

Program EVALPLOT  
(Version 2021-1)

by

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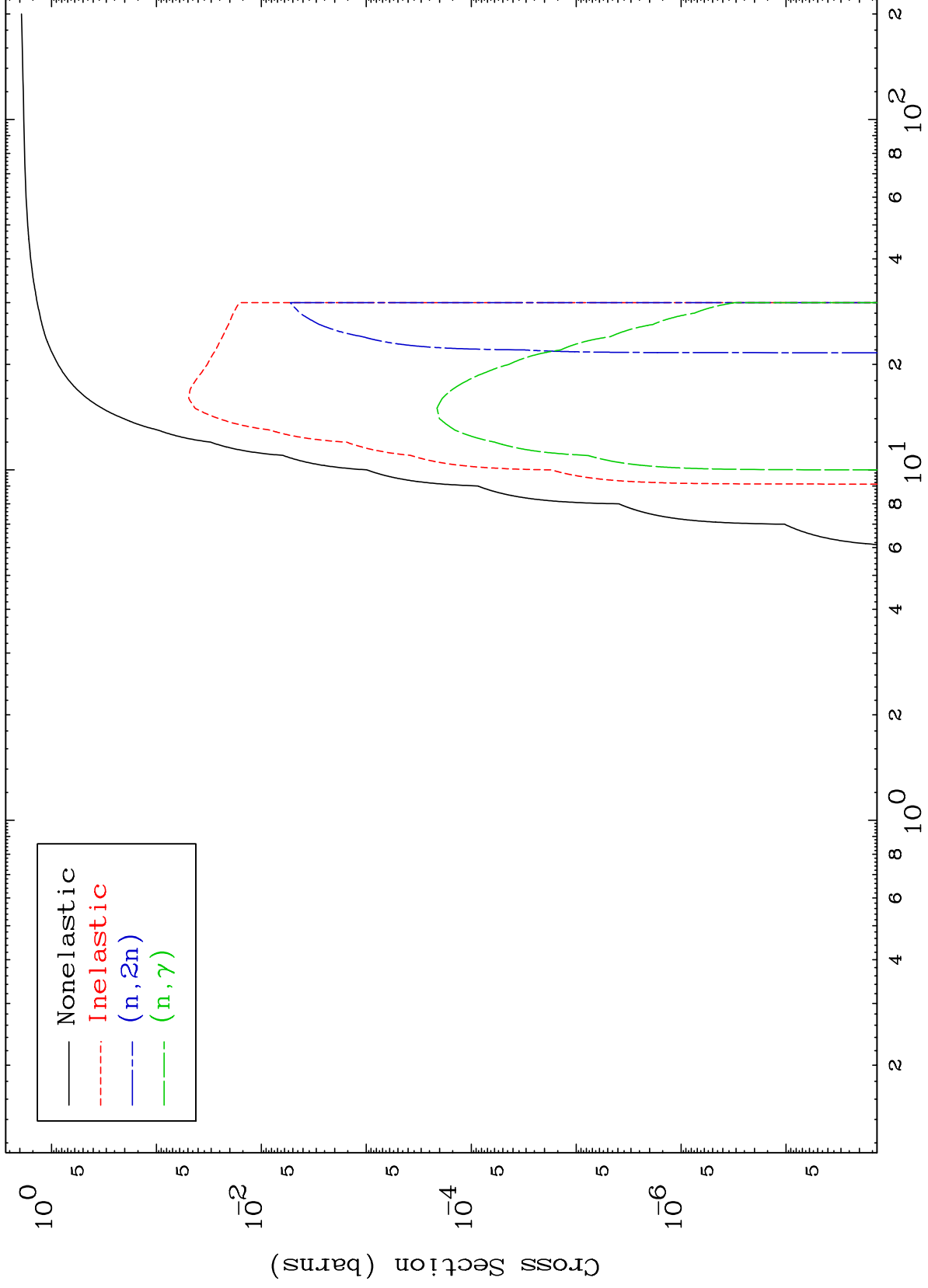
Press Mouse Button to Start

MAT 4301

$\alpha$  Major

43-Tc-91

0 Kelvin Cross Sections



1

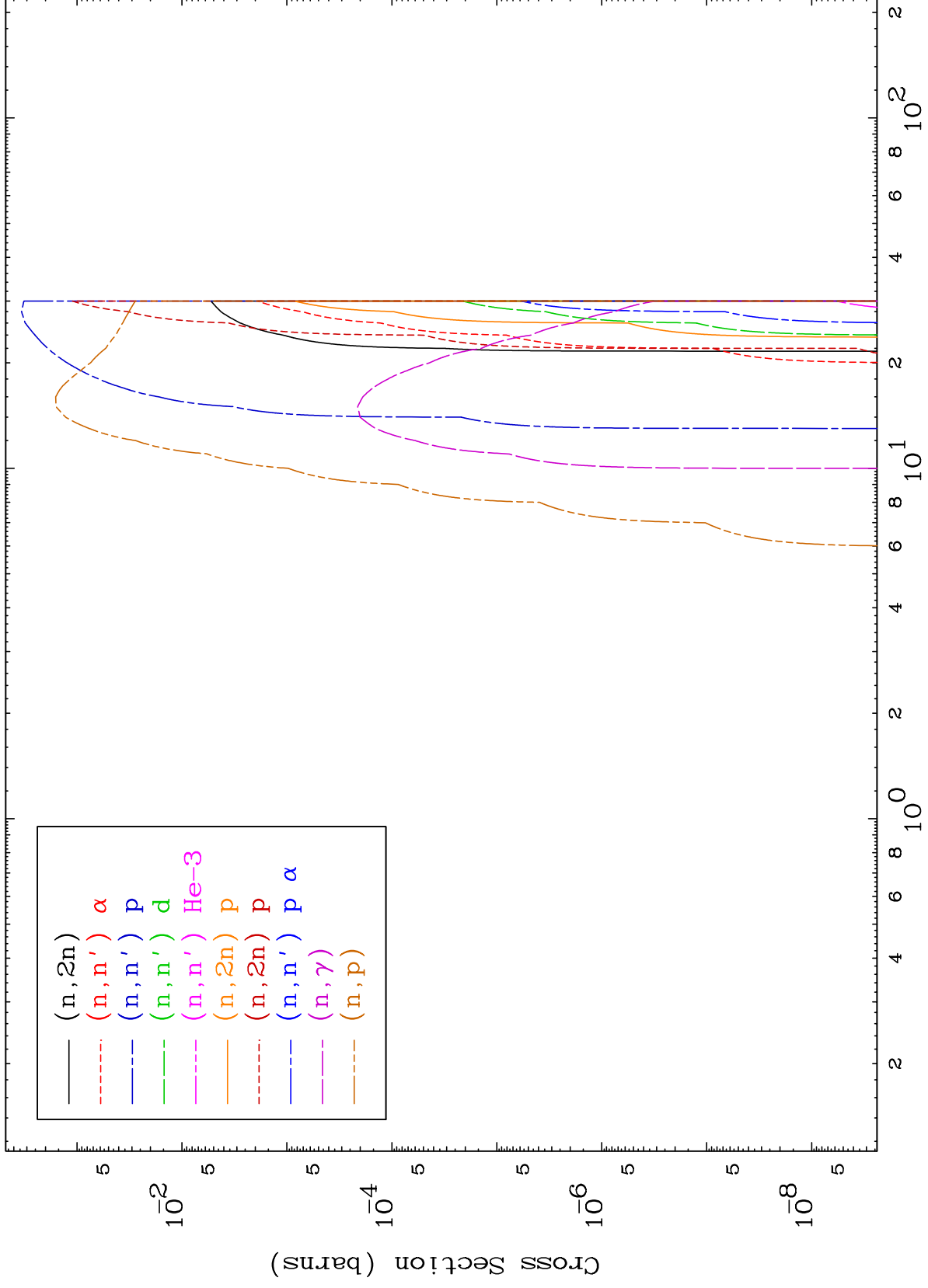
Incident Energy (MeV)

43-Tc-91

MAT 4301

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

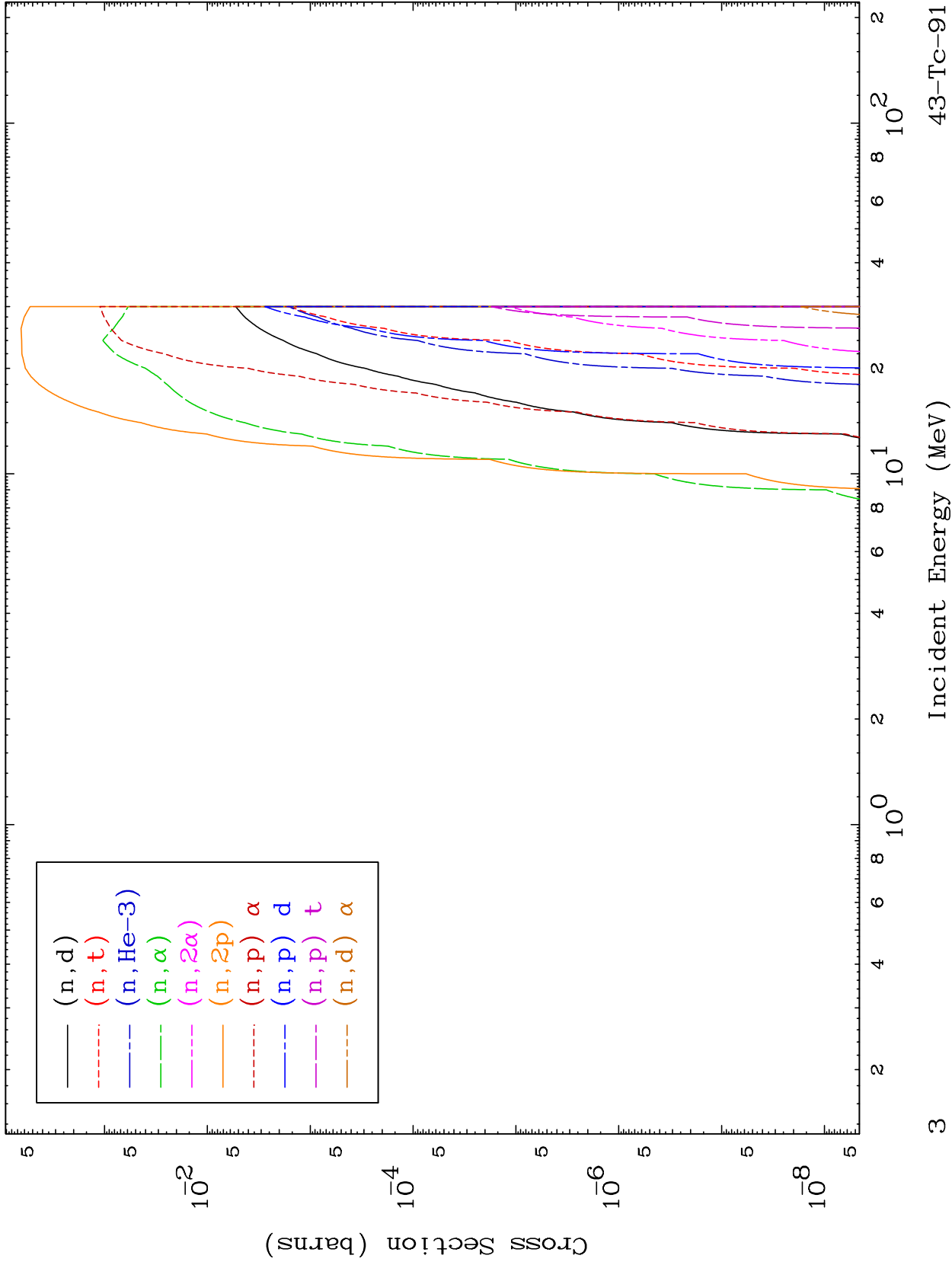
43-Tc-91



MAT 4301

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

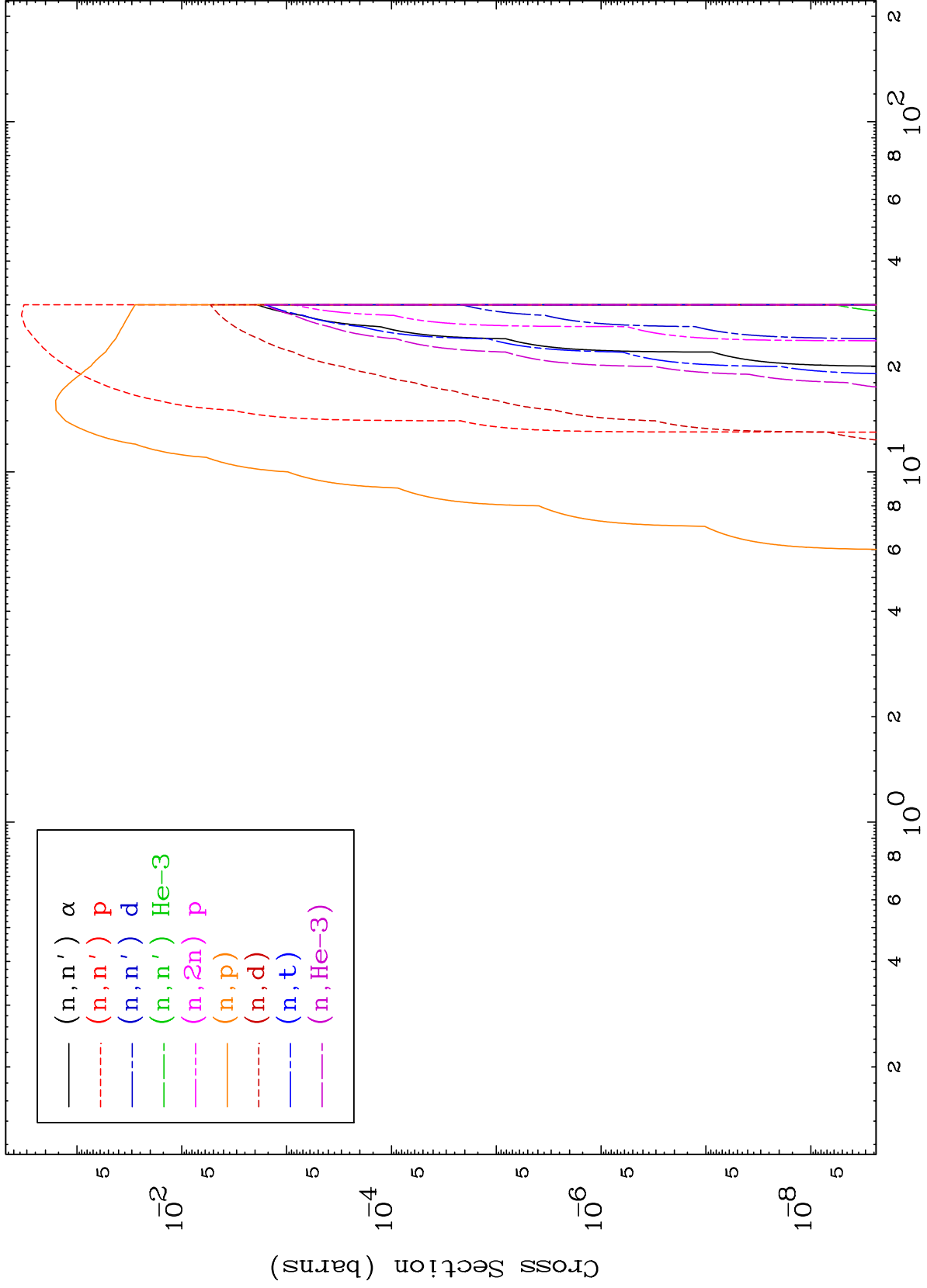
43-Tc-91

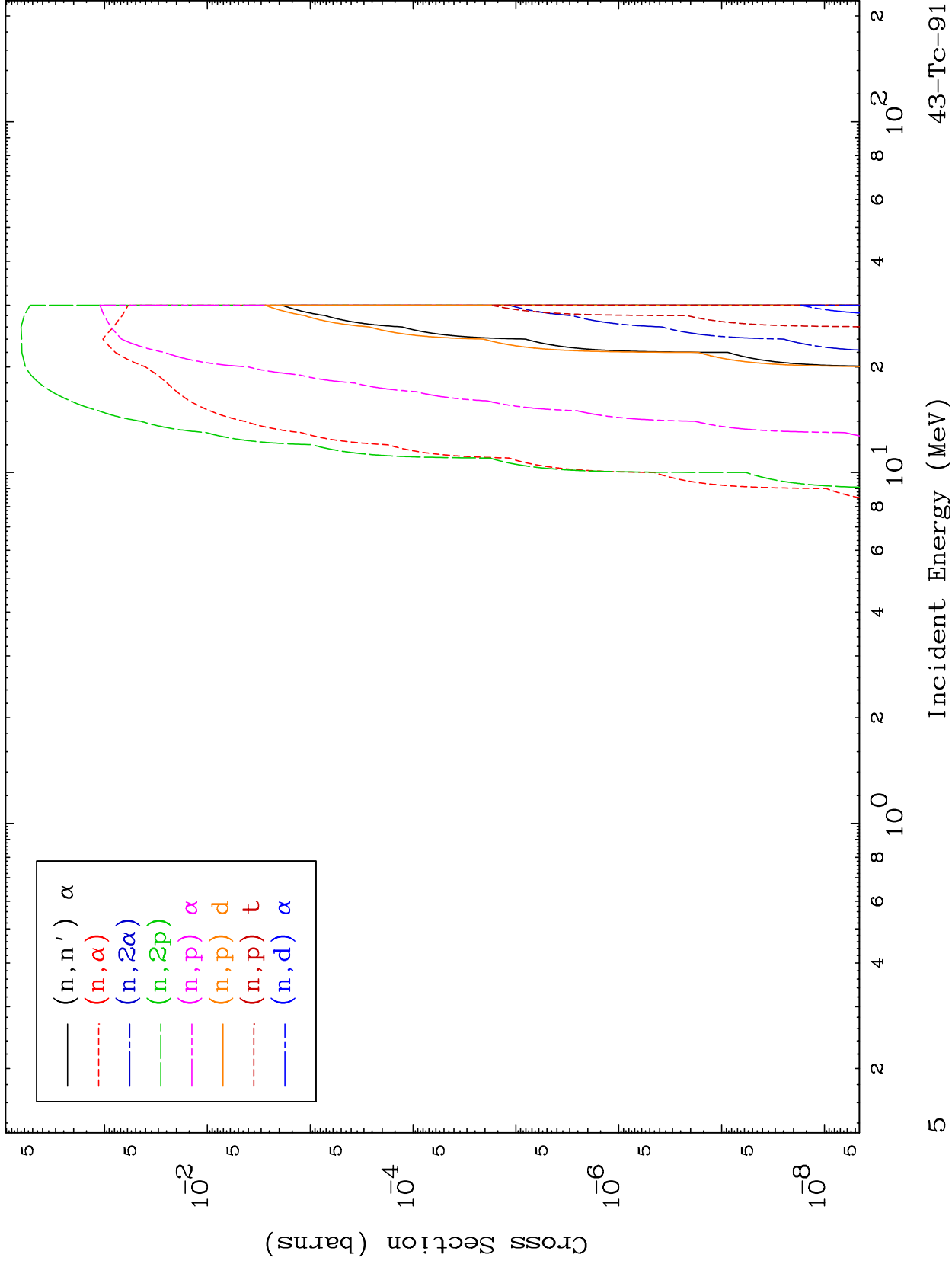


MAT 4301

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

43-Tc-91



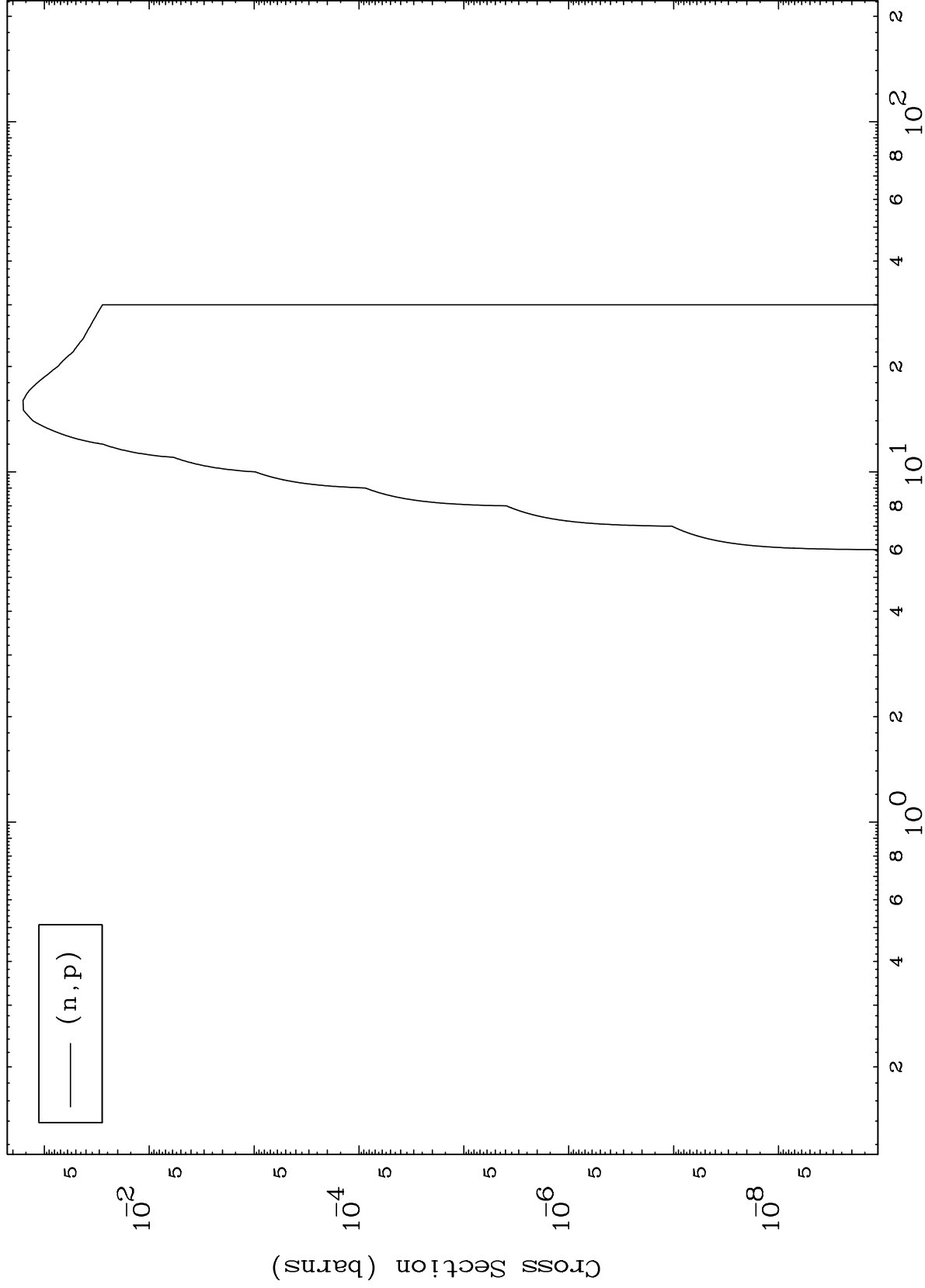


MAT 4301

( $\alpha, p$ ) Levels

43-Tc-91

0 Kelvin Cross Sections

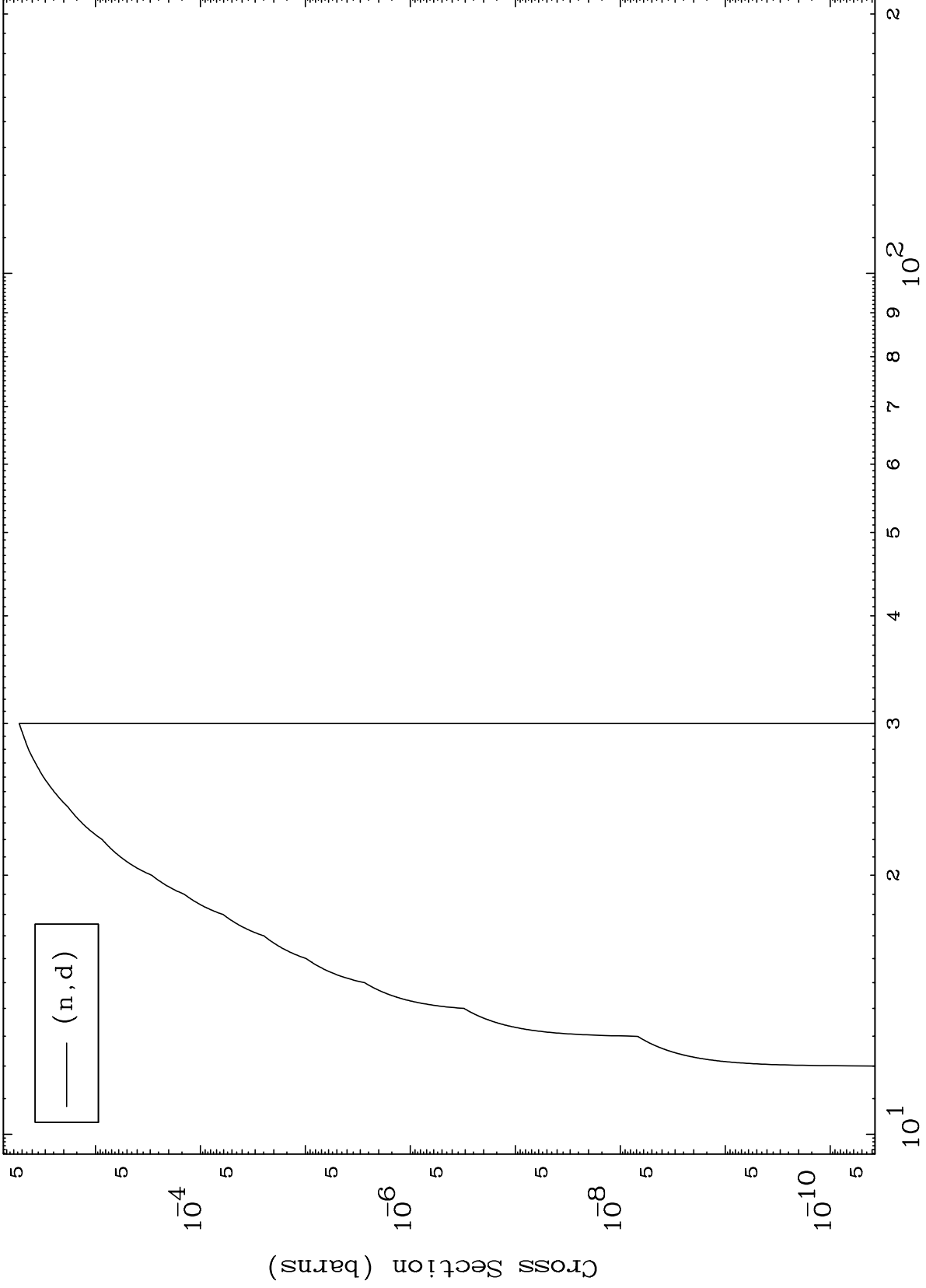


MAT 4301

( $\alpha, d$ ) Levels

43-Tc-91

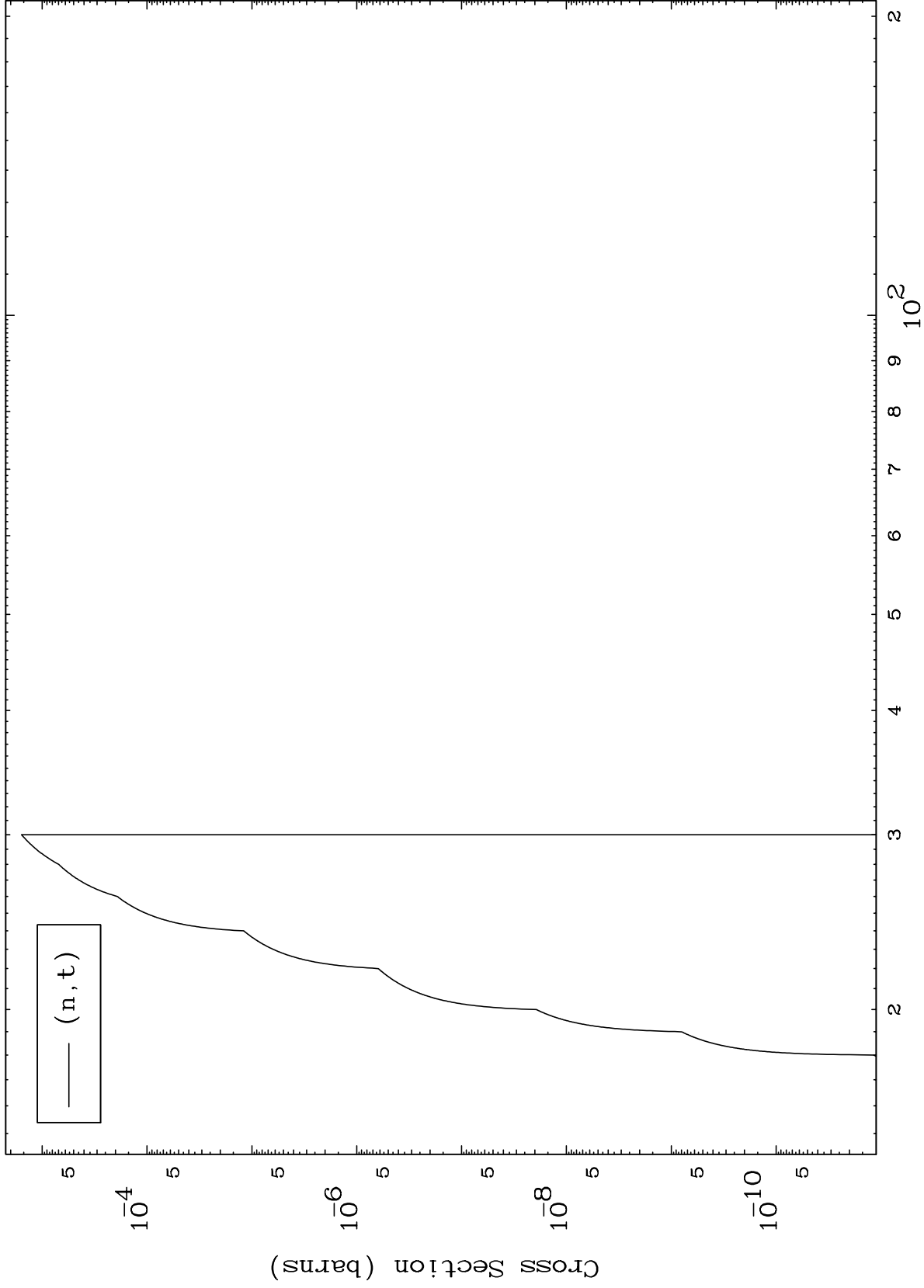
0 Kelvin Cross Sections



Incident Energy (MeV)

43-Tc-91

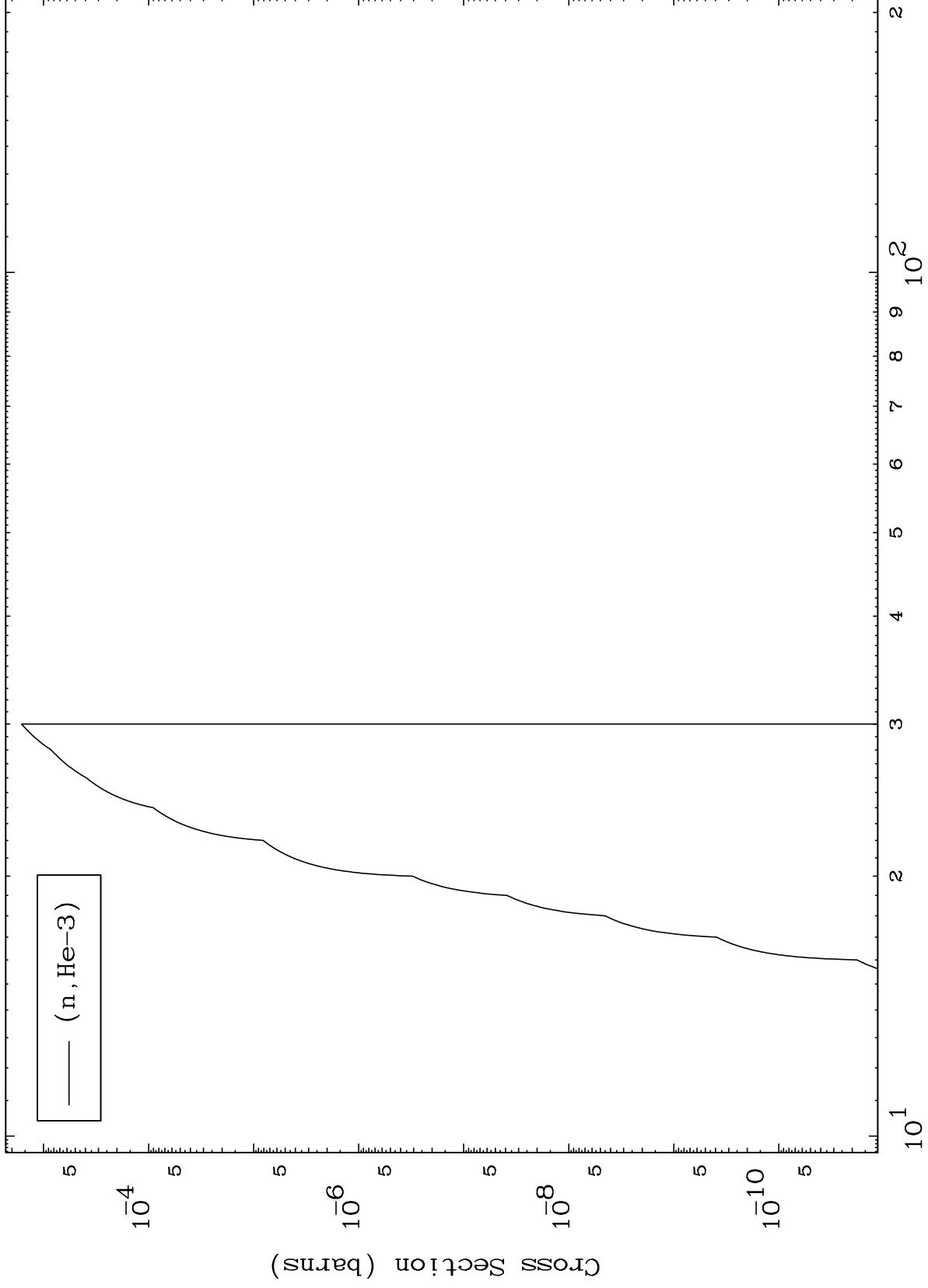




MAT 4301

43-Tc-91

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections



43-Tc-91

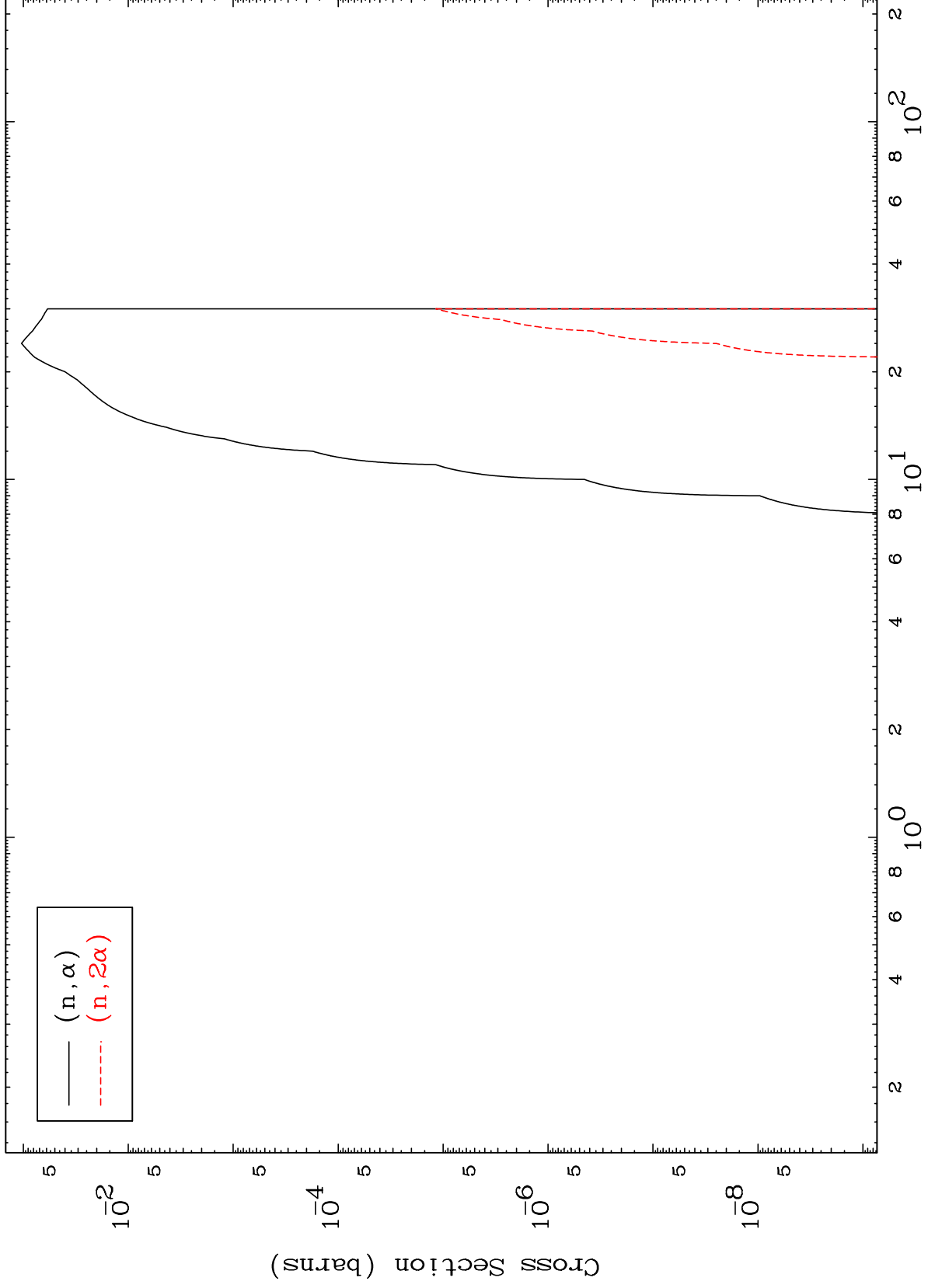
Incident Energy (MeV)

MAT 4301

( $\alpha, \alpha$ ) Levels

43-Tc-91

0 Kelvin Cross Sections



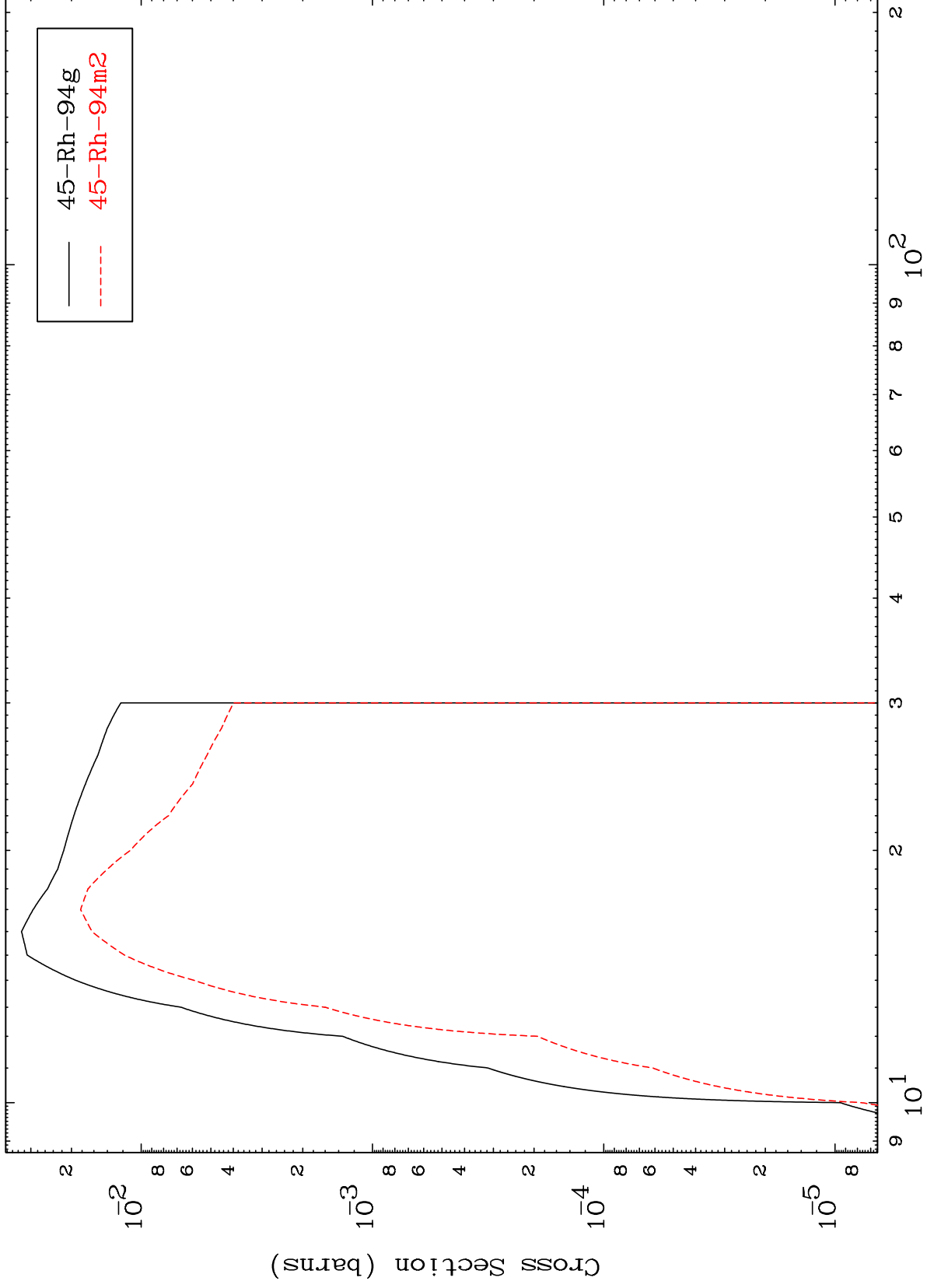
— (n,  $\alpha$ )  
- - - (n,  $2\alpha$ )

10

Incident Energy (MeV)

43-Tc-91

Inelastic  
Radionuclide Production Cross Section

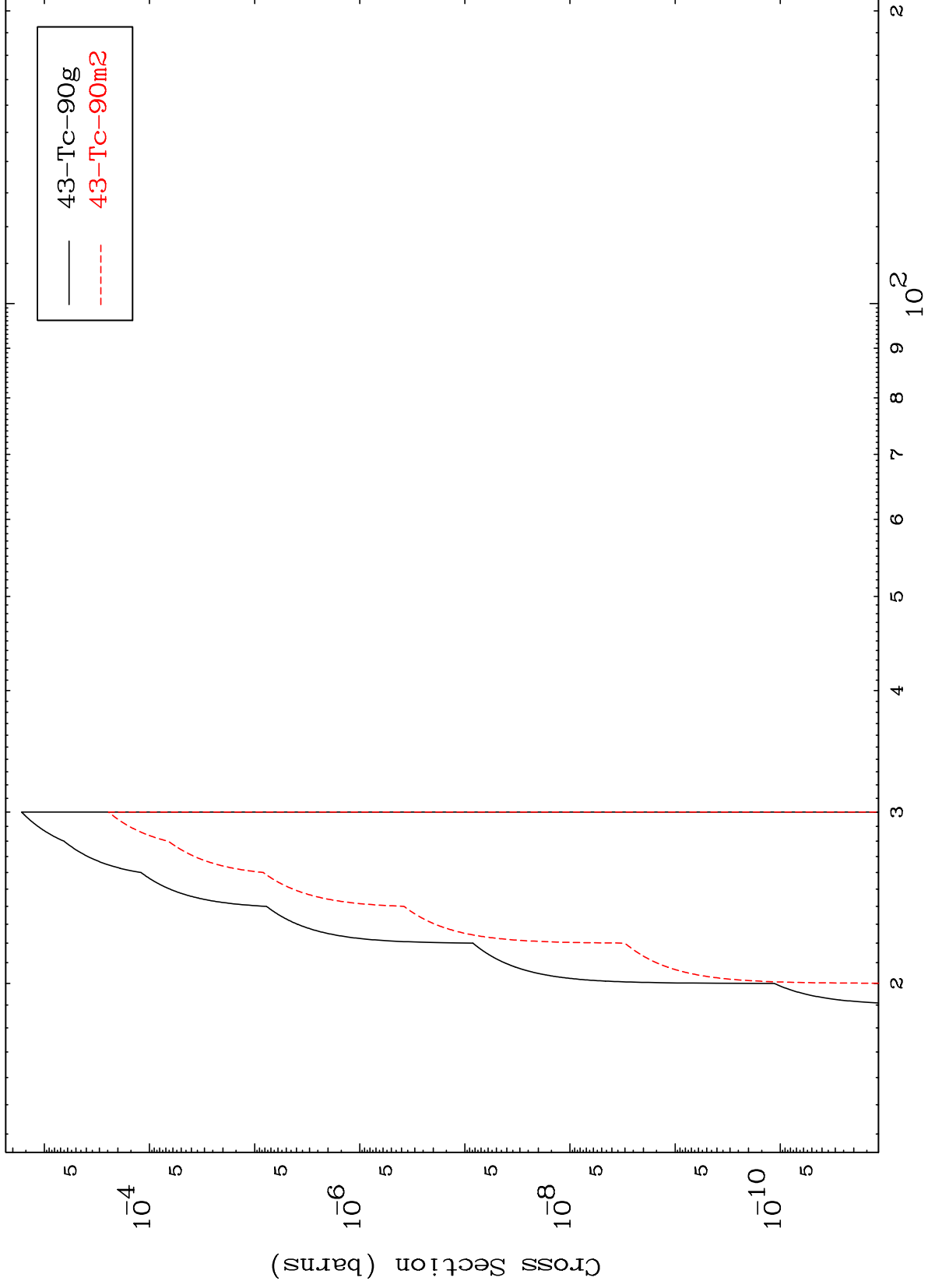


MAT 4301

$(n, n')$   $\alpha$

43-Tc-91

Radionuclide Production Cross Section

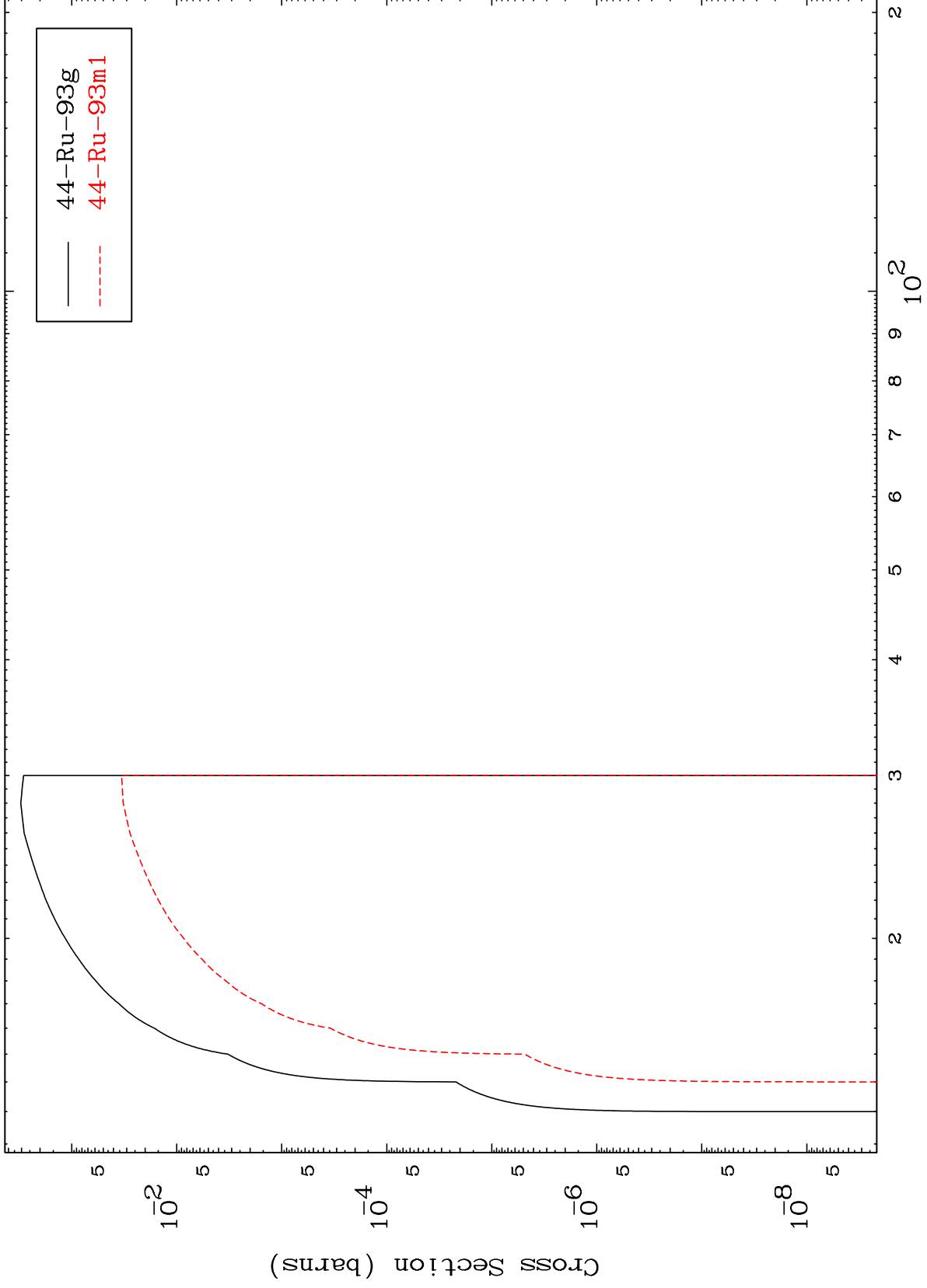


12

Incident Energy (MeV)

43-Tc-91

Radionuclide Production Cross Section

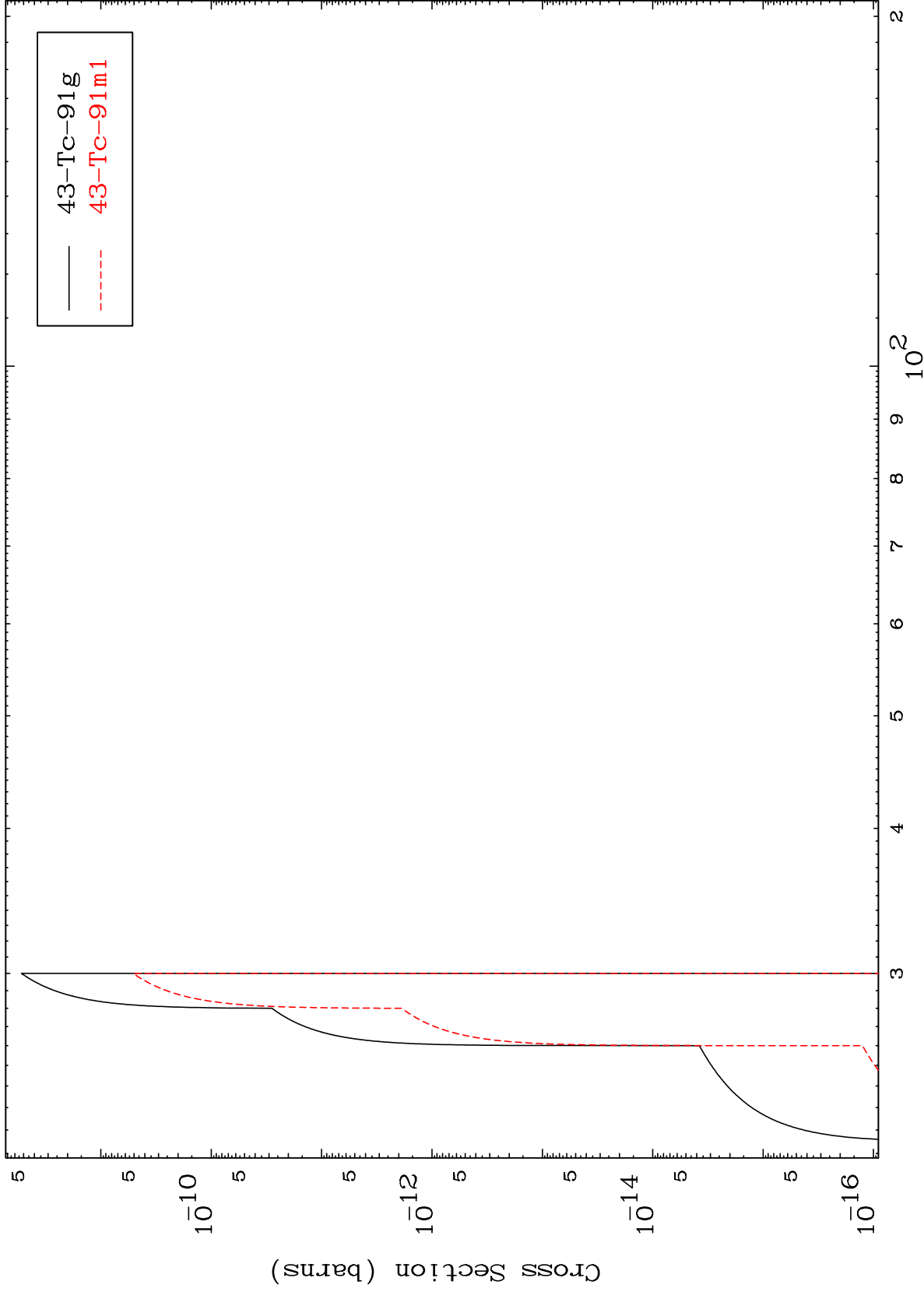


MAT 4301

(n,n') He-3

43-Tc-91

Radionuclide Production Cross Section



14

Incident Energy (MeV)

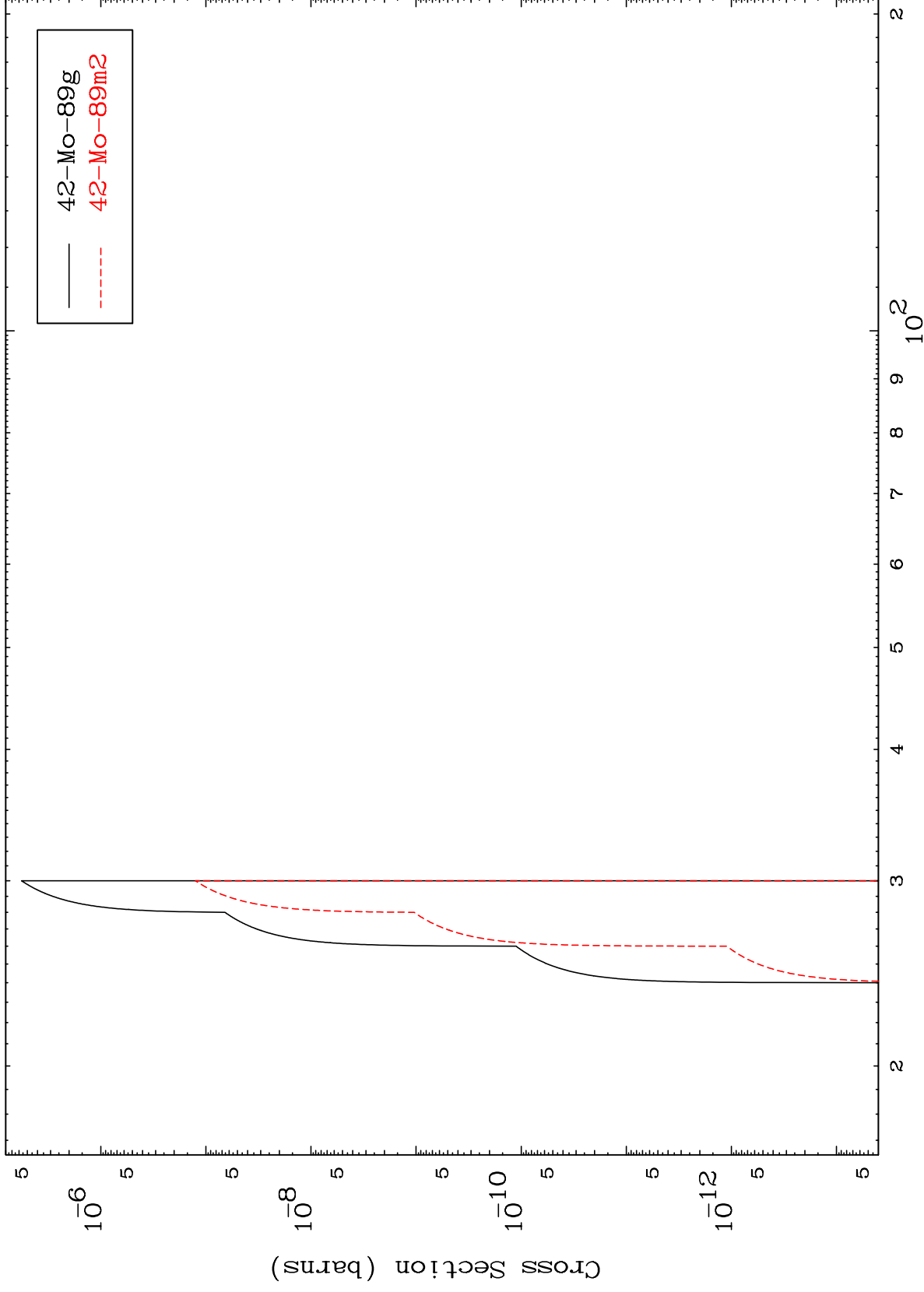
43-Tc-91

MAT 4301

(n,n') p  $\alpha$

43-Tc-91

Radionuclide Production Cross Section



15

Incident Energy (MeV)

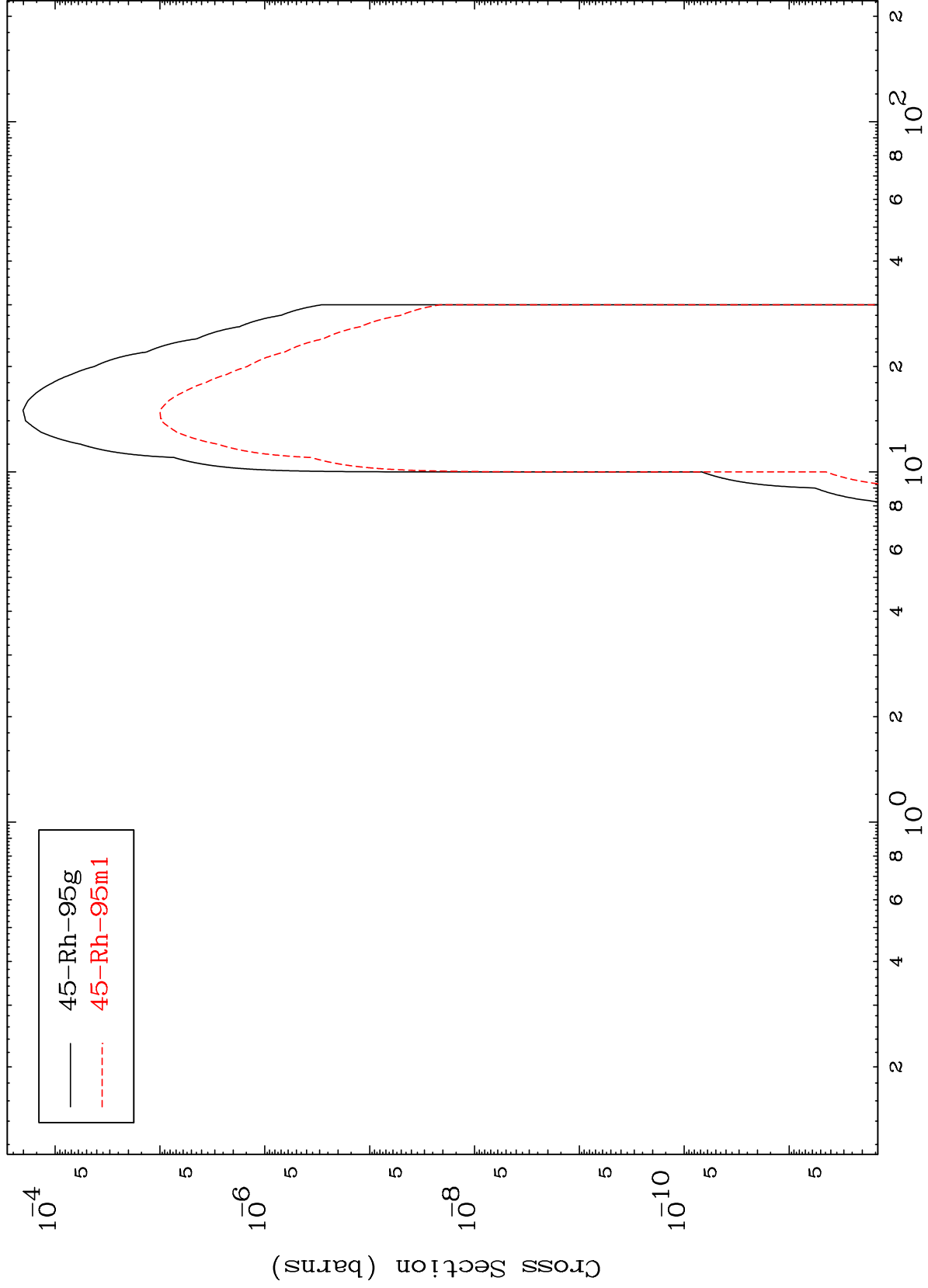
43-Tc-91



MAT 4301

43-Tc-91

(n,  $\gamma$ )  
Radionuclide Production Cross Section



16

43-Tc-91

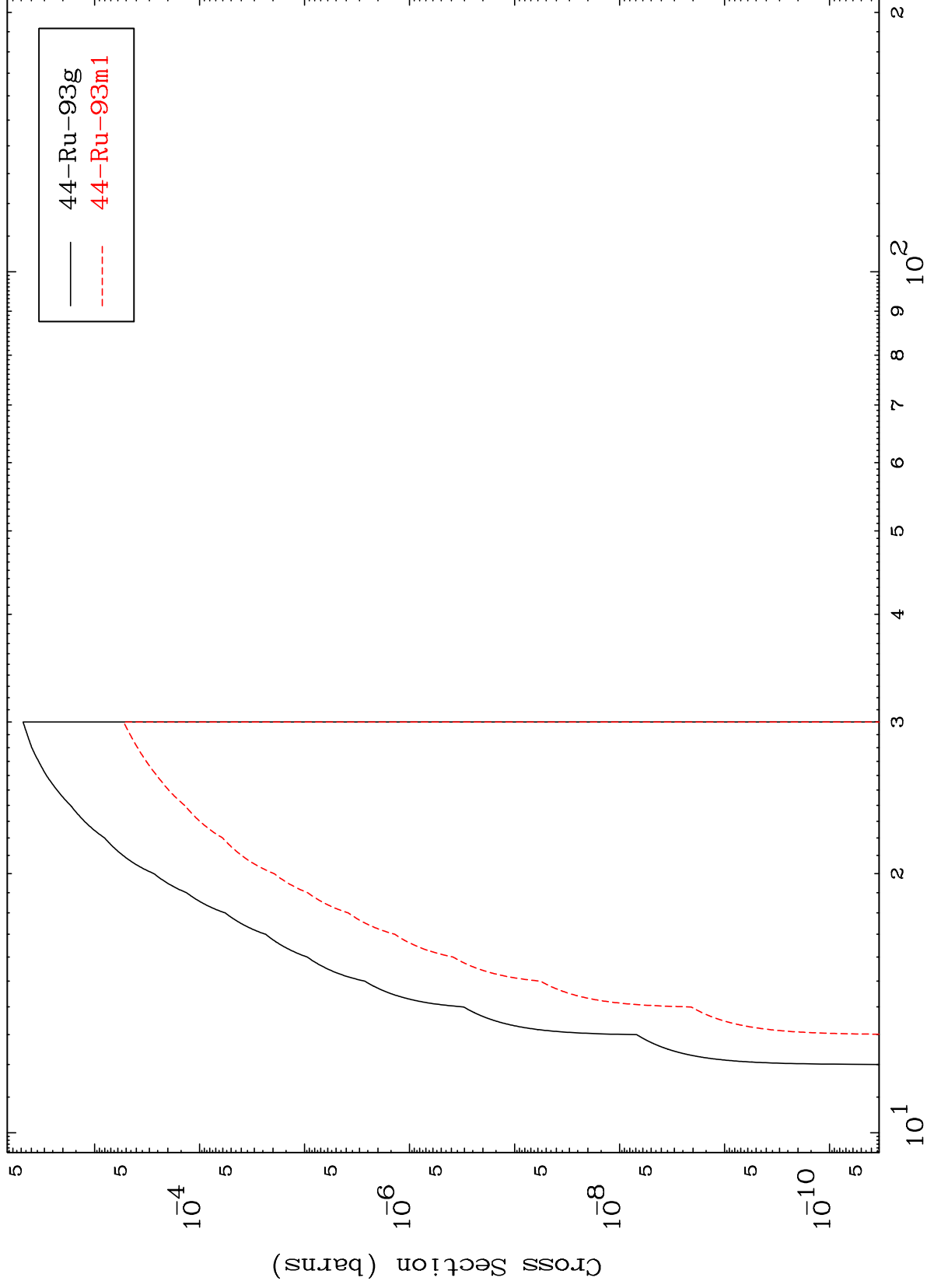
Incident Energy (MeV)

MAT 4301

(n,d)

43-Tc-91

Radionuclide Production Cross Section



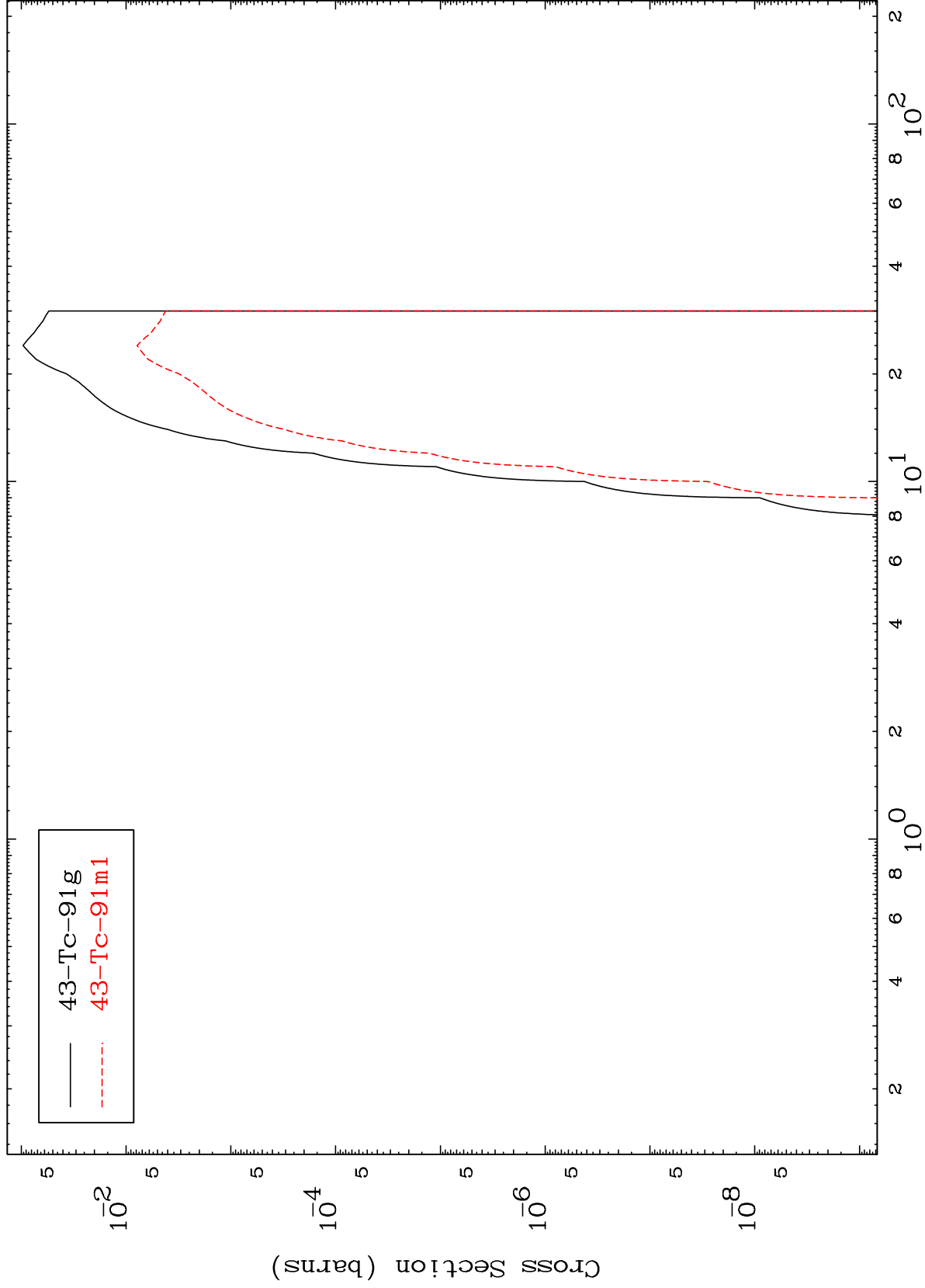
Incident Energy (MeV)

43-Tc-91

MAT 4301

43-Tc-91

(n,  $\alpha$ )  
Radionuclide Production Cross Section



18

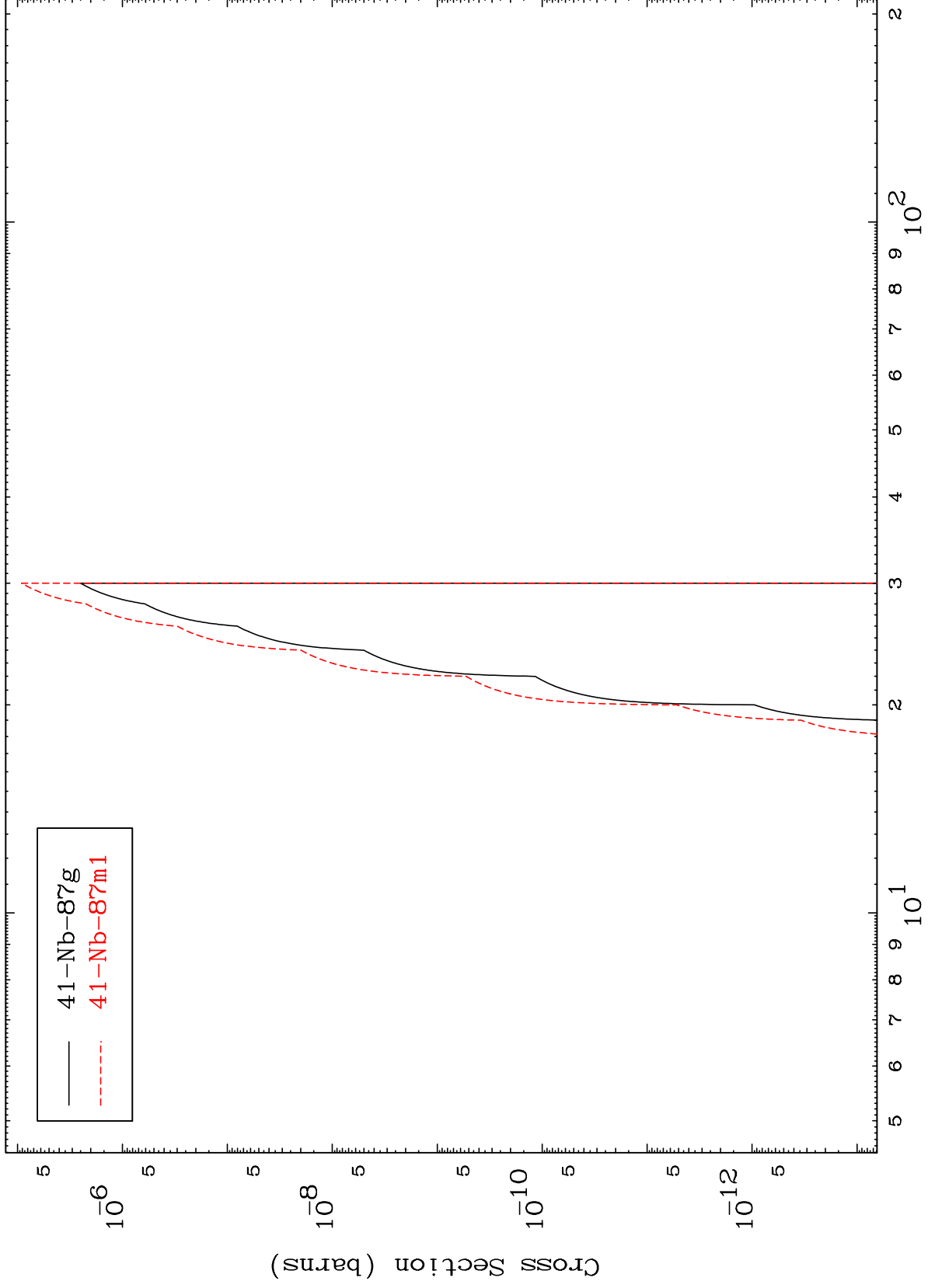
Incident Energy (MeV)

43-Tc-91

MAT 4301

43-Tc-91

(n,2α)  
Radionuclide Production Cross Section



19

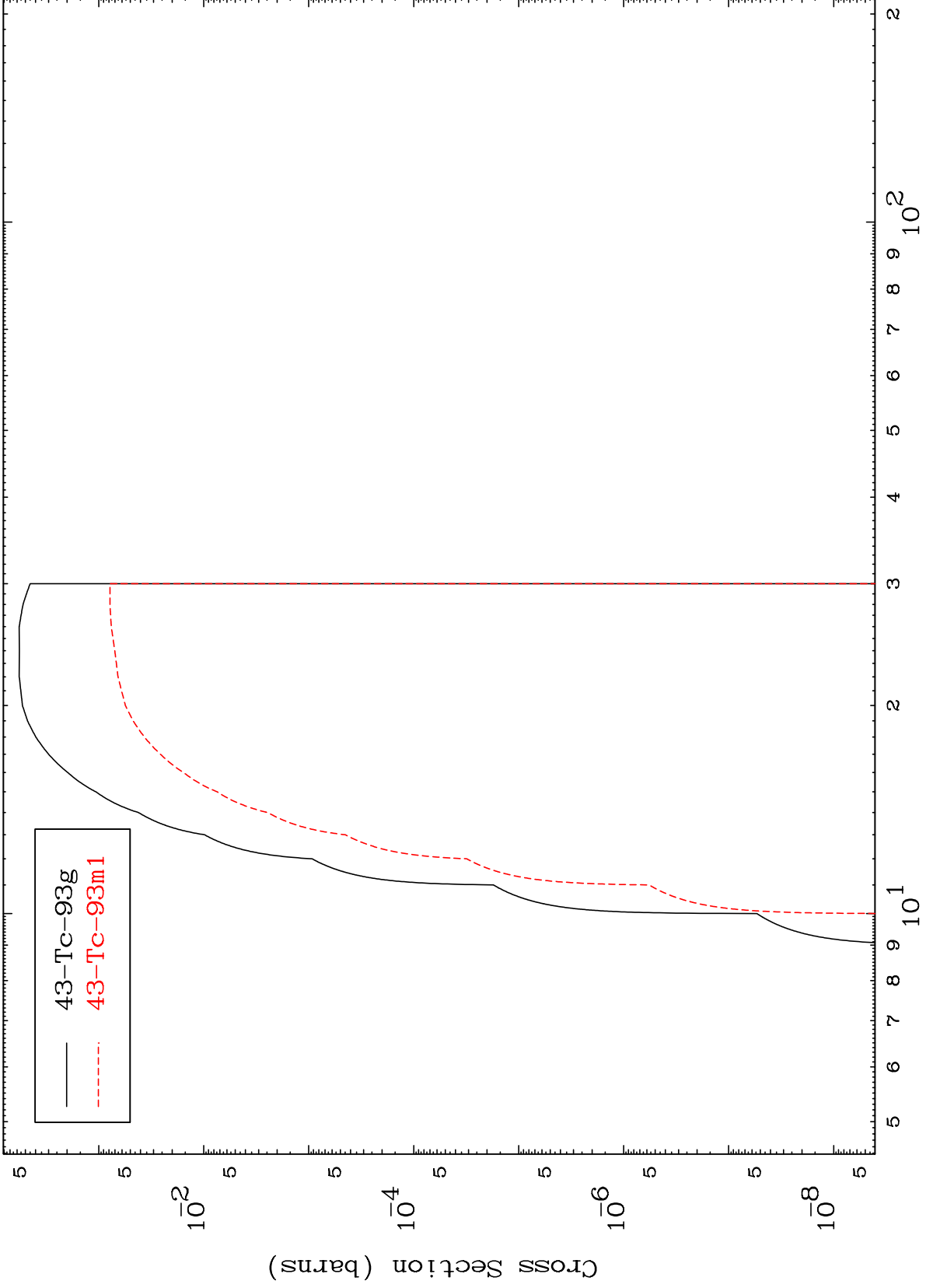
Incident Energy (MeV)

43-Tc-91

MAT 4301

43-Tc-91

(n,2p)  
Radionuclide Production Cross Section

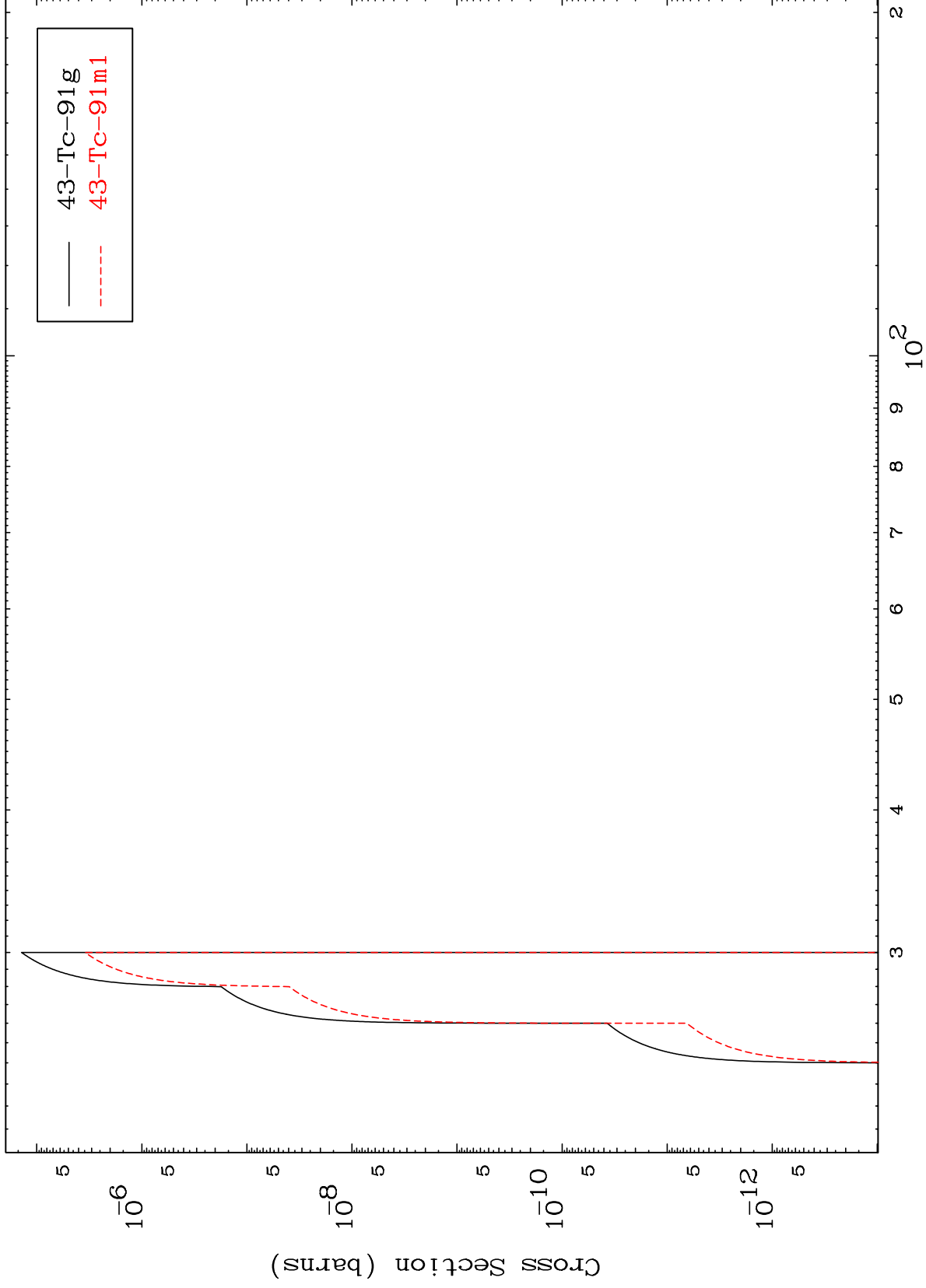


20

Incident Energy (MeV)

43-Tc-91

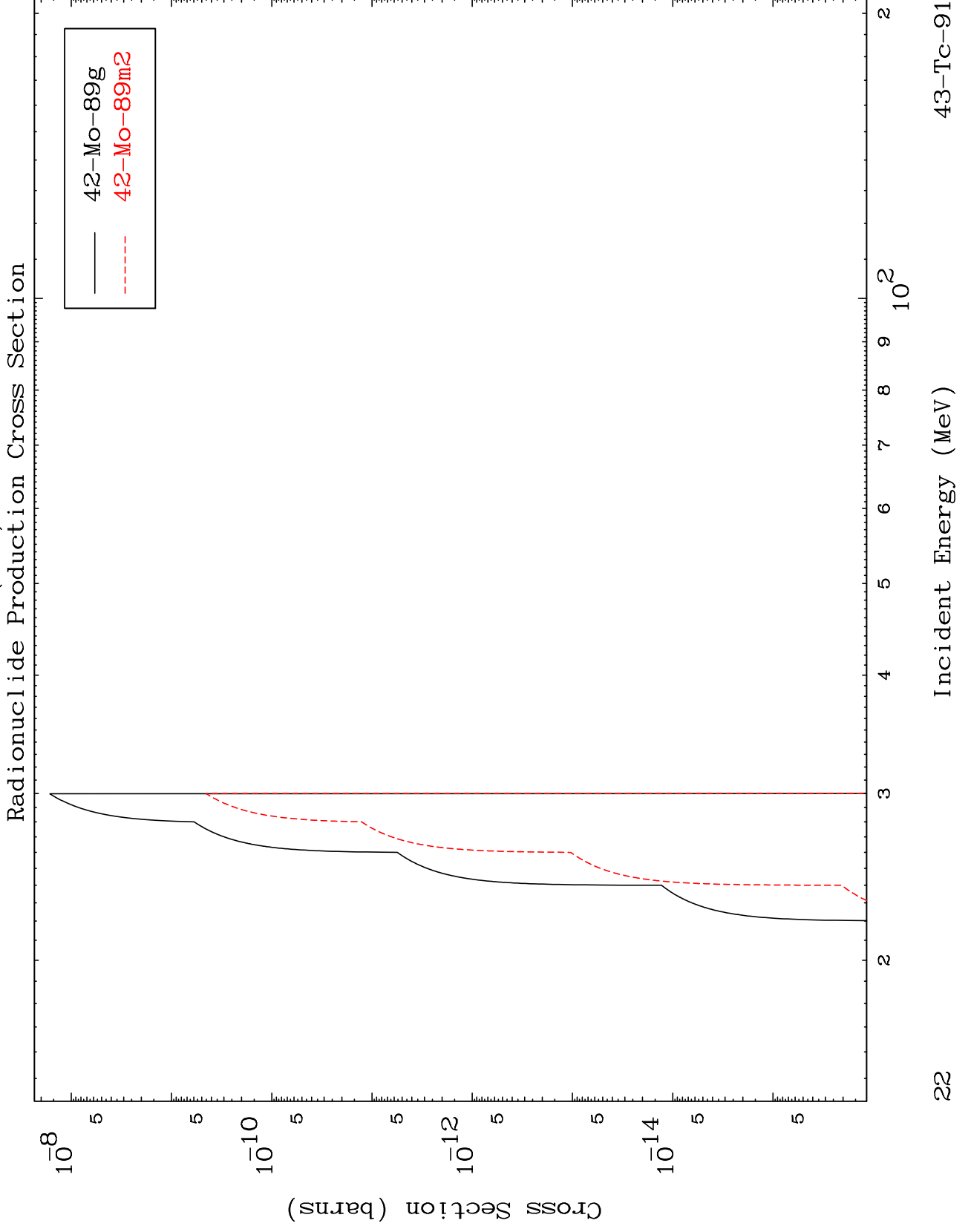
Radionuclide Production Cross Section



MAT 4301

(n,d)  $\alpha$

43-Tc-91



22

Incident Energy (MeV)

43-Tc-91