

Program EVALPLOT
(Version 2021-1)

by

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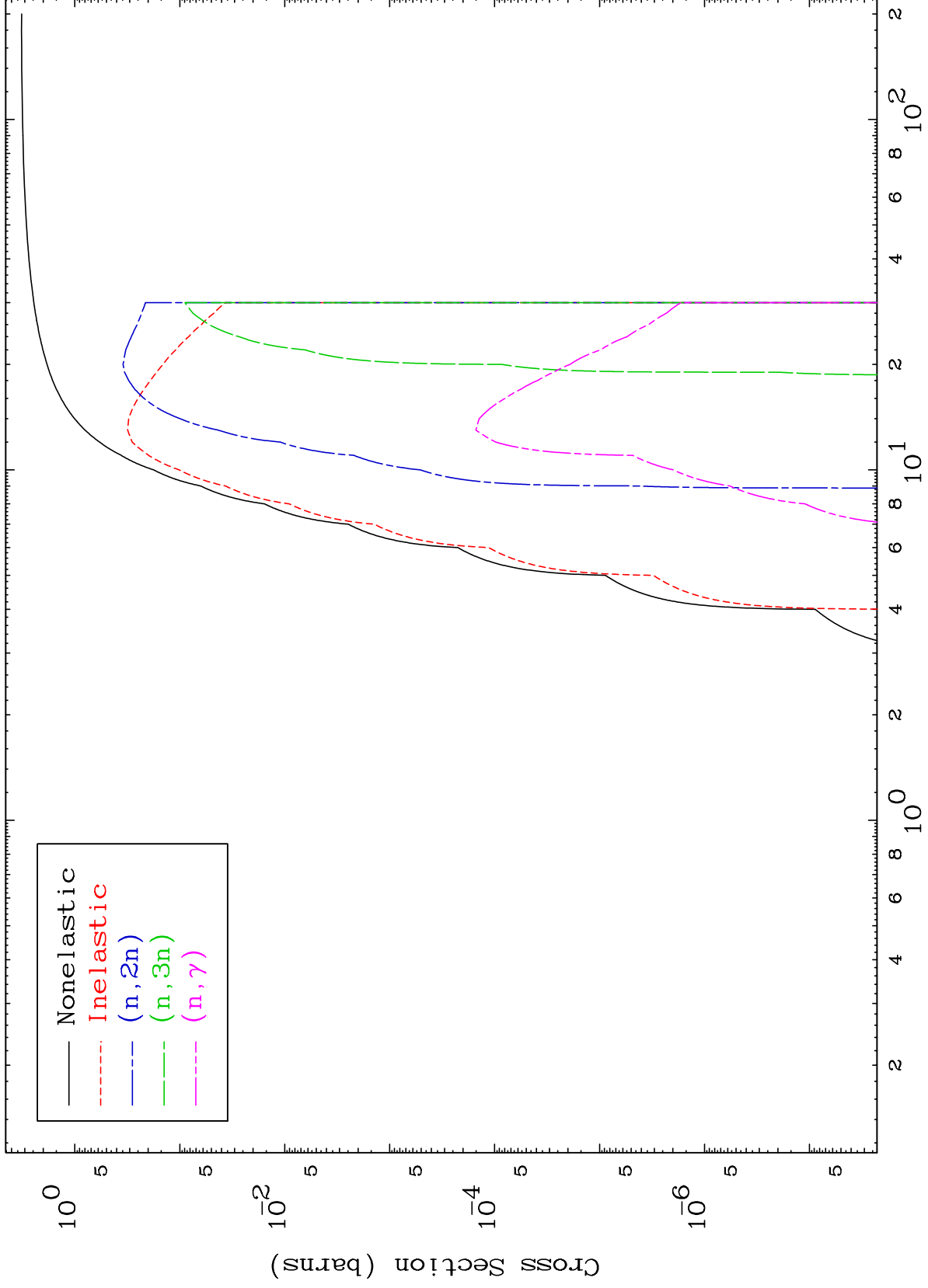
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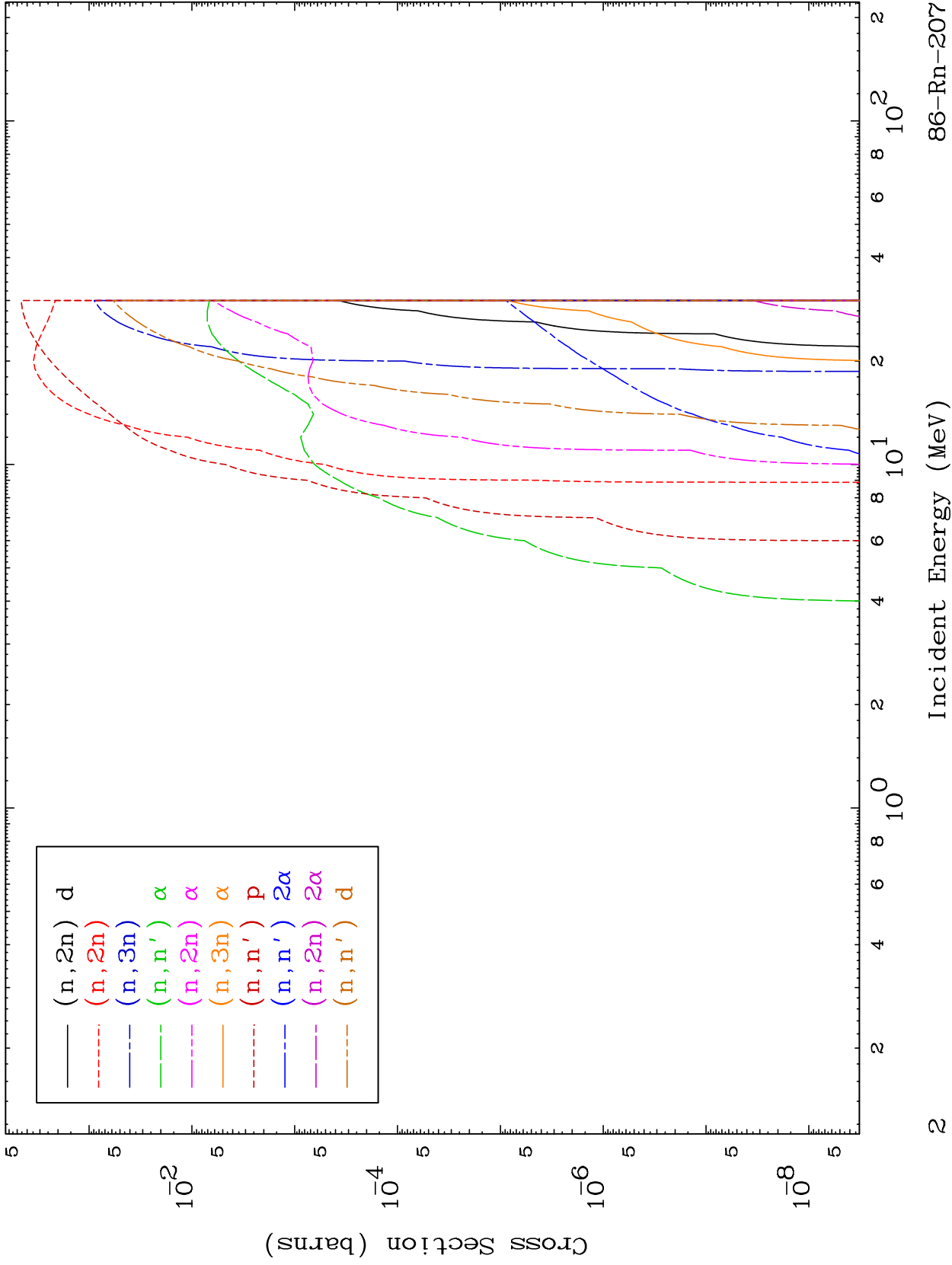
Press Mouse Button to Start

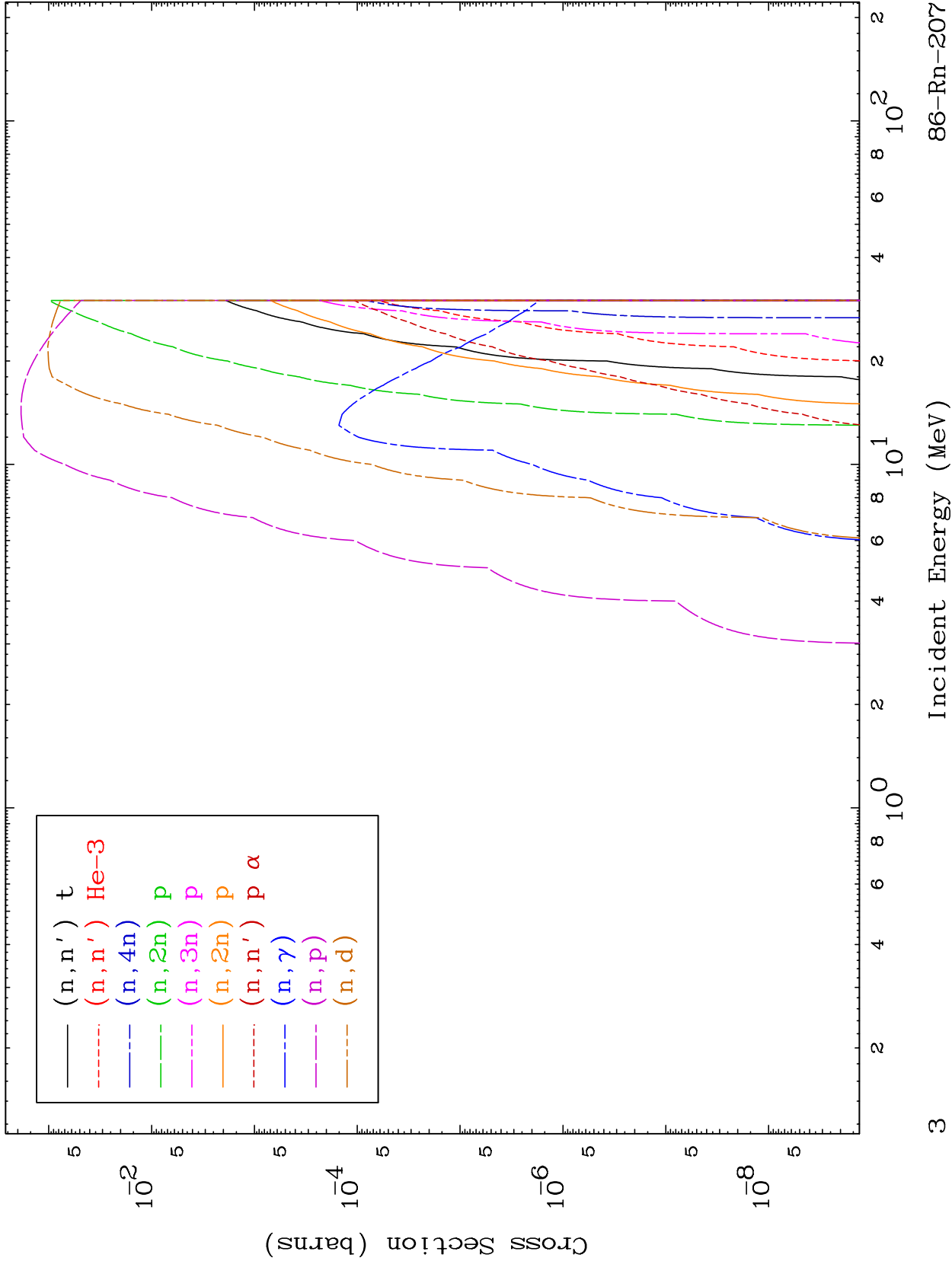
MAT 8613

Deuteron Major
0 Kelvin Cross Sections

86-Rn-207



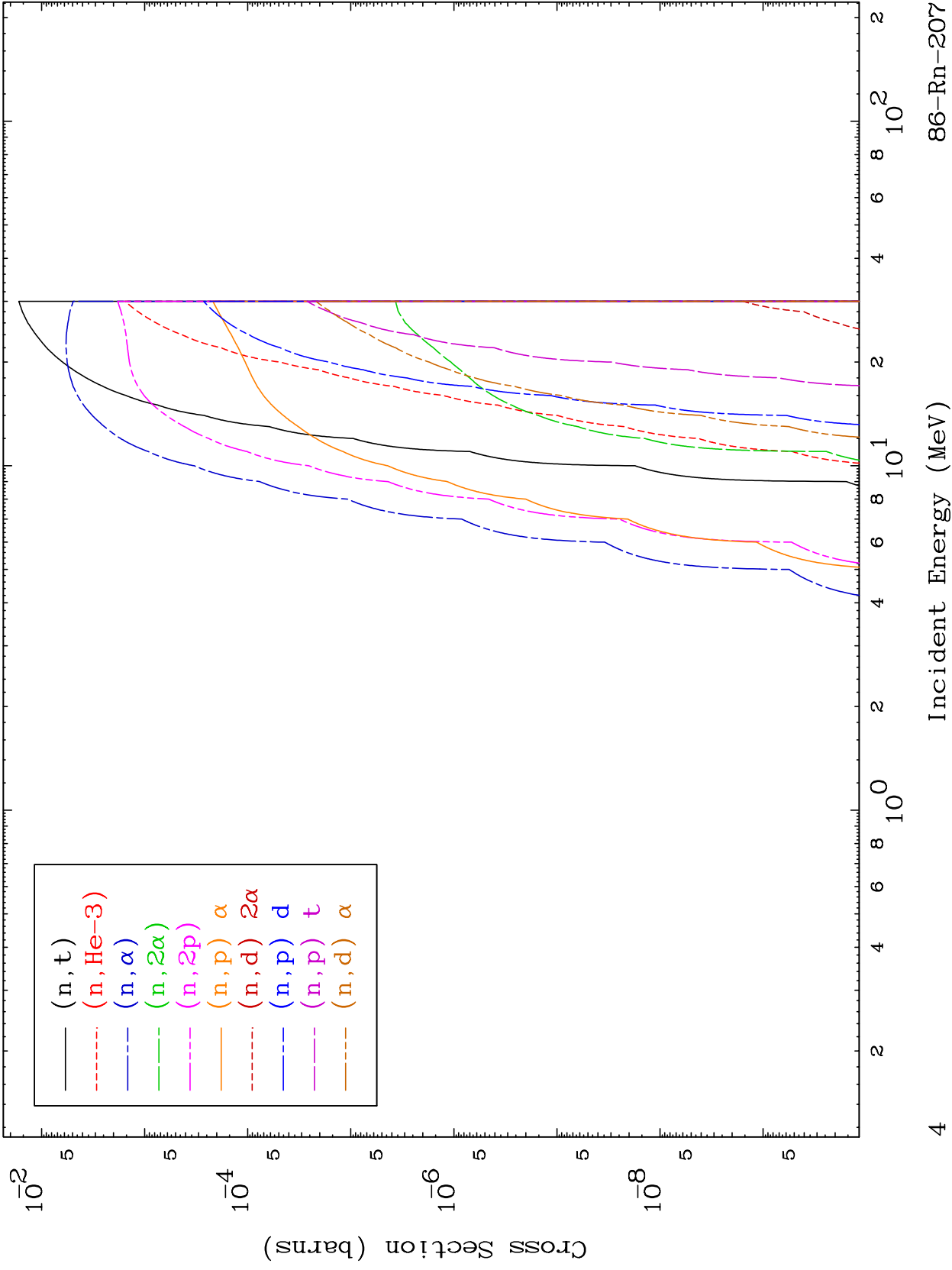


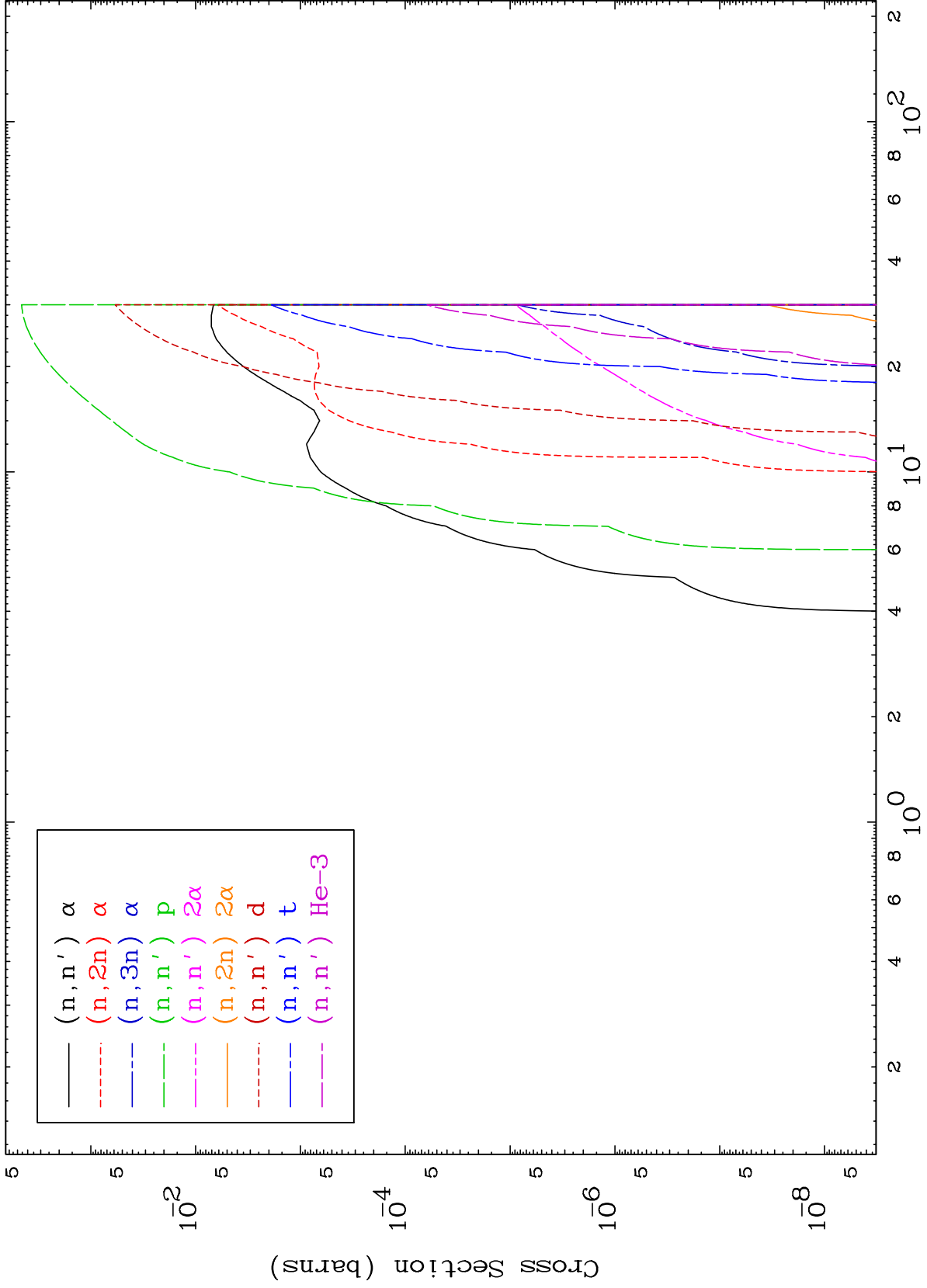


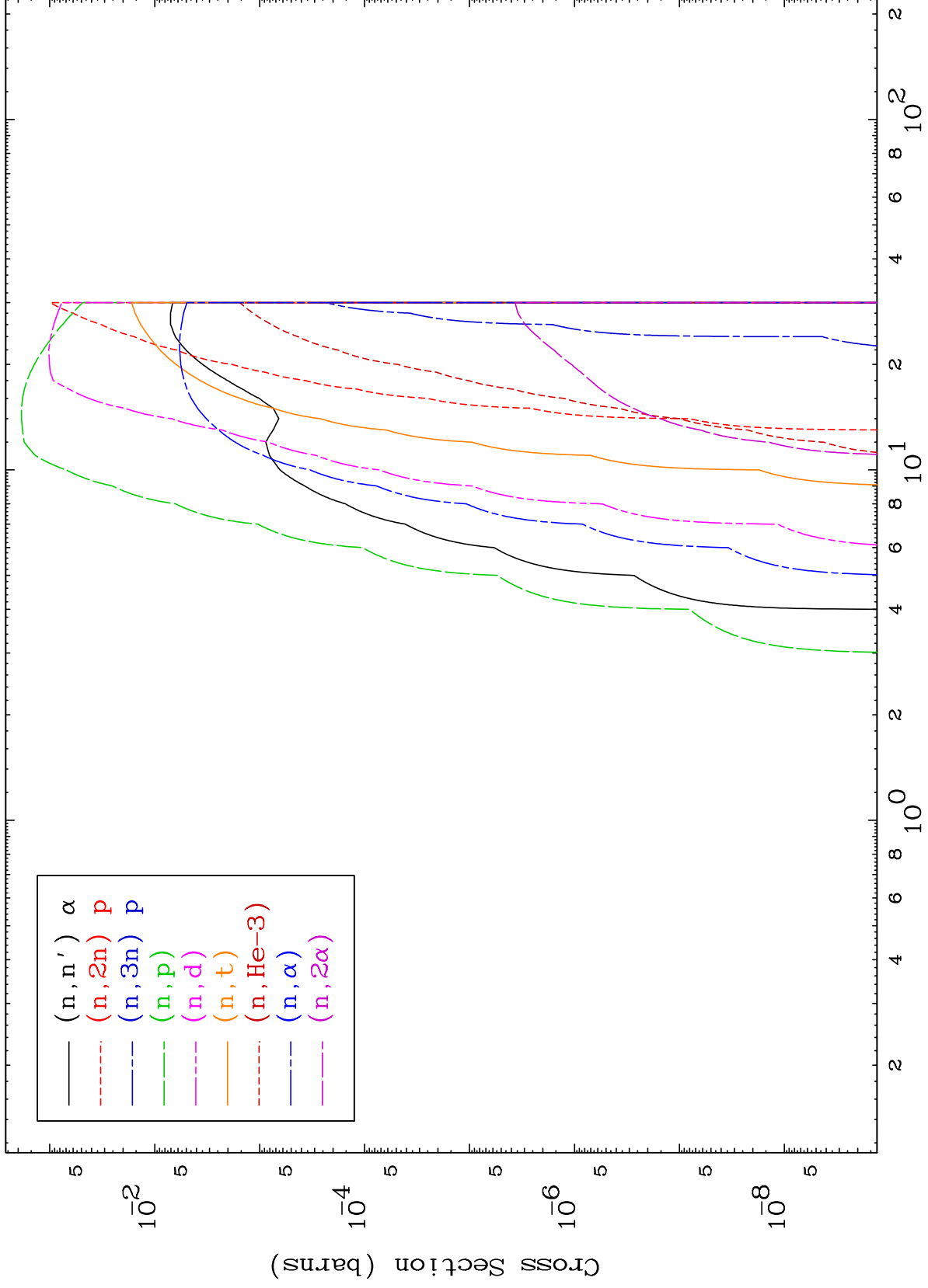
MAT 8613

Deuteron Neutron Absorption
0 Kelvin Cross Sections

86-Rn-207



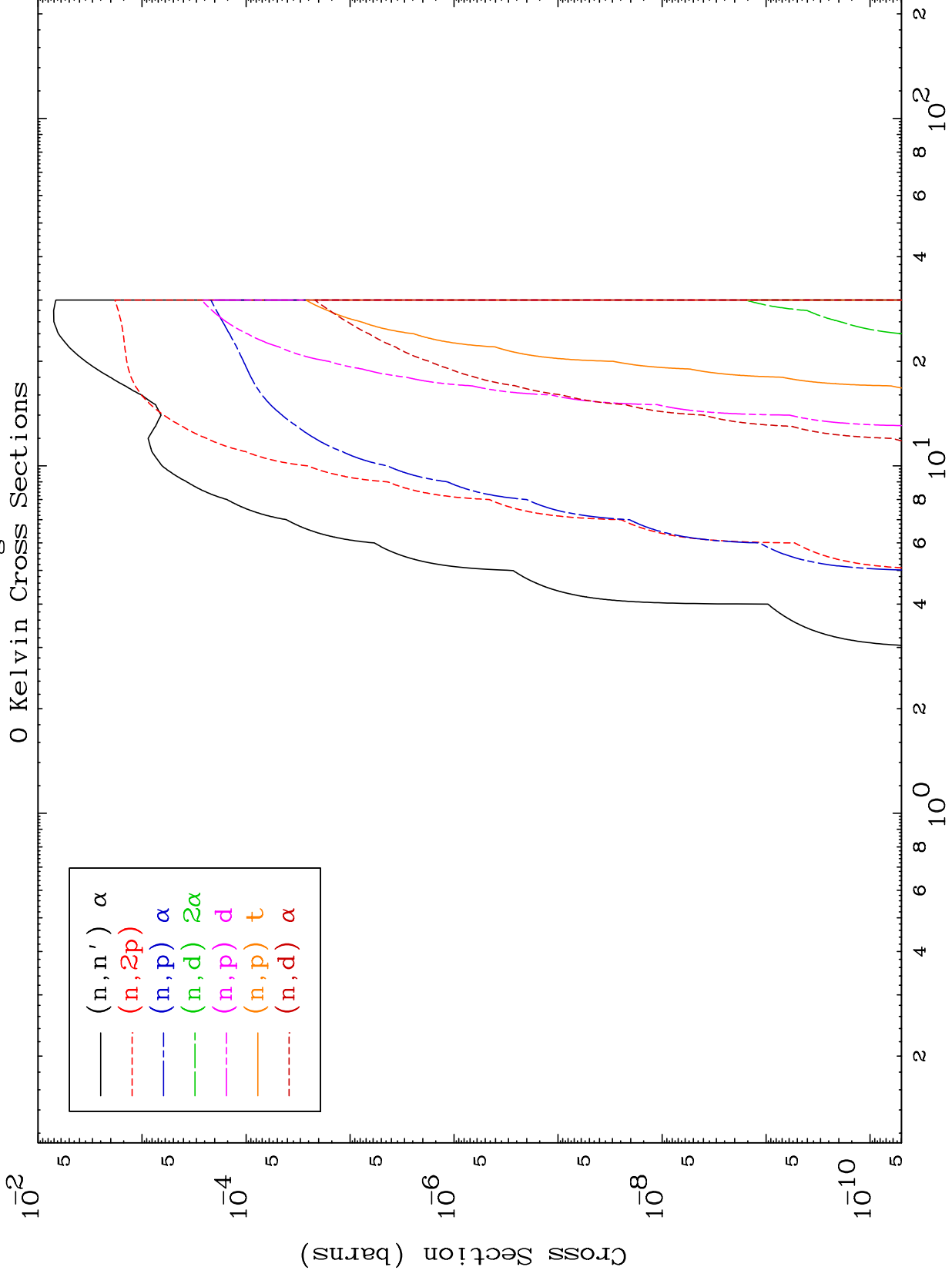




MAT 8613

Deuteron Charged Particle
0 Kelvin Cross Sections

86-Rn-207

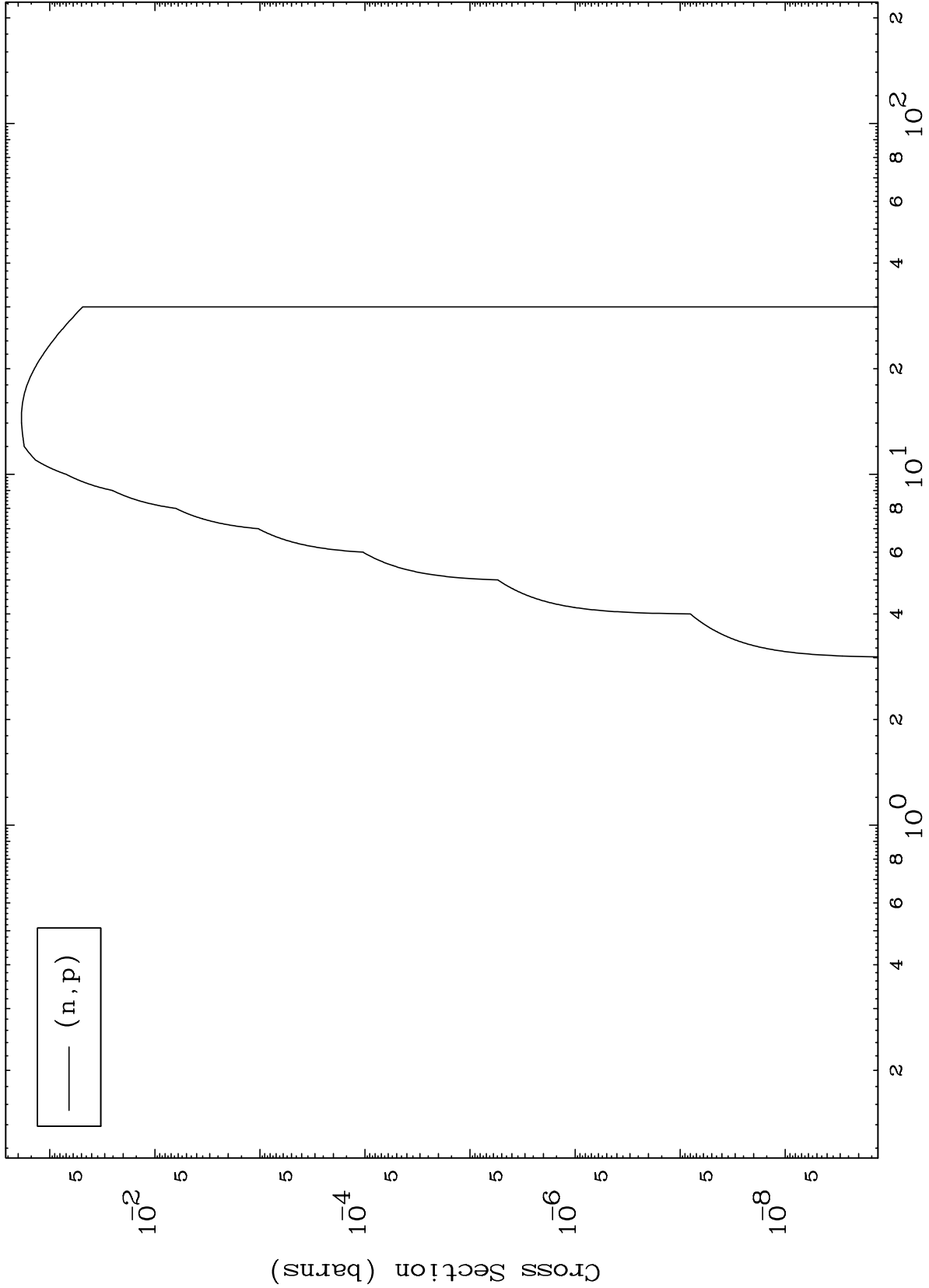


MAT 8613

(d,p) Levels

86-Rn-207

0 Kelvin Cross Sections

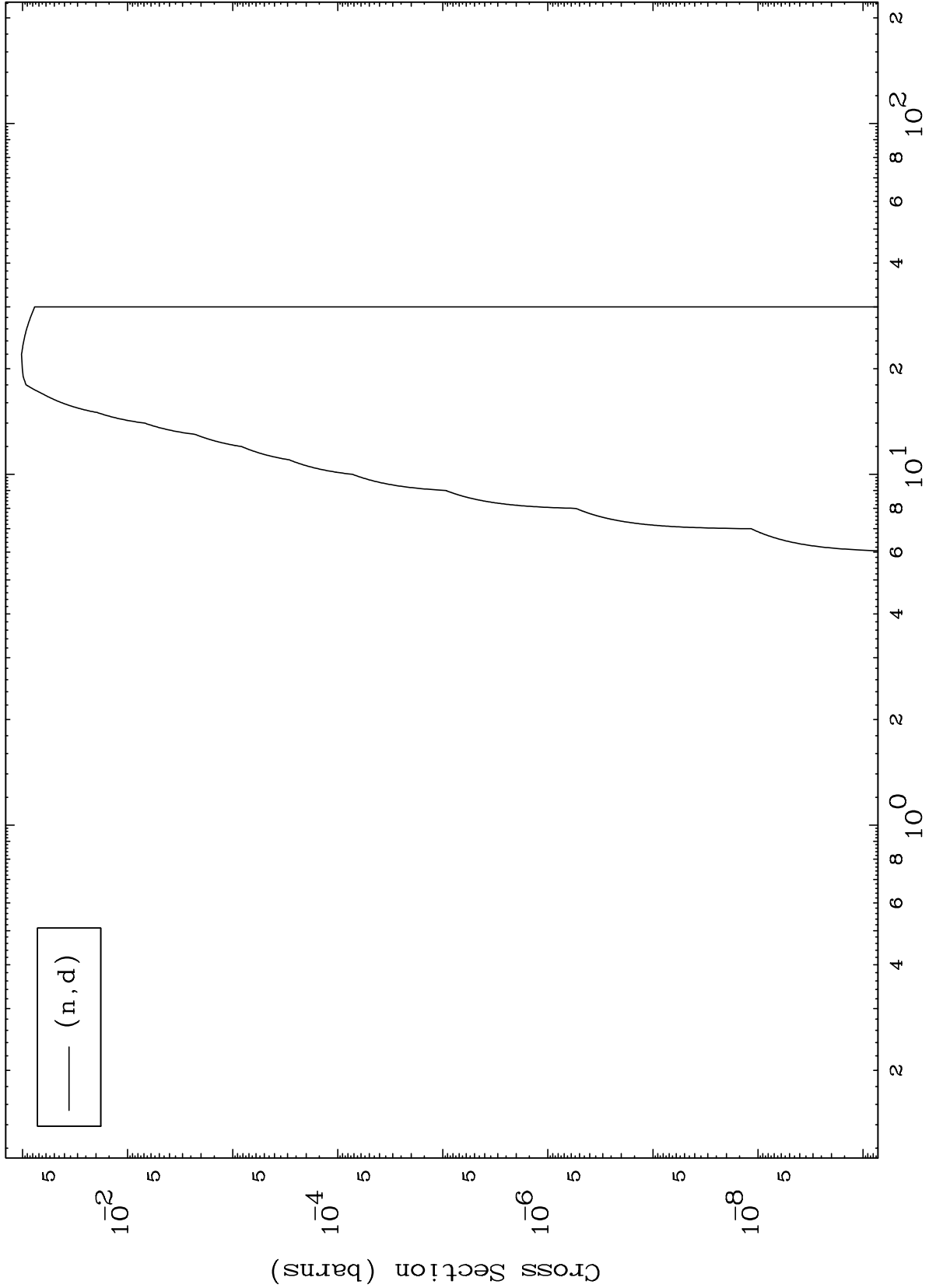


MAT 8613

(d,d) Levels

86-Rn-207

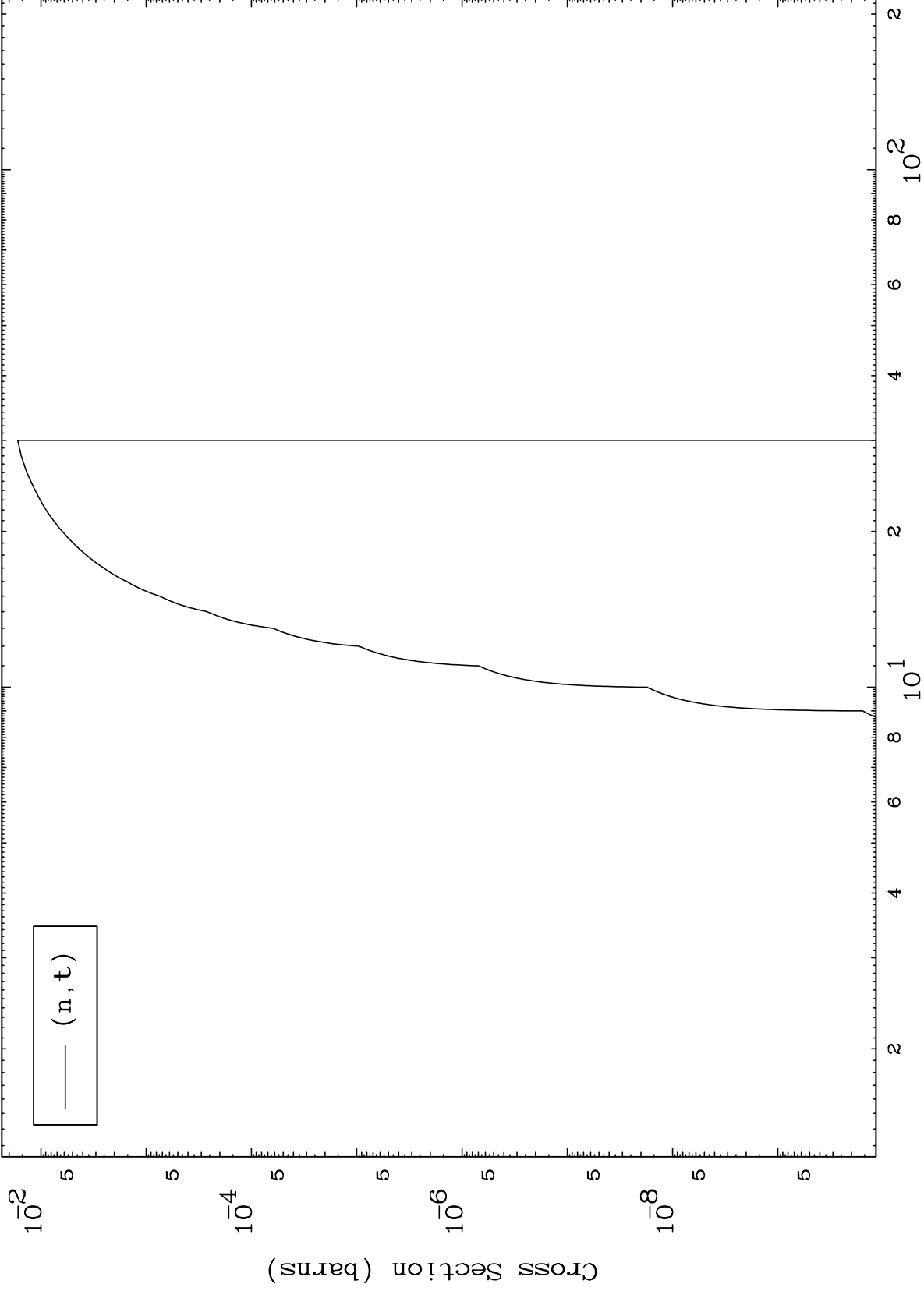
0 Kelvin Cross Sections



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(d,t) Levels
0 Kelvin Cross Sections

86-Rn-207



10

Incident Energy (MeV)

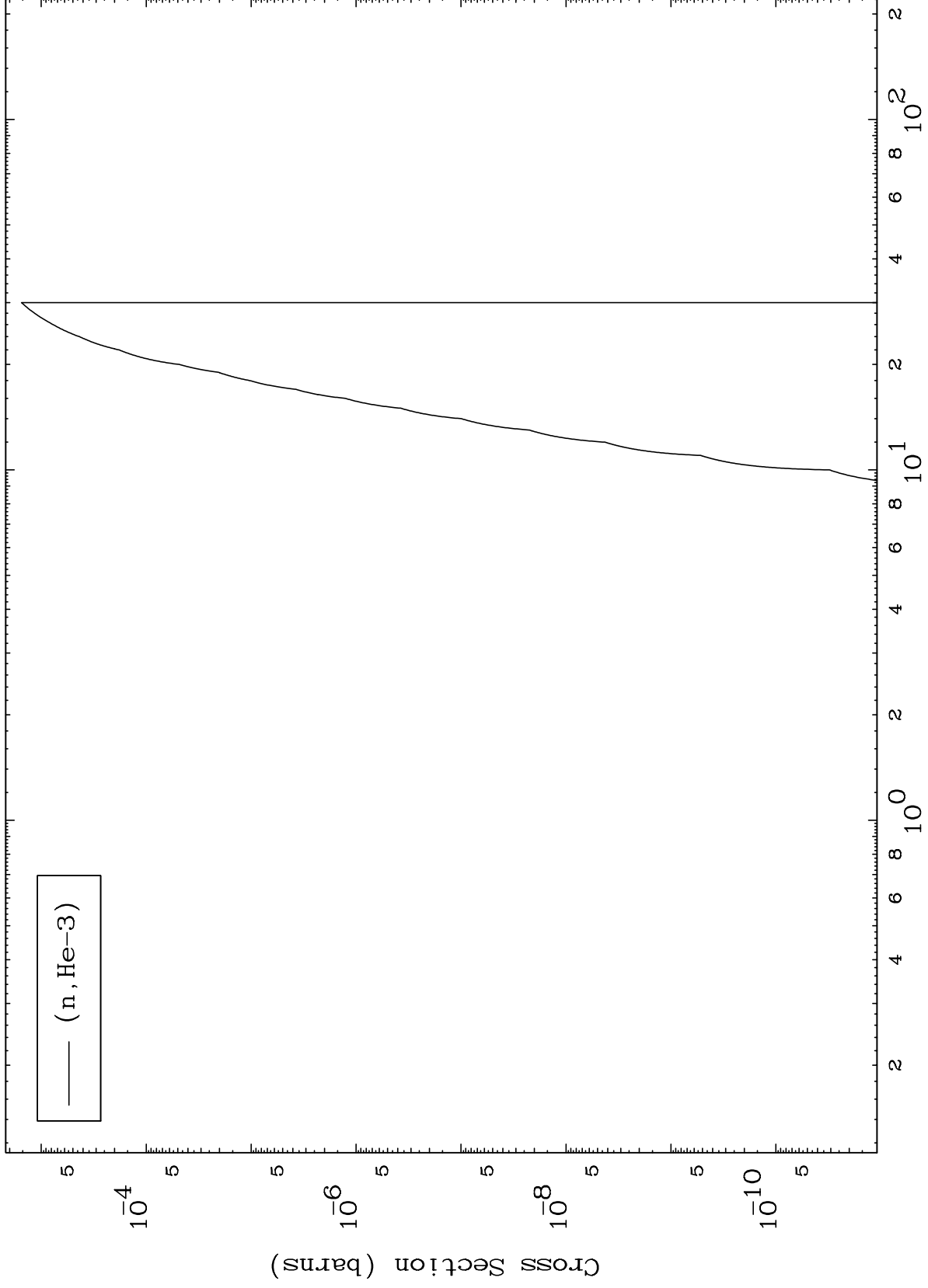
86-Rn-207

MAT 8613

(d,He3) Levels

86-Rn-207

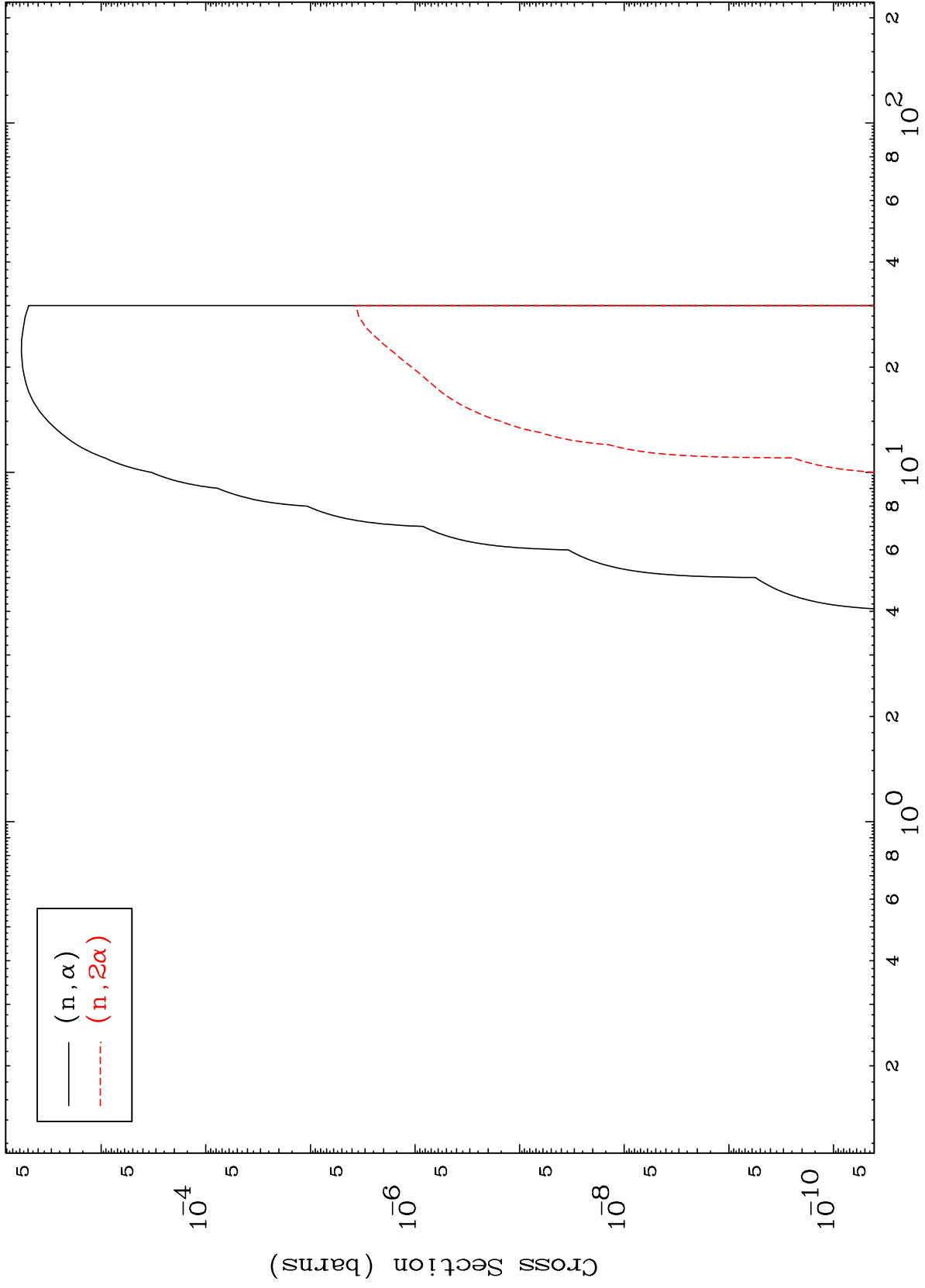
0 Kelvin Cross Sections



MAT 8613

(d, α) Levels
0 Kelvin Cross Sections

86-Rn-207



12

Incident Energy (MeV)

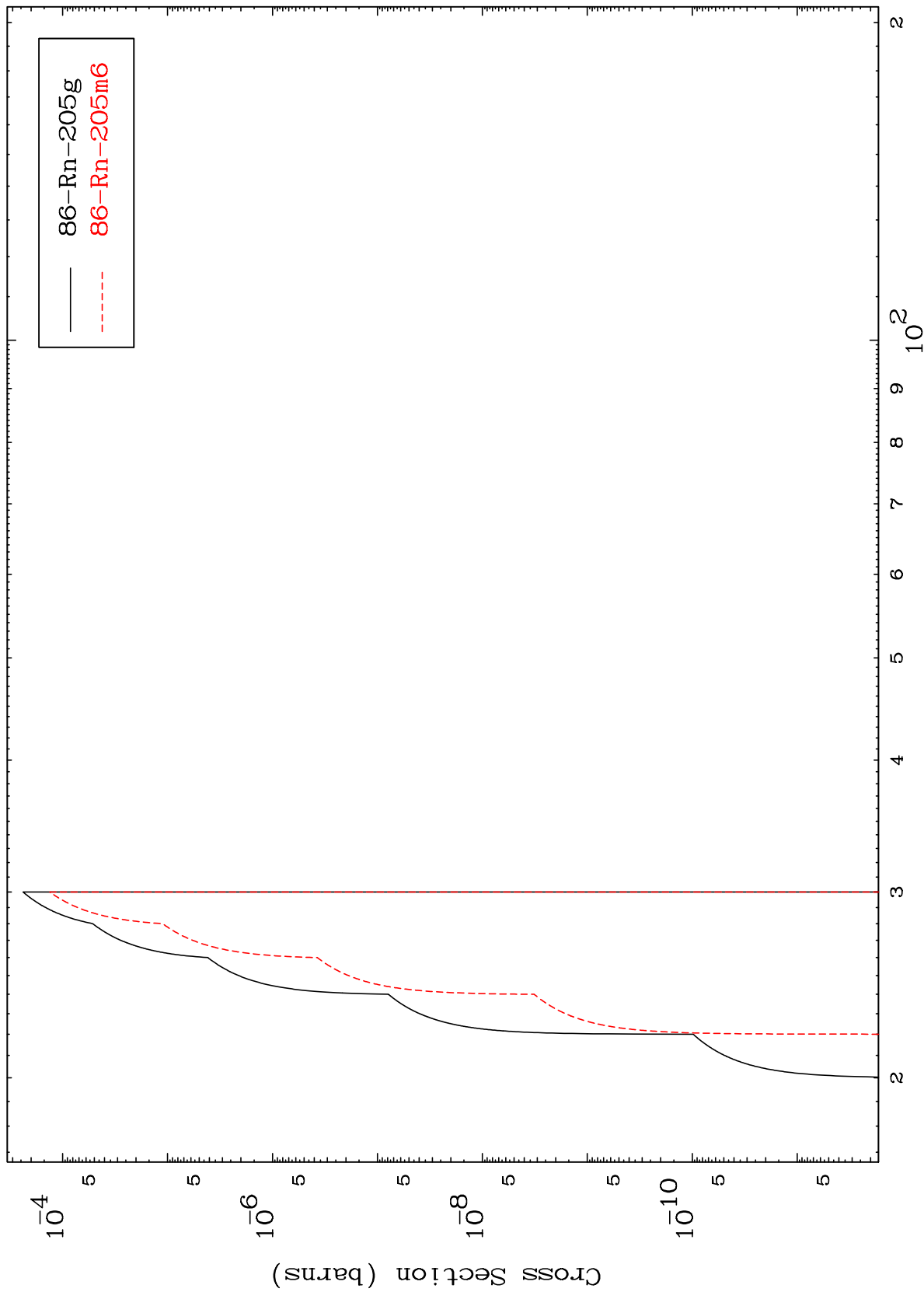
86-Rn-207

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(n,2n) d

86-Rn-207

Radionuclide Production Cross Section



13

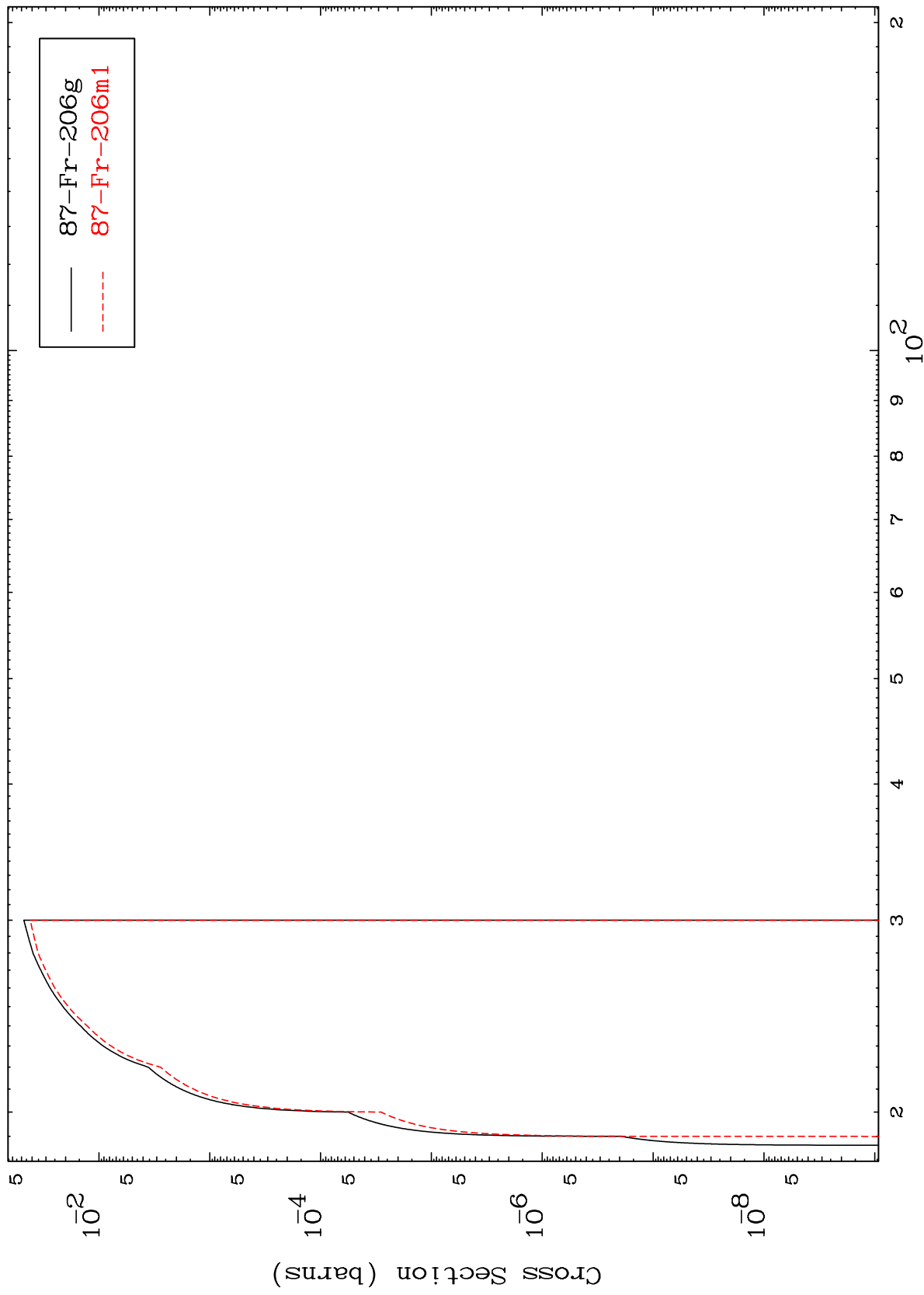
Incident Energy (MeV)

86-Rn-207

MAT 8613

86-Rn-207

(n,3n)
Radionuclide Production Cross Section



86-Rn-207

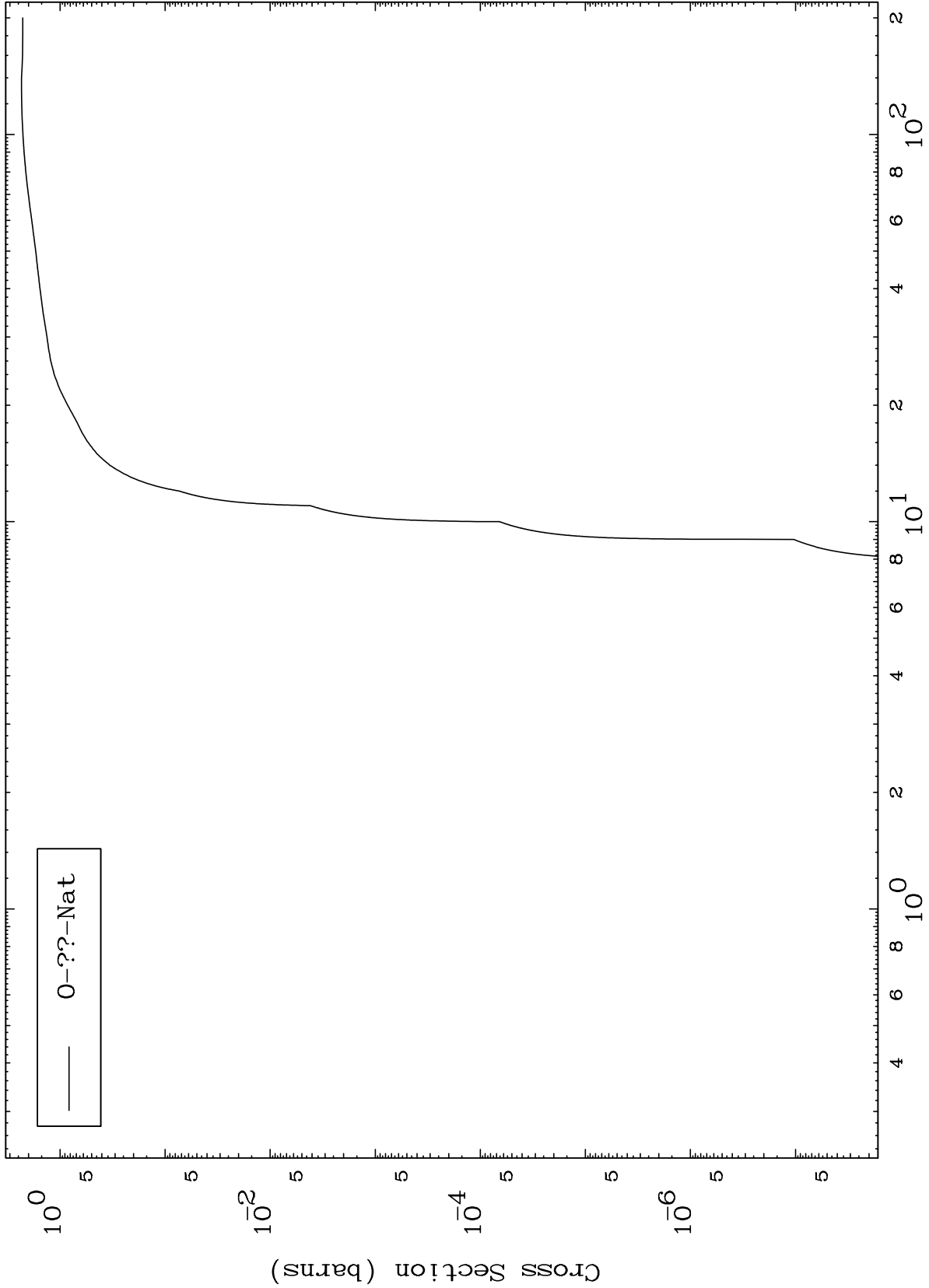
Incident Energy (MeV)

14

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86-Rn-207

Fission
Radionuclide Production Cross Section

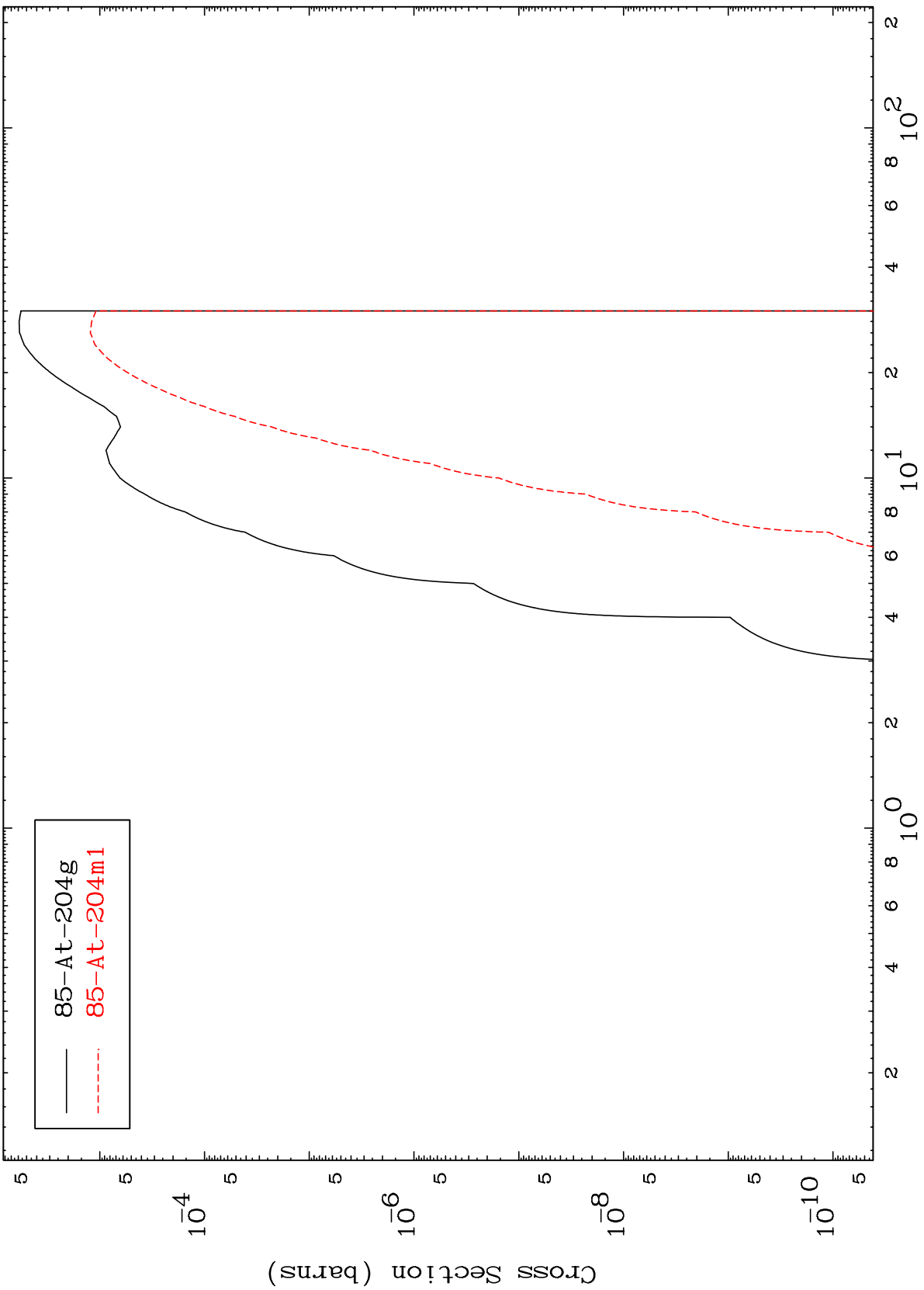


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$(n, n') \alpha$

86-Rn-207

Radionuclide Production Cross Section



16

Incident Energy (MeV)

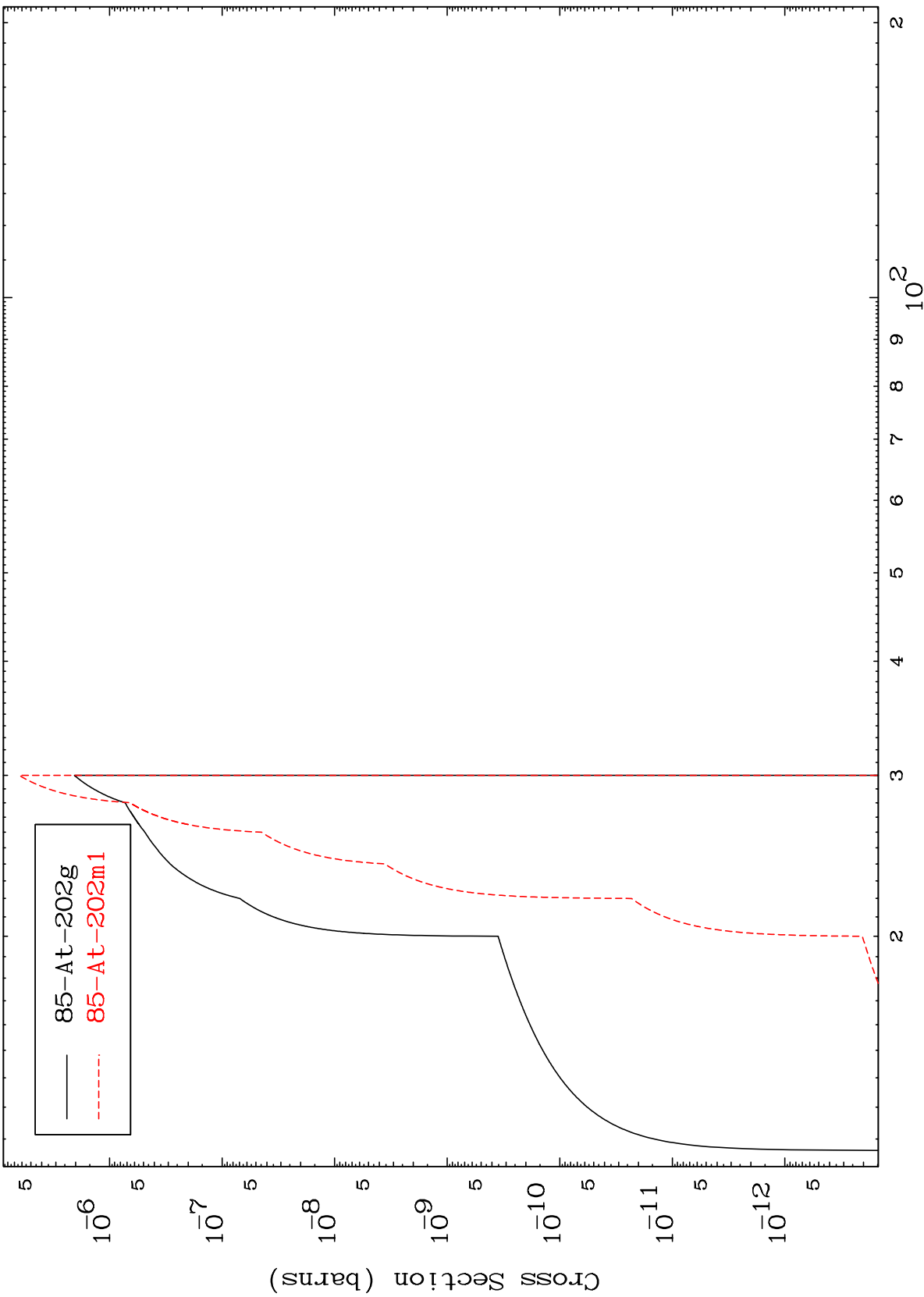
86-Rn-207

MAT 8613

(n,3n) α

86-Rn-207

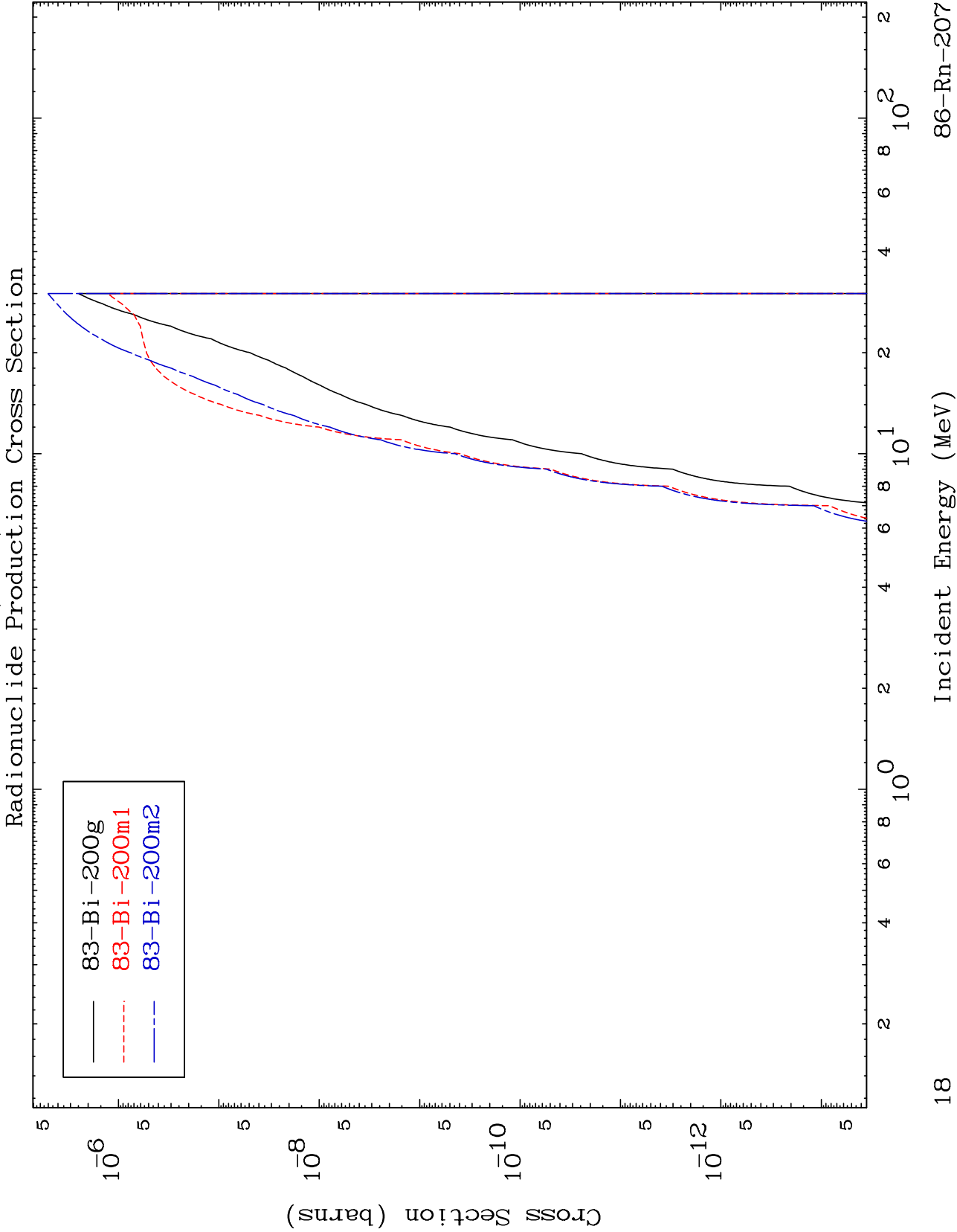
Radionuclide Production Cross Section



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(n,n') 2 α

86-Rn-207



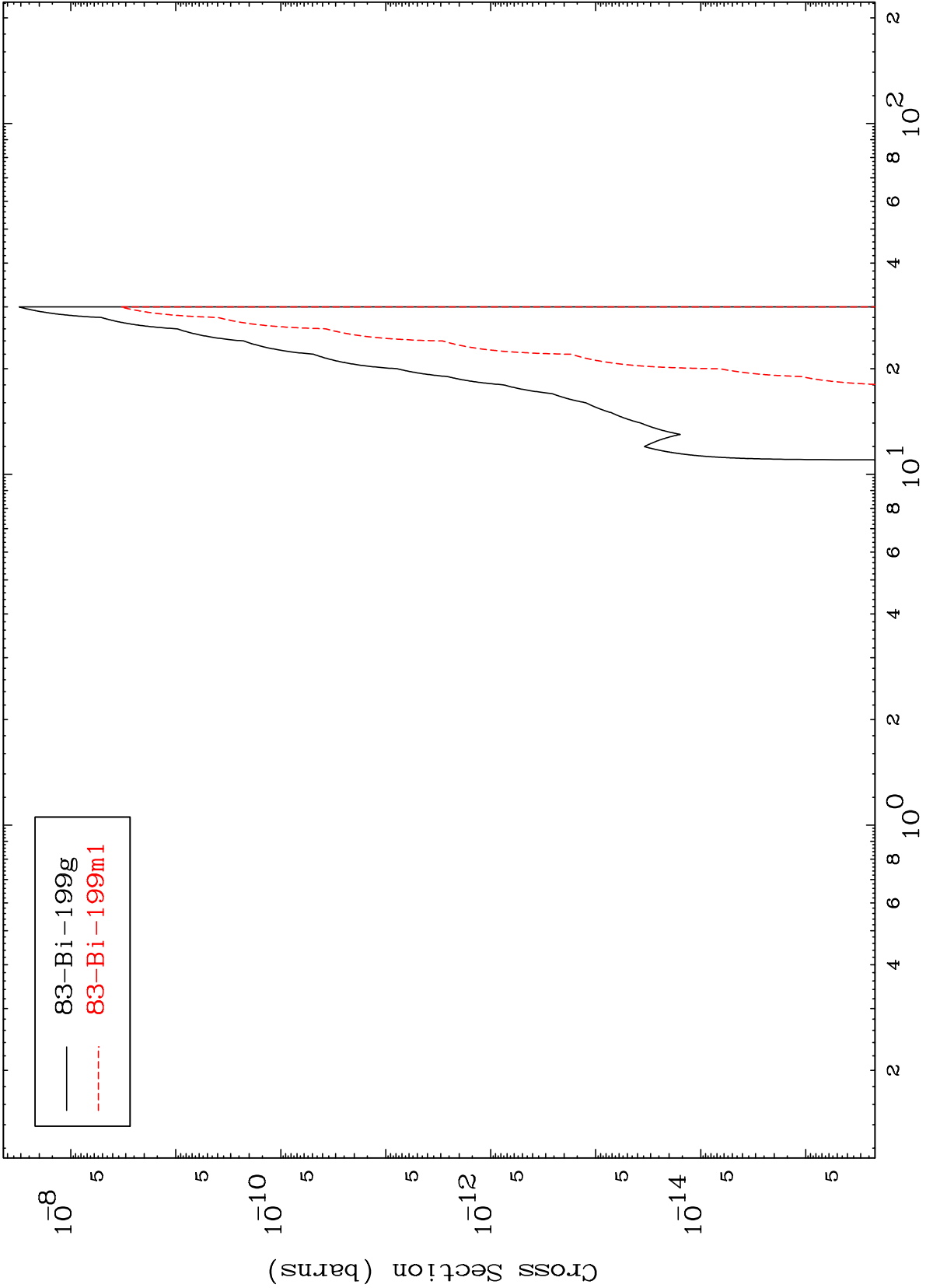
83-Bi-200g
83-Bi-200m1
83-Bi-200m2

MAT 8613

(n,2n) 2α

86-Rn-207

Radionuclide Production Cross Section



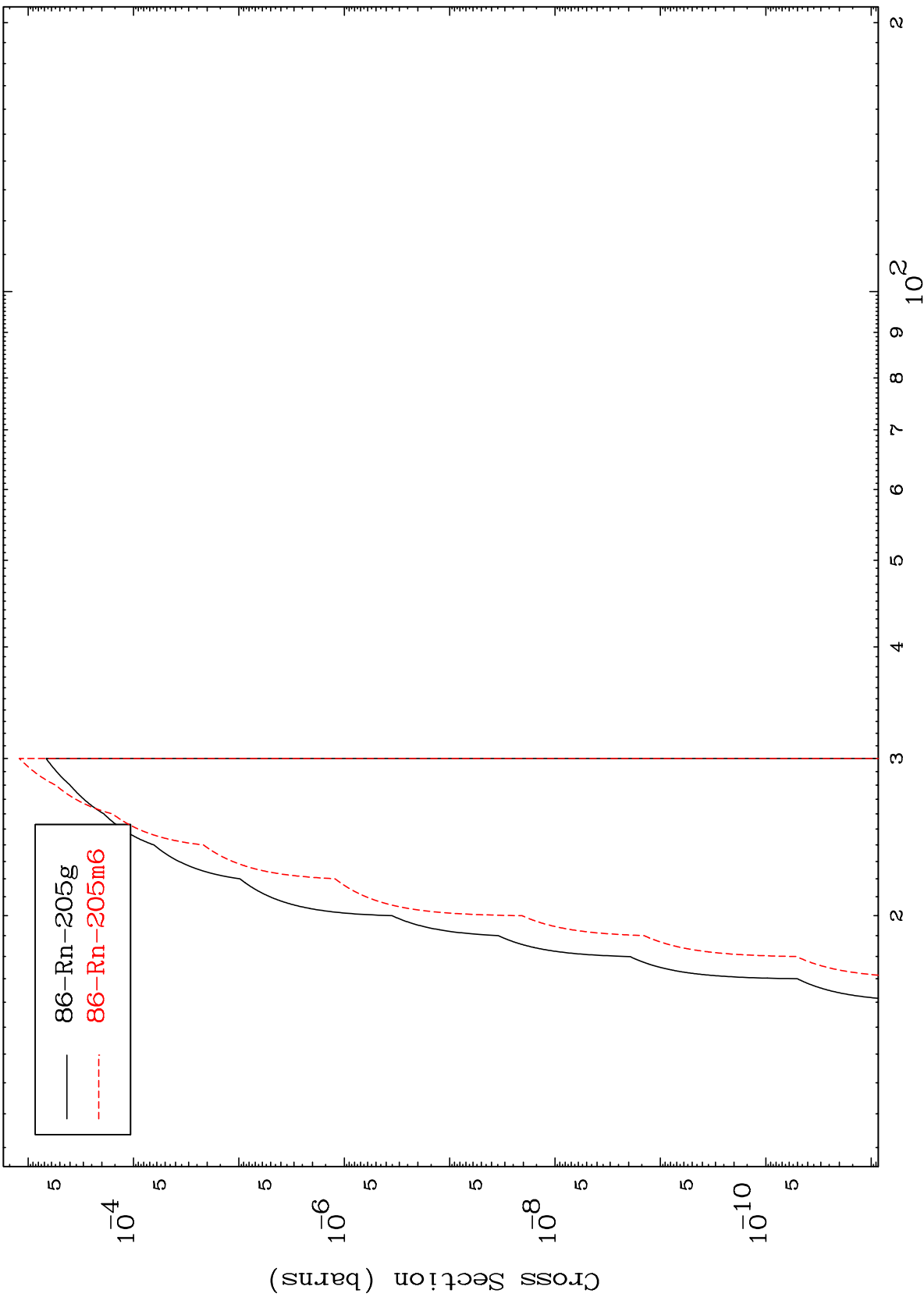
— 83-Bi-199g
- - - 83-Bi-199m1

MAT 8613

(n,n') t

86-Rn-207

Radionuclide Production Cross Section



20

Incident Energy (MeV)

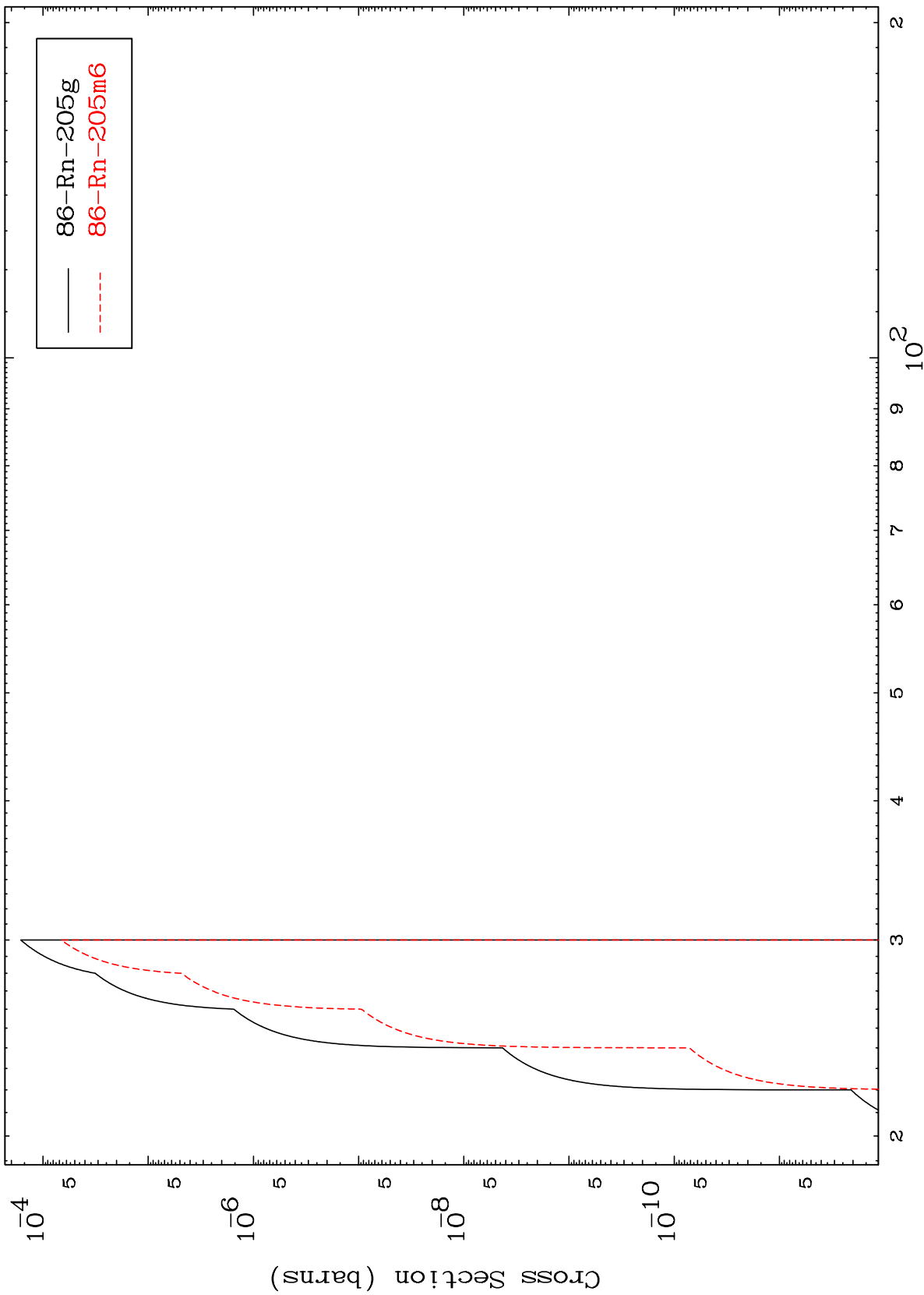
86-Rn-207

MAT 8613

(n,3n) p

86-Rn-207

Radionuclide Production Cross Section



21

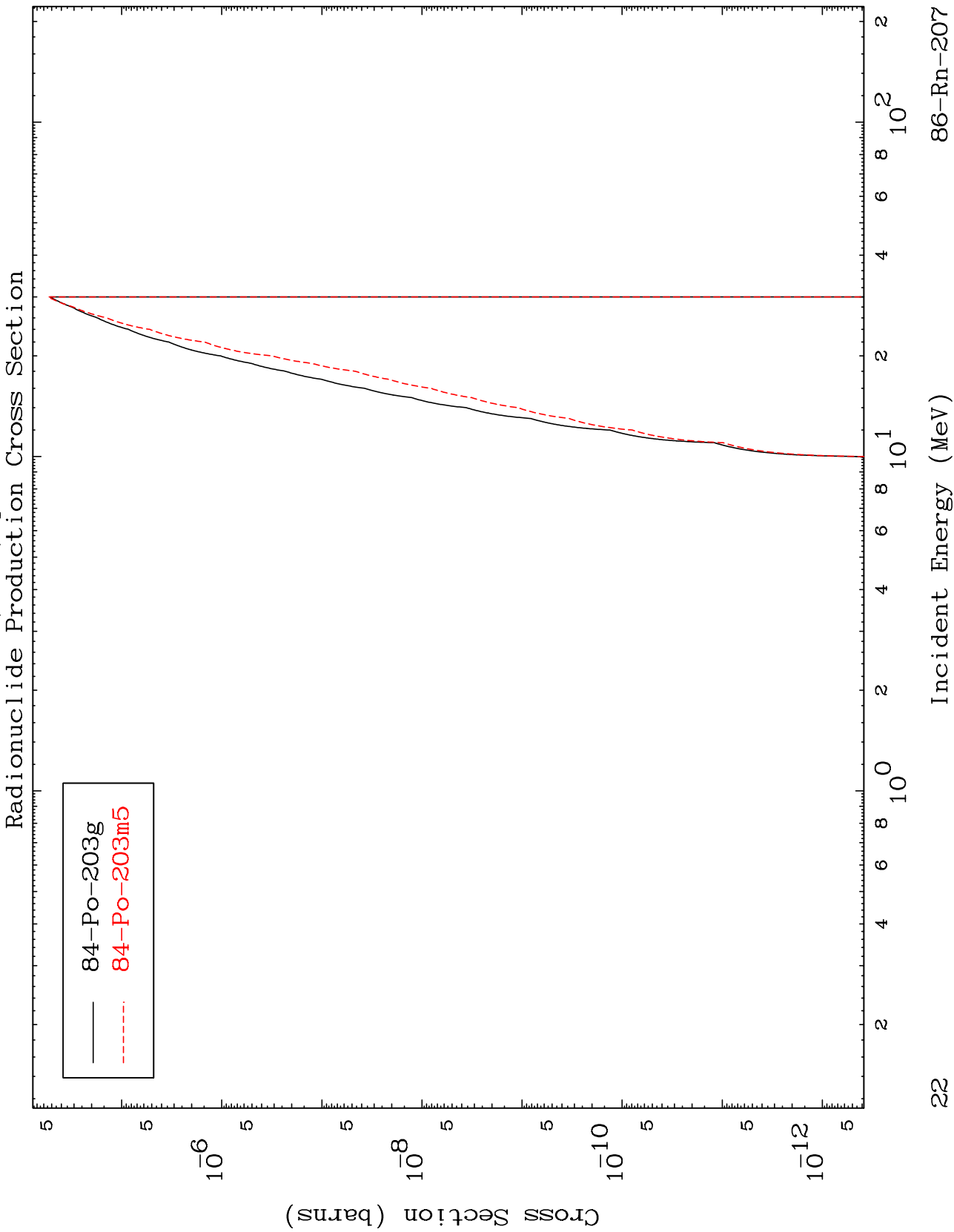
Incident Energy (MeV)

86-Rn-207

MAT 8613

(n,n') p α

86-Rn-207

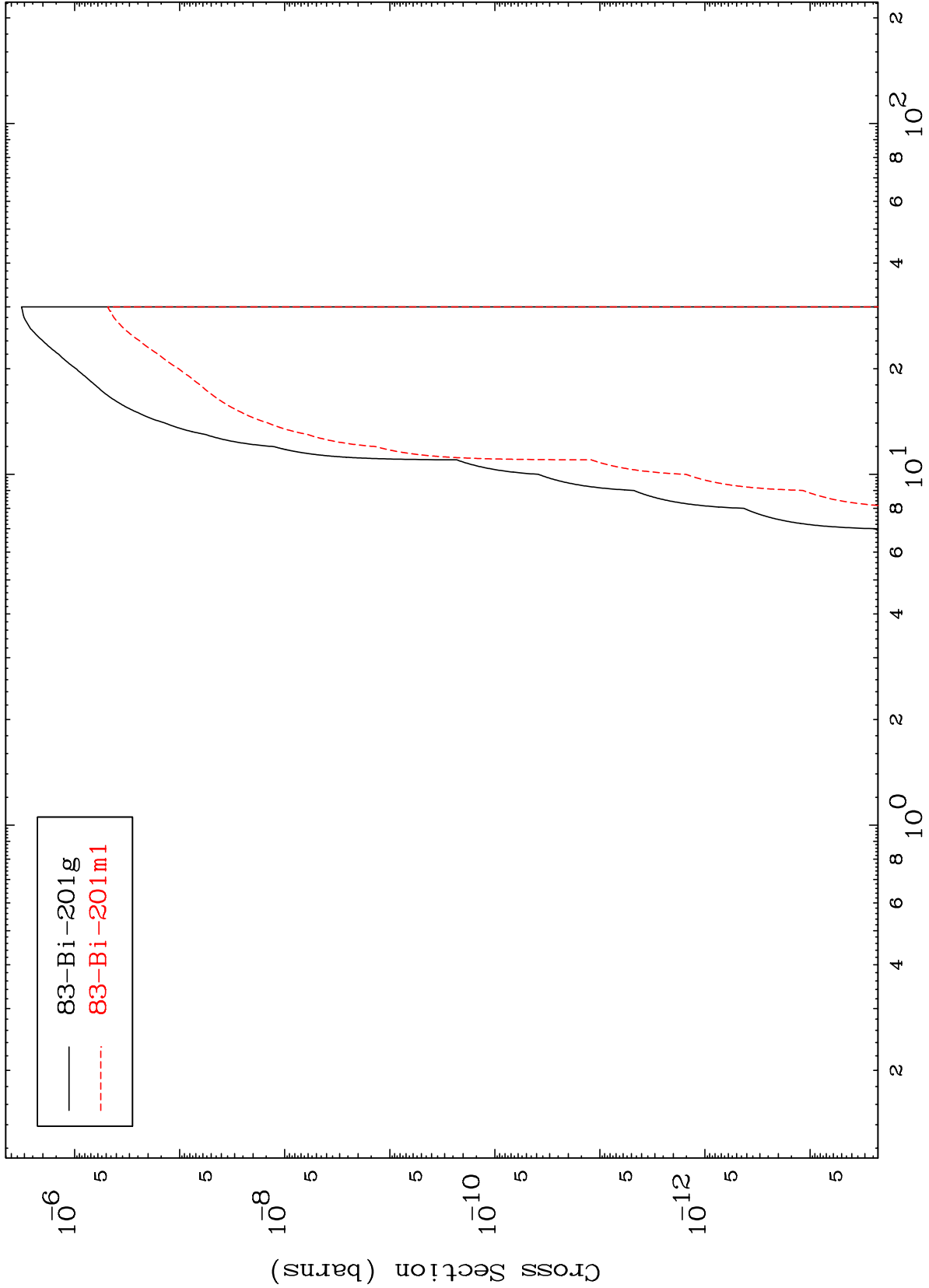


MAT 8613

(n,2α)

86-Rn-207

Radionuclide Production Cross Section

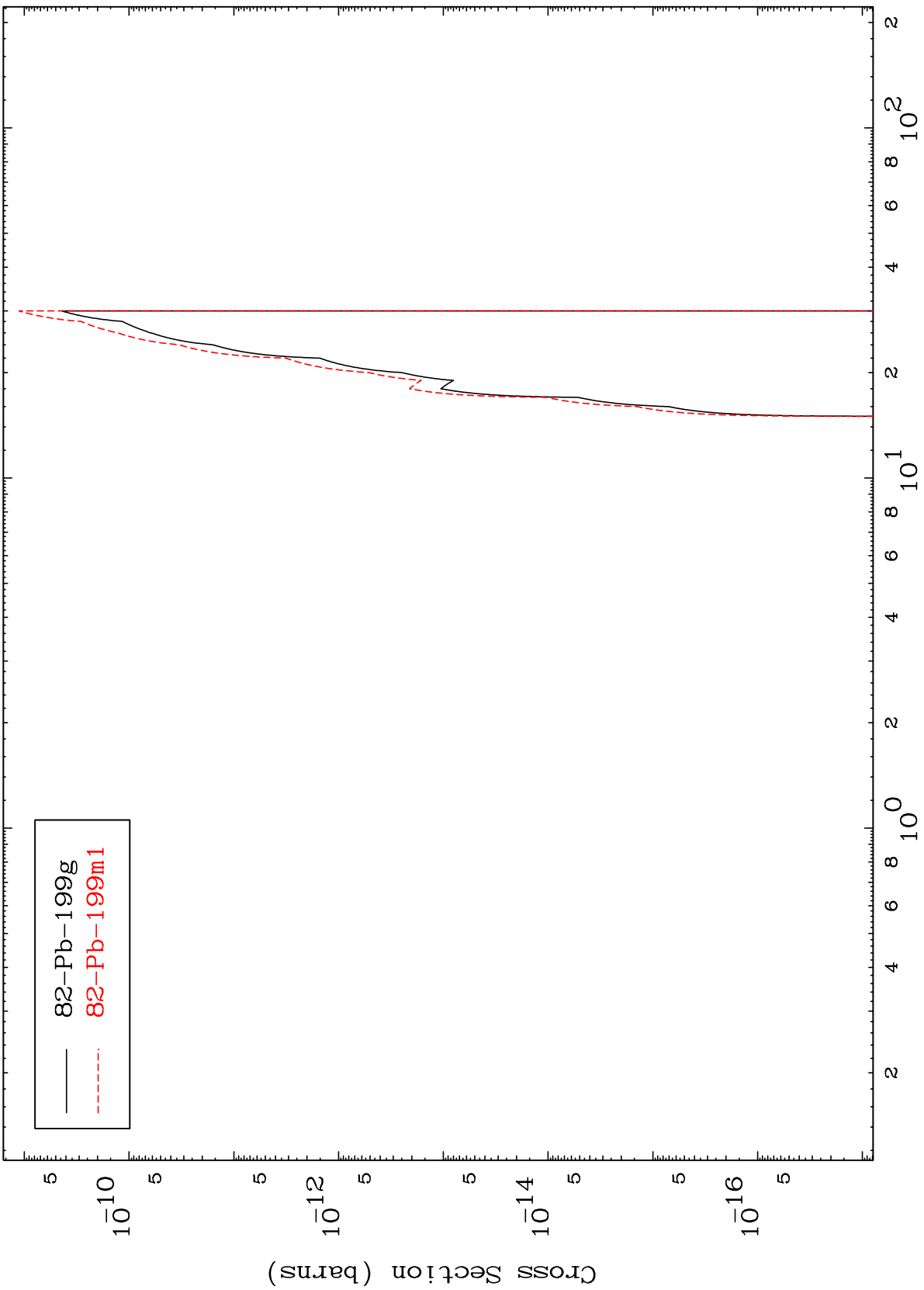


MAT 8613

(n,d) 2α

86-Rn-207

Radionuclide Production Cross Section



24

Incident Energy (MeV)

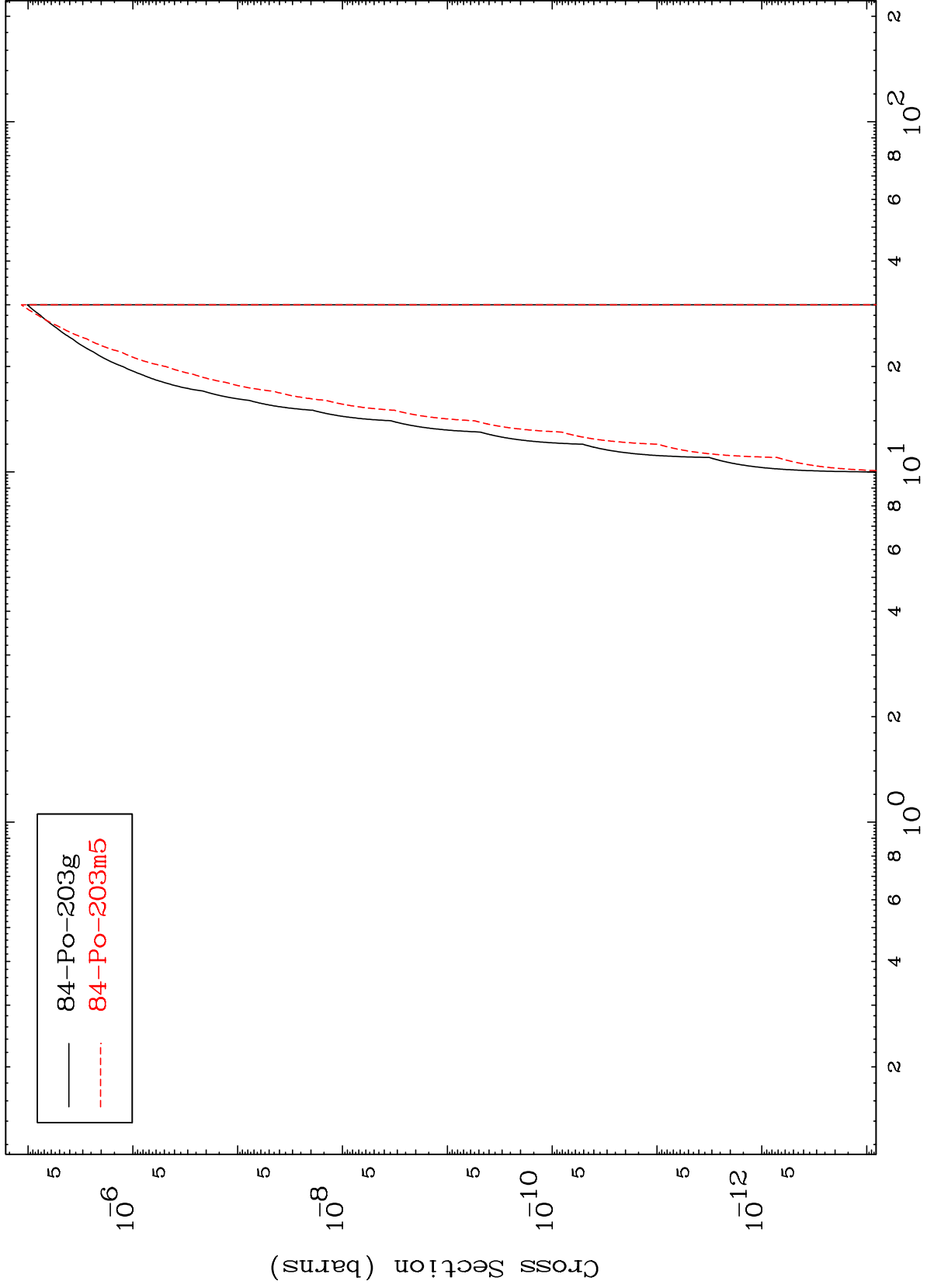
86-Rn-207

MAT 8613

(n,d) α

86-Rn-207

Radionuclide Production Cross Section



25

Incident Energy (MeV)

86-Rn-207