

Program EVALPLOT  
(Version 2021-1)

by

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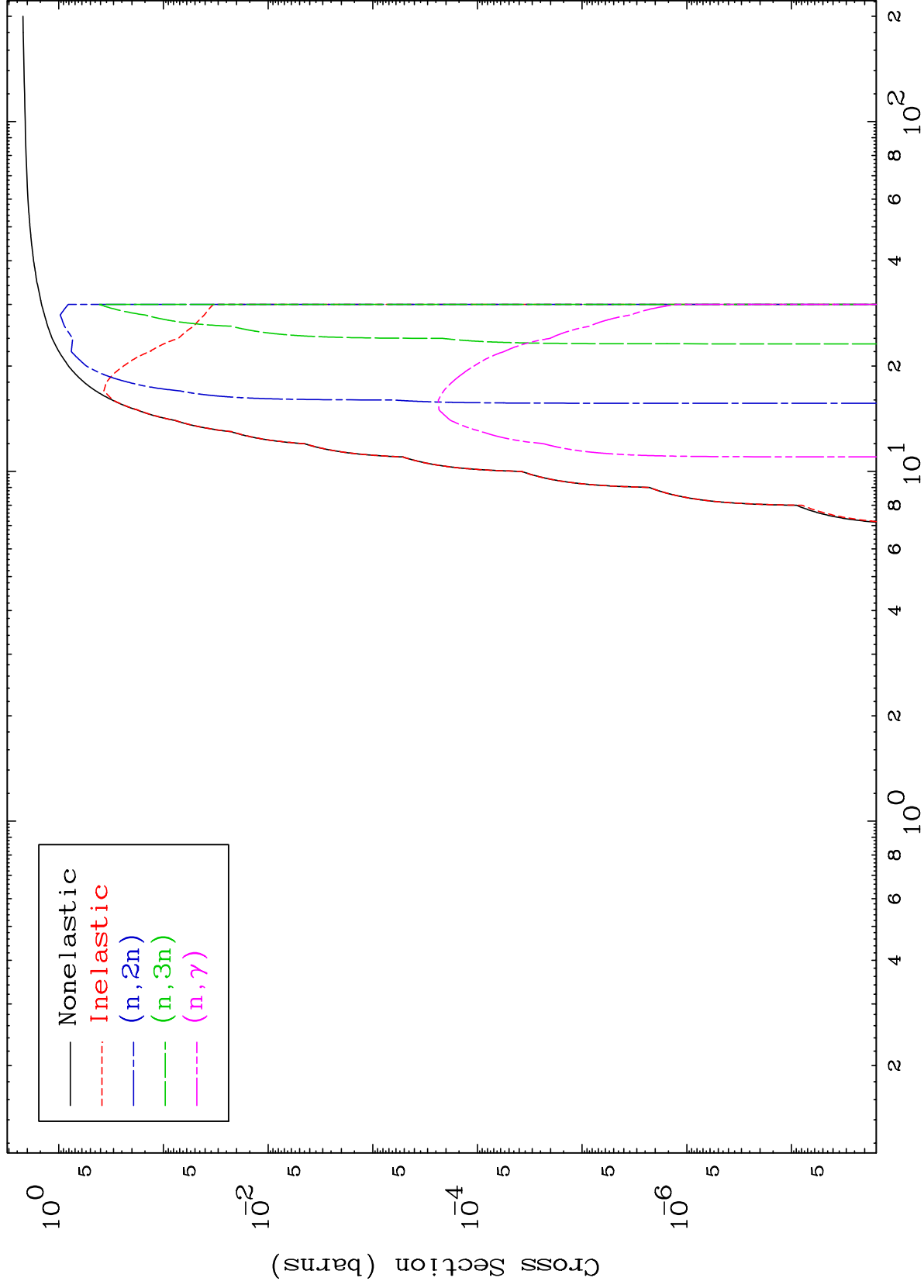
Press Mouse Button to Start

MAT 5047

$\alpha$  Major

0 Kelvin Cross Sections

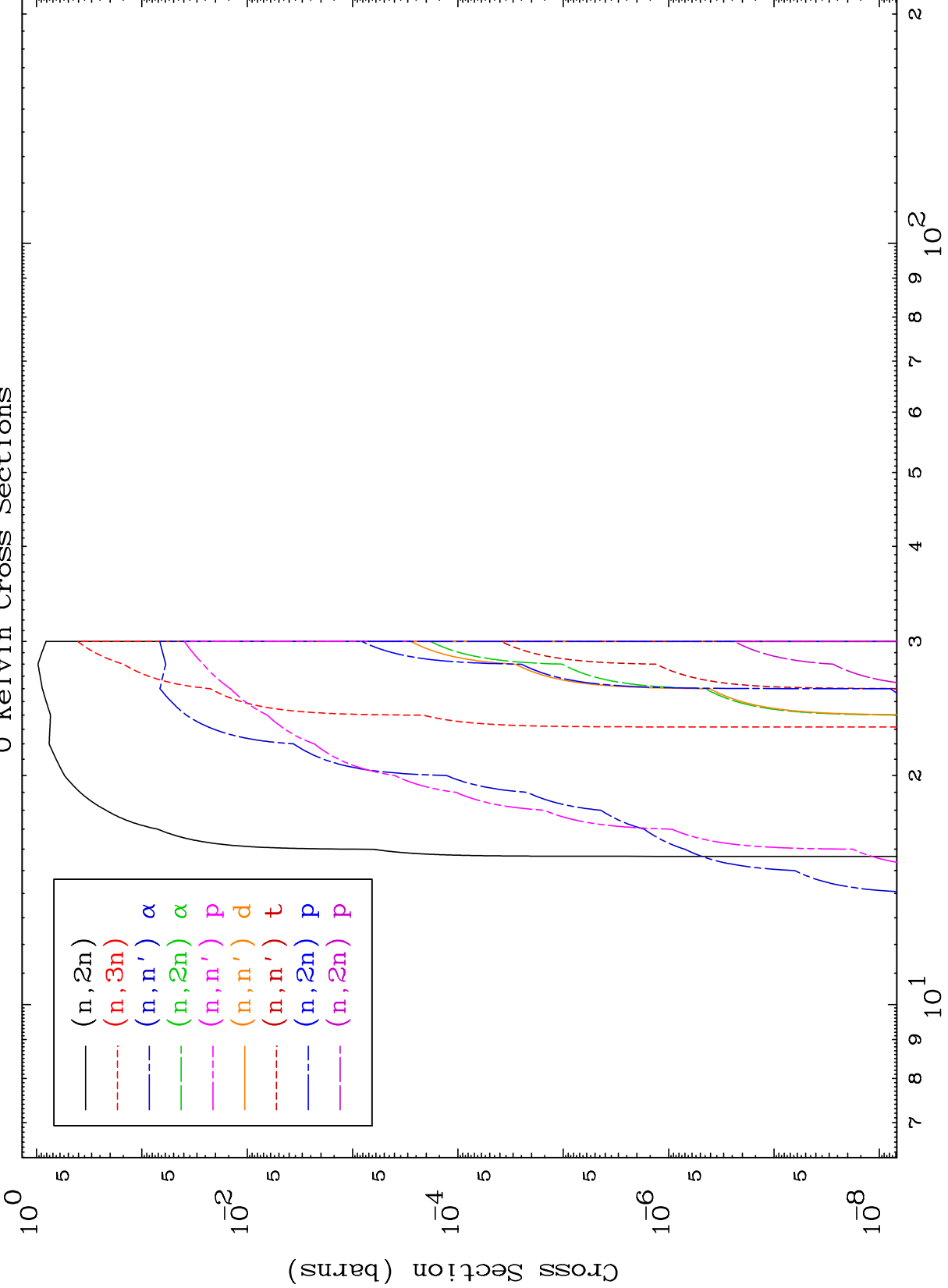
50-Sn-119m

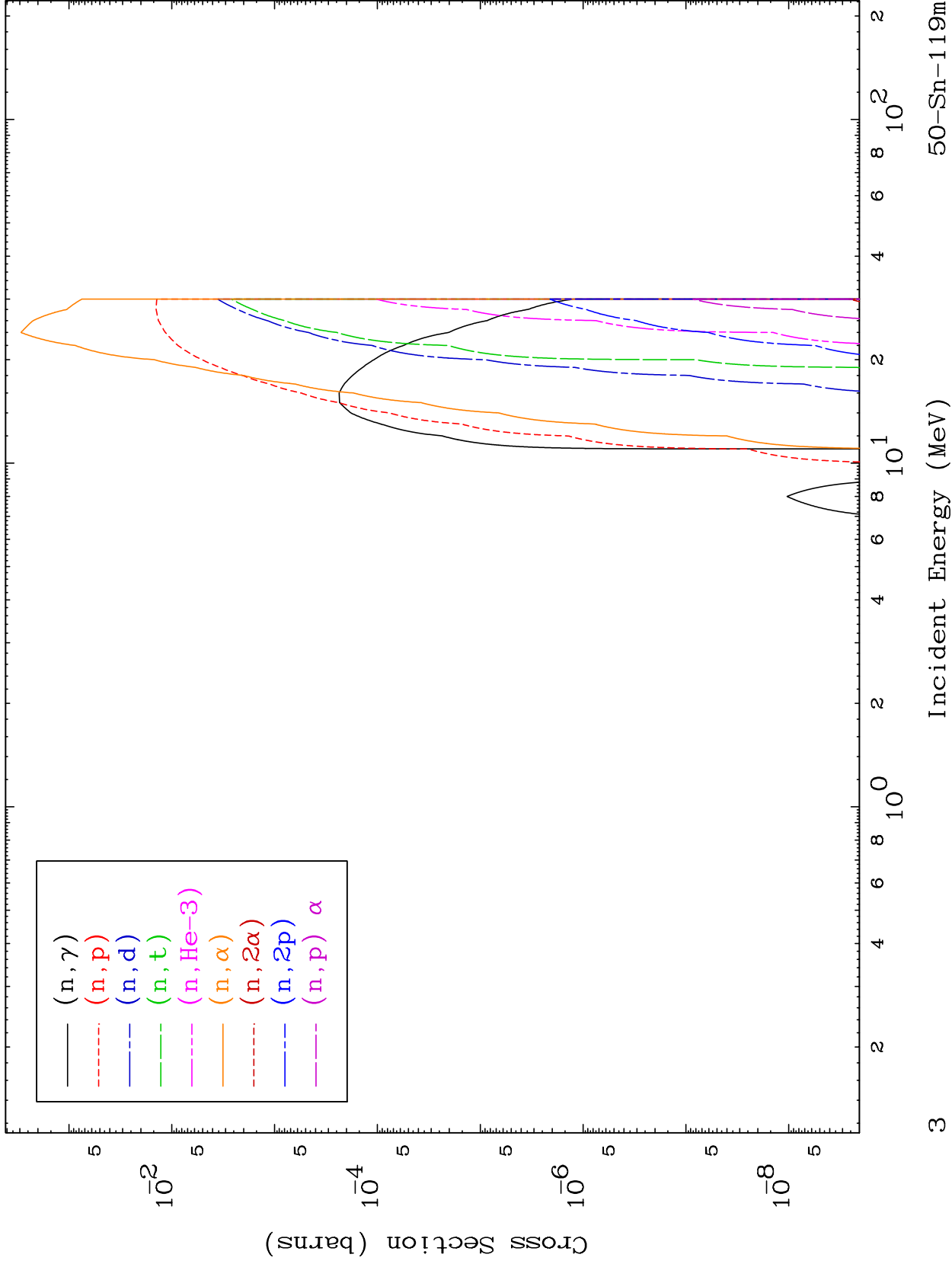


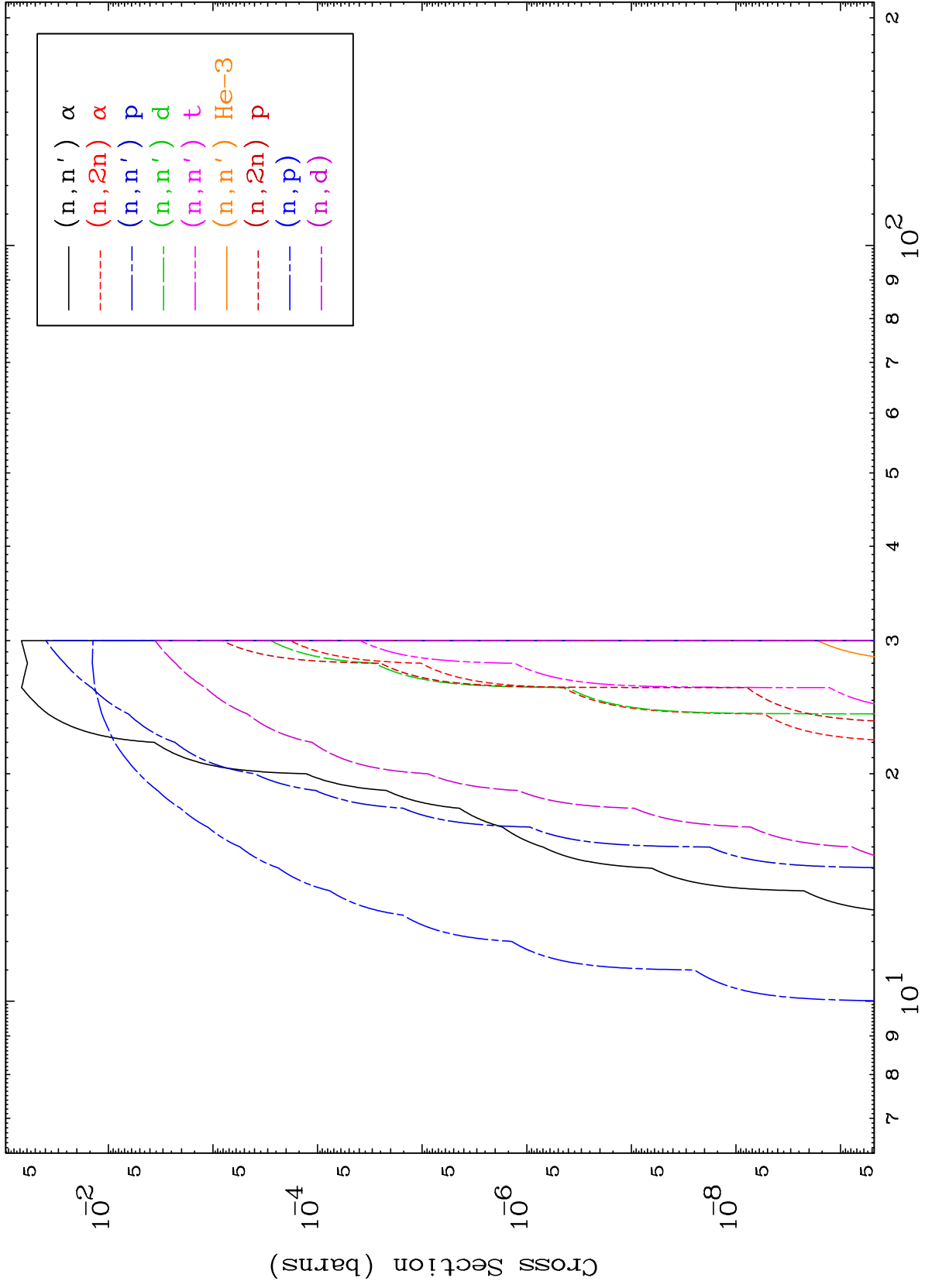
MAT 5047

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

50-Sn-119m



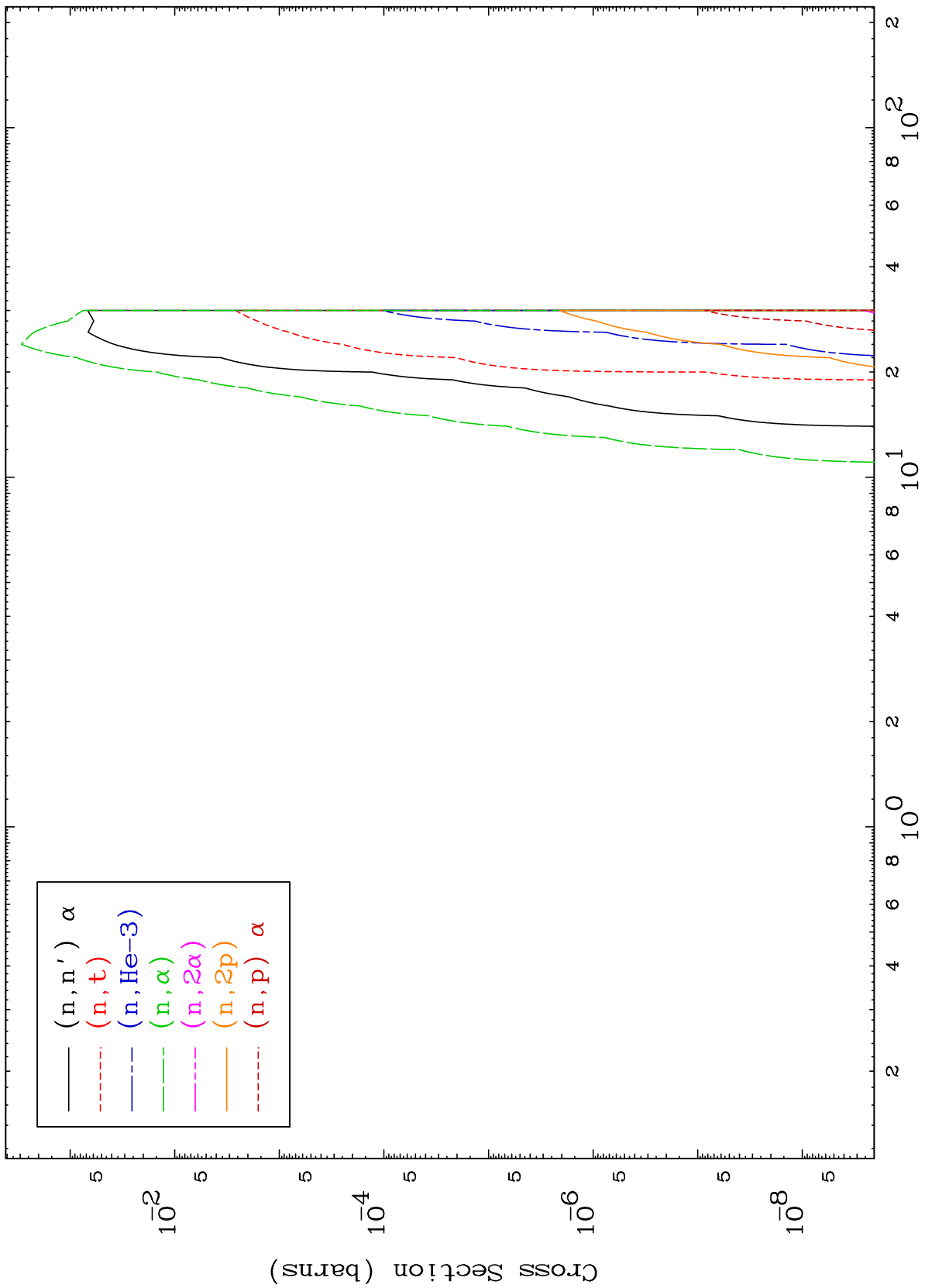




MAT 5047

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

50-Sn-119m

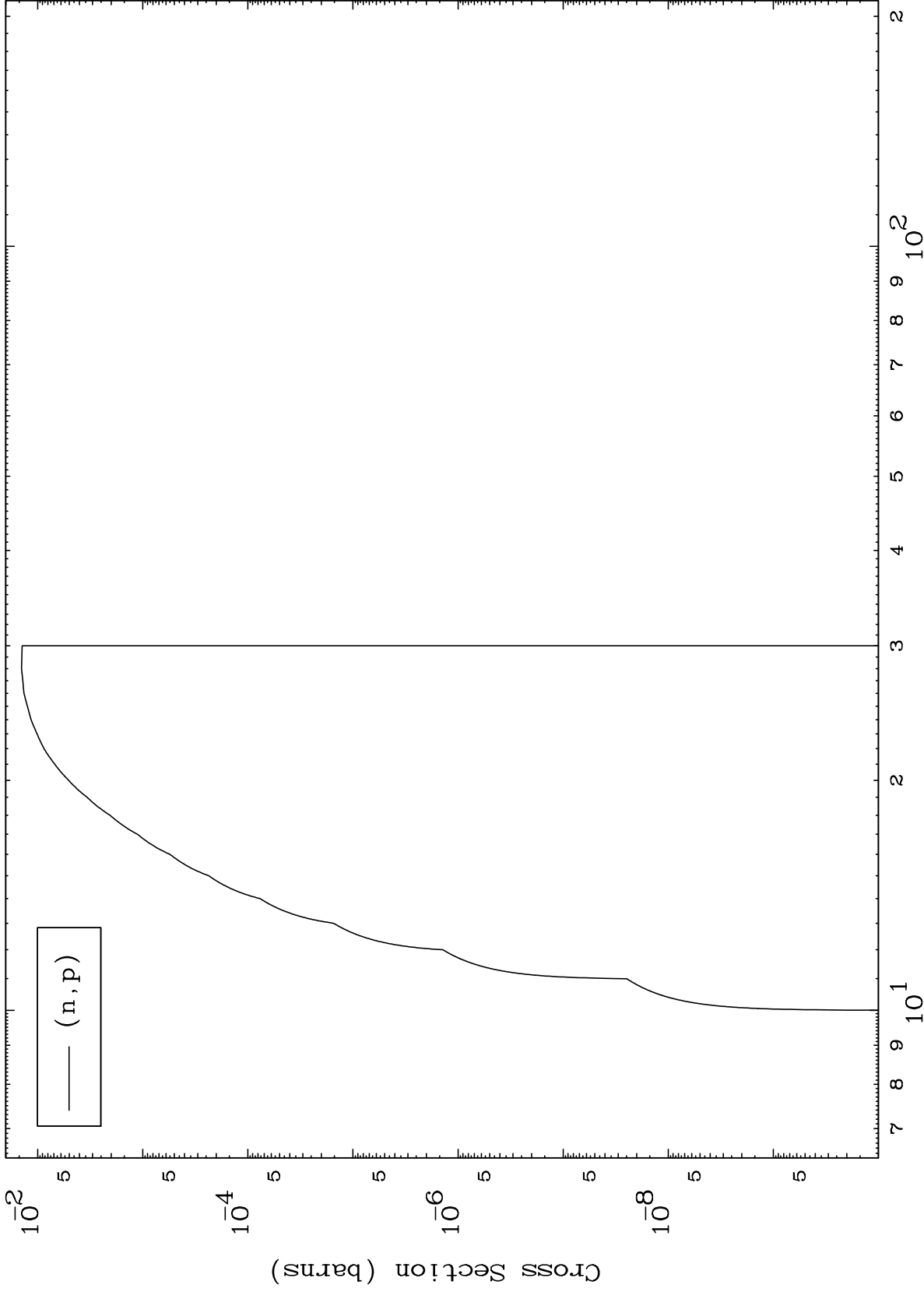


MAT 5047

( $\alpha, p$ ) Levels

50-Sn-119m

0 Kelvin Cross Sections



Incident Energy (MeV)

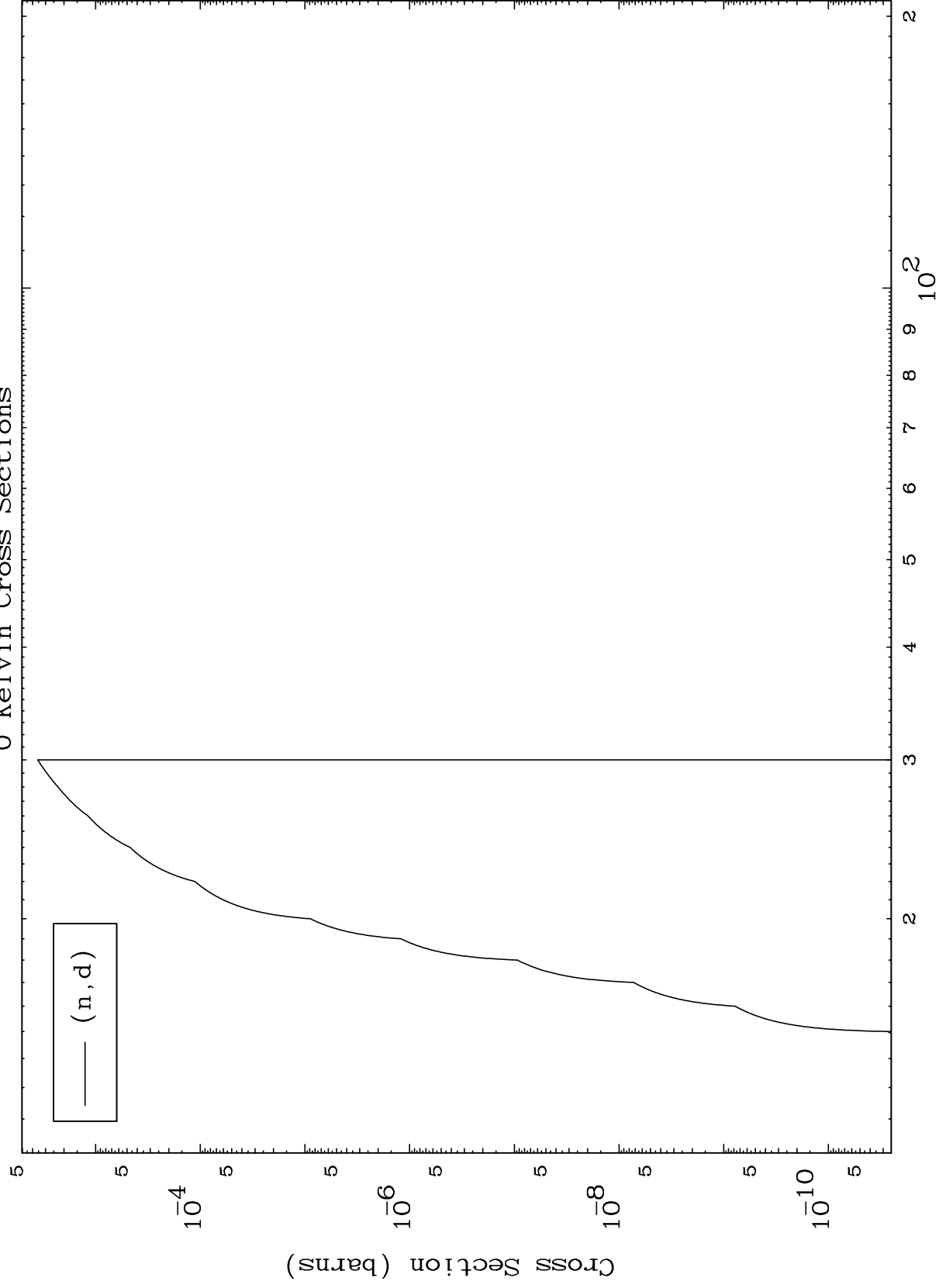
50-Sn-119m

6

MAT 5047

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

50-Sn-119m



7

Incident Energy (MeV)

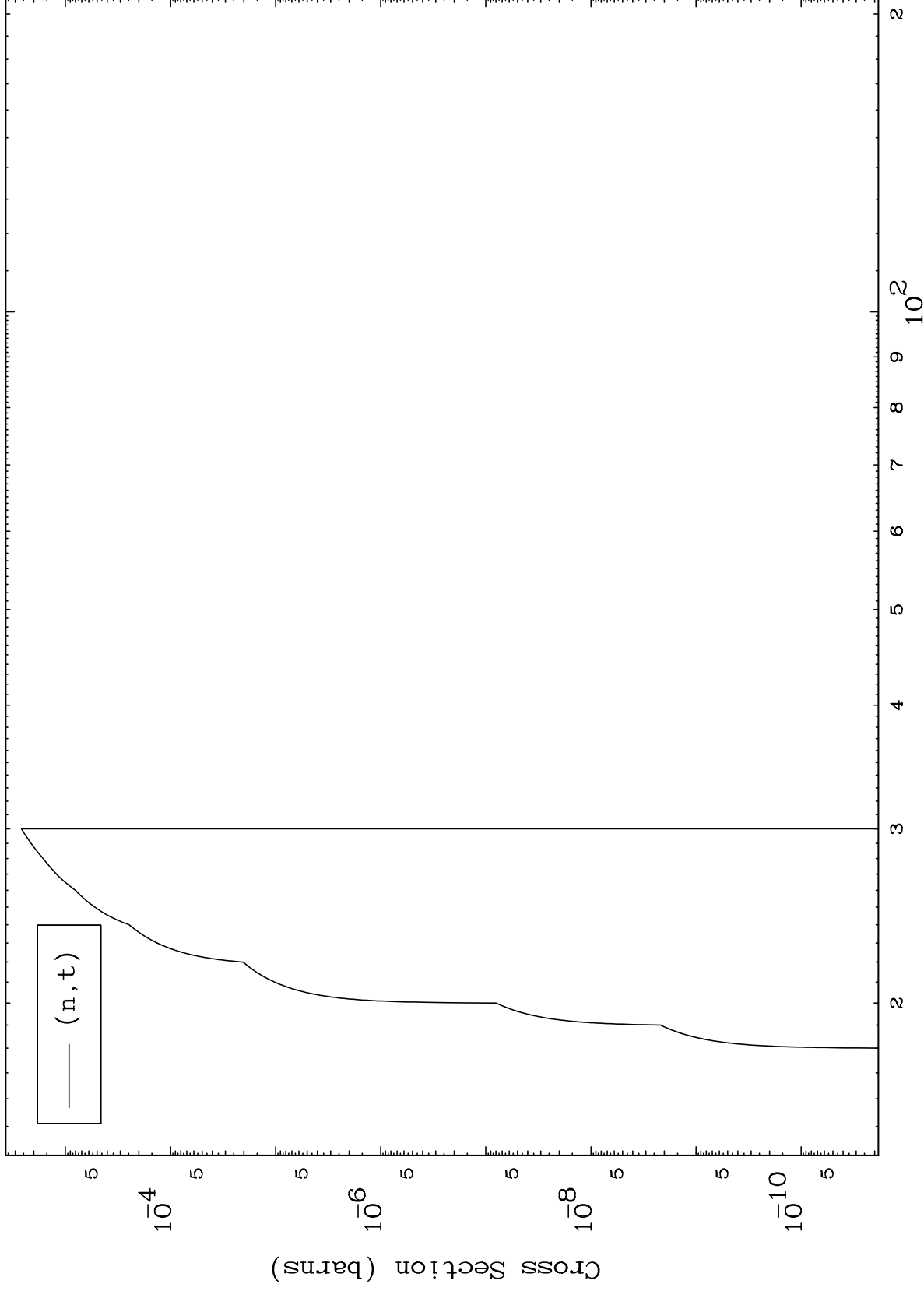
50-Sn-119m



MAT 5047

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

50-Sn-119m



8

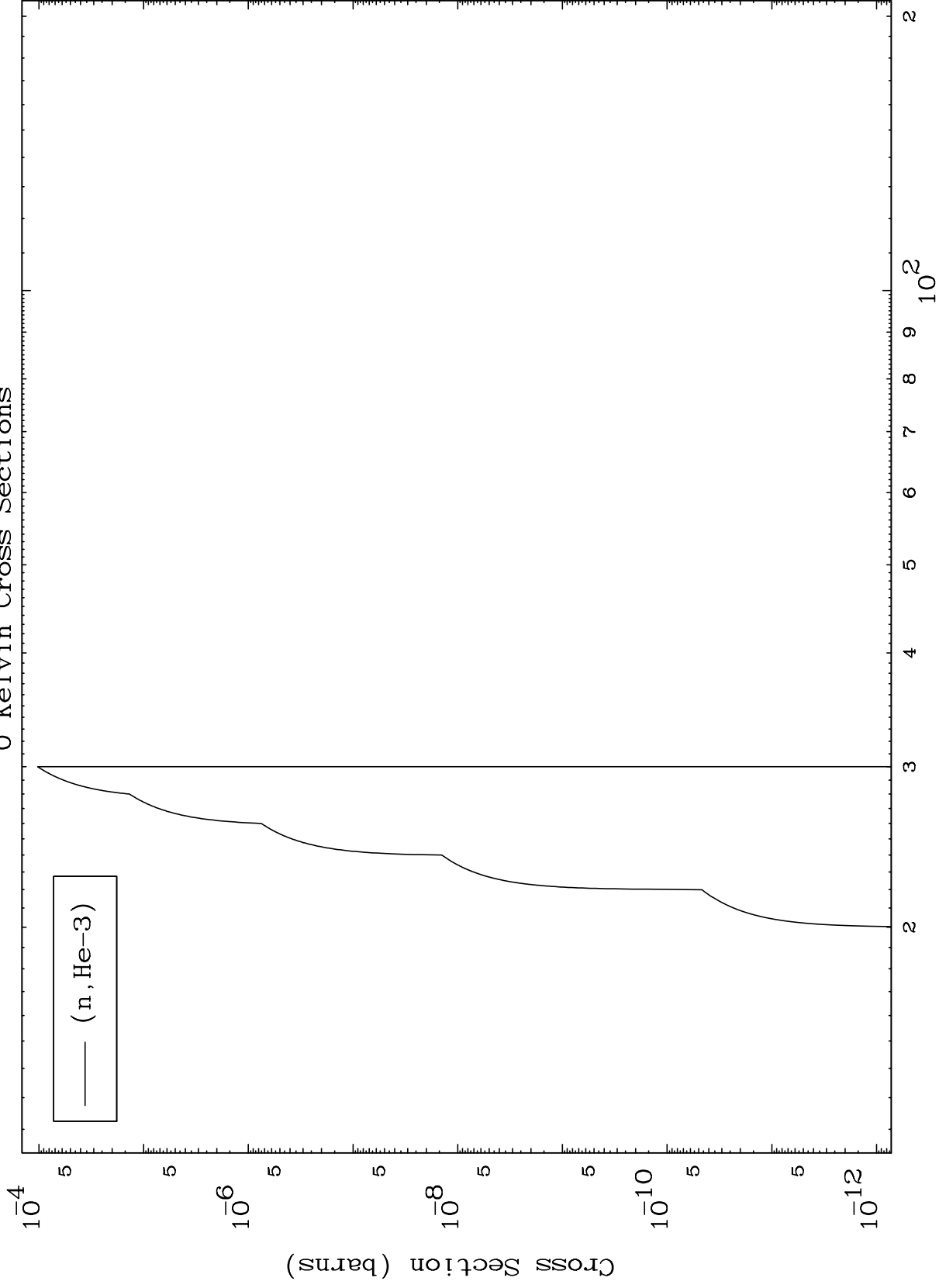
Incident Energy (MeV)

50-Sn-119m

MAT 5047

50-Sn-119m

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

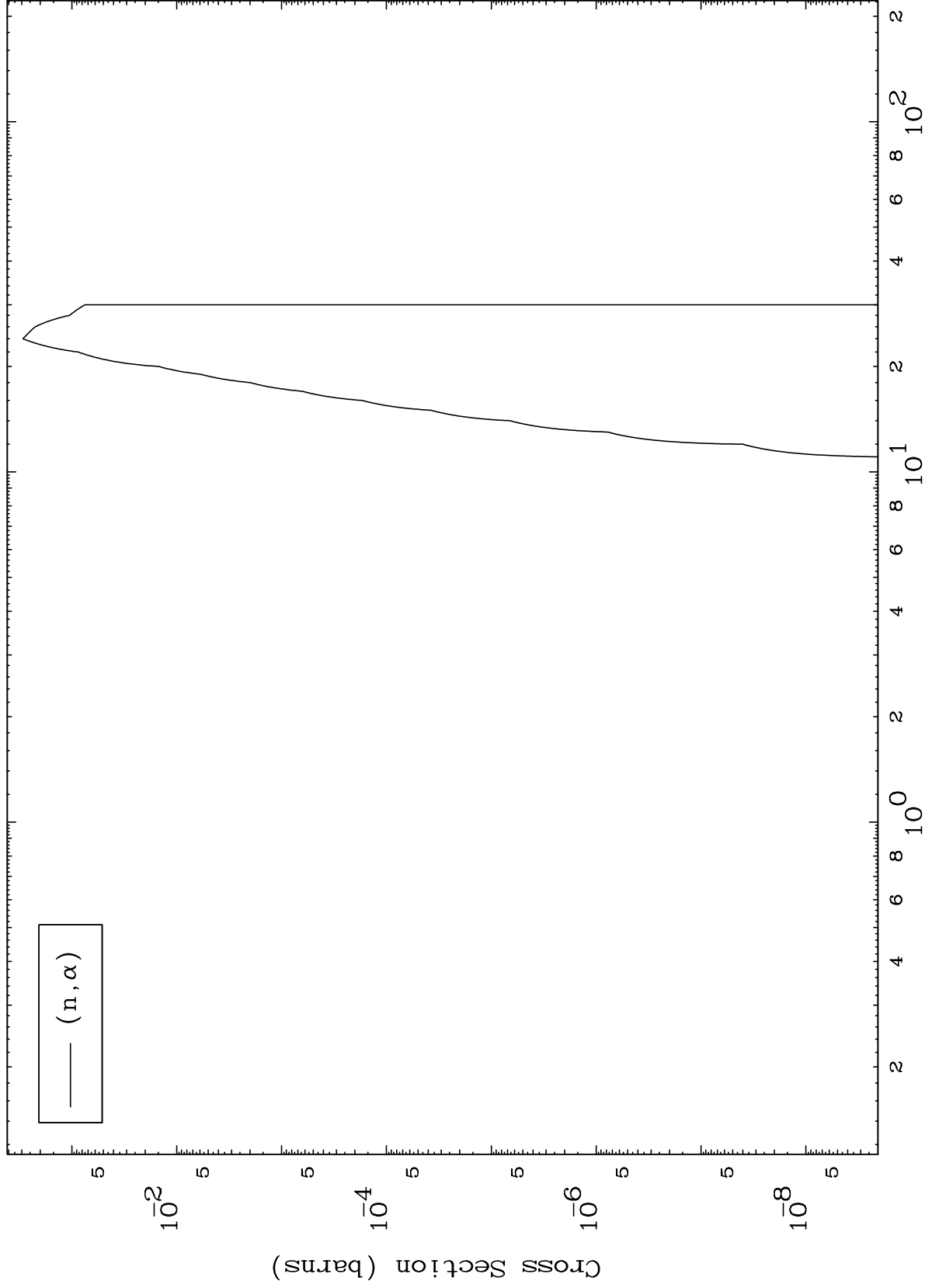


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( $\alpha, \alpha$ ) Levels

50-Sn-119m

0 Kelvin Cross Sections



(n,  $\alpha$ )

10

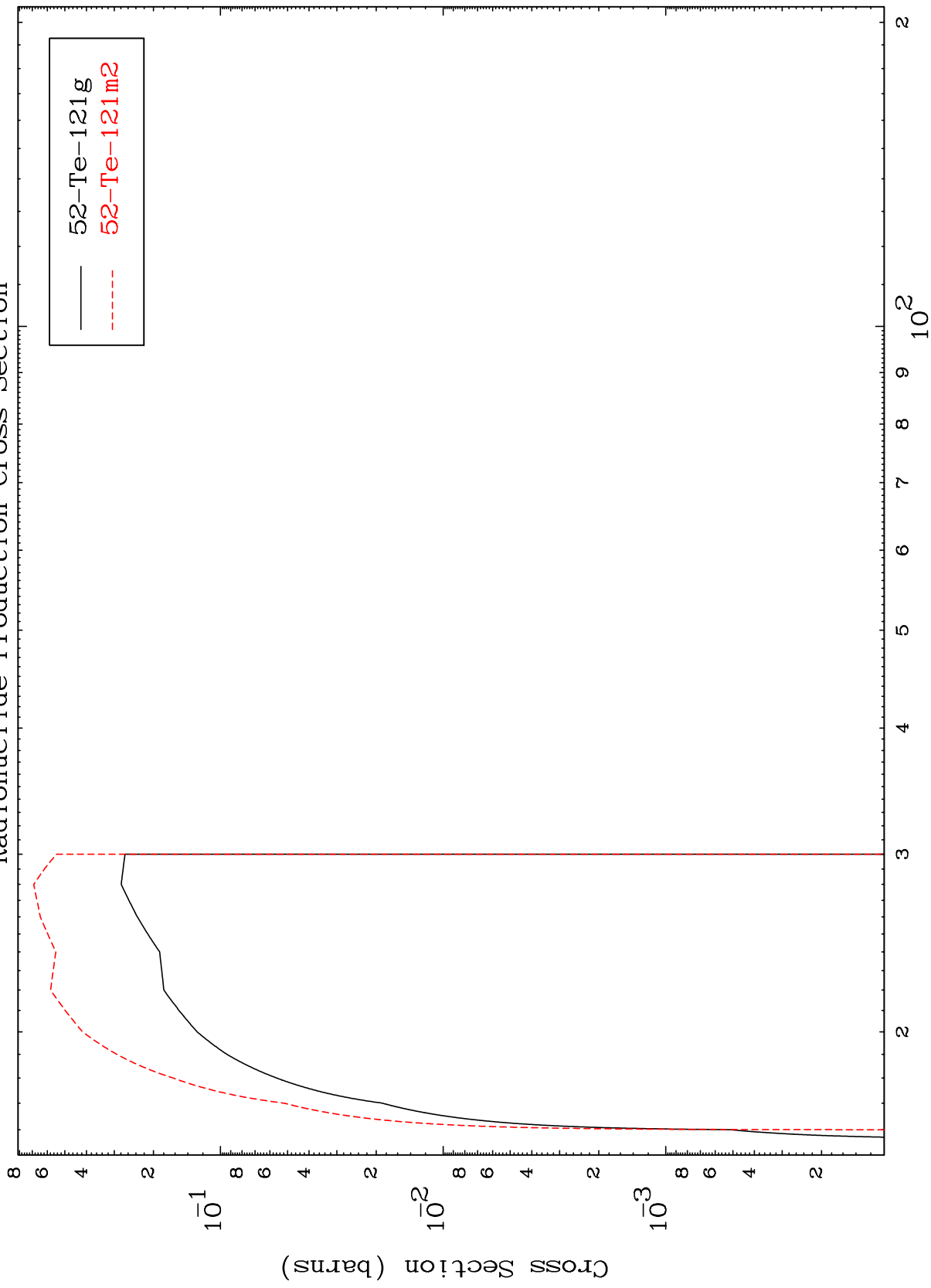
Incident Energy (MeV)

50-Sn-119m

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50-Sn-119m

(n,2n)  
Radionuclide Production Cross Section



50-Sn-119m

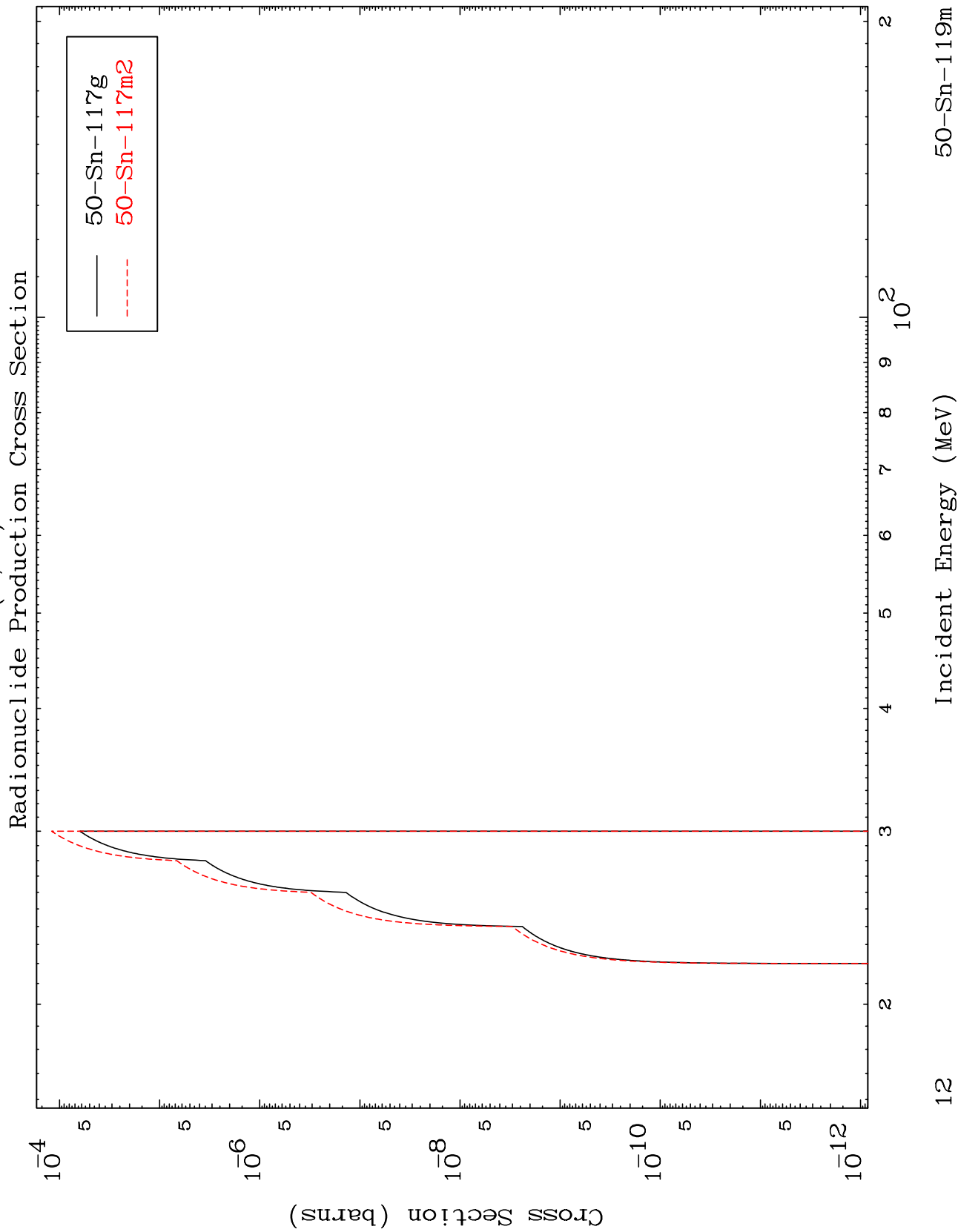
Incident Energy (MeV)

11

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(n,2n)  $\alpha$

50-Sn-119m



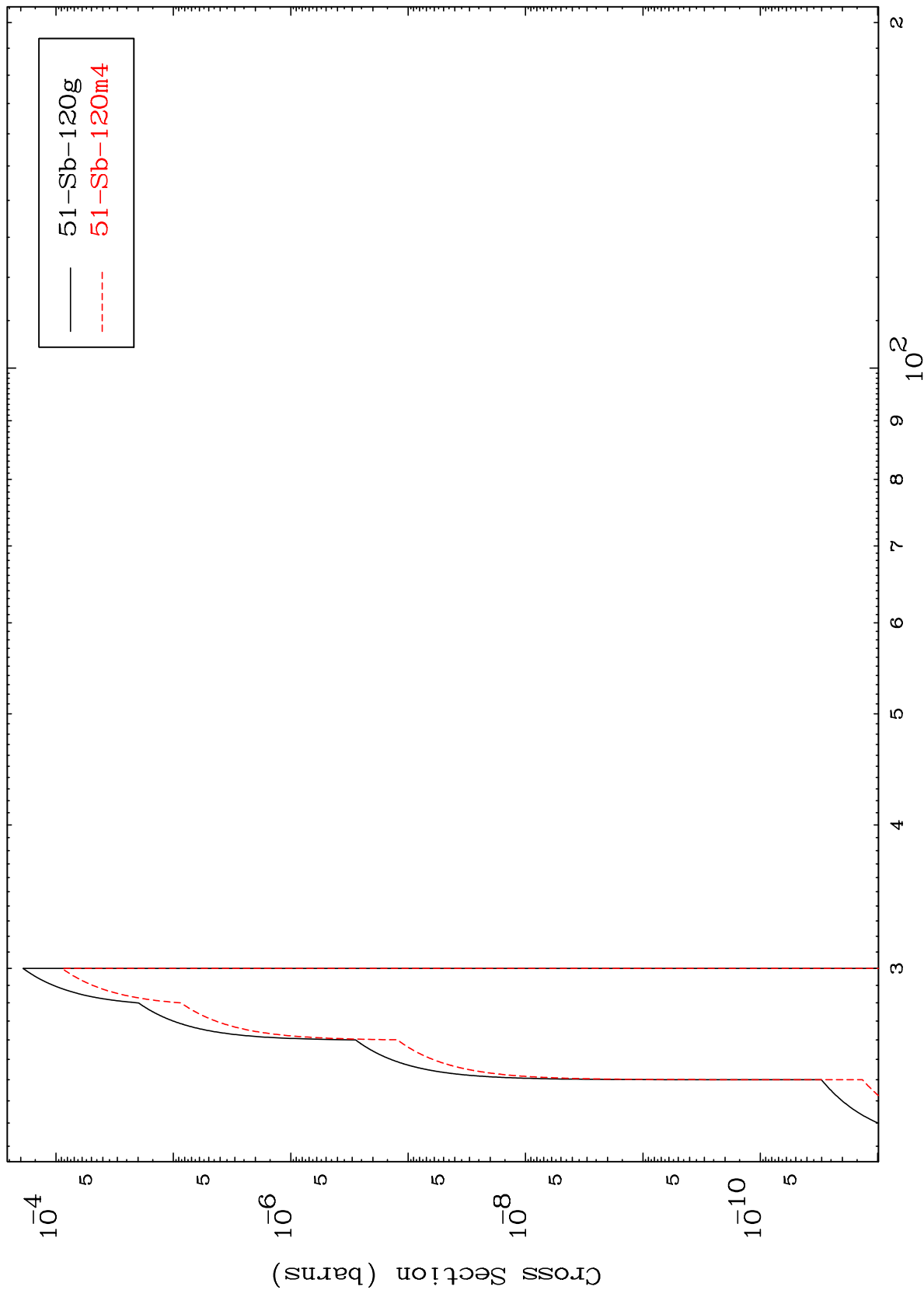
12

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(n,n') d

50-Sn-119m

Radionuclide Production Cross Section



13

Incident Energy (MeV)

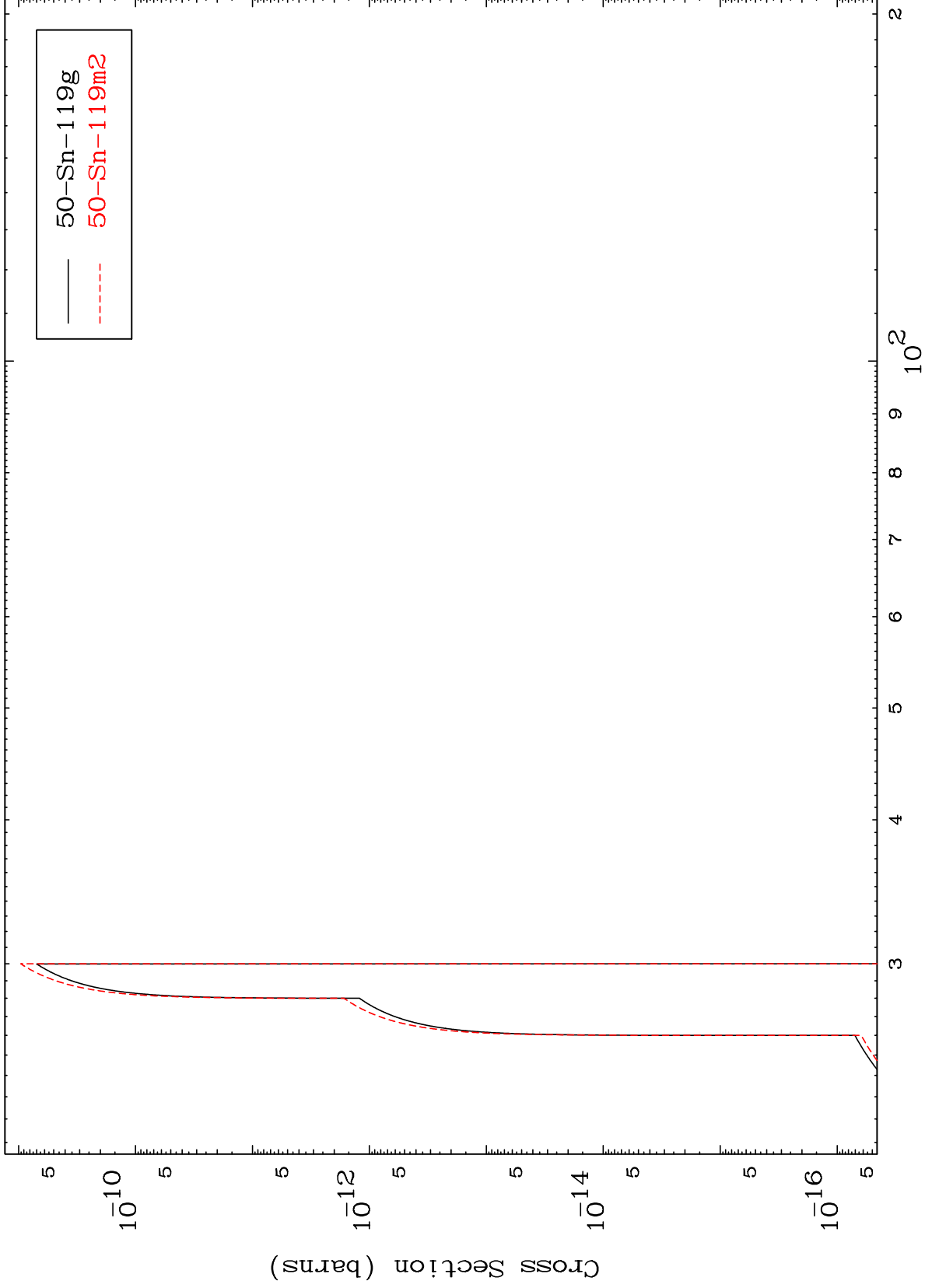
50-Sn-119m

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(n,n') He-3

50-Sn-119m

Radionuclide Production Cross Section



14

Incident Energy (MeV)

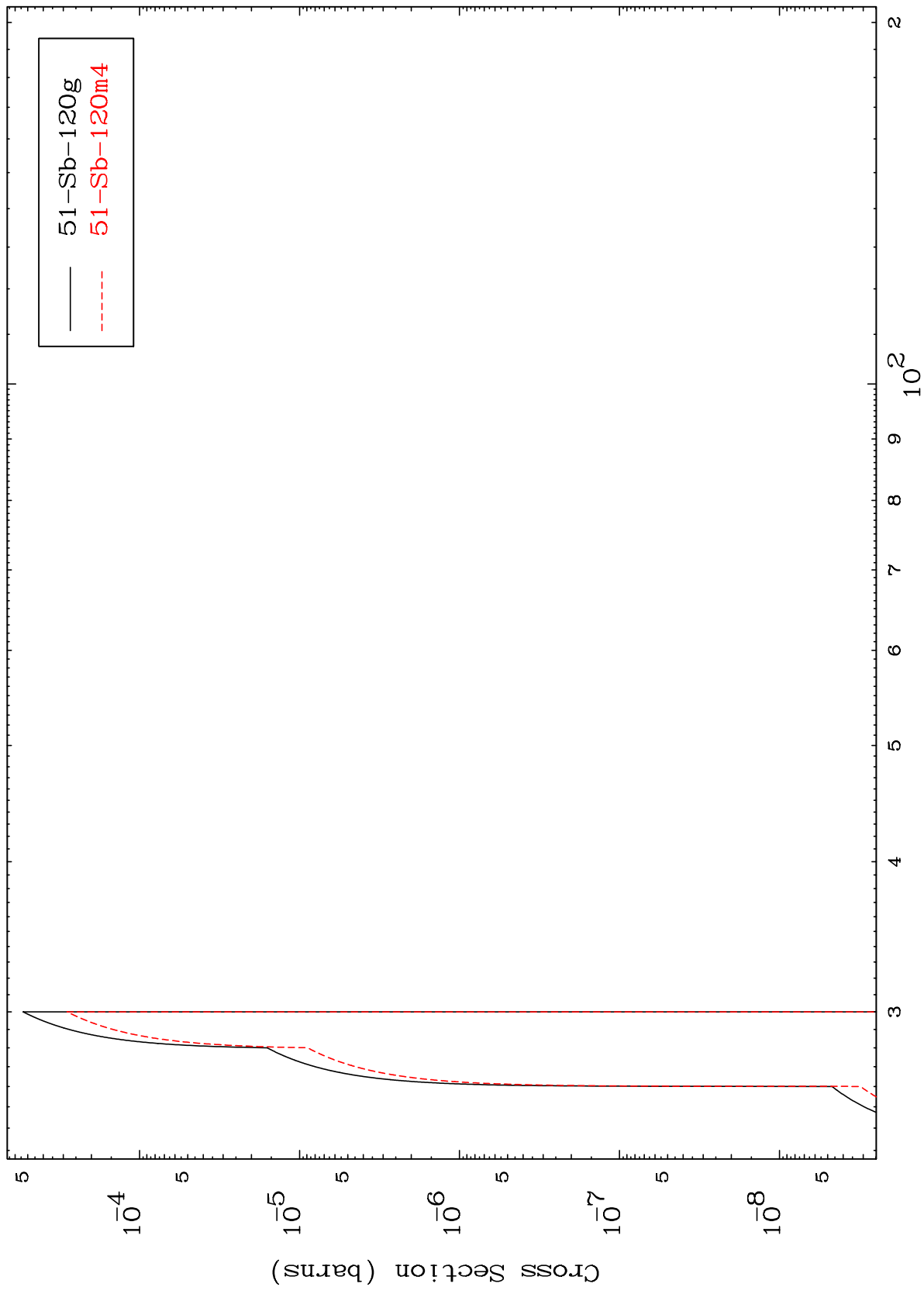
50-Sn-119m

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(n,2n) p

50-Sn-119m

Radionuclide Production Cross Section



15

Incident Energy (MeV)

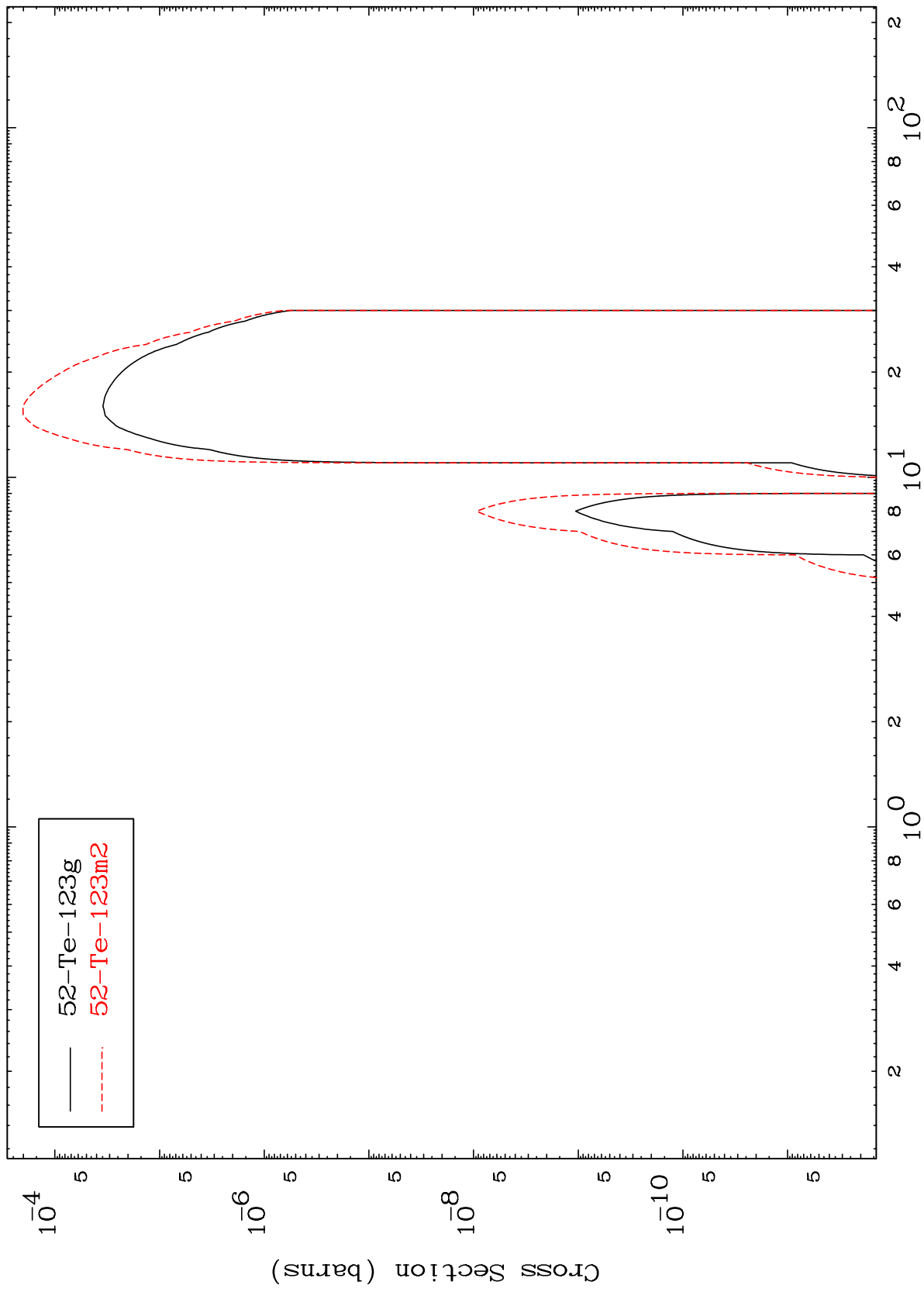
50-Sn-119m



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50-Sn-119m

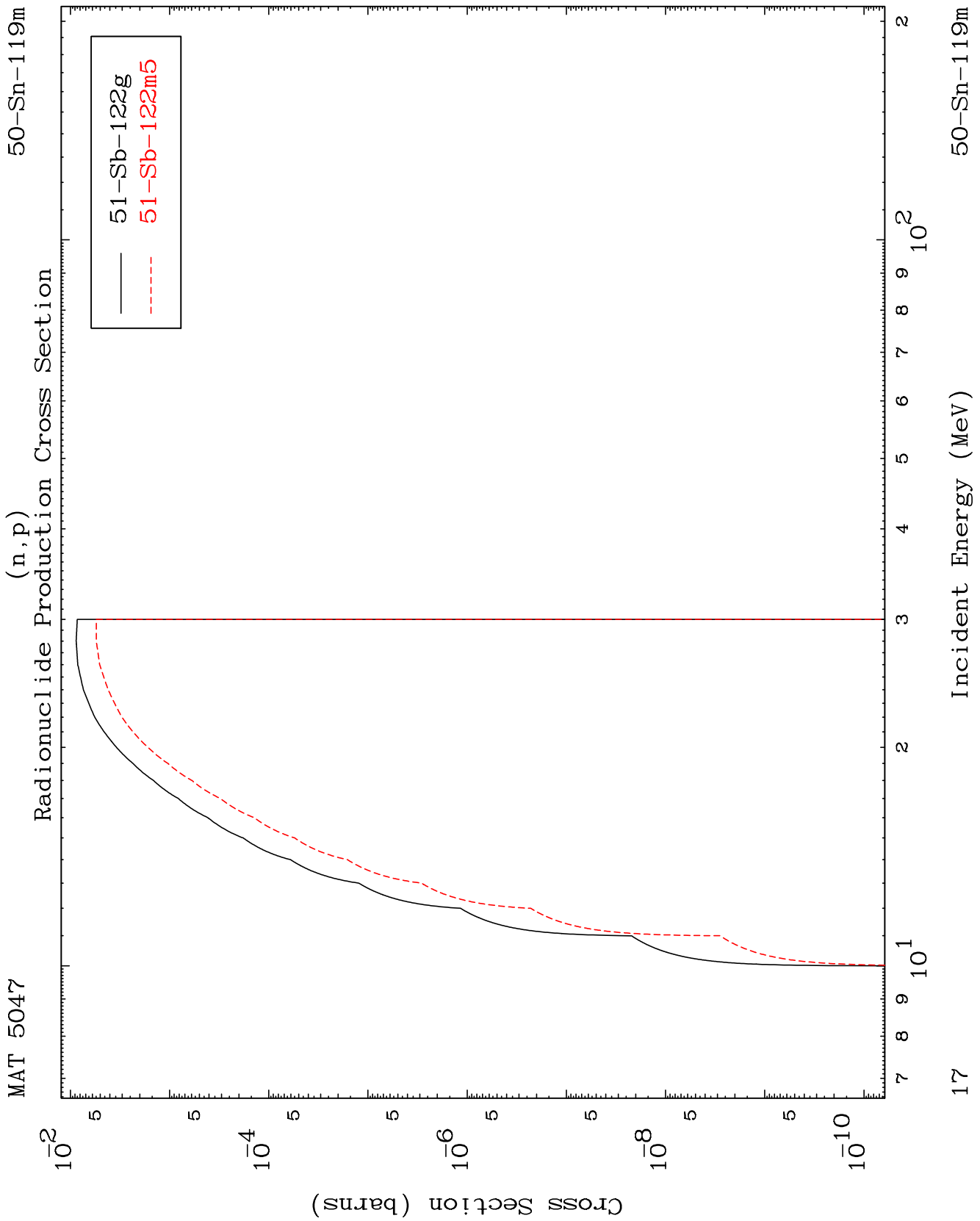
(n,  $\gamma$ )  
Radionuclide Production Cross Section



16

Incident Energy (MeV)

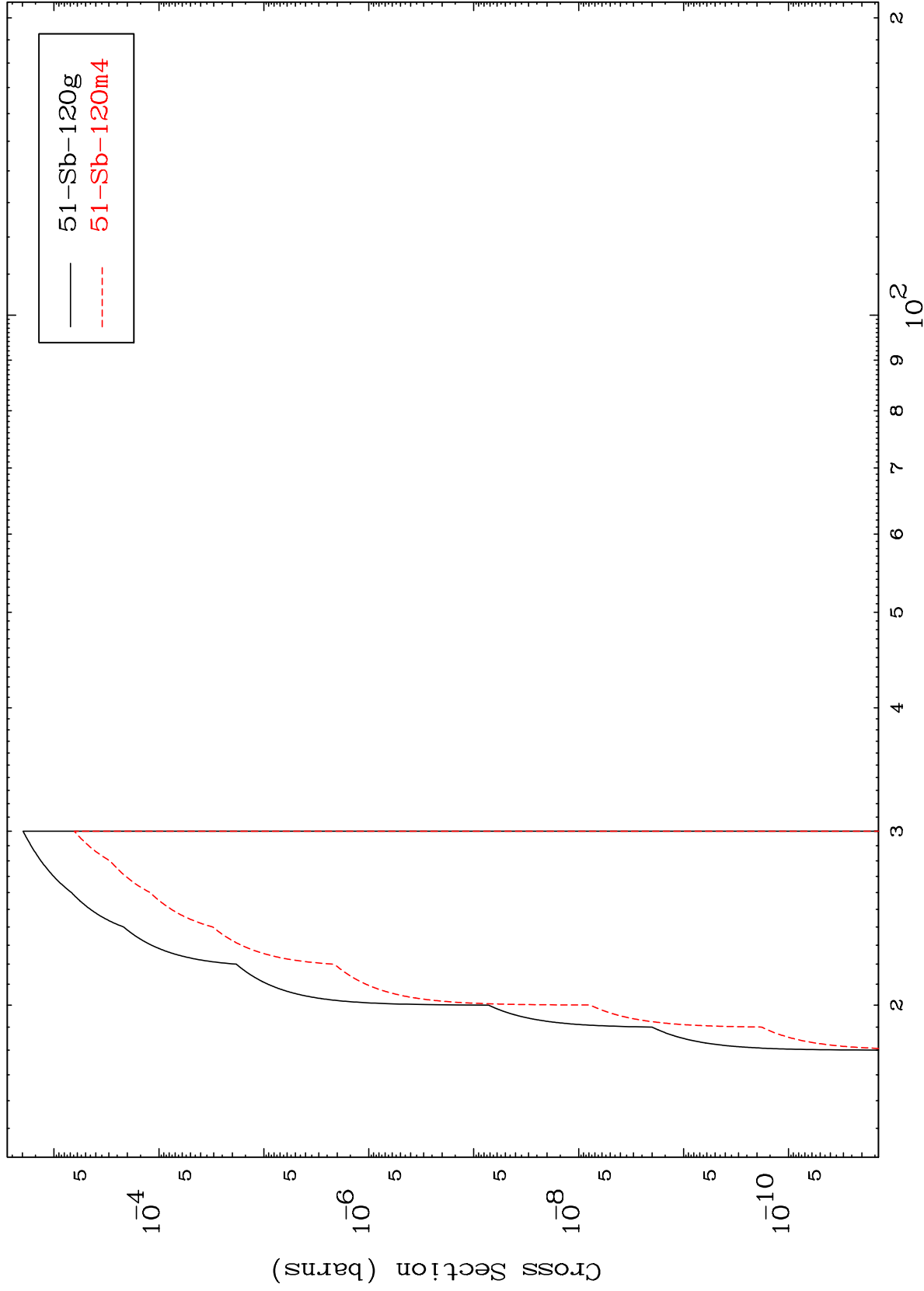
50-Sn-119m



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50-Sn-119m

(n,t)  
Radionuclide Production Cross Section



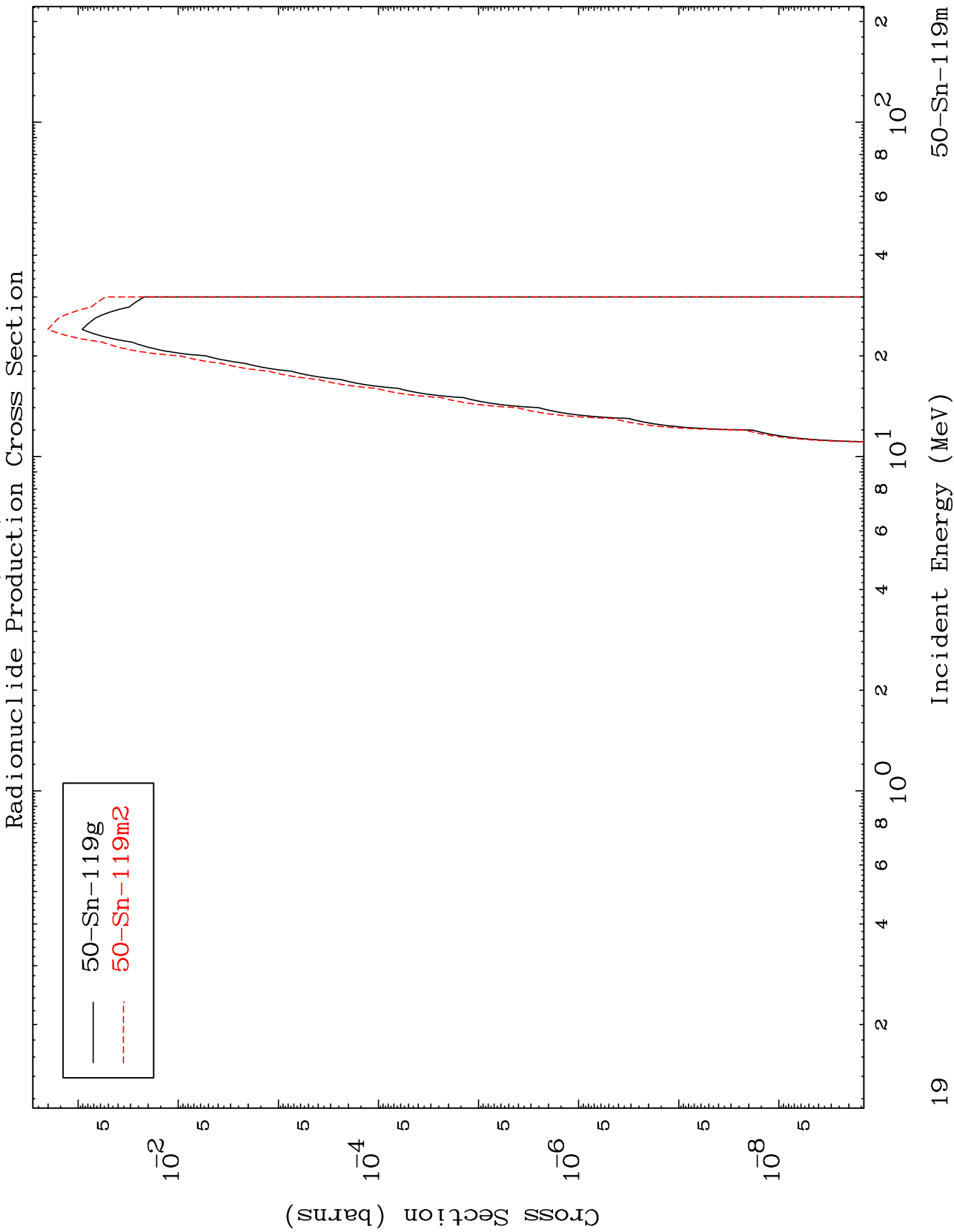
18

Incident Energy (MeV)

50-Sn-119m

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50-Sn-119m

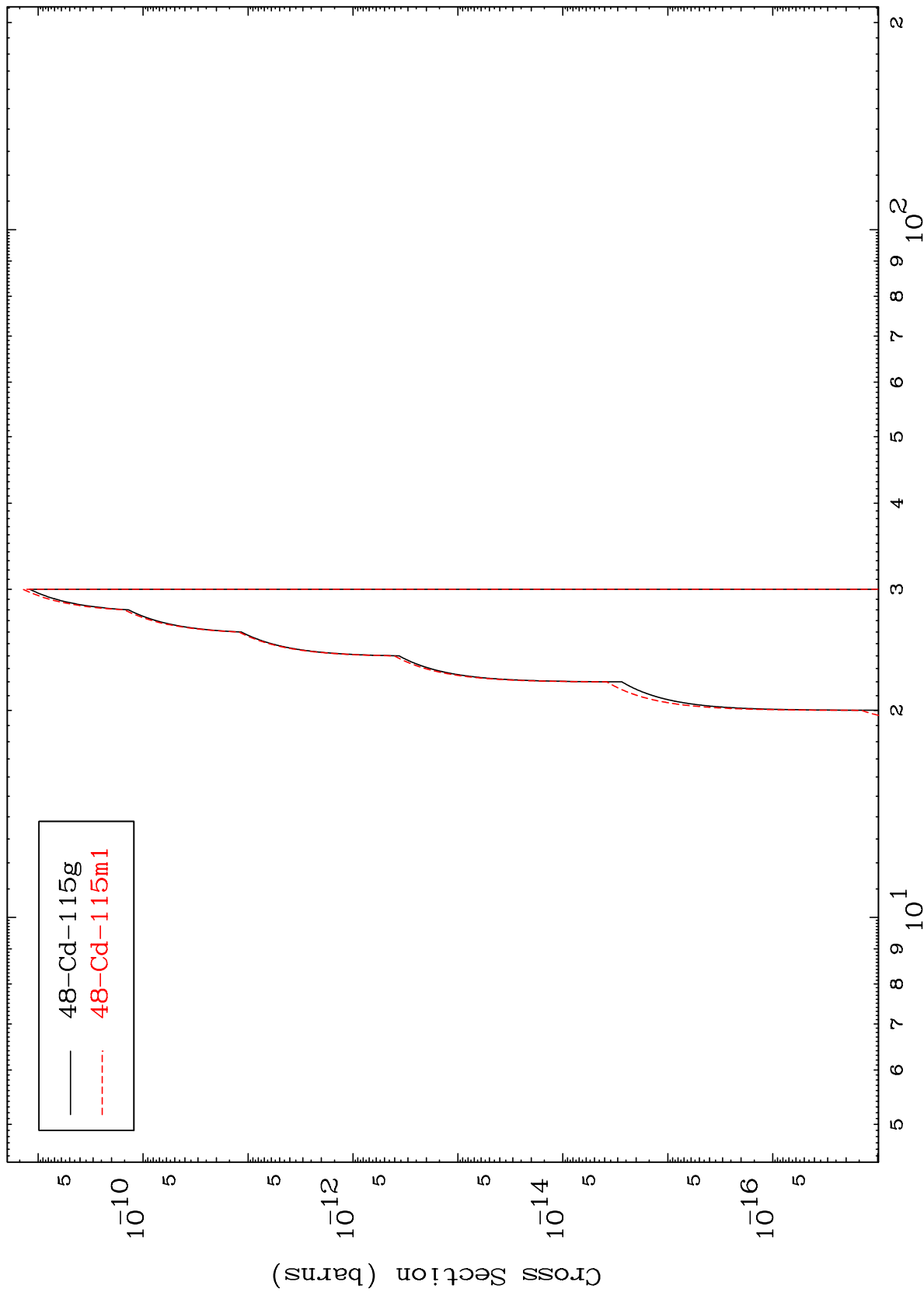


50-Sn-119g  
50-Sn-119m2

MAT 5047

50-Sn-119m

Radionuclide Production Cross Section  
(n, 2 $\alpha$ )



50-Sn-119m

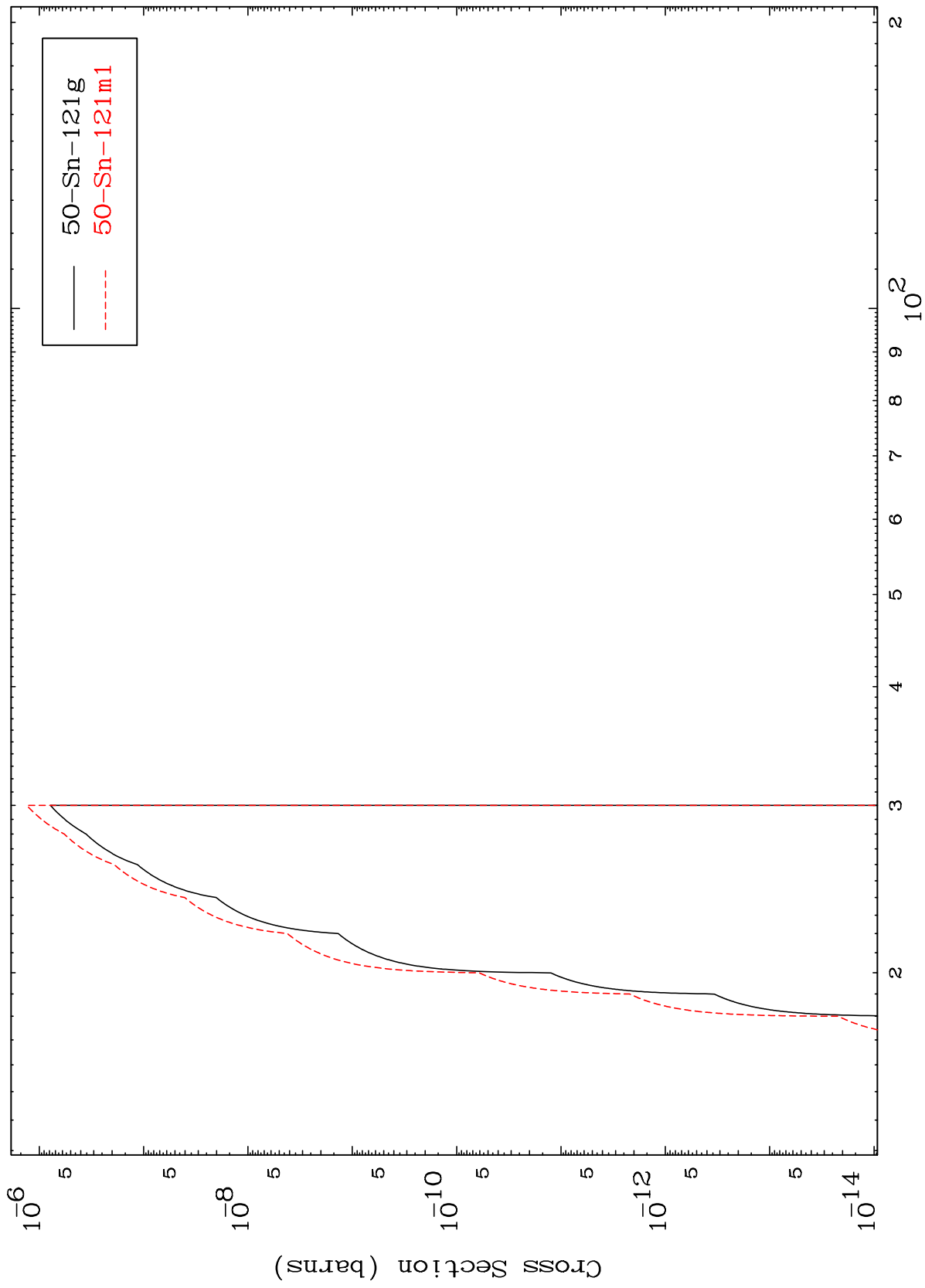
Incident Energy (MeV)

20

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50-Sn-119m

(n,2p)  
Radionuclide Production Cross Section



21

Incident Energy (MeV)

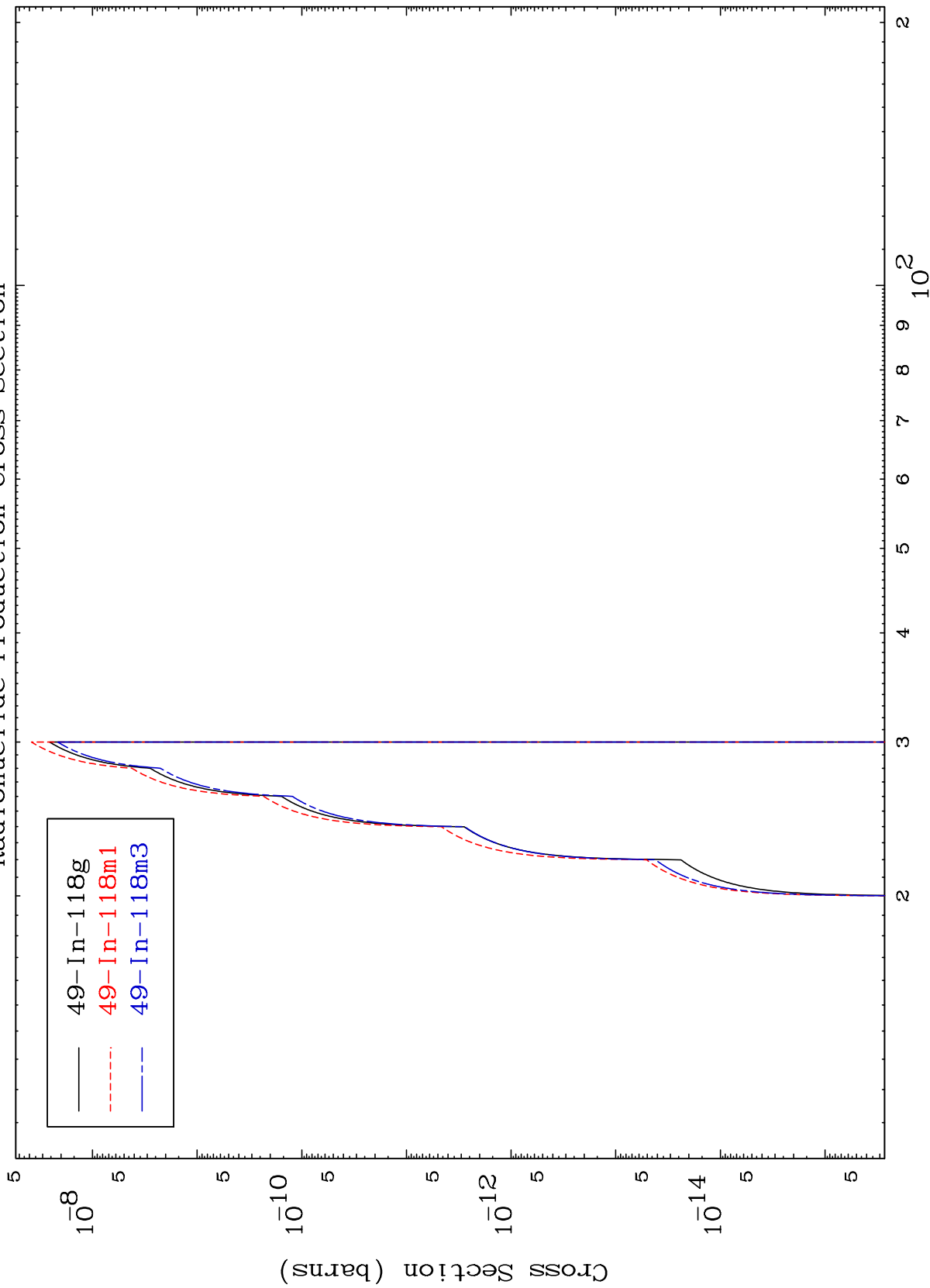
50-Sn-119m

MAT 5047

(n,p)  $\alpha$

50-Sn-119m

Radionuclide Production Cross Section



Incident Energy (MeV)

50-Sn-119m