

Program EVALPLOT  
(Version 2021-1)

by

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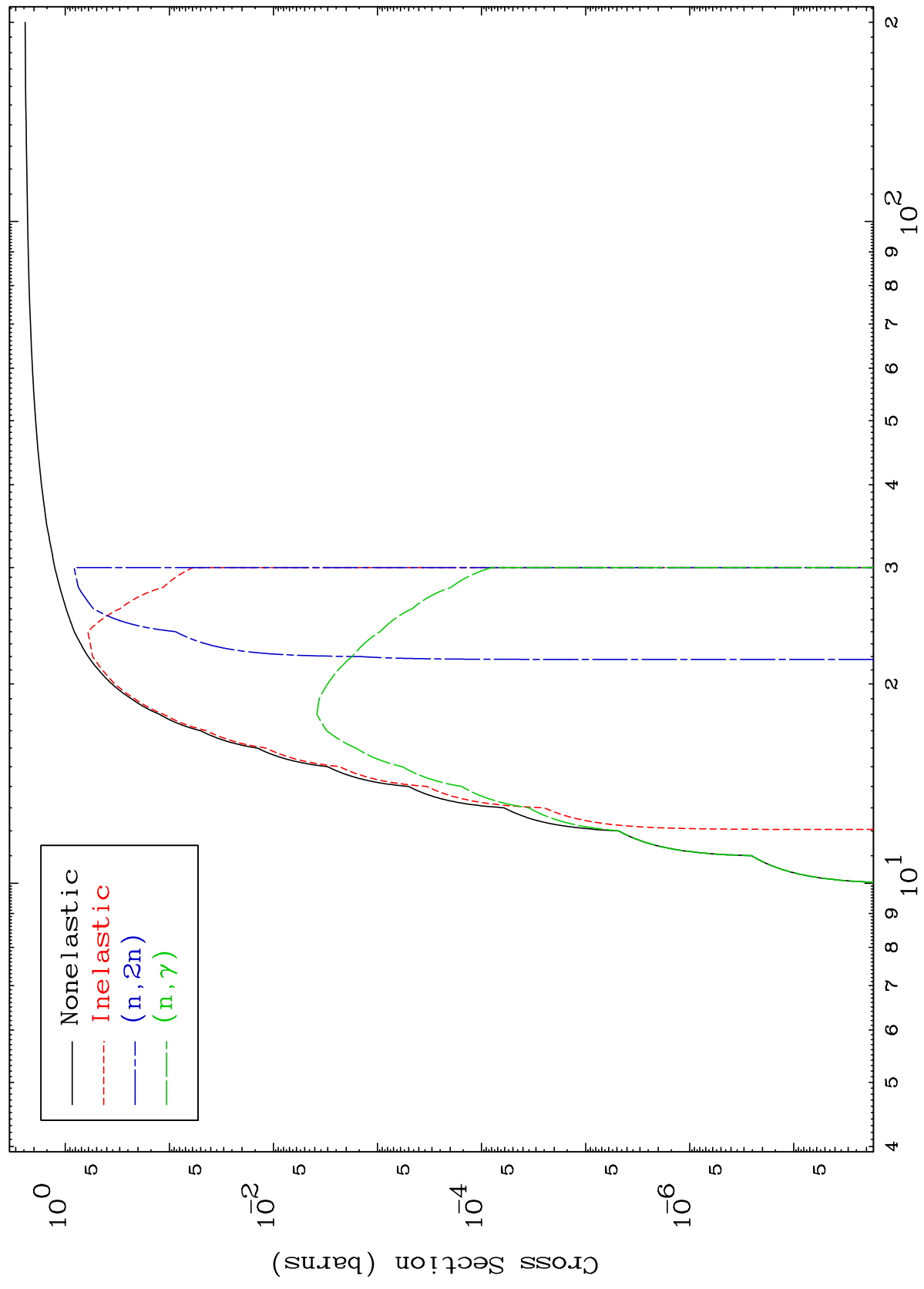
Press Mouse Button to Start

MAT 6498

0 Kelvin

$\alpha$  Major

65-Tb-150



1

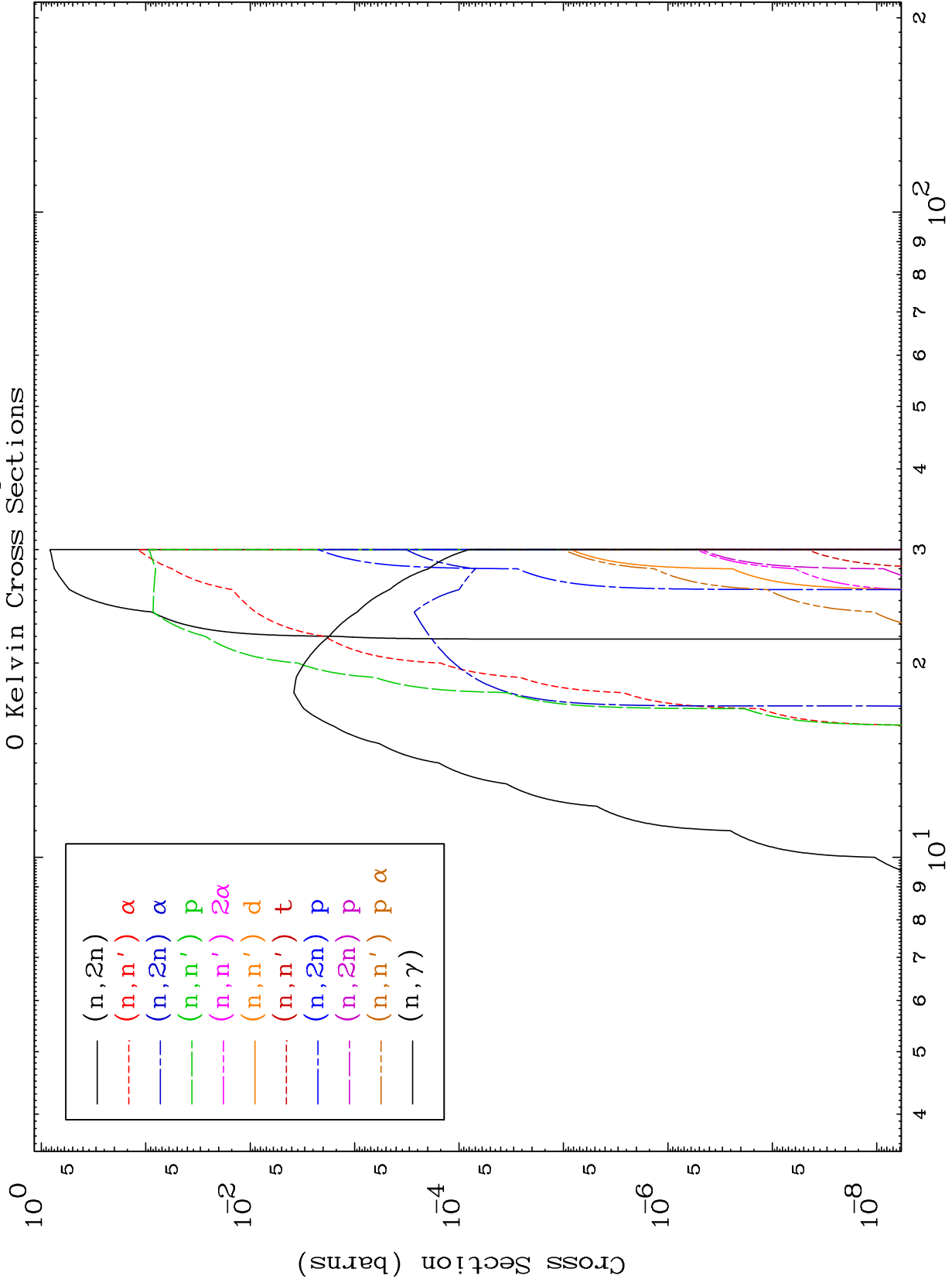
Incident Energy (MeV)

65-Tb-150

MAT 6498

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

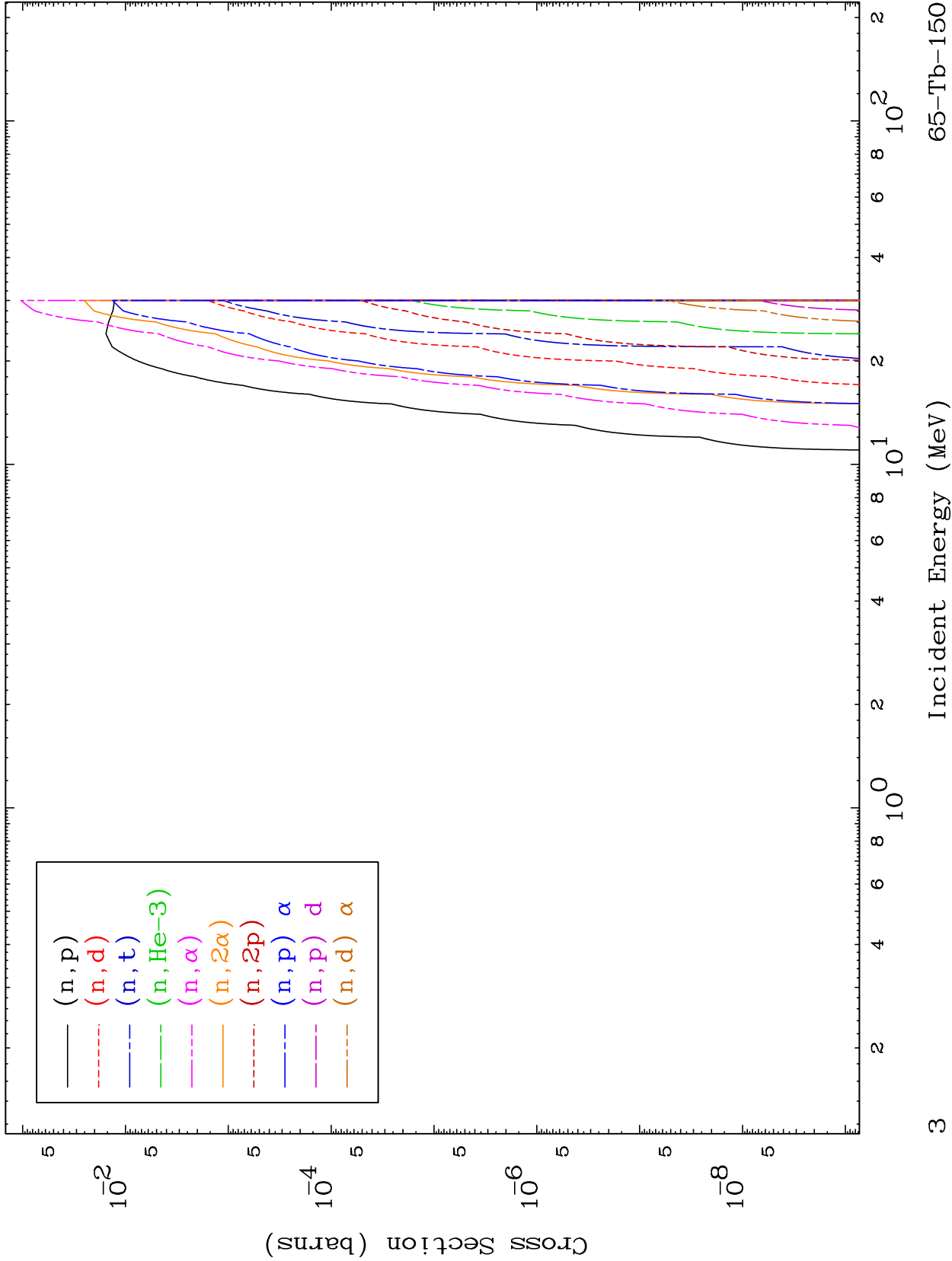
65-Tb-150

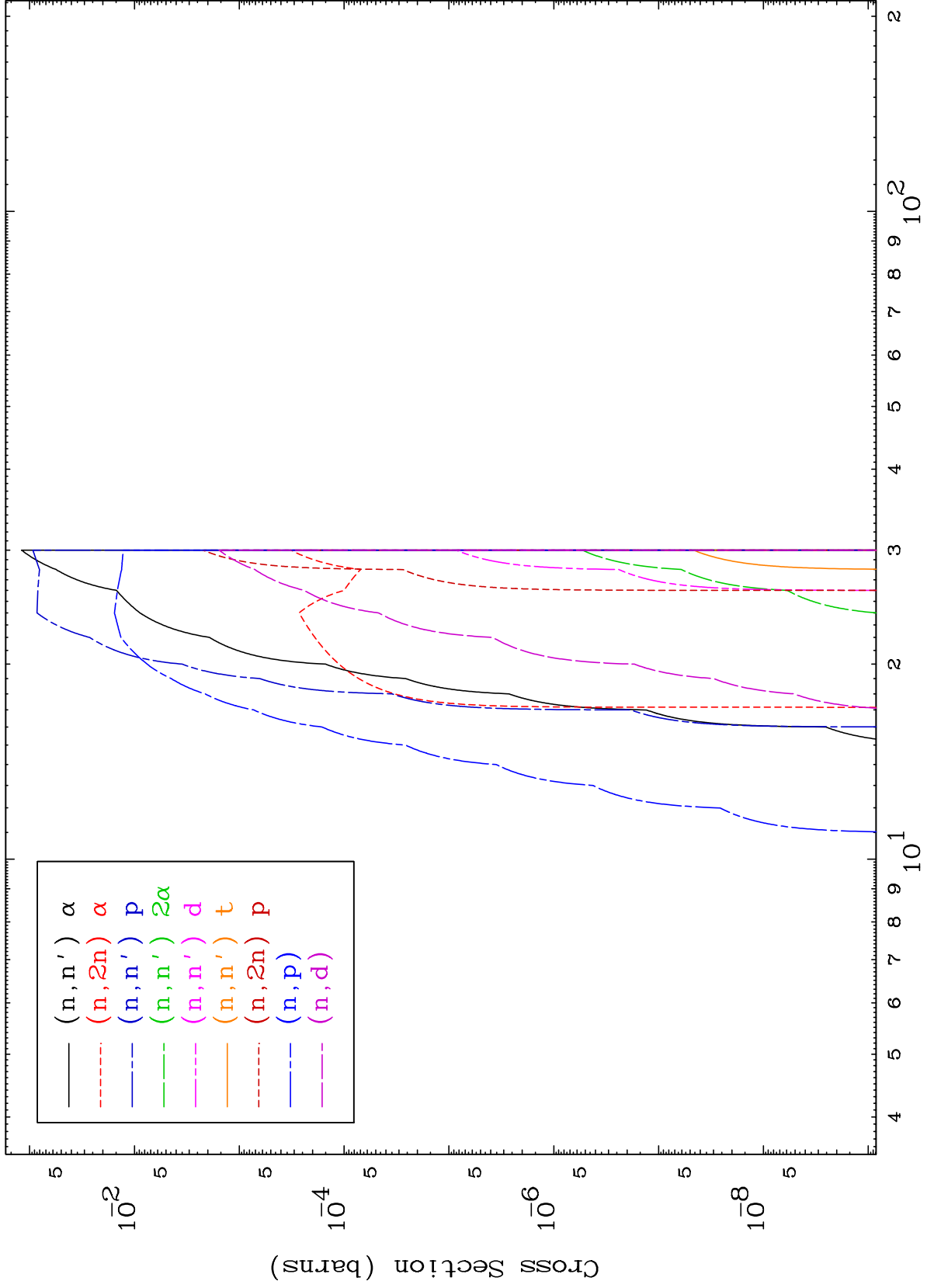


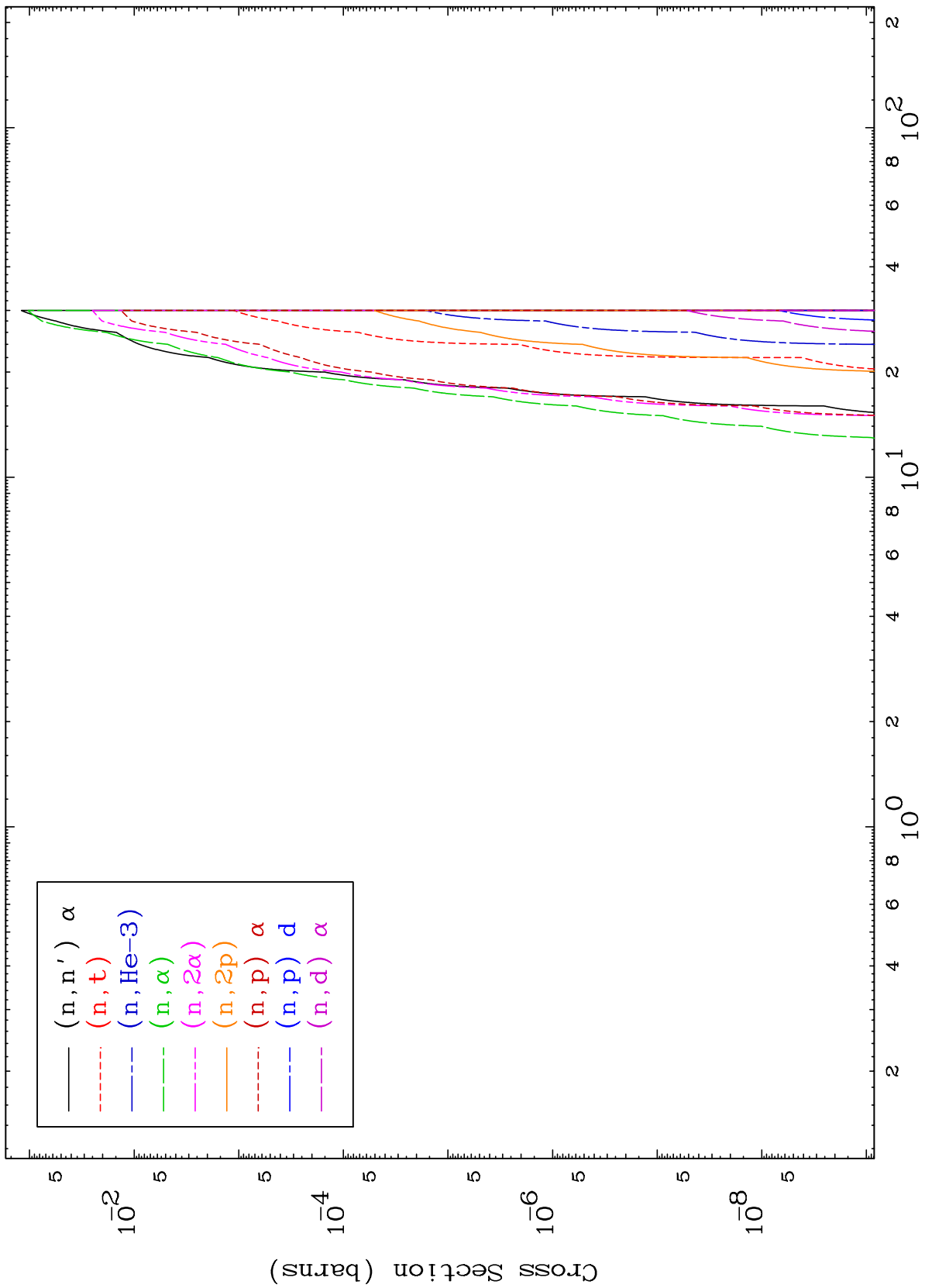
MAT 6498

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

65-Tb-150



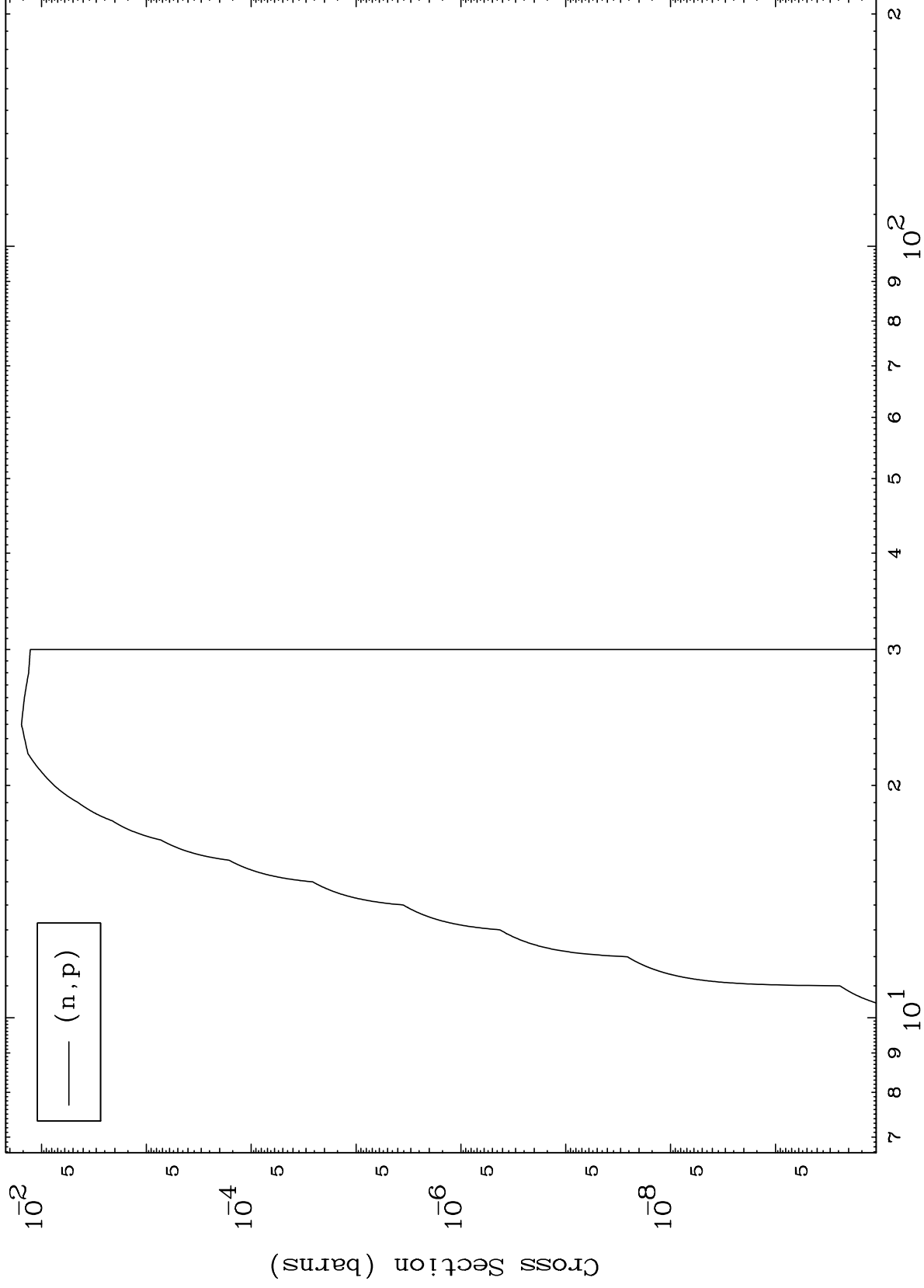




MAT 6498

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

65-Tb-150



6

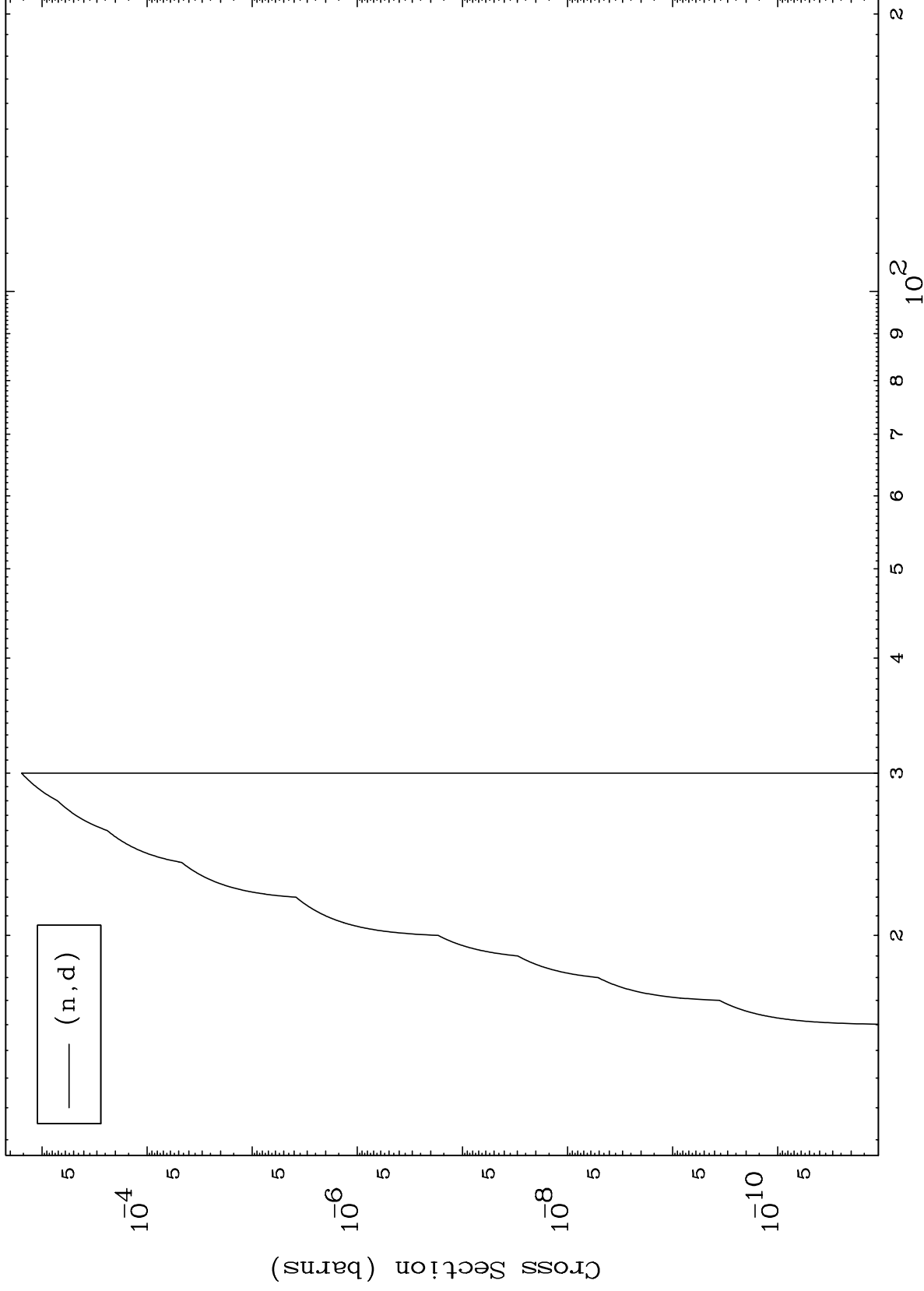
Incident Energy (MeV)

65-Tb-150

MAT 6498

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

65-Tb-150



7

Incident Energy (MeV)

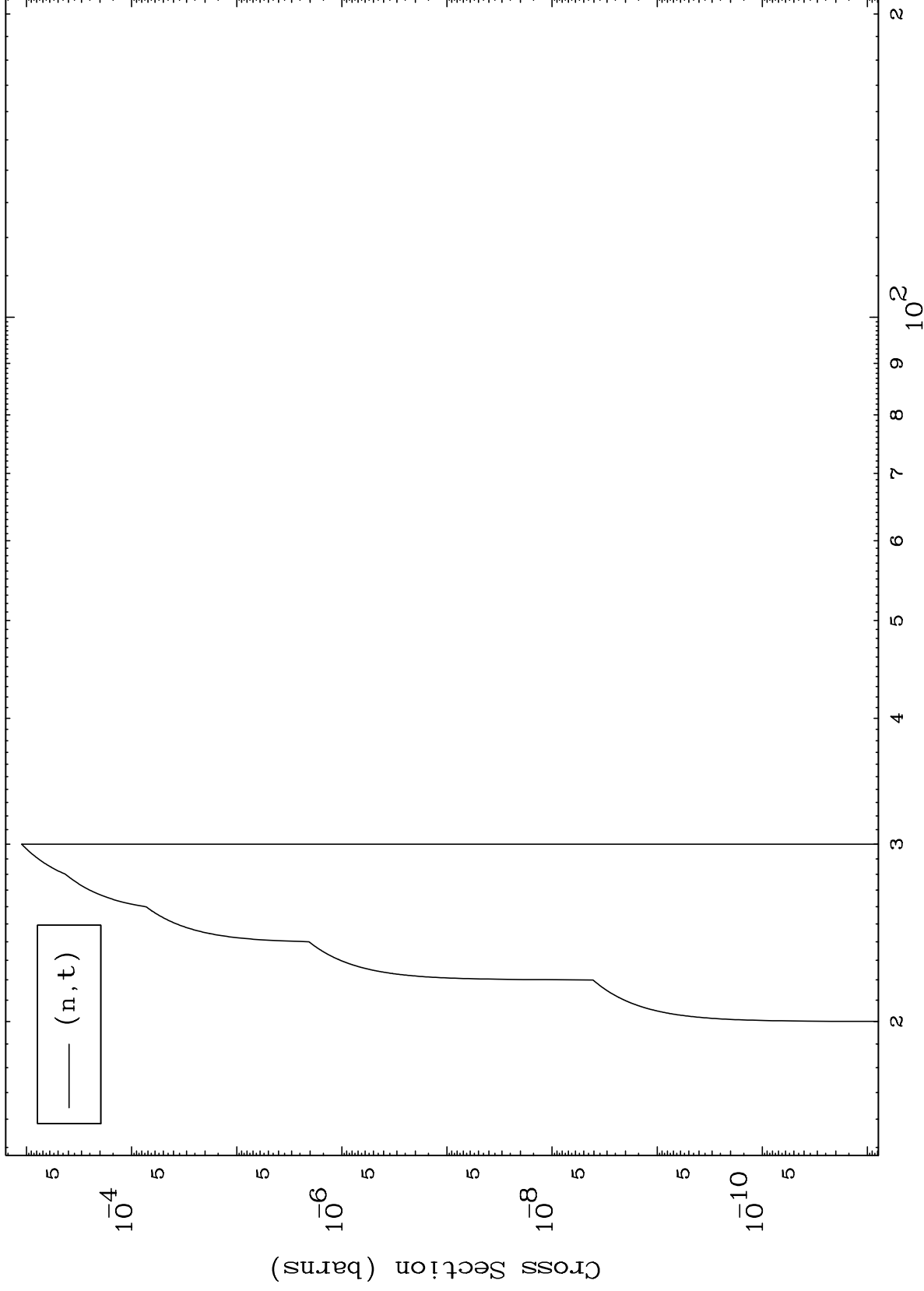
65-Tb-150

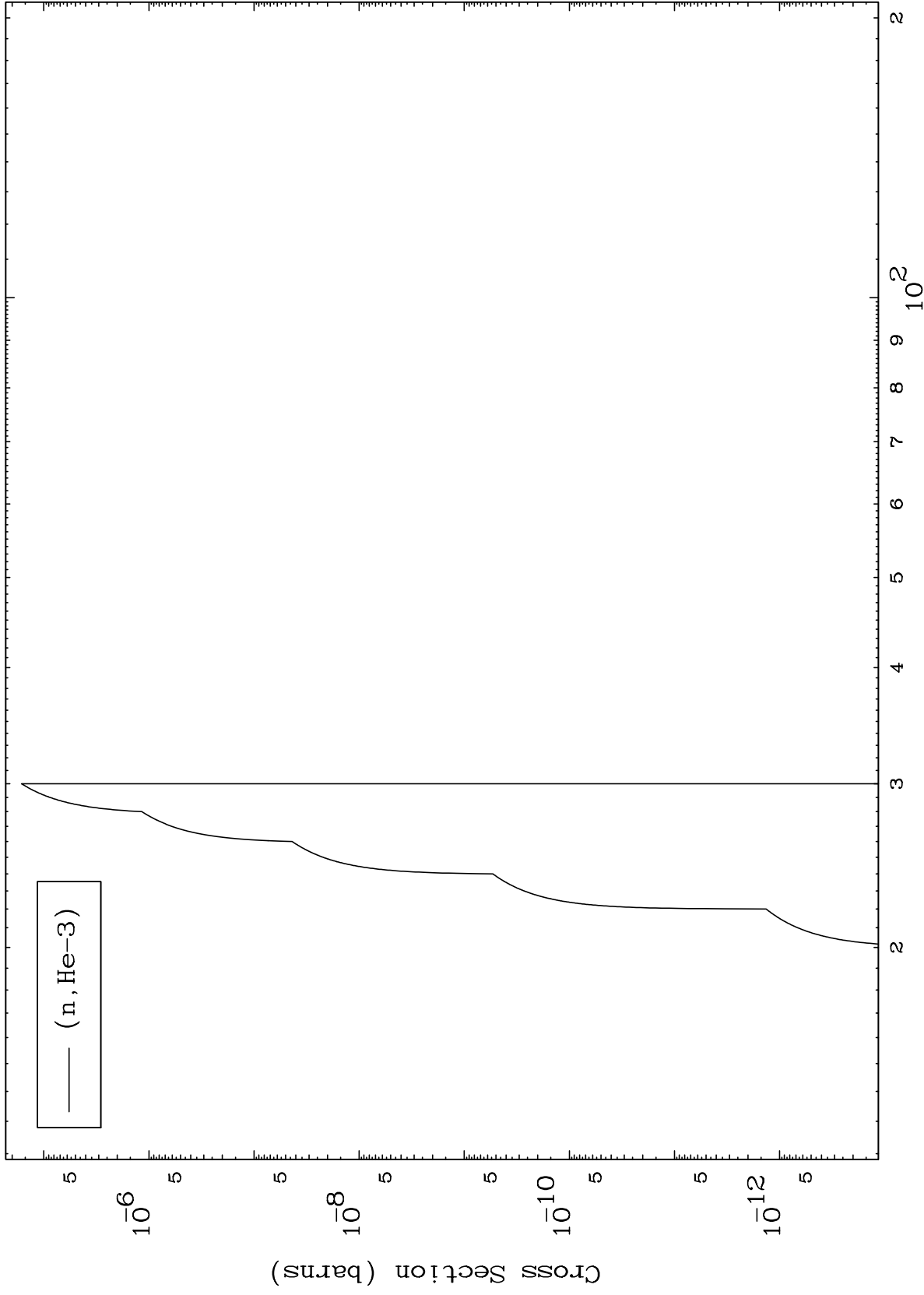


MAT 6498

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

65-Tb-150



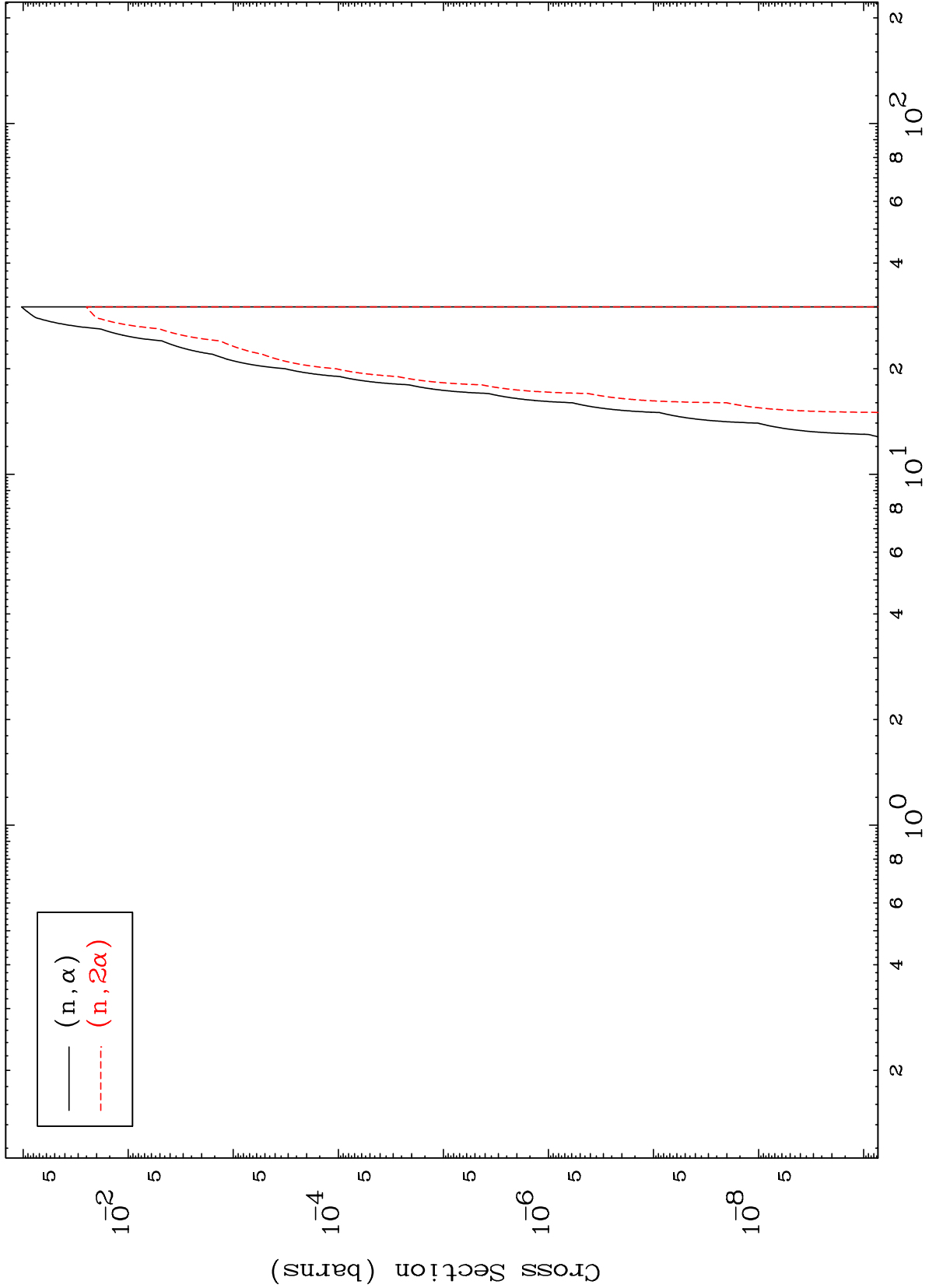


MAT 6498

( $\alpha, \alpha$ ) Levels

65-Tb-150

0 Kelvin Cross Sections

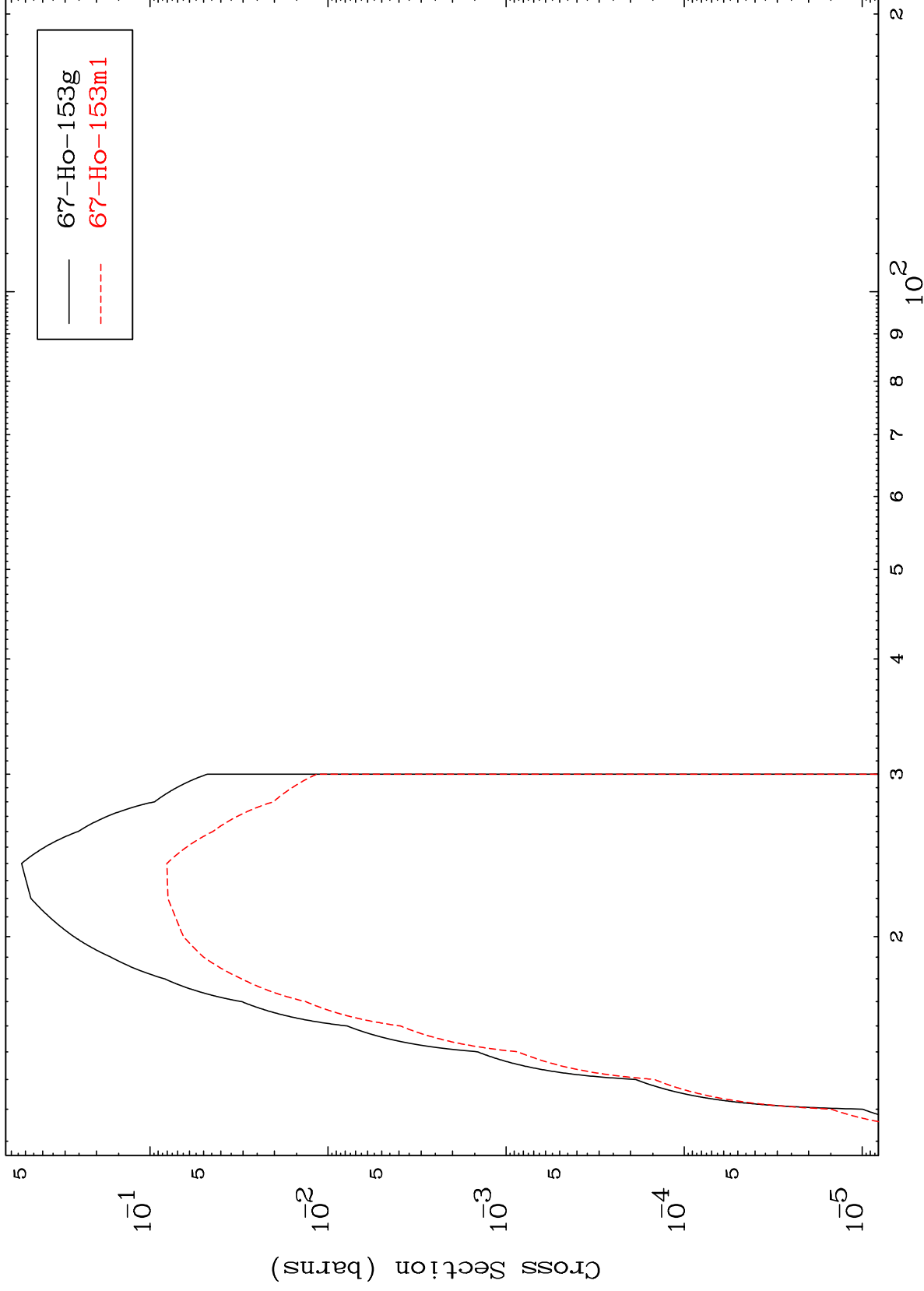


10

Incident Energy (MeV)

65-Tb-150

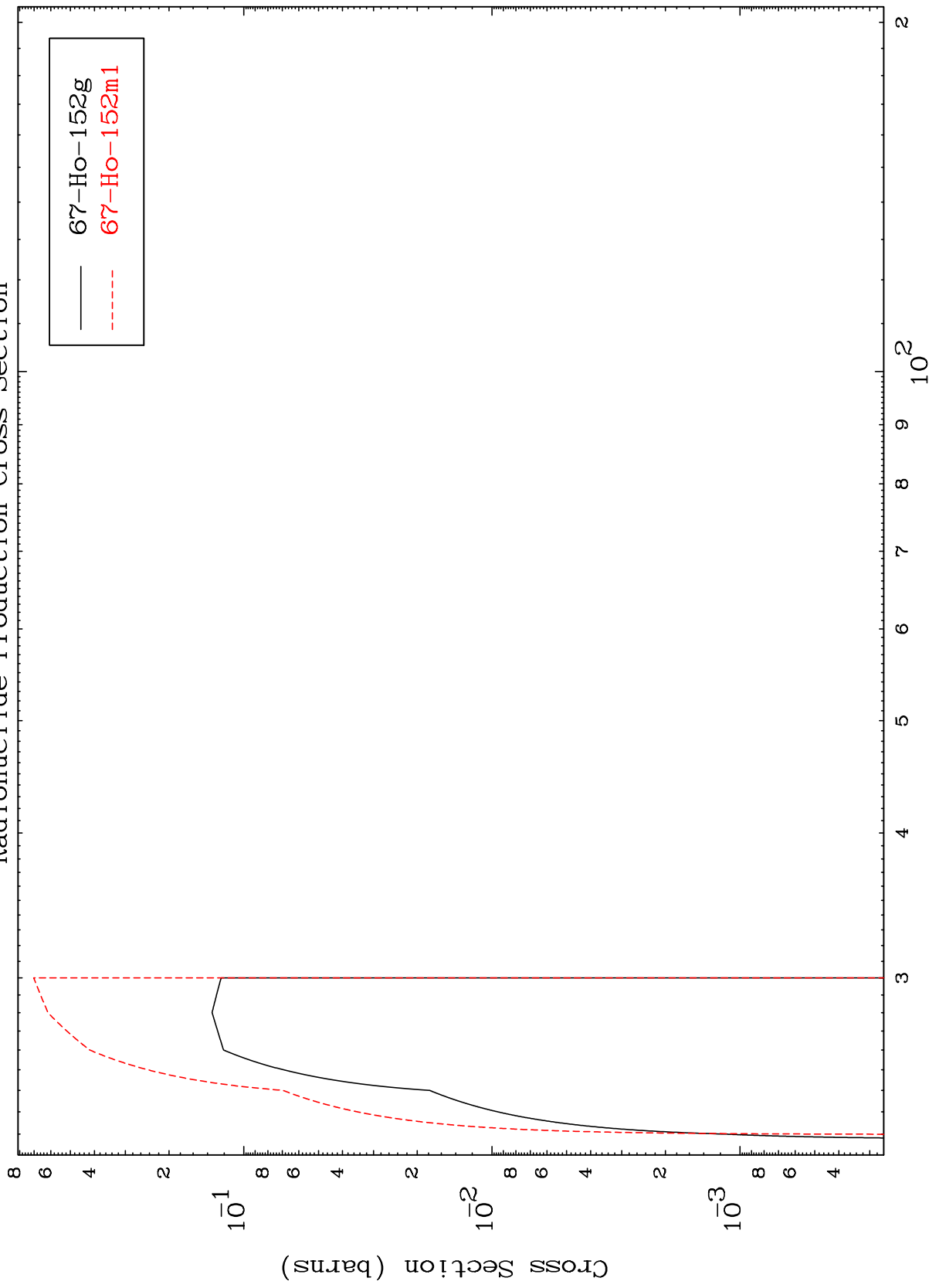
Inelastic  
Radionuclide Production Cross Section



MAT 6498

65-Tb-150

(n,2n)  
Radionuclide Production Cross Section



12

Incident Energy (MeV)

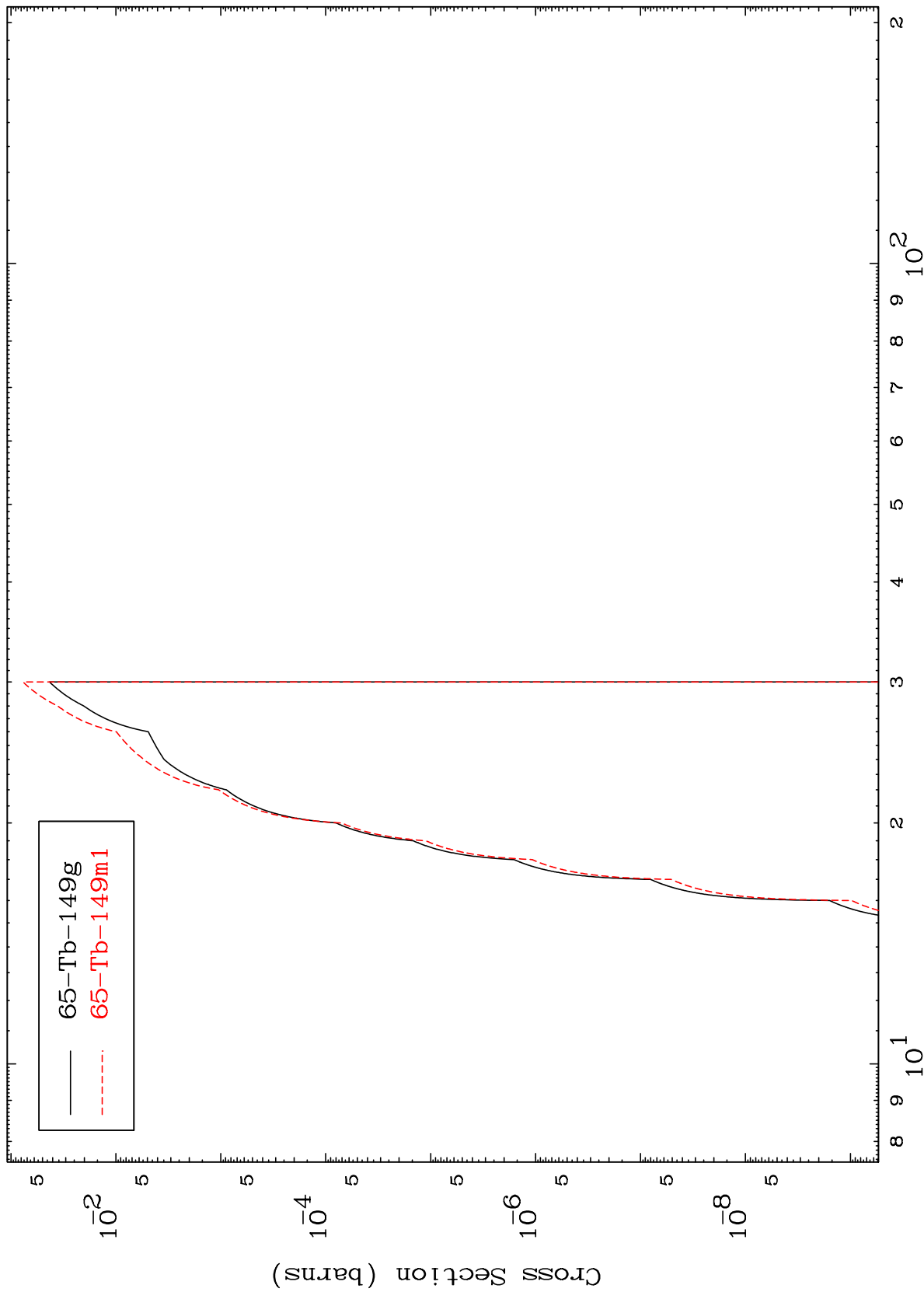
65-Tb-150

MAT 6498

$(n, n') \alpha$

65-Tb-150

Radionuclide Production Cross Section



13

Incident Energy (MeV)

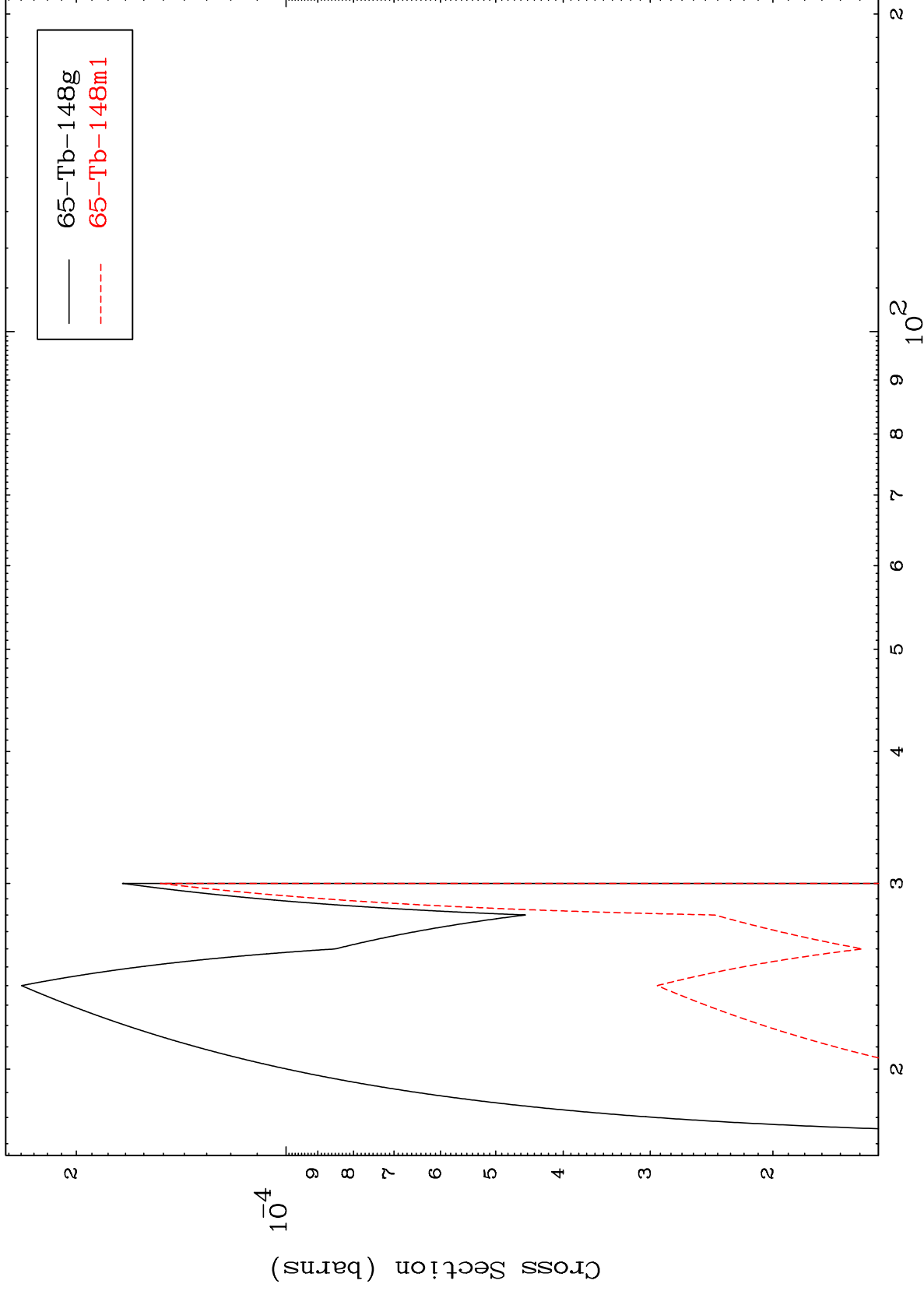
65-Tb-150

MAT 6498

$(n,2n) \alpha$

65-Tb-150

Radionuclide Production Cross Section



14

Incident Energy (MeV)

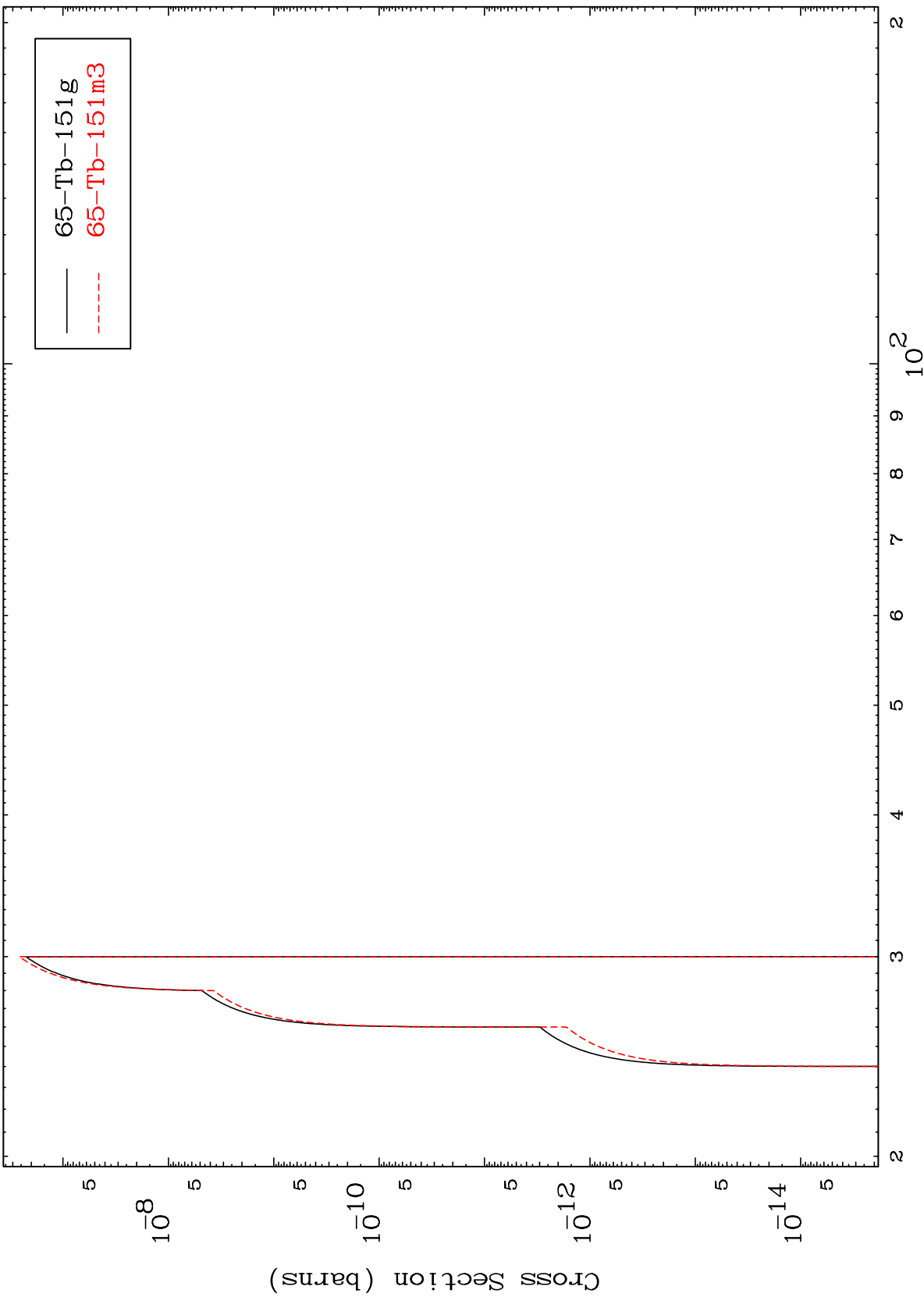
65-Tb-150

MAT 6498

(n,2n) p

65-Tb-150

Radionuclide Production Cross Section



15

Incident Energy (MeV)

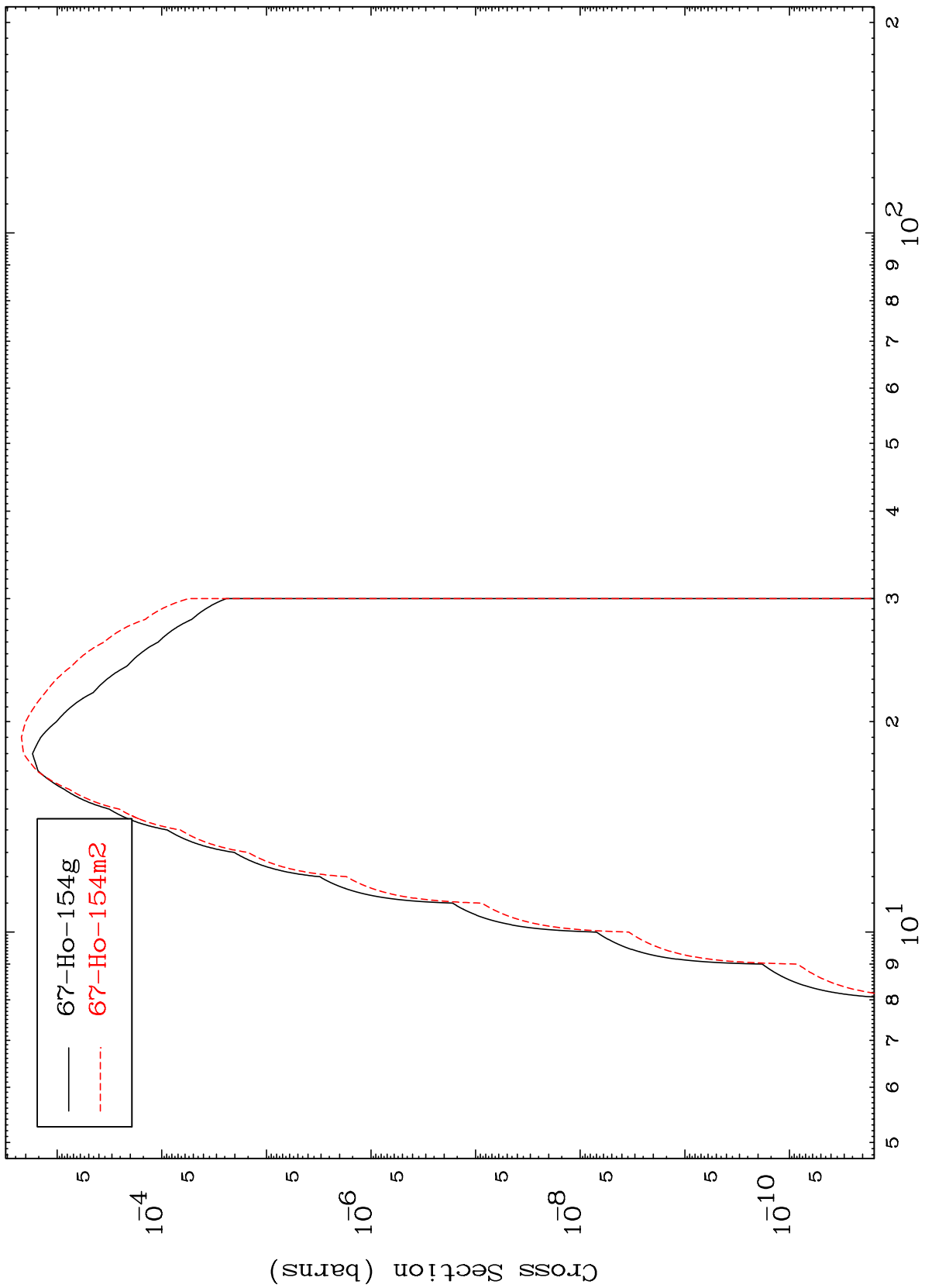
65-Tb-150



MAT 6498

65-Tb-150

(n,γ)  
Radionuclide Production Cross Section



16

Incident Energy (MeV)

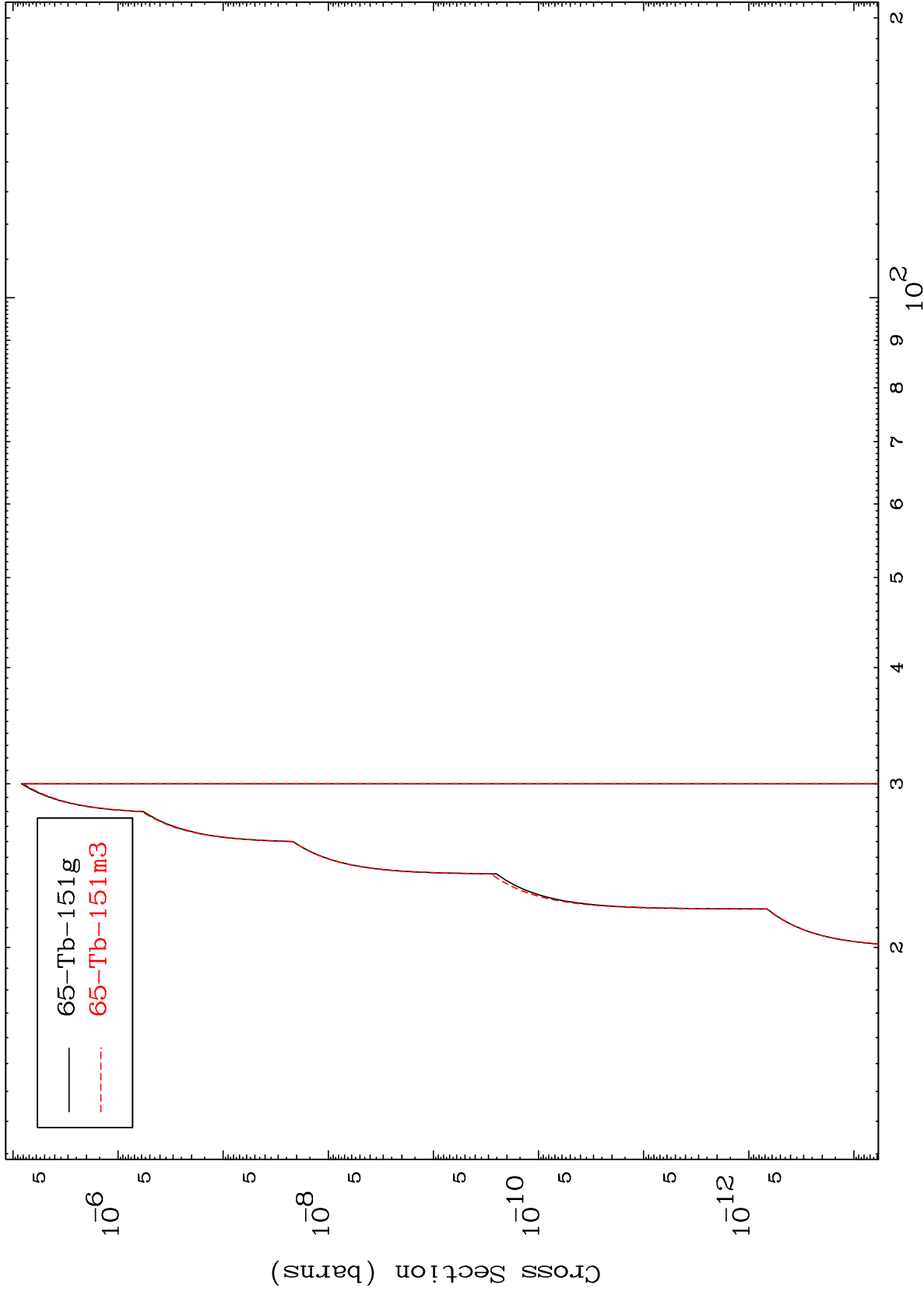
65-Tb-150

MAT 6498

(n,He-3)

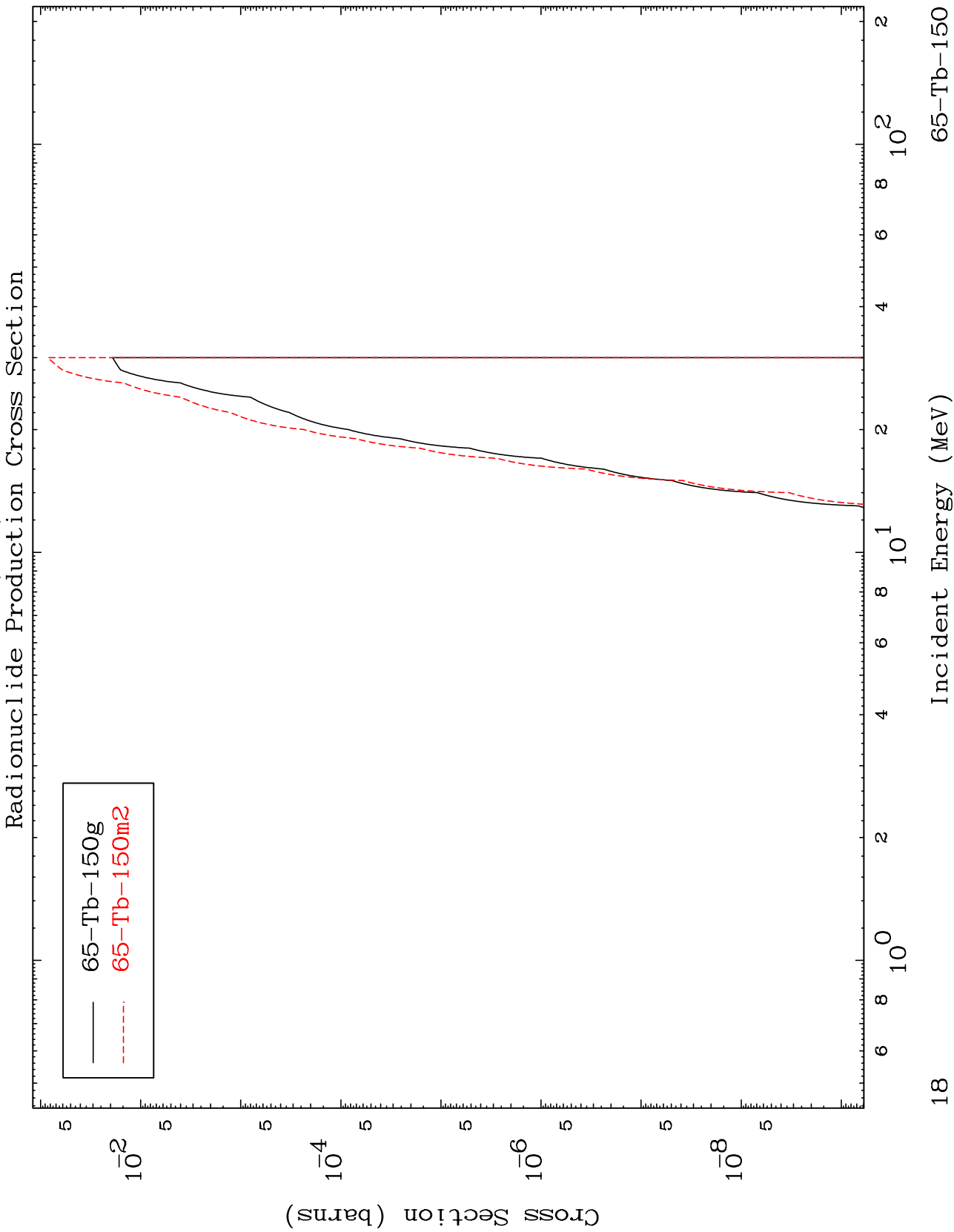
65-Tb-150

Radionuclide Production Cross Section



MAT 6498

65-Tb-150



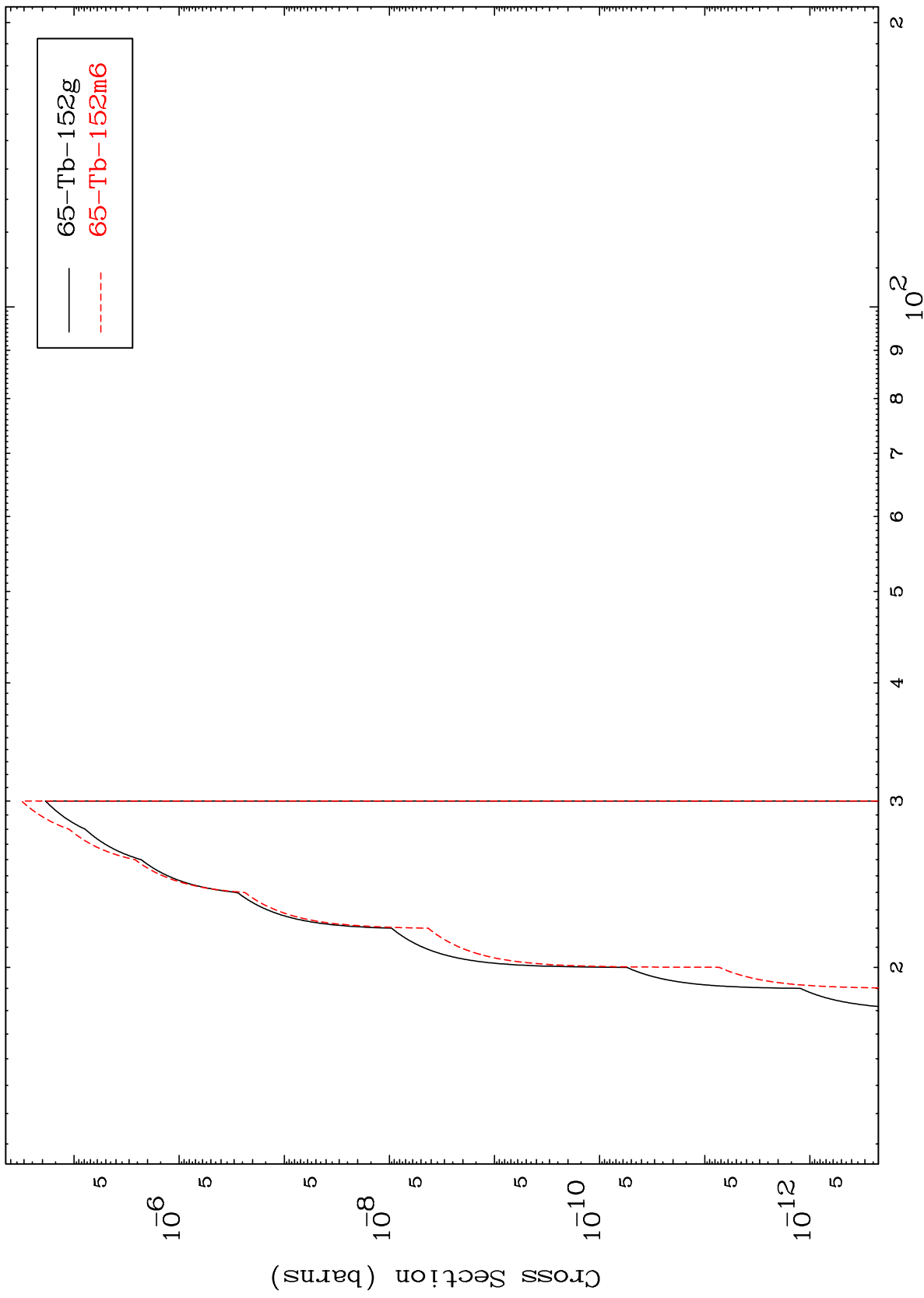
18

65-Tb-150

MAT 6498

65-Tb-150

(n,2p)  
Radionuclide Production Cross Section



19

Incident Energy (MeV)

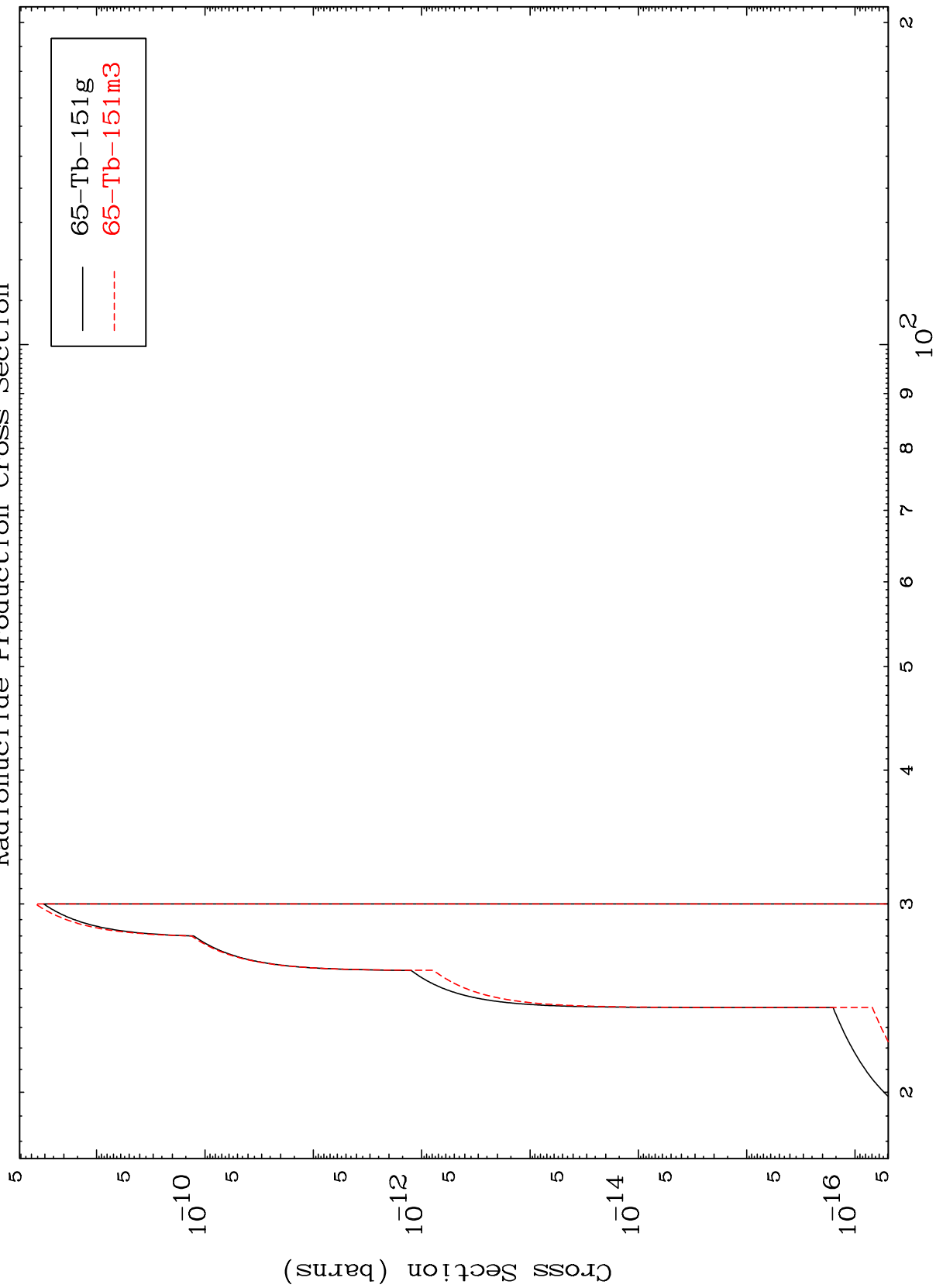
65-Tb-150

MAT 6498

(n,p) d

65-Tb-150

Radionuclide Production Cross Section



20

Incident Energy (MeV)

65-Tb-150