

Program EVALPLOT  
(Version 2021-1)

by

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(Present Contact Information)

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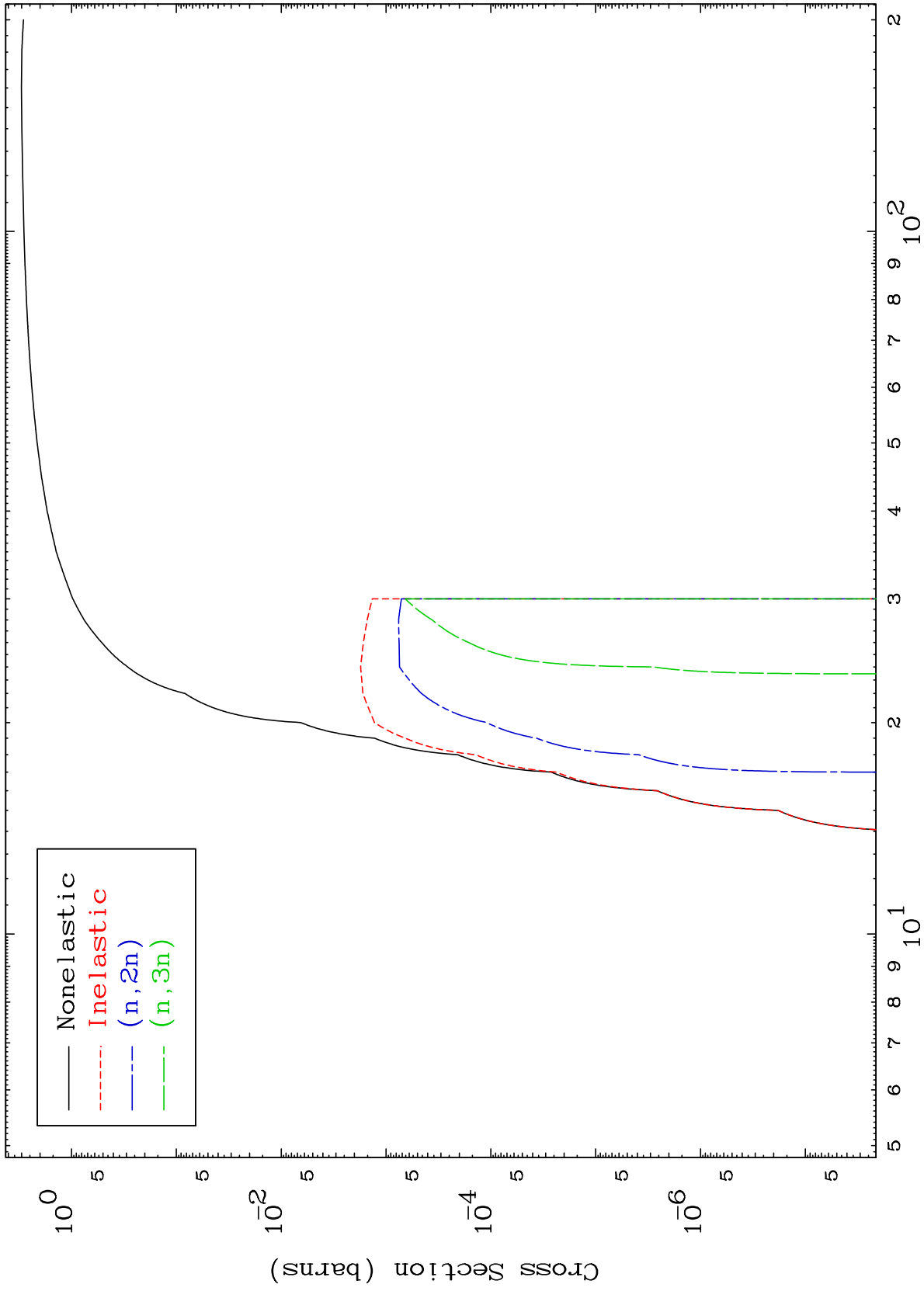
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9237

0 Kelvin  $\alpha$  Major  
Cross Sections

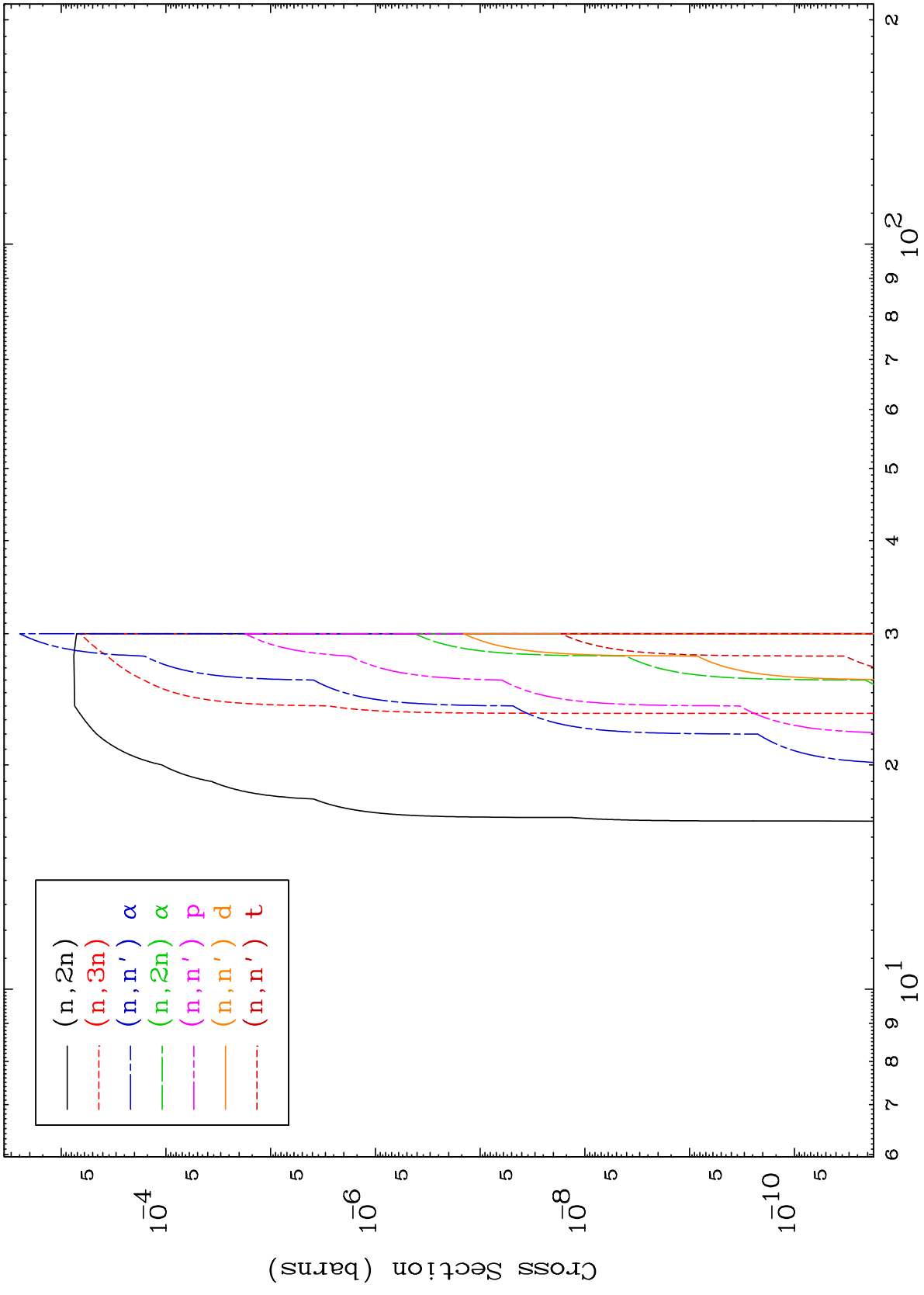
92-U -238



1

Incident Energy (MeV)

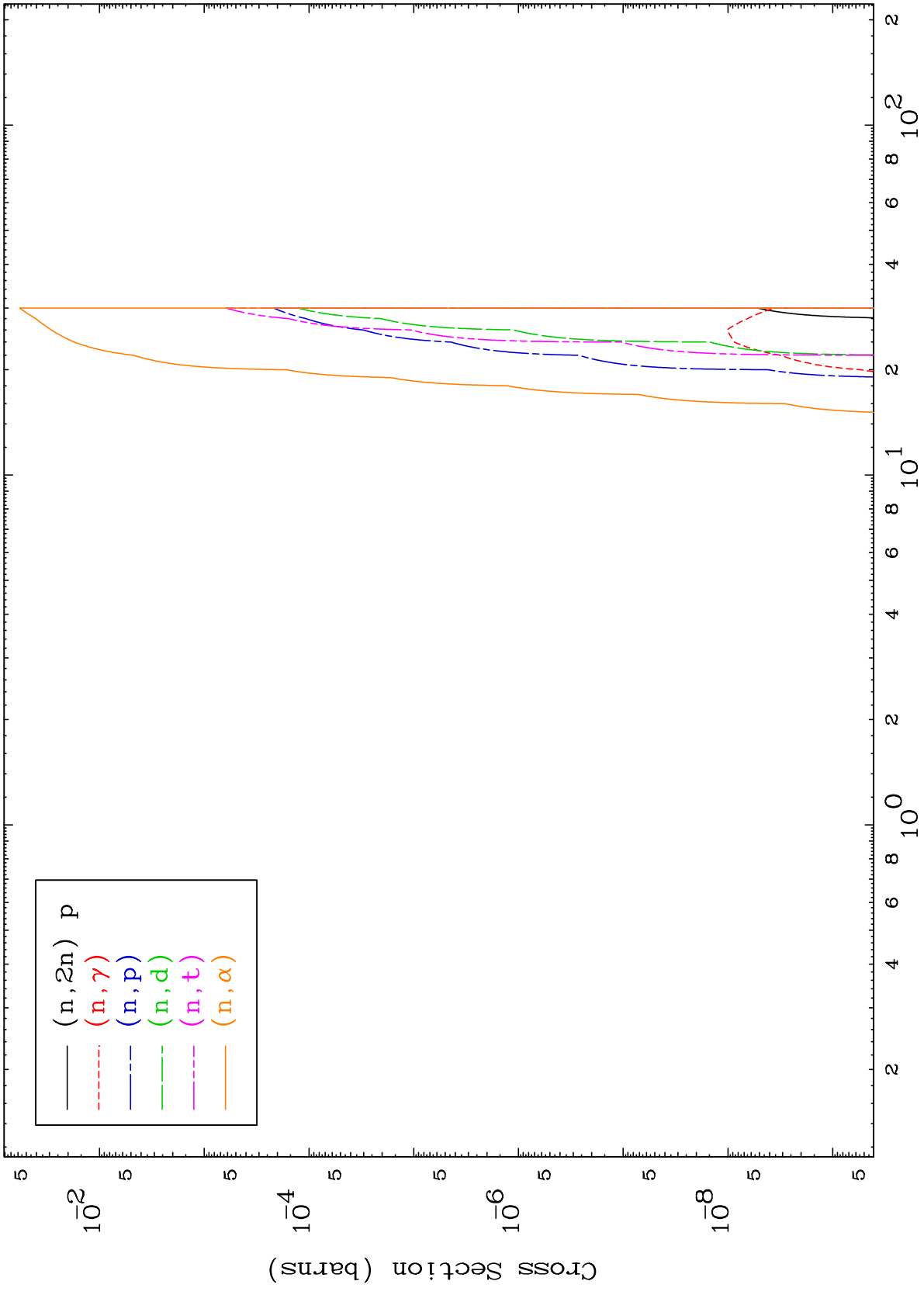
92-U -238

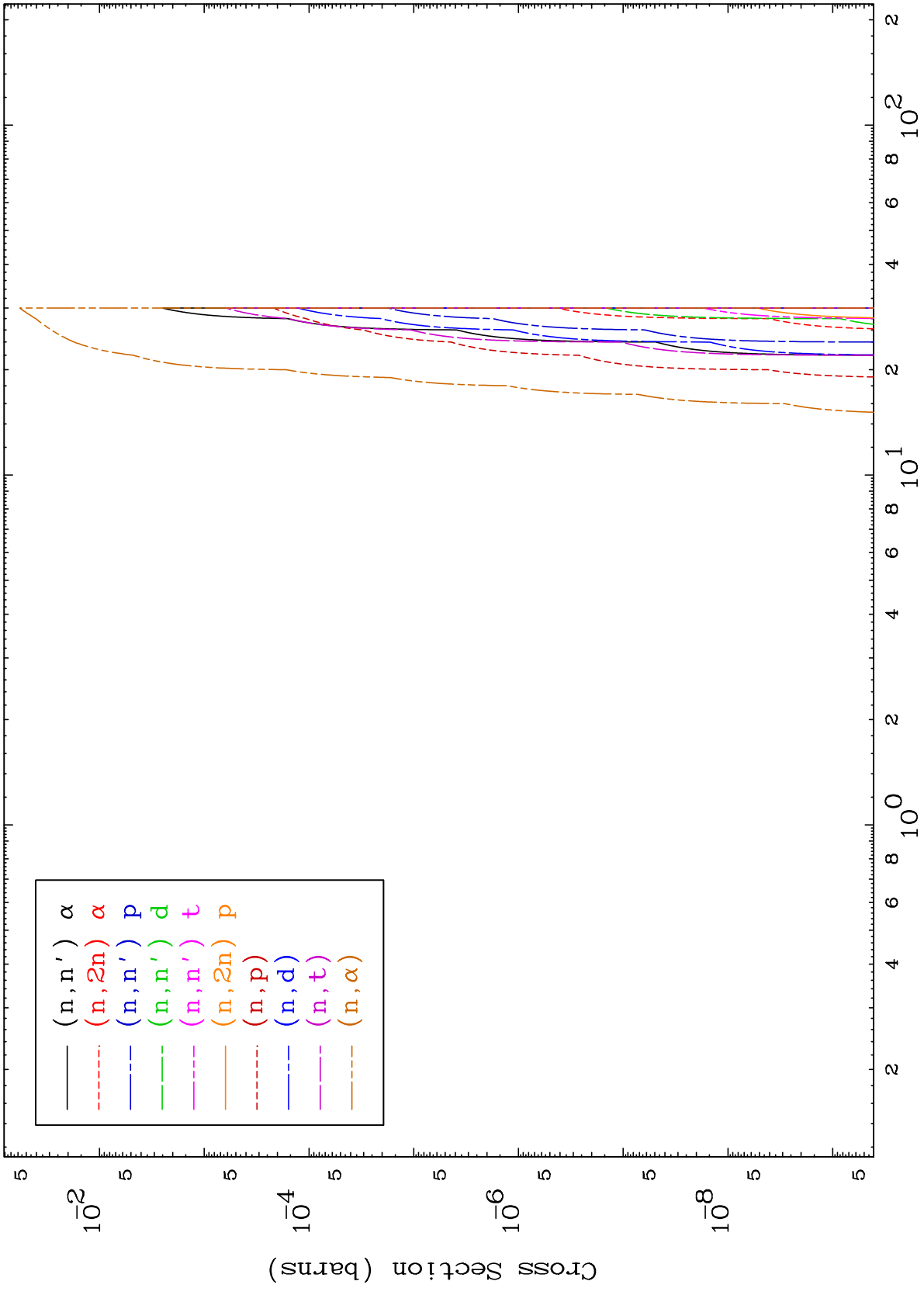


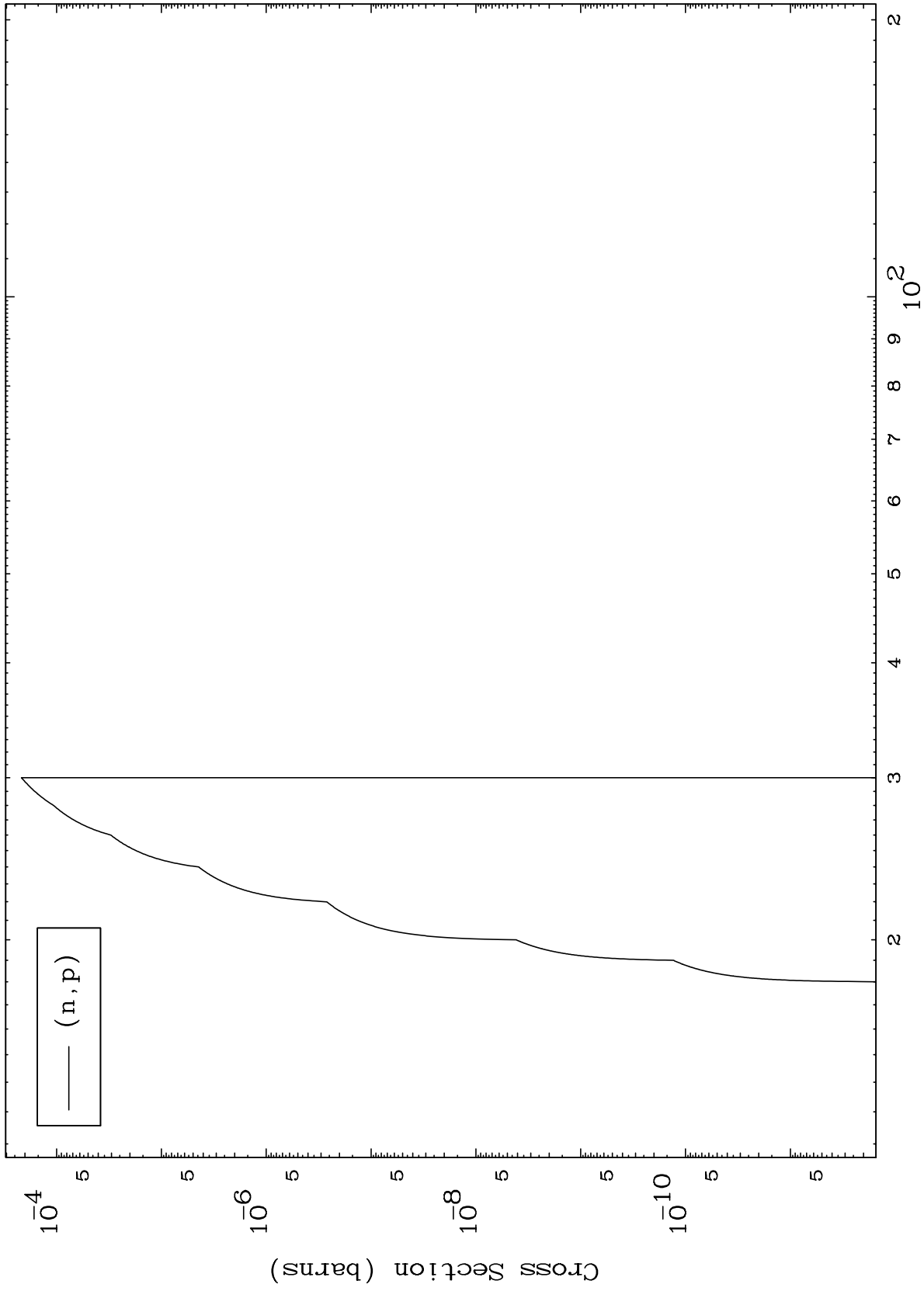
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$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

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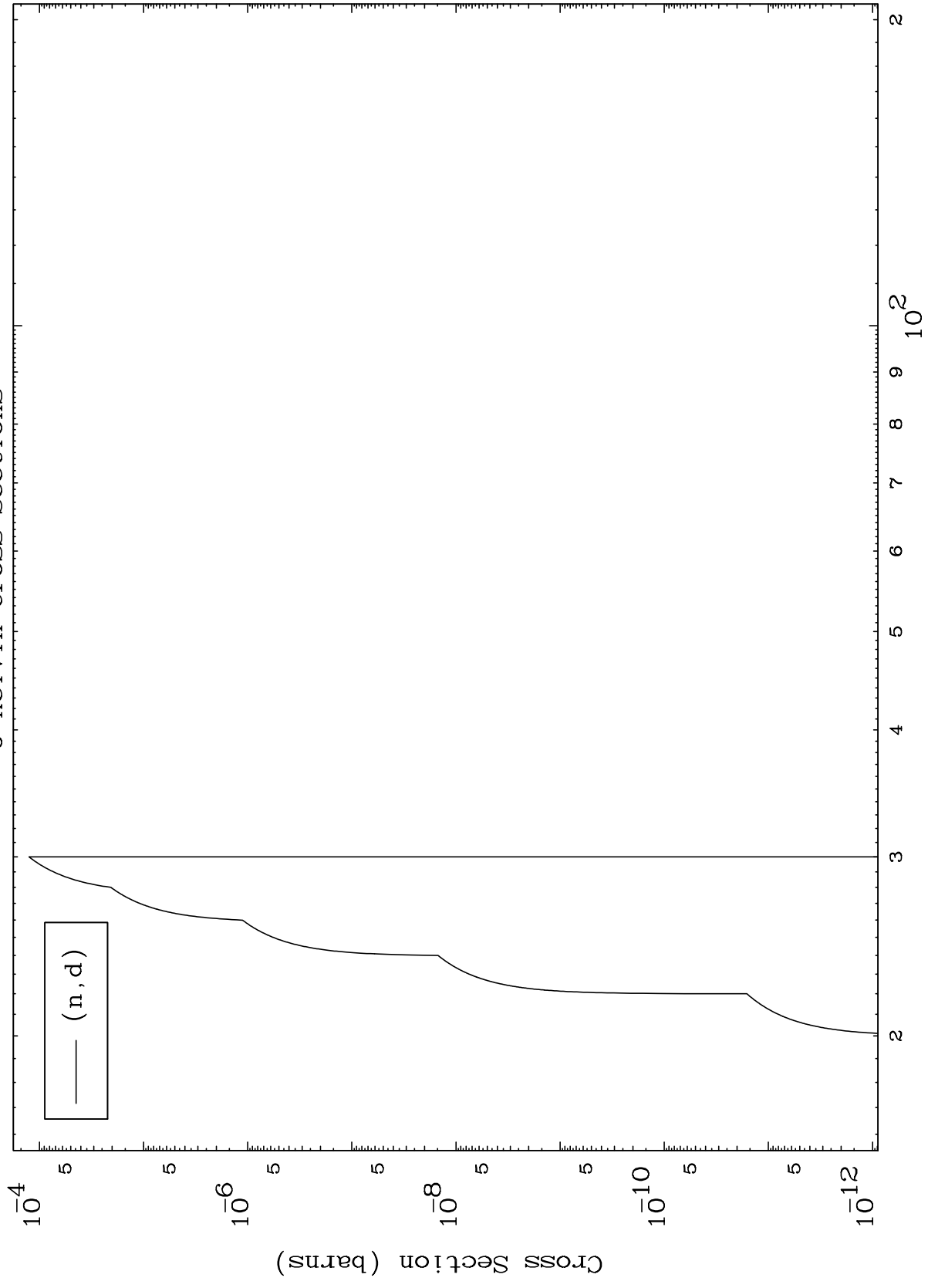




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( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

92-U -238



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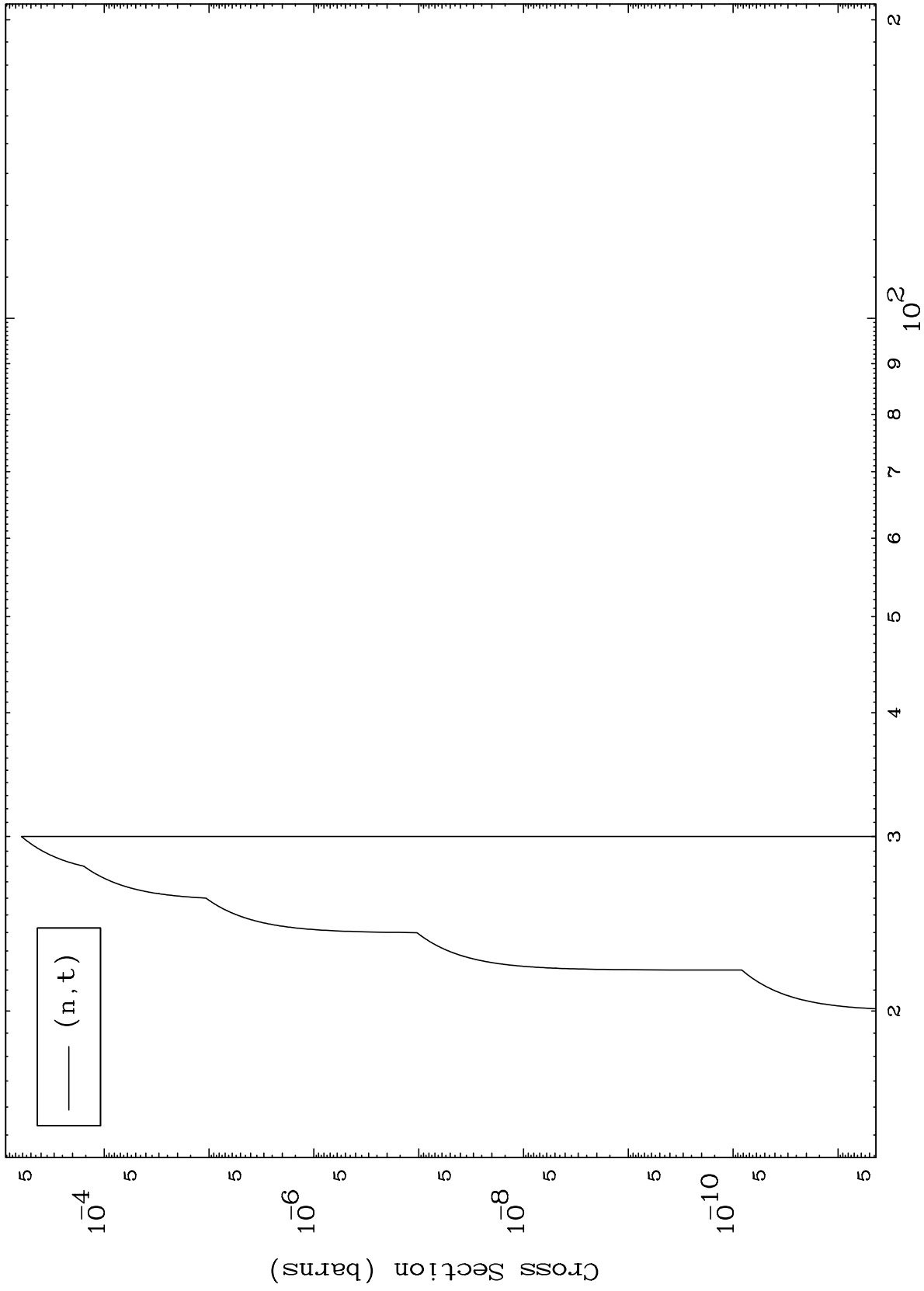
Incident Energy (MeV)

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( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

92-U -238



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Incident Energy (MeV)

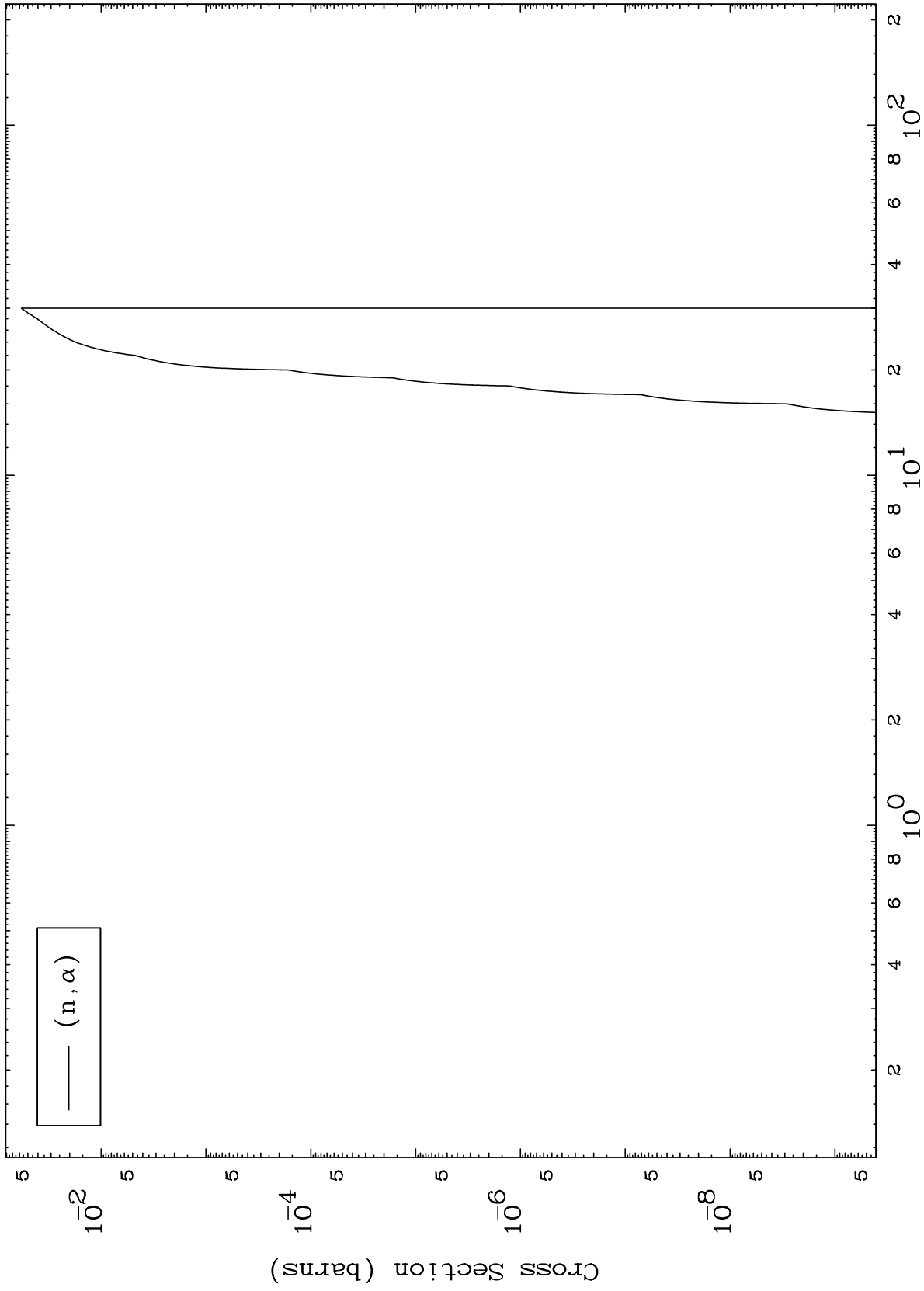
92-U -238



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( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

92-U -238



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Incident Energy (MeV)

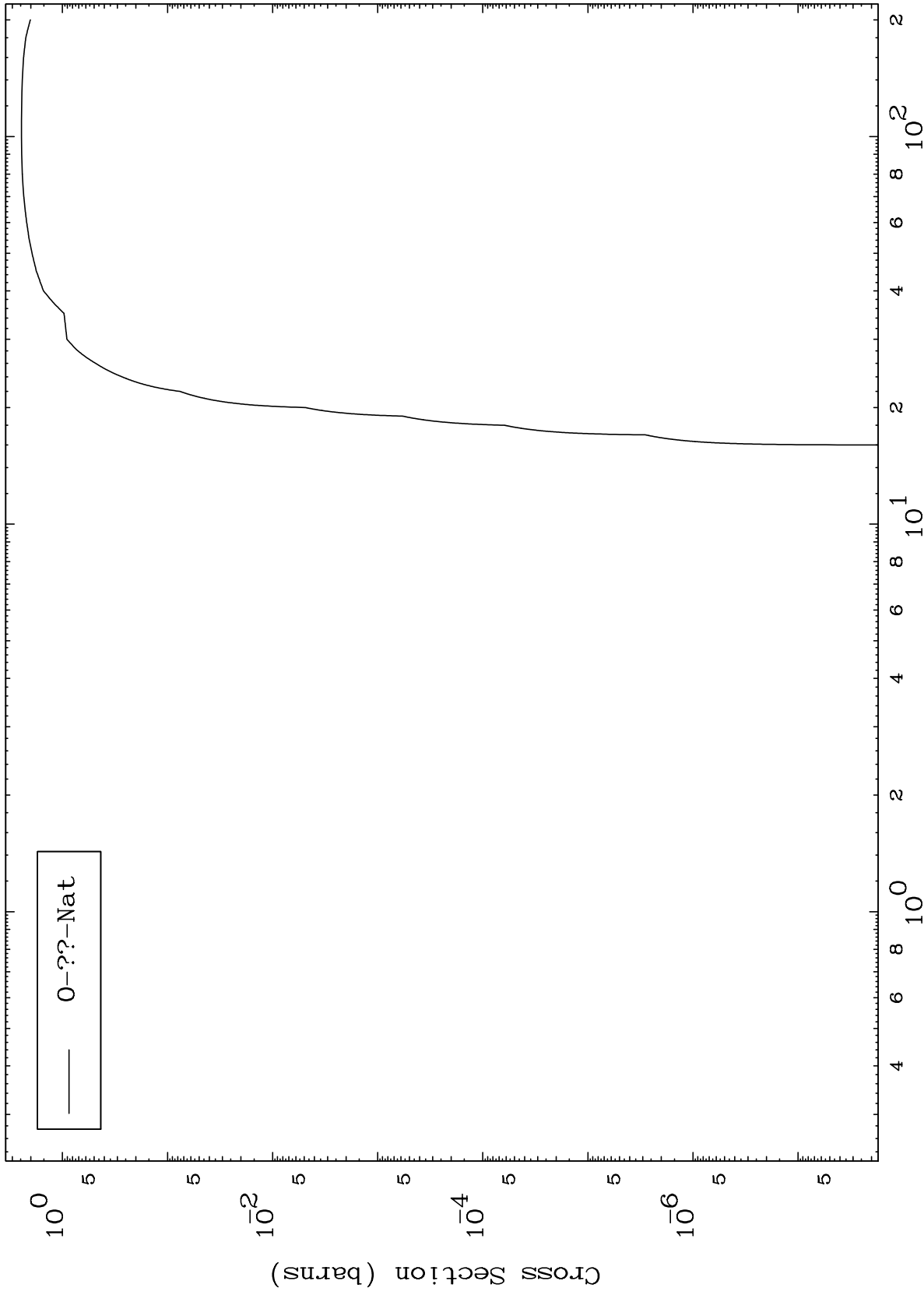
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Fission

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Radionuclide Production Cross Section

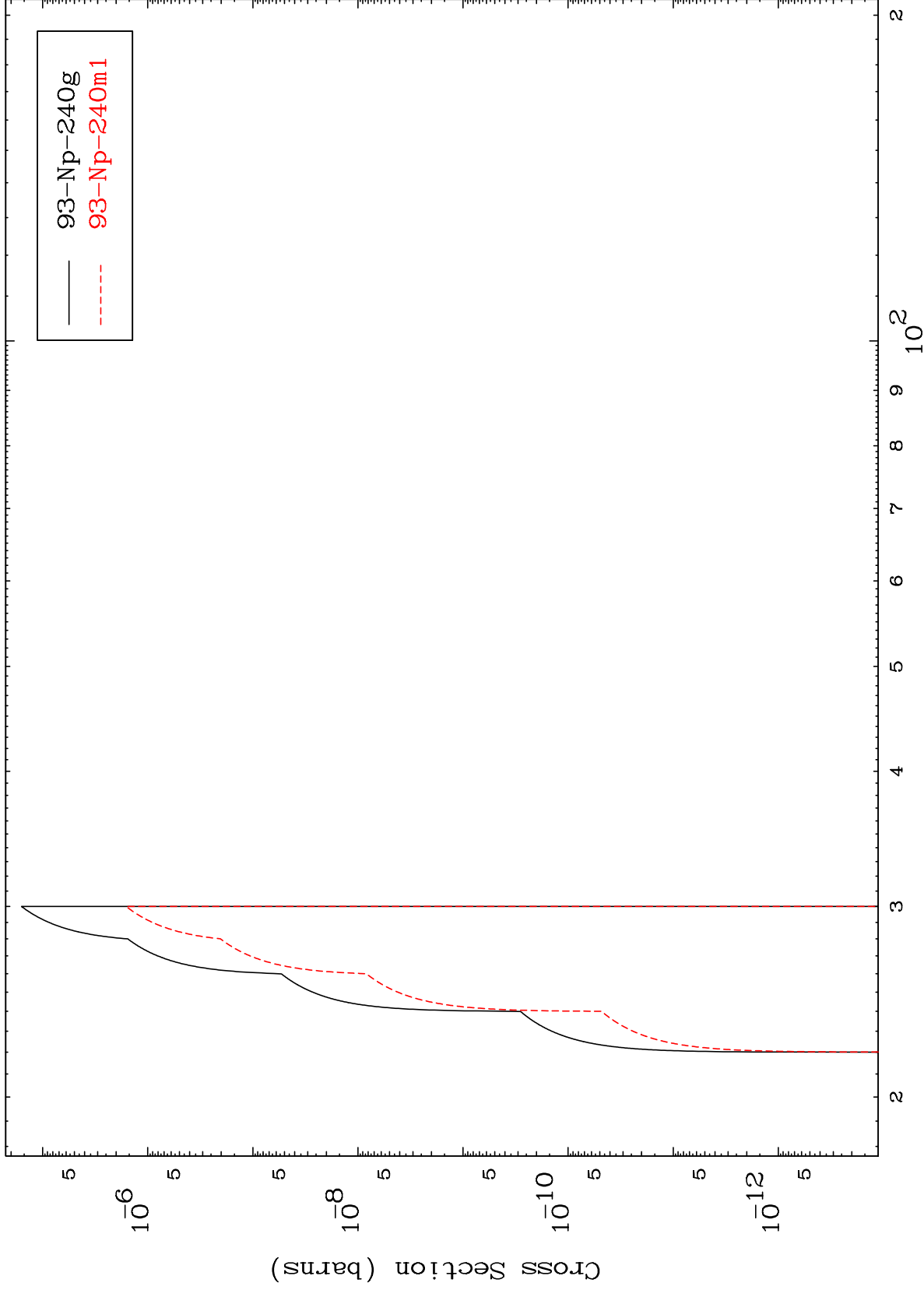


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(n,n') p

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Radionuclide Production Cross Section



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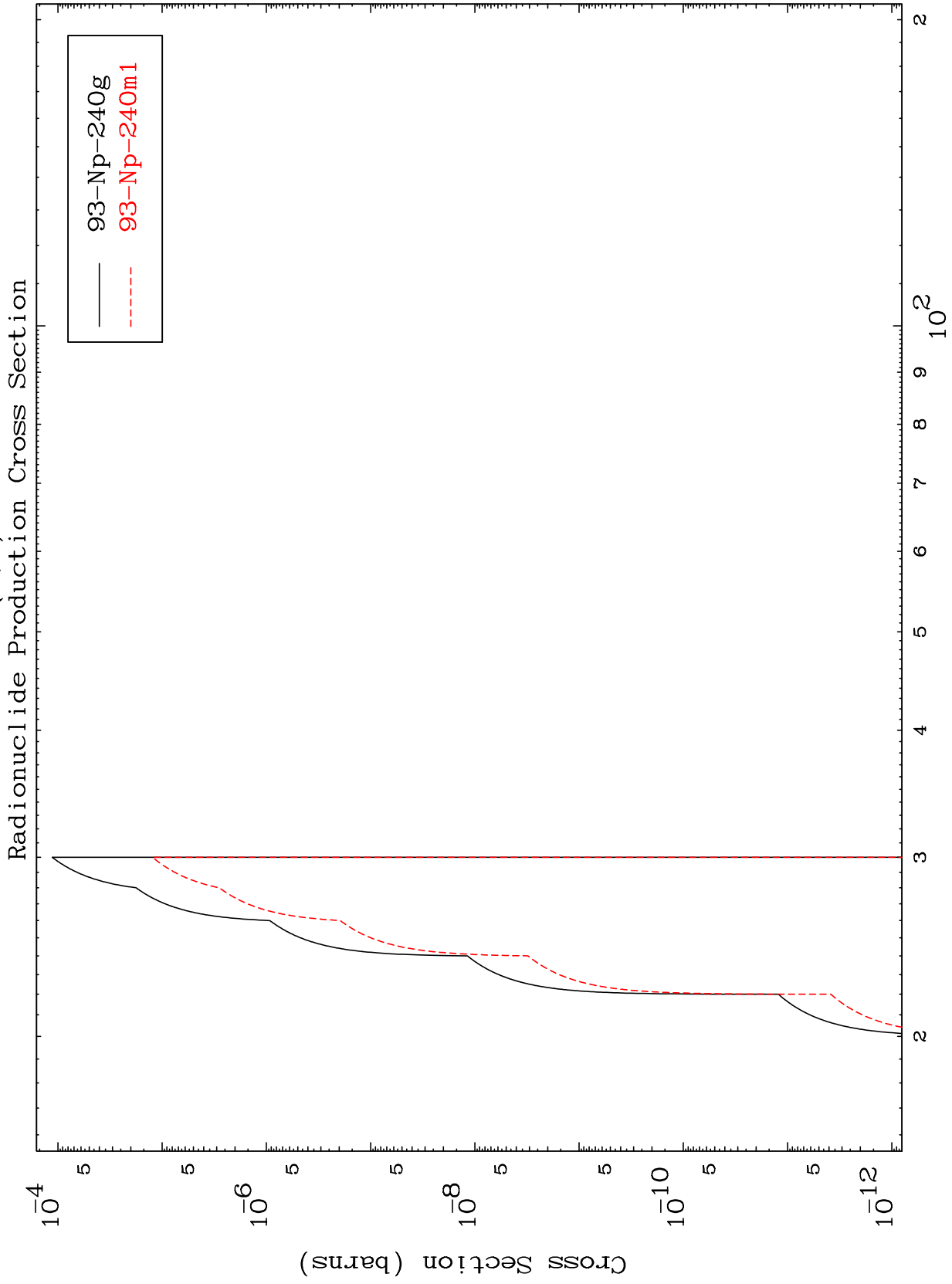
Incident Energy (MeV)

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(n,d)

92-U -238



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Incident Energy (MeV)

92-U -238