

Program EVALPLOT  
(Version 2021-1)

by

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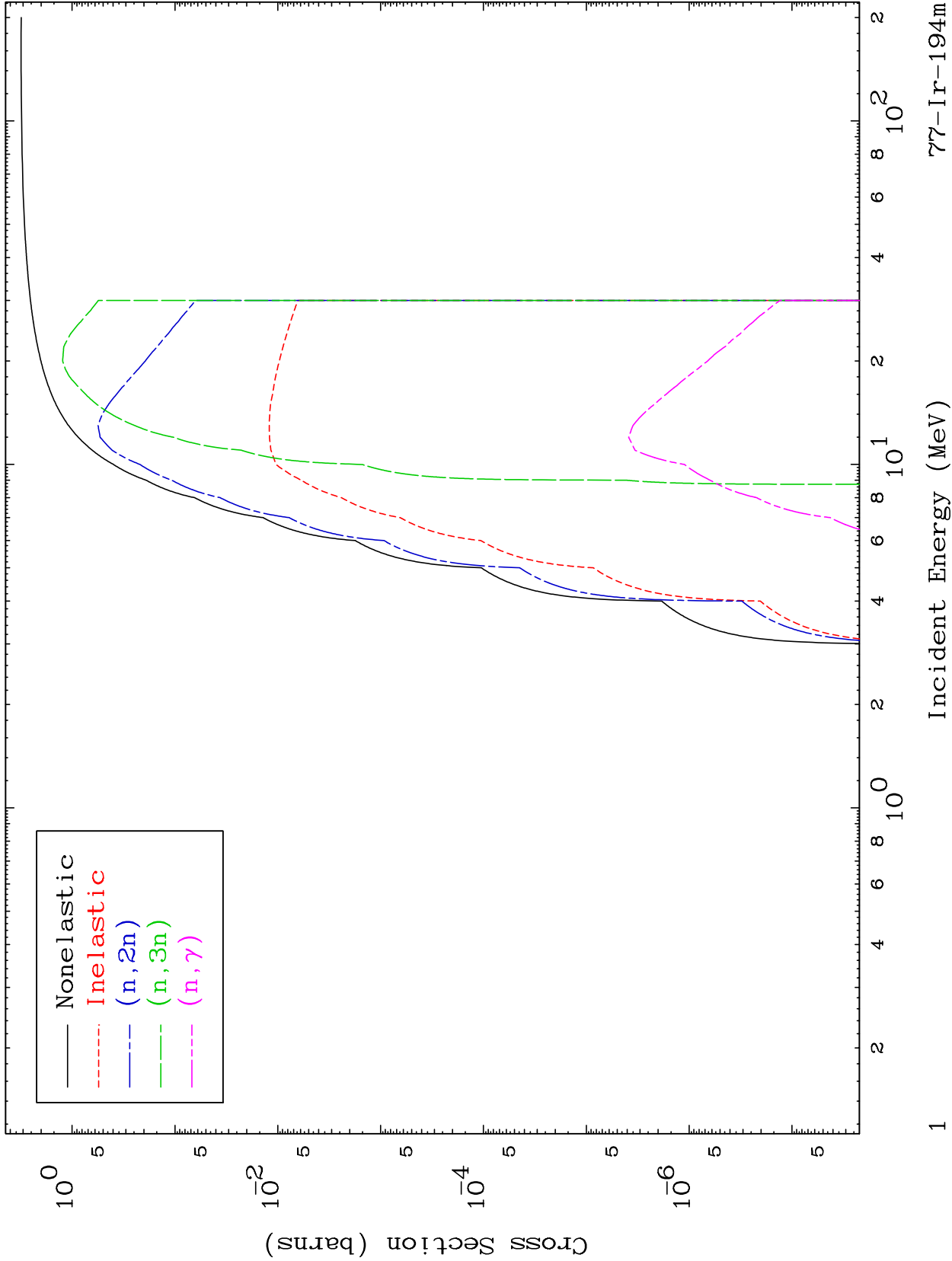
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7735

Deuteron Major  
0 Kelvin Cross Sections

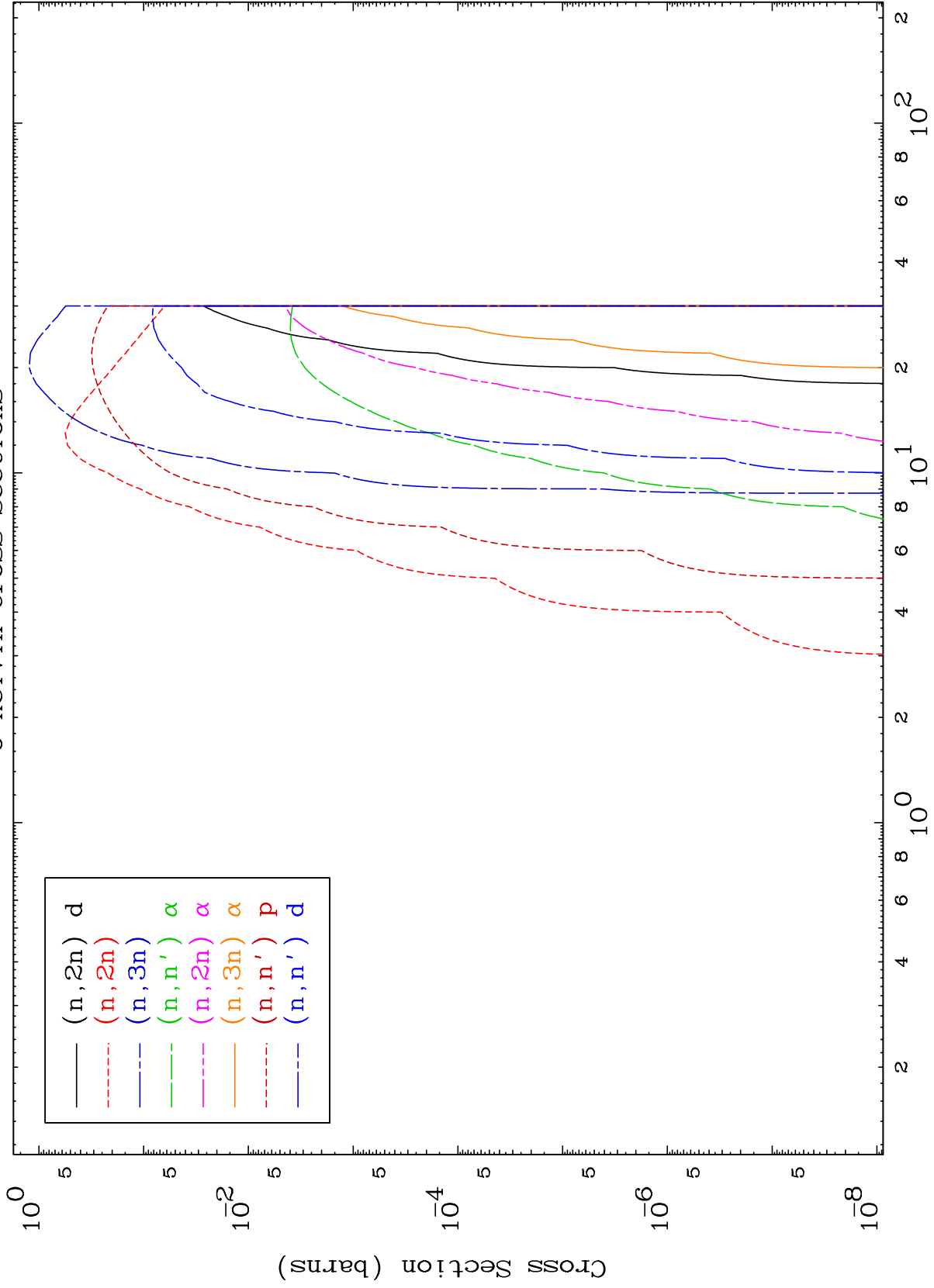
77-Ir-194m



MAT 7735

Deuteron Neutron Absorption  
0 Kelvin Cross Sections

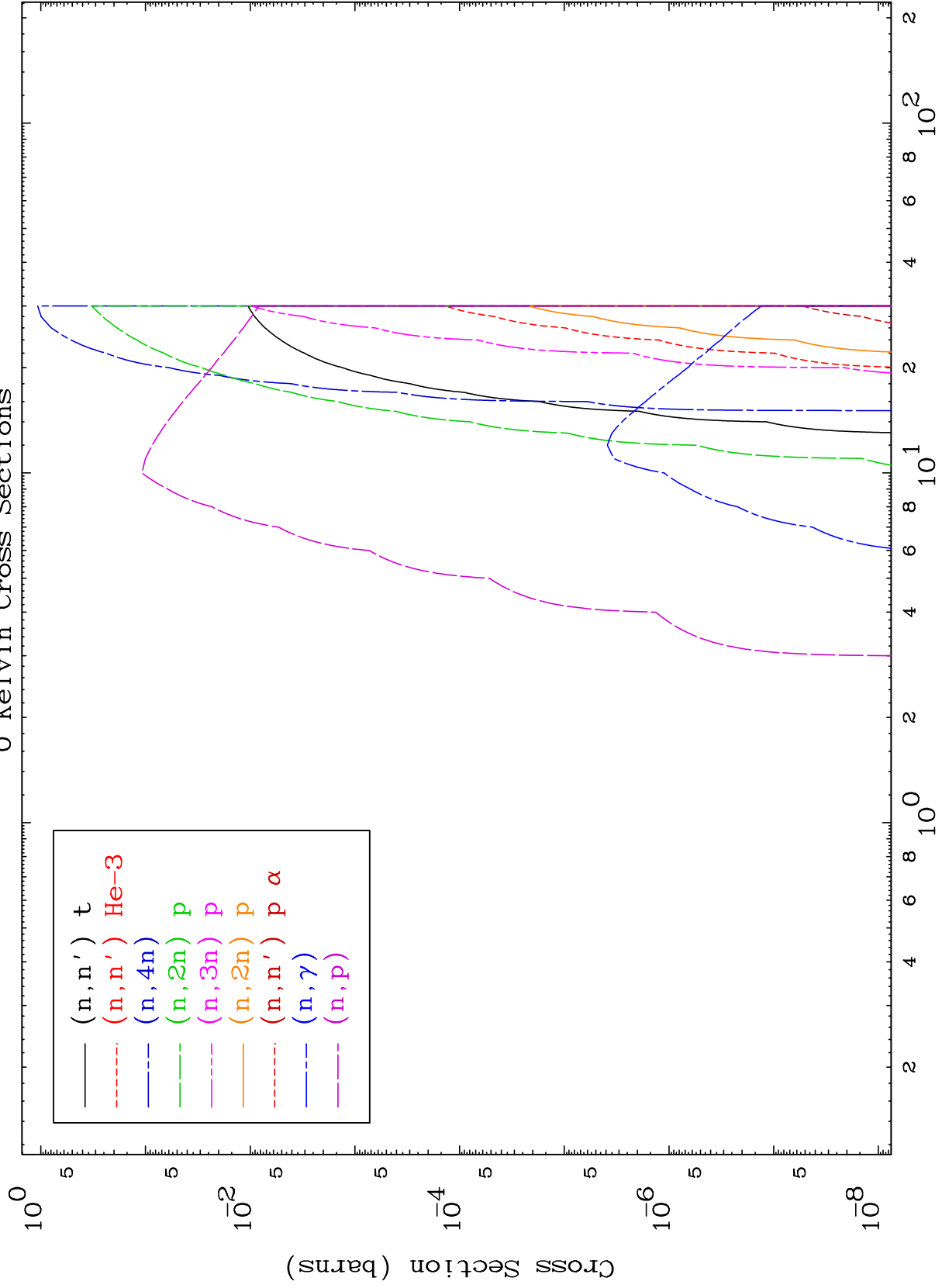
77-Ir-194m



MAT 7735

Deuteron Neutron Absorption  
0 Kelvin Cross Sections

77-Ir-194m



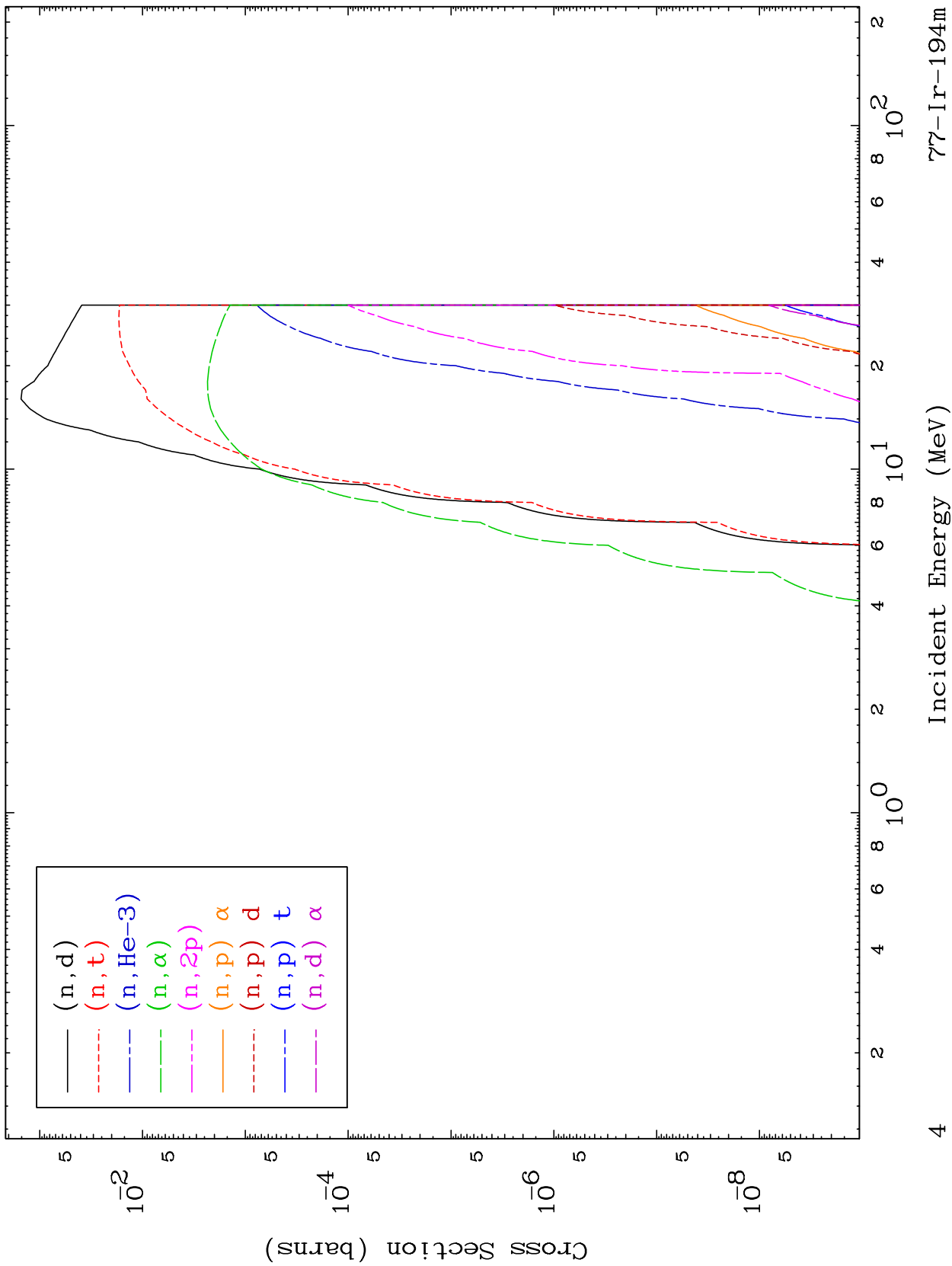
77-Ir-194m

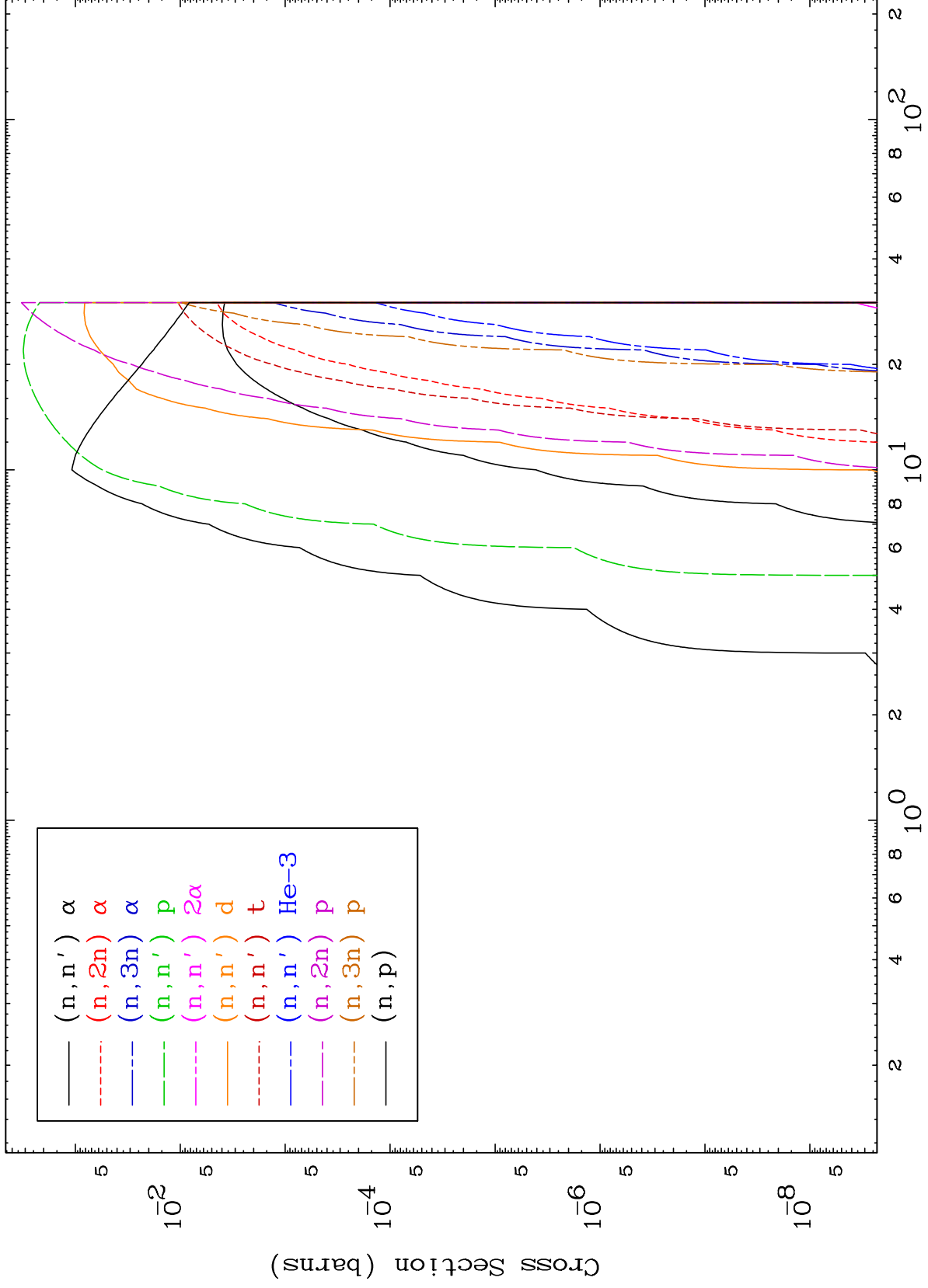
Incident Energy (MeV)

MAT 7735

Deuteron Neutron Absorption  
0 Kelvin Cross Sections

77-Ir-194m

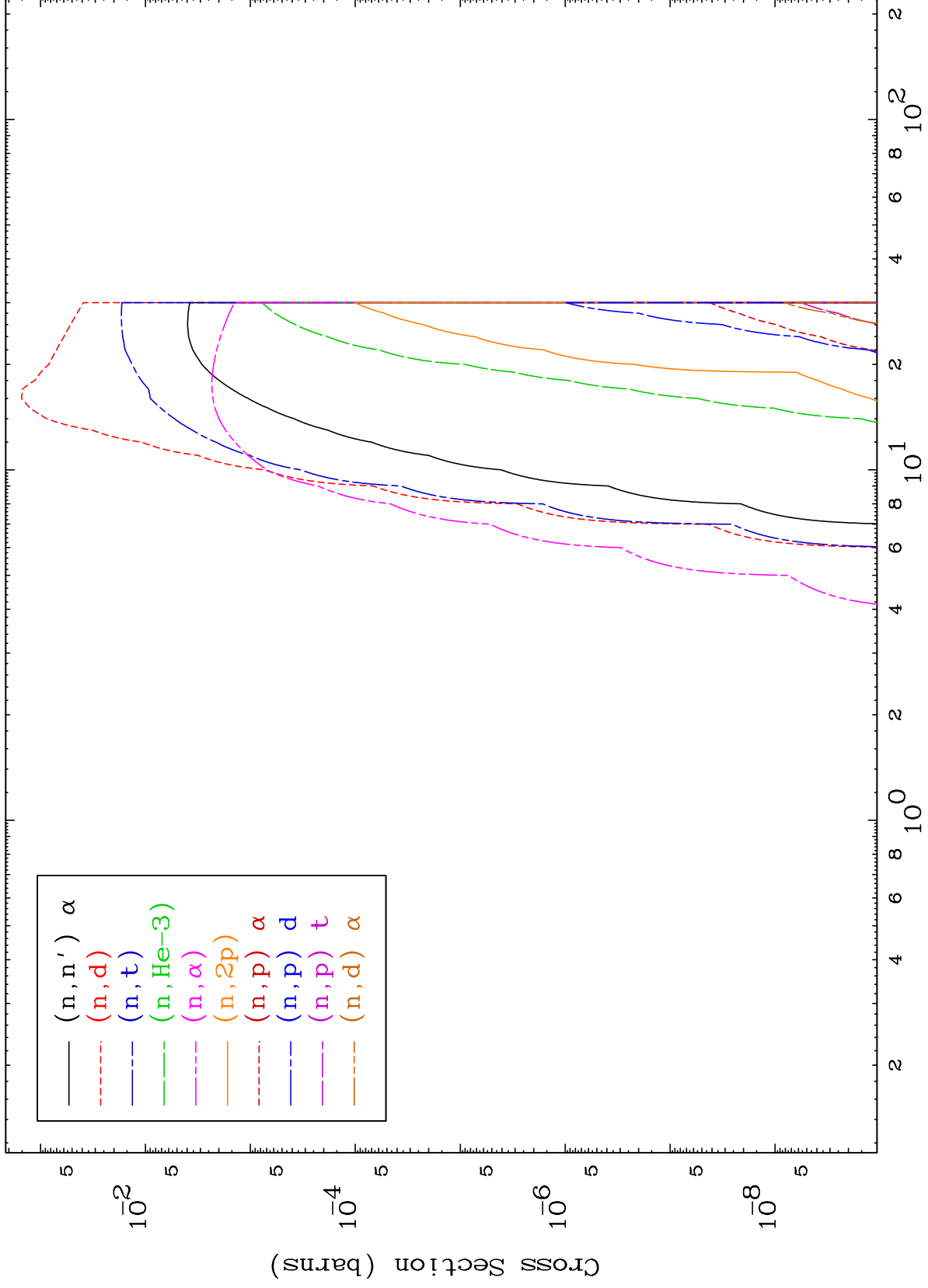




MAT 7735

Deuteron Charged Particle  
0 Kelvin Cross Sections

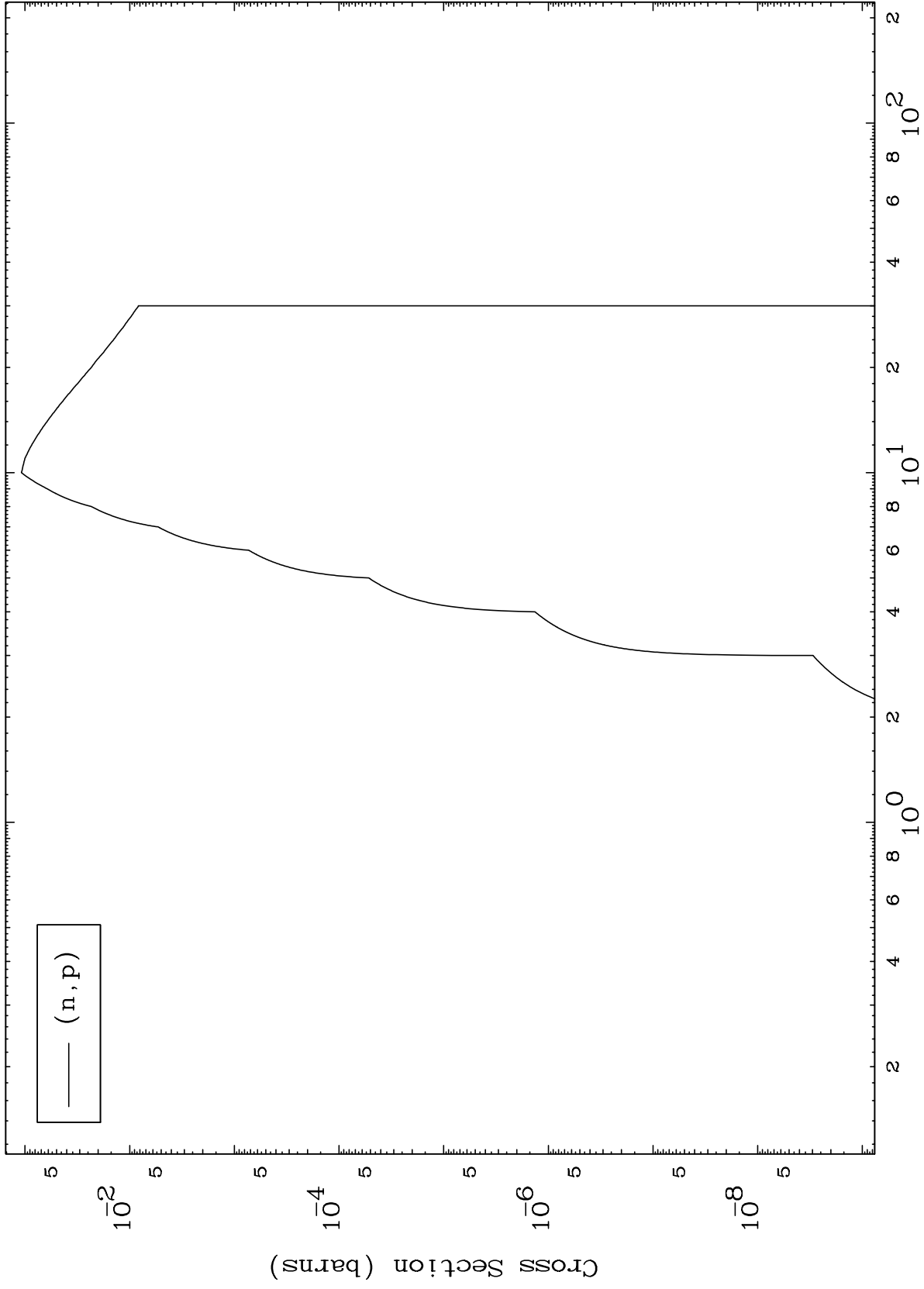
77-Ir-194m



MAT 7735

(d,p) Levels  
0 Kelvin Cross Sections

77-Ir-194m



7

Incident Energy (MeV)

77-Ir-194m

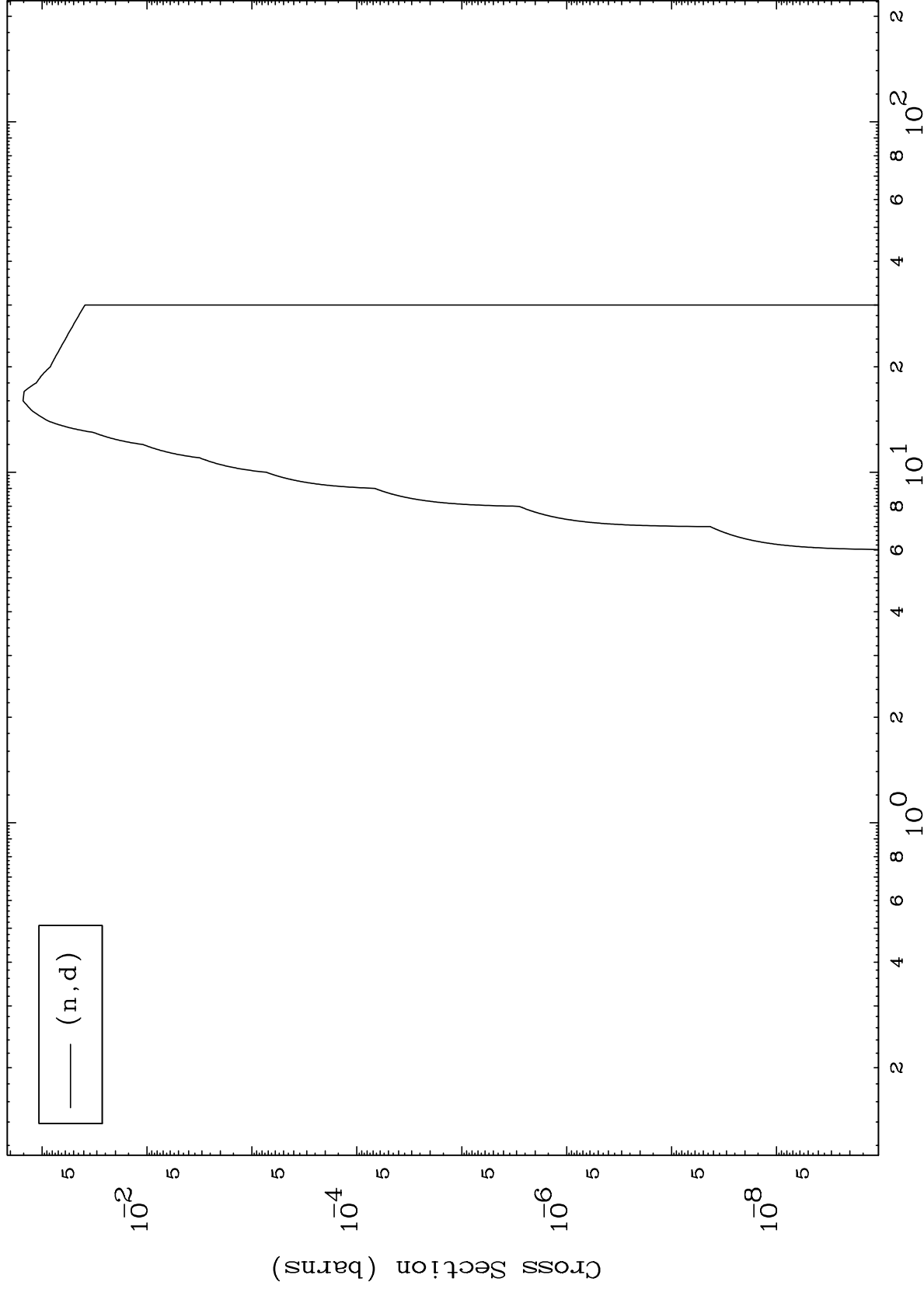


MAT 7735

(d,d) Levels

77-Ir-194m

0 Kelvin Cross Sections

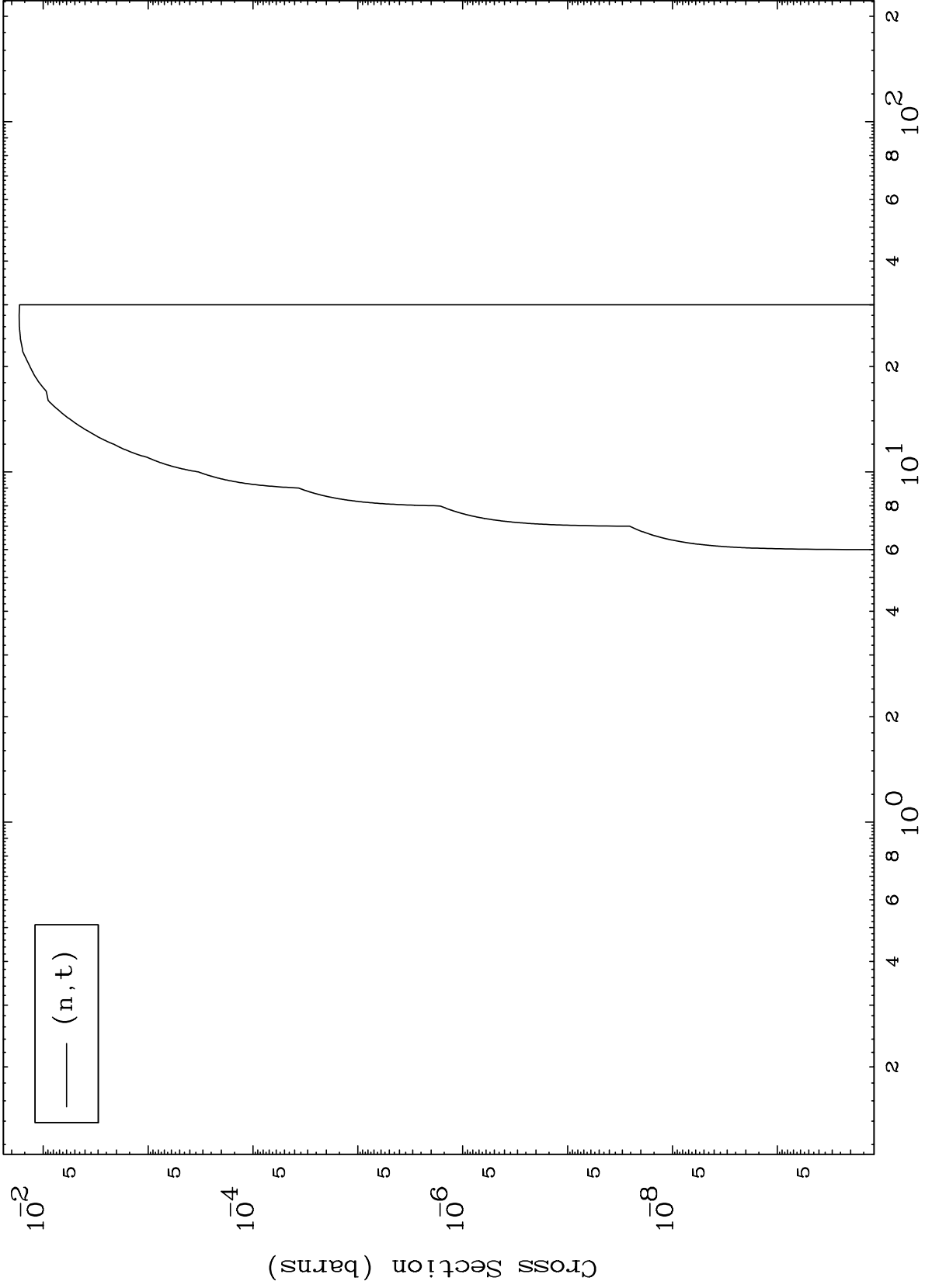


MAT 7735

(d, t) Levels

77-Ir-194m

0 Kelvin Cross Sections

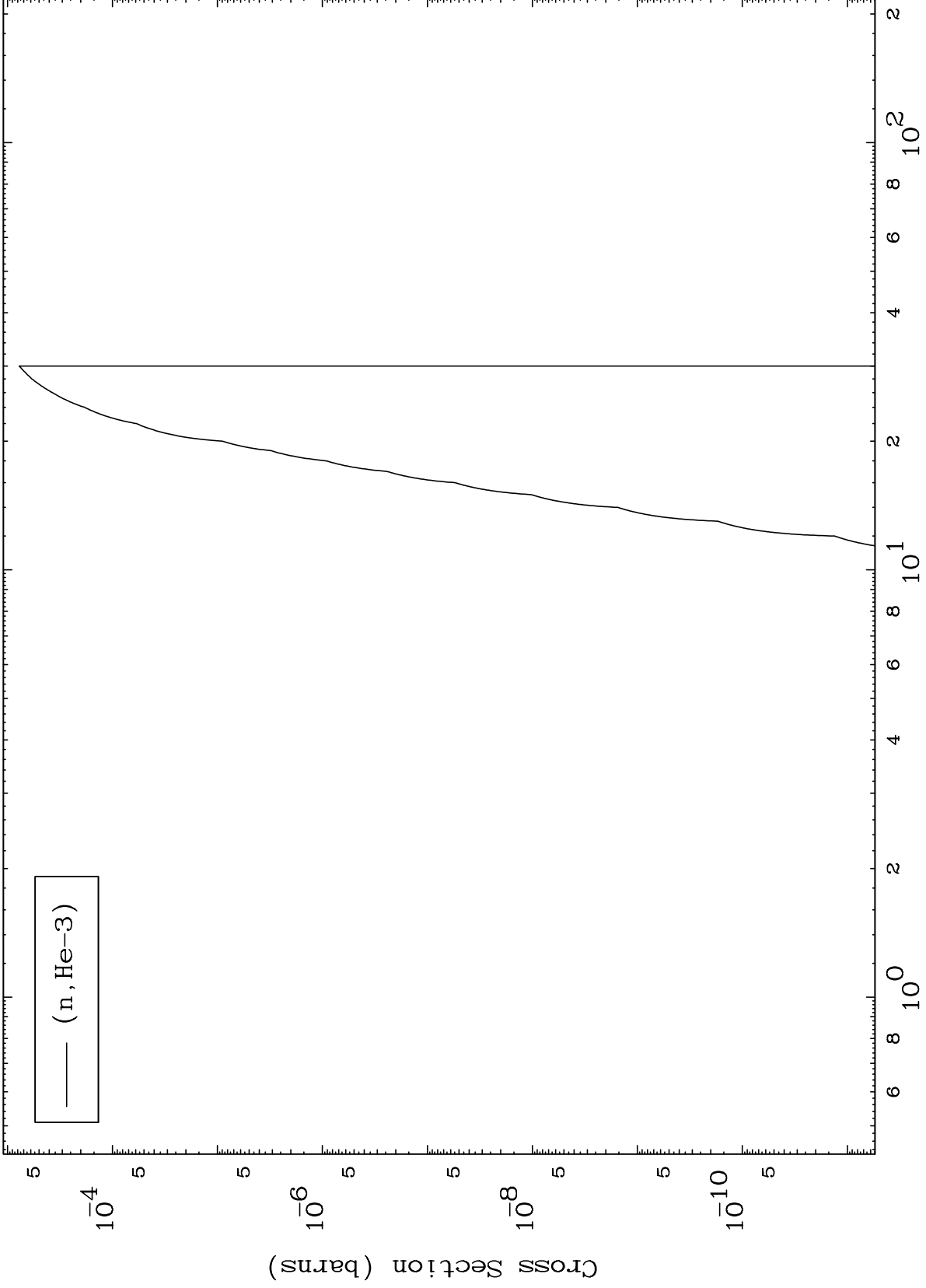


MAT 7735

(d,He3) Levels

77-Ir-194m

0 Kelvin Cross Sections



10

Incident Energy (MeV)

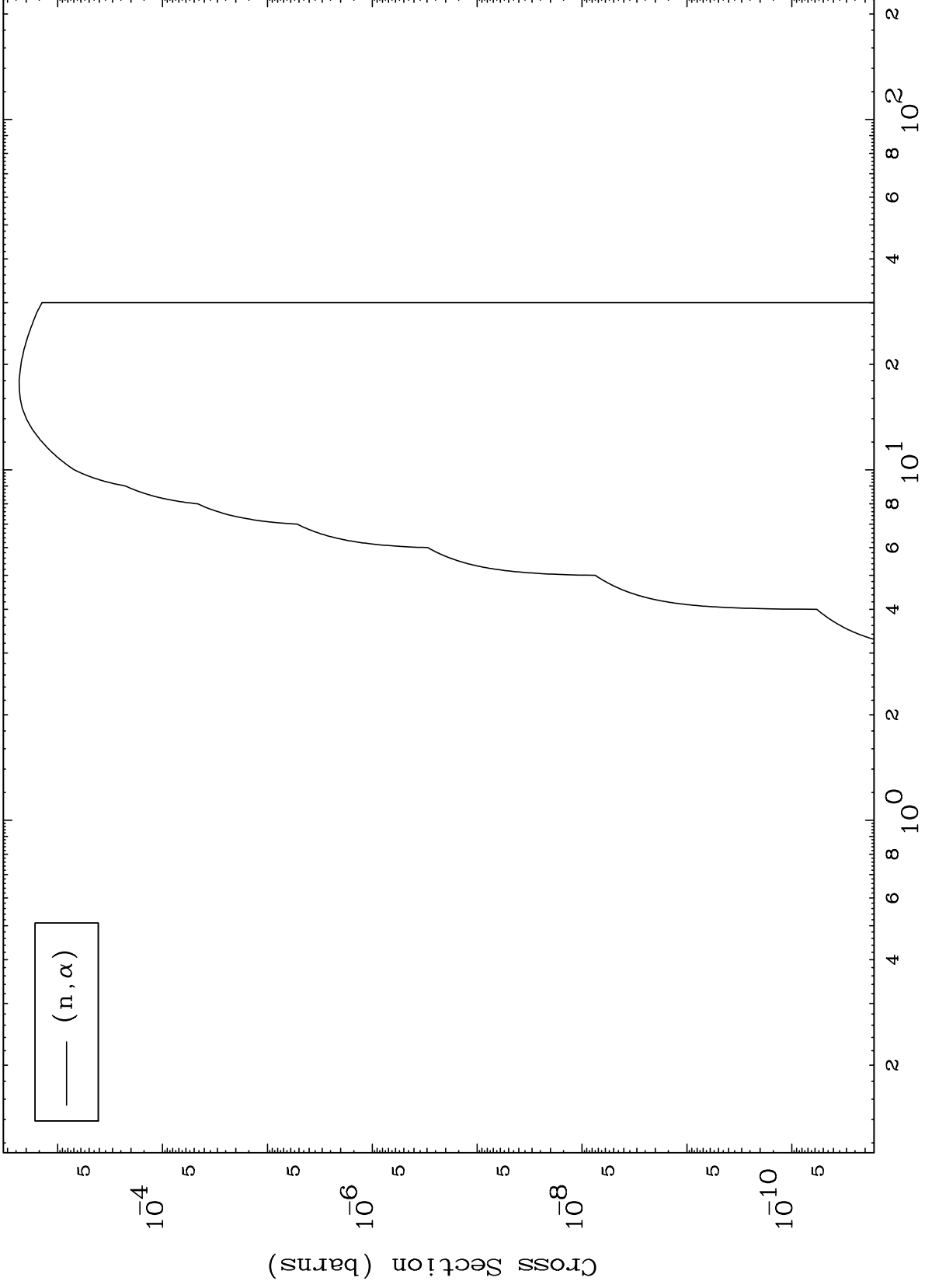
77-Ir-194m

MAT 7735

(d,  $\alpha$ ) Levels

<sup>77</sup>Ir-194m

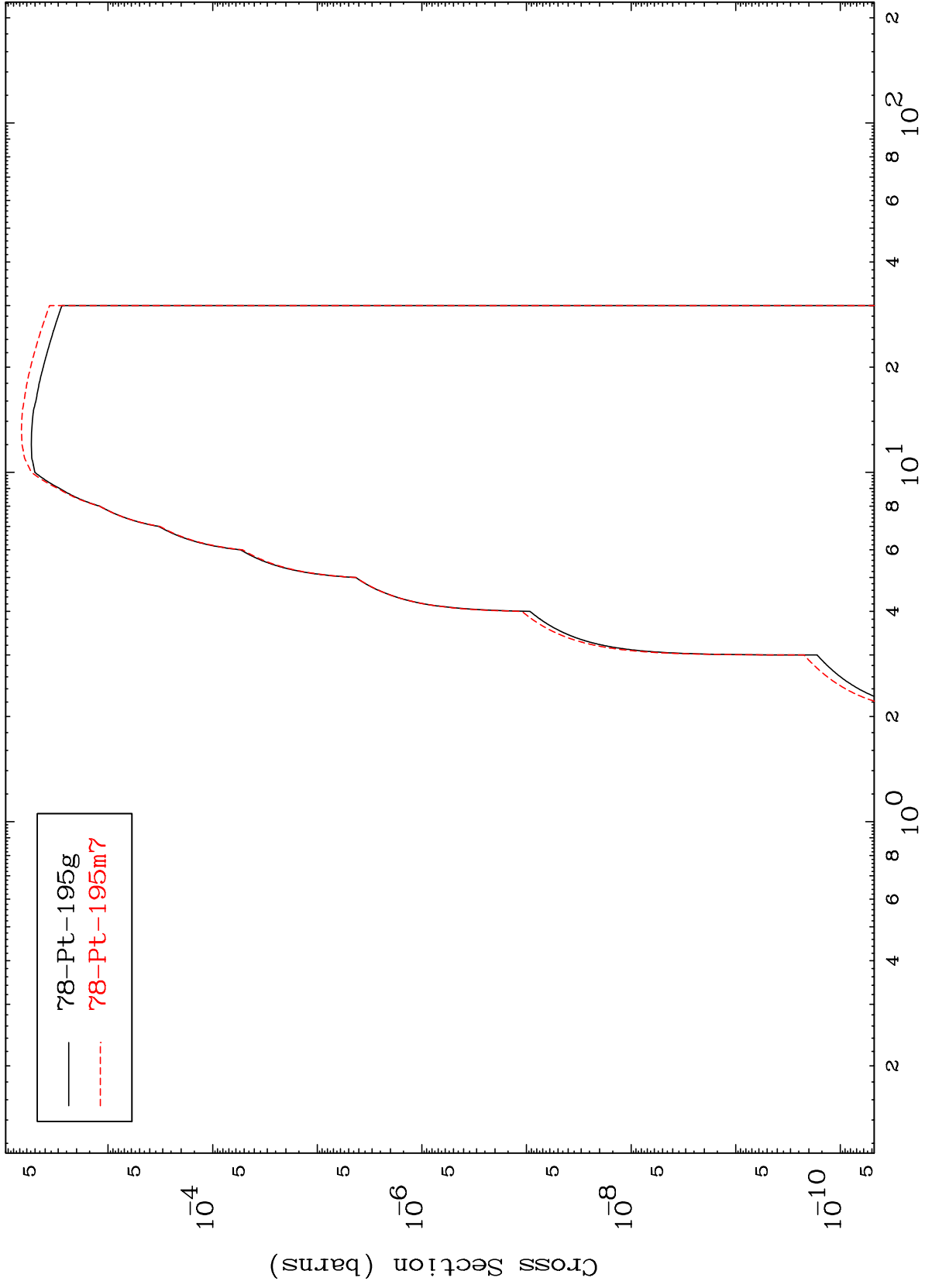
0 Kelvin Cross Sections



MAT 7735

Inelastic  
Radionuclide Production Cross Section

<sup>77</sup>Ir-194m

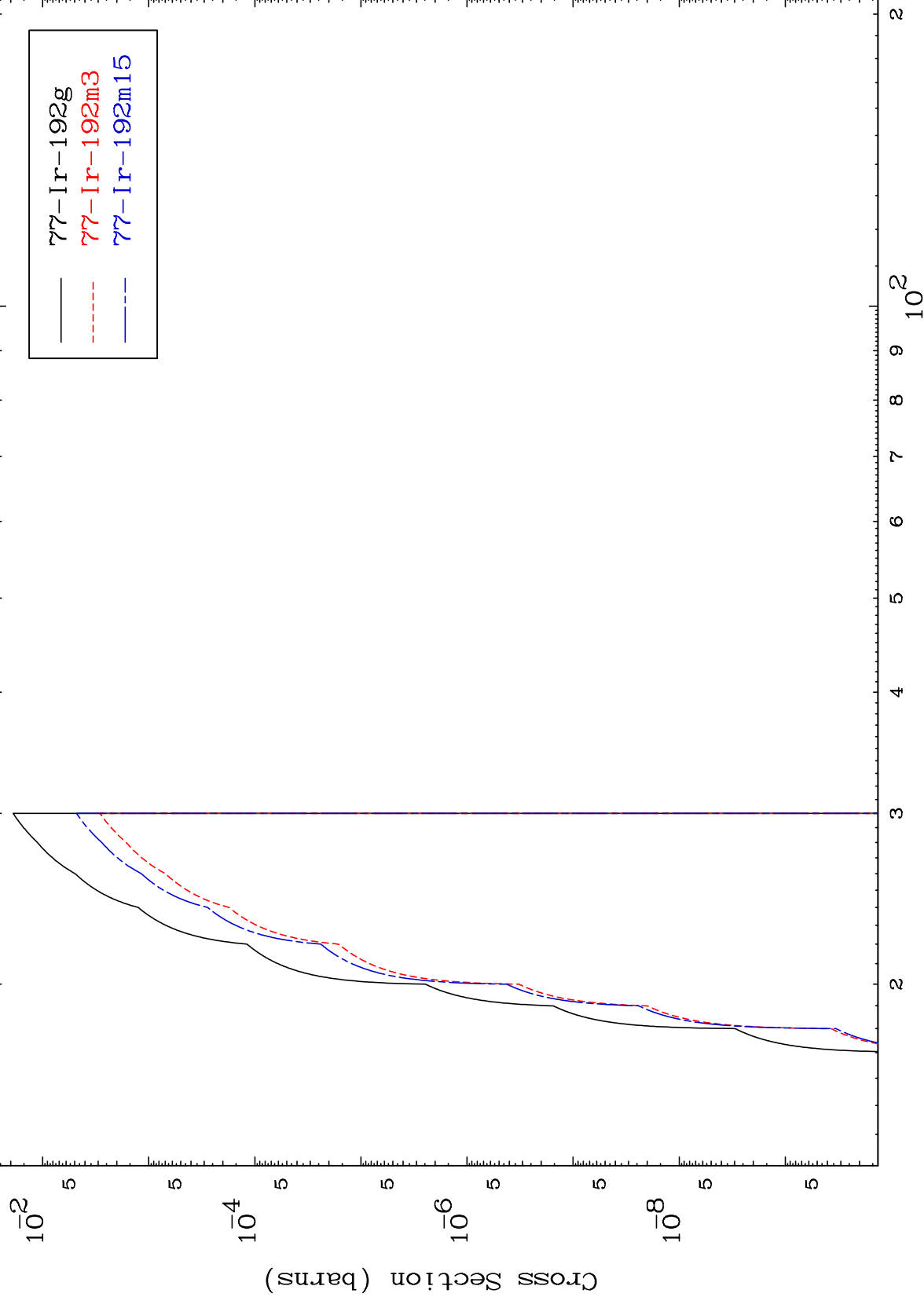


MAT 7735

(n,2n) d

77-Ir-194m

Radionuclide Production Cross Section

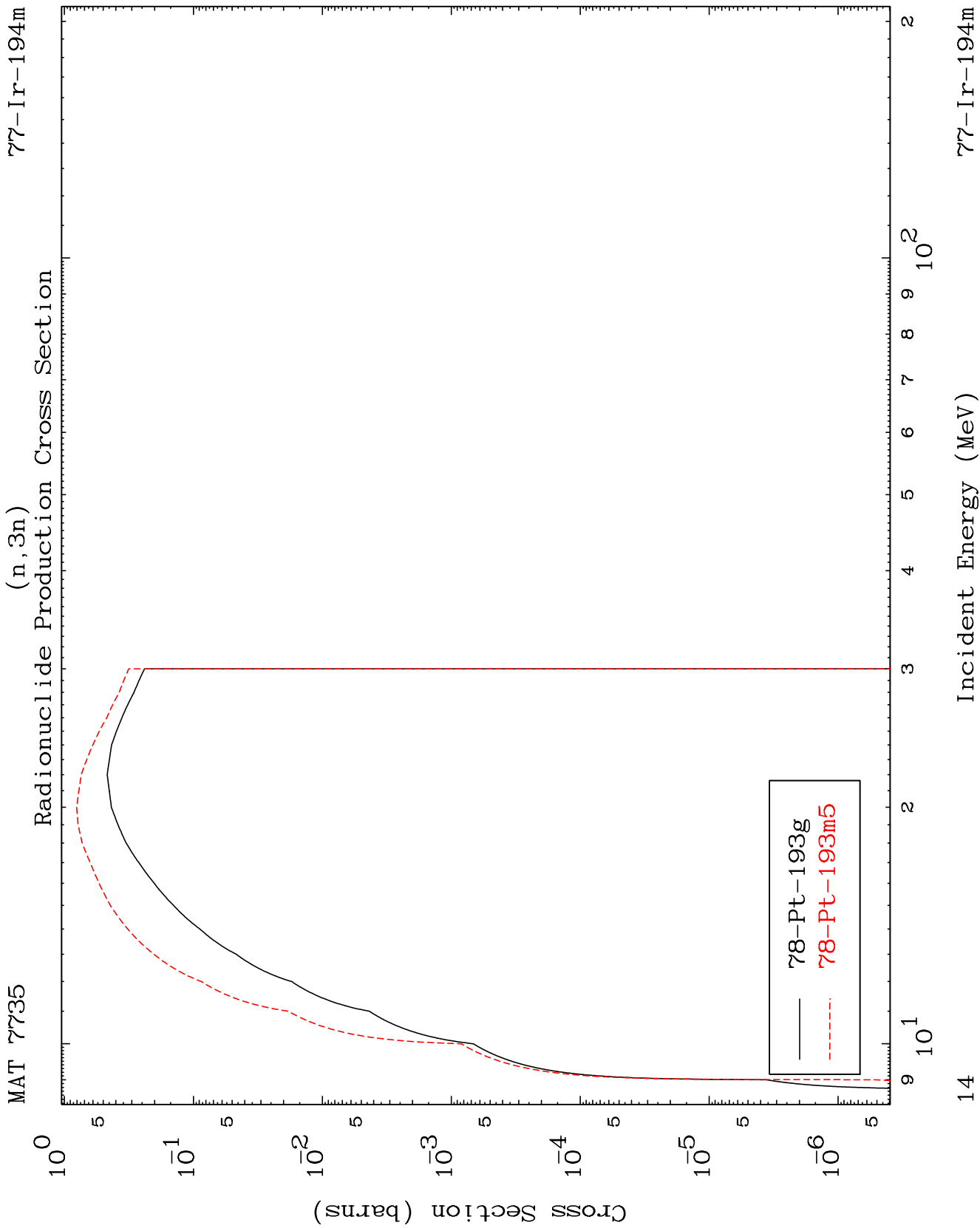


77-Ir-192g  
77-Ir-192m3  
77-Ir-192m15

13

Incident Energy (MeV)

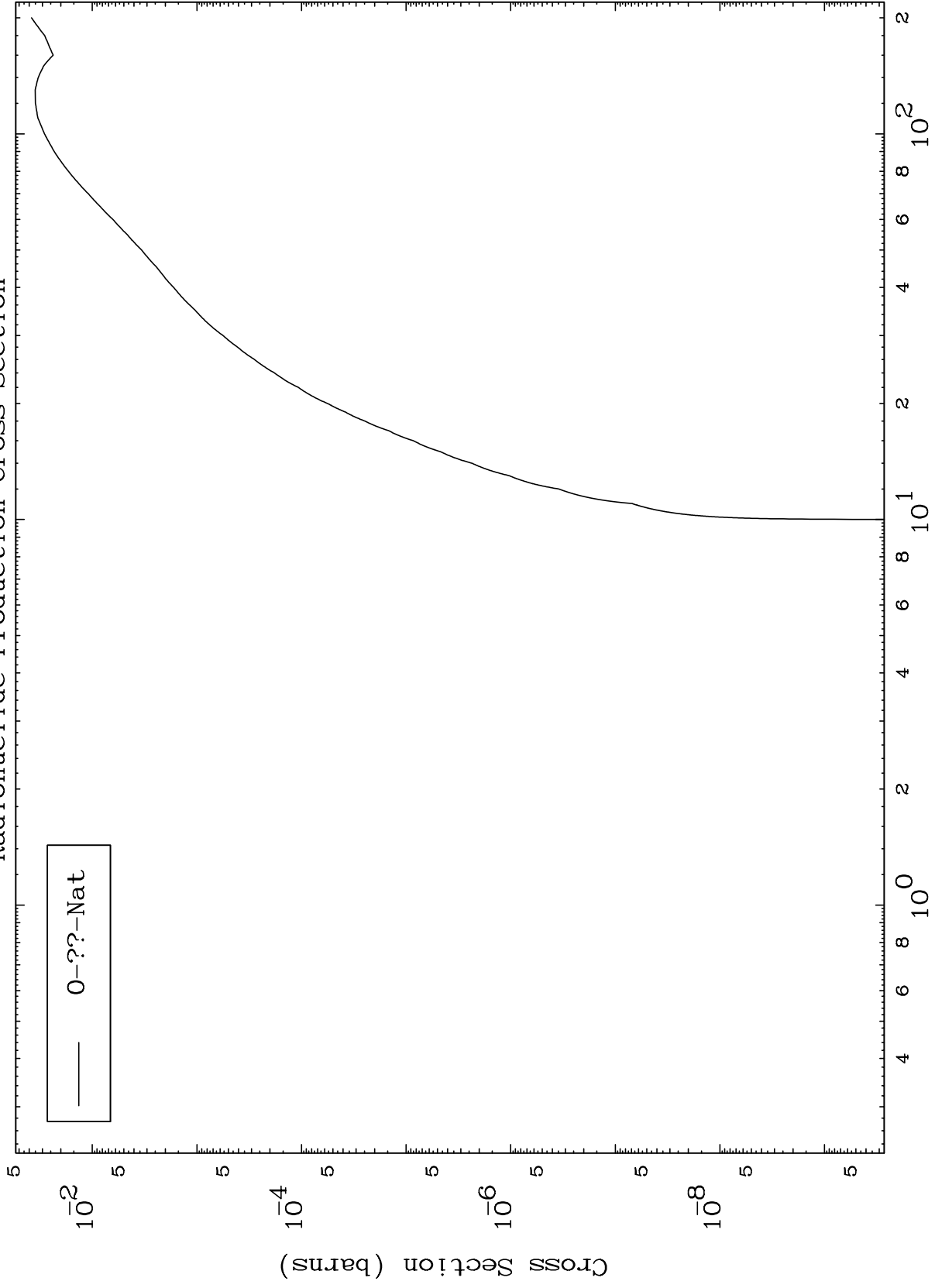
77-Ir-194m



MAT 7735

Fission  
Radionuclide Production Cross Section

<sup>77</sup>Ir-194m



15

Incident Energy (MeV)

<sup>77</sup>Ir-194m

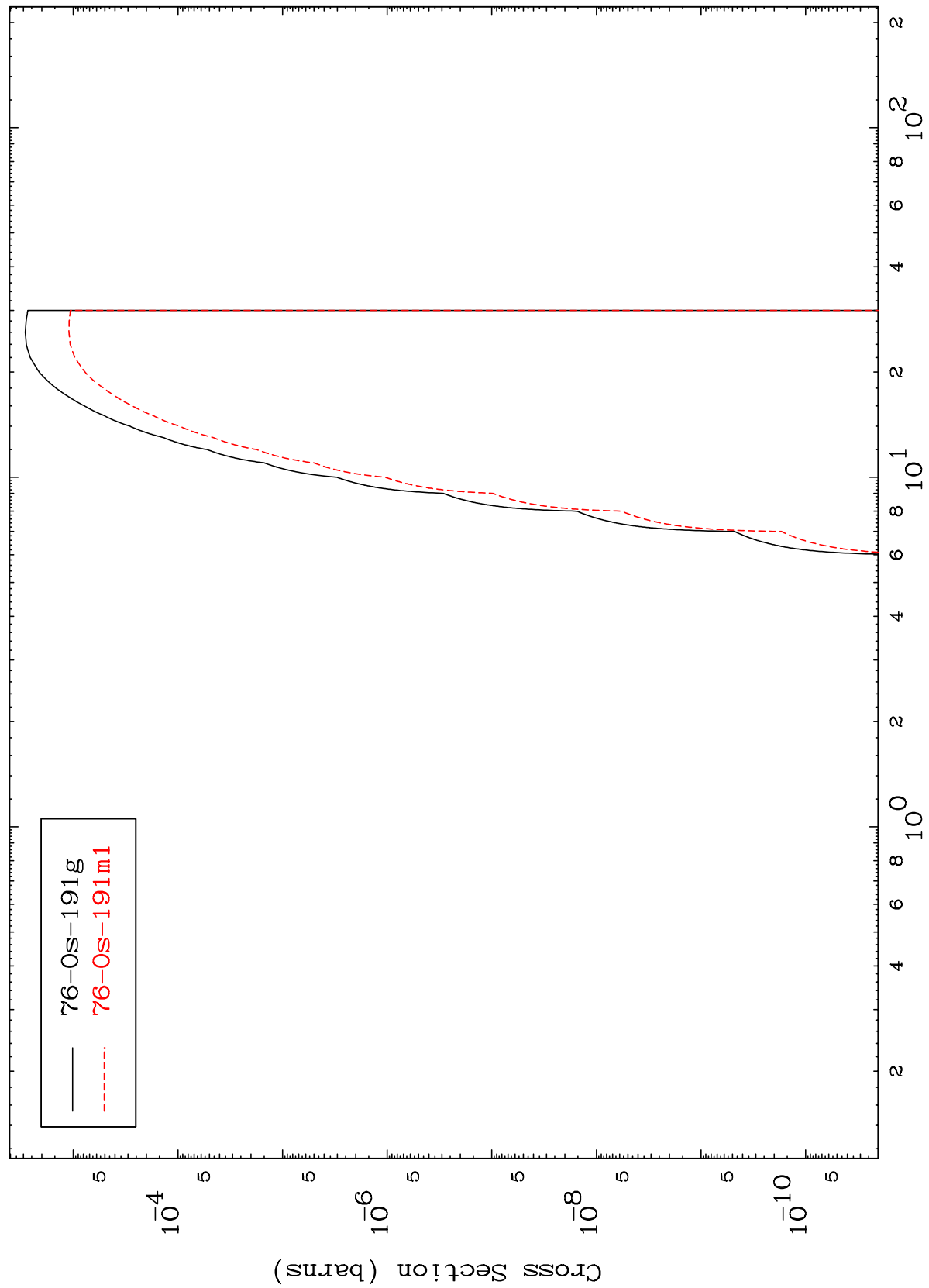


MAT 7735

$(n, n') \alpha$

$^{77}\text{Ir-194m}$

Radionuclide Production Cross Section

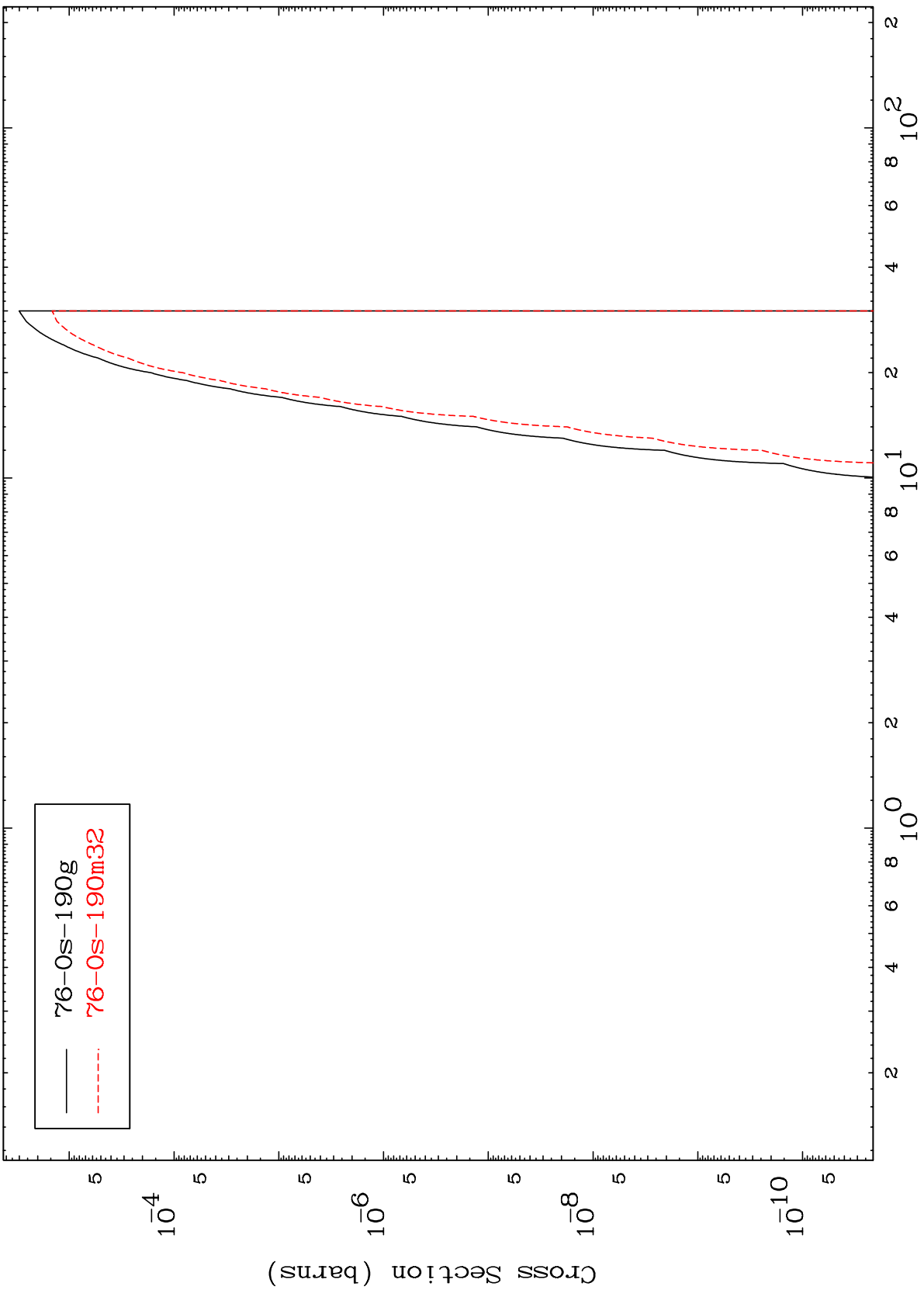


MAT 7735

(n,2n)  $\alpha$

<sup>77</sup>Ir-194m

Radionuclide Production Cross Section



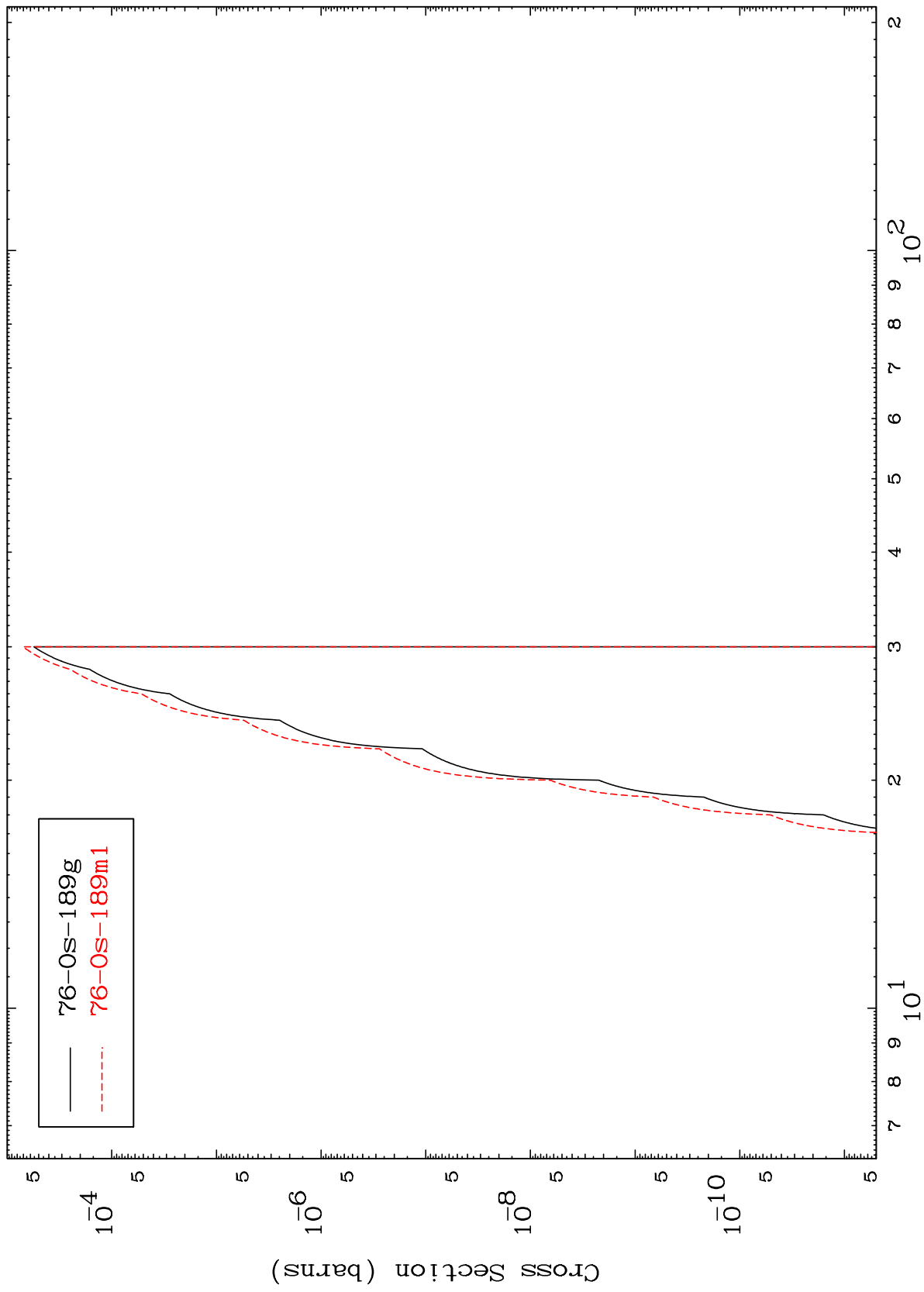
76-0s-190g  
76-0s-190m32

MAT 7735

(n,3n)  $\alpha$

77-Ir-194m

Radionuclide Production Cross Section



18

Incident Energy (MeV)

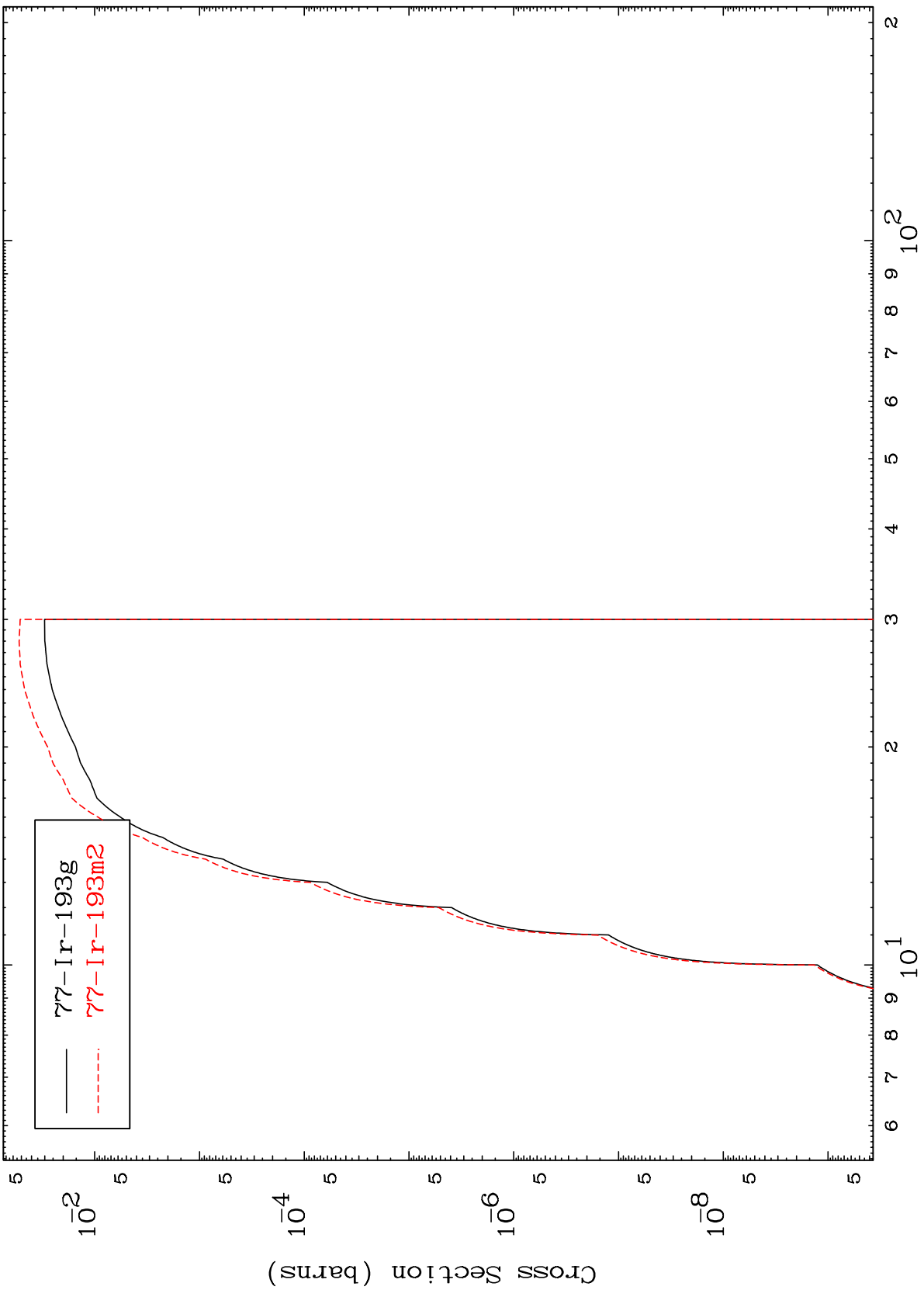
77-Ir-194m

MAT 7735

(n,n') d

<sup>77</sup>Ir-194m

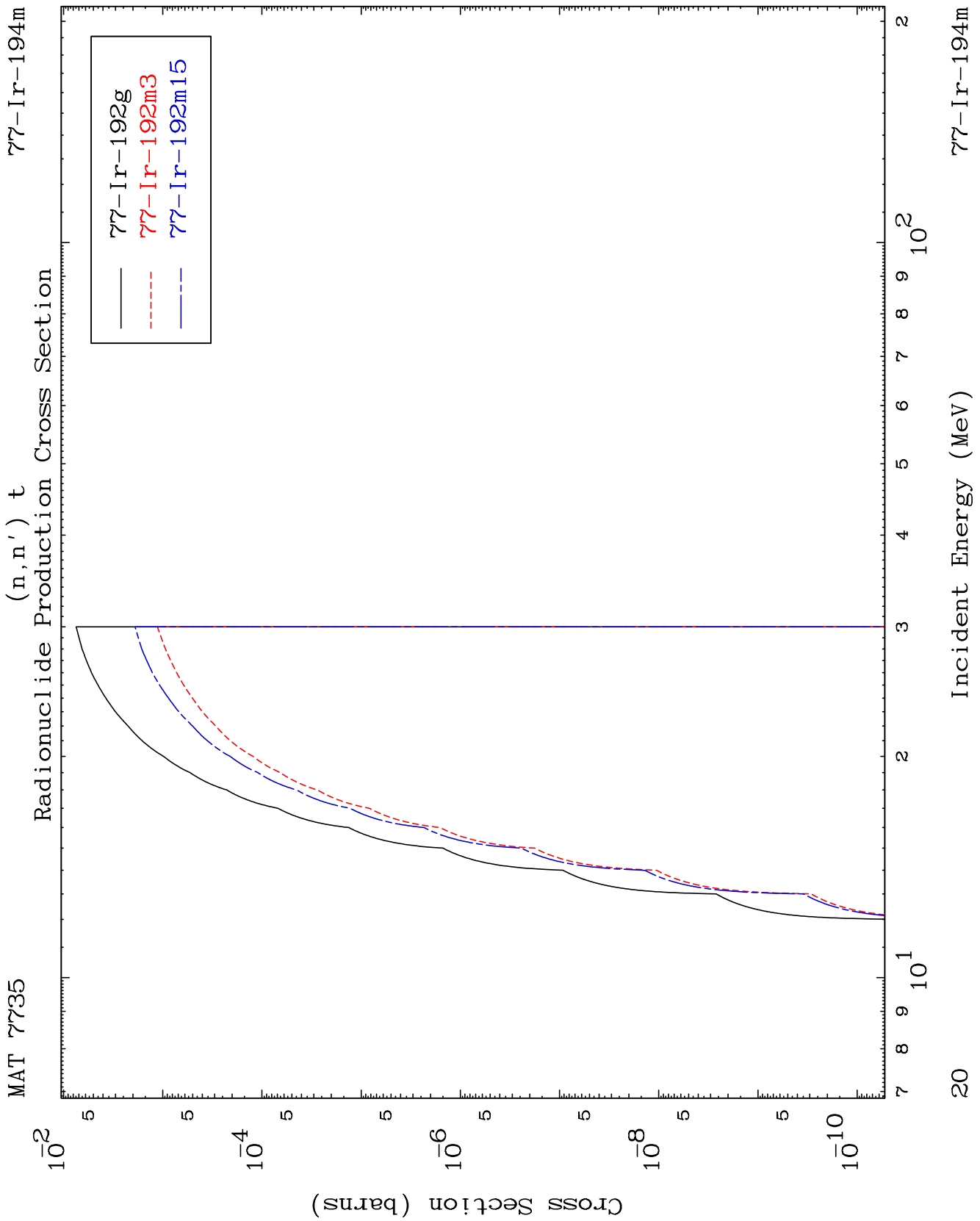
Radionuclide Production Cross Section



19

Incident Energy (MeV)

<sup>77</sup>Ir-194m

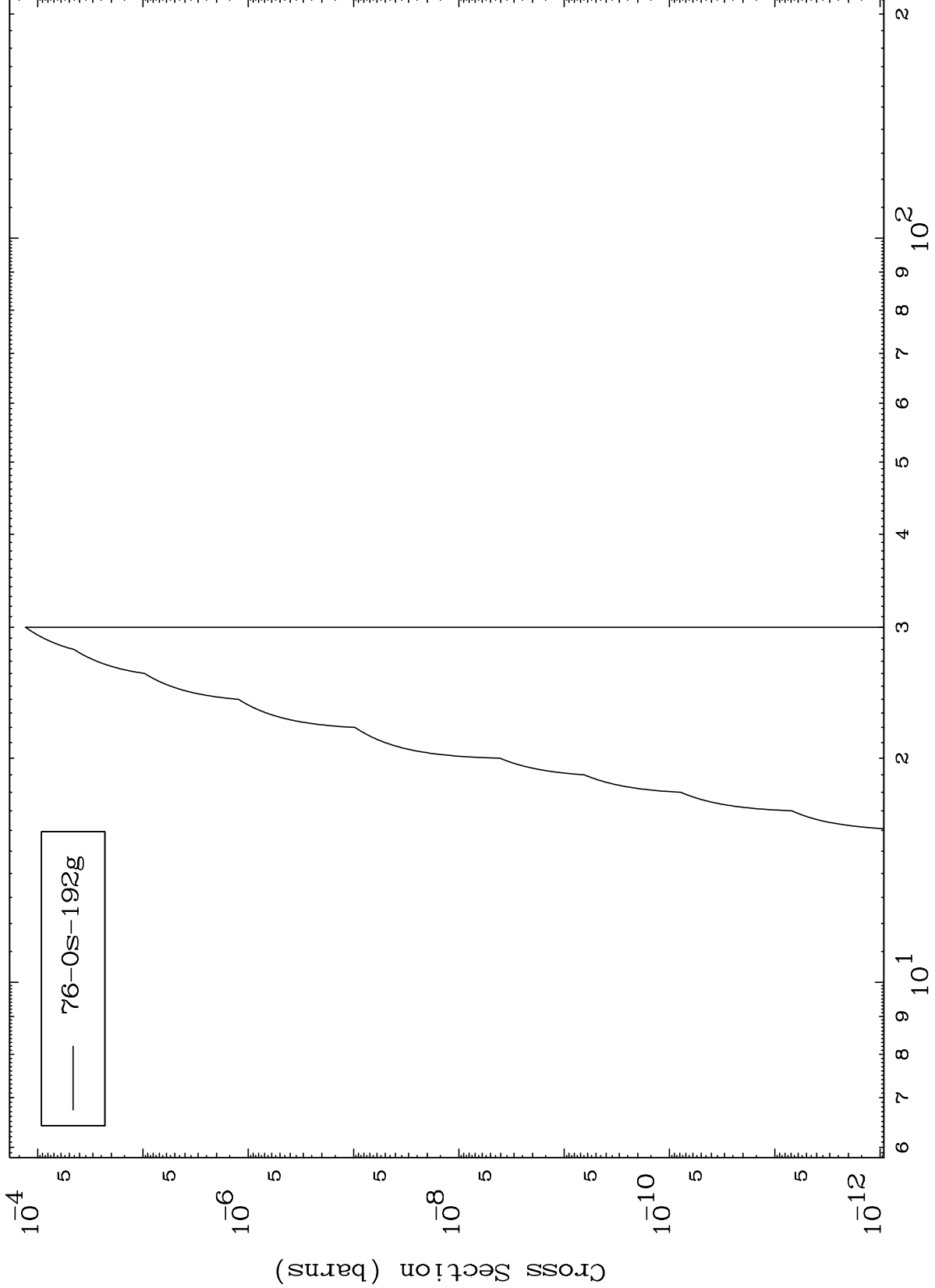


MAT 7735

(n,n') He-3

77-Ir-194m

Radionuclide Production Cross Section



76-Os-192g

Incident Energy (MeV)

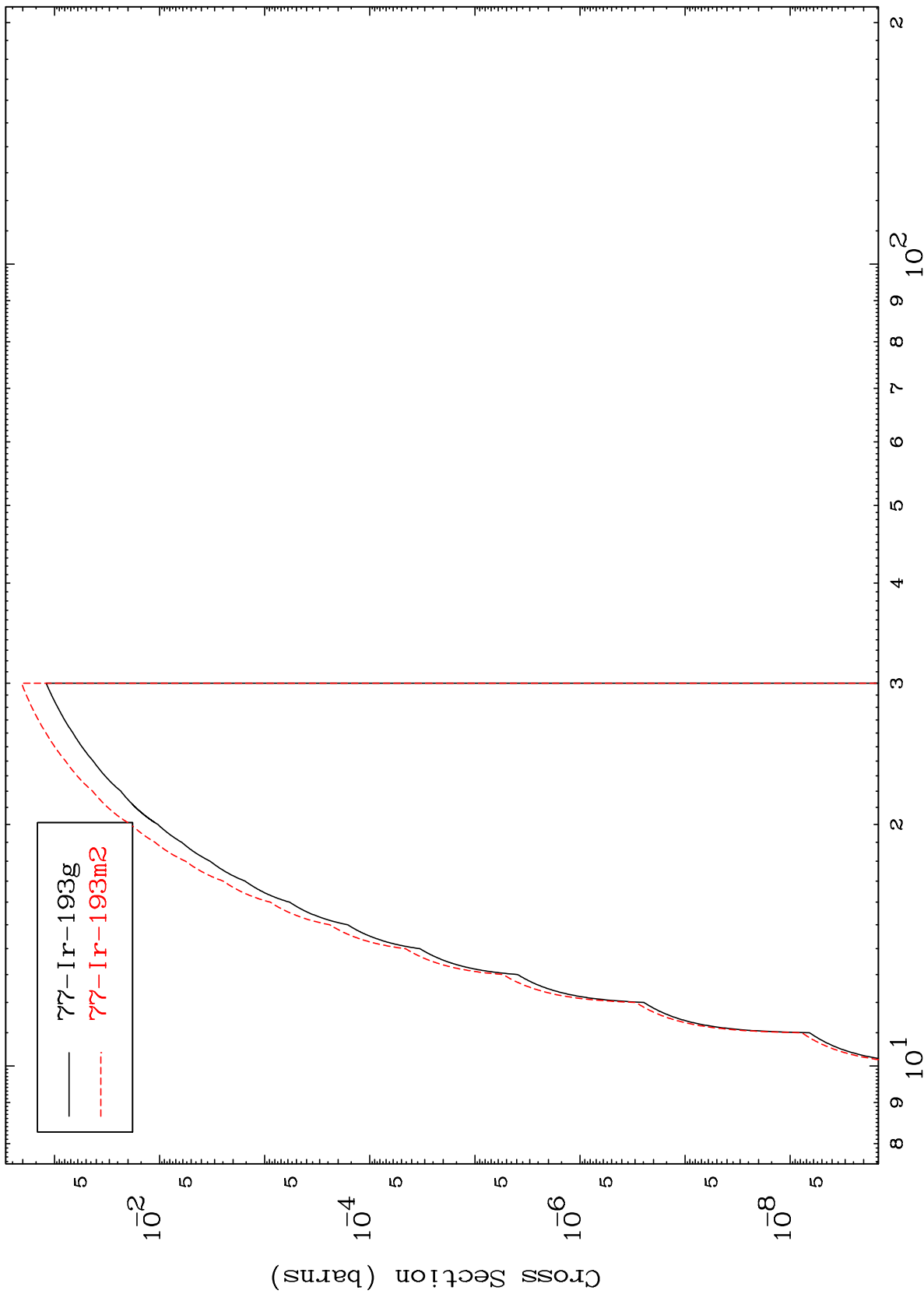
77-Ir-194m

MAT 7735

(n,2n) p

<sup>77</sup>Ir-194m

Radionuclide Production Cross Section



— <sup>77</sup>Ir-193g  
- - - <sup>77</sup>Ir-193m2

Incident Energy (MeV)

<sup>77</sup>Ir-194m

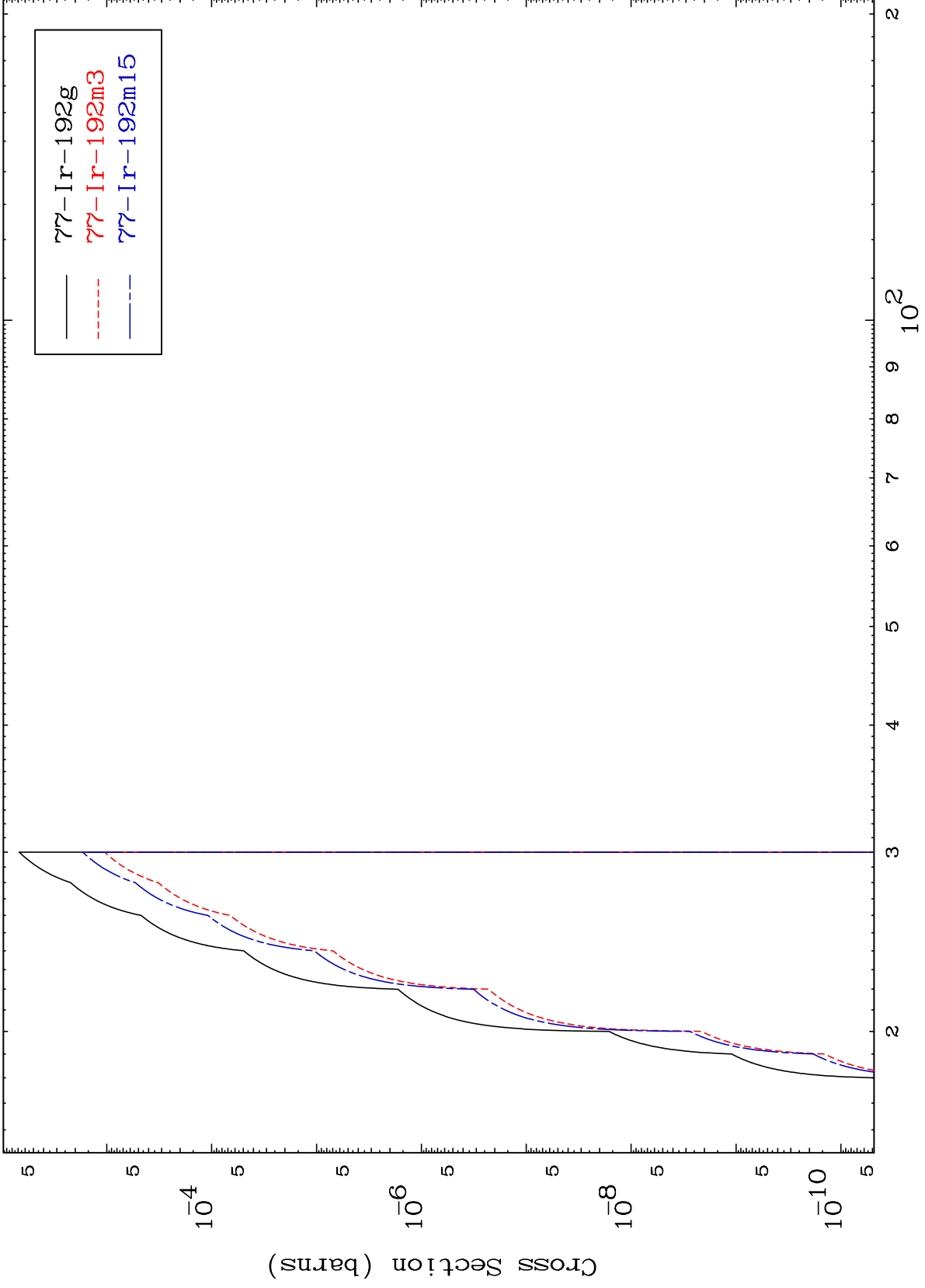
22

MAT 7735

(n,3n) p

<sup>77</sup>Ir-194m

Radionuclide Production Cross Section



23

Incident Energy (MeV)

<sup>77</sup>Ir-194m

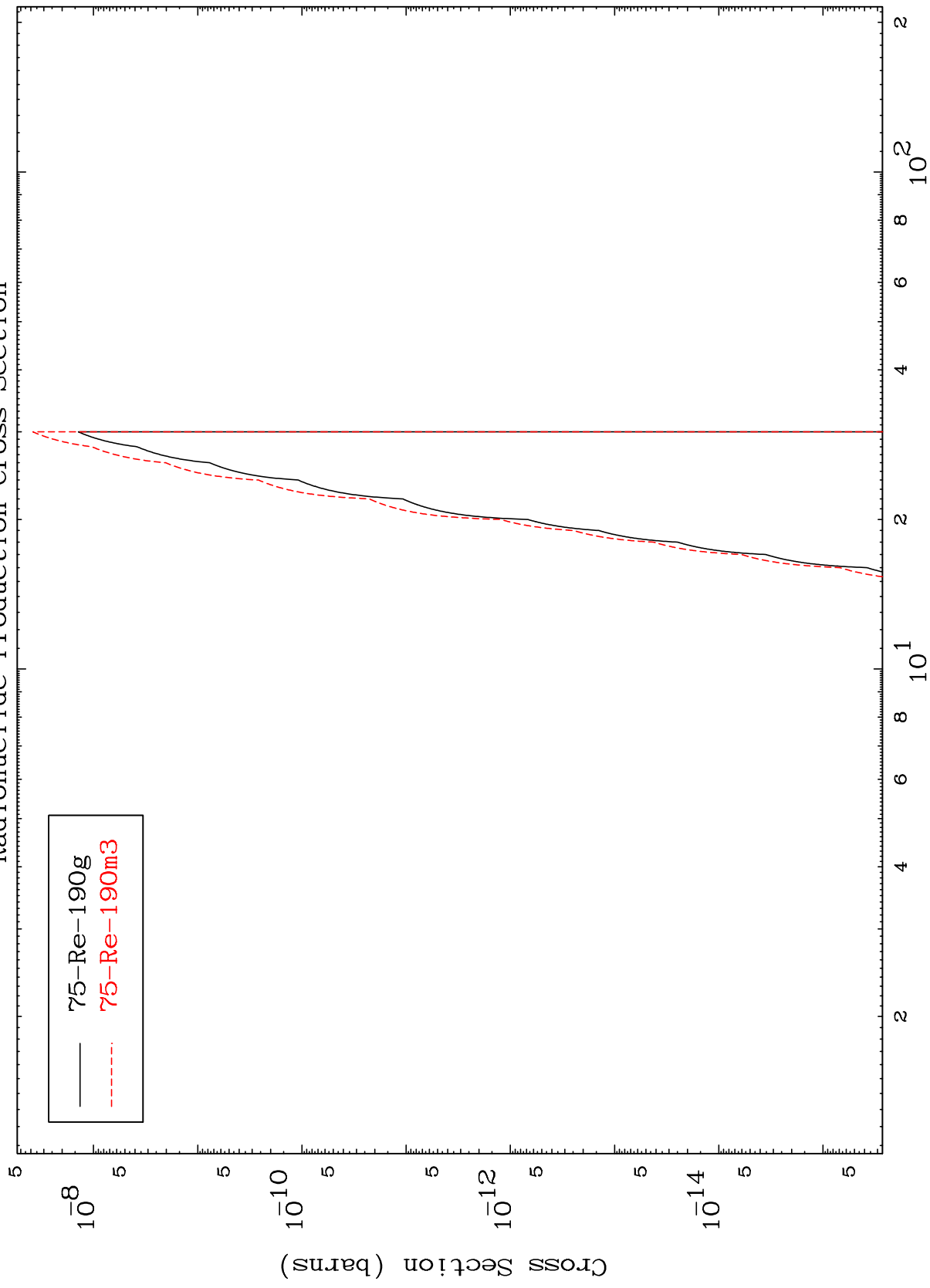


MAT 7735

(n,n') p  $\alpha$

77-Ir-194m

Radionuclide Production Cross Section



24

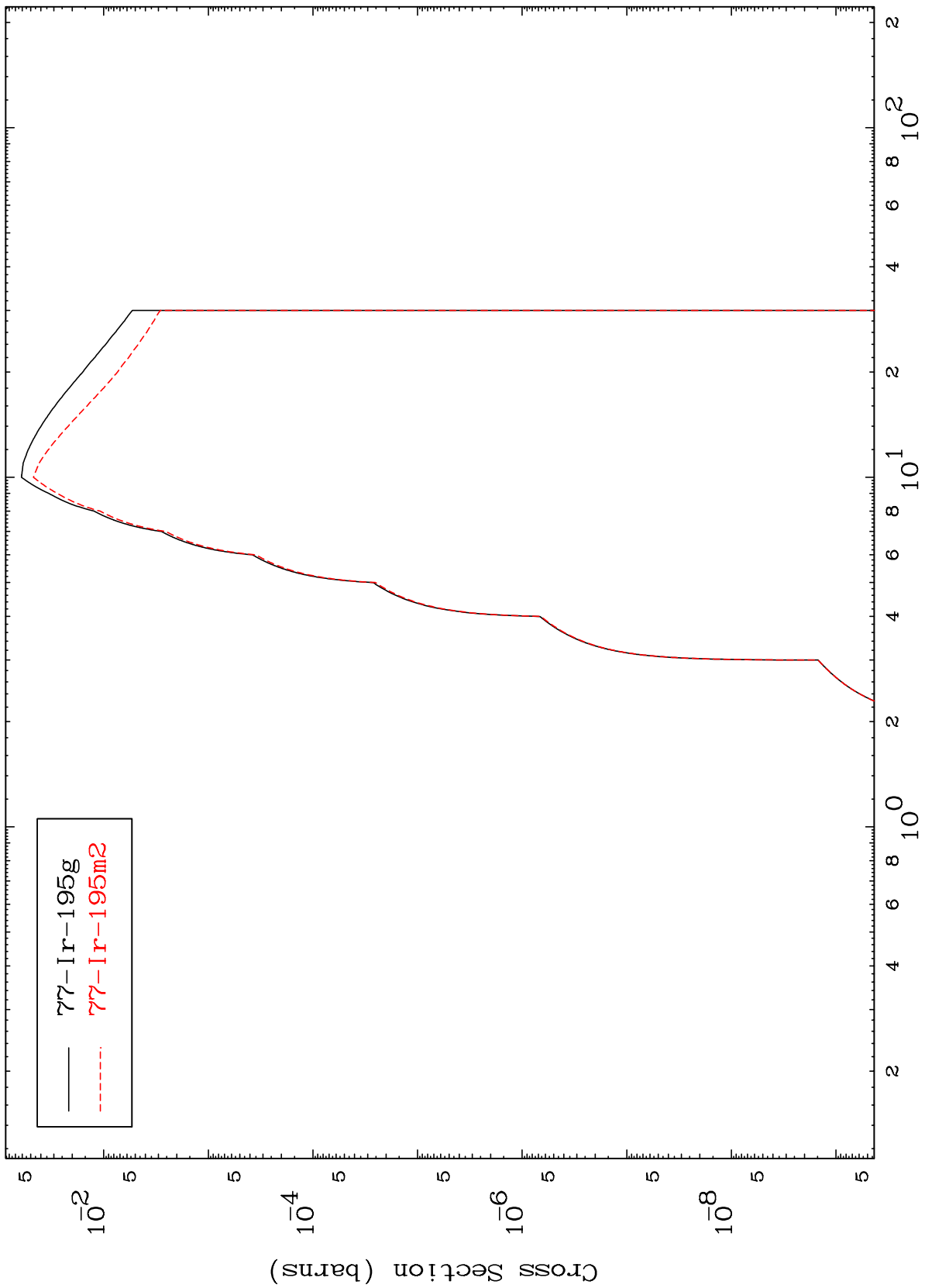
Incident Energy (MeV)

77-Ir-194m

MAT 7735

77-Ir-194m

(n,p)  
Radionuclide Production Cross Section



77-Ir-194m

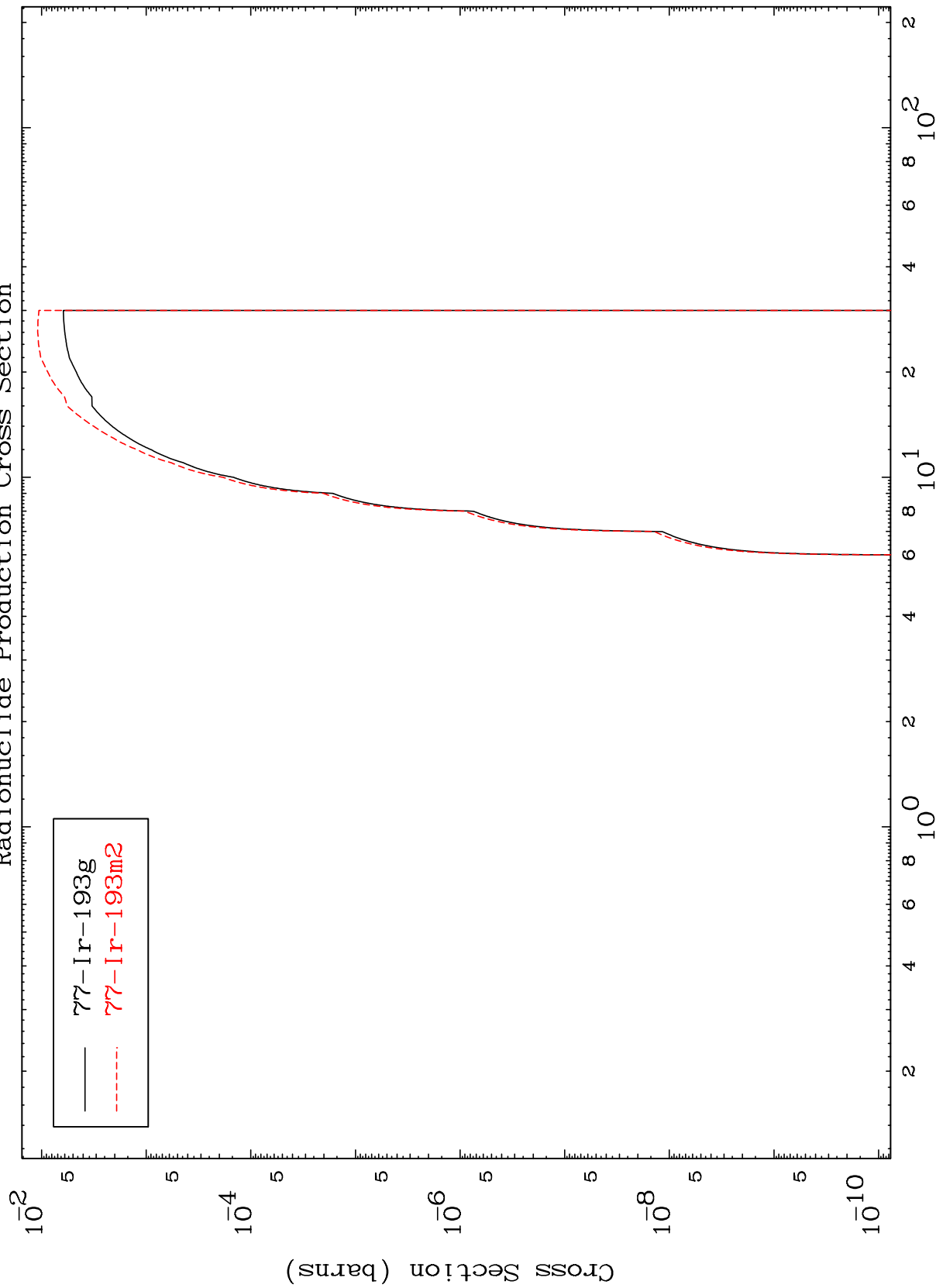
Incident Energy (MeV)

25

MAT 7735

77-Ir-194m

Radionuclide Production Cross Section



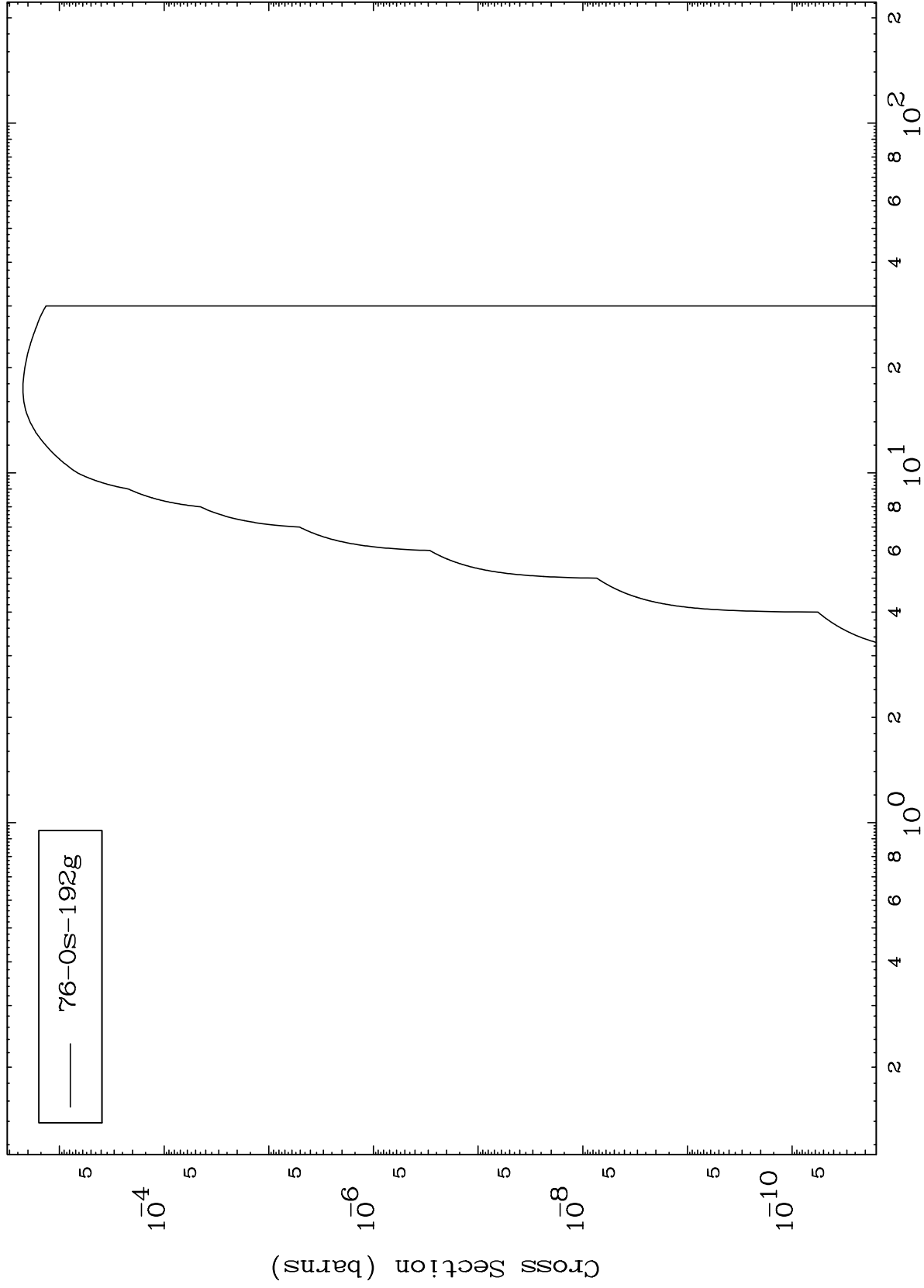
Incident Energy (MeV)

77-Ir-194m

MAT 7735

77-Ir-194m

Radionuclide Production Cross Section

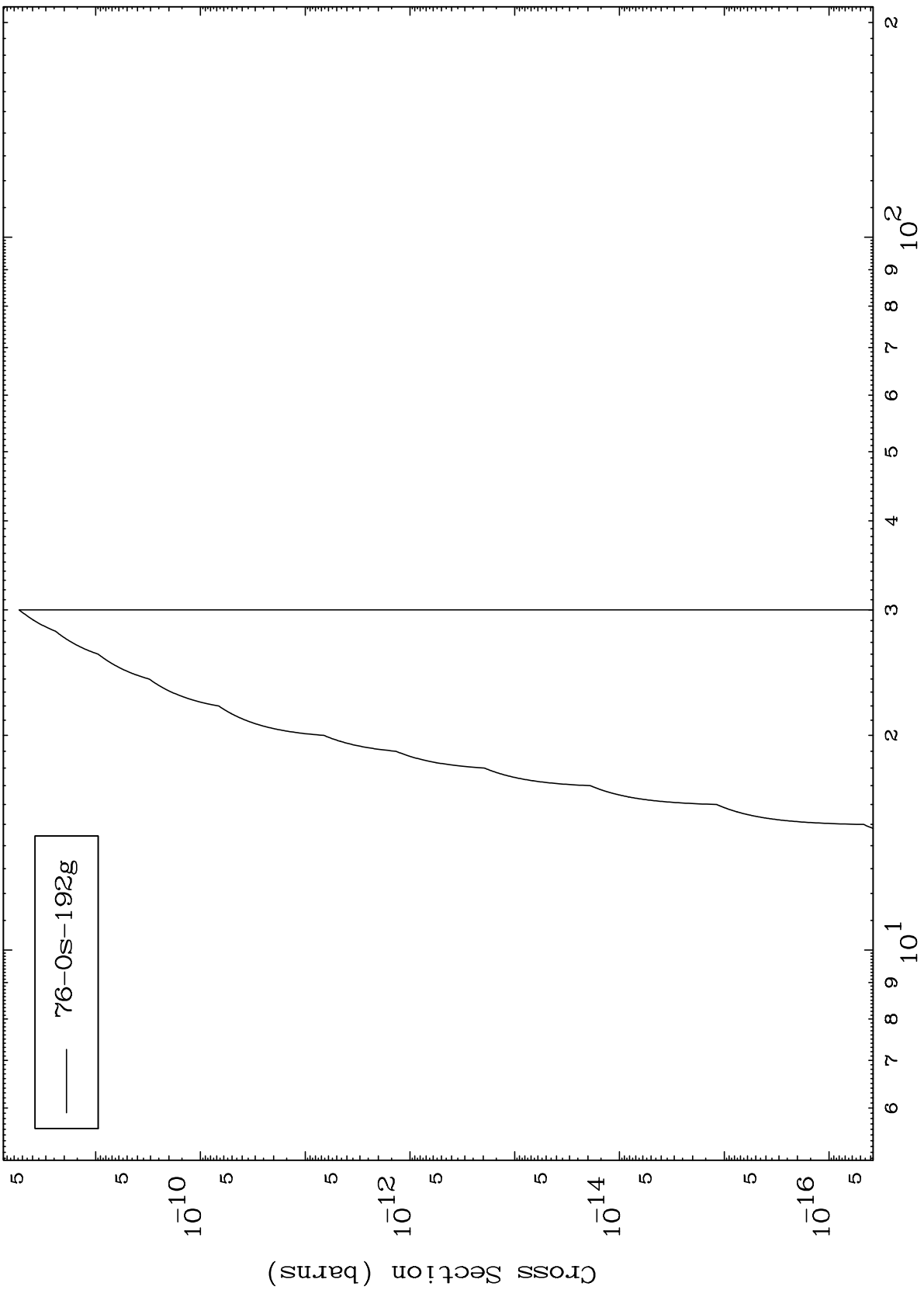


MAT 7735

(n,p) t

<sup>77</sup>Ir-194m

Radionuclide Production Cross Section



28

Incident Energy (MeV)

<sup>77</sup>Ir-194m

MAT 7735

(n,d)  $\alpha$

77-Ir-194m

Radionuclide Production Cross Section

