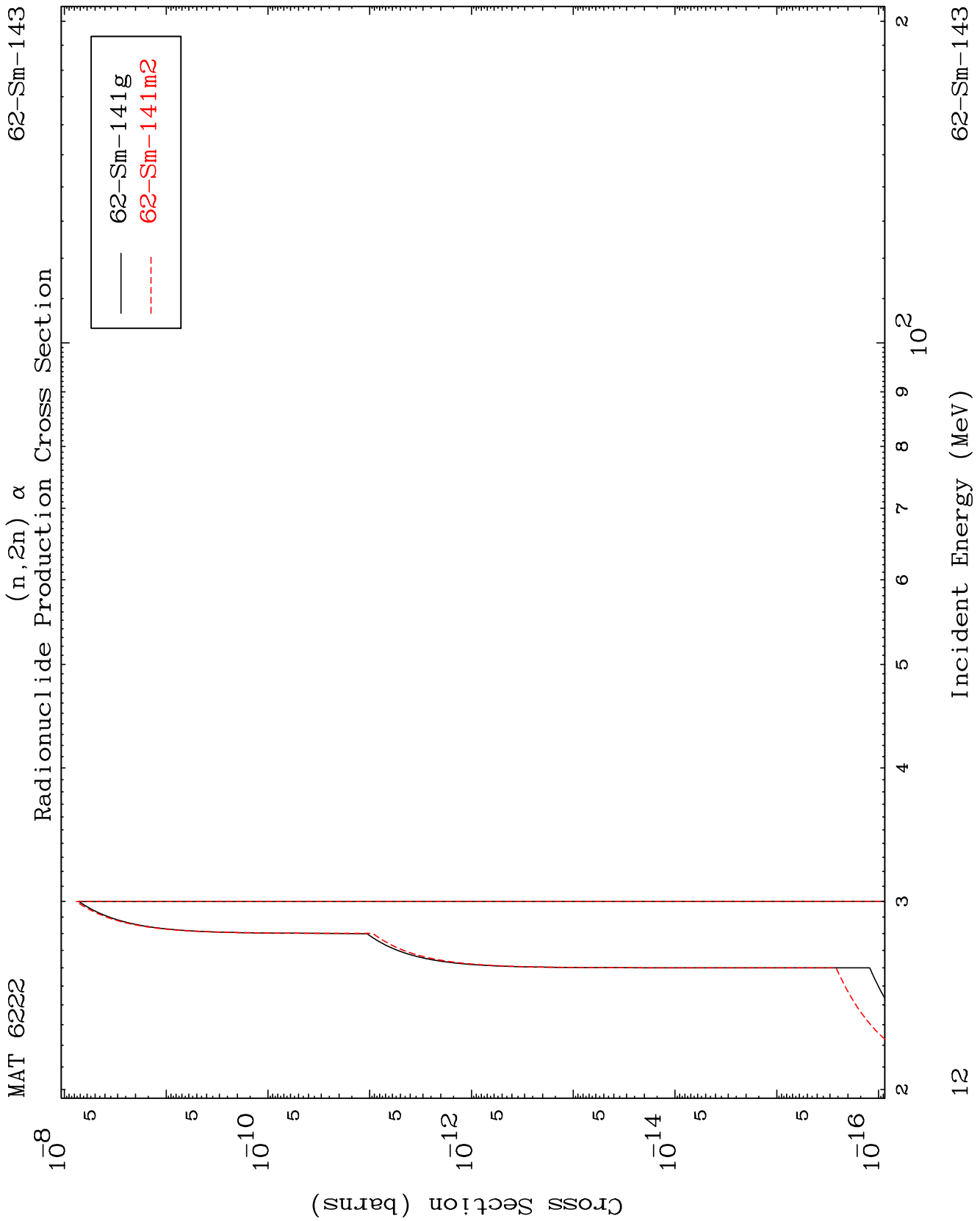
Radionuclide Production Cross Section



11

Incident Energy (MeV)

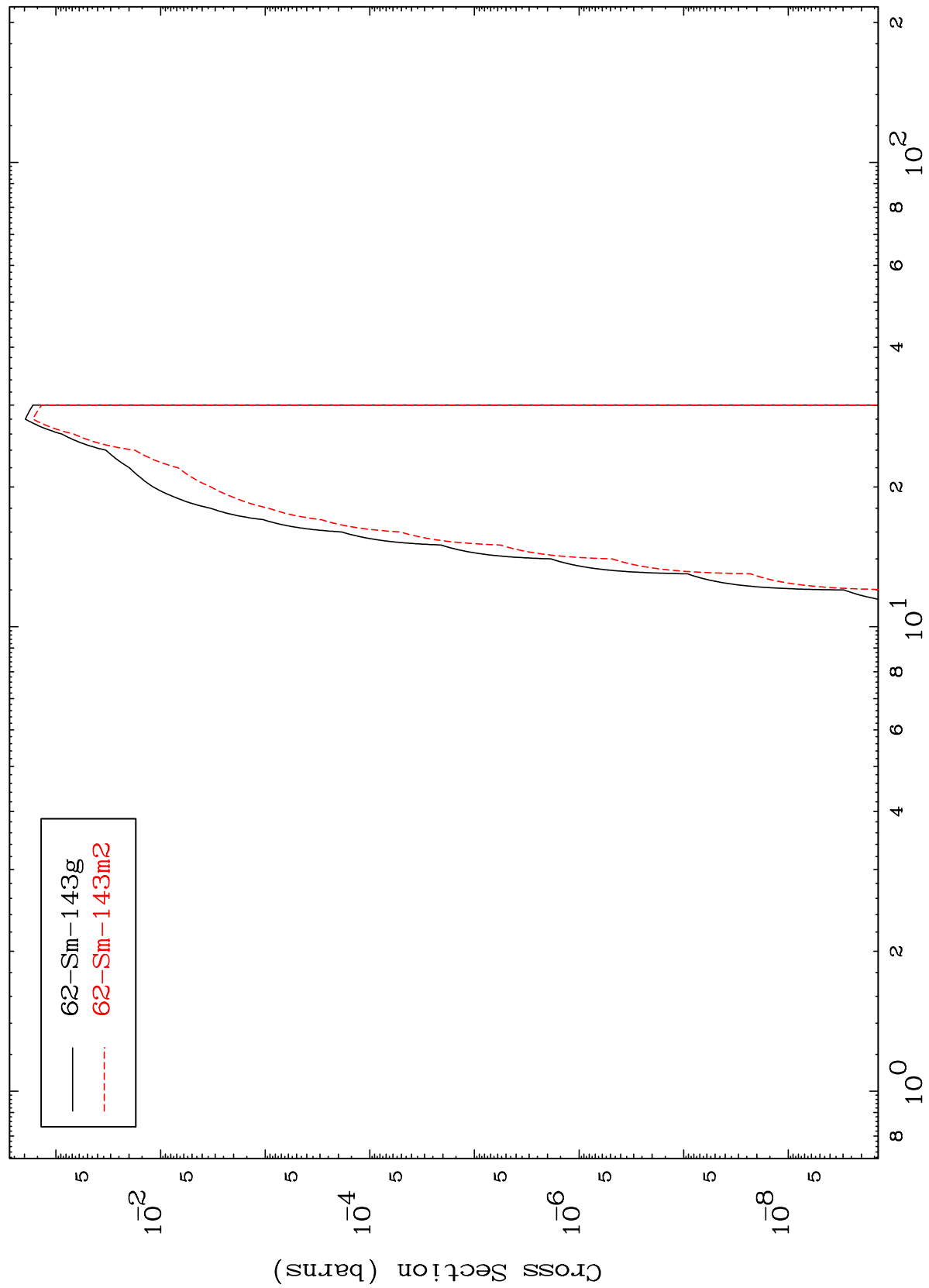
62-Sm-143



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62-Sm-143

Radionuclide Production Cross Section
(n, α)



62-Sm-143g
62-Sm-143m2

13

Incident Energy (MeV)

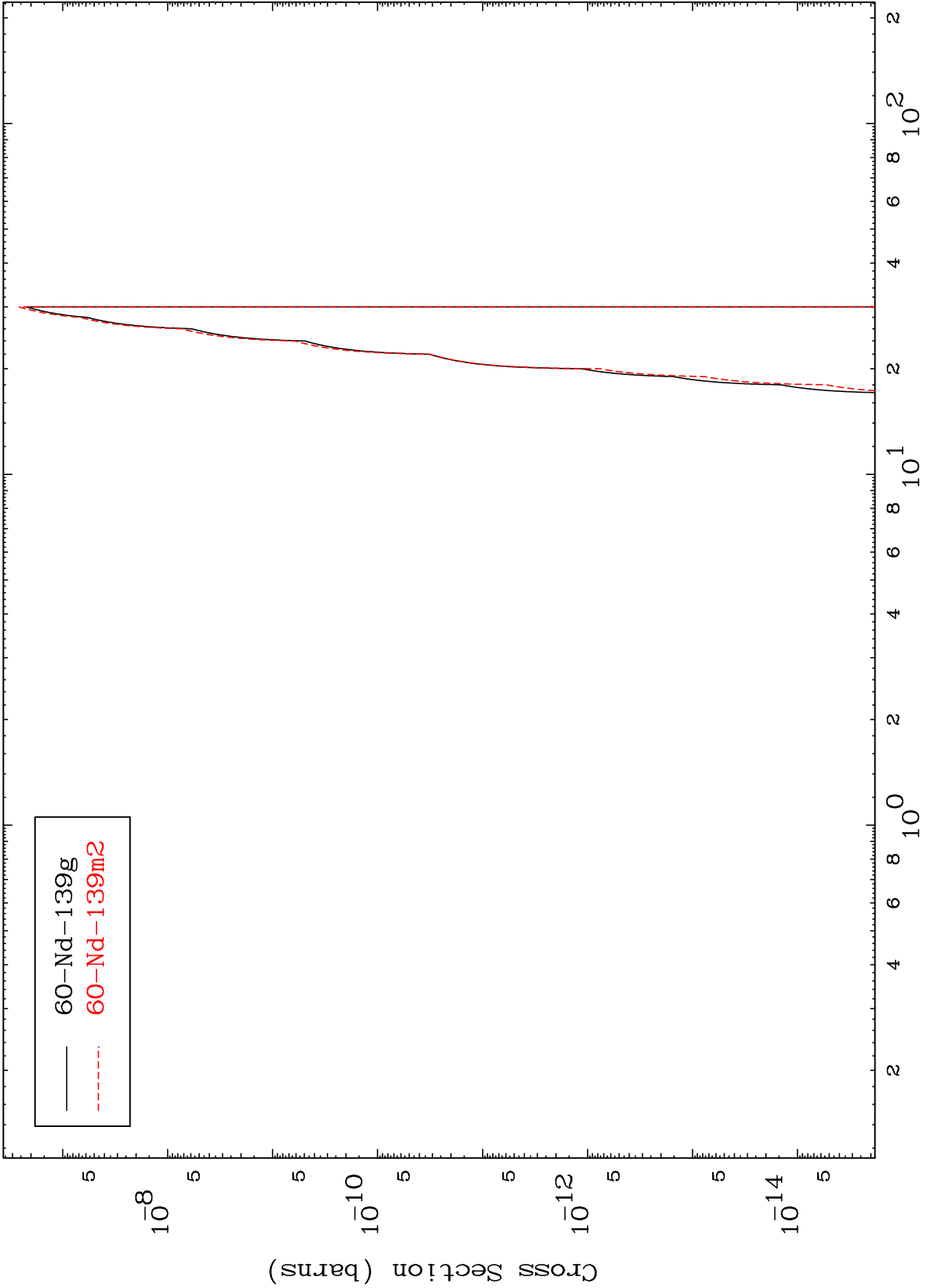
62-Sm-143

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(n,2α)

62-Sm-143

Radionuclide Production Cross Section



14

Incident Energy (MeV)

62-Sm-143