

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

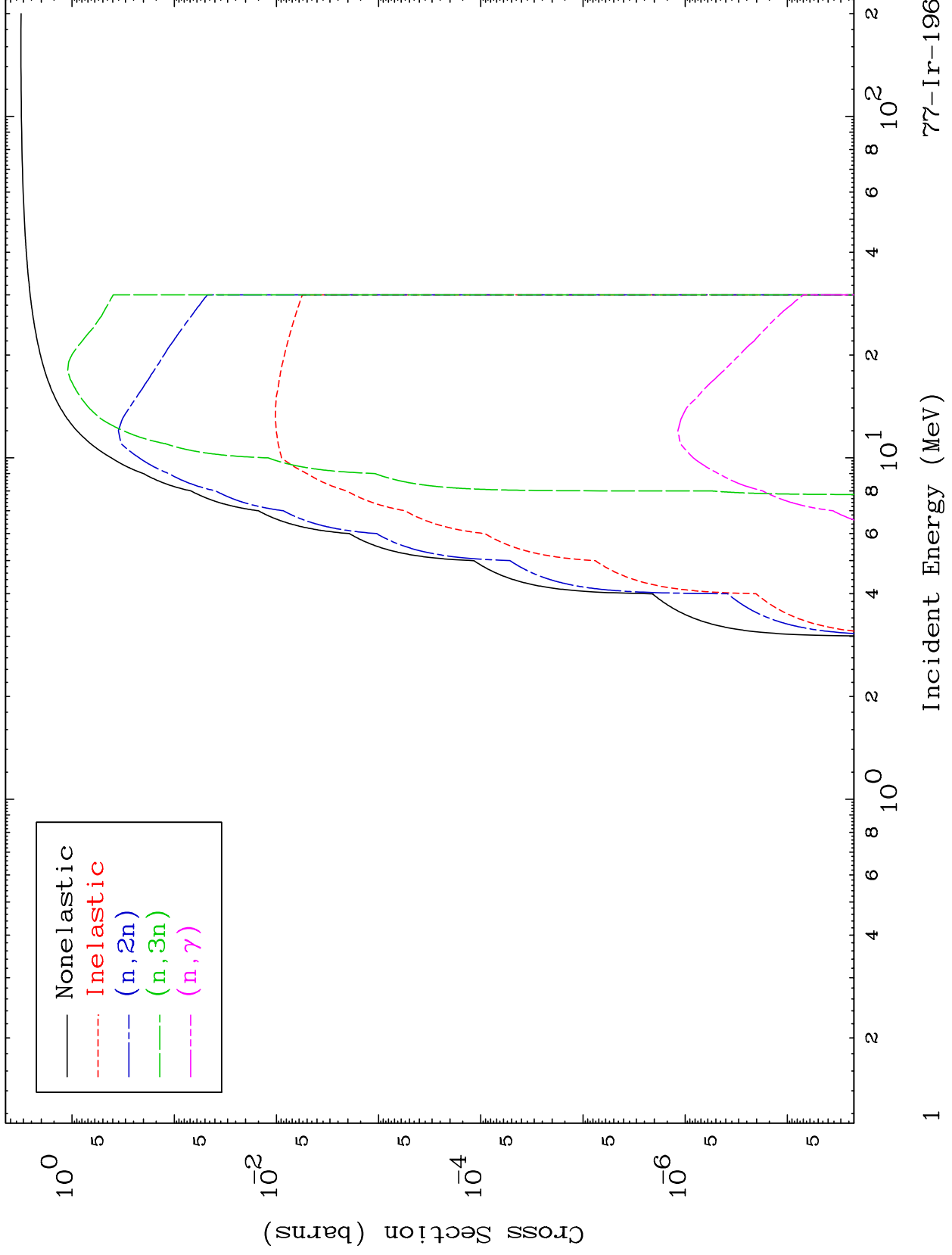
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7740

Deuteron Major  
0 Kelvin Cross Sections

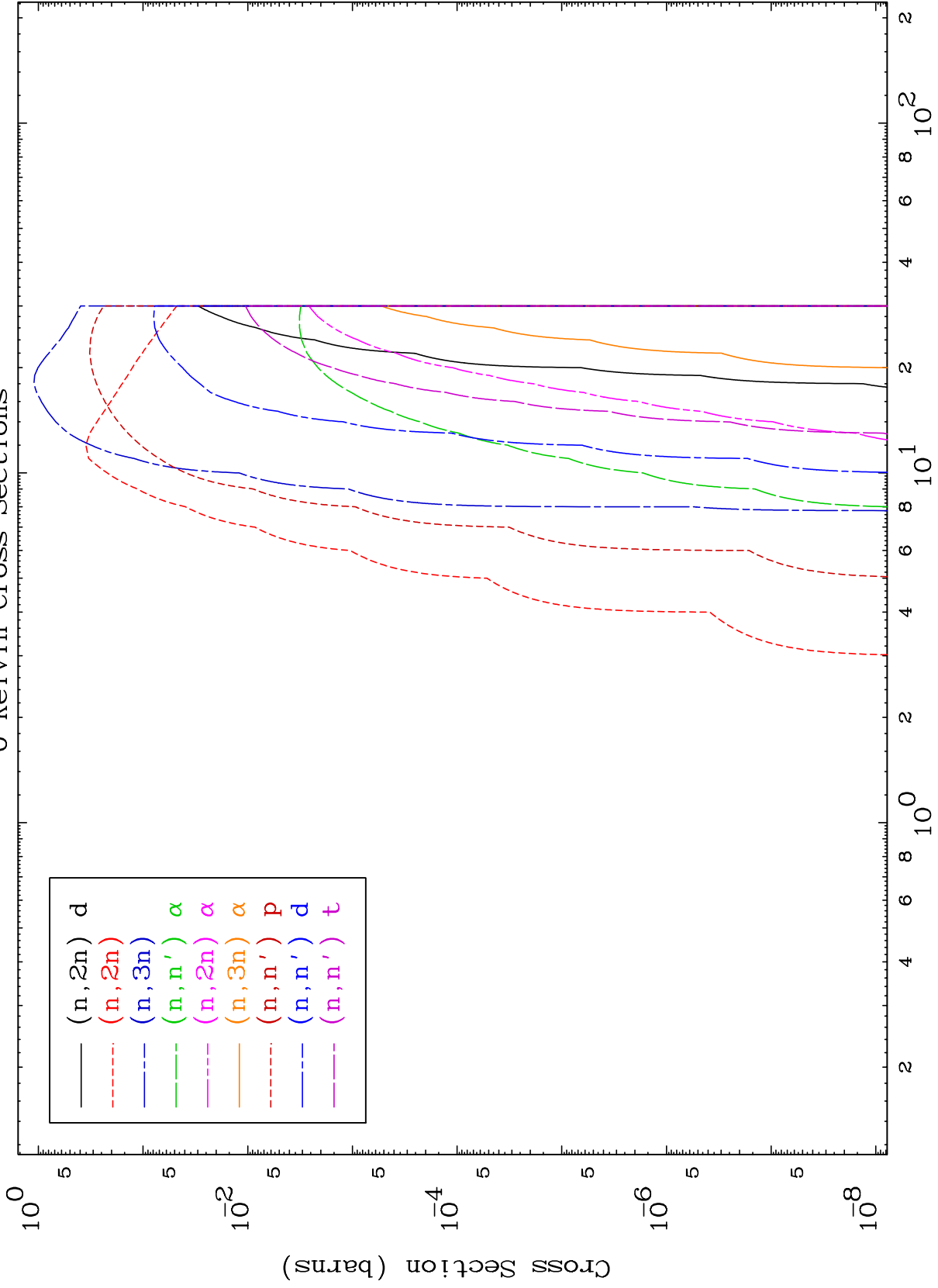
77-Ir-196

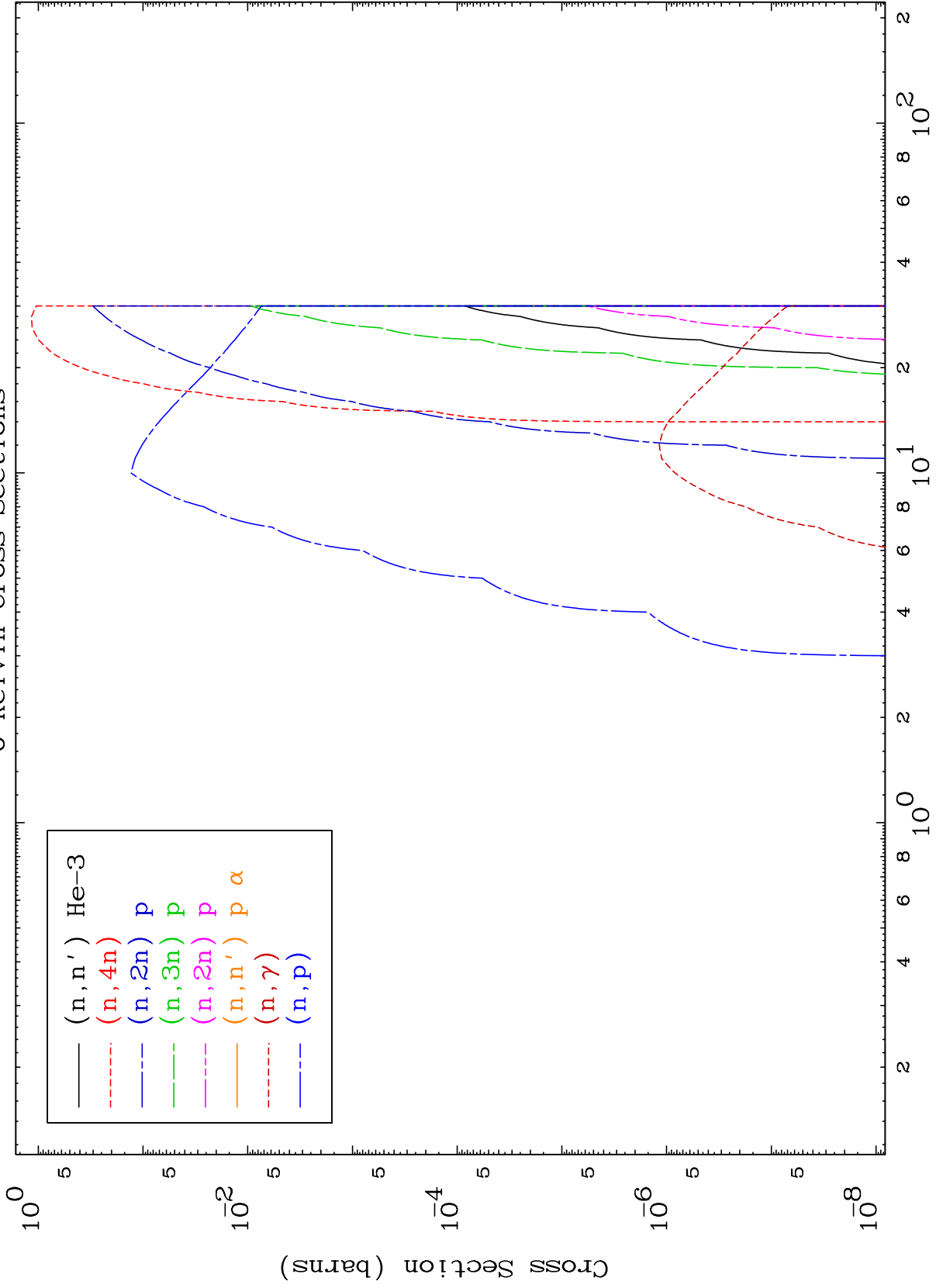


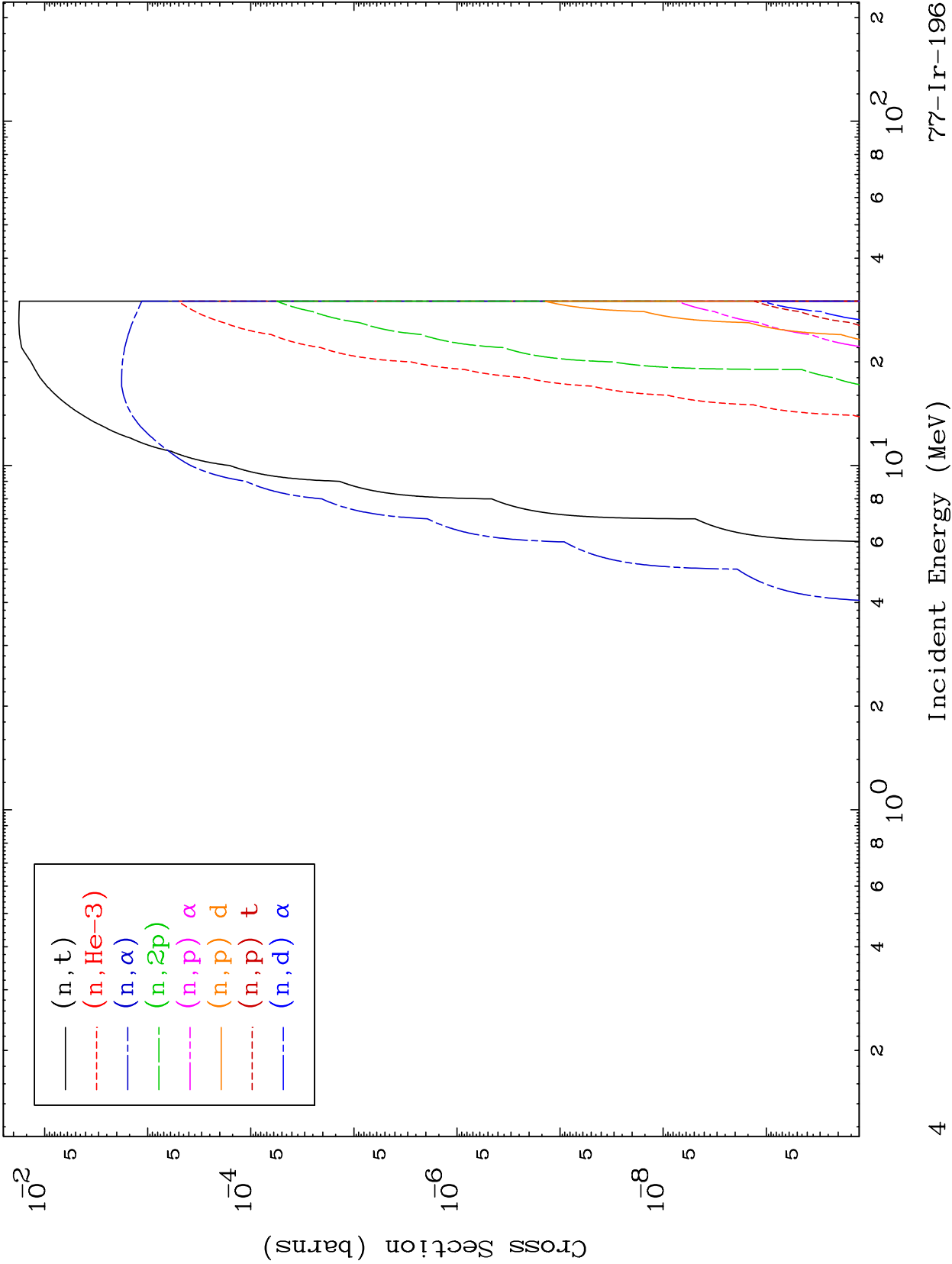
MAT 7740

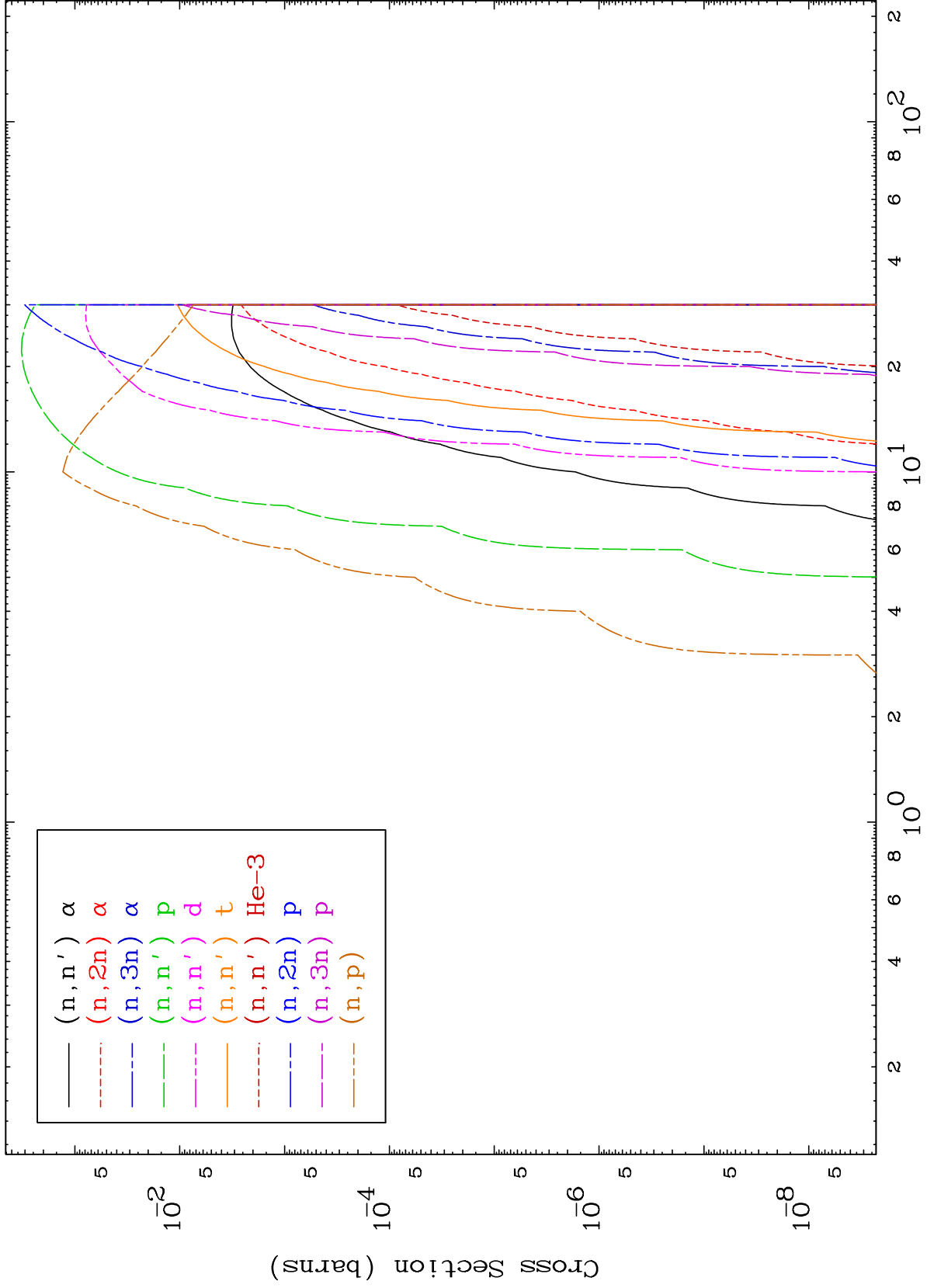
Deuteron Neutron Absorption  
0 Kelvin Cross Sections

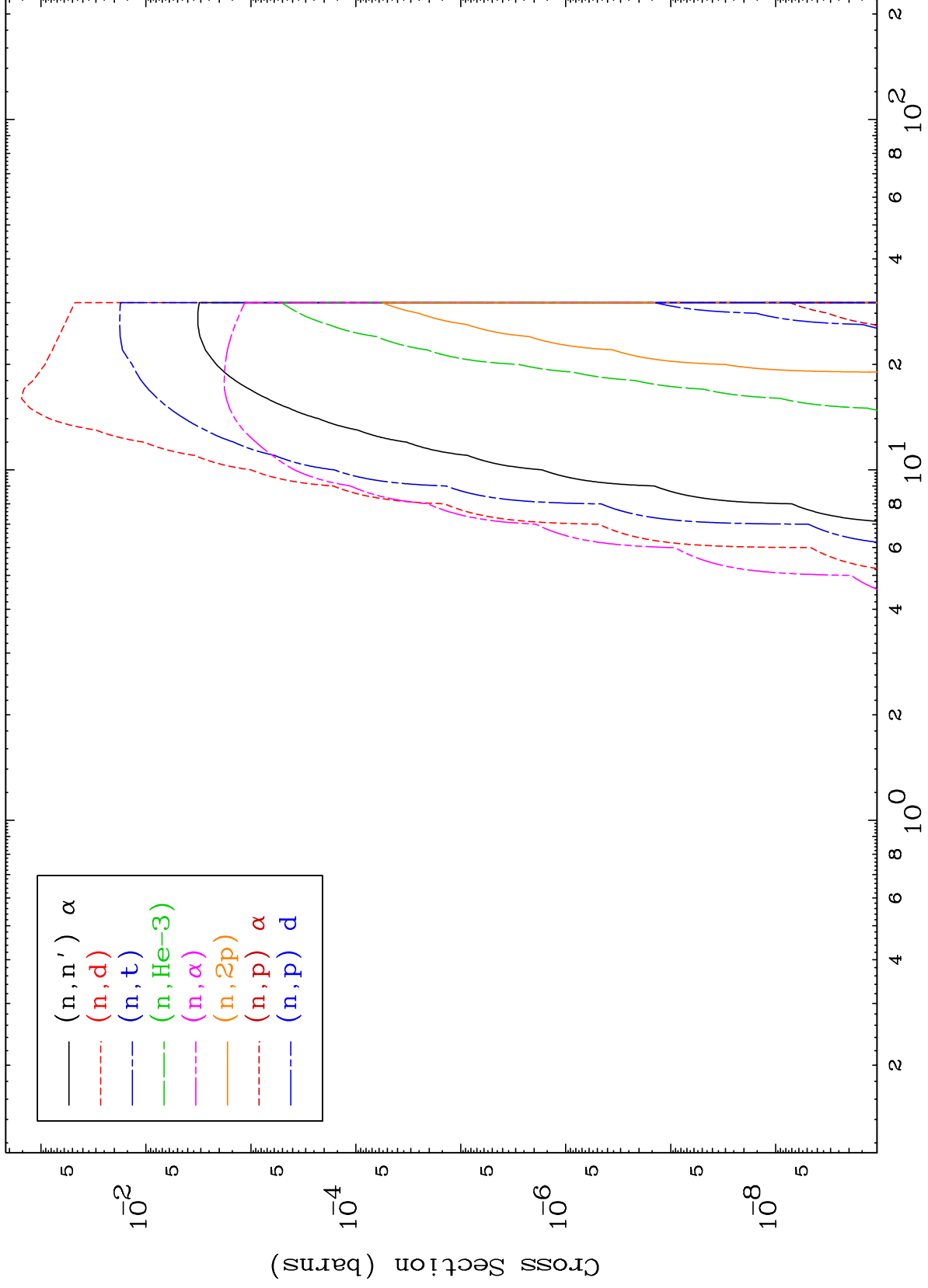
77-Ir-196







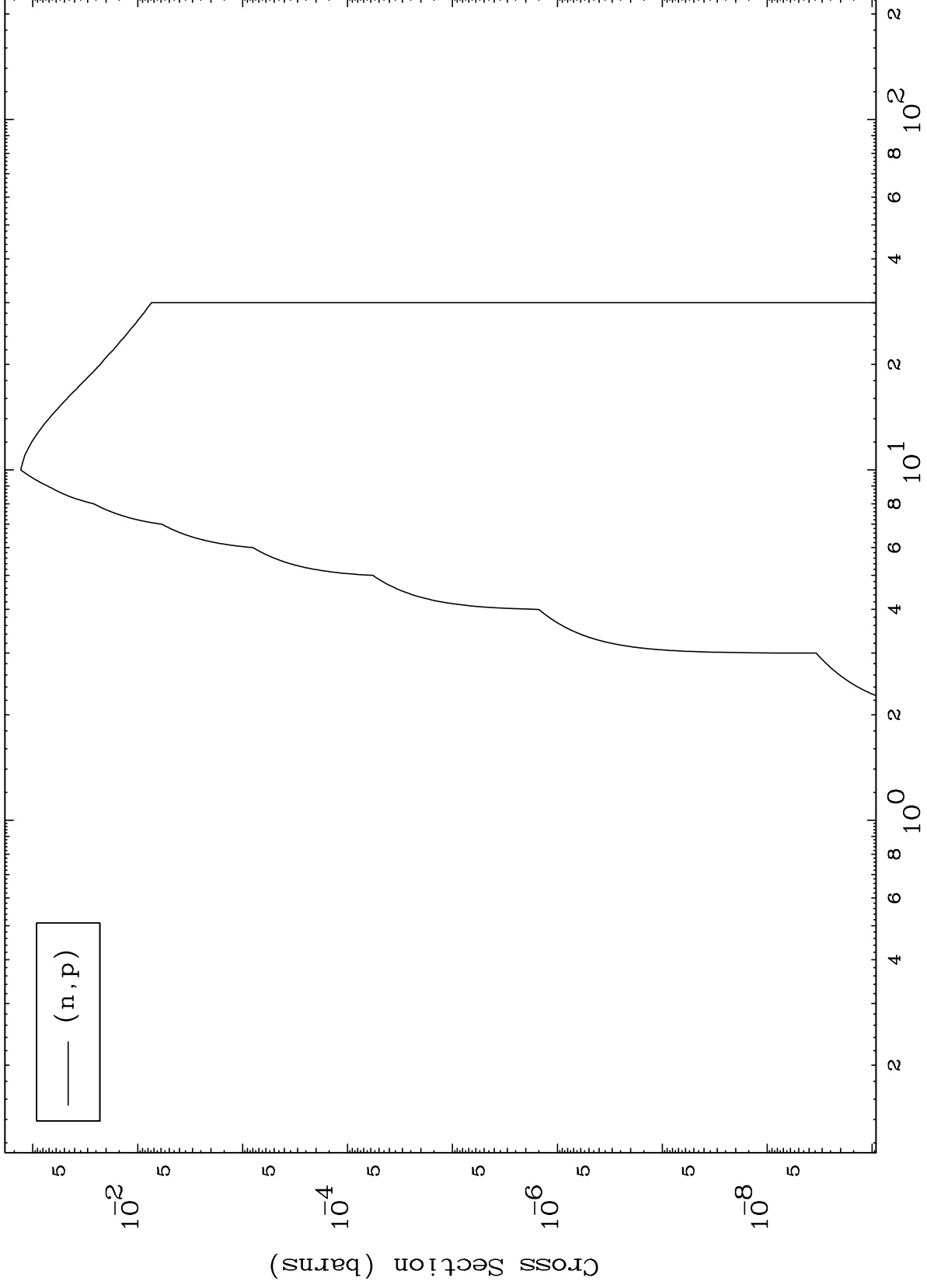




MAT 7740

(d,p) Levels  
0 Kelvin Cross Sections

77-Ir-196



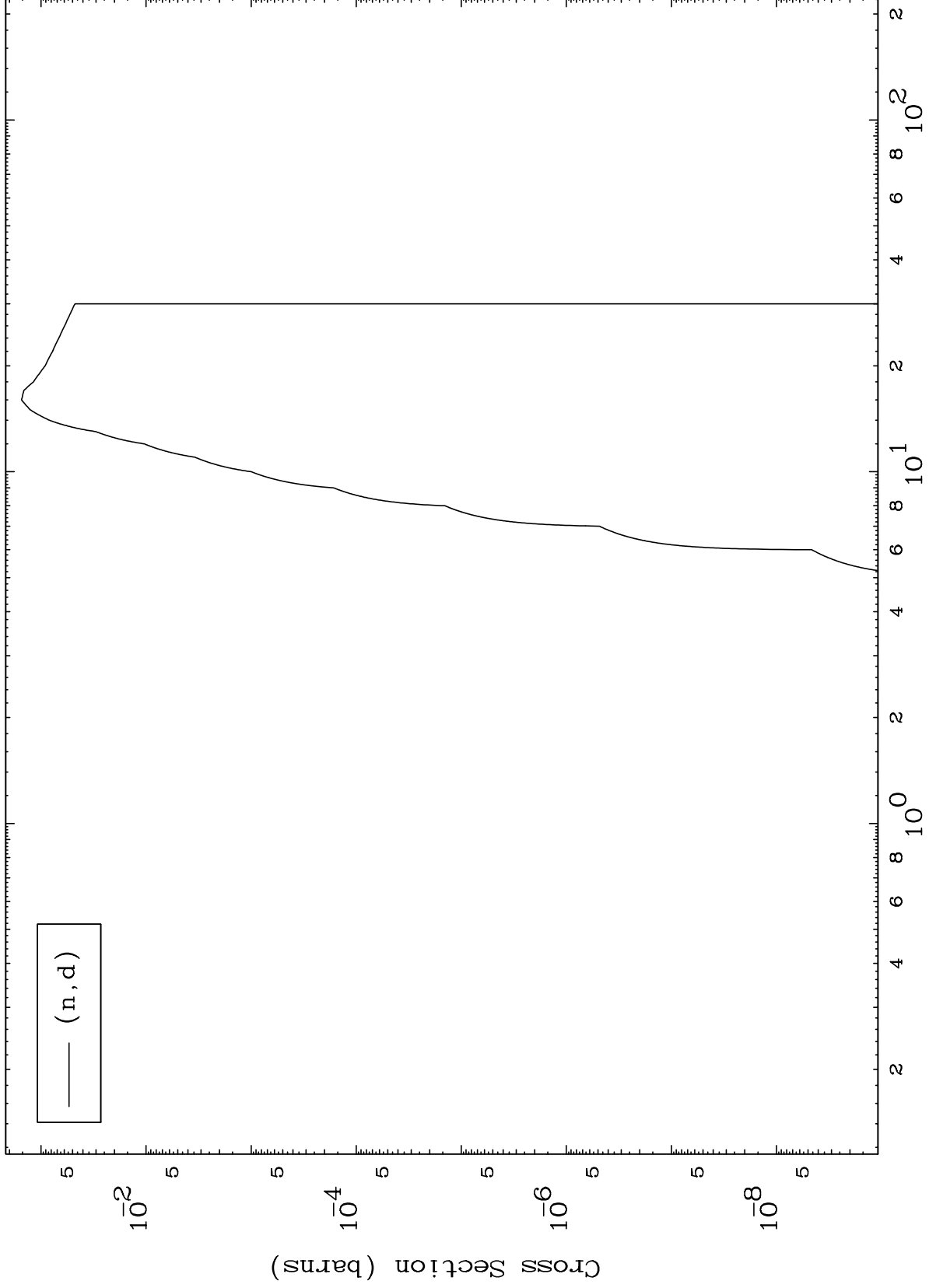


MAT 7740

(d,d) Levels

77-Ir-196

0 Kelvin Cross Sections

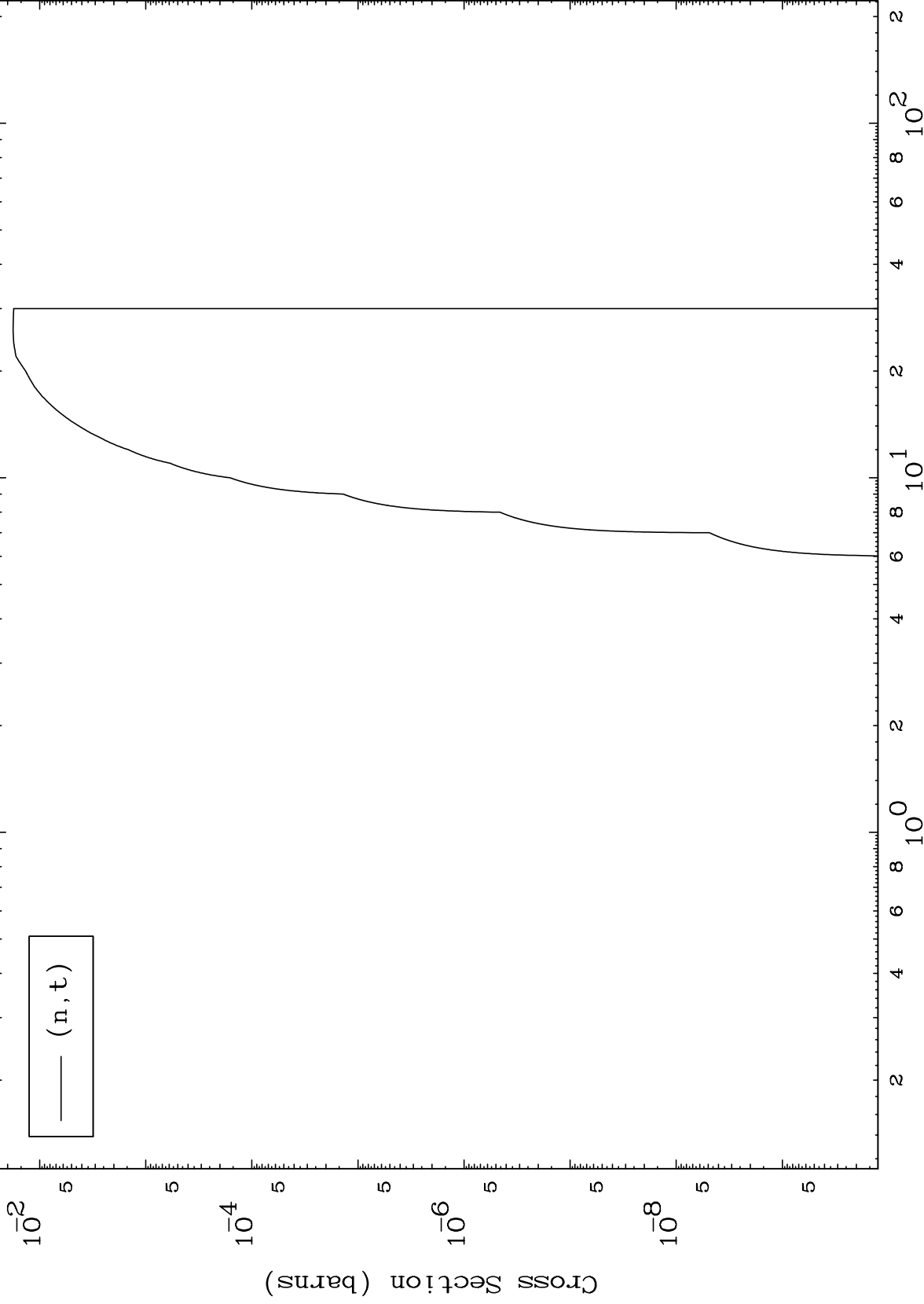


MAT 7740

(d, t) Levels

77-Ir-196

0 Kelvin Cross Sections

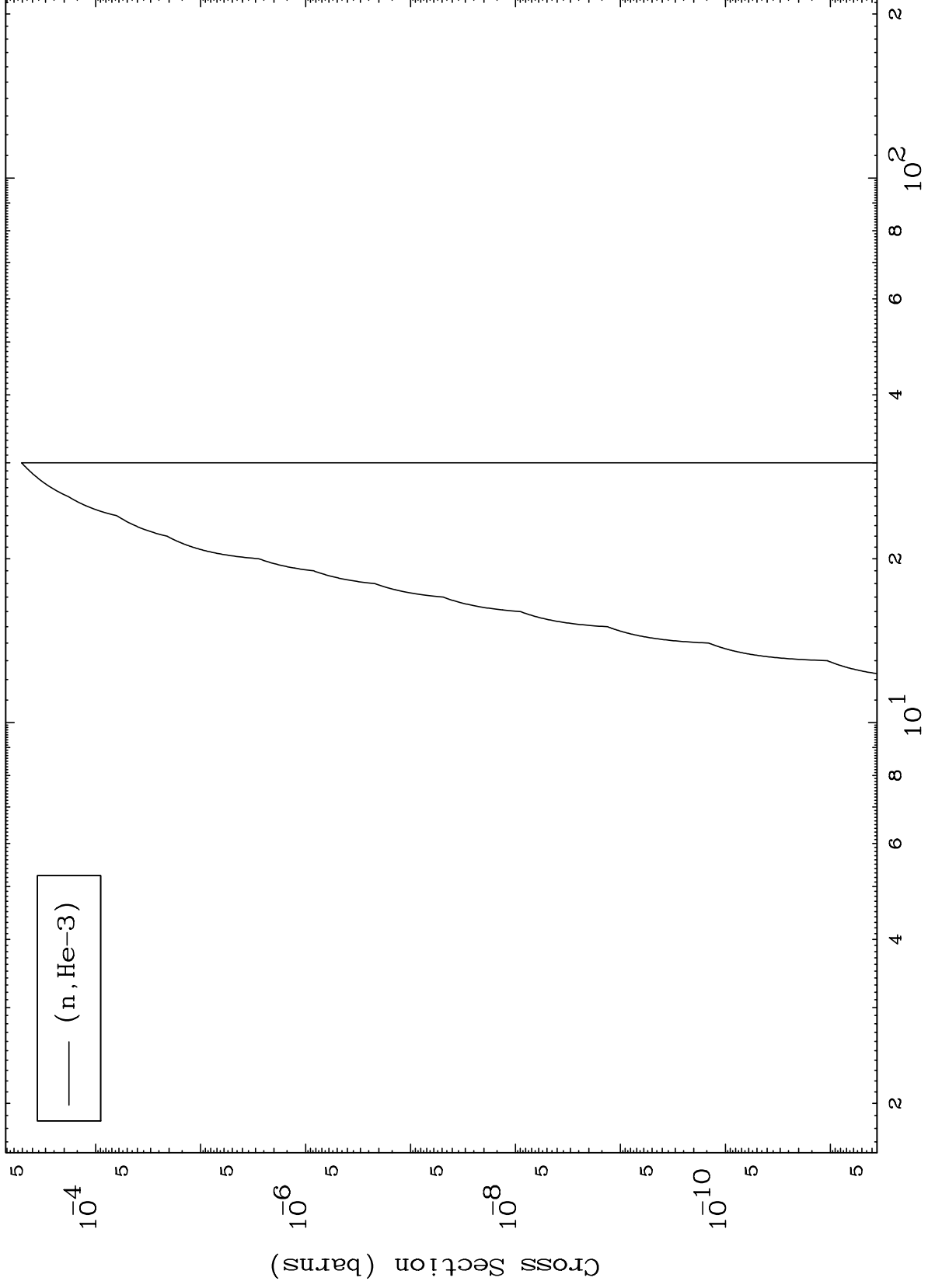


MAT 7740

(d,He3) Levels

77-Ir-196

0 Kelvin Cross Sections



10

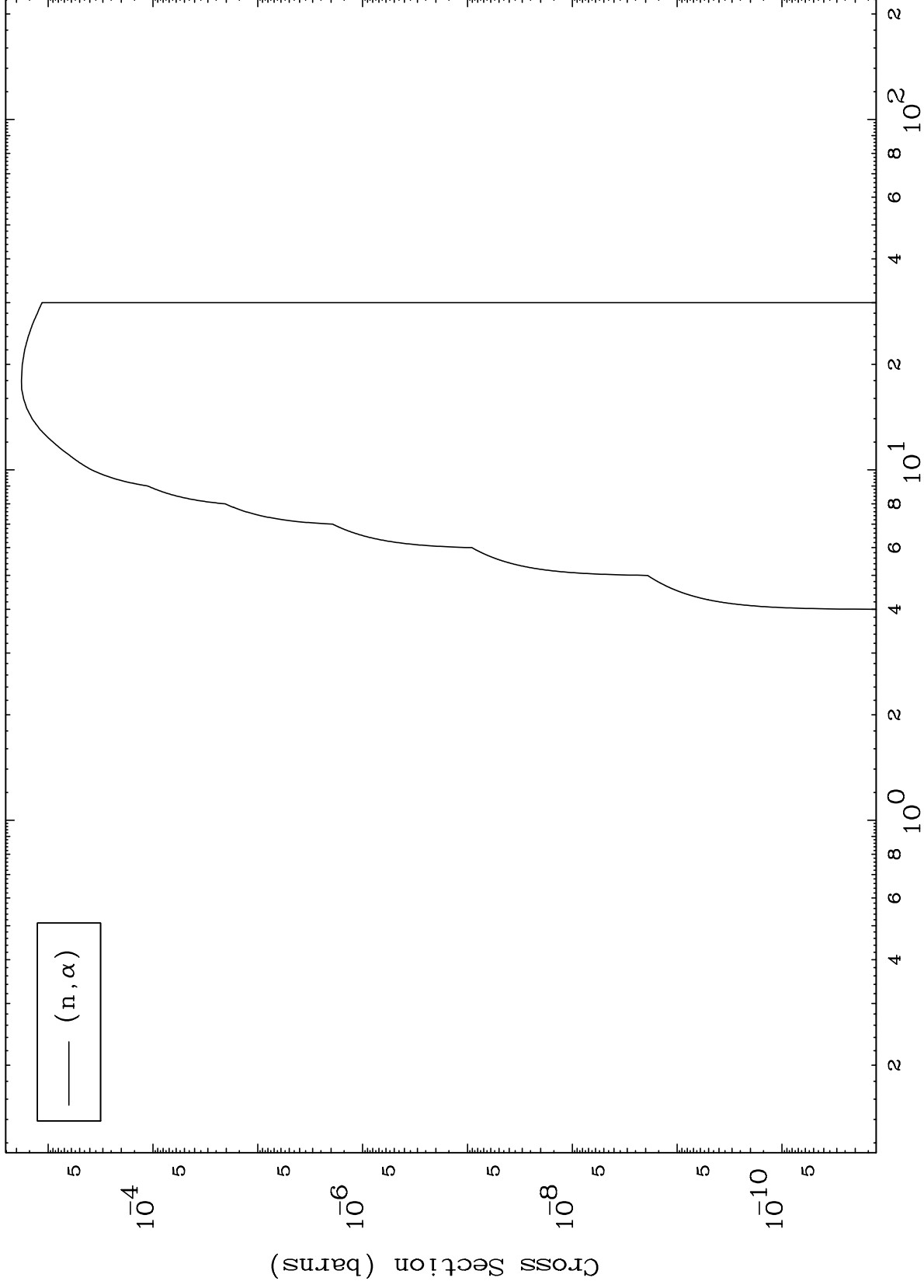
Incident Energy (MeV)

77-Ir-196

MAT 7740

77-Ir-196

(d,  $\alpha$ ) Levels  
0 Kelvin Cross Sections



77-Ir-196

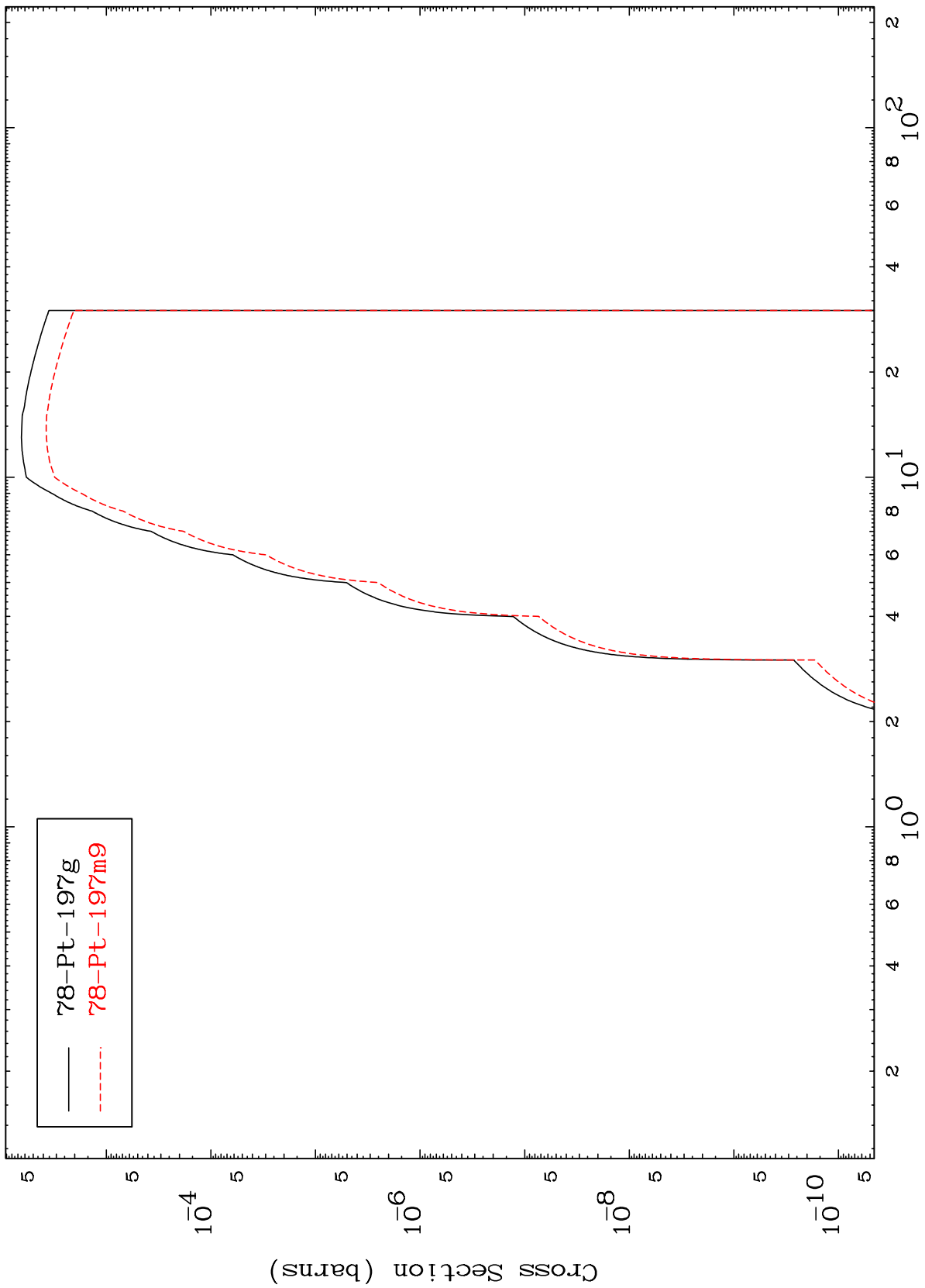
Incident Energy (MeV)

11

MAT 7740

77-Ir-196

Inelastic  
Radionuclide Production Cross Section



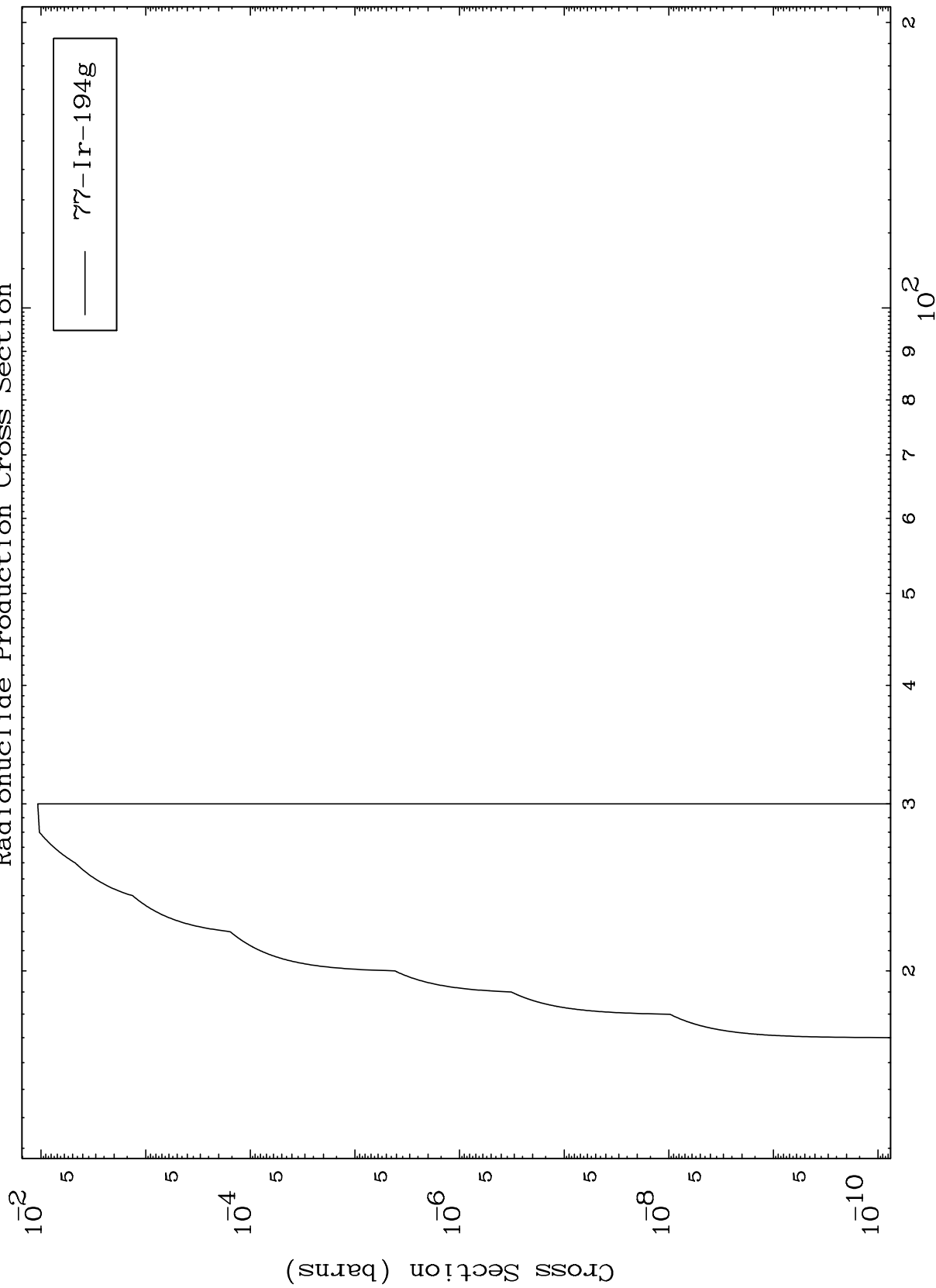
78-Pt-197g  
78-Pt-197m9

MAT 7740

(n,2n) d

77-Ir-196

Radionuclide Production Cross Section

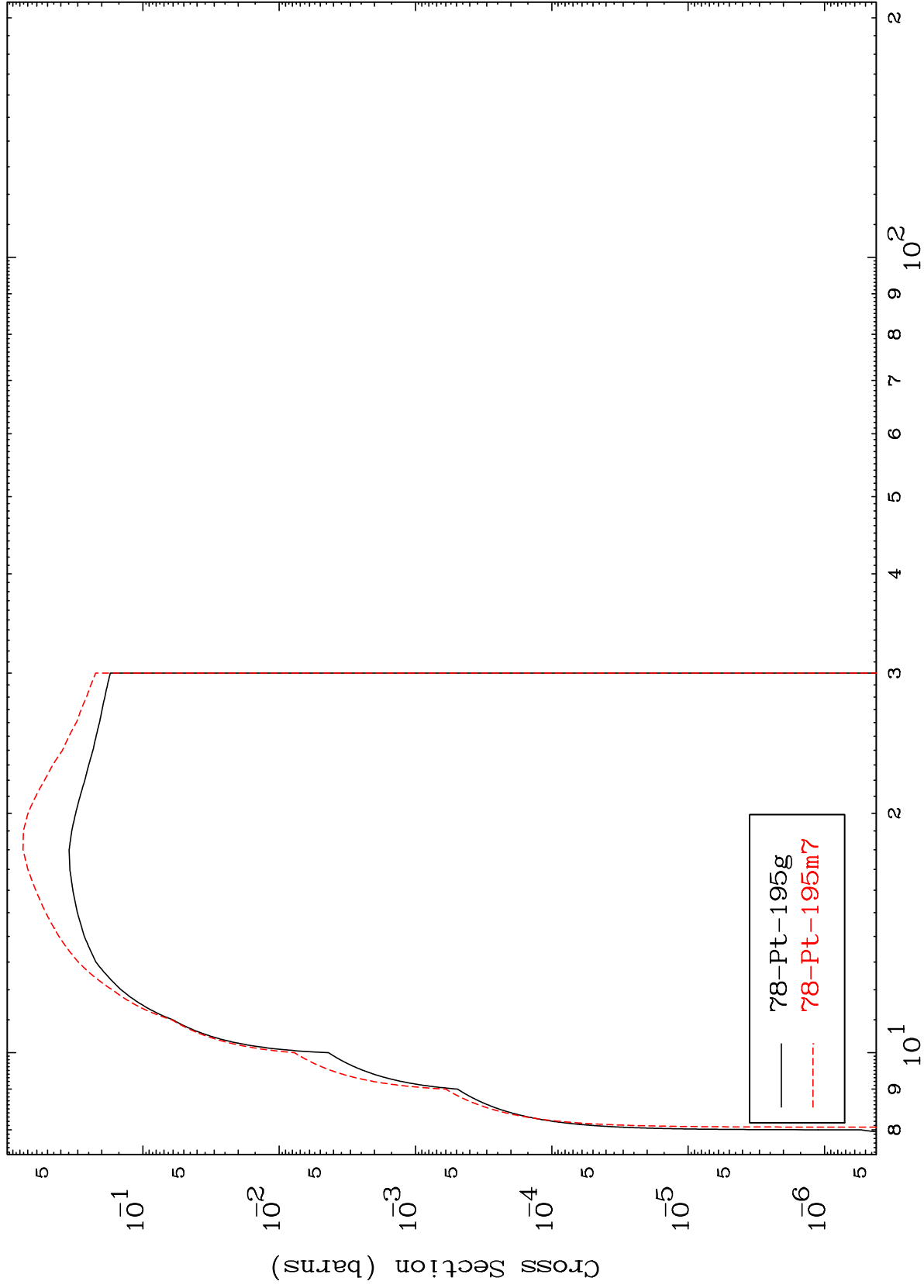


MAT 7740

(n,3n)

77-Ir-196

Radionuclide Production Cross Section



14

Incident Energy (MeV)

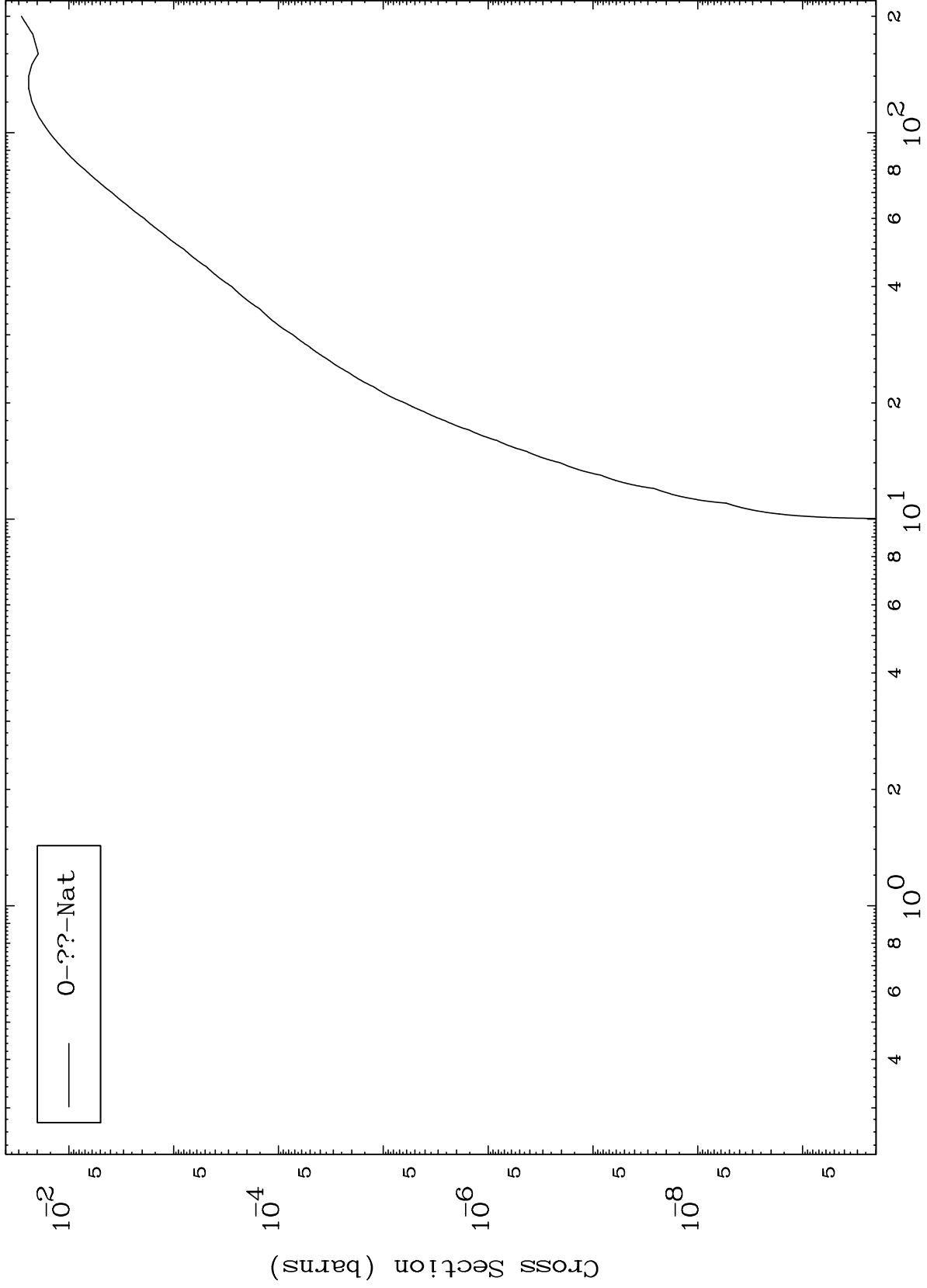
77-Ir-196

MAT 7740

Fission

77-Ir-196

Radionuclide Production Cross Section



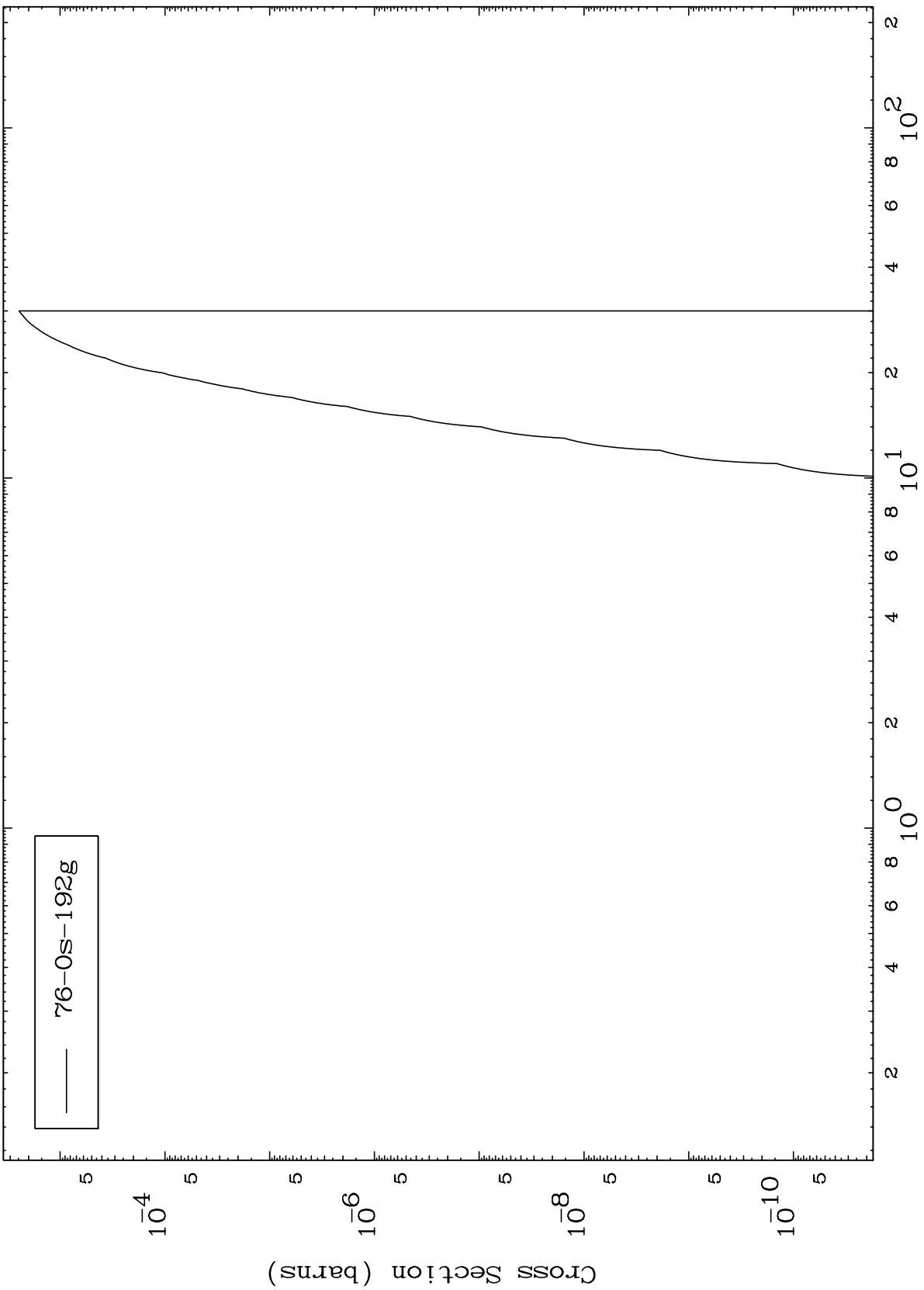


MAT 7740

(n,2n)  $\alpha$

77-Ir-196

Radionuclide Production Cross Section



16

Incident Energy (MeV)

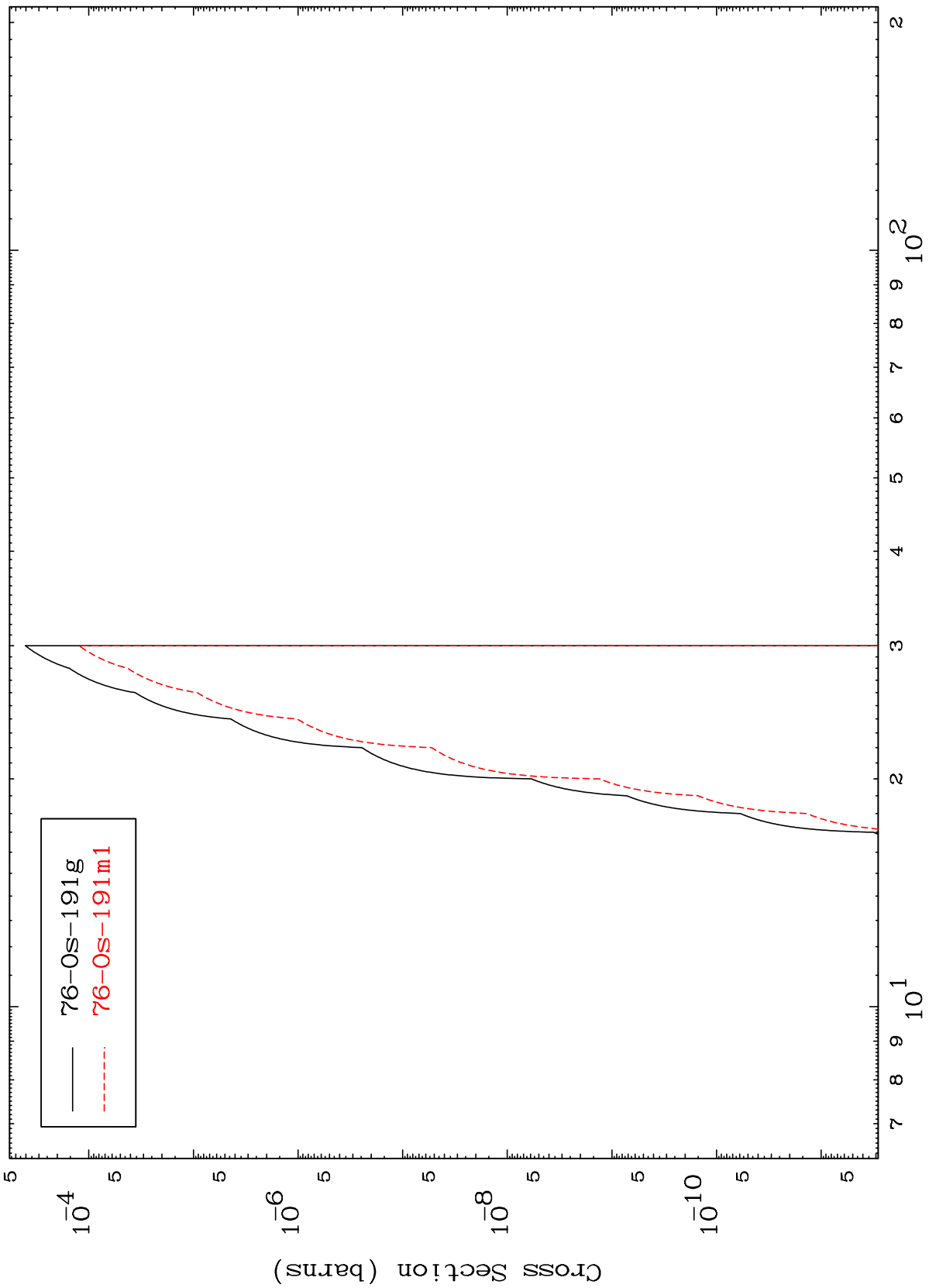
77-Ir-196

MAT 7740

(n,3n)  $\alpha$

77-Ir-196

Radionuclide Production Cross Section



17

Incident Energy (MeV)

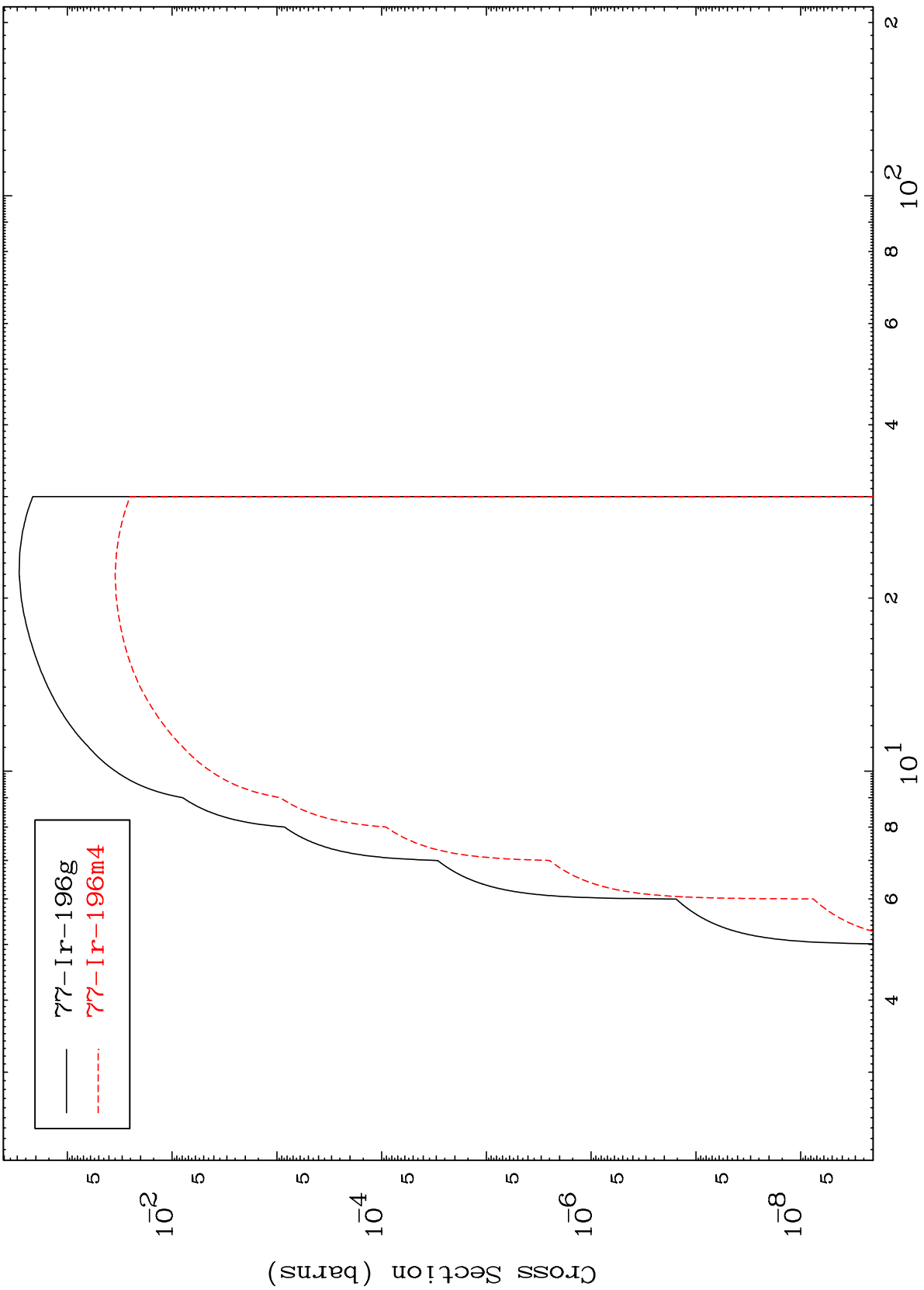
77-Ir-196

MAT 7740

(n,n') p

77-Ir-196

Radionuclide Production Cross Section



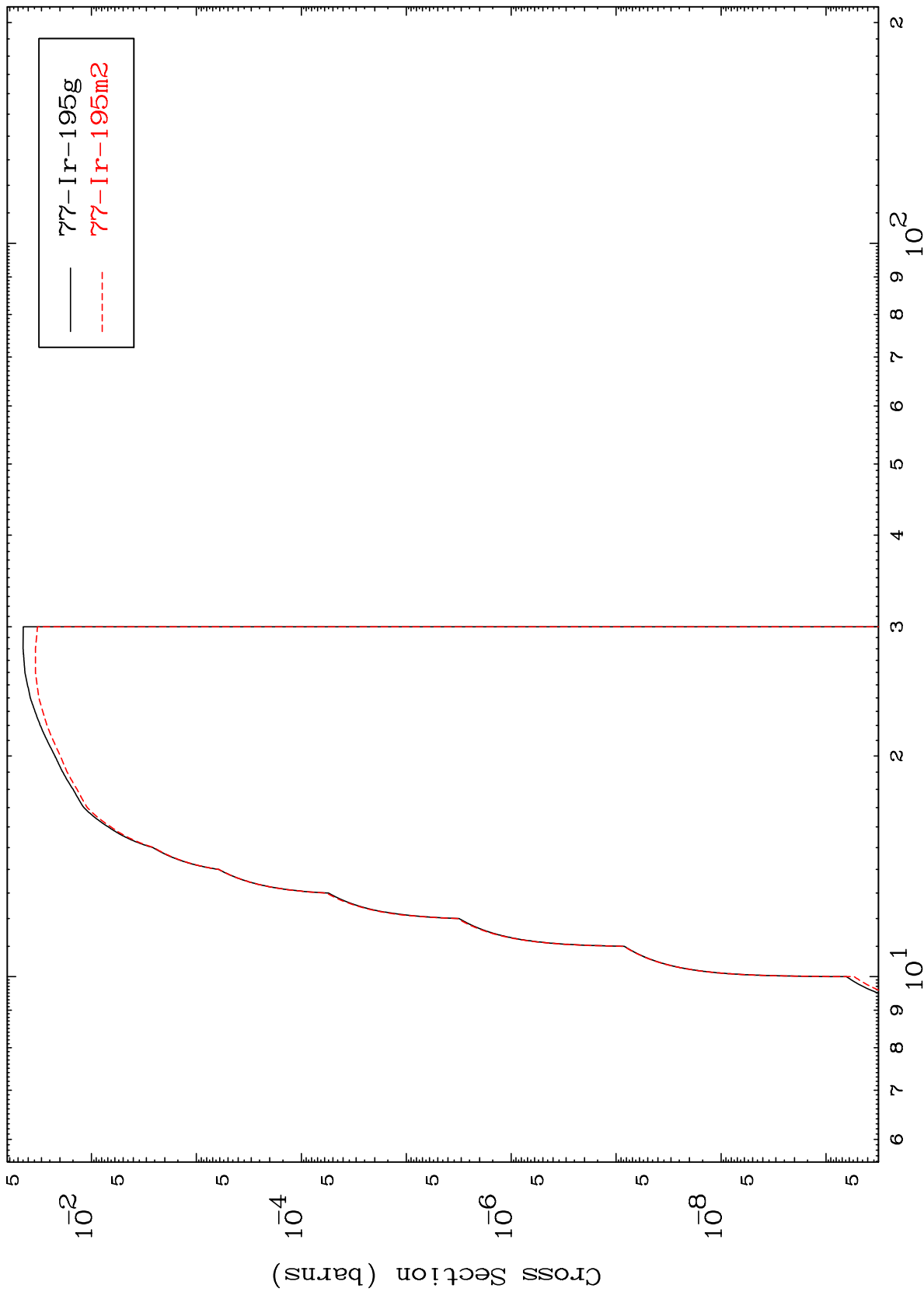
Legend:  
—  $^{77}\text{Ir-196g}$   
- - -  $^{77}\text{Ir-196m4}$

MAT 7740

(n,n') d

77-Ir-196

Radionuclide Production Cross Section



19

Incident Energy (MeV)

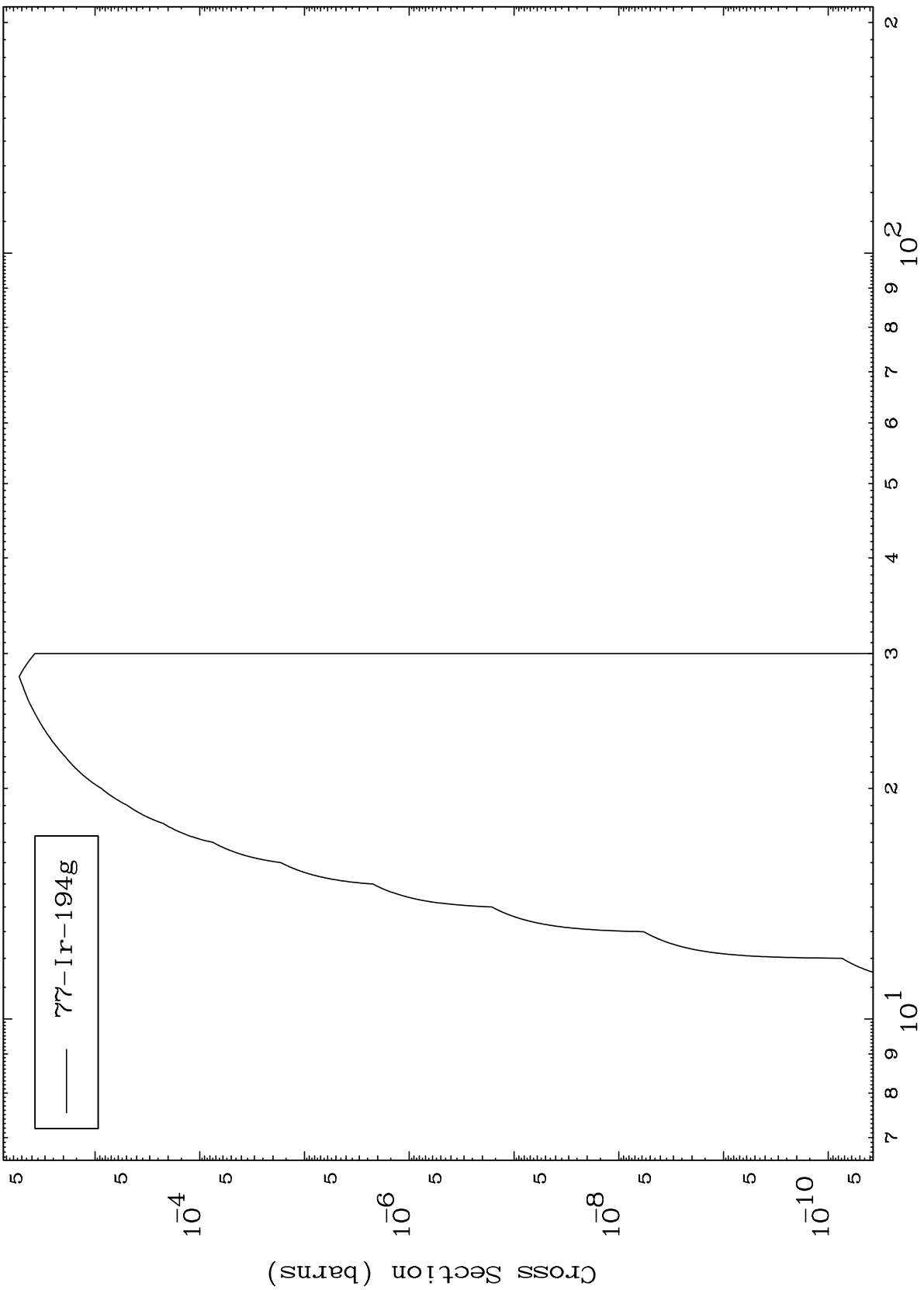
77-Ir-196

MAT 7740

(n,n') t

77-Ir-196

Radionuclide Production Cross Section



77-Ir-194g

20

Incident Energy (MeV)

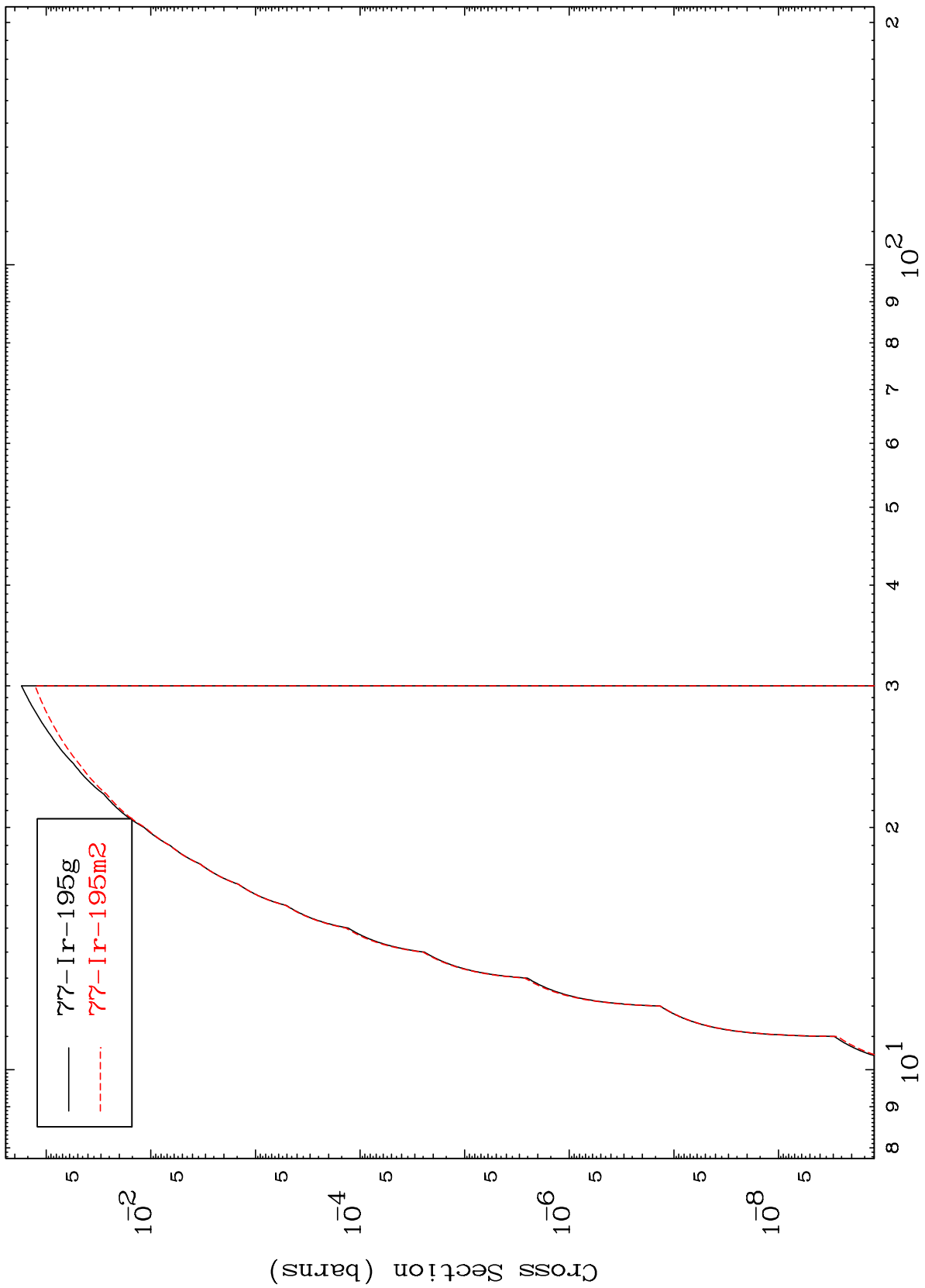
77-Ir-196

MAT 7740

(n,2n) p

<sup>77</sup>Ir-196

Radionuclide Production Cross Section

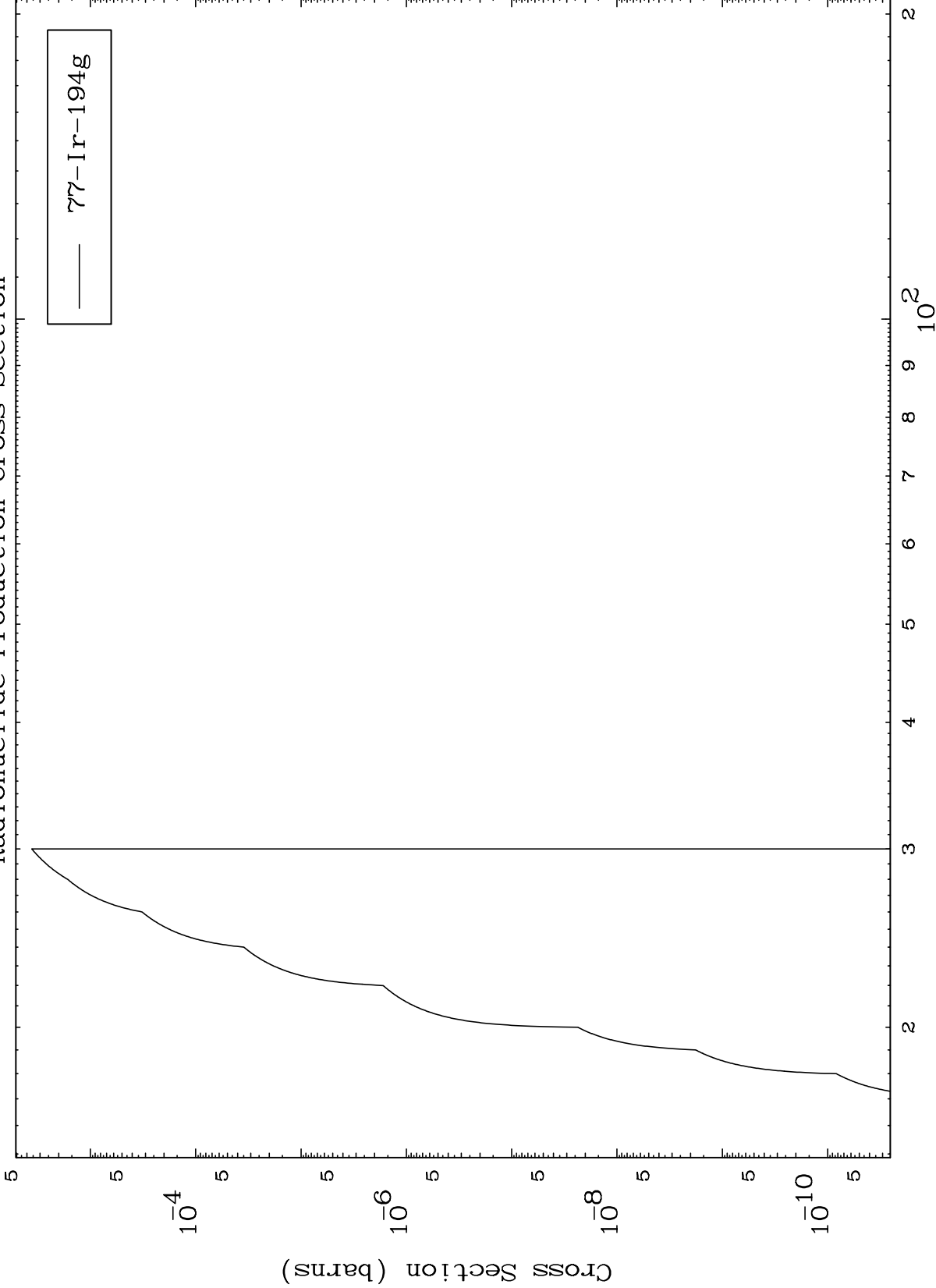


MAT 7740

(n,3n) p

77-Ir-196

Radionuclide Production Cross Section



22

Incident Energy (MeV)

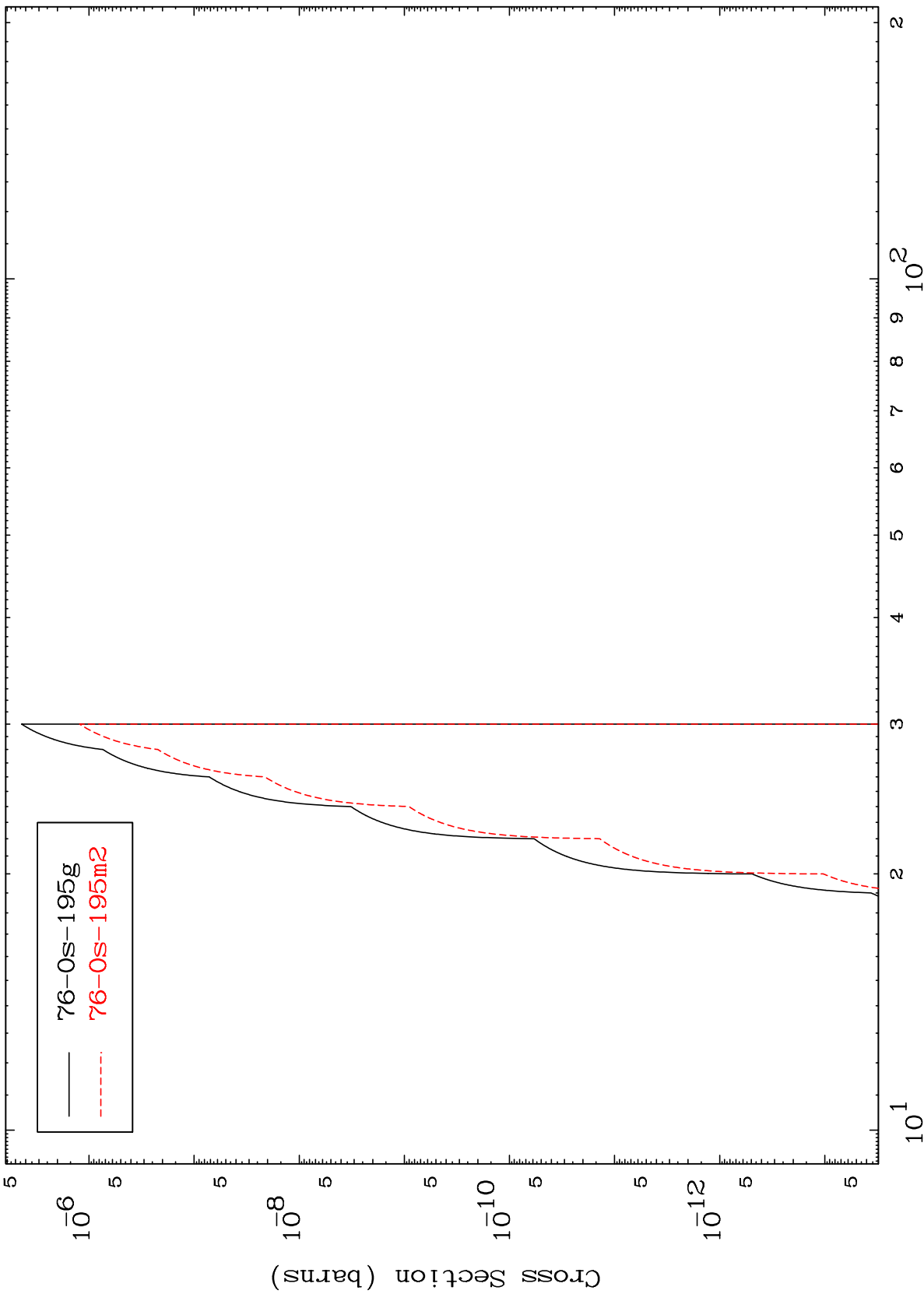
77-Ir-196

MAT 7740

(n,2n) p

<sup>77</sup>Ir-196

Radionuclide Production Cross Section



76-0s-195g  
76-0s-195m2

Incident Energy (MeV)

<sup>77</sup>Ir-196

23

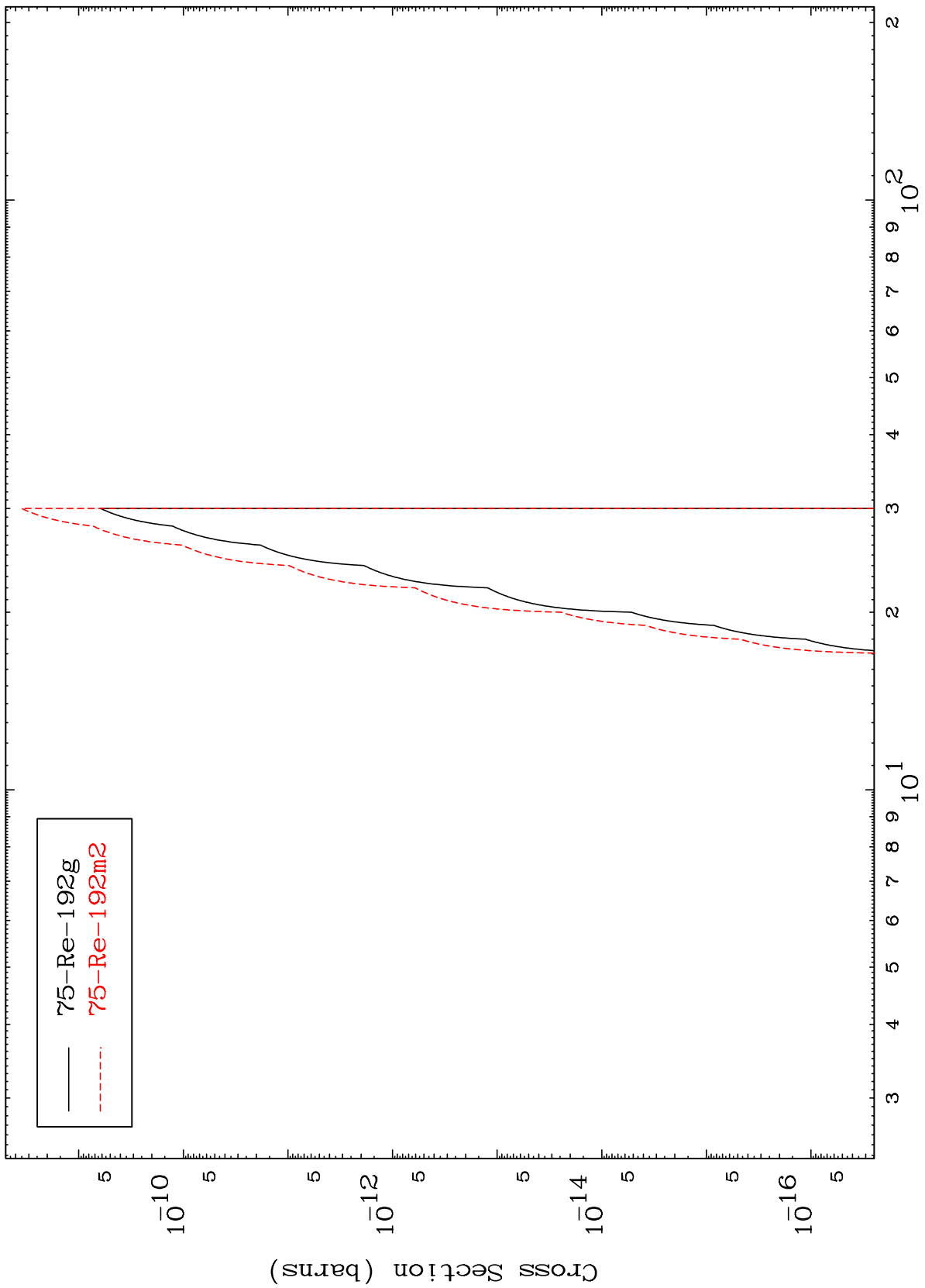


MAT 7740

(n,n') p  $\alpha$

77-Ir-196

Radionuclide Production Cross Section



24

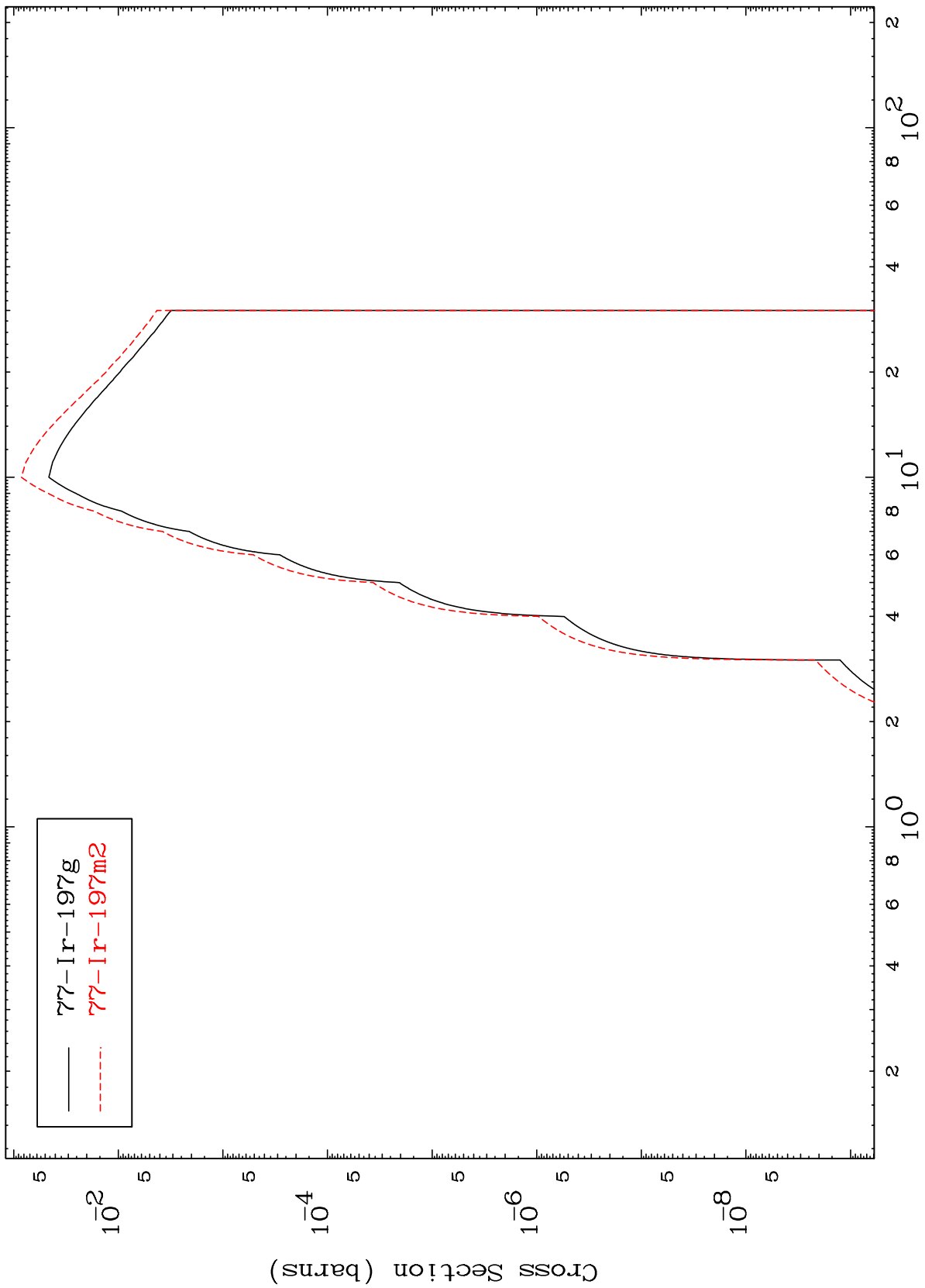
Incident Energy (MeV)

77-Ir-196

MAT 7740

77-Ir-196

(n,p)  
Radionuclide Production Cross Section

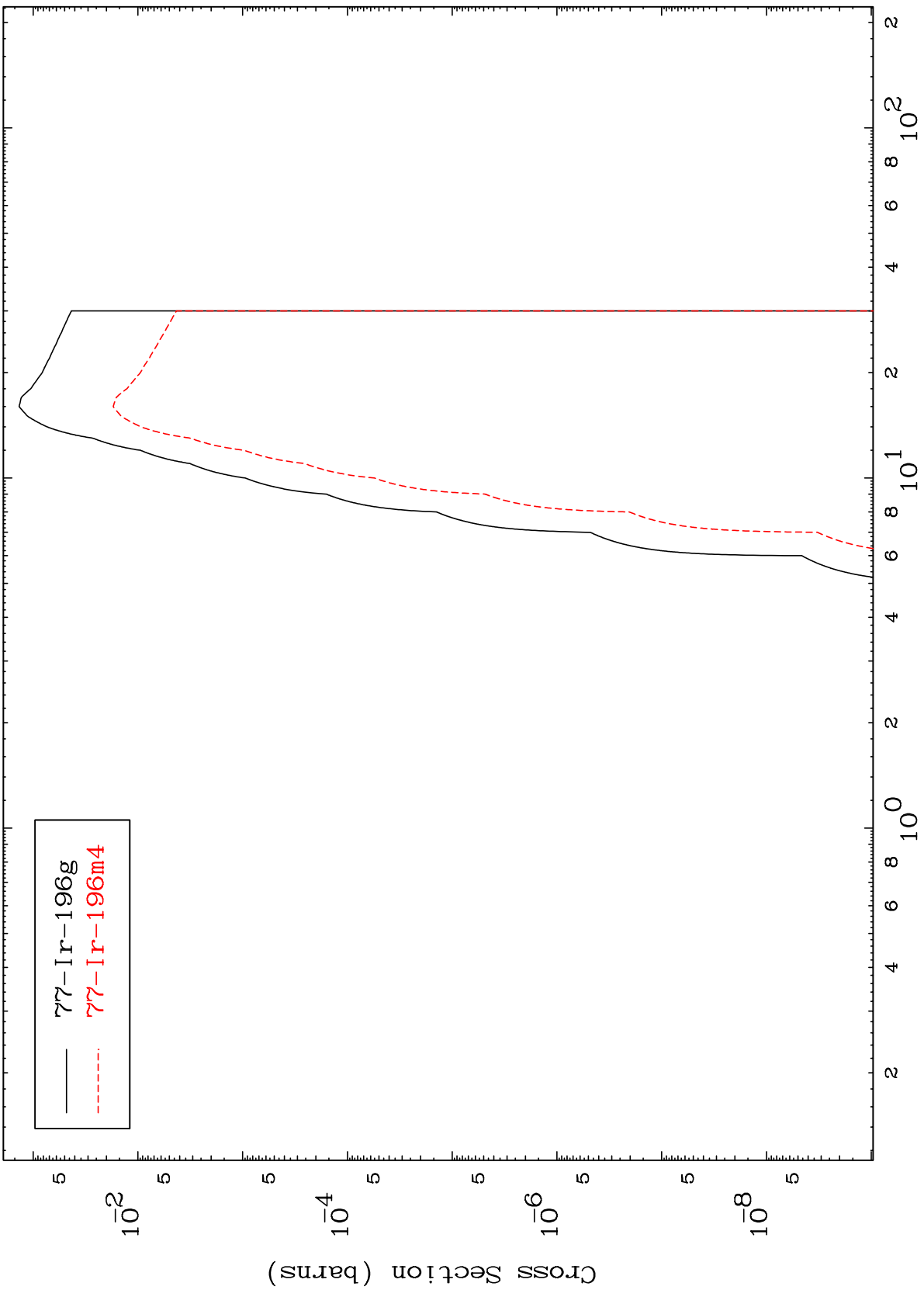


MAT 7740

(n,d)

<sup>77</sup>Ir-196

Radionuclide Production Cross Section

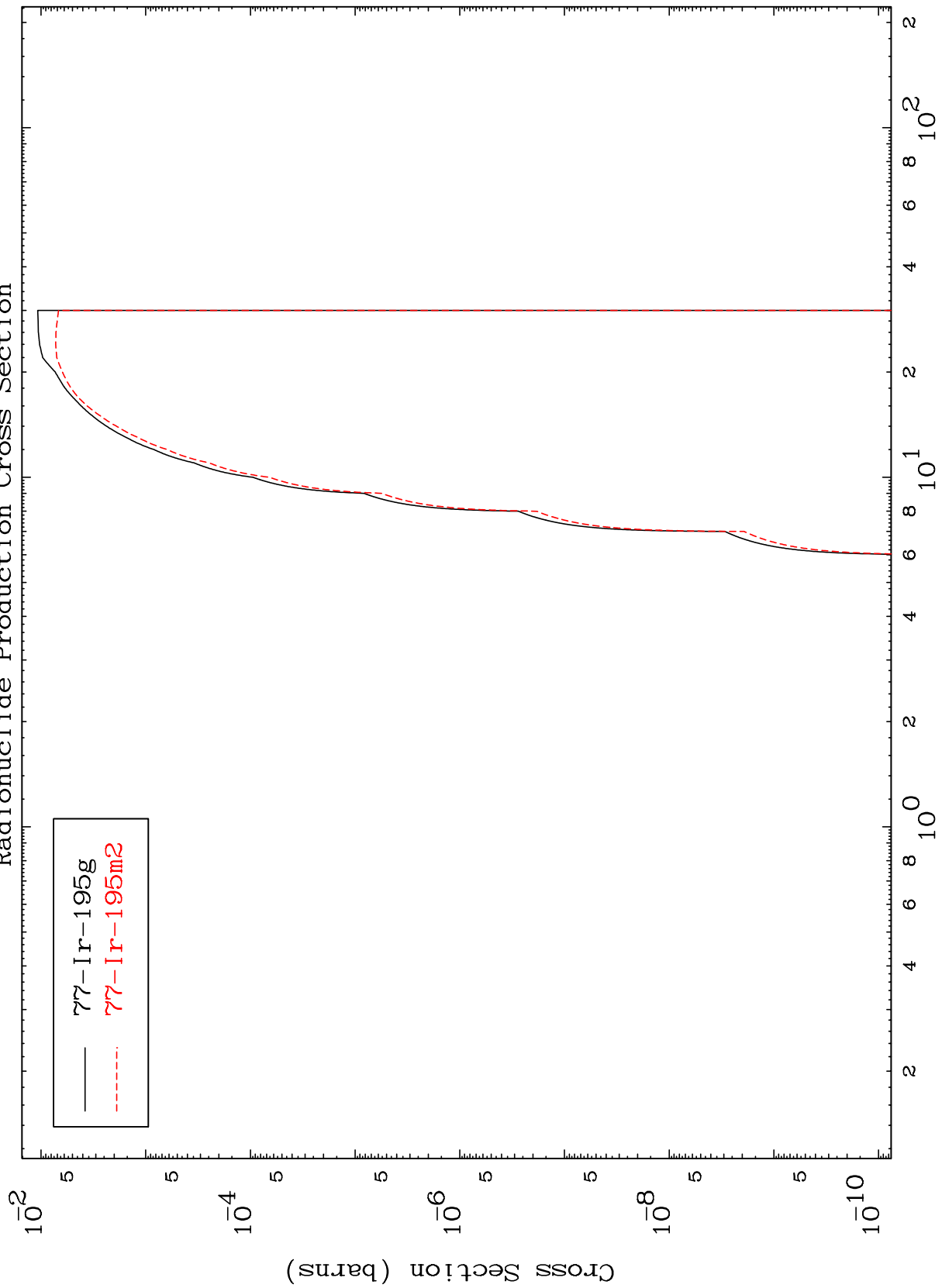


— <sup>77</sup>Ir-196g  
- - - <sup>77</sup>Ir-196m4

MAT 7740

<sup>77</sup>Ir-196

Radionuclide Production Cross Section



Incident Energy (MeV)

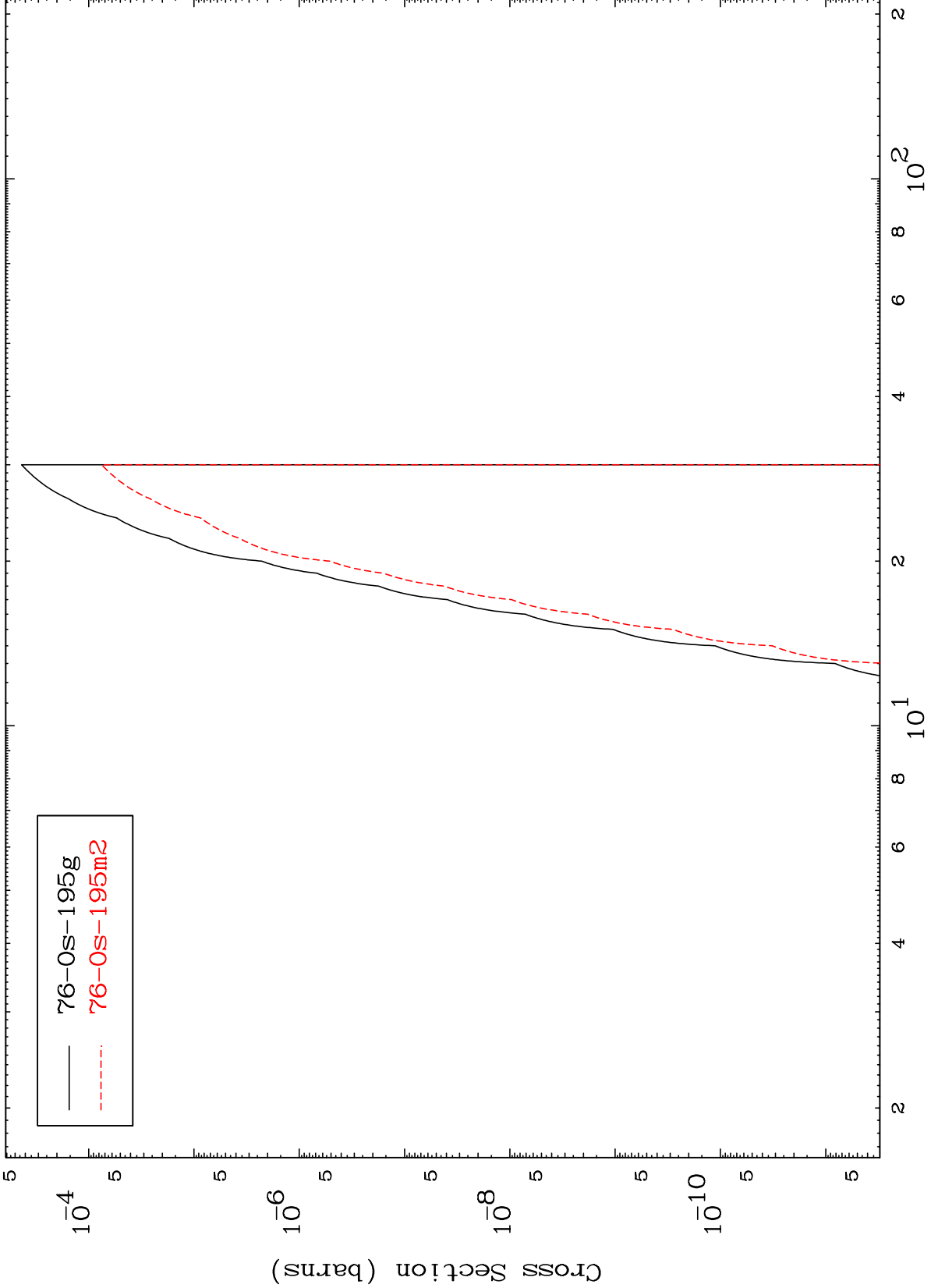
<sup>77</sup>Ir-196

MAT 7740

(n,He-3)

77-Ir-196

Radionuclide Production Cross Section



76-Os-195g  
76-Os-195m2

28

Incident Energy (MeV)

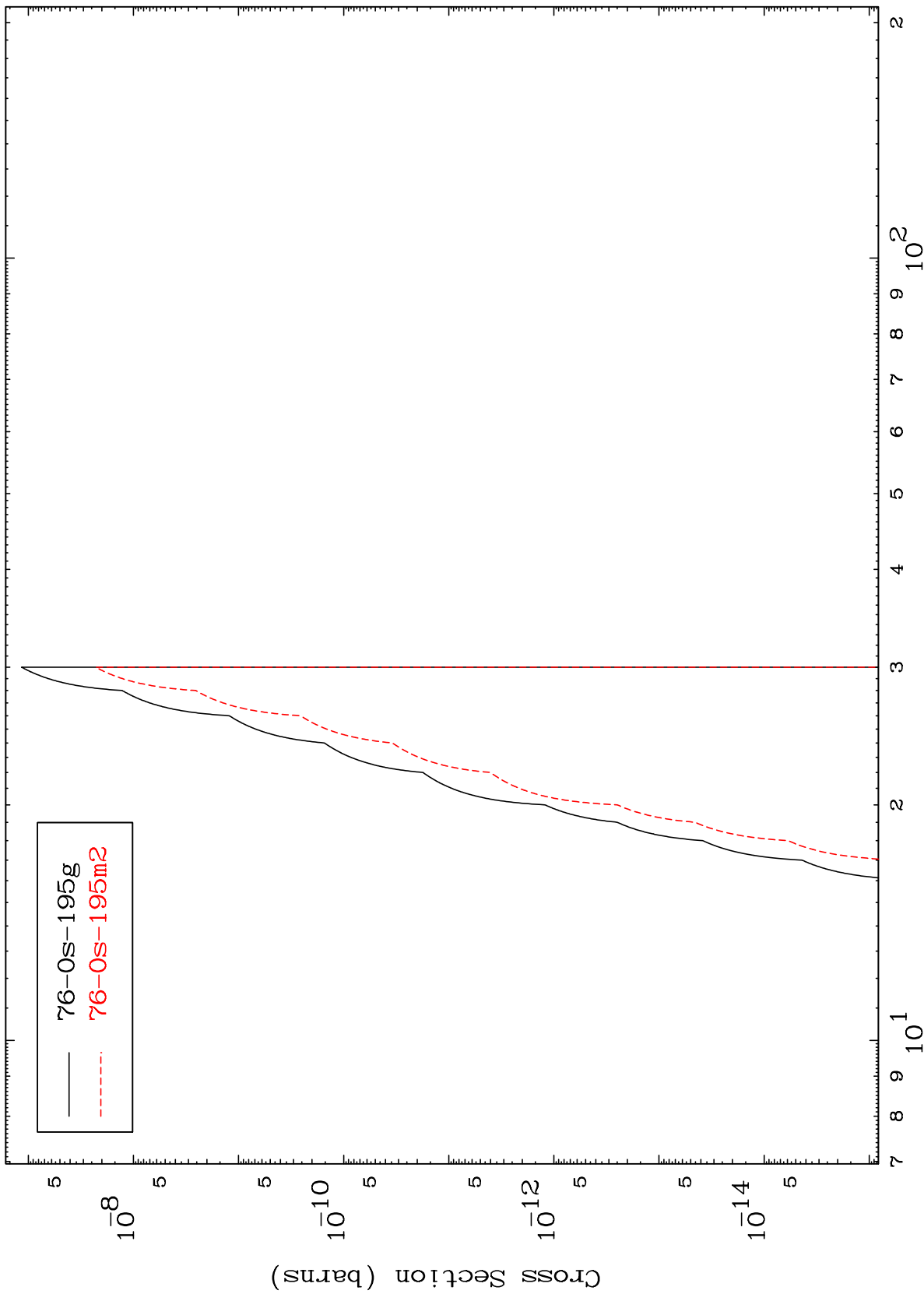
77-Ir-196

MAT 7740

(n,p) d

<sup>77</sup>Ir-196

Radionuclide Production Cross Section



29

Incident Energy (MeV)

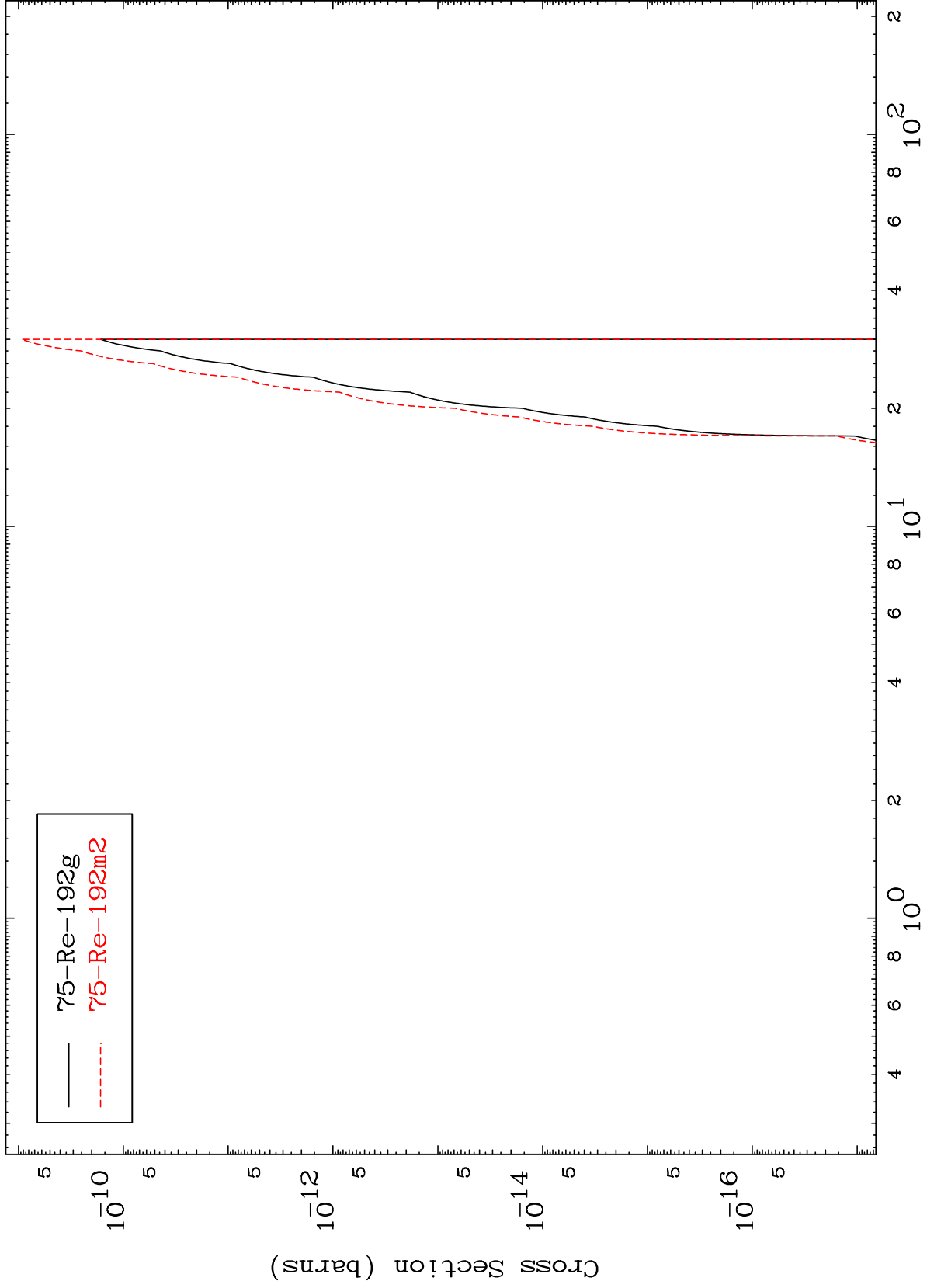
<sup>77</sup>Ir-196

MAT 7740

(n,d)  $\alpha$

<sup>77</sup>Ir-196

Radionuclide Production Cross Section



30

Incident Energy (MeV)

<sup>77</sup>Ir-196