

Program EVALPLOT  
(Version 2021-1)

by

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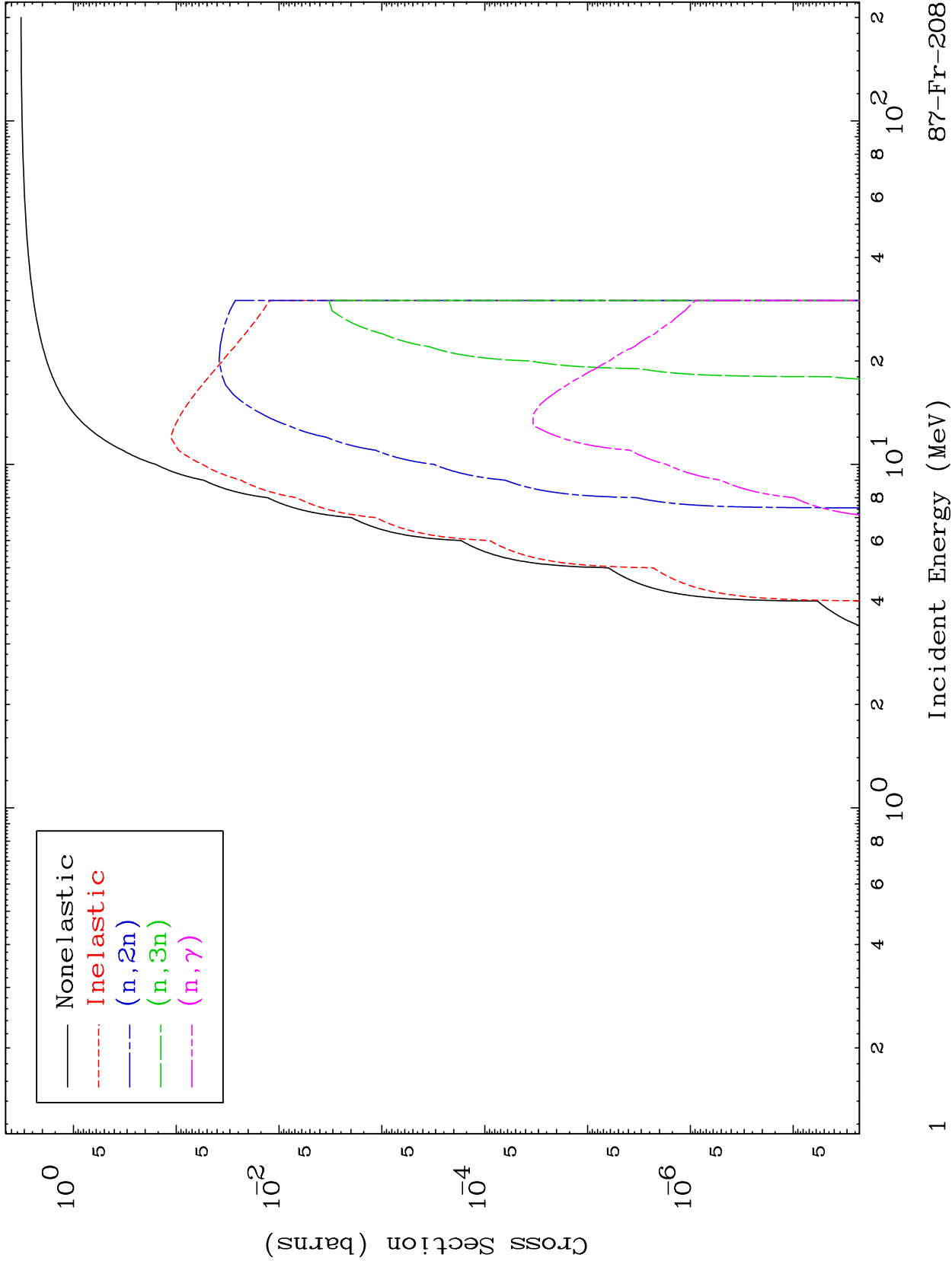
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Press Mouse Button to Start

MAT 8713

Deuteron Major  
0 Kelvin Cross Sections

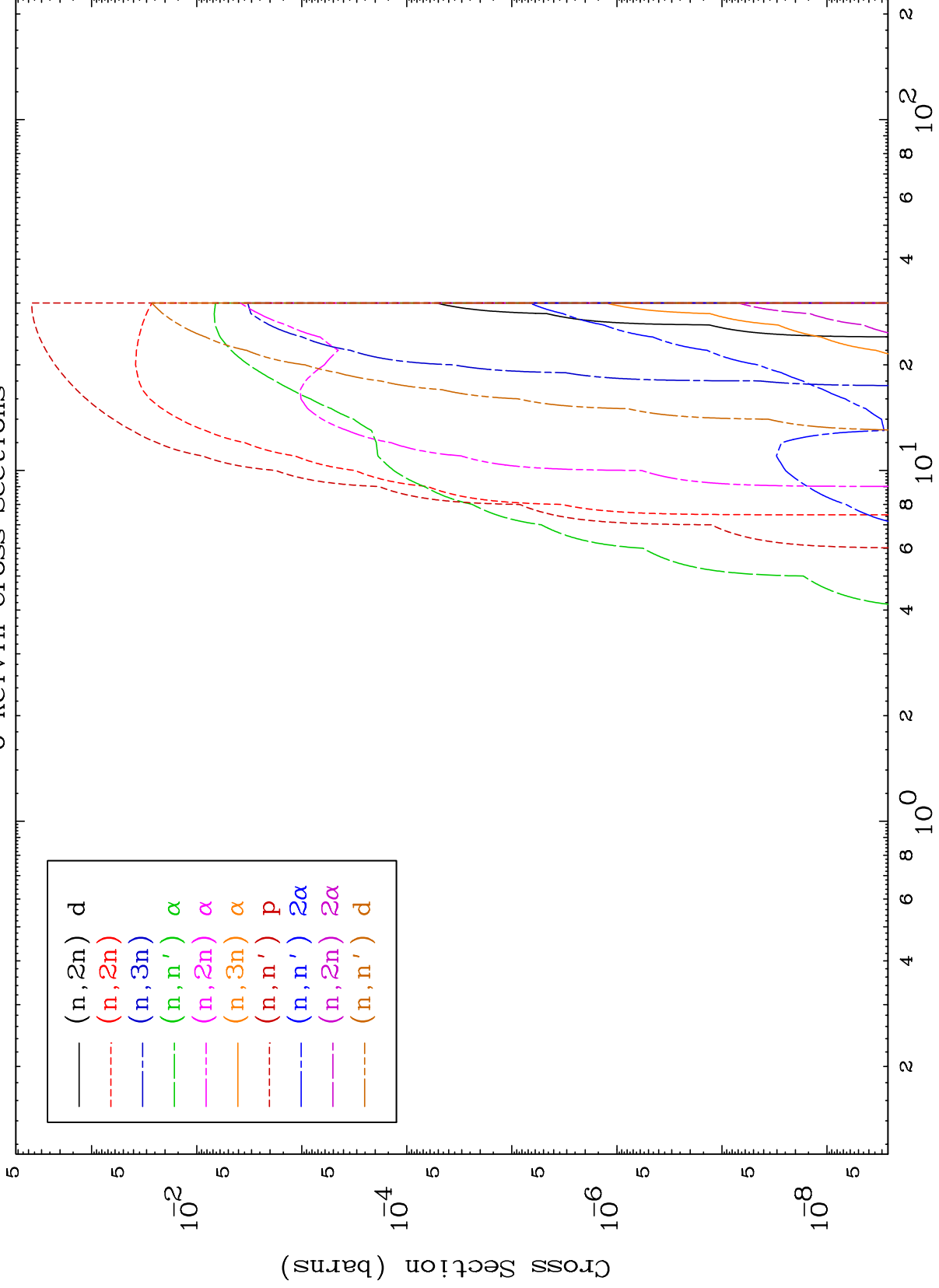
87-Fr-208

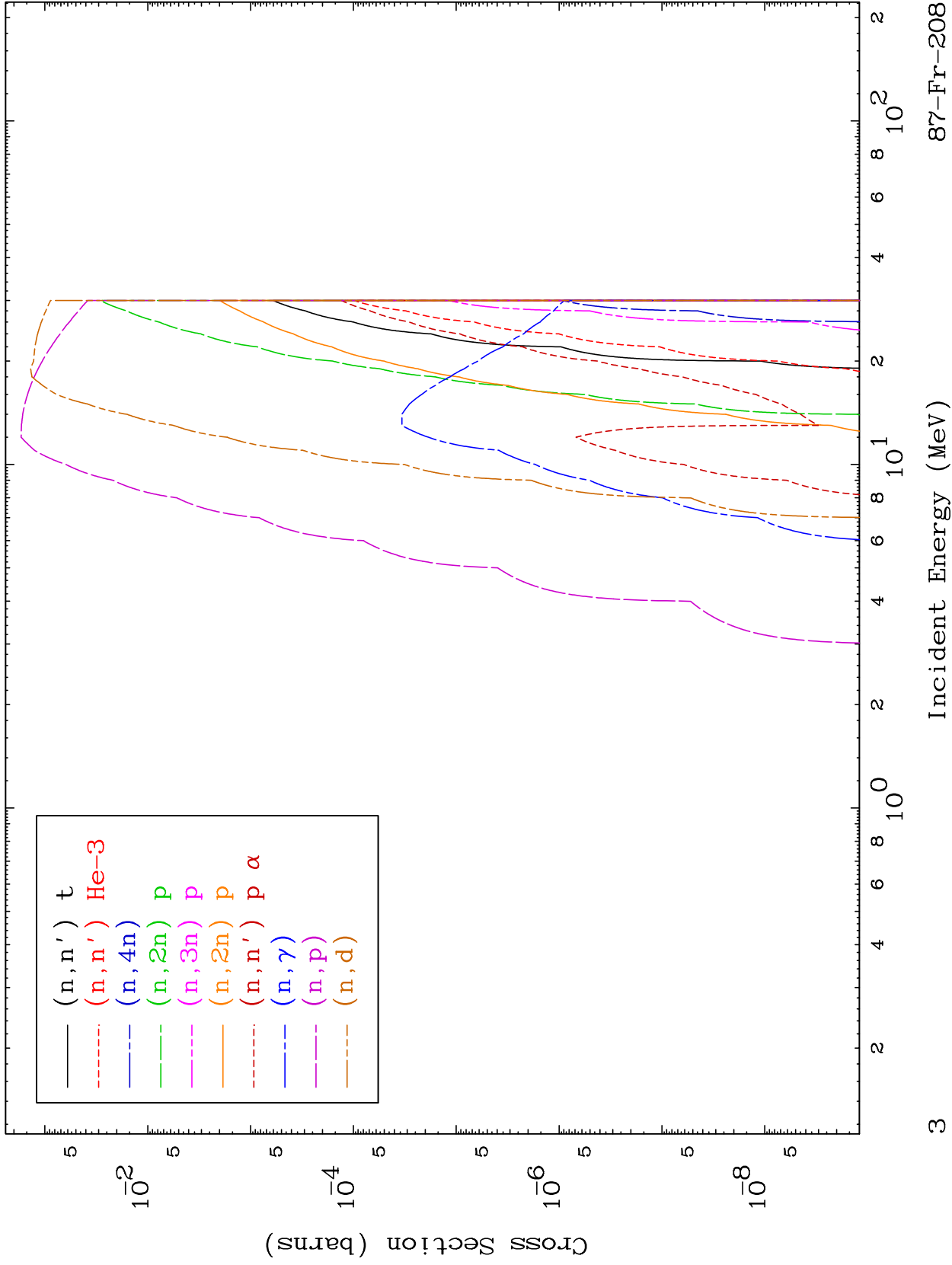


MAT 8713

Deuteron Neutron Absorption  
0 Kelvin Cross Sections

87-Fr-208

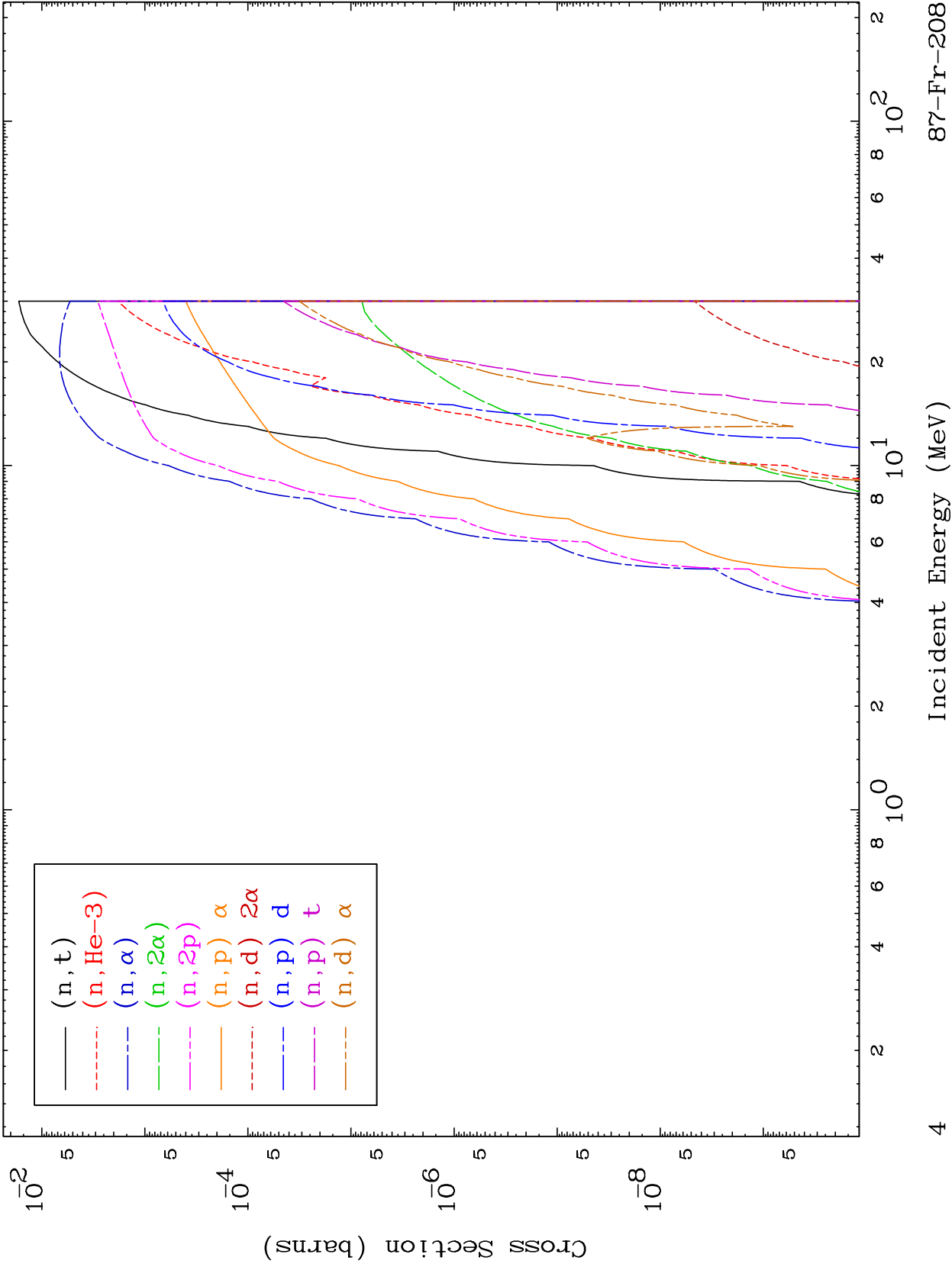




MAT 8713

Deuteron Neutron Absorption  
0 Kelvin Cross Sections

87-Fr-208



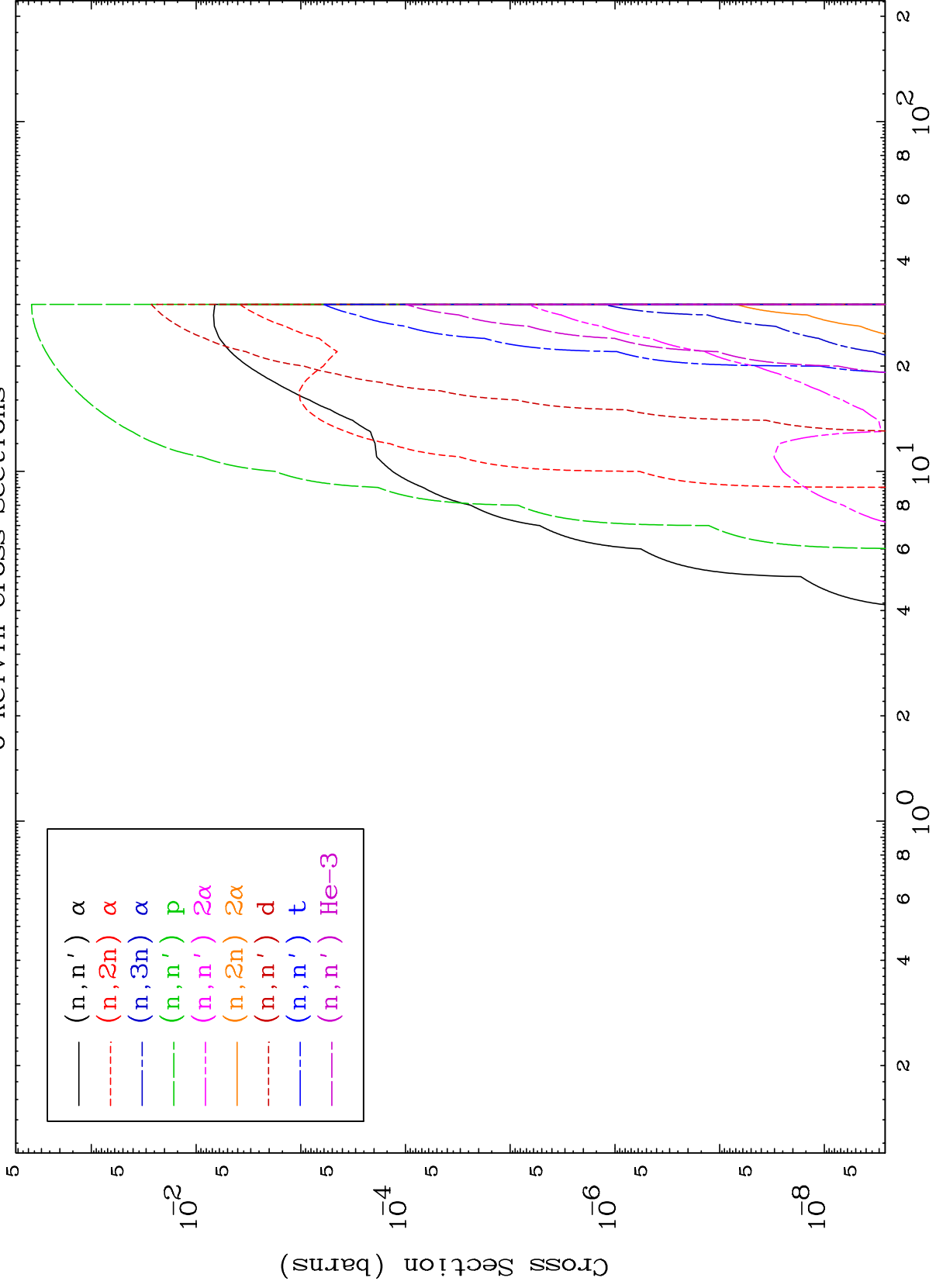
87-Fr-208

Incident Energy (MeV)

MAT 8713

Deuteron Charged Particle  
0 Kelvin Cross Sections

87-Fr-208



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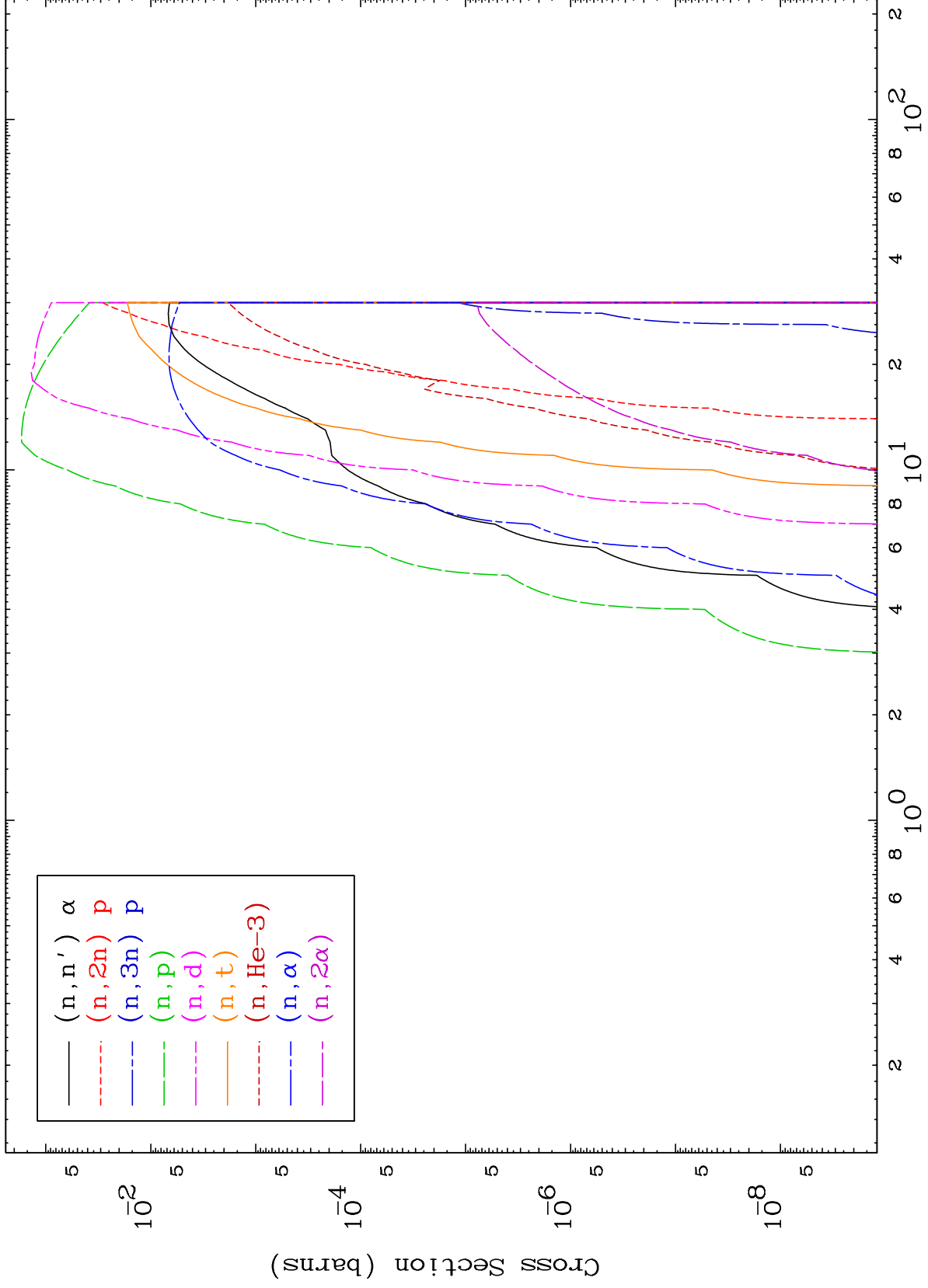
Incident Energy (MeV)

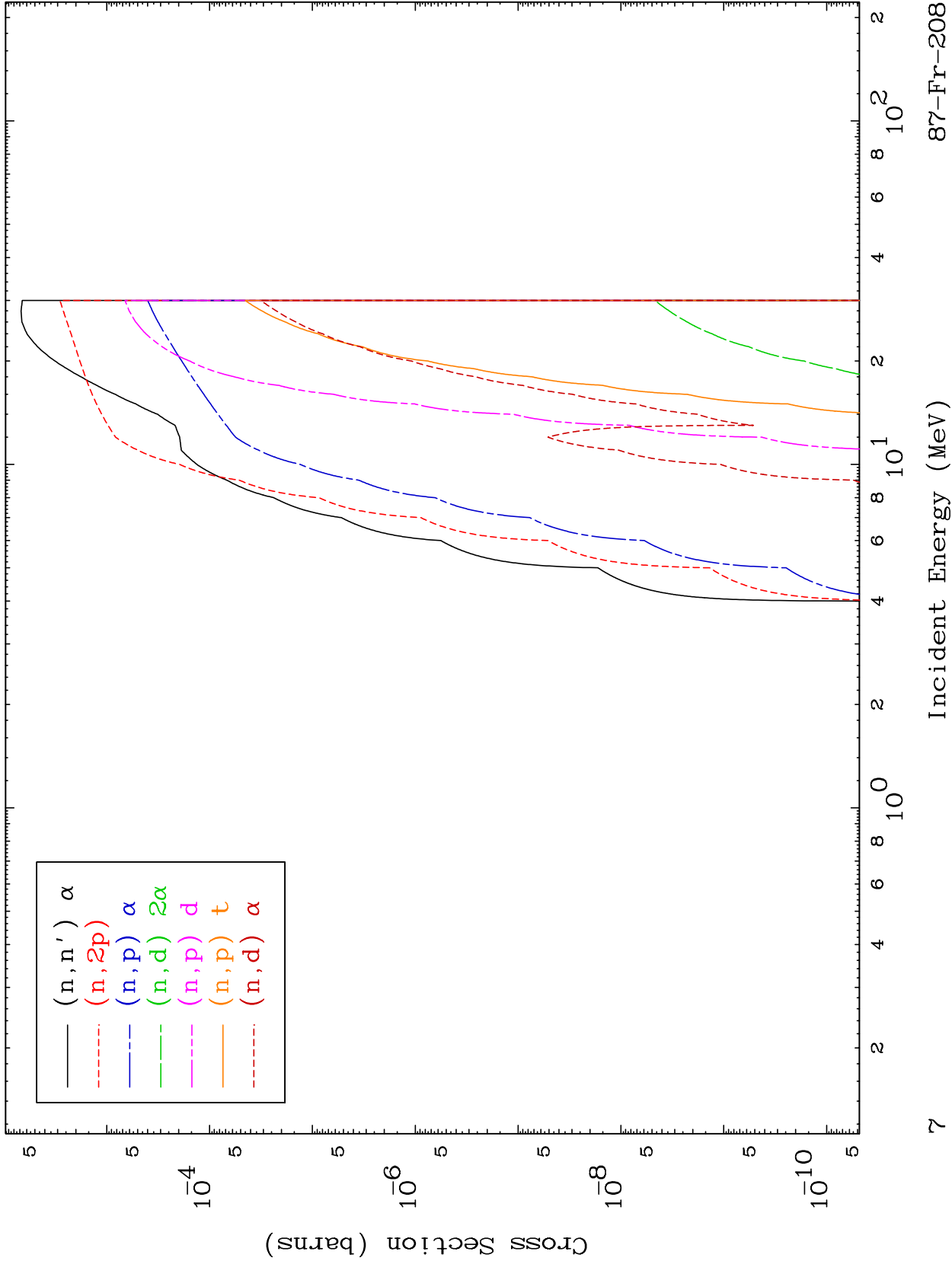
87-Fr-208

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Deuteron Charged Particle  
0 Kelvin Cross Sections

87-Fr-208





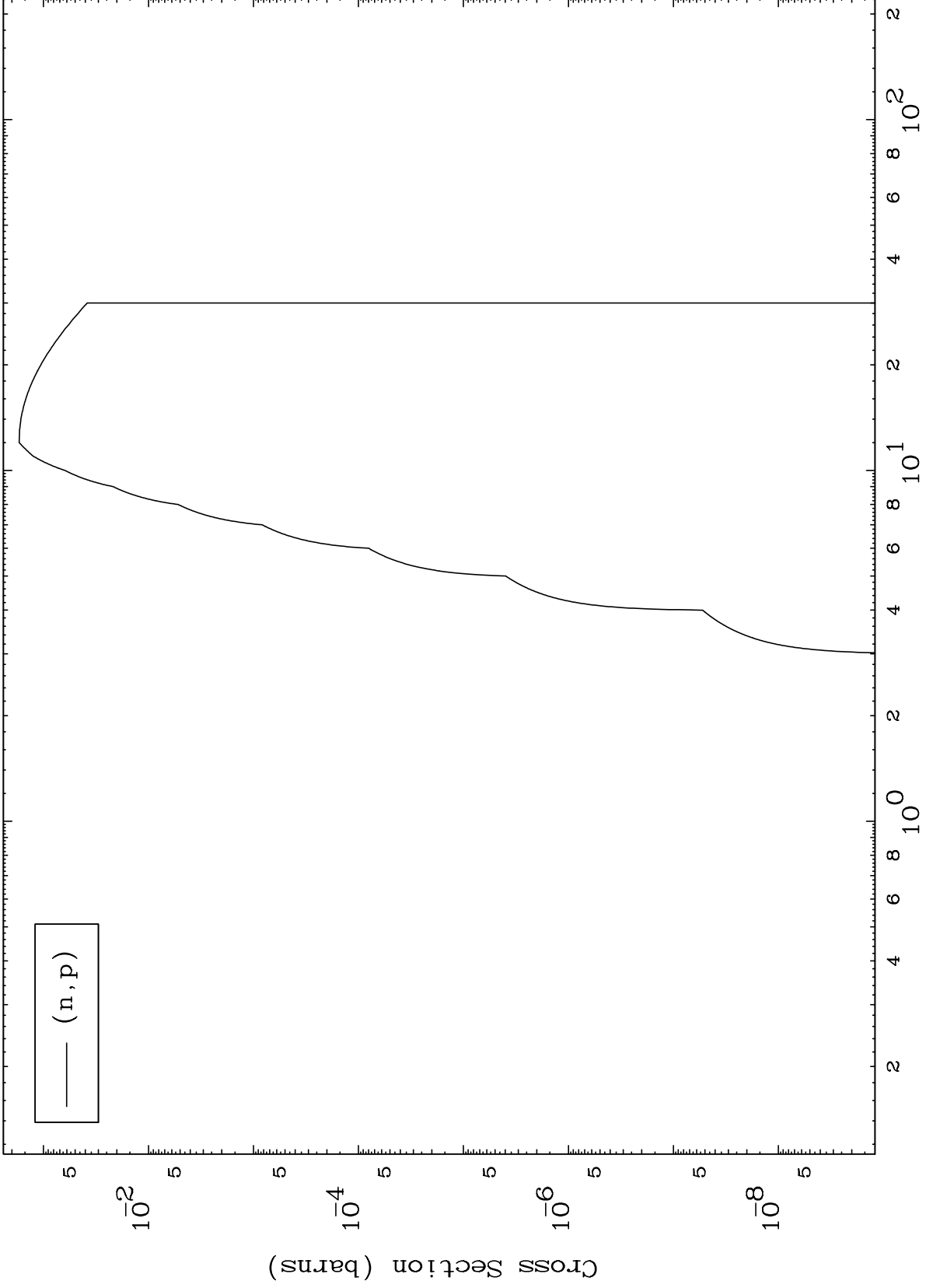


MAT 8713

(d,p) Levels

87-Fr-208

0 Kelvin Cross Sections

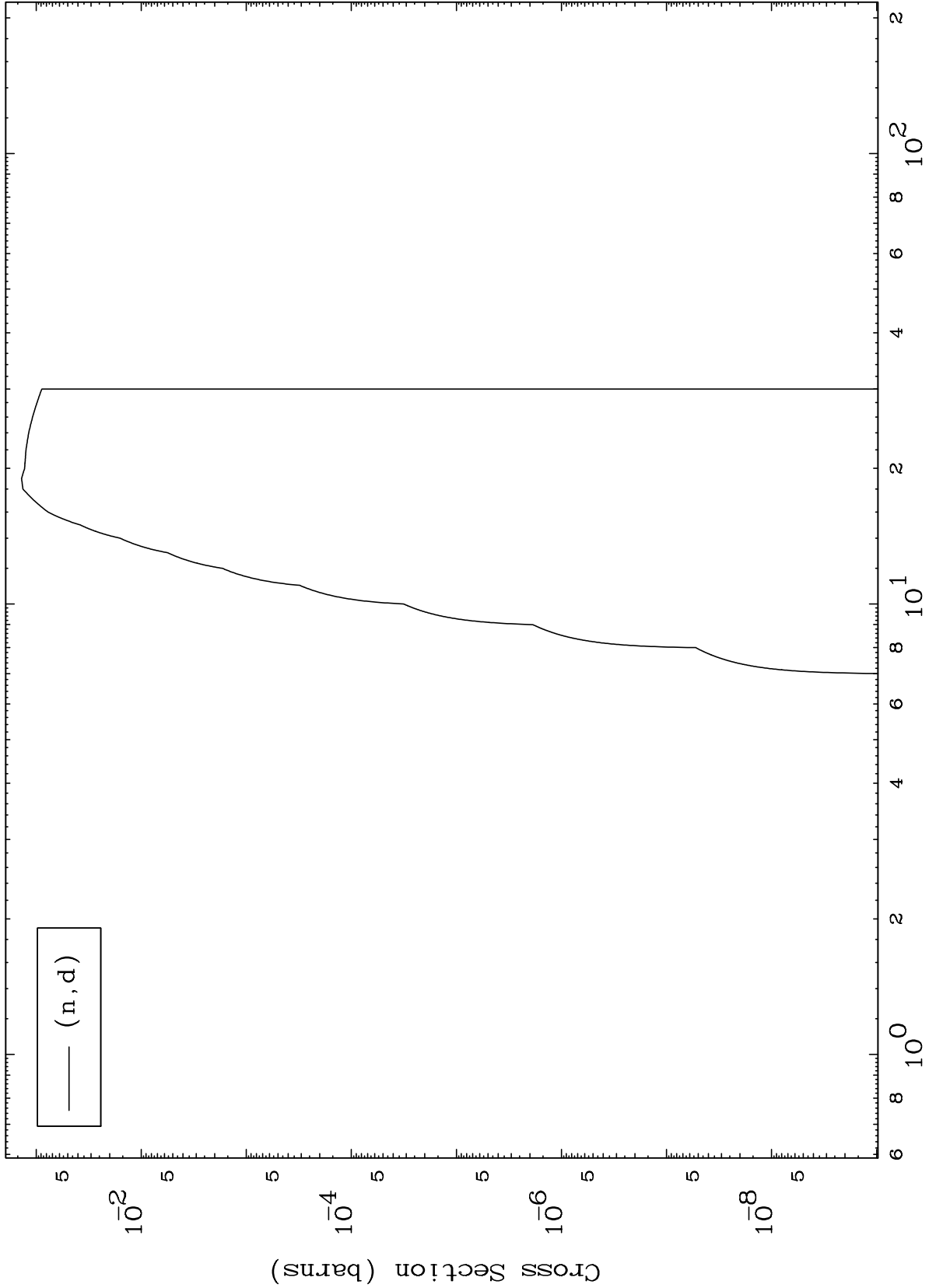


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(d,d) Levels

87-Fr-208

0 Kelvin Cross Sections



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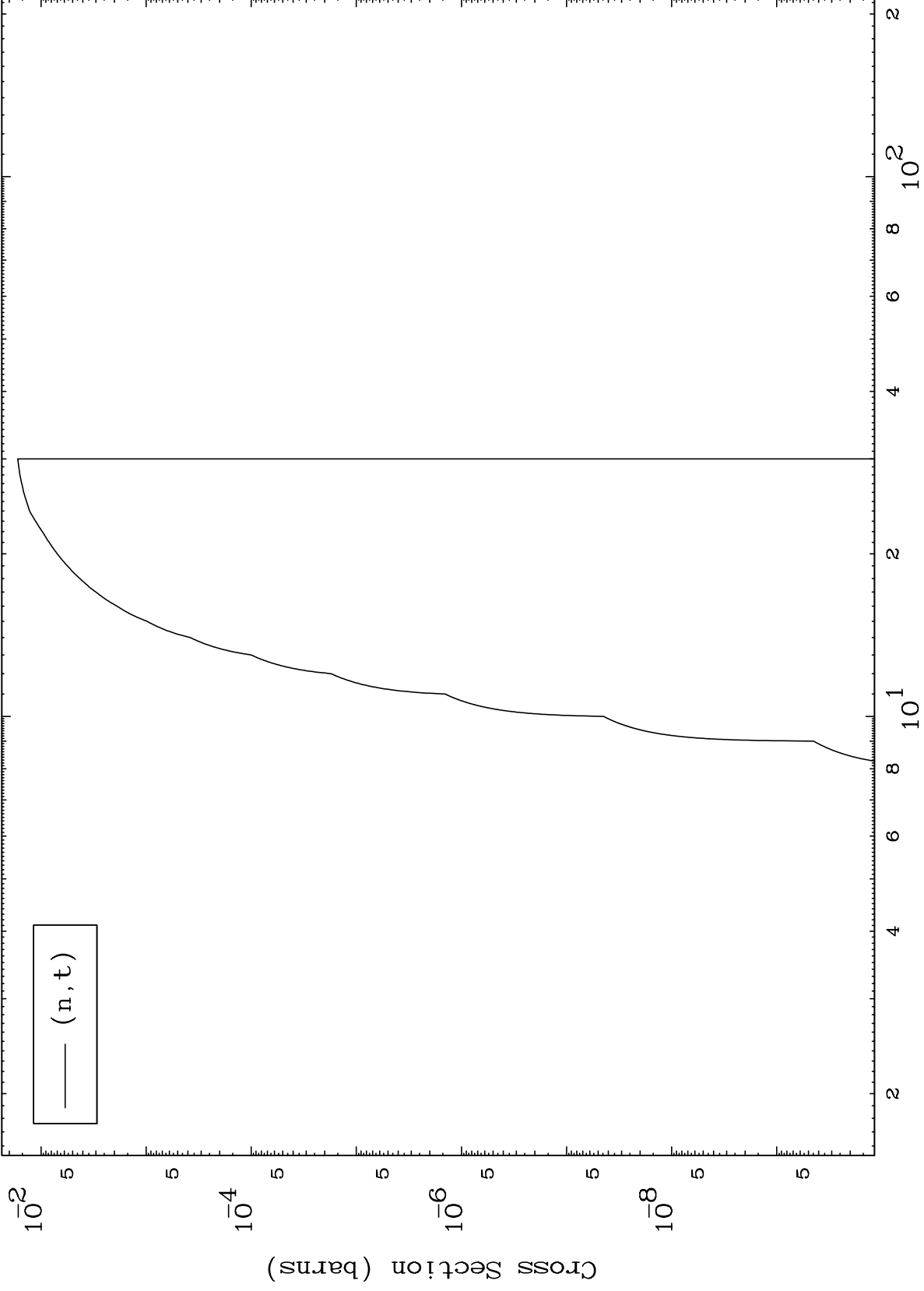
Incident Energy (MeV)

87-Fr-208

MAT 8713

(d, t) Levels  
0 Kelvin Cross Sections

87-Fr-208



10

Incident Energy (MeV)

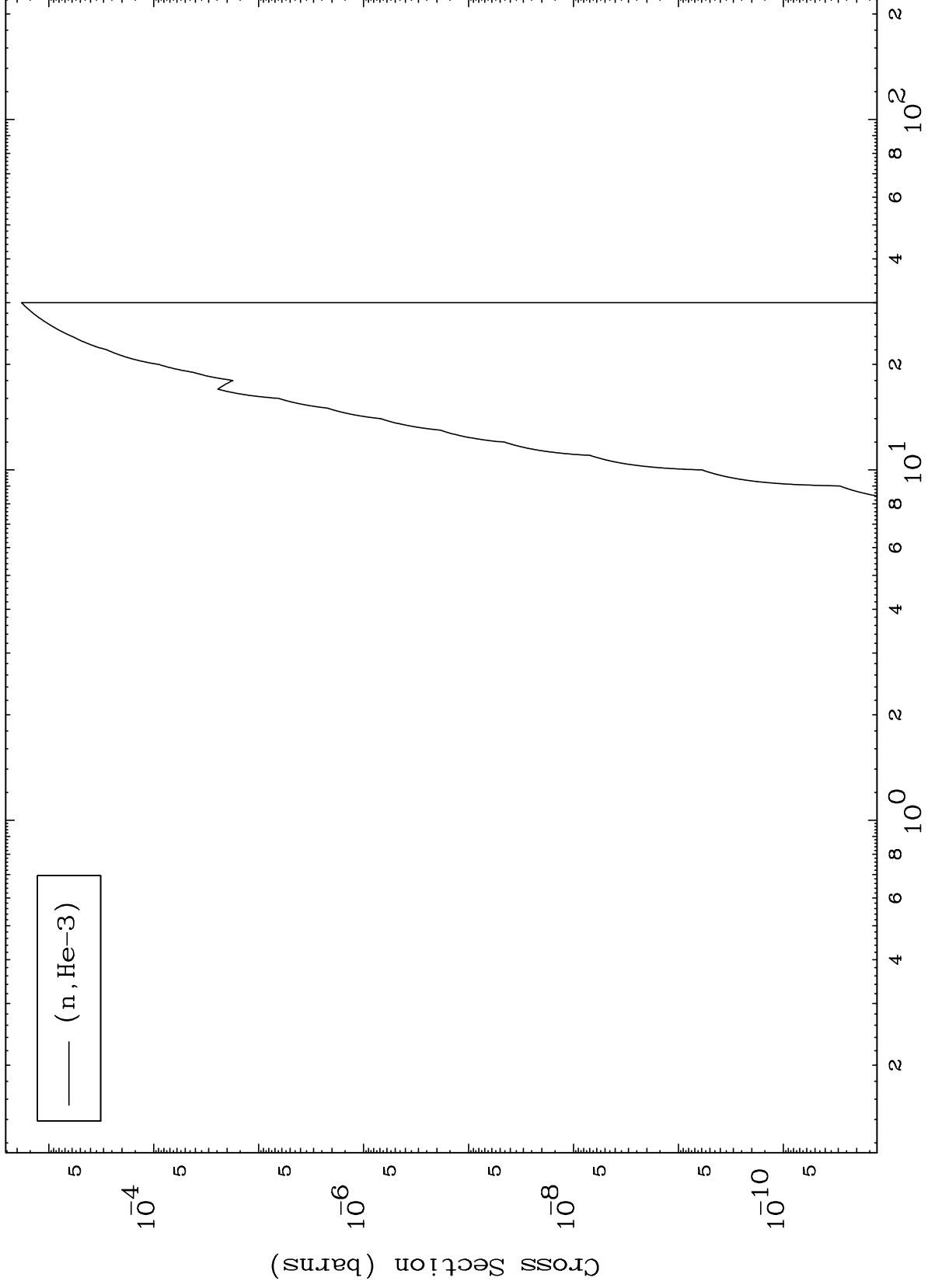
87-Fr-208

MAT 8713

(d,He3) Levels

87-Fr-208

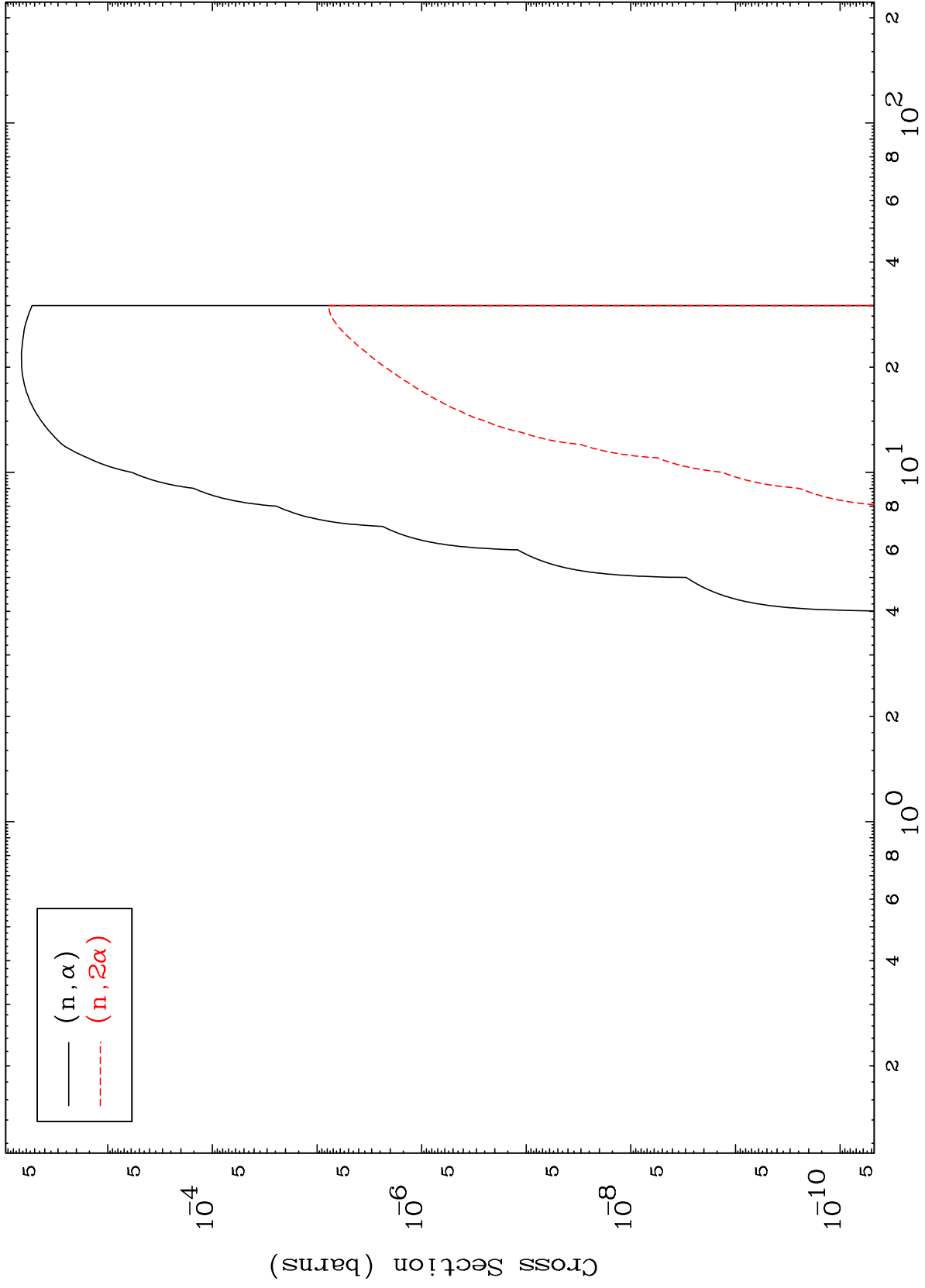
0 Kelvin Cross Sections



MAT 8713

(d,  $\alpha$ ) Levels  
0 Kelvin Cross Sections

87-Fr-208

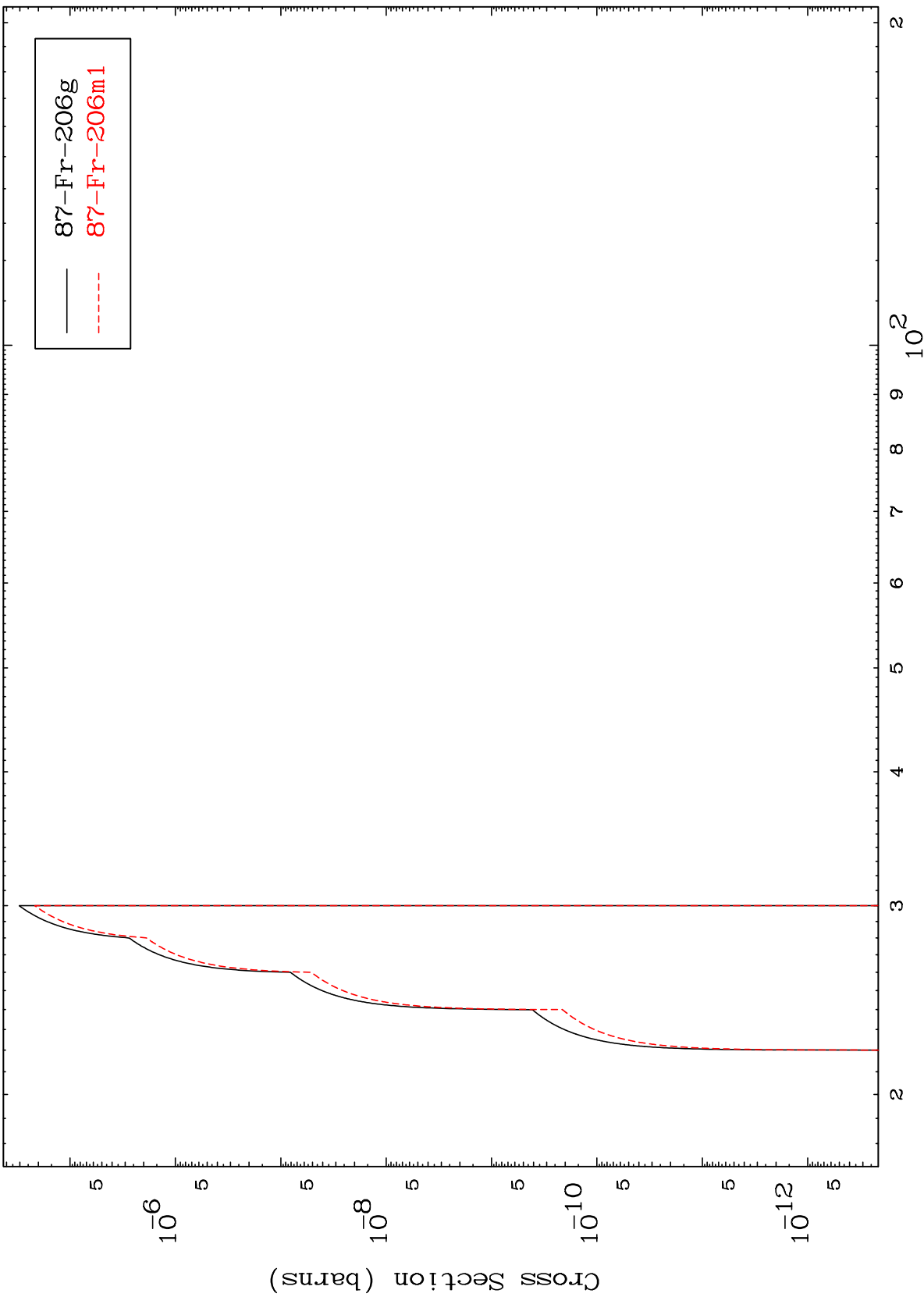


12

Incident Energy (MeV)

87-Fr-208

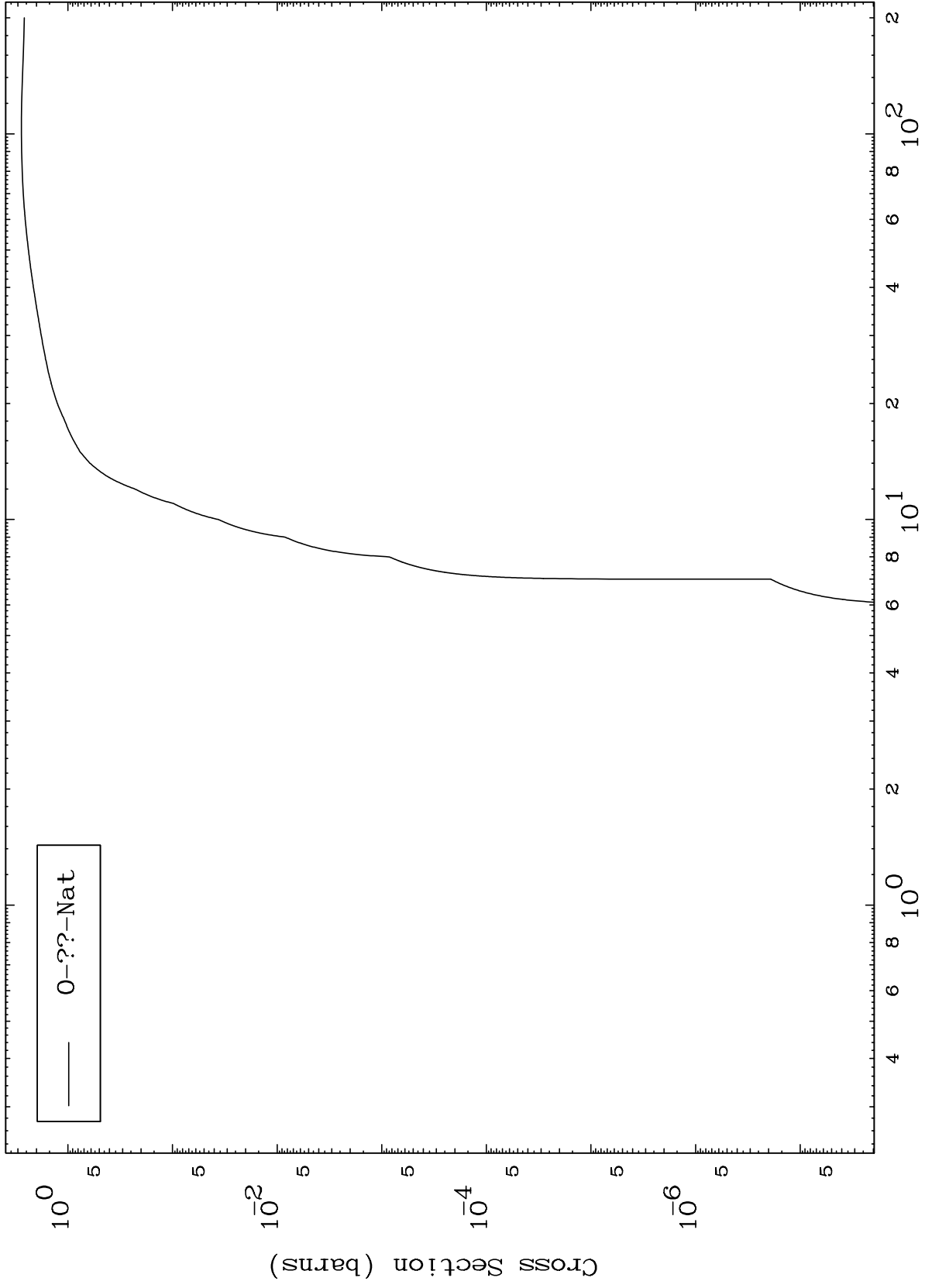
Radionuclide Production Cross Section



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87-Fr-208

Fission  
Radionuclide Production Cross Section



14

Incident Energy (MeV)

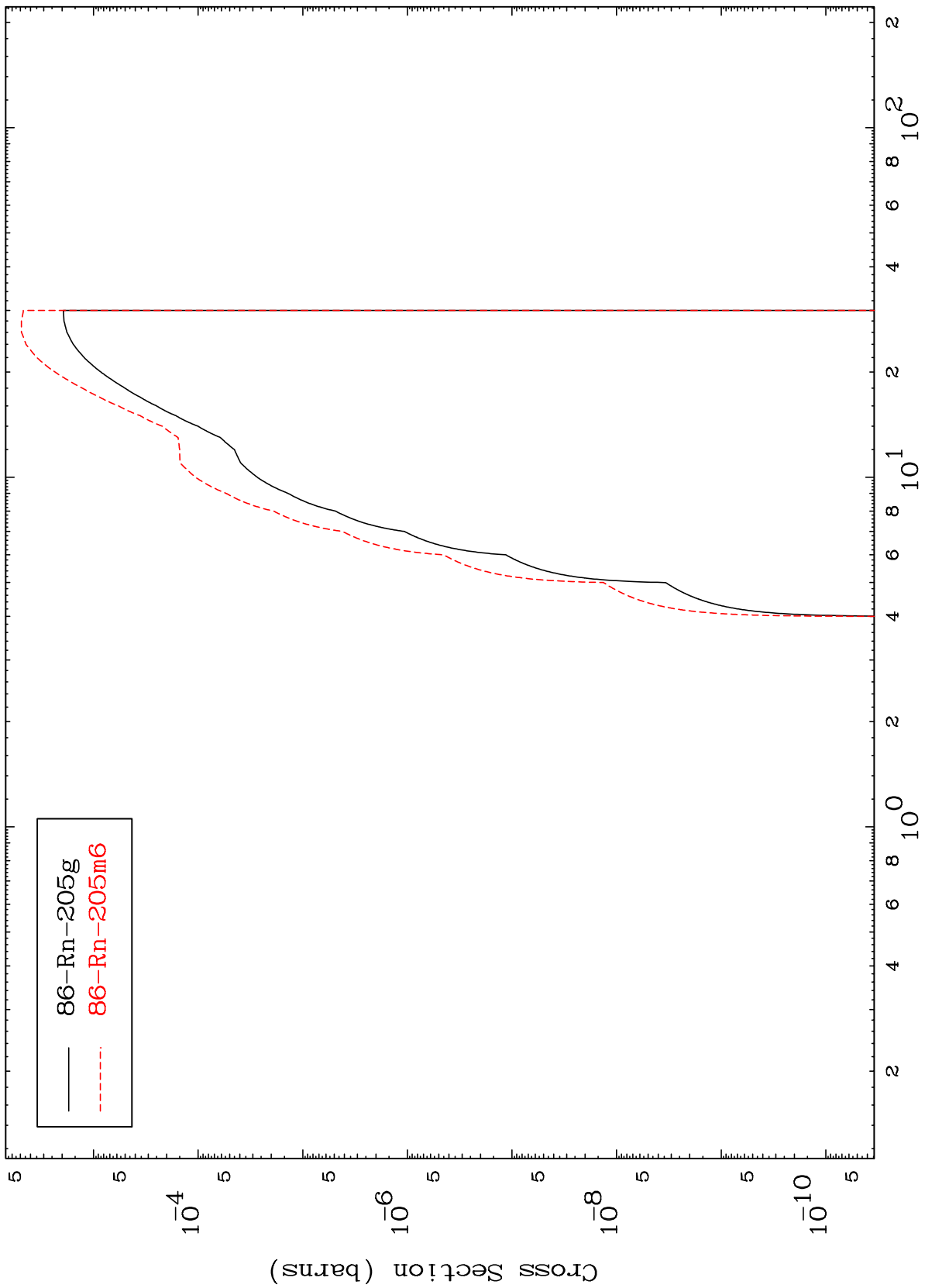
87-Fr-208

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$(n, n') \alpha$

87-Fr-208

Radionuclide Production Cross Section



15

Incident Energy (MeV)

87-Fr-208

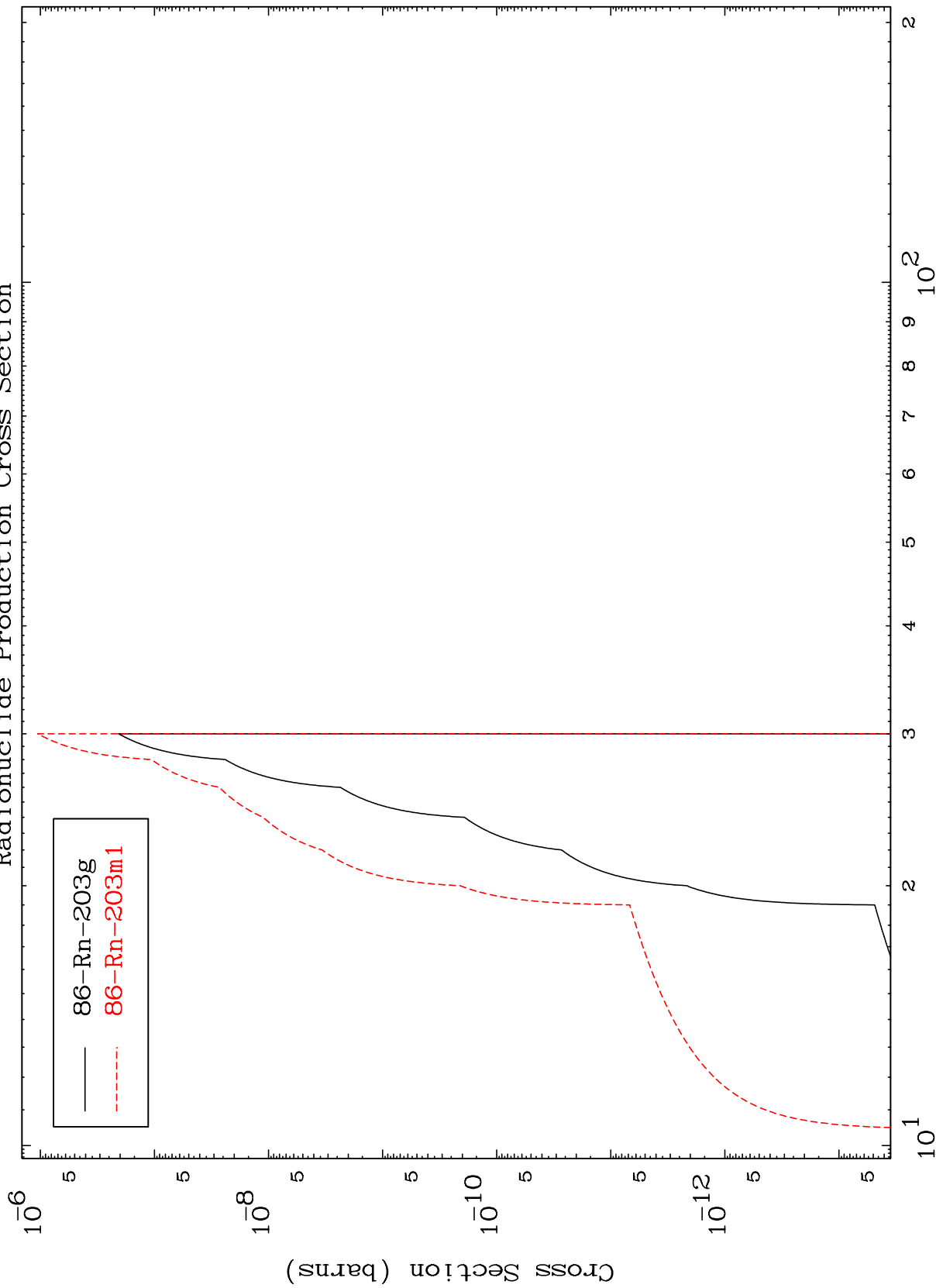


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(n,3n)  $\alpha$

87-Fr-208

Radionuclide Production Cross Section



Incident Energy (MeV)

87-Fr-208

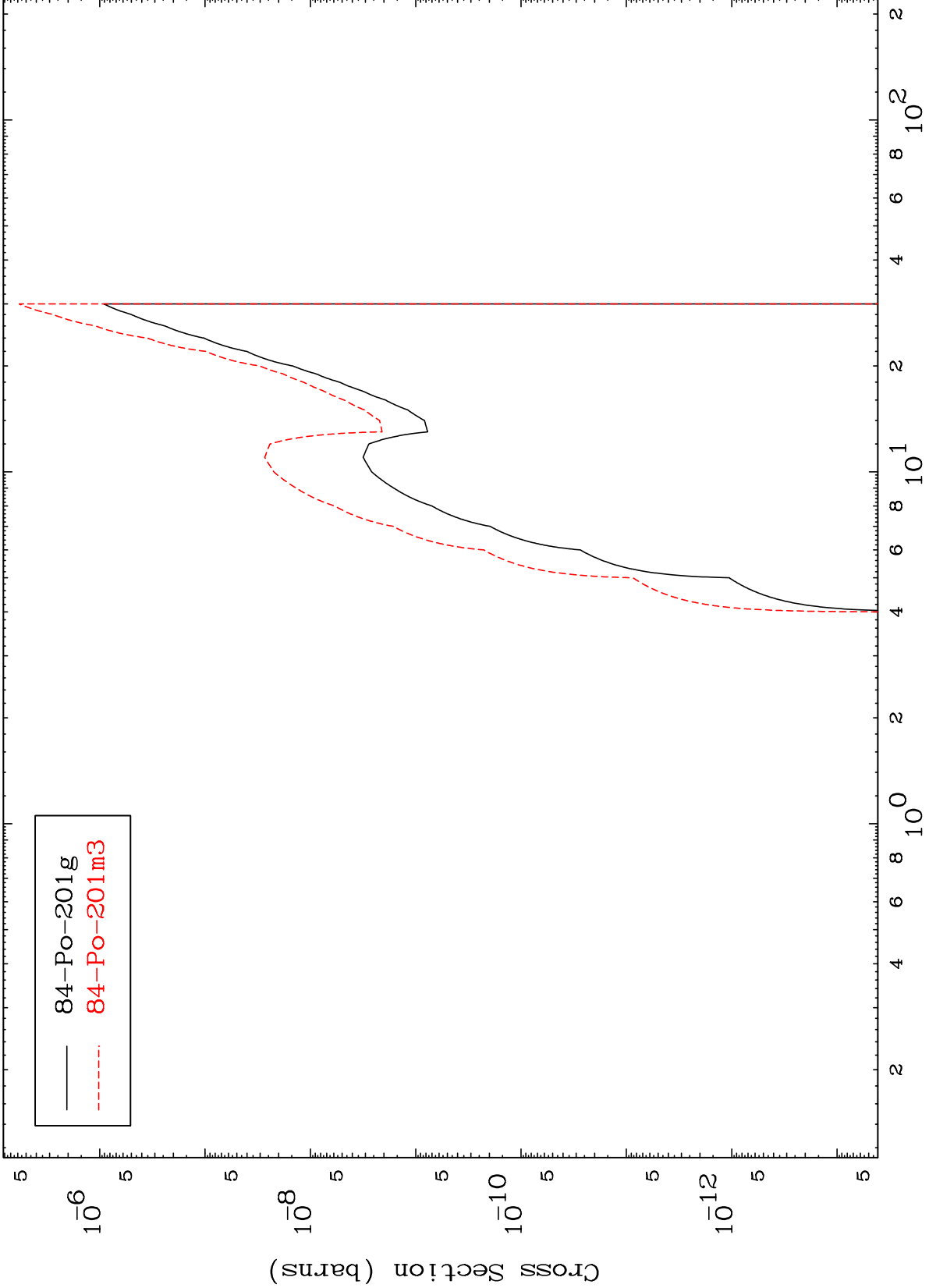
16

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(n,n') 2 $\alpha$

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Radionuclide Production Cross Section

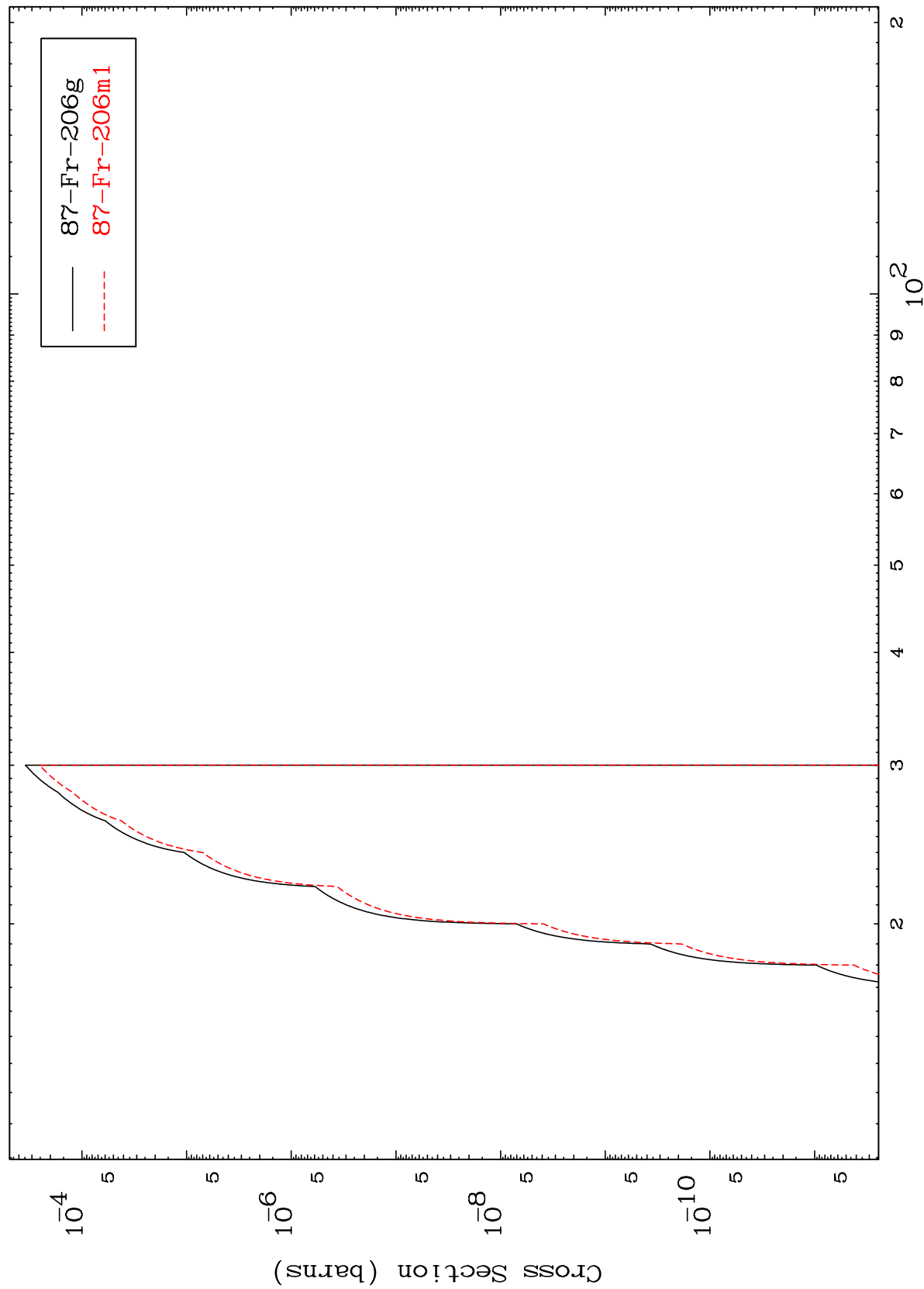


MAT 8713

(n,n') t

87-Fr-208

Radionuclide Production Cross Section



18

Incident Energy (MeV)

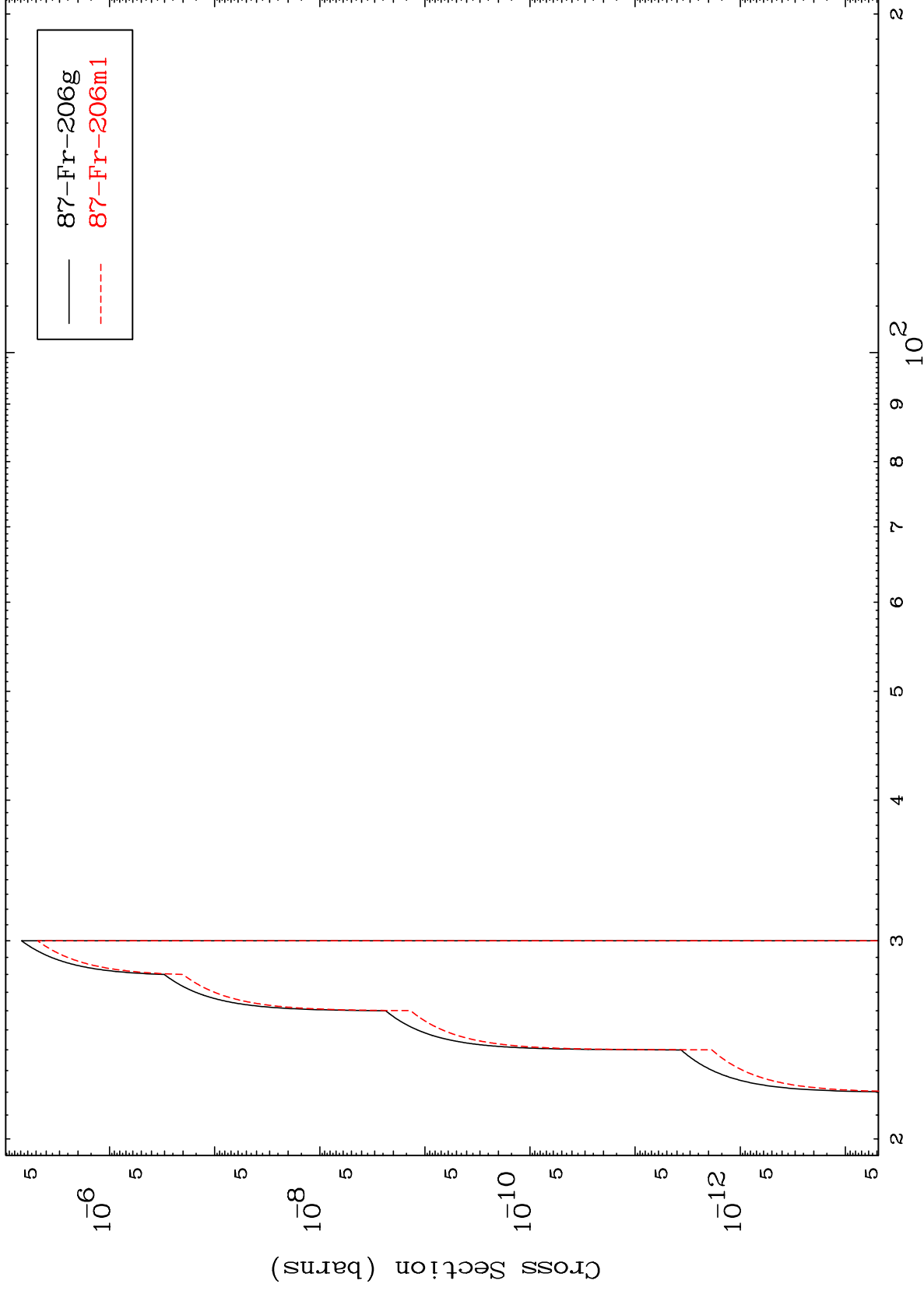
87-Fr-208

MAT 8713

(n,3n) p

87-Fr-208

Radionuclide Production Cross Section



19

Incident Energy (MeV)

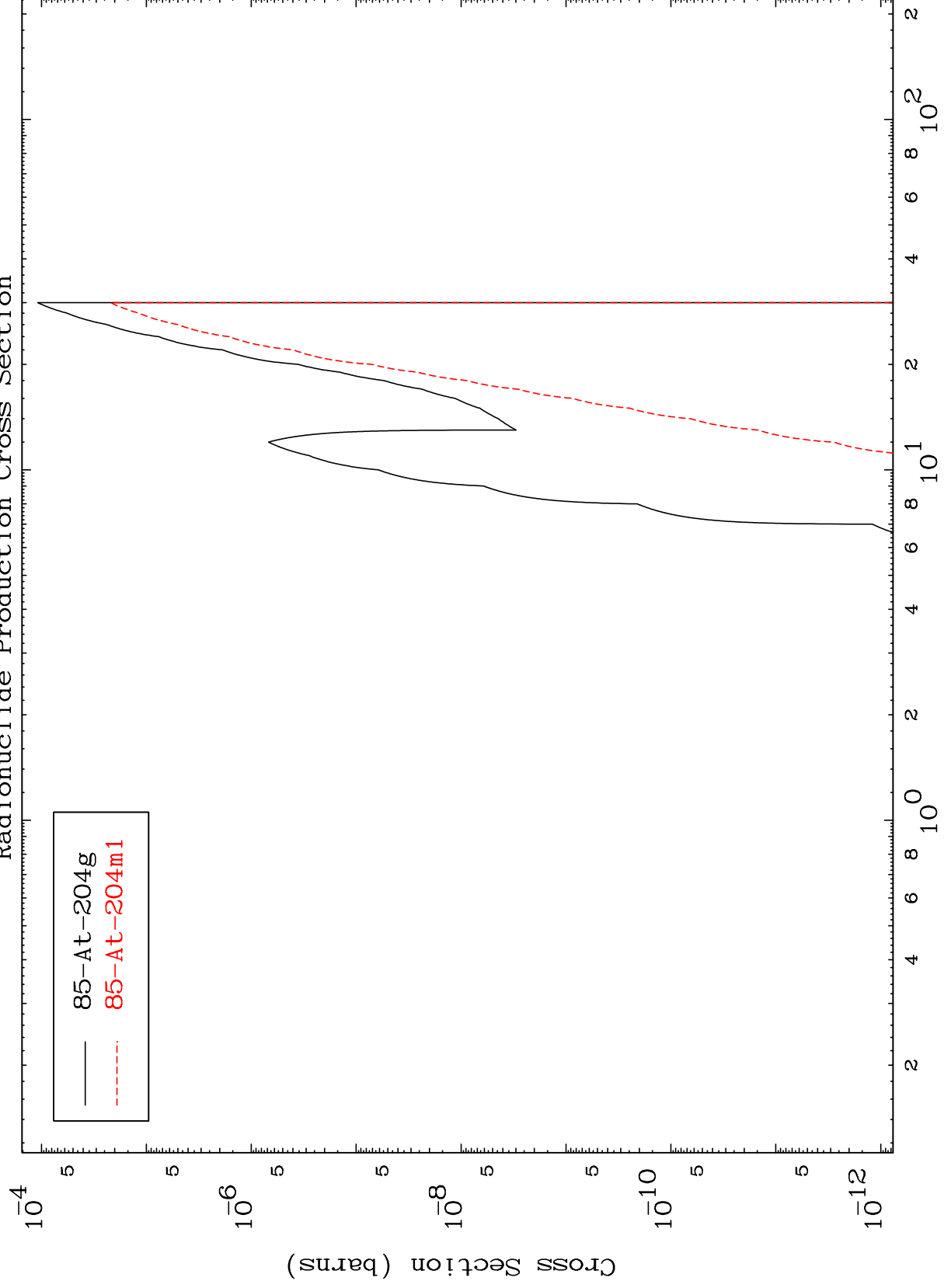
87-Fr-208

MAT 8713

(n,n') p  $\alpha$

87-Fr-208

Radionuclide Production Cross Section



85-At-204g  
85-At-204m1

20

Incident Energy (MeV)

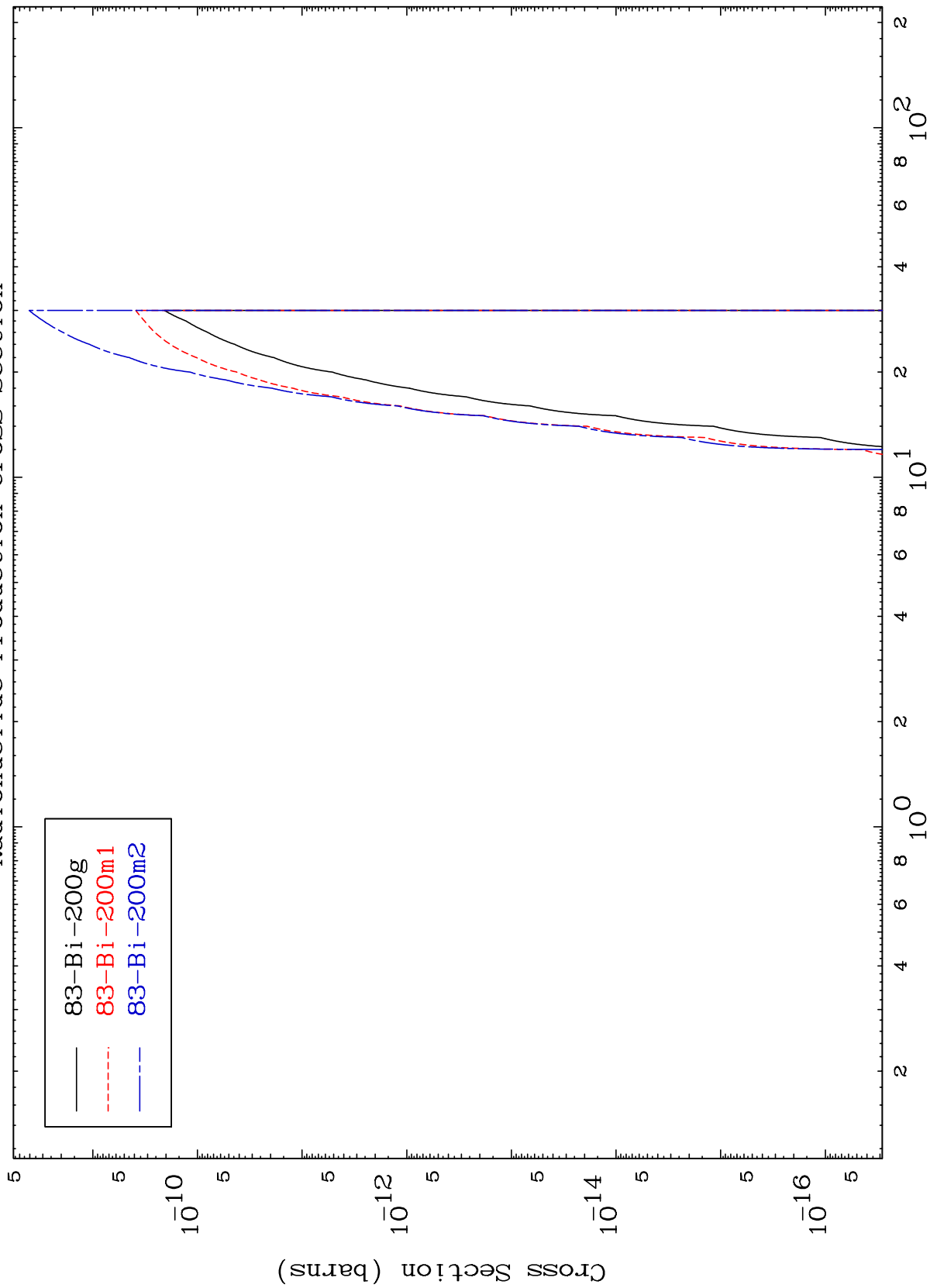
87-Fr-208

MAT 8713

$(n,d) 2\alpha$

87-Fr-208

Radionuclide Production Cross Section



21

Incident Energy (MeV)

87-Fr-208

MAT 8713

(n,d)  $\alpha$

87-Fr-208

Radionuclide Production Cross Section

