

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)

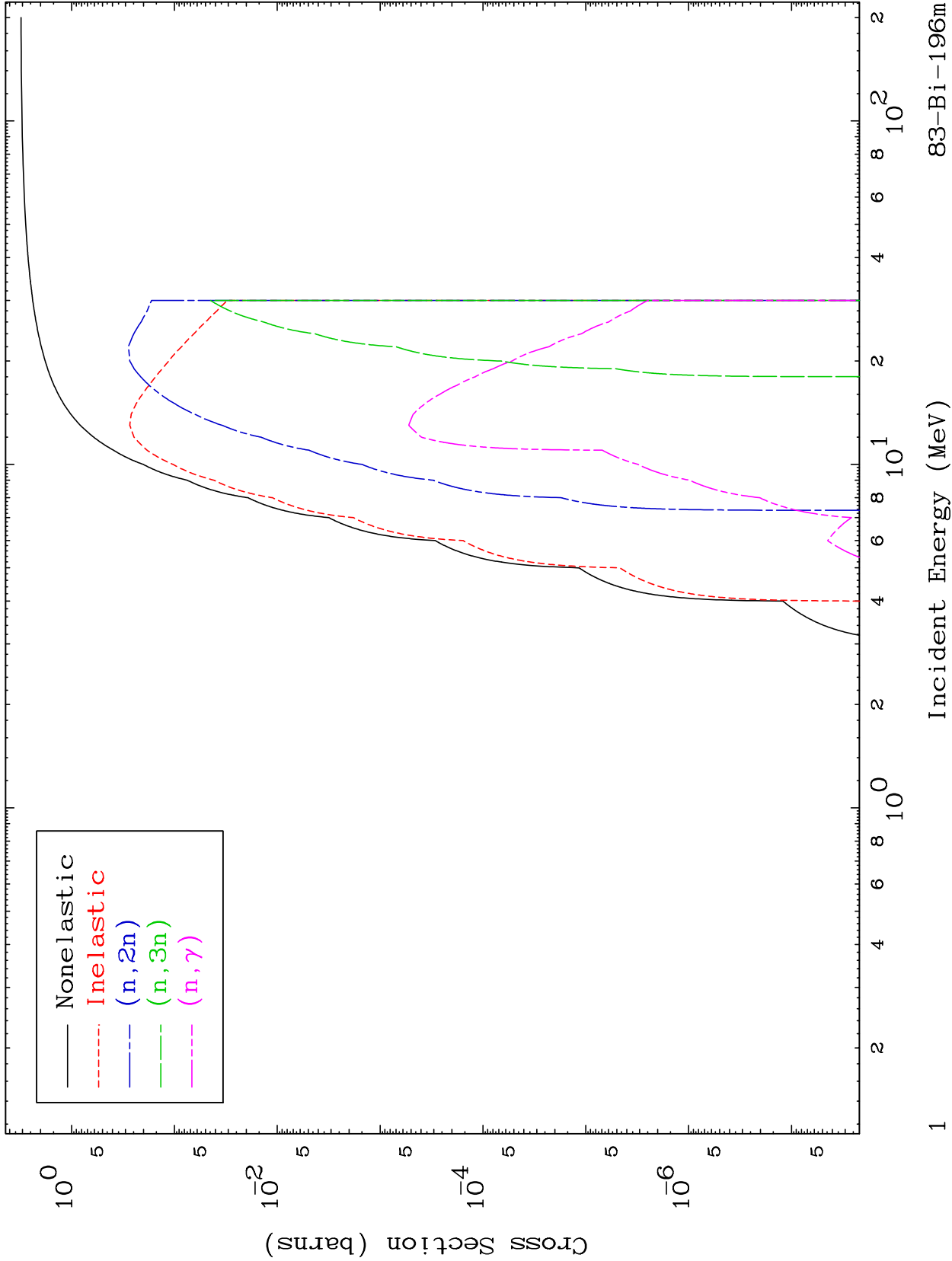
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 8287

Deuteron Major  
0 Kelvin Cross Sections

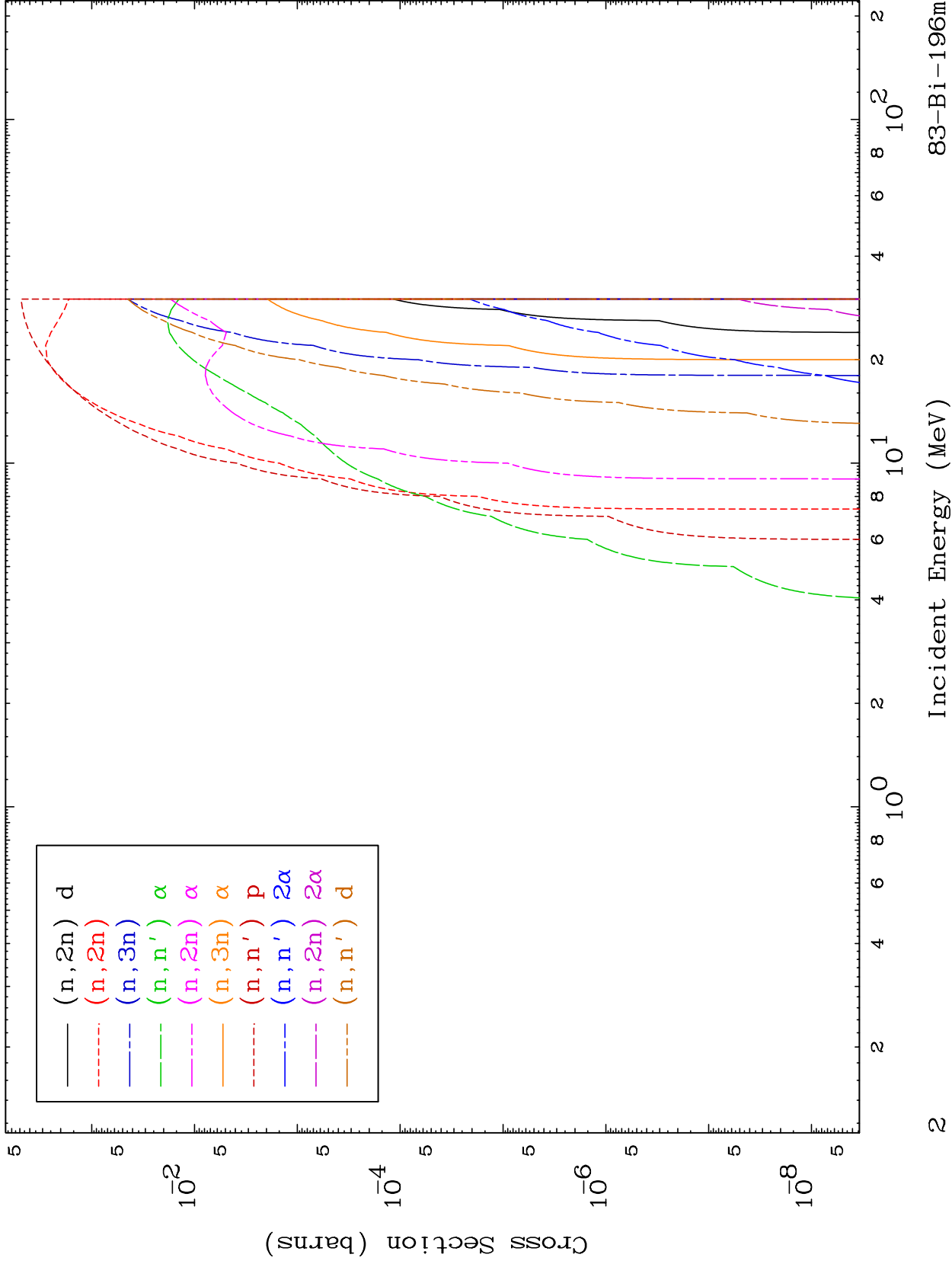
83-Bi-196m

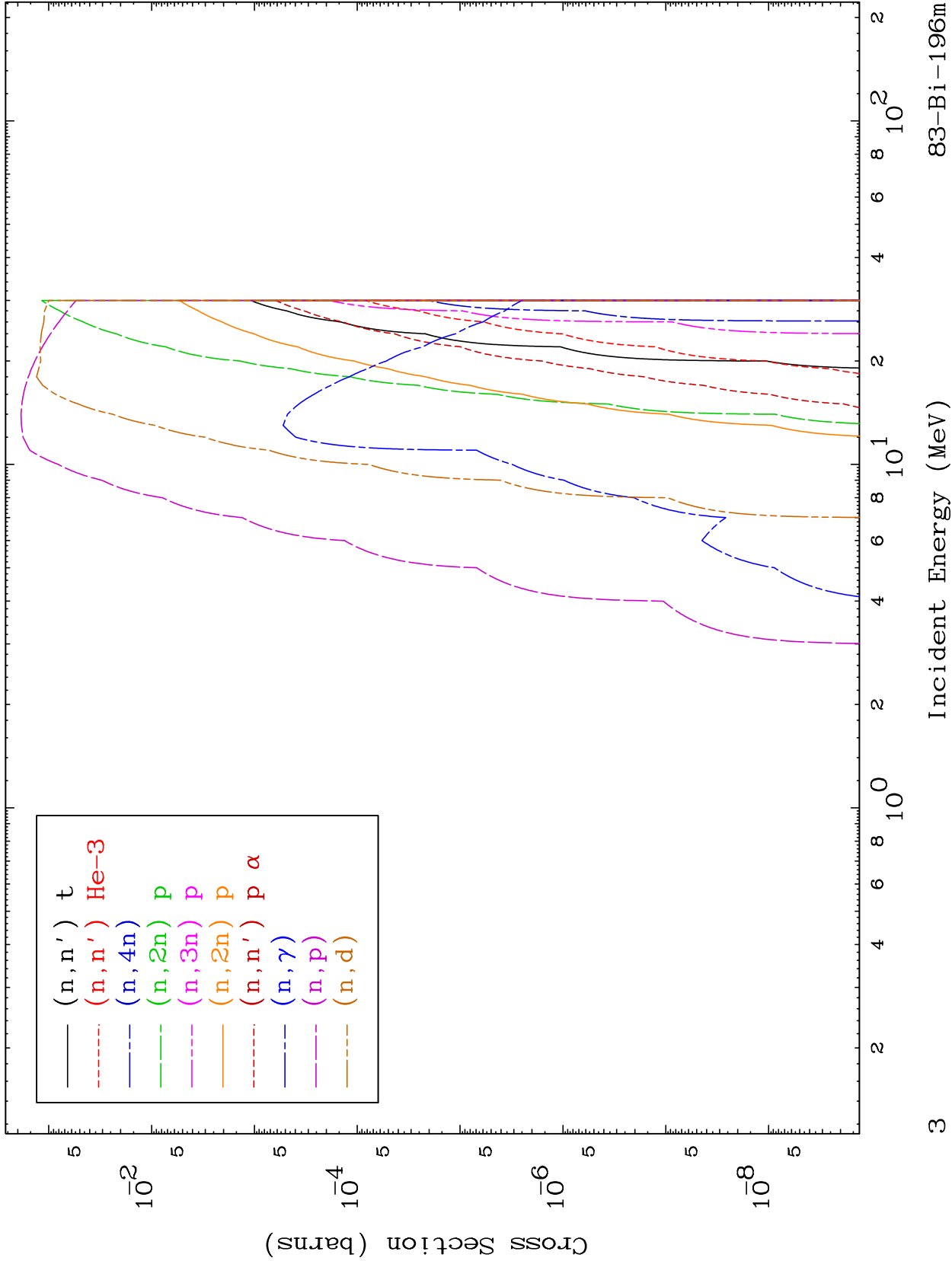


MAT 8287

Deuteron Neutron Absorption  
0 Kelvin Cross Sections

83-Bi-196m

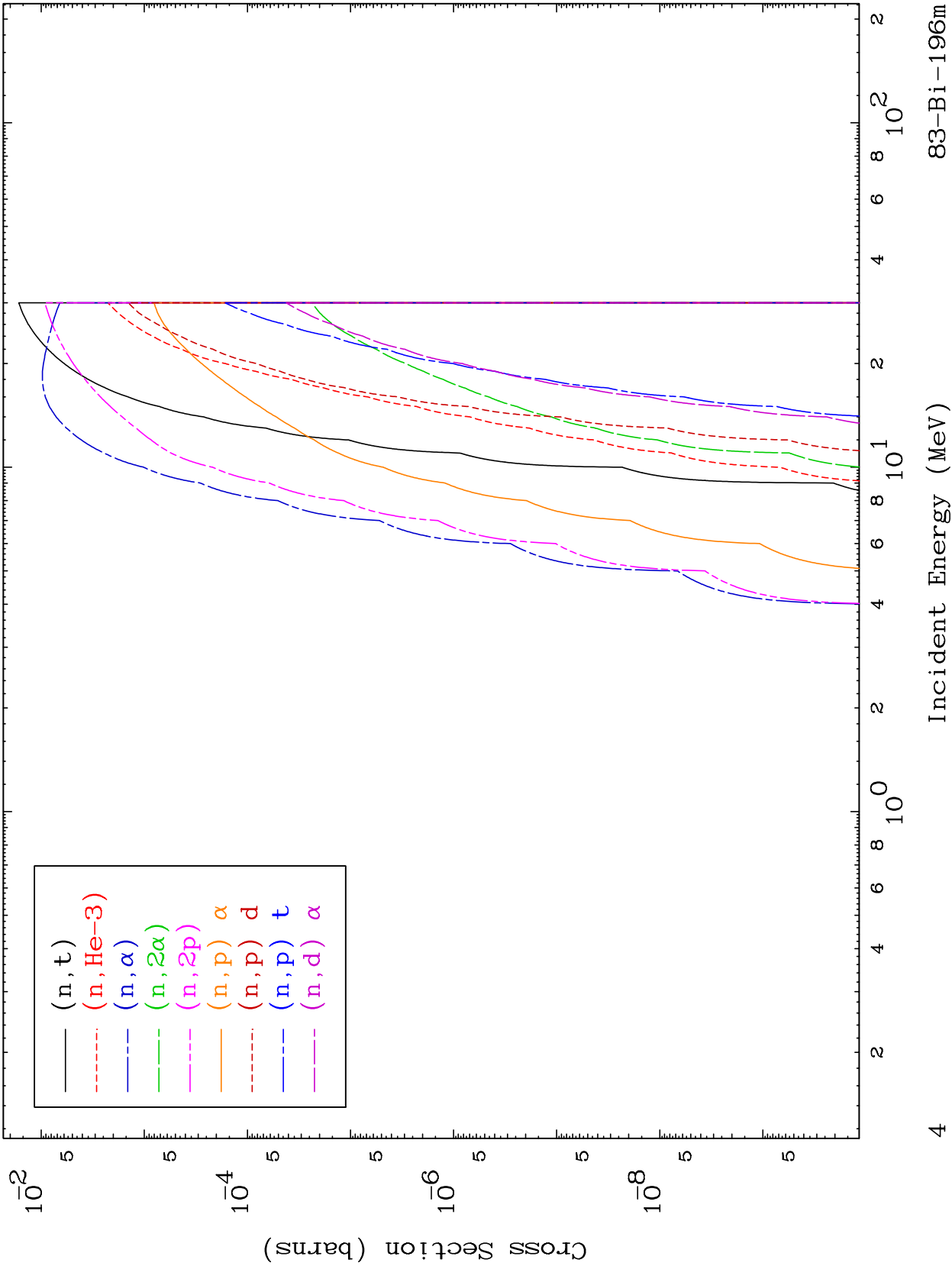




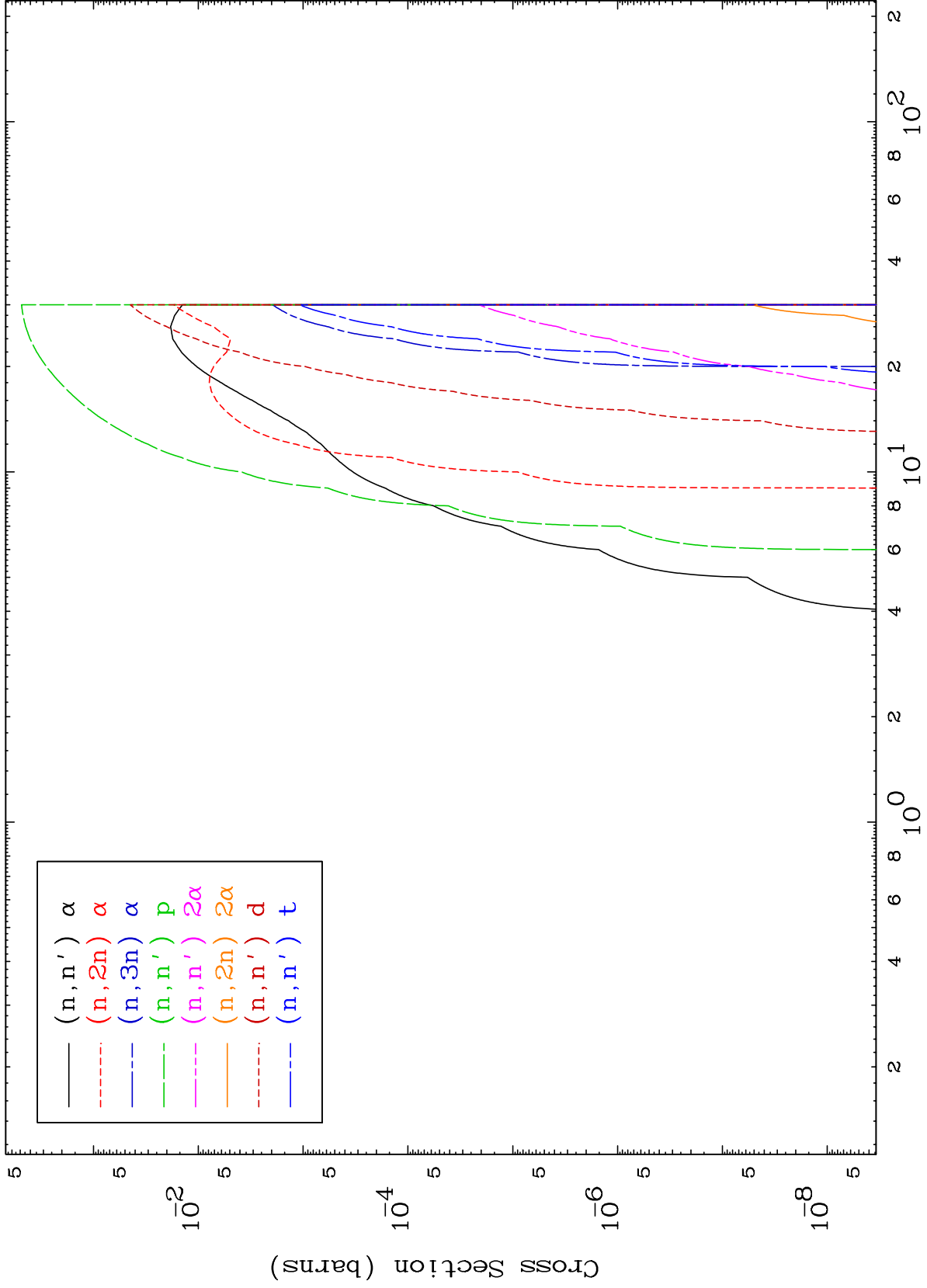
MAT 8287

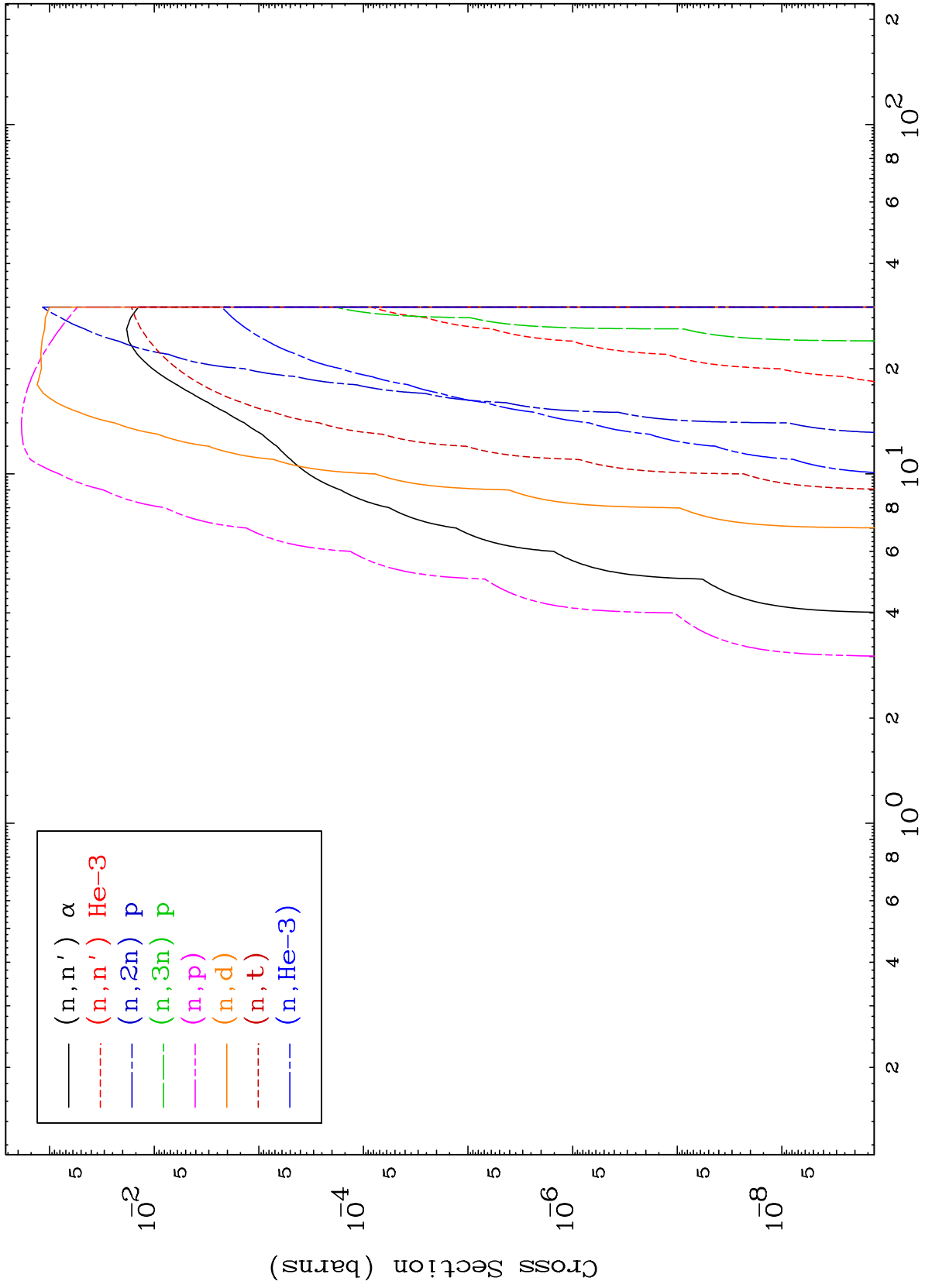
Deuteron Neutron Absorption  
0 Kelvin Cross Sections

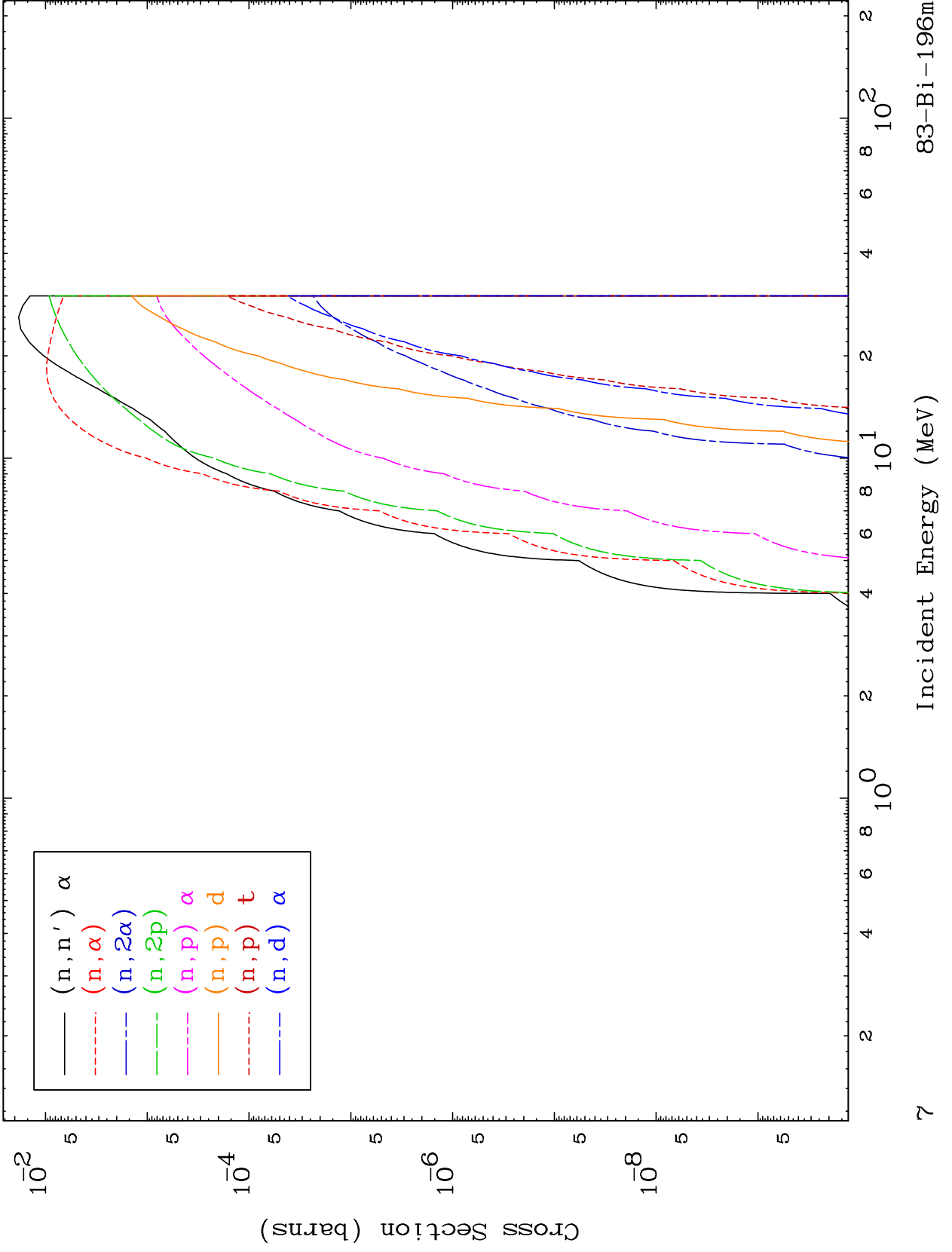
83-Bi-196m



83-Bi-196m







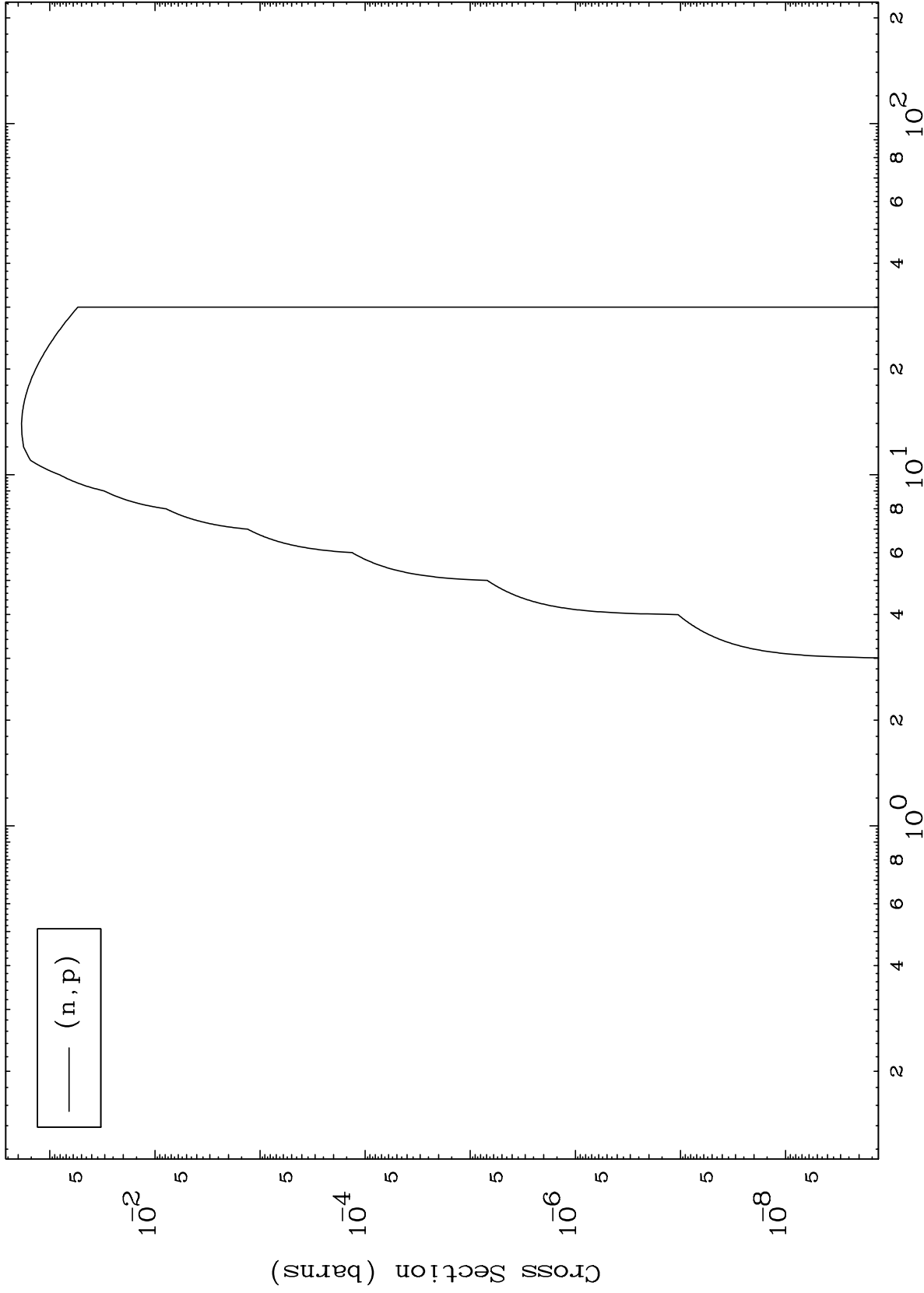


MAT 8287

(d,p) Levels

83-Bi-196m

0 Kelvin Cross Sections

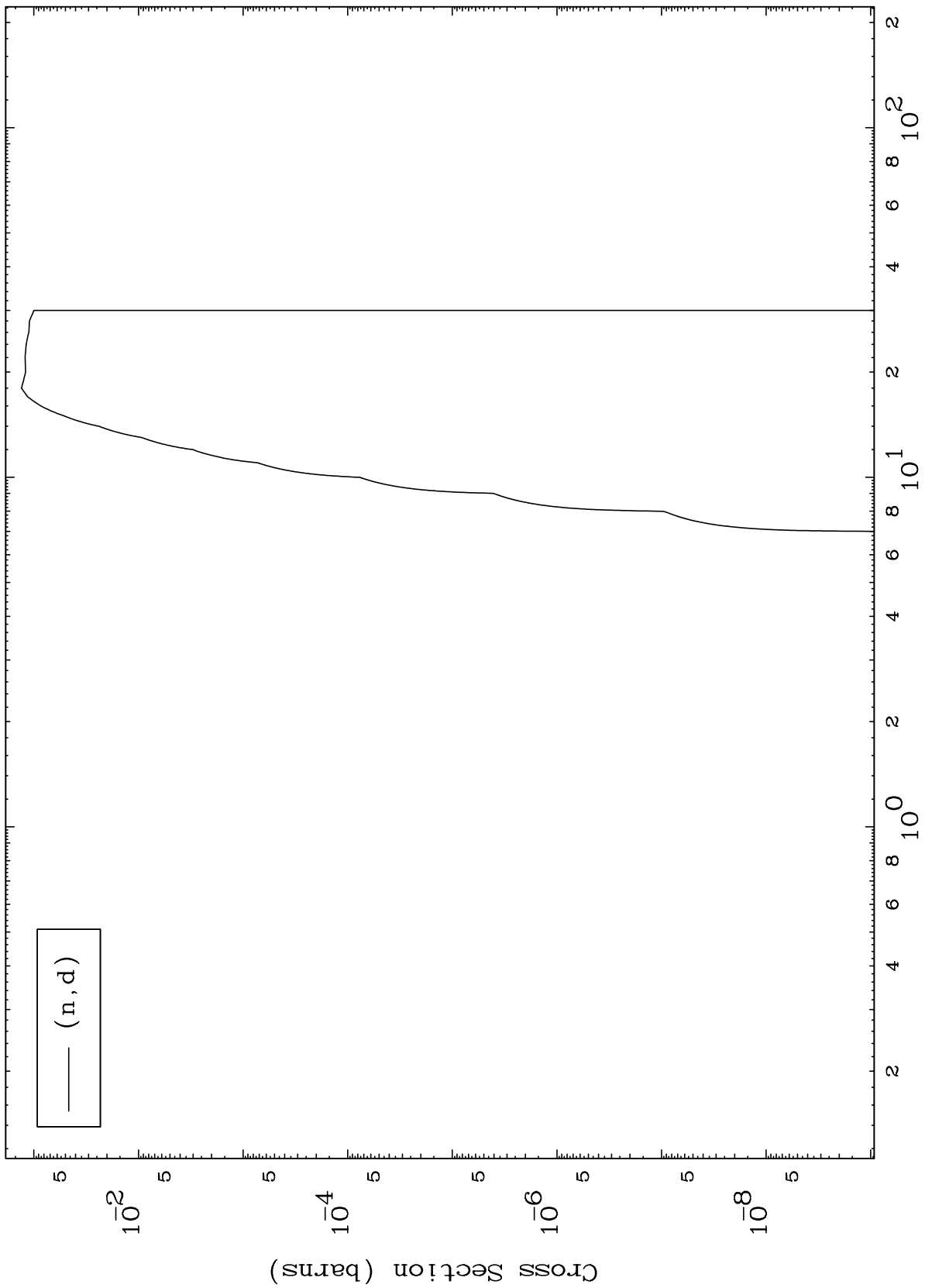


MAT 8287

(d,d) Levels

83-Bi-196m

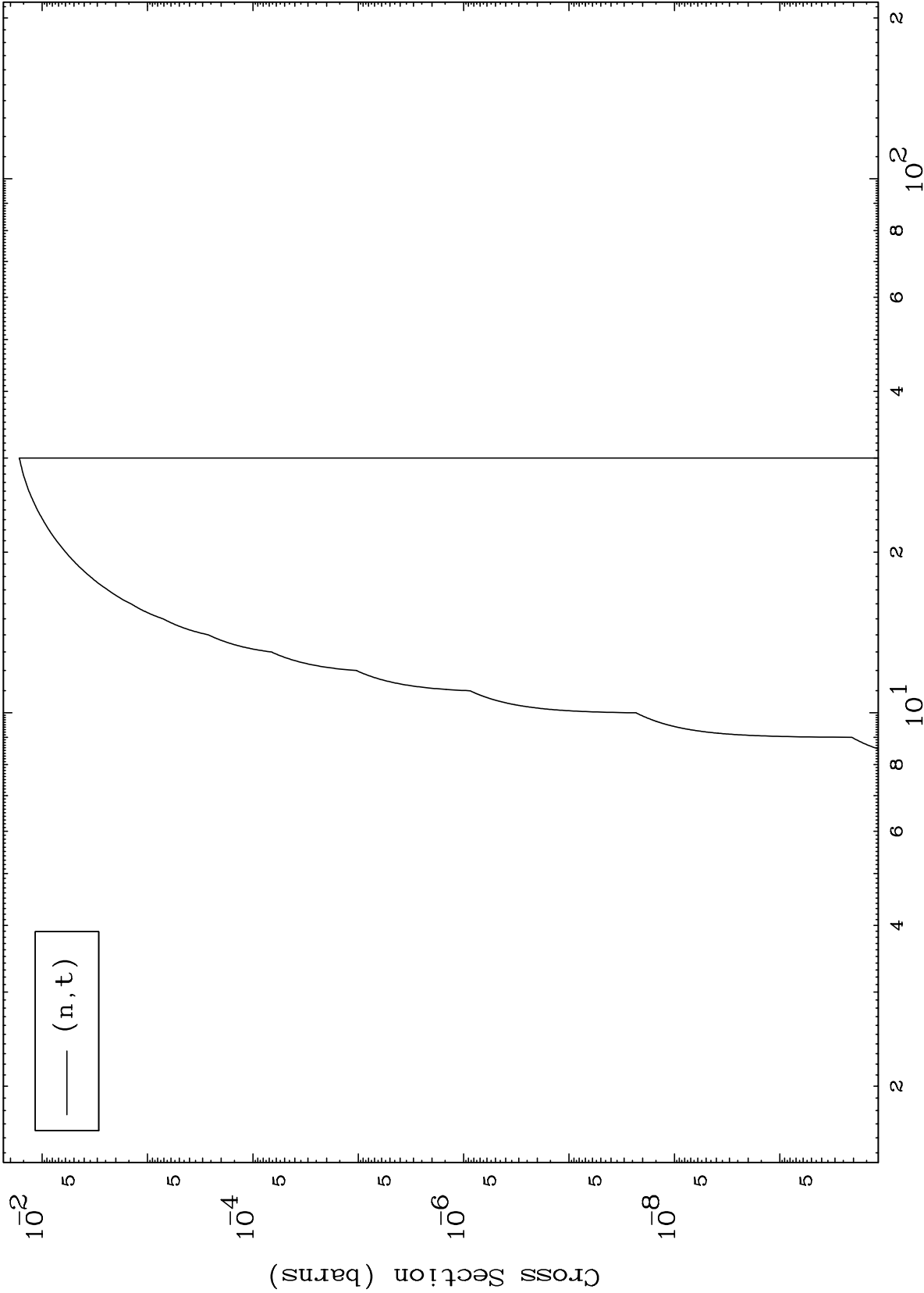
0 Kelvin Cross Sections



MAT 8287

(d,t) Levels  
0 Kelvin Cross Sections

83-Bi-196m



10

Incident Energy (MeV)

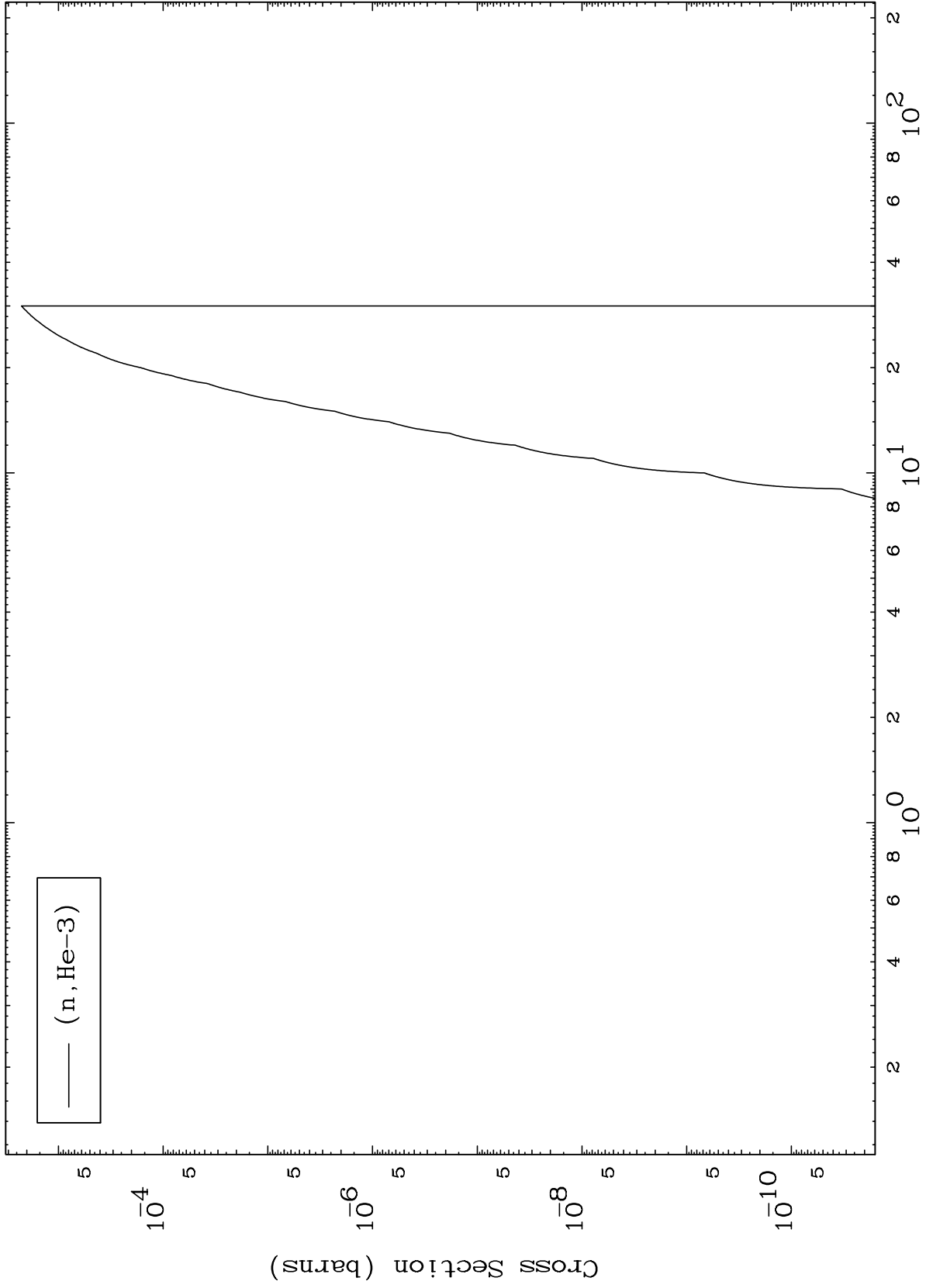
83-Bi-196m

MAT 8287

(d,He3) Levels

83-Bi-196m

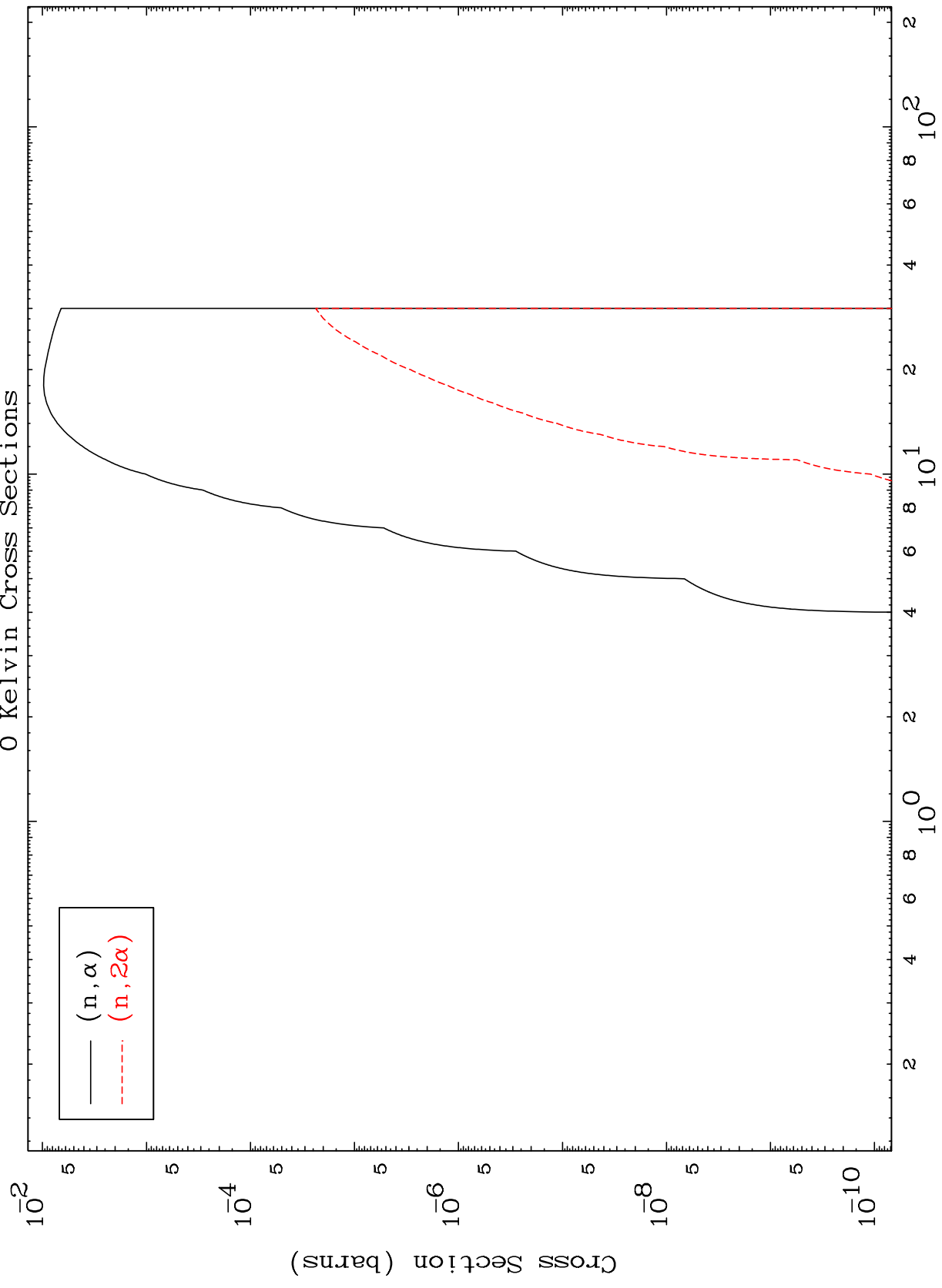
0 Kelvin Cross Sections



MAT 8287

$^{83}\text{Bi-196m}$

(d,  $\alpha$ ) Levels  
0 Kelvin Cross Sections



$^{83}\text{Bi-196m}$

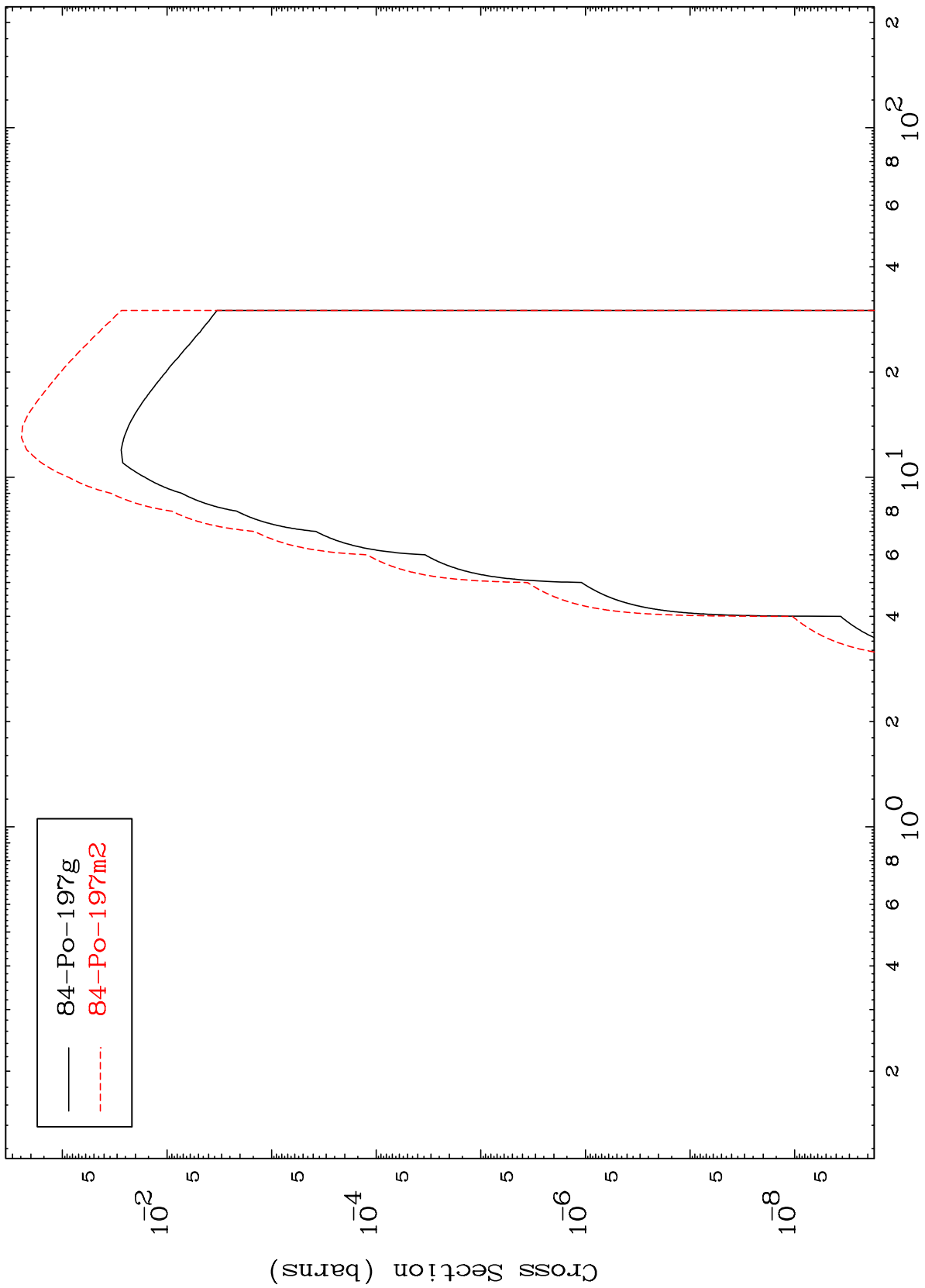
Incident Energy (MeV)

12

MAT 8287

83-Bi-196m

Inelastic  
Radionuclide Production Cross Section



84-Po-197g  
84-Po-197m2

83-Bi-196m

Incident Energy (MeV)

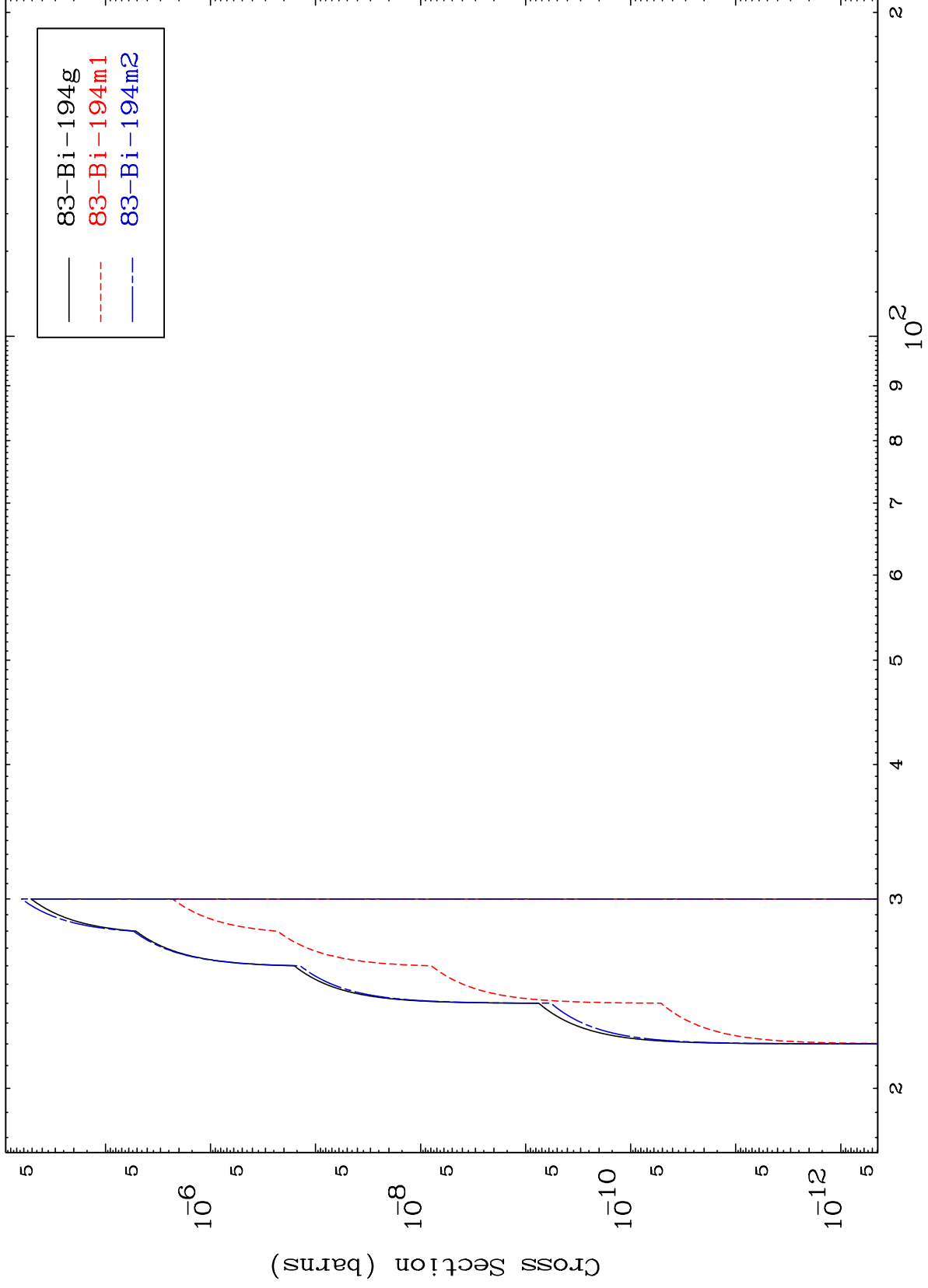
13

MAT 8287

(n,2n) d

83-Bi-196m

Radionuclide Production Cross Section



14

Incident Energy (MeV)

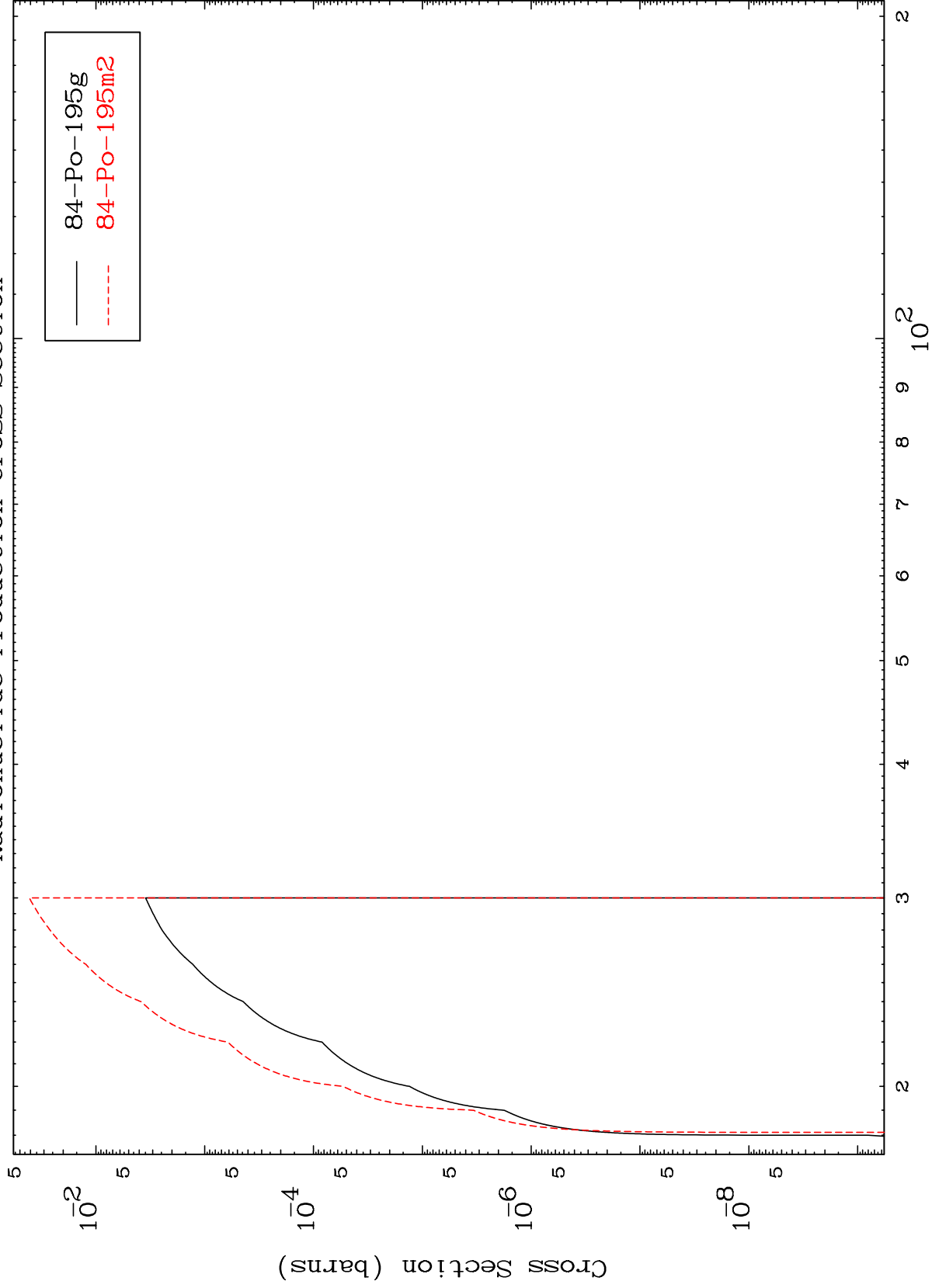
83-Bi-196m

MAT 8287

(n,3n)

83-Bi-196m

Radionuclide Production Cross Section



15

Incident Energy (MeV)

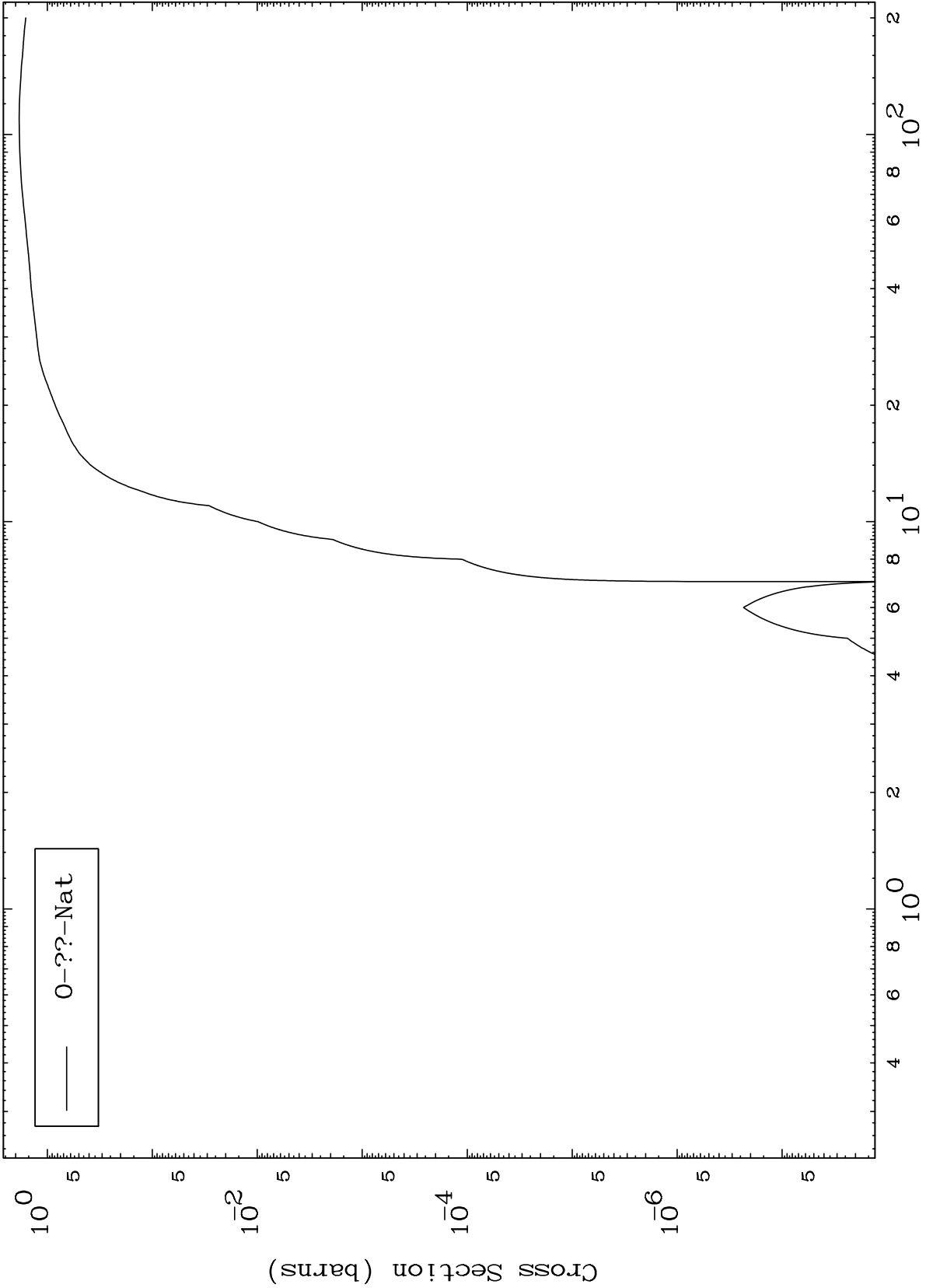
83-Bi-196m



MAT 8287

83-Bi-196m

Fission  
Radionuclide Production Cross Section



83-Bi-196m

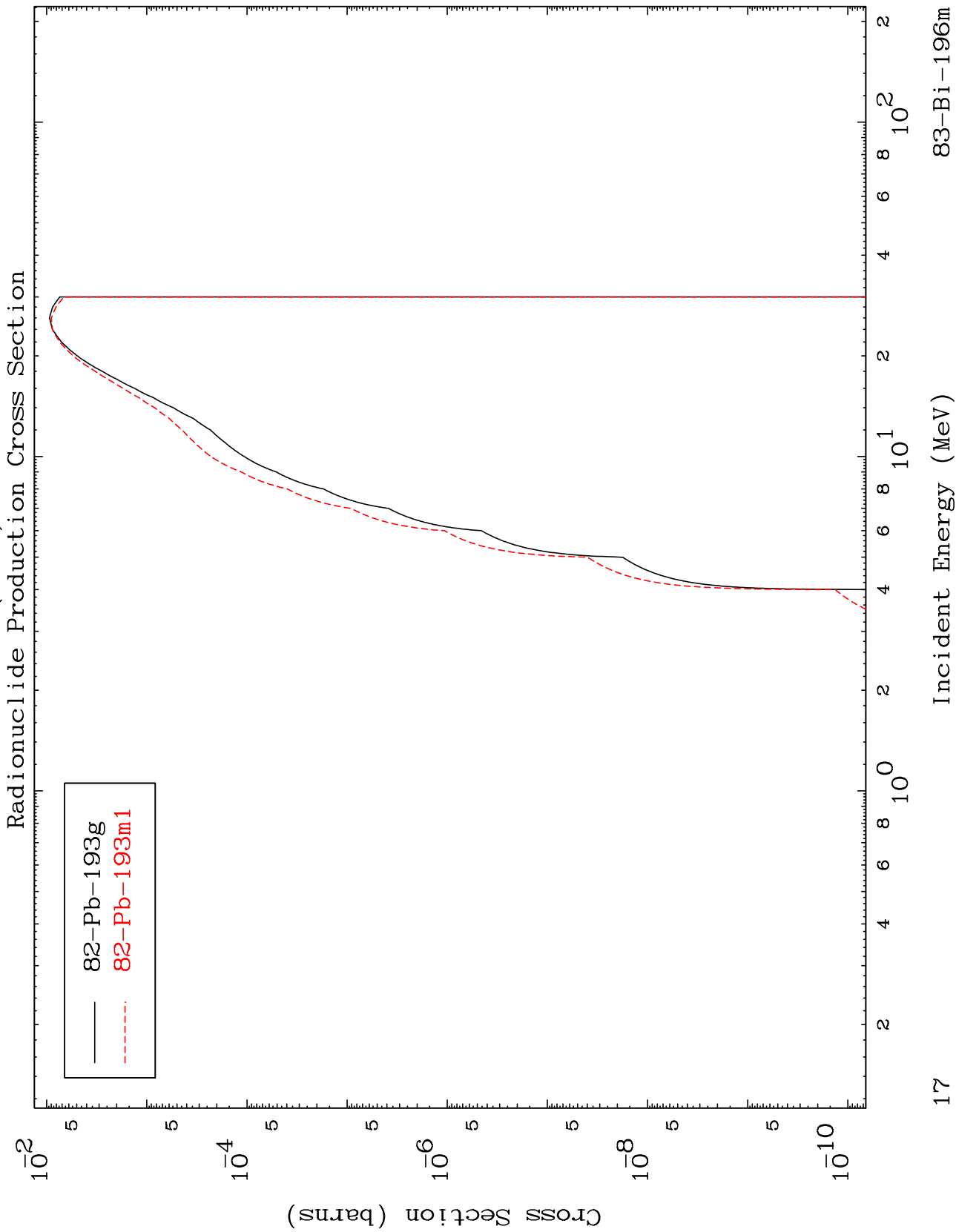
Incident Energy (MeV)

16

MAT 8287

$(n, n') \alpha$

83-Bi-196m

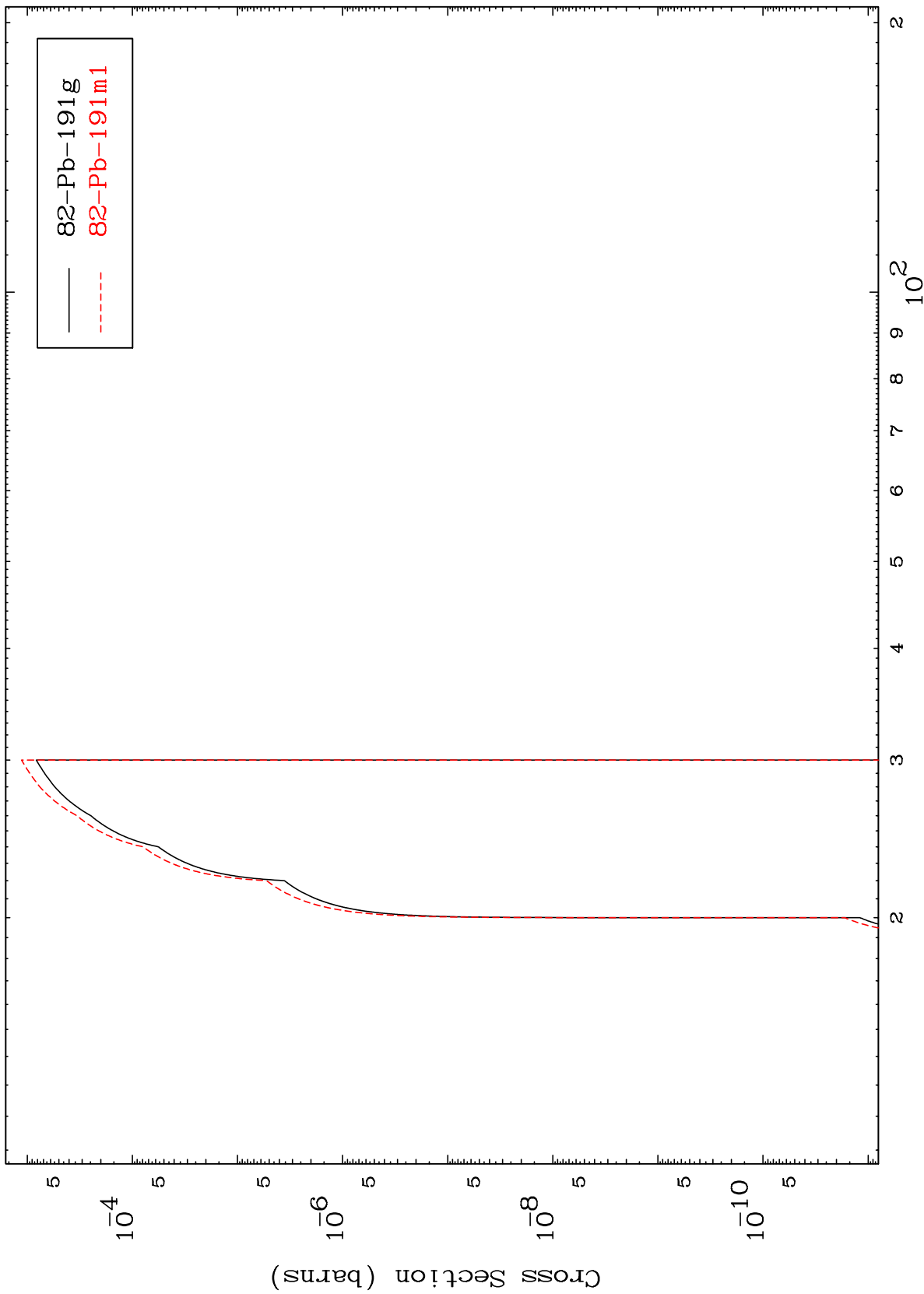


MAT 8287

(n,3n)  $\alpha$

83-Bi-196m

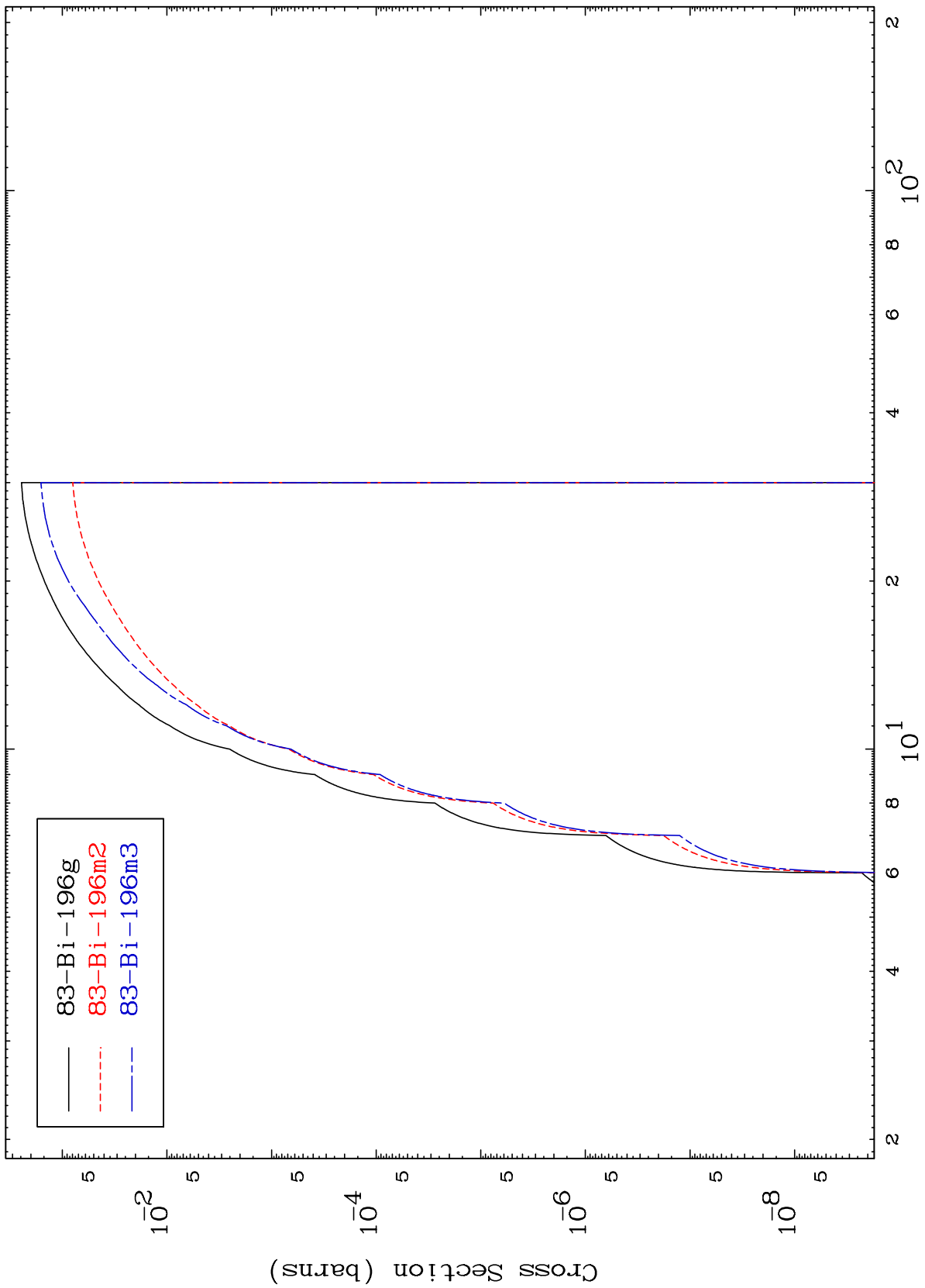
Radionuclide Production Cross Section



MAT 8287

$^{83}\text{Bi}-196\text{m}$

$(n, n')$  p  
Radionuclide Production Cross Section



19

$^{83}\text{Bi}-196\text{m}$

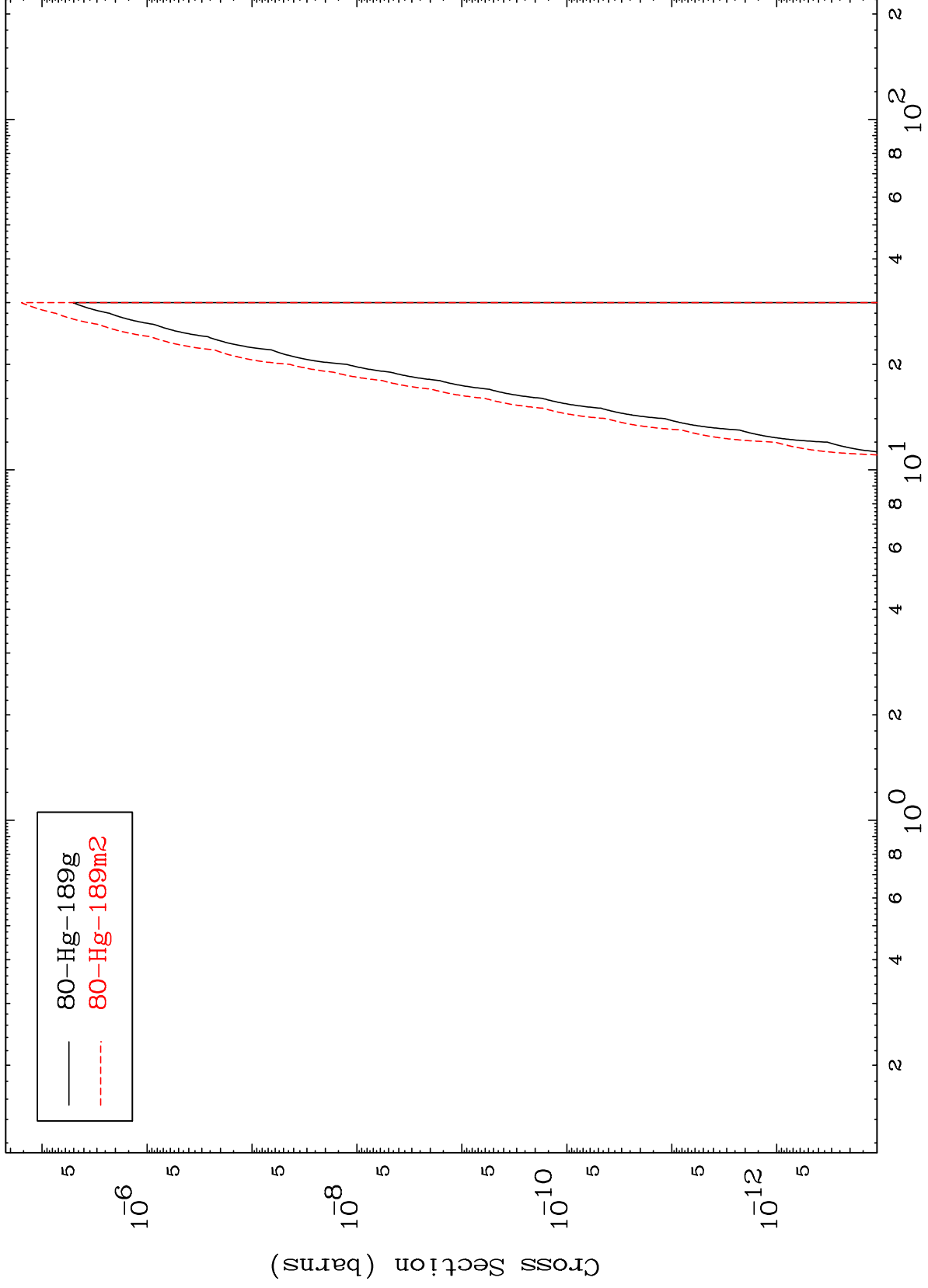
Incident Energy (MeV)

MAT 8287

$(n, n')$   $2\alpha$

$^{83}\text{Bi-196m}$

Radionuclide Production Cross Section



20

Incident Energy (MeV)

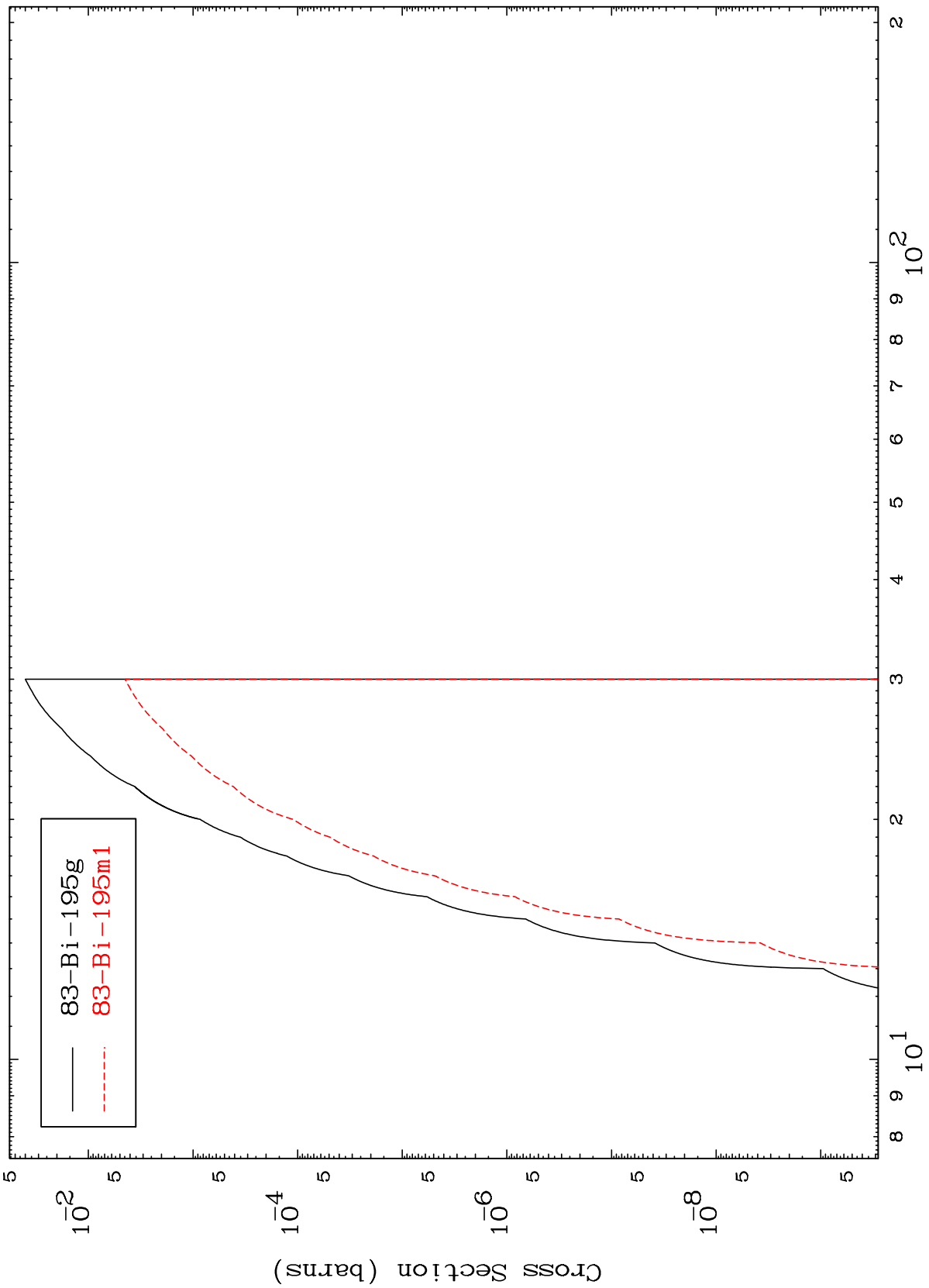
$^{83}\text{Bi-196m}$

MAT 8287

(n,n') d

83-Bi-196m

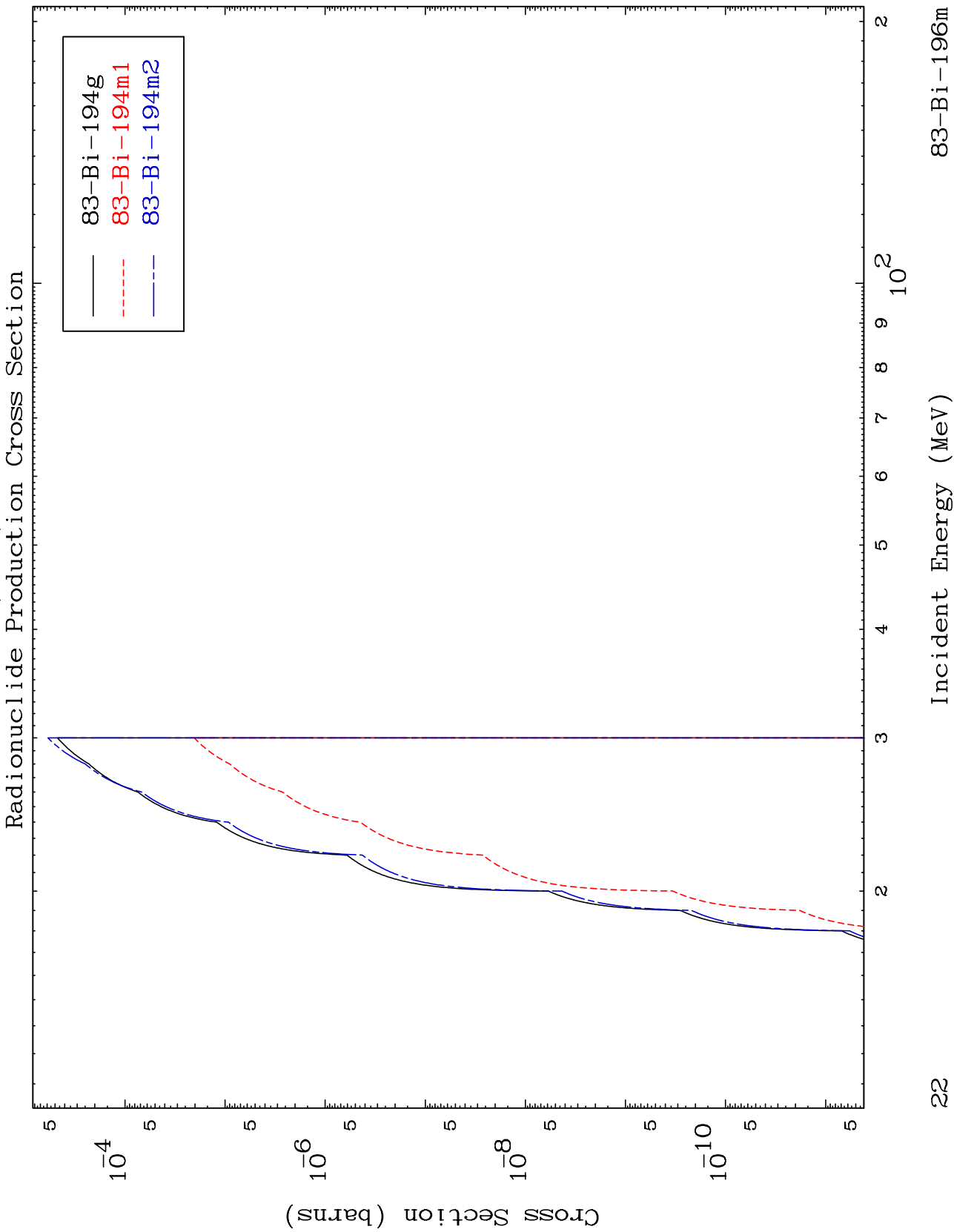
Radionuclide Production Cross Section



21

Incident Energy (MeV)

83-Bi-196m

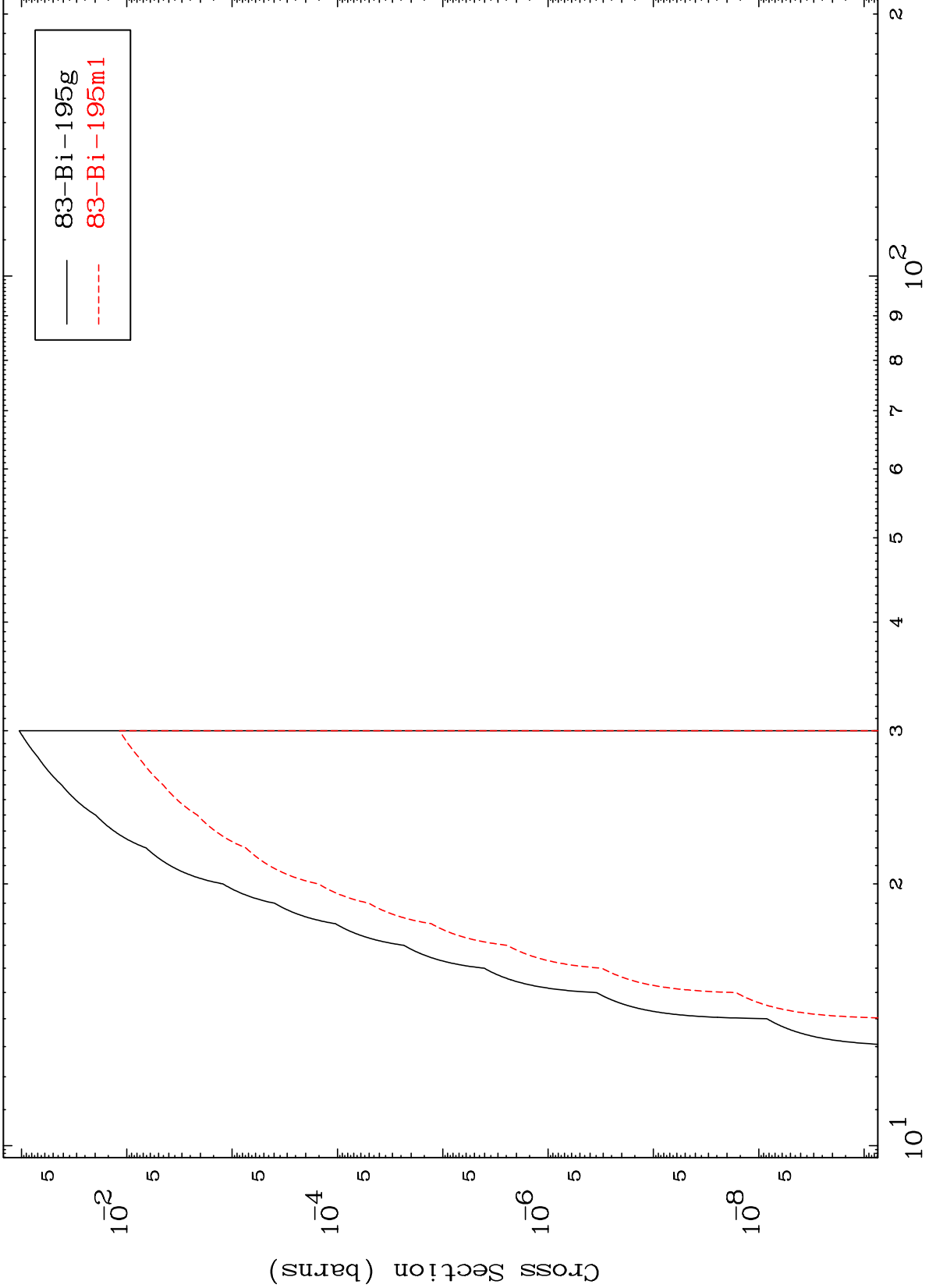


MAT 8287

$(n,2n)$  p

$^{83}\text{Bi}-196\text{m}$

Radionuclide Production Cross Section



23

Incident Energy (MeV)

$^{83}\text{Bi}-196\text{m}$

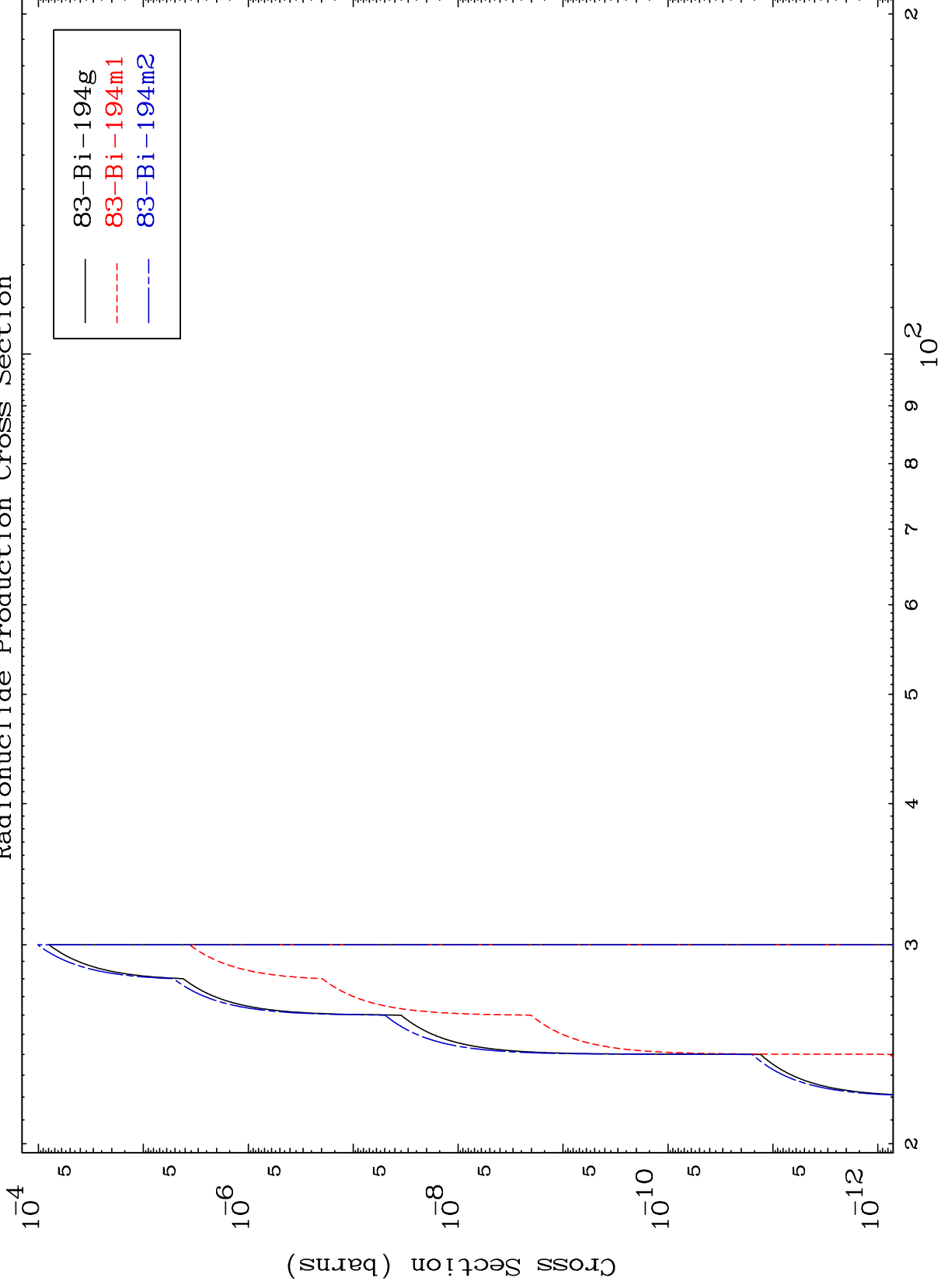


MAT 8287

$(n,3n)$  p

$^{83}\text{Bi}-196\text{m}$

Radionuclide Production Cross Section



24

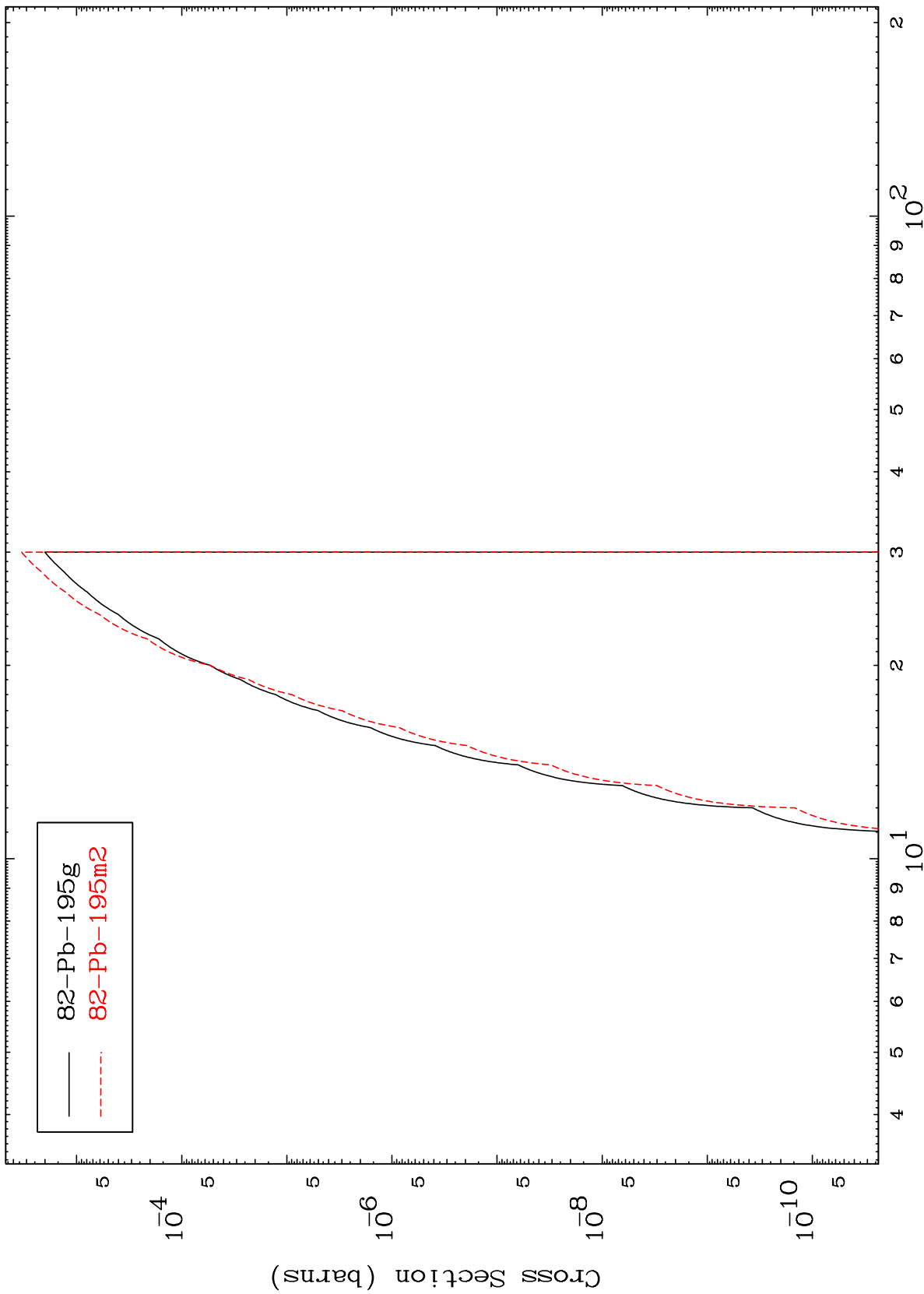
Incident Energy (MeV)

$^{83}\text{Bi}-196\text{m}$

MAT 8287

83-Bi-196m

(n,2n) p  
Radionuclide Production Cross Section



25

Incident Energy (MeV)

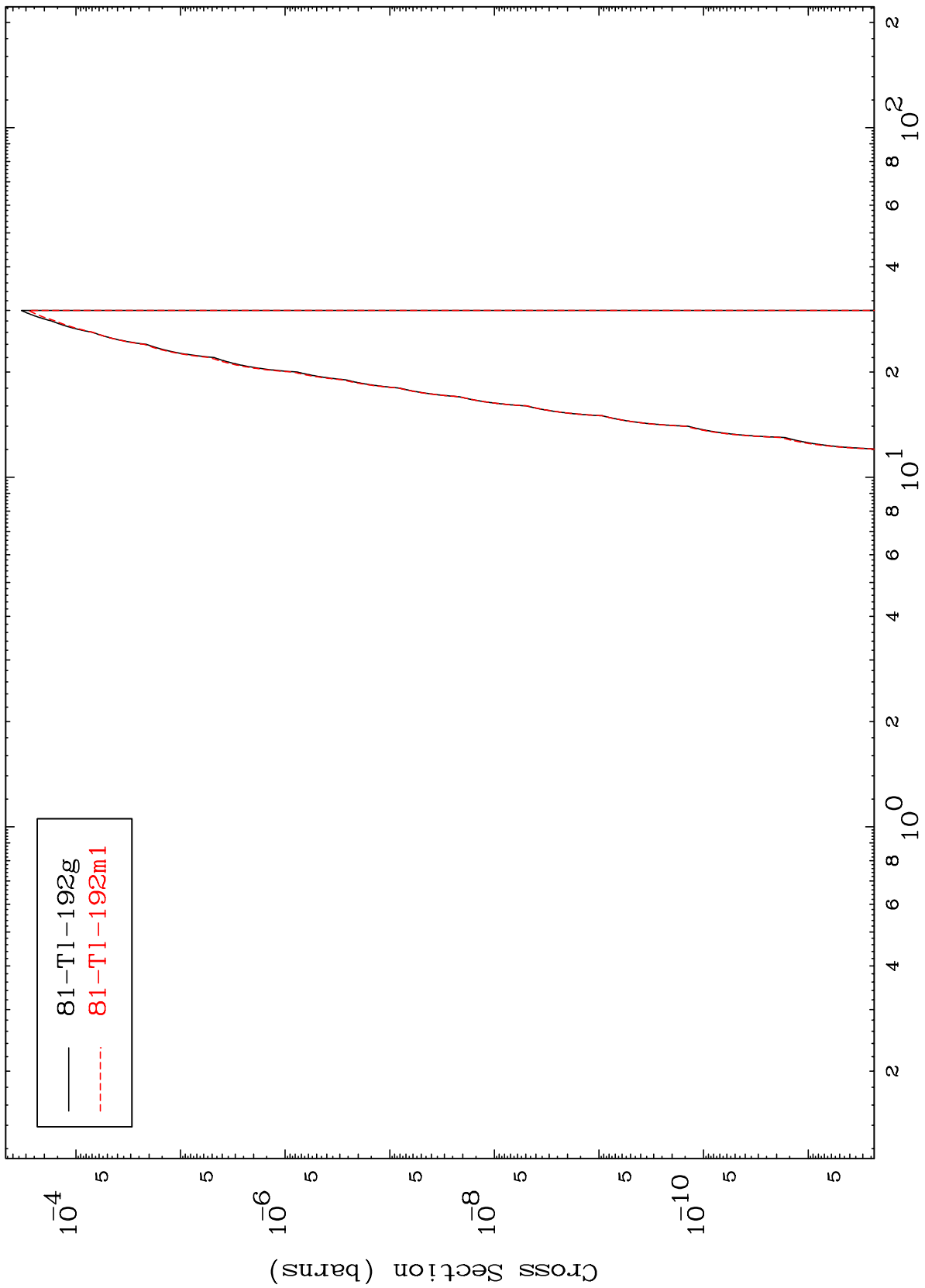
83-Bi-196m

MAT 8287

(n,n') p  $\alpha$

83-Bi-196m

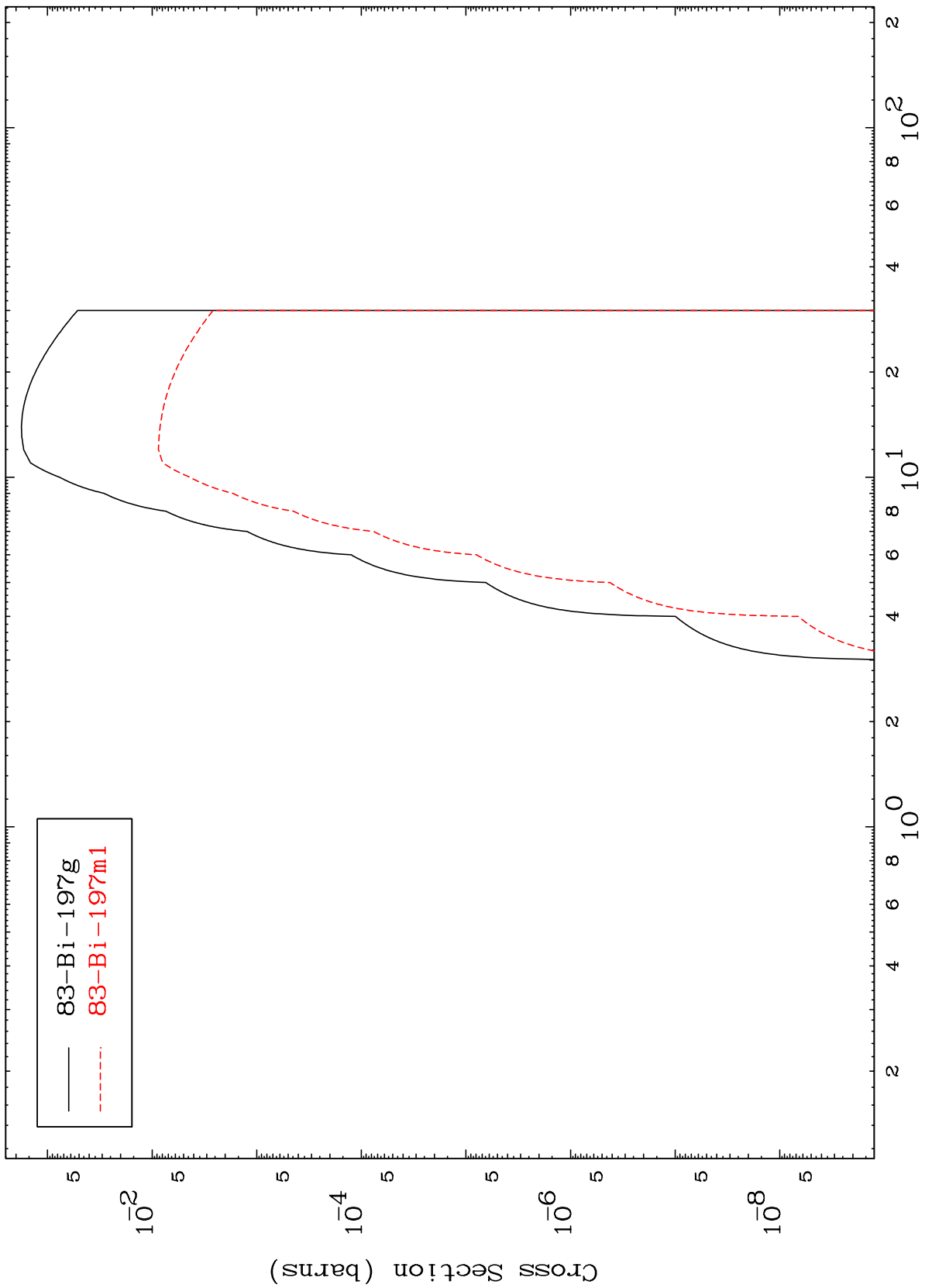
Radionuclide Production Cross Section



MAT 8287

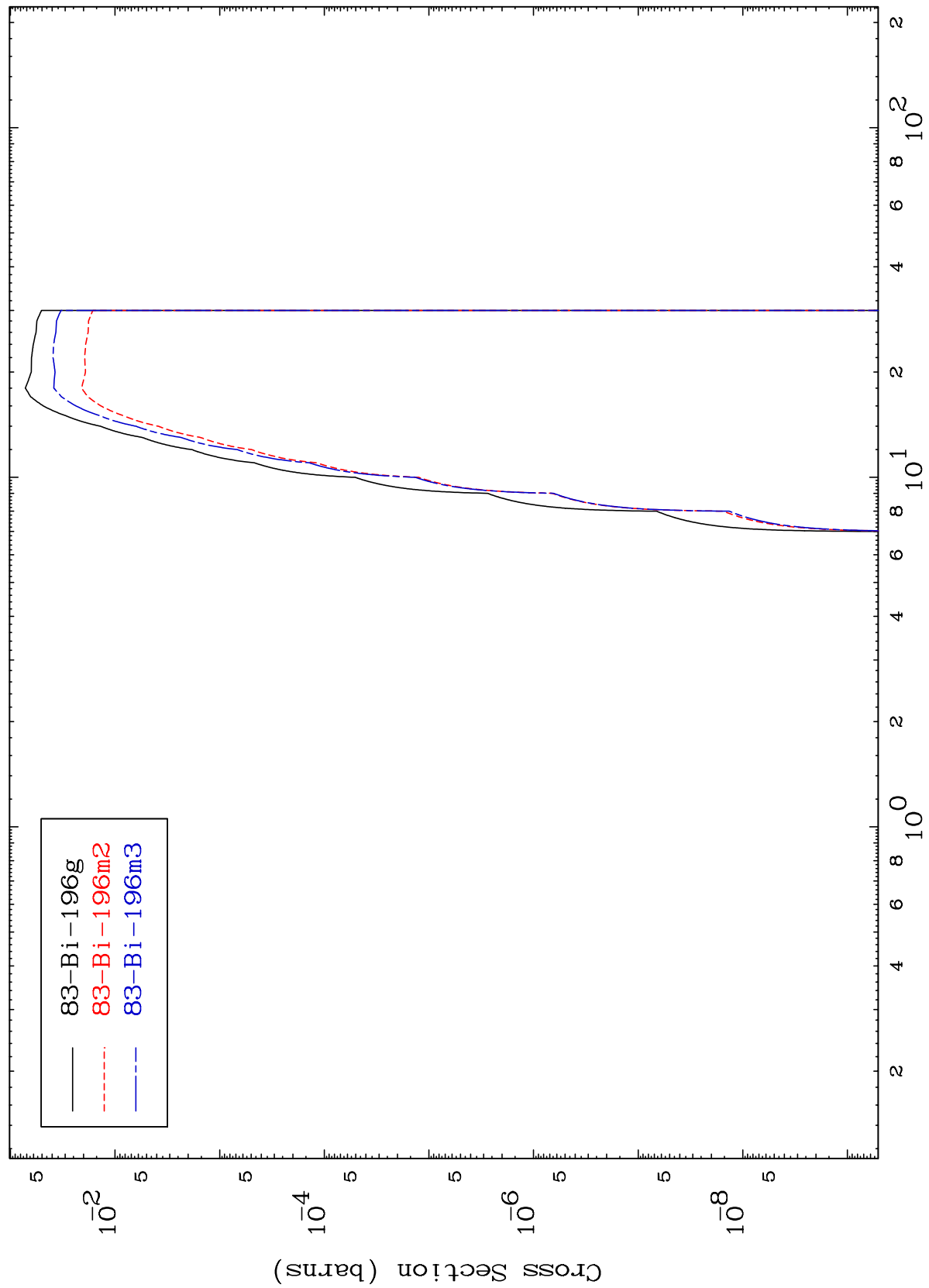
<sup>83</sup>Bi-196m

(n,p)  
Radionuclide Production Cross Section



— 83-Bi-197g  
- - - 83-Bi-197m1

Radionuclide Production Cross Section

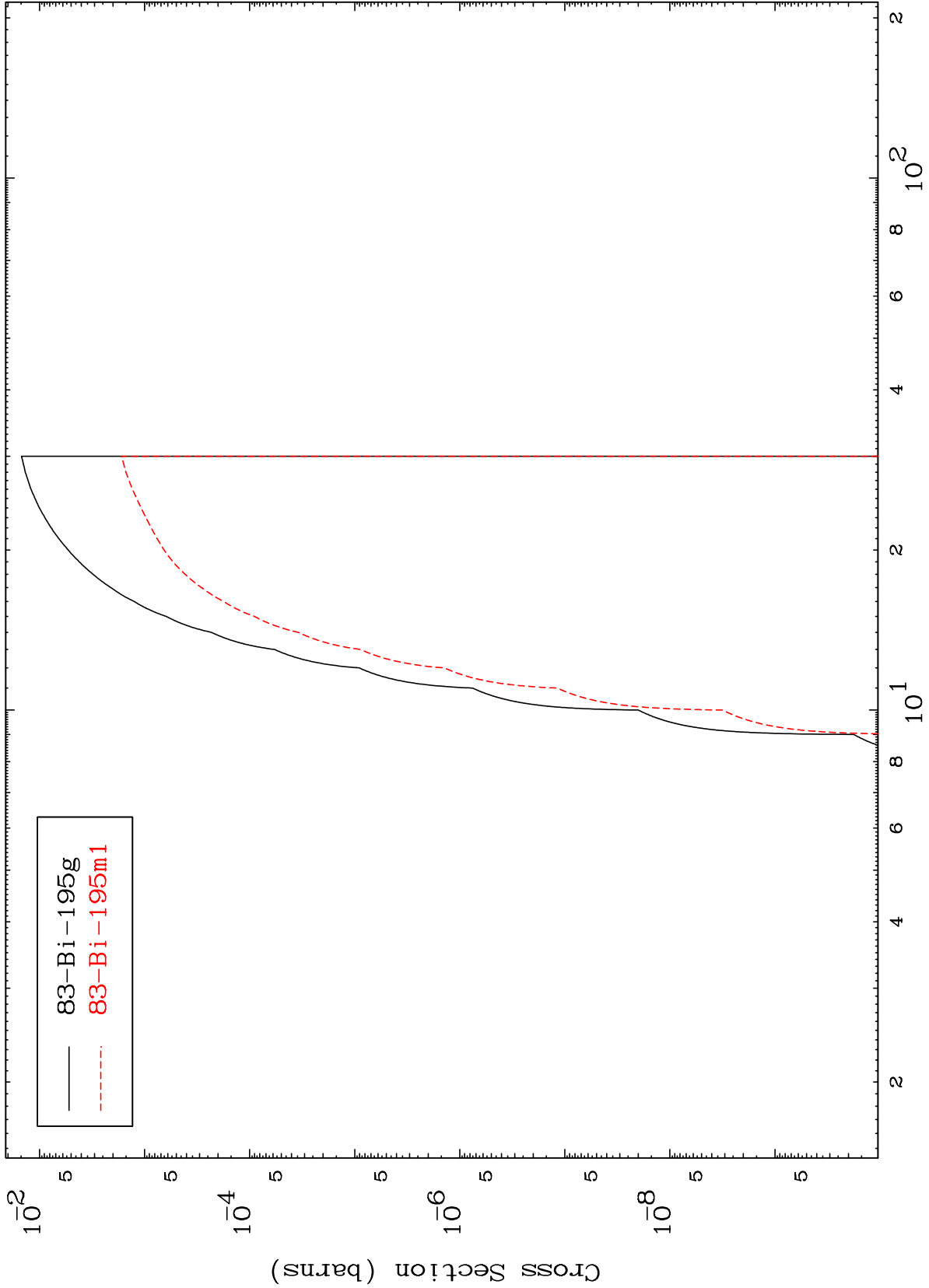


83-Bi-196g  
83-Bi-196m2  
83-Bi-196m3

MAT 8287

83-Bi-196m

(n, t)  
Radionuclide Production Cross Section

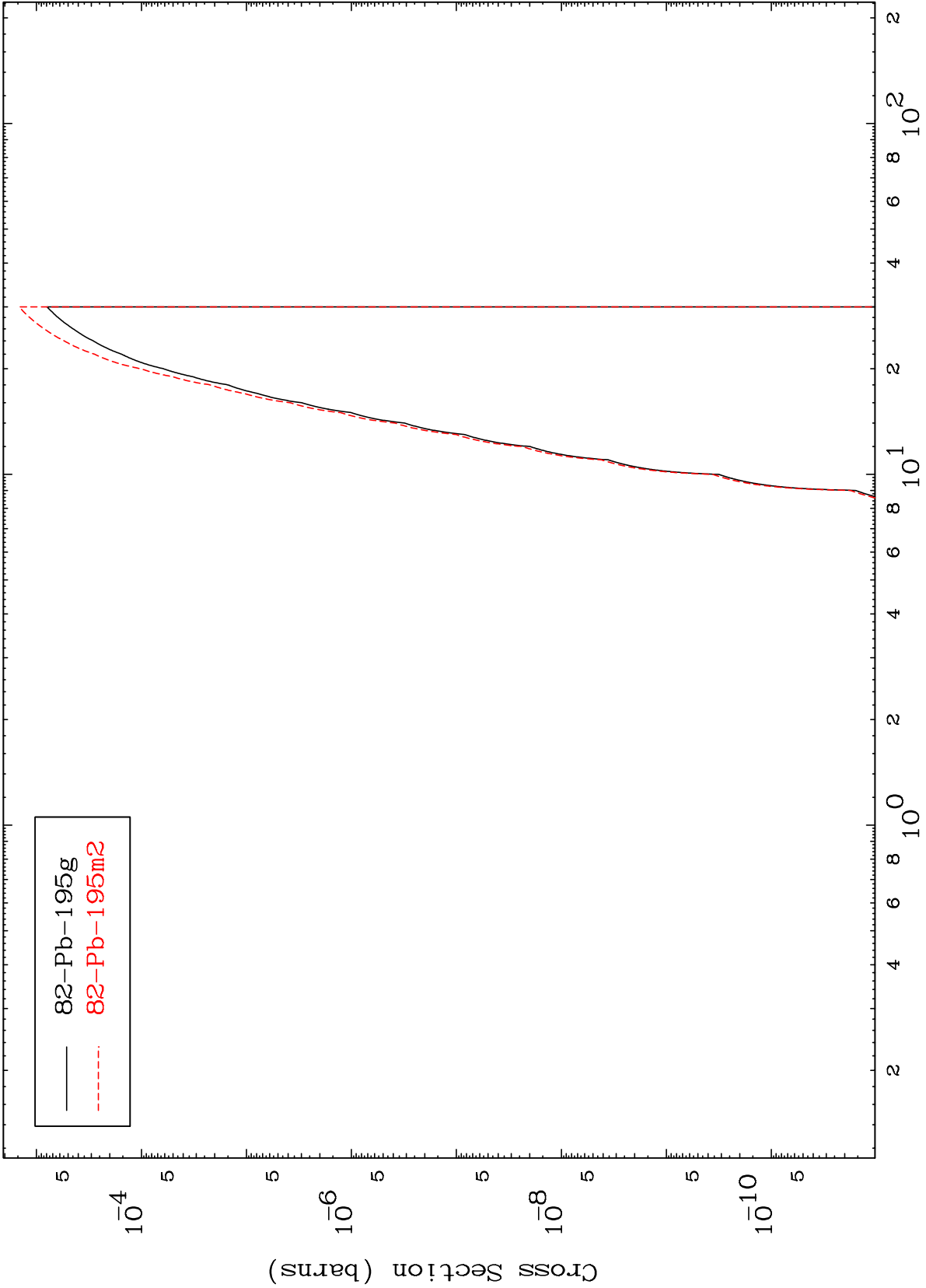


MAT 8287

(n,He-3)

83-Bi-196m

Radionuclide Production Cross Section



30

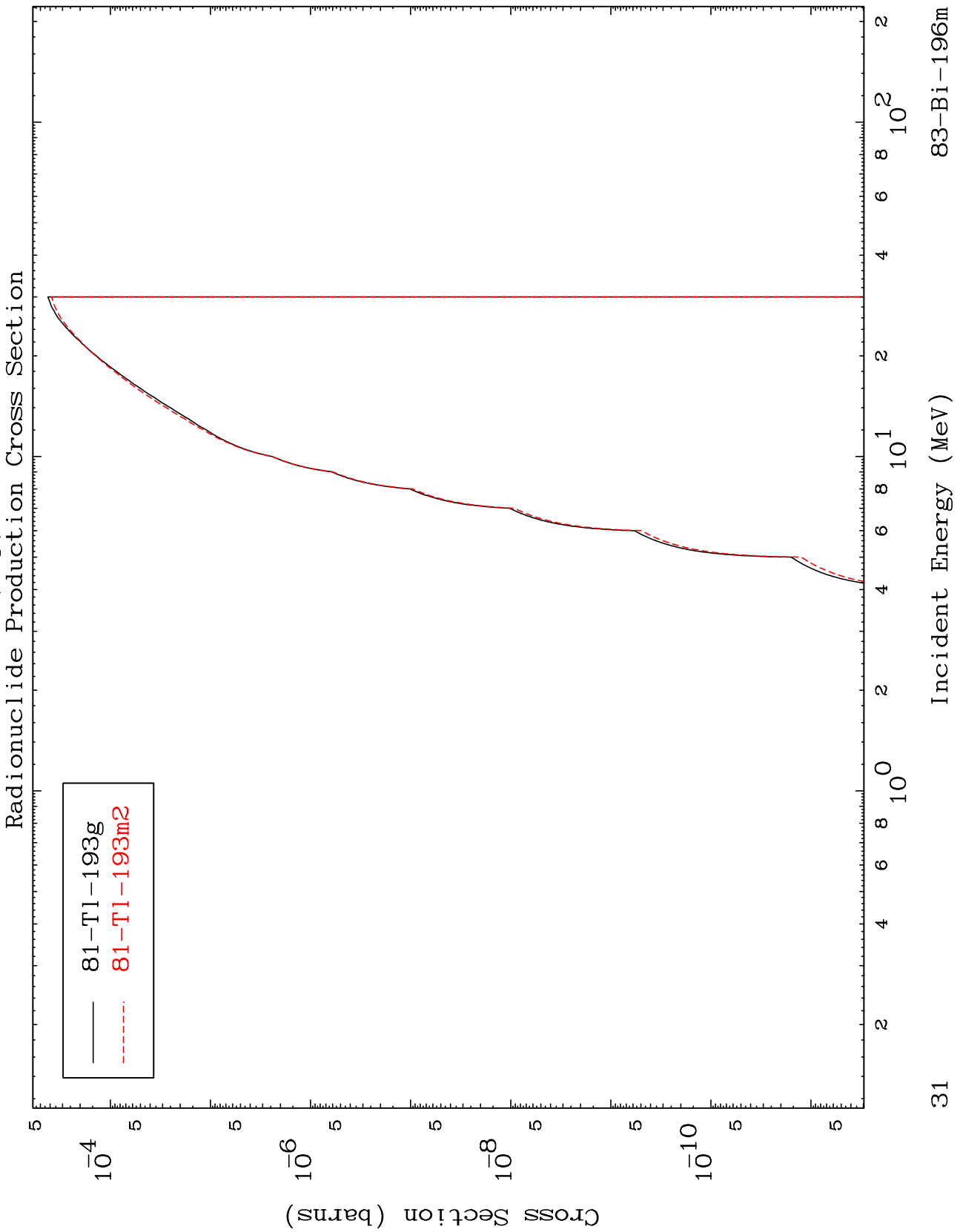
Incident Energy (MeV)

83-Bi-196m

MAT 8287

(n,p)  $\alpha$

83-Bi-196m



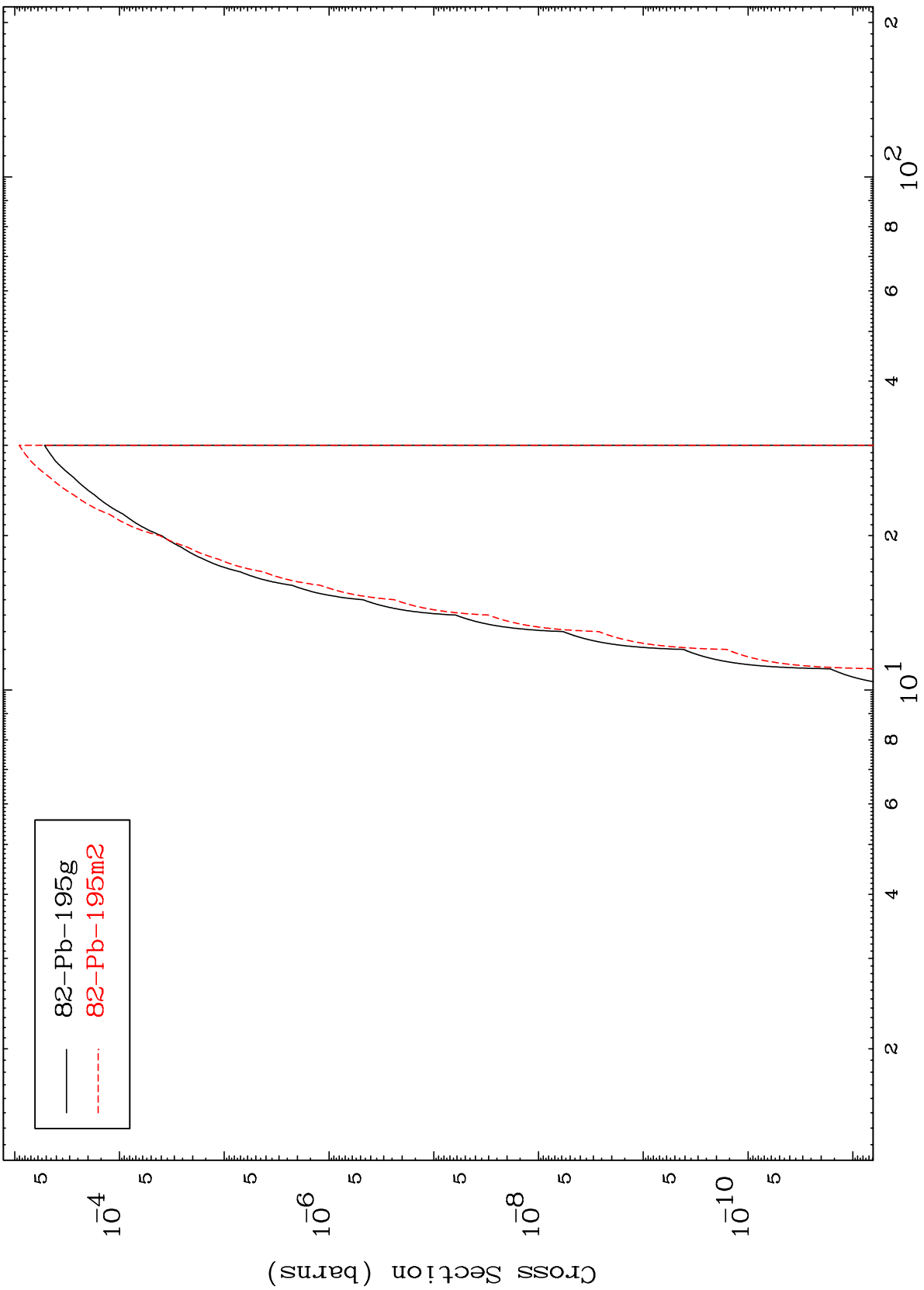


MAT 8287

(n,p) d

83-Bi-196m

Radionuclide Production Cross Section



32

Incident Energy (MeV)

83-Bi-196m

MAT 8287

(n,d)  $\alpha$

$^{83}\text{Bi}-196\text{m}$

