

Program EVALPLOT  
(Version 2021-1)

by

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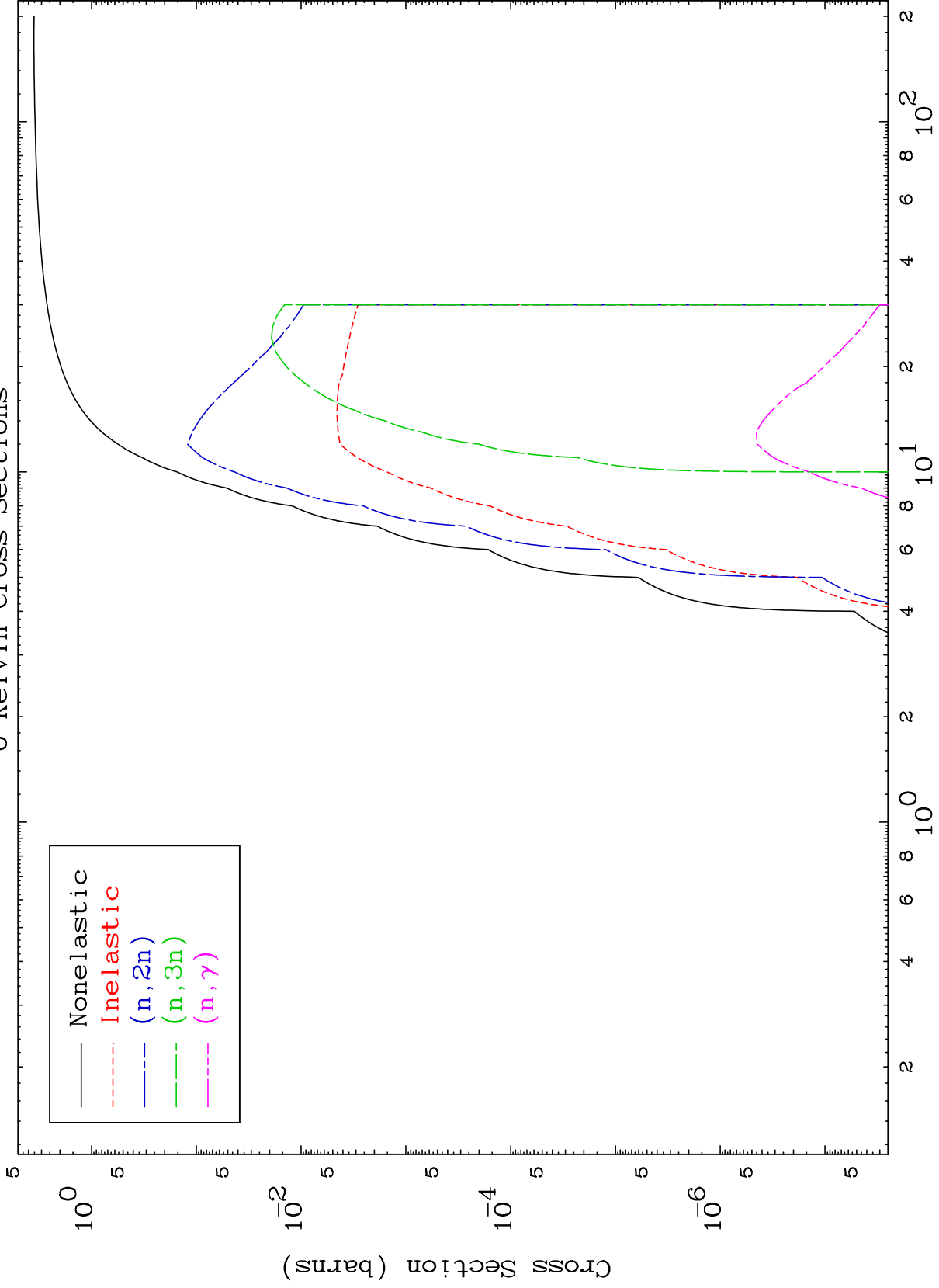
Press Mouse Button to Start

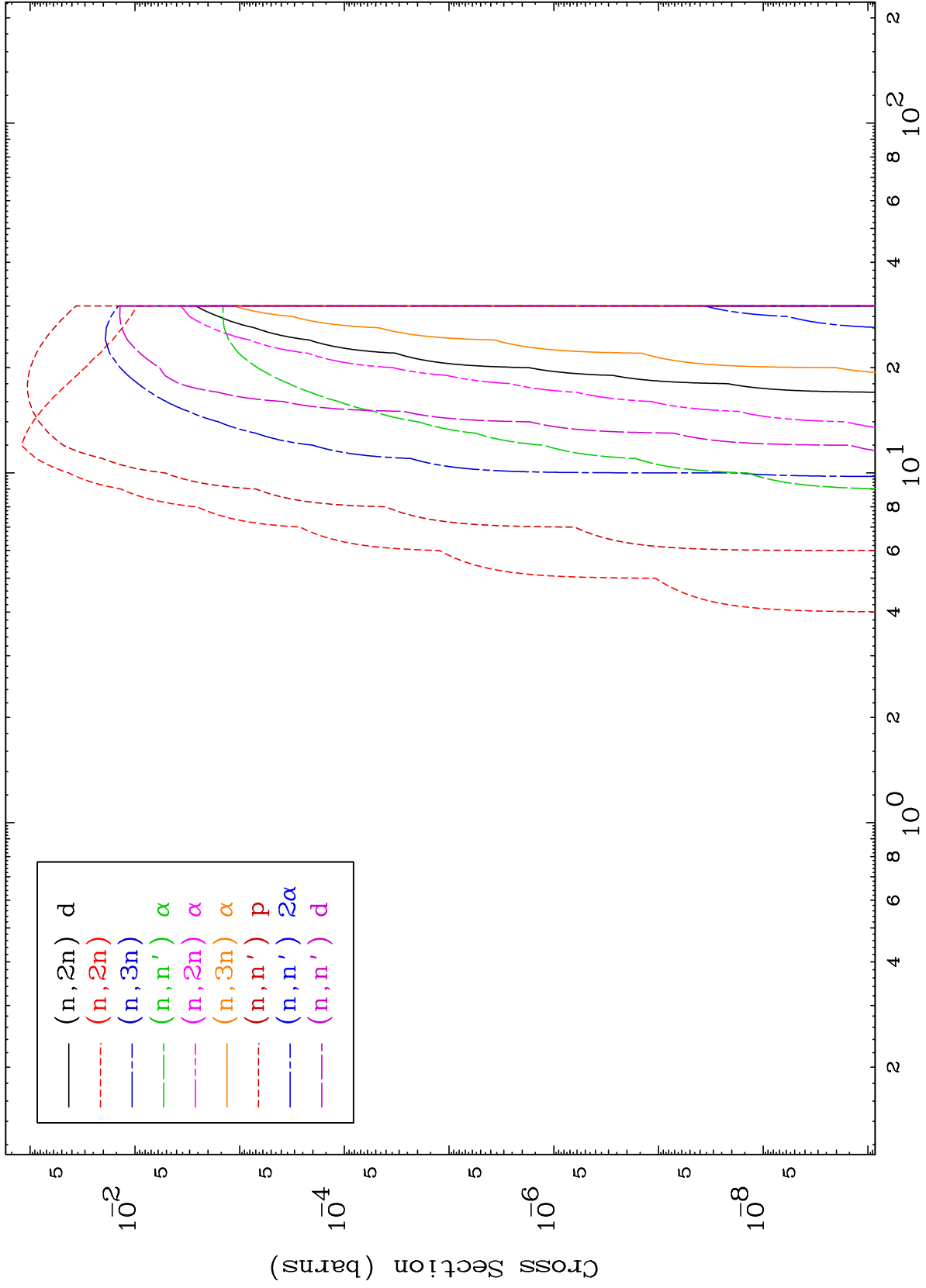
MAT 9231

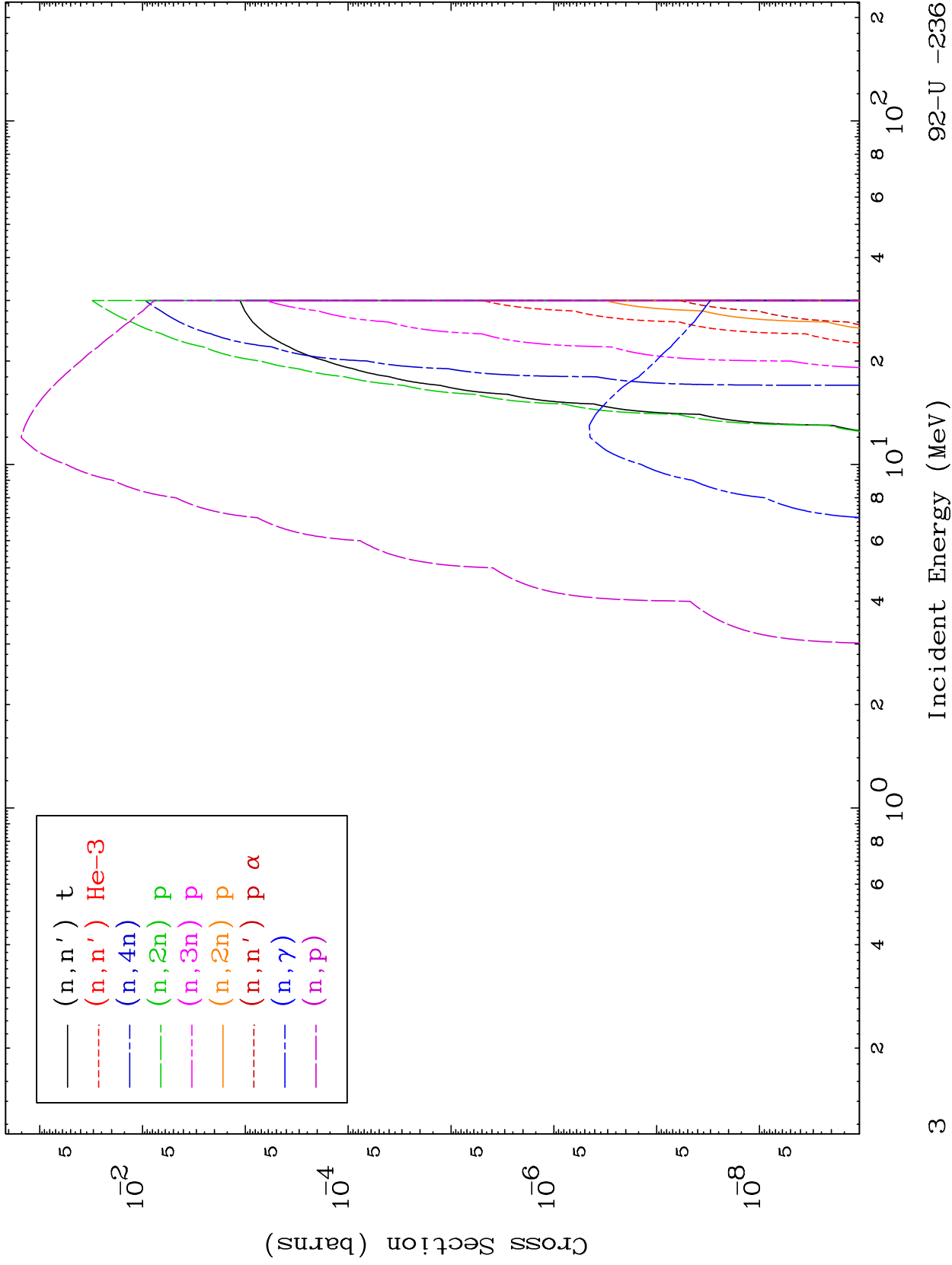
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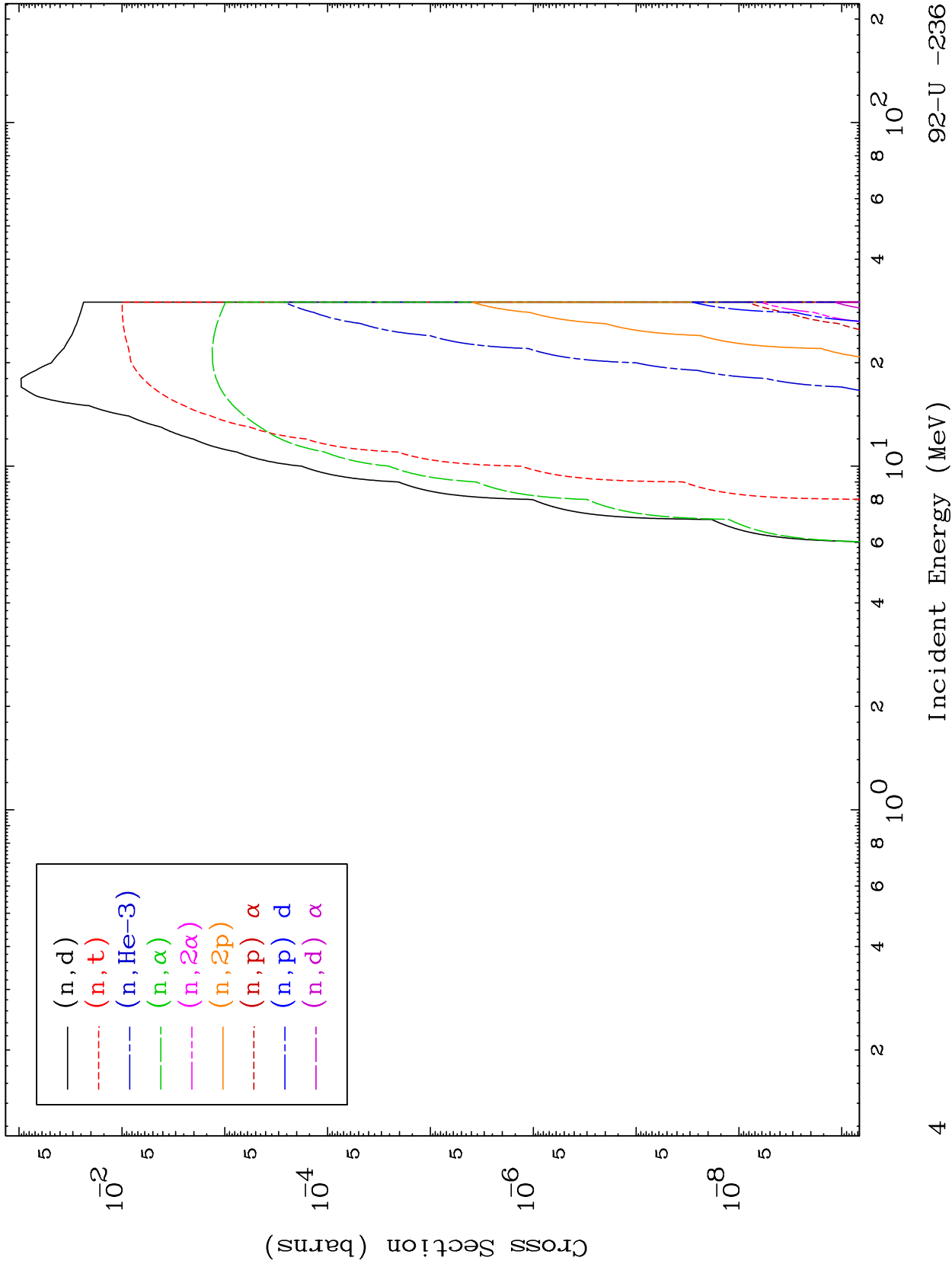
92-U -236

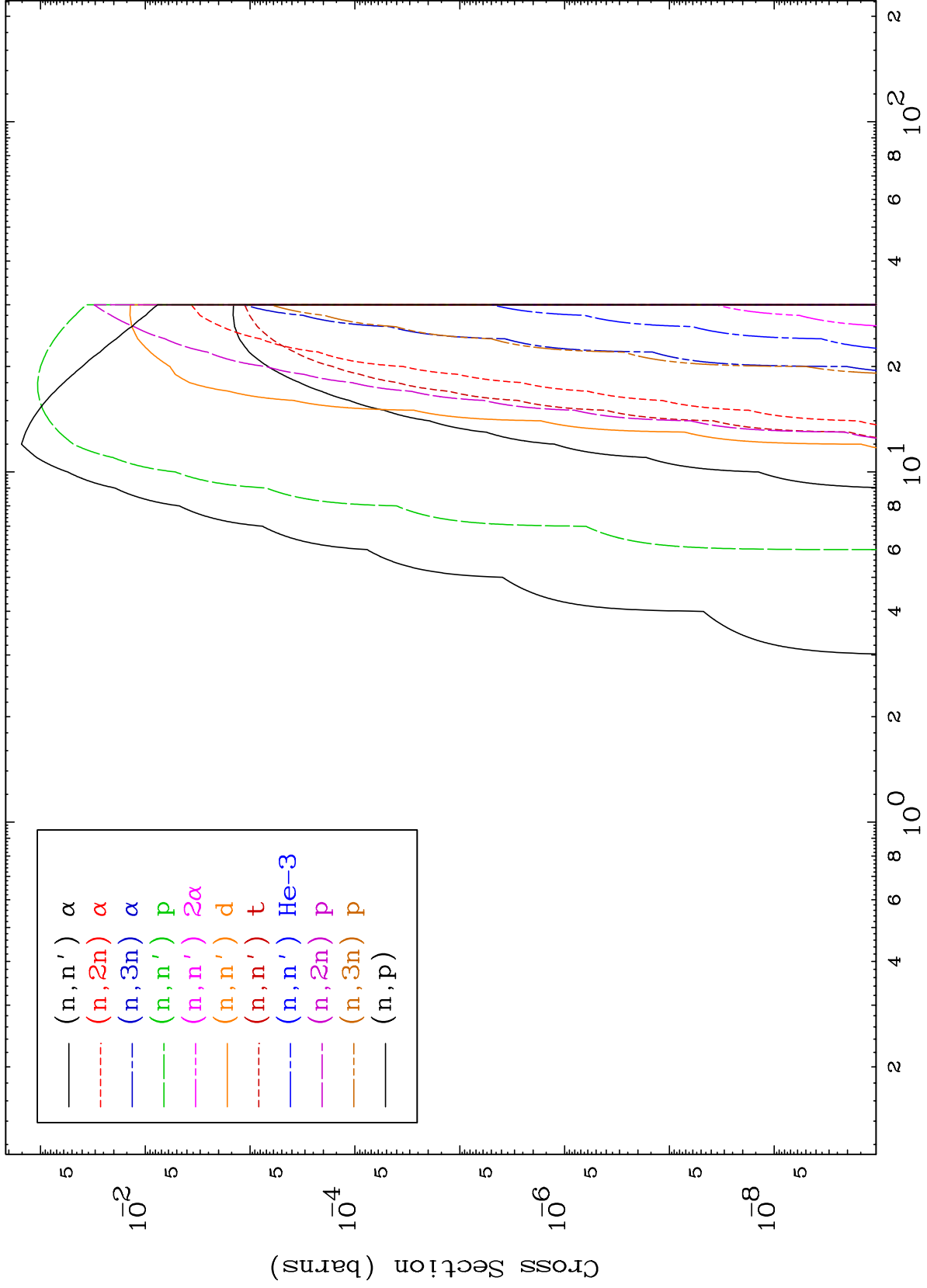
0 Kelvin Cross Sections

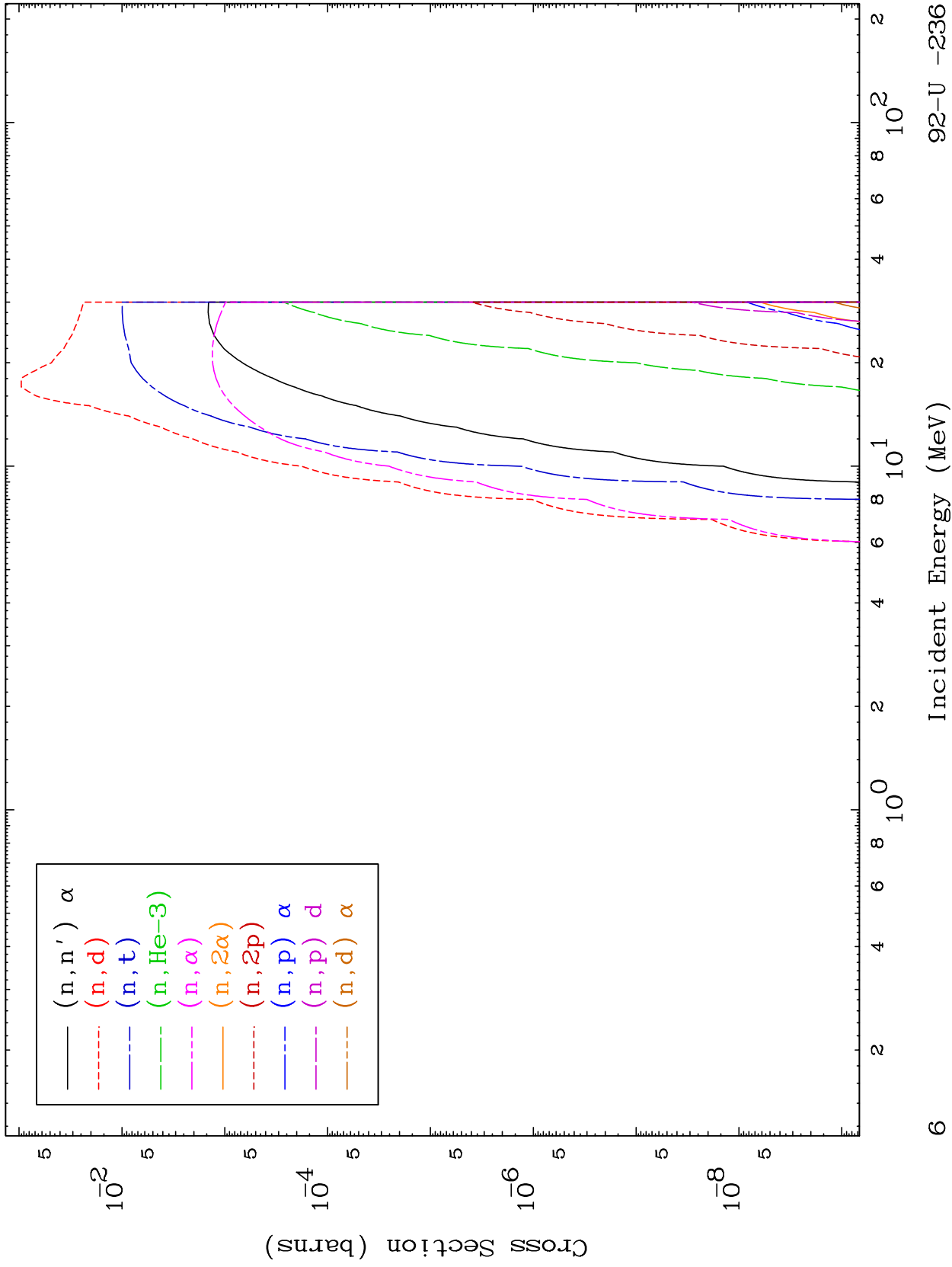










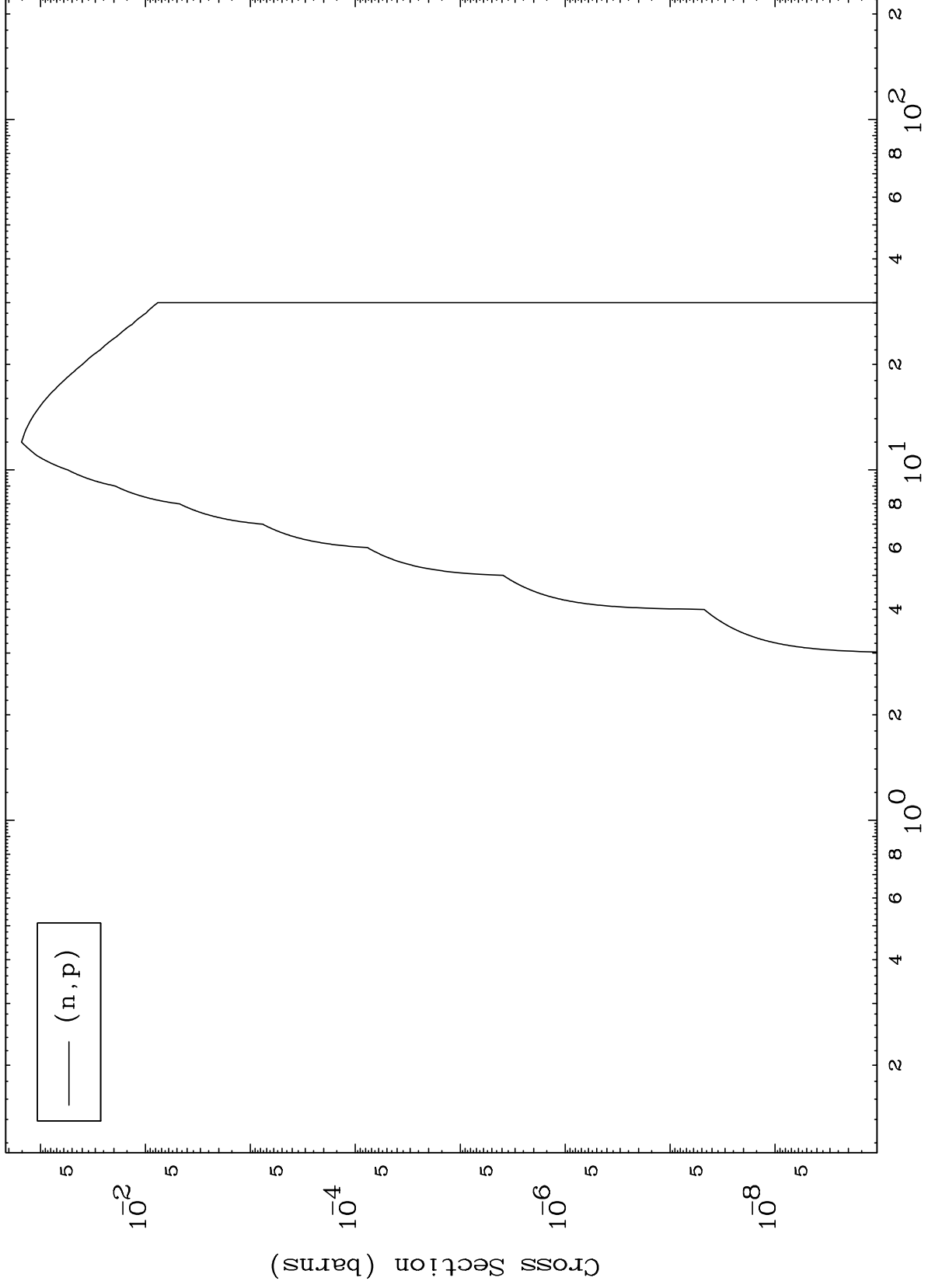


MAT 9231

(d,p) Levels

92-U -236

0 Kelvin Cross Sections



(n,p)

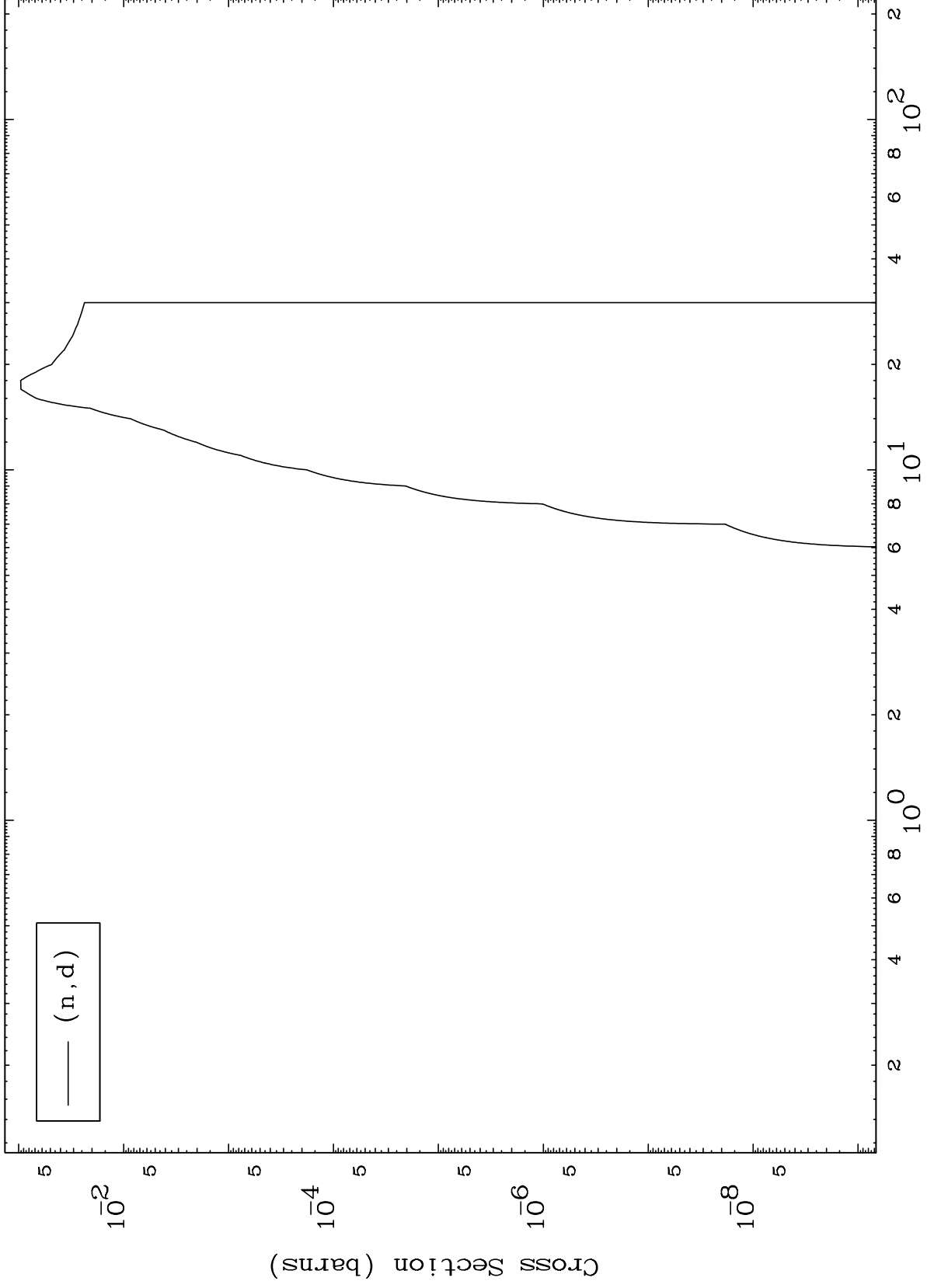


MAT 9231

(d,d) Levels

92-U -236

0 Kelvin Cross Sections

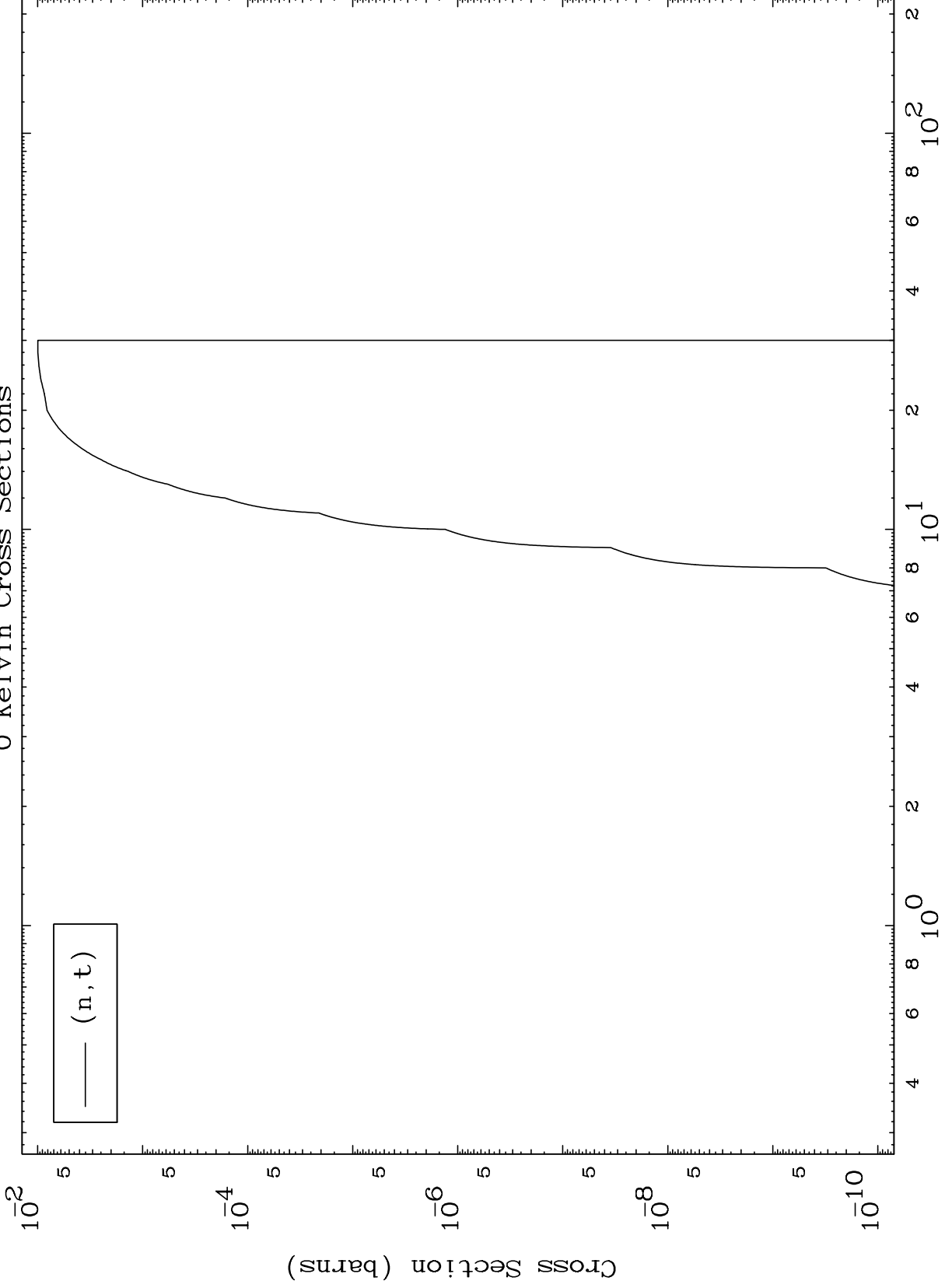


MAT 9231

(d,t) Levels

92-U -236

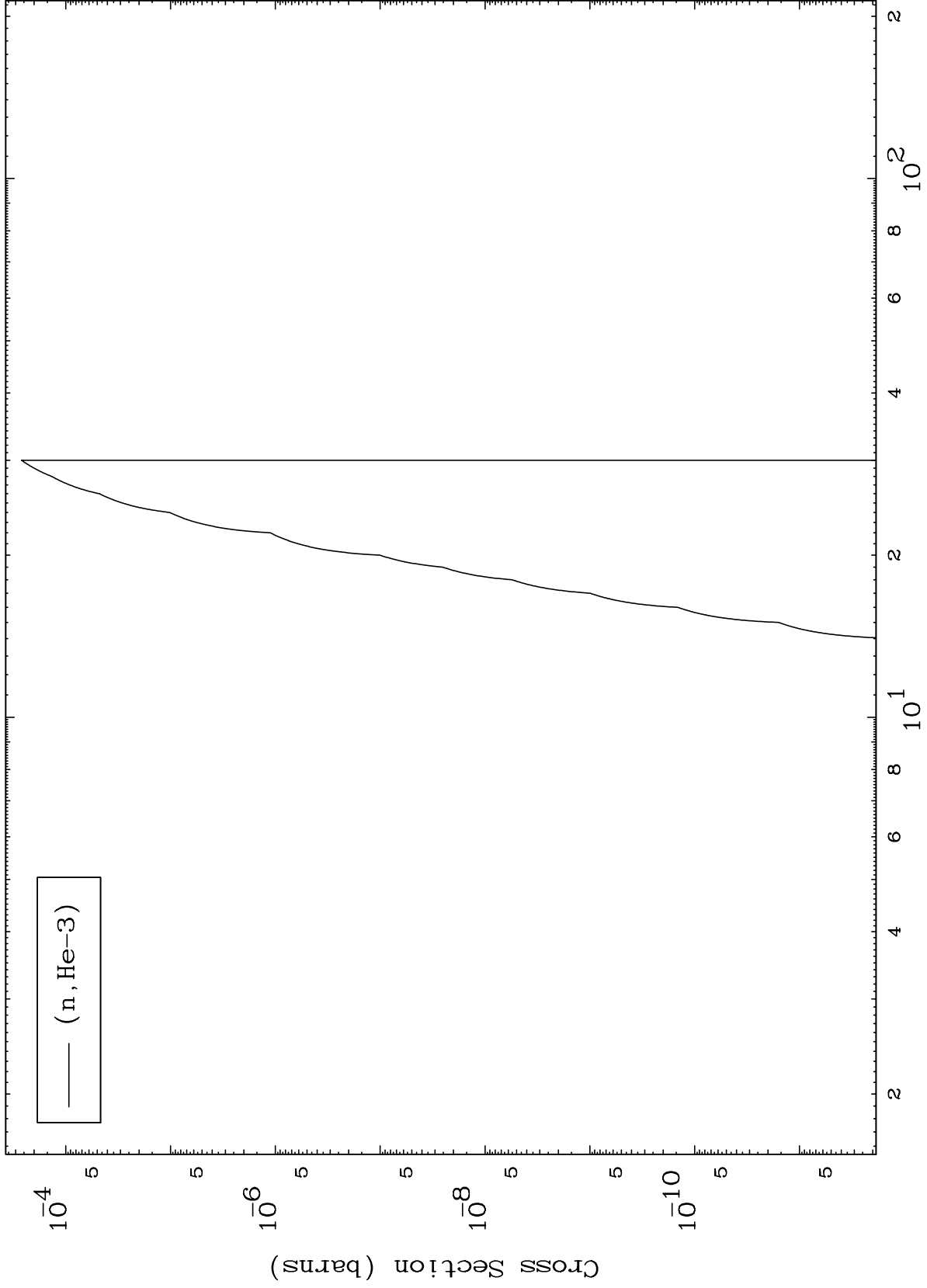
0 Kelvin Cross Sections



MAT 9231

(d,He3) Levels  
0 Kelvin Cross Sections

92-U -236



10

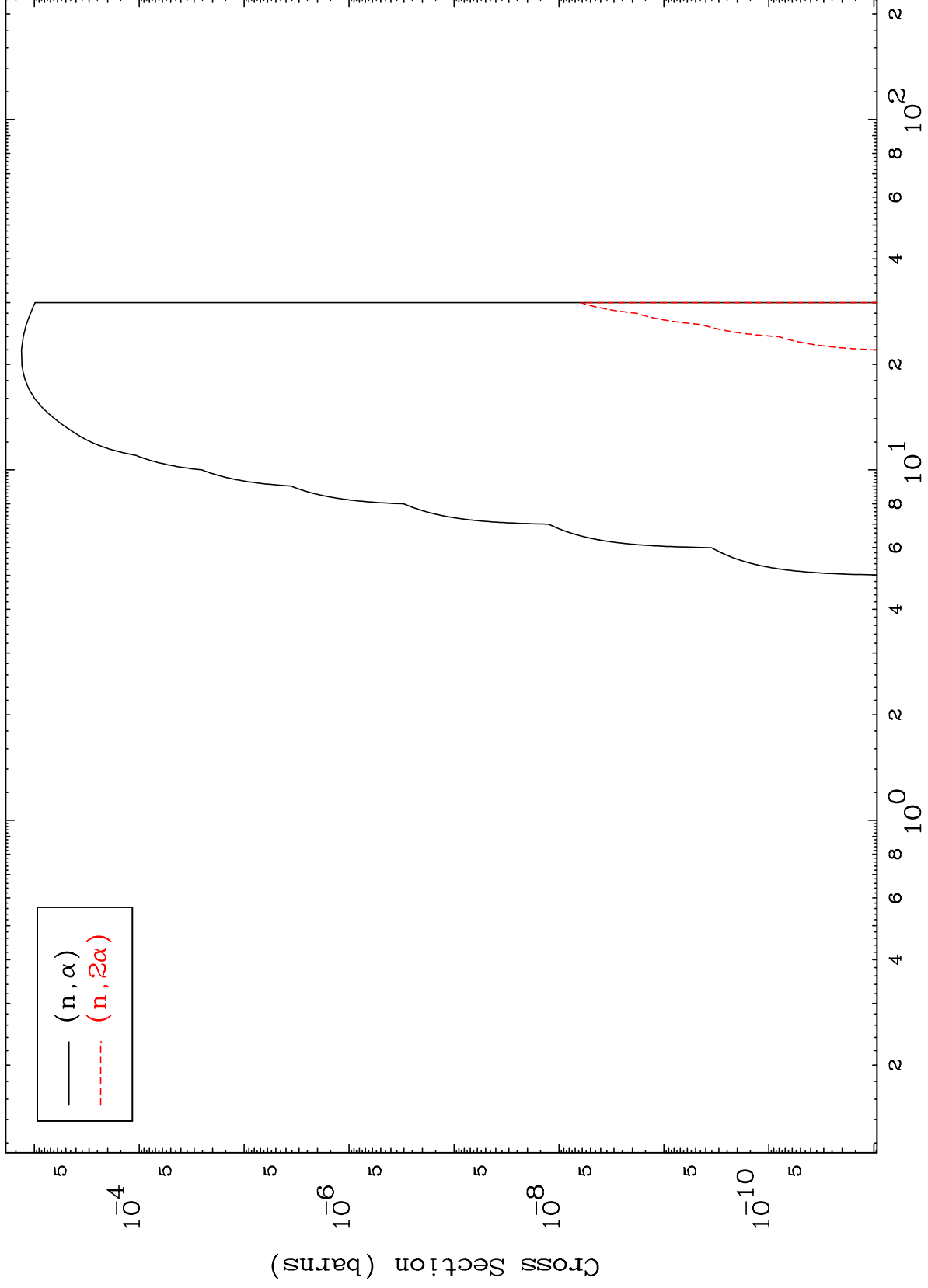
Incident Energy (MeV)

92-U -236

MAT 9231

(d,  $\alpha$ ) Levels  
0 Kelvin Cross Sections

92-U -236

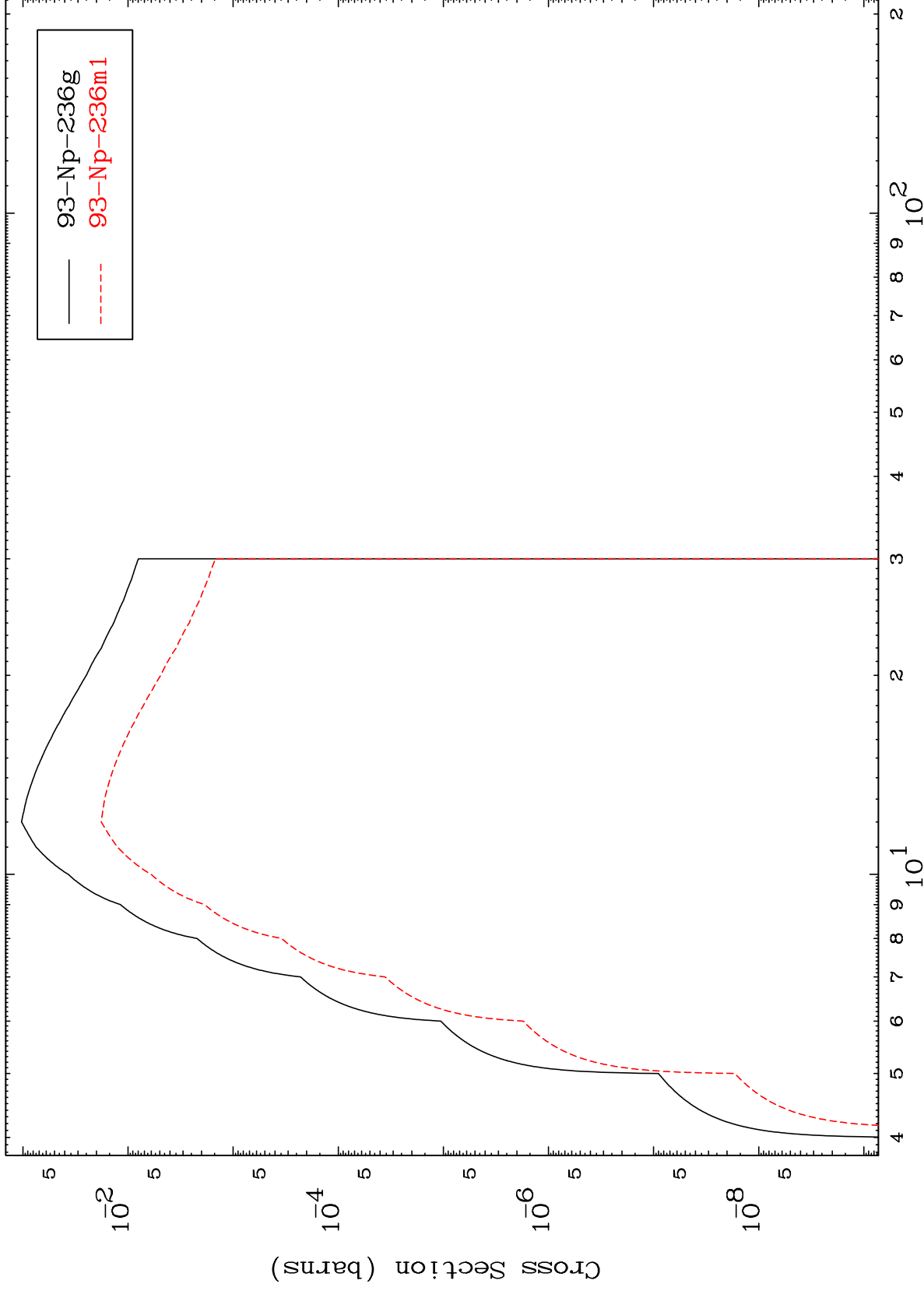


92-U -236

MAT 9231

92-U -236

(n,2n)  
Radionuclide Production Cross Section



92-U -236

Incident Energy (MeV)

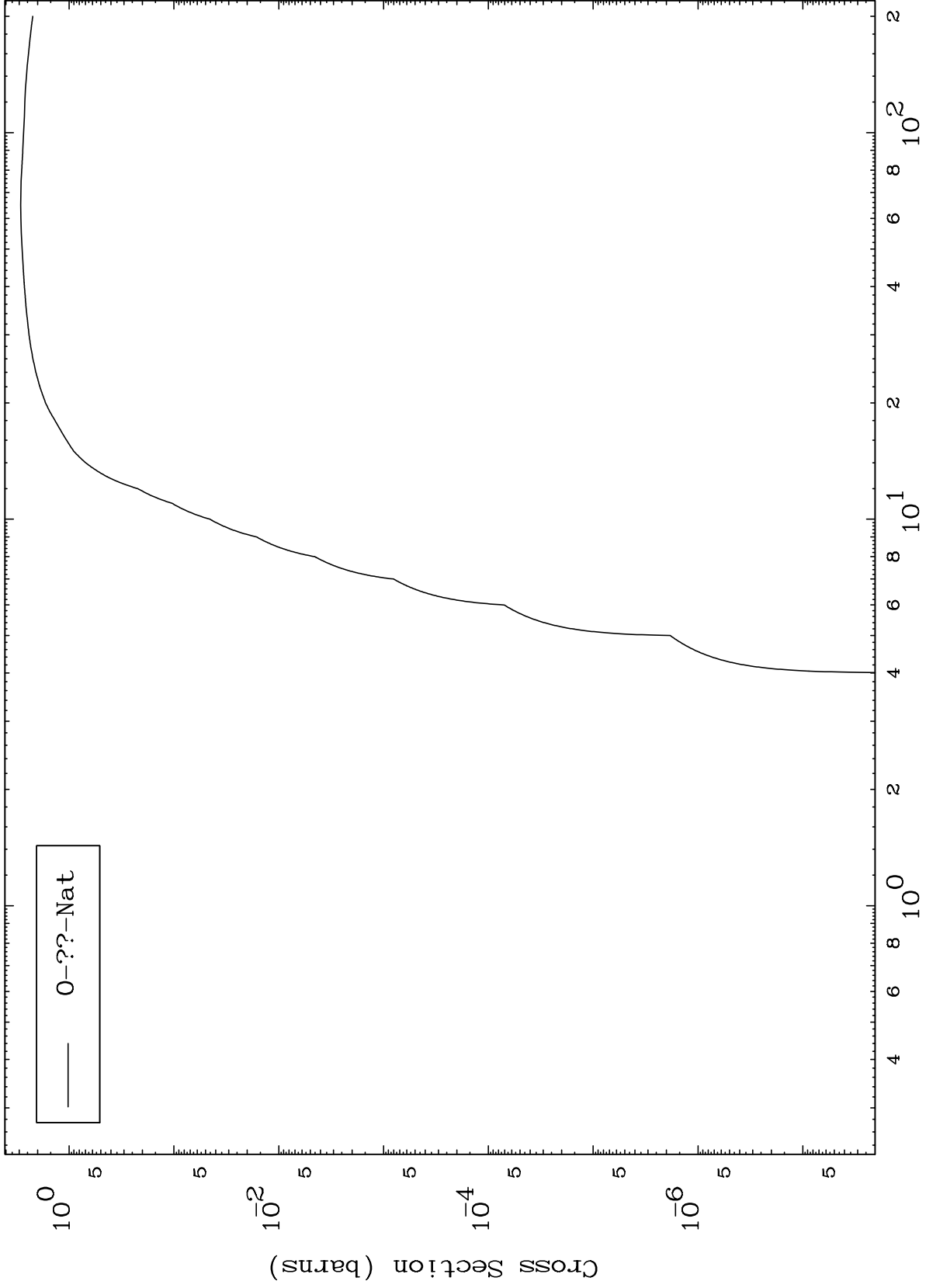
12

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92-U -236

Fission  
Radionuclide Production Cross Section

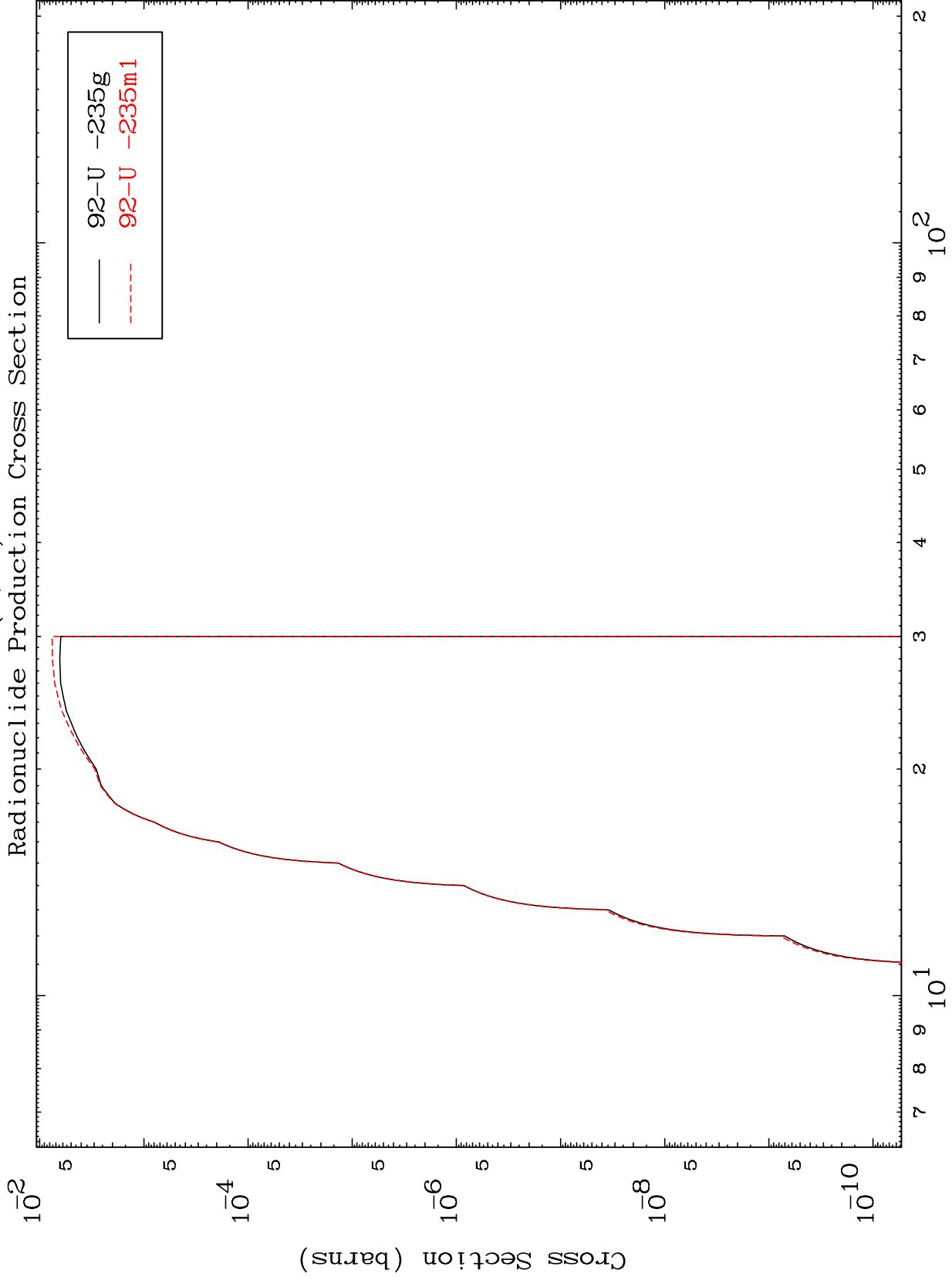
— 0-??-Nat



MAT 9231

(n,n') d

92-U -236



14

92-U -236

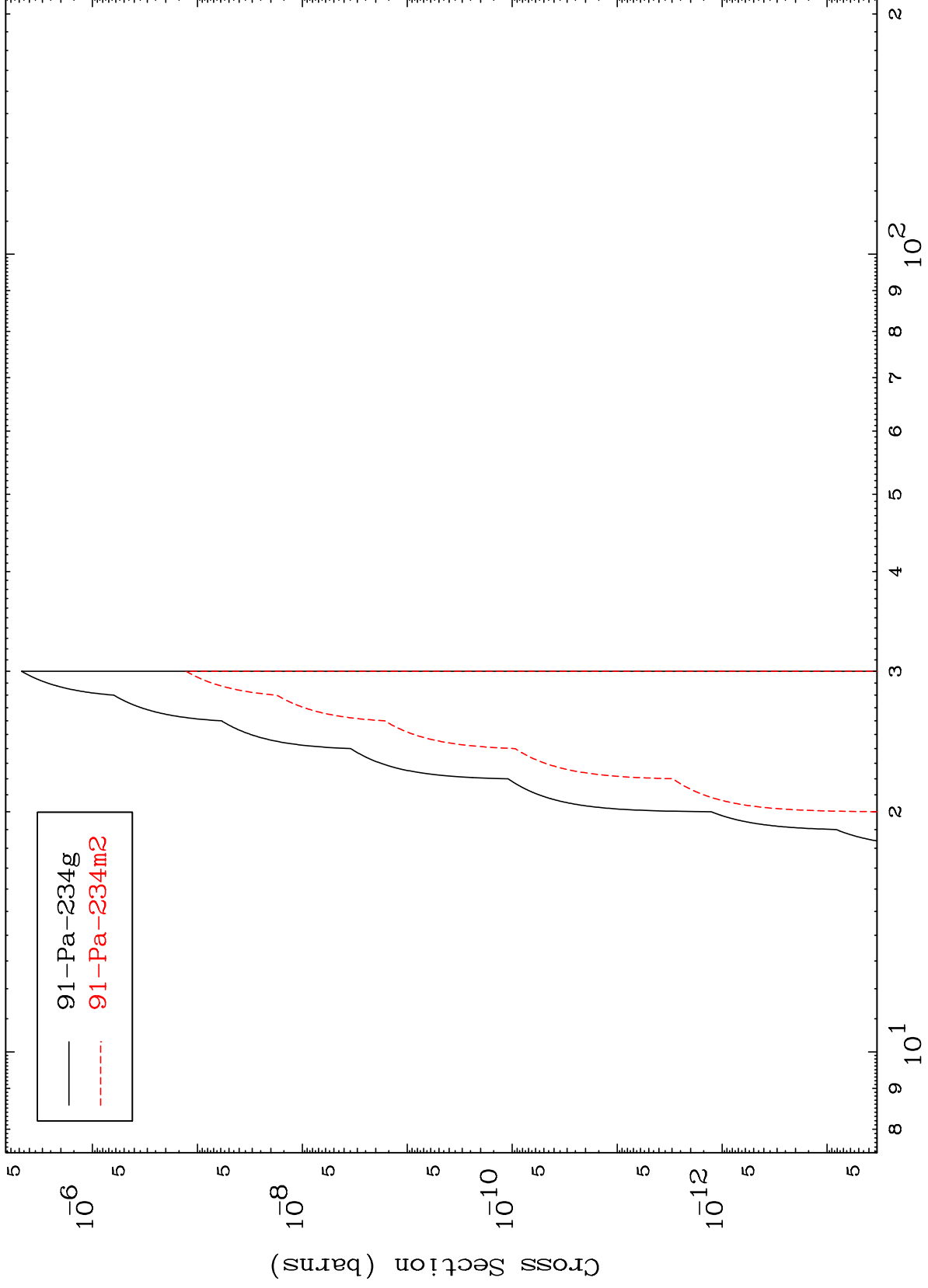
92-U -236

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(n,n') He-3

92-U -236

Radionuclide Production Cross Section



91-Pa-234g  
91-Pa-234m2

15

Incident Energy (MeV)

92-U -236

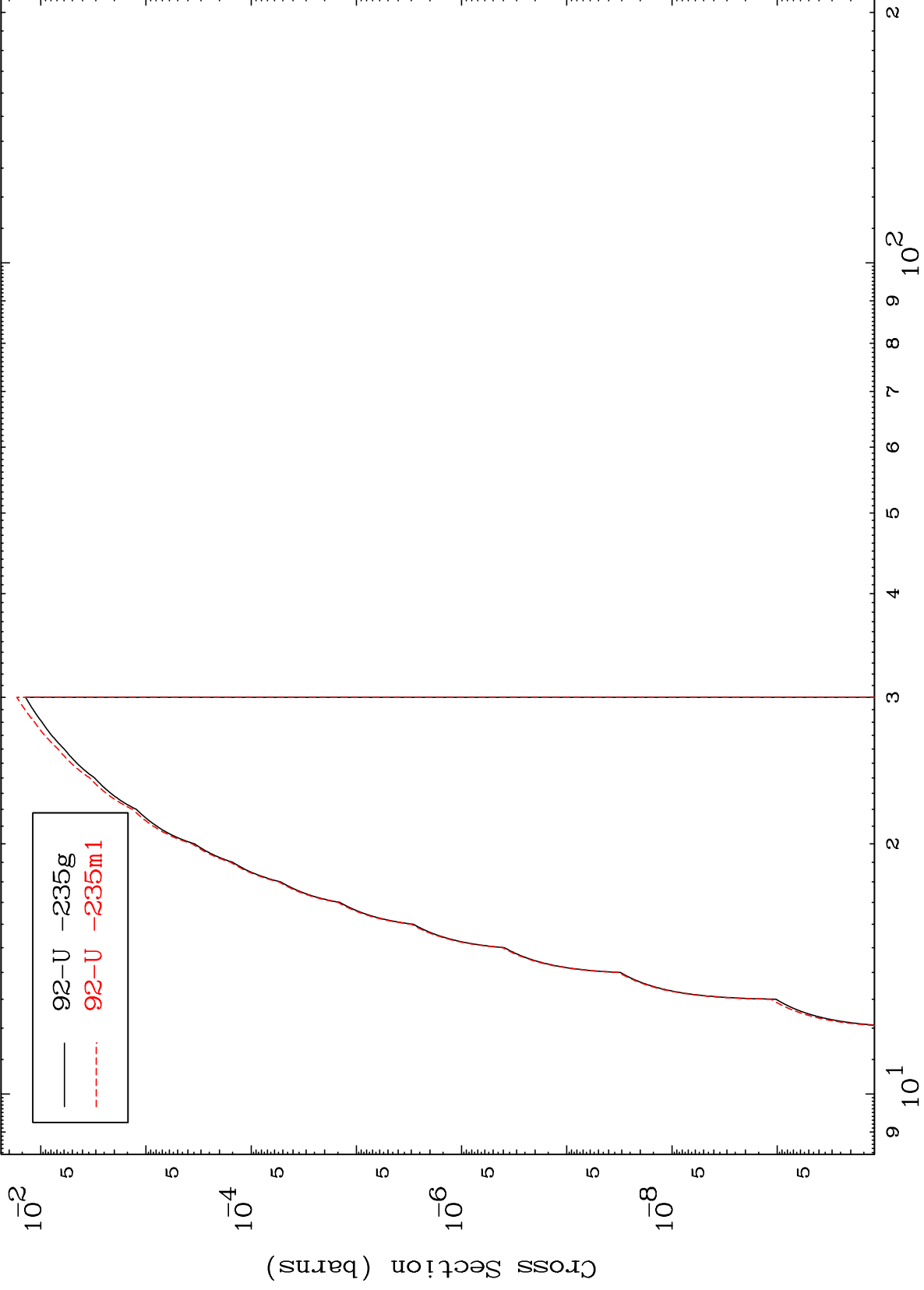


MAT 9231

(n,2n) p

92-U -236

Radionuclide Production Cross Section



16

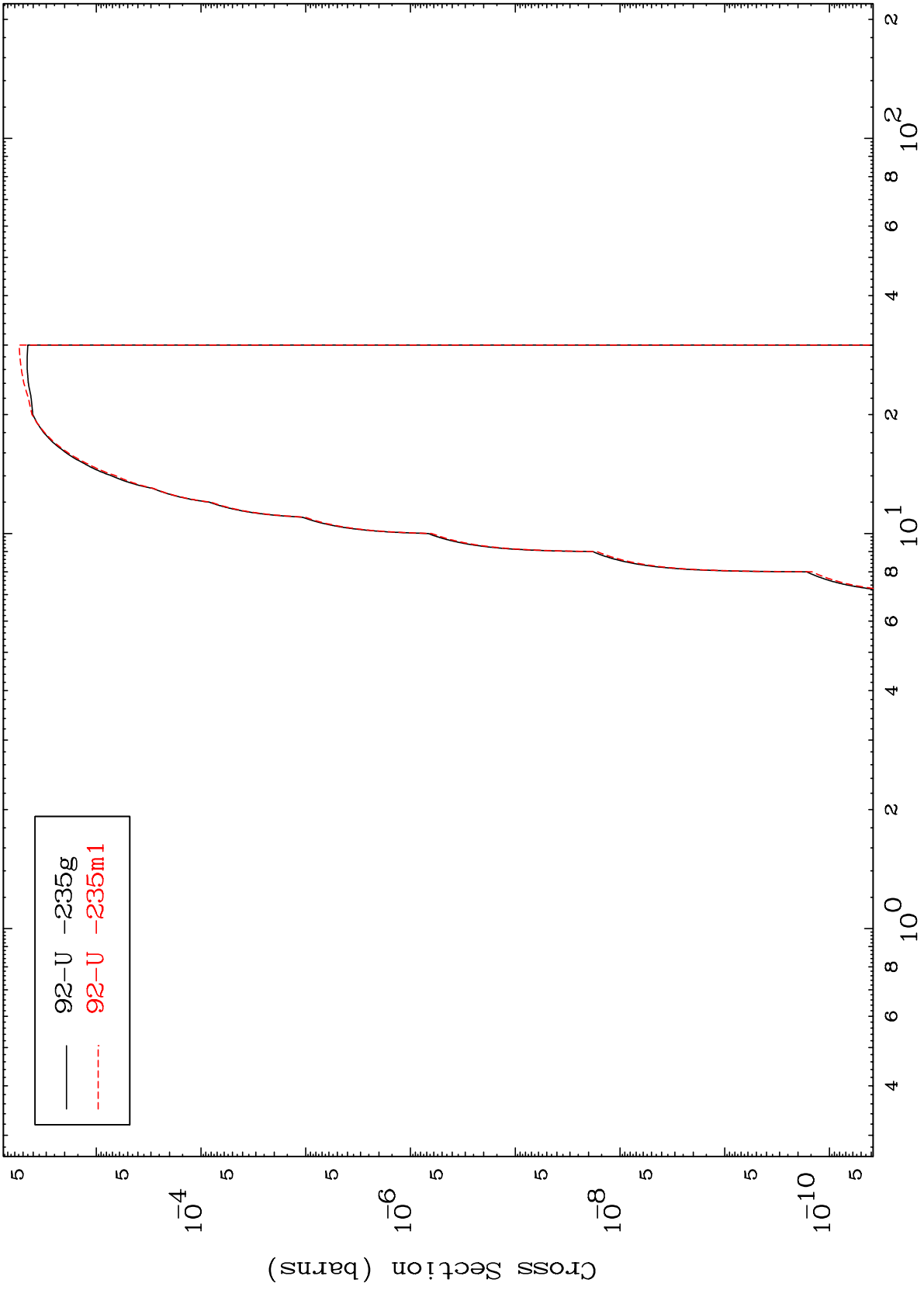
Incident Energy (MeV)

92-U -236

MAT 9231

92-U -236

(n,t)  
Radionuclide Production Cross Section



17

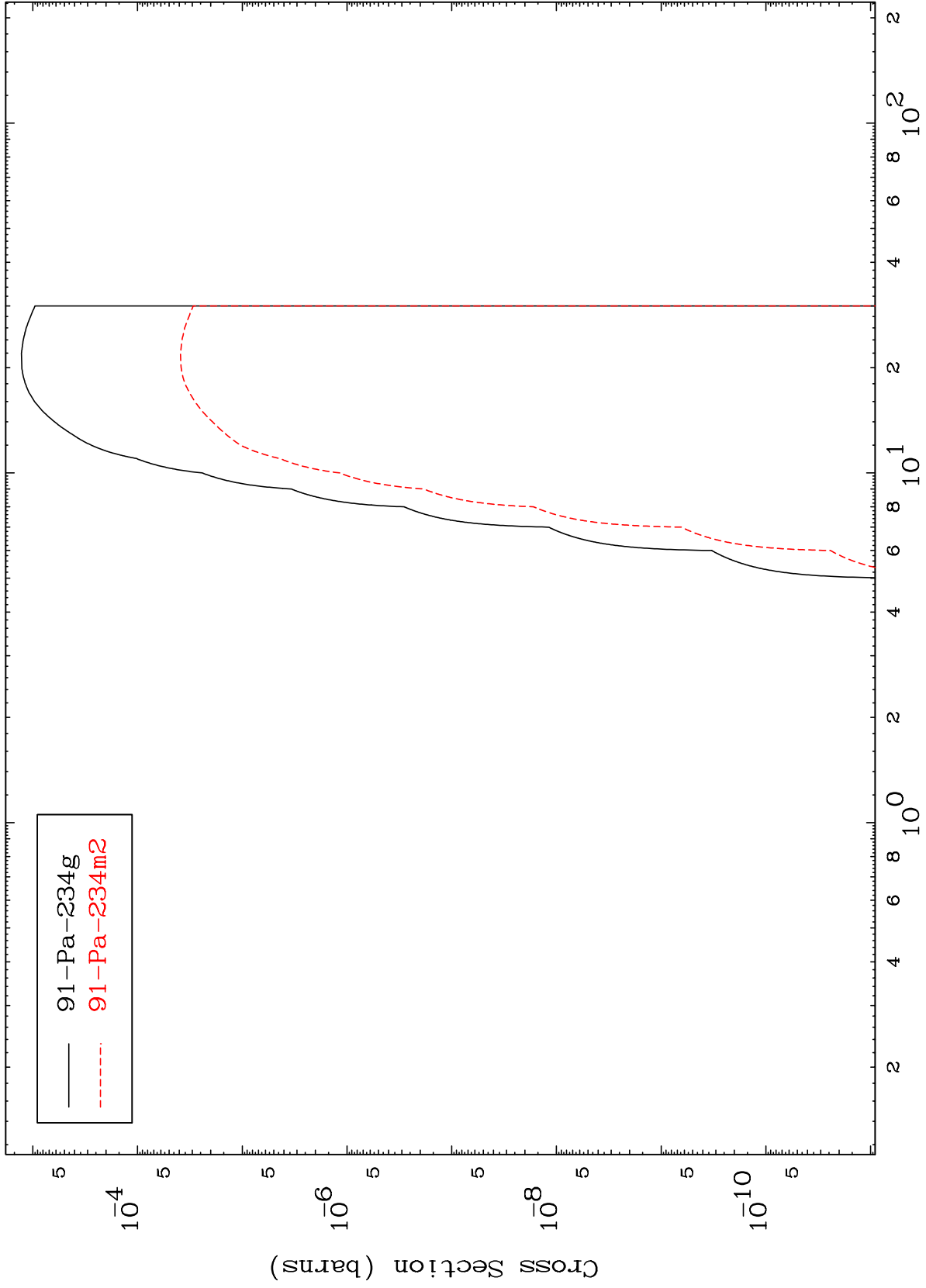
92-U -236

Incident Energy (MeV)

MAT 9231

92-U -236

(n,  $\alpha$ )  
Radionuclide Production Cross Section



18

92-U -236

Incident Energy (MeV)