Program EVALPLOT
(Version 2021-1)
by
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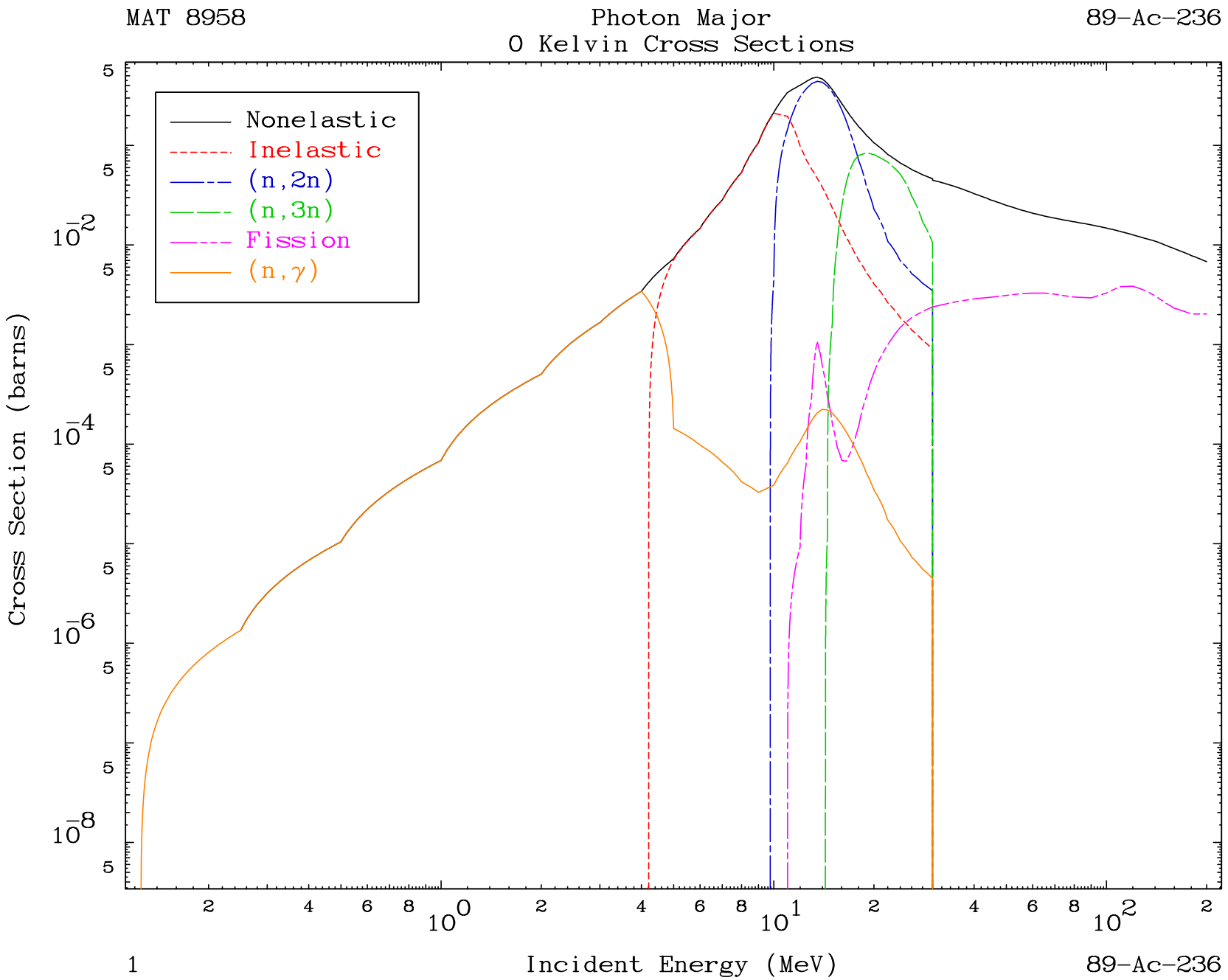
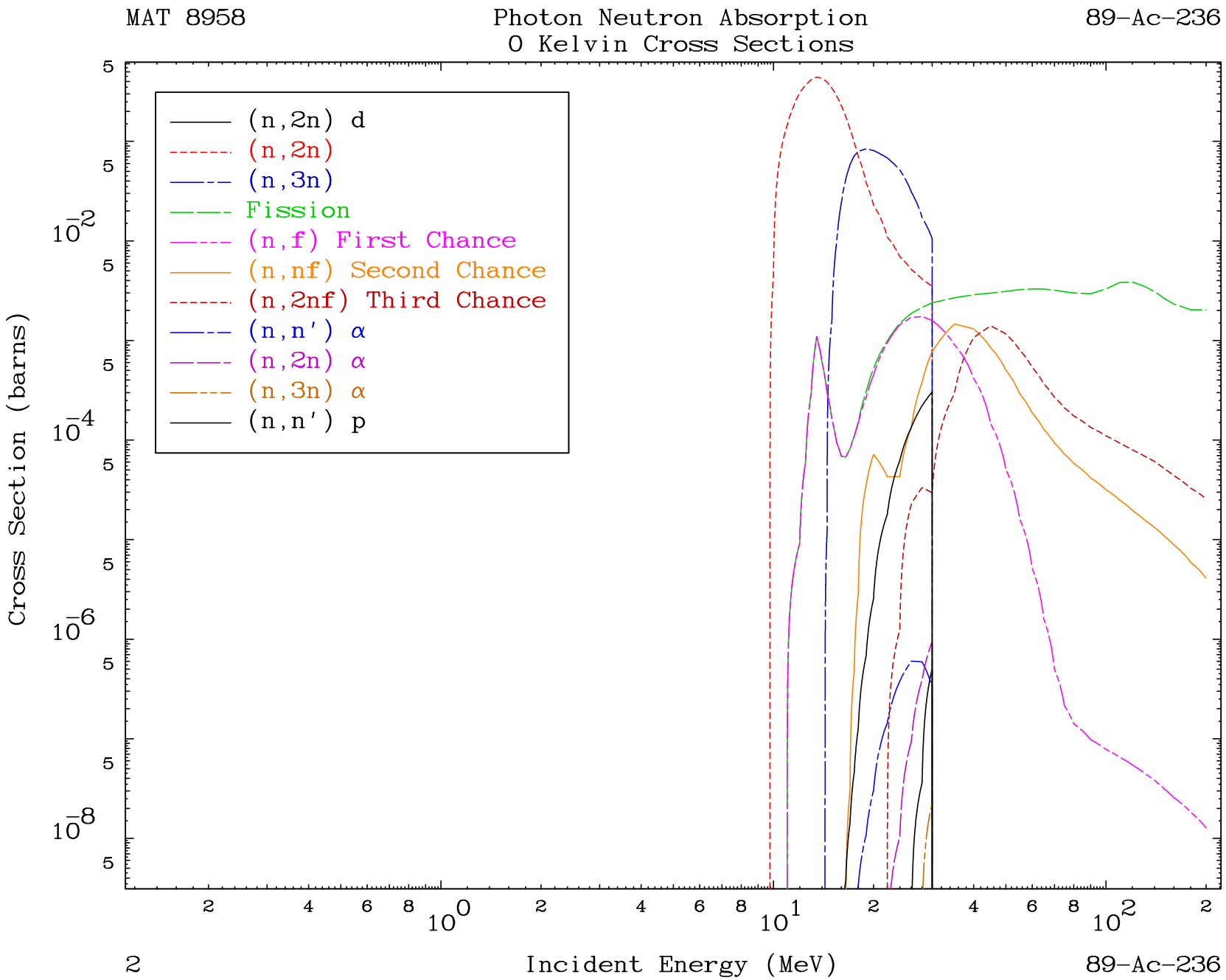
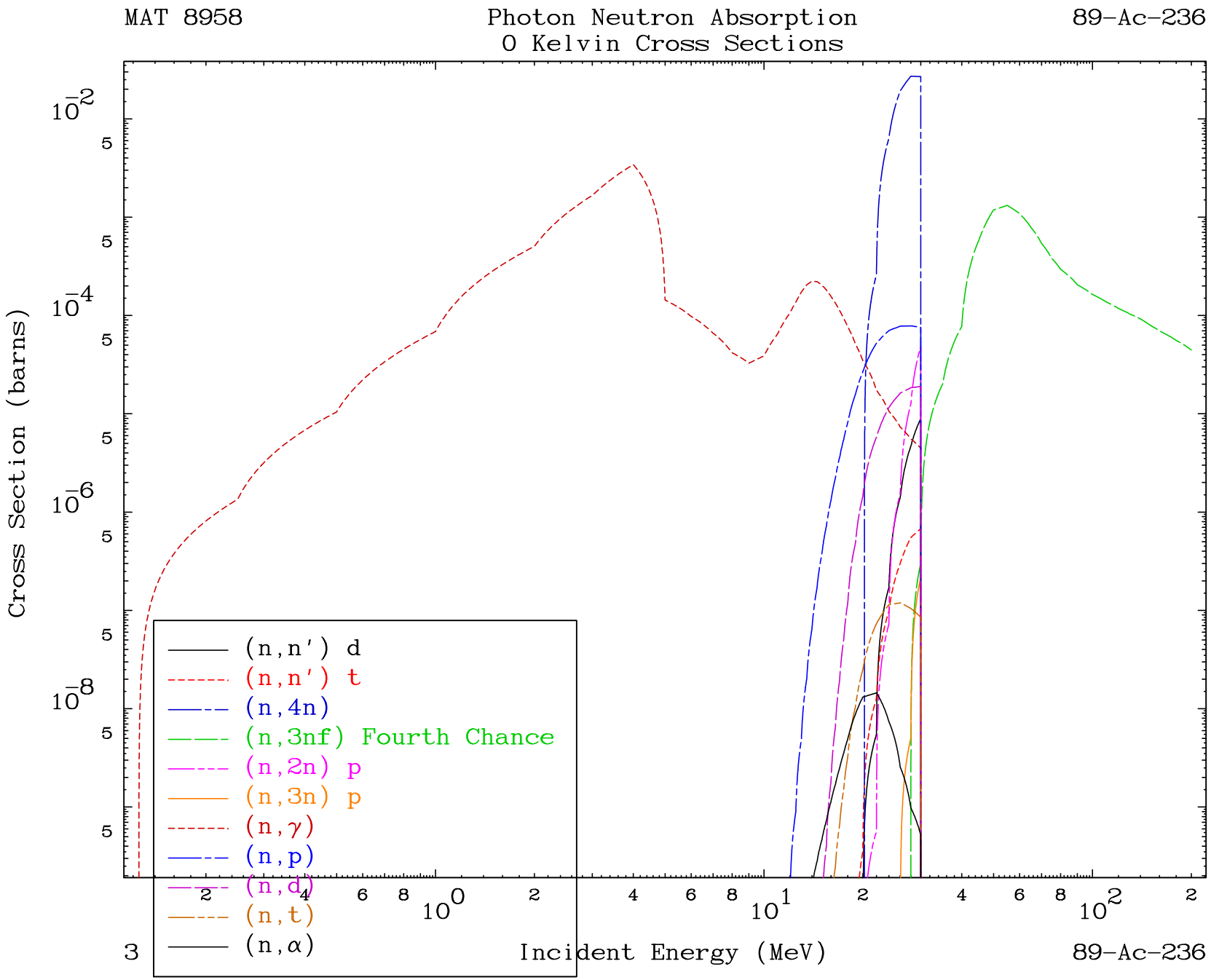
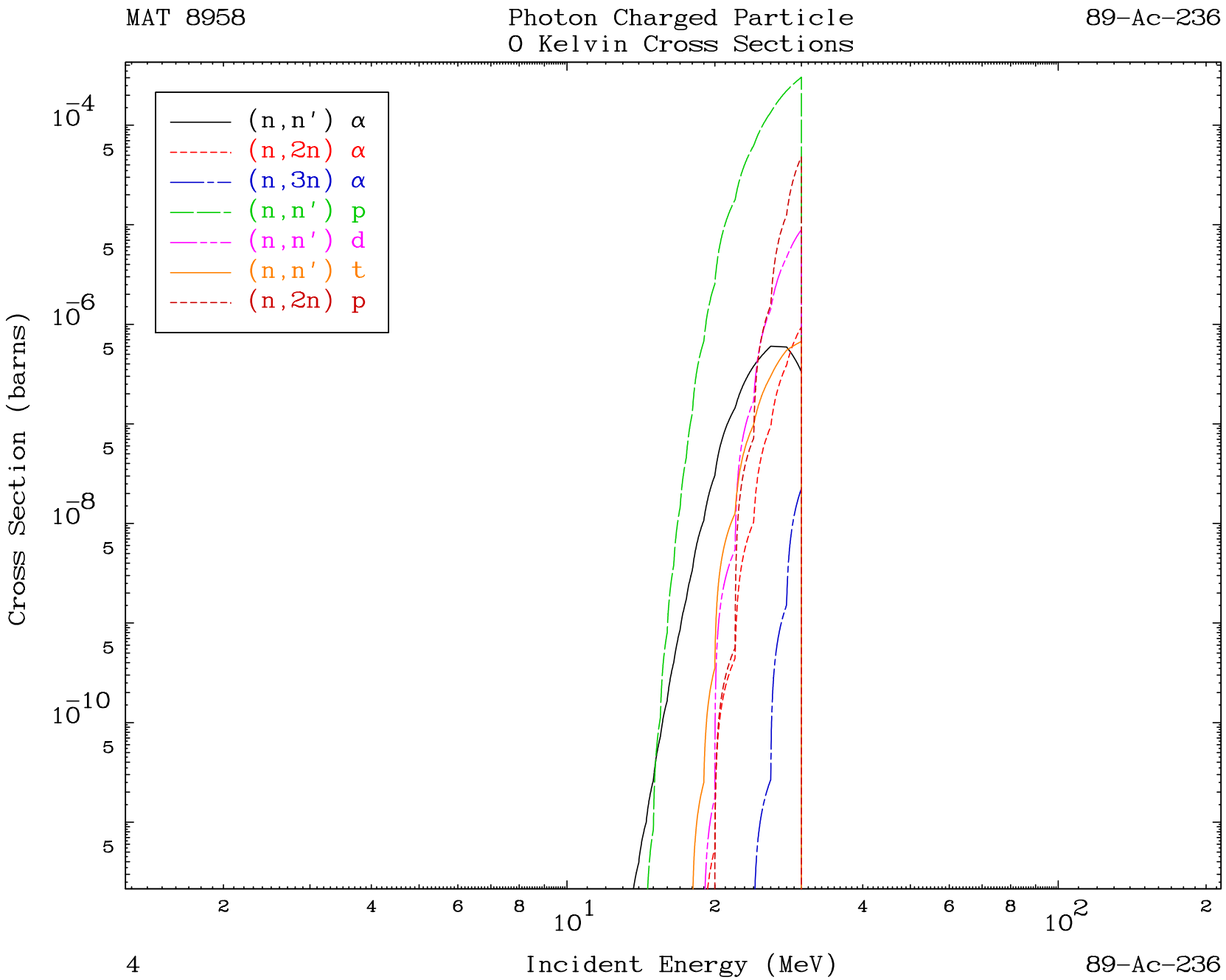
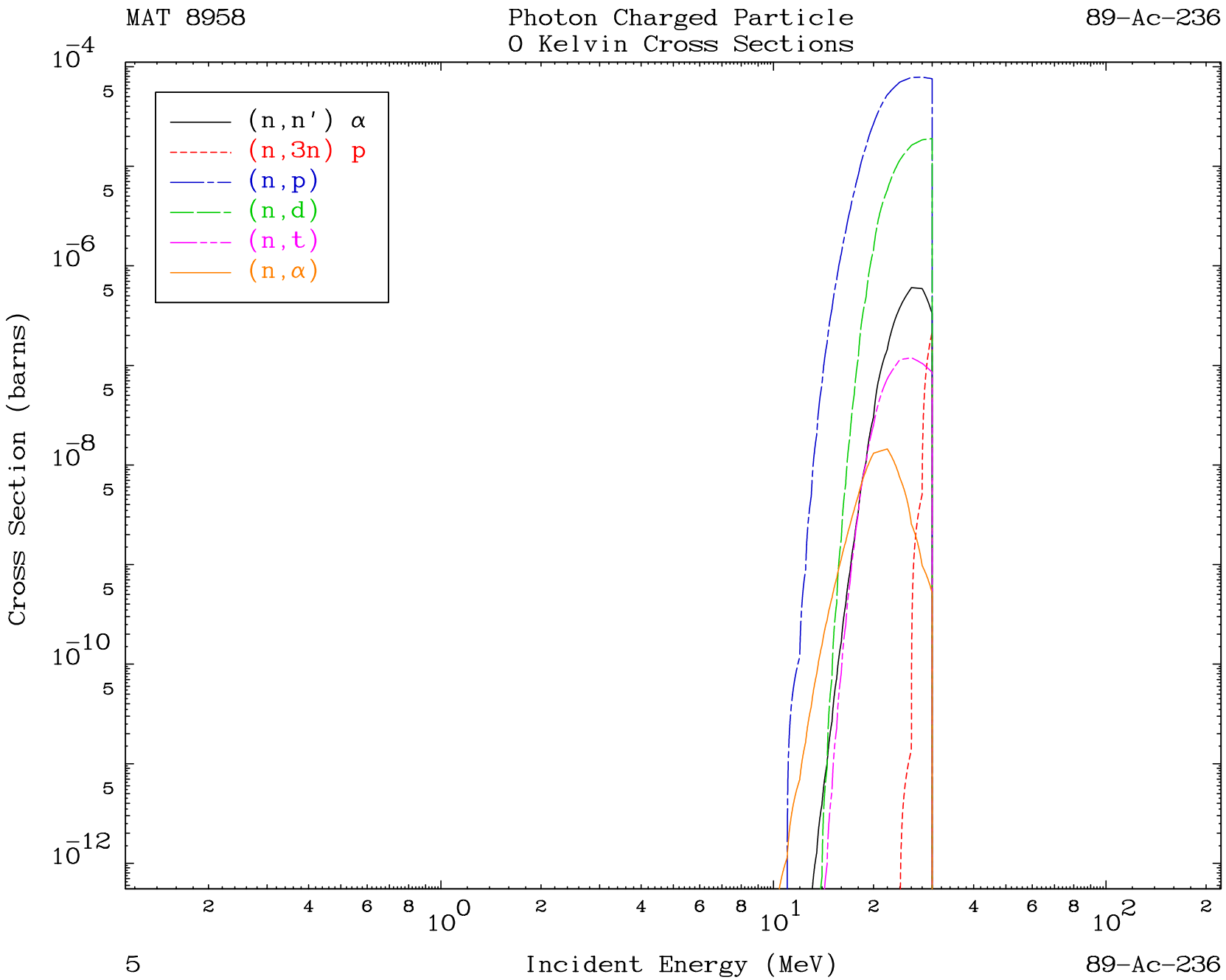
Press Mouse Button to Start

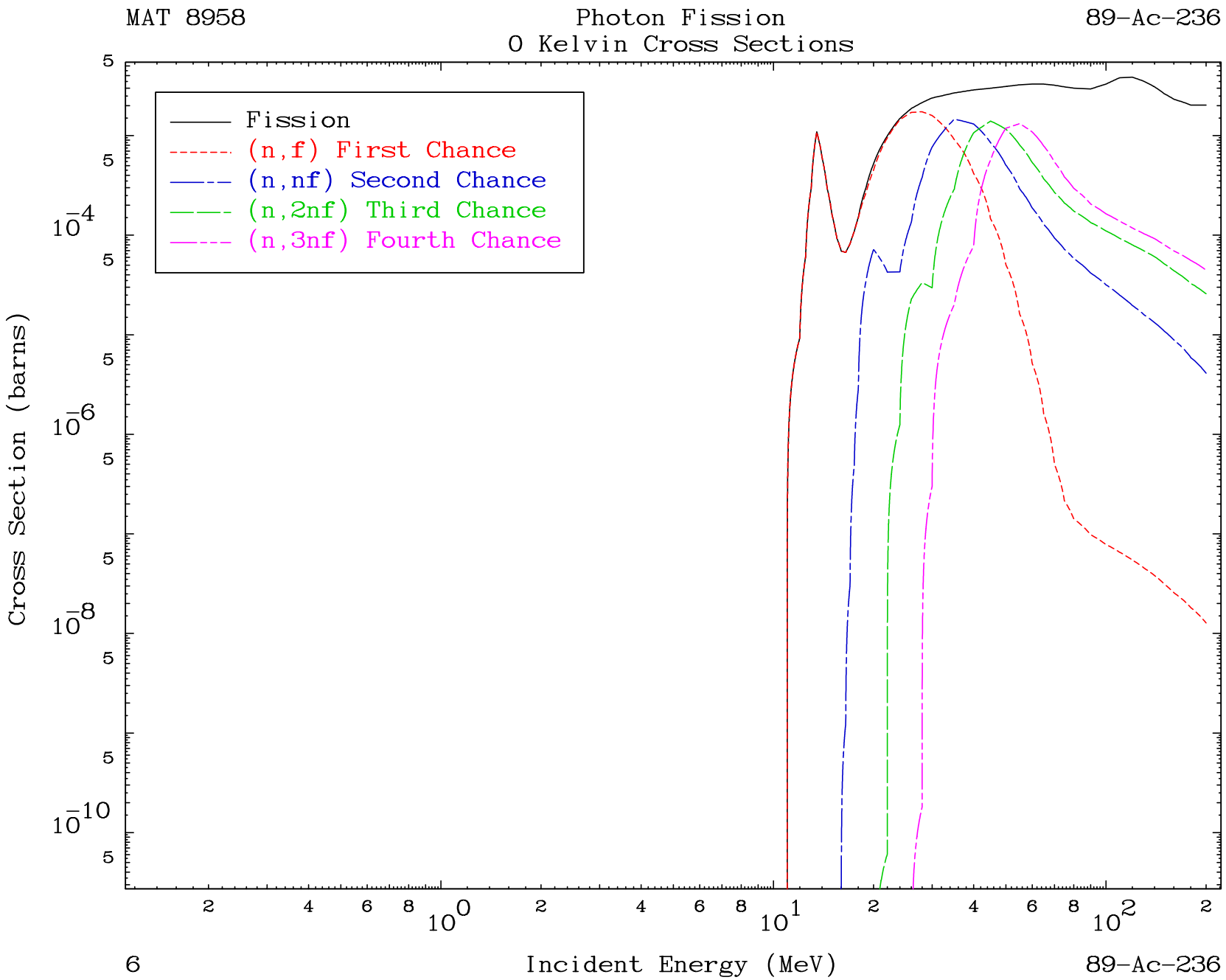

MAT 8958
Photon Major
89-Ac-236
0 Kelvin Cross Sections
Nonelastic
Inelastic
(n,2n)
(n,3n)
Fission
(n,γ)
Cross Section (barns)
Incident Energy (MeV)
89-Ac-236
1

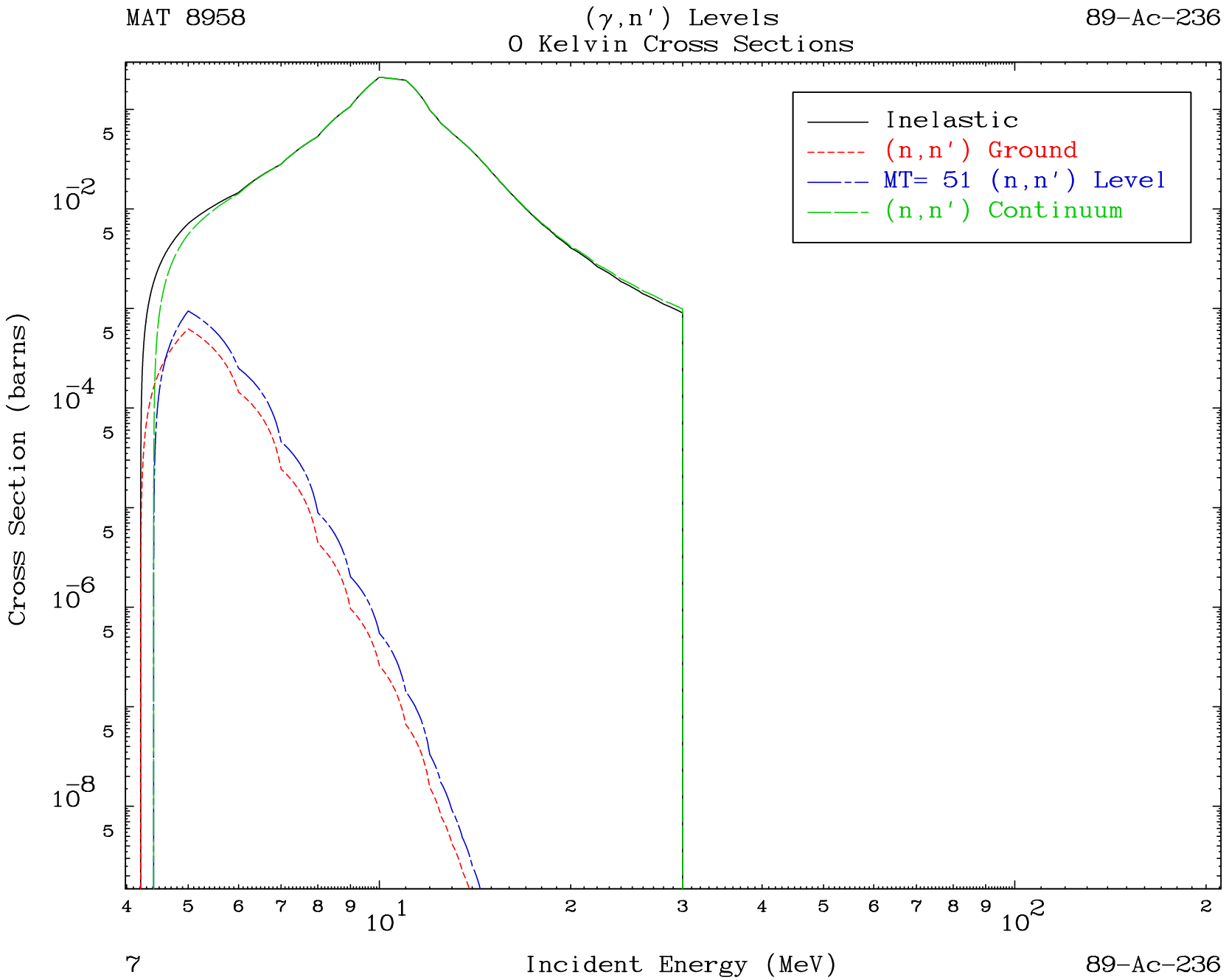

MAT 8958
Photon Neutron Absorption
0 Kelvin Cross Sections
89-Ac-236
(n,2n) d
(n,2n)
(n,3n)
Fission
(n,f) First Chance
(n,nf) Second Chance
(n,2nf) Third Chance
(n,n') α
(n,2n) α
(n,3n) α
(n,n') p
Cross Section (barns)
Incident Energy (MeV)
89-Ac-236

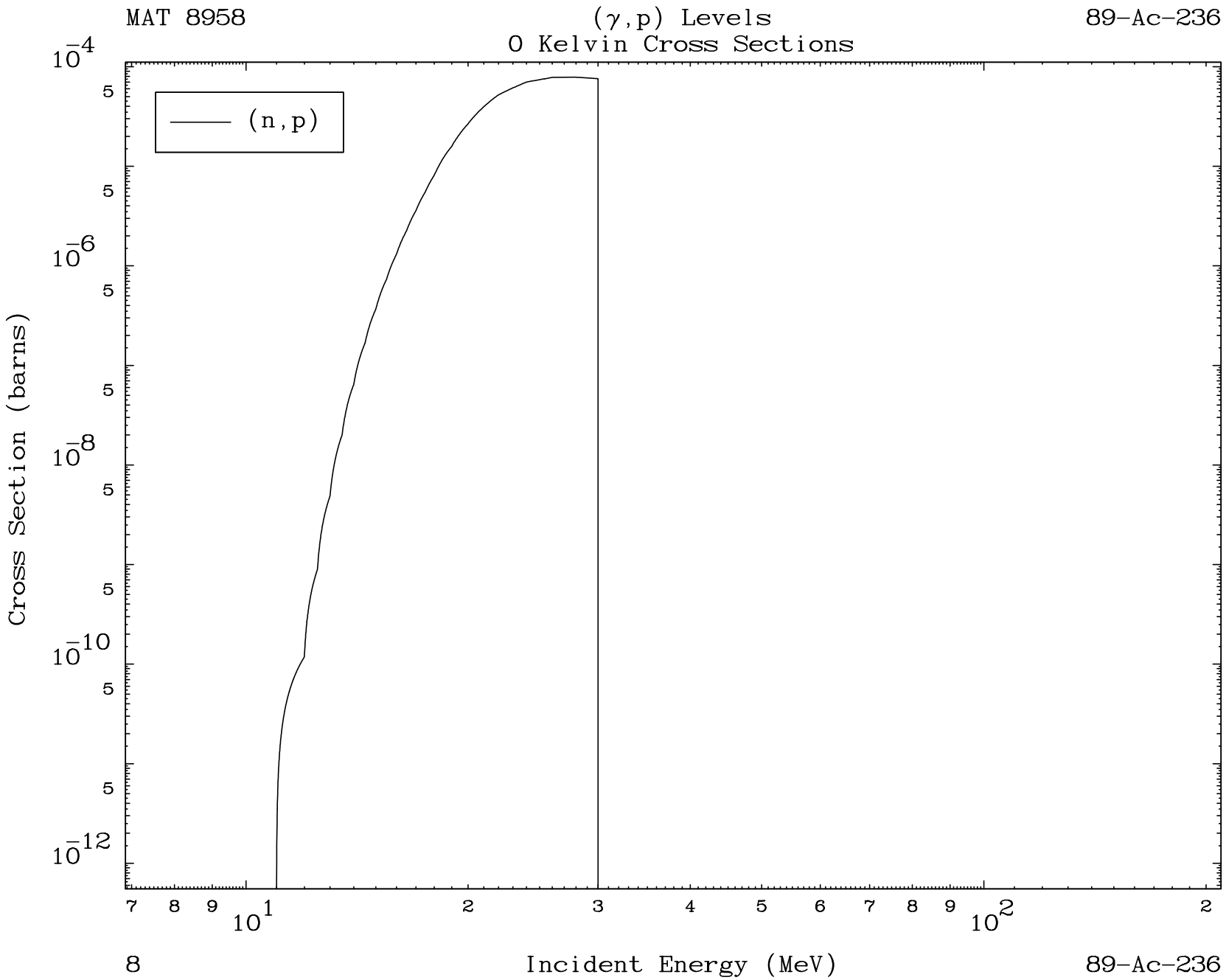

MAT 8958
Photon Neutron Absorption
0 Kelvin Cross Sections
89-Ac-236
Cross Section (barns)
Incident Energy (MeV)
89-Ac-236
(n,n') d
(n,n') t
(n,4n)
(n,3nf) Fourth Chance
(n,2n) p
(n,3n) p
(n,γ)
(n,p)
(n,d)
(n,t)
(n,α)

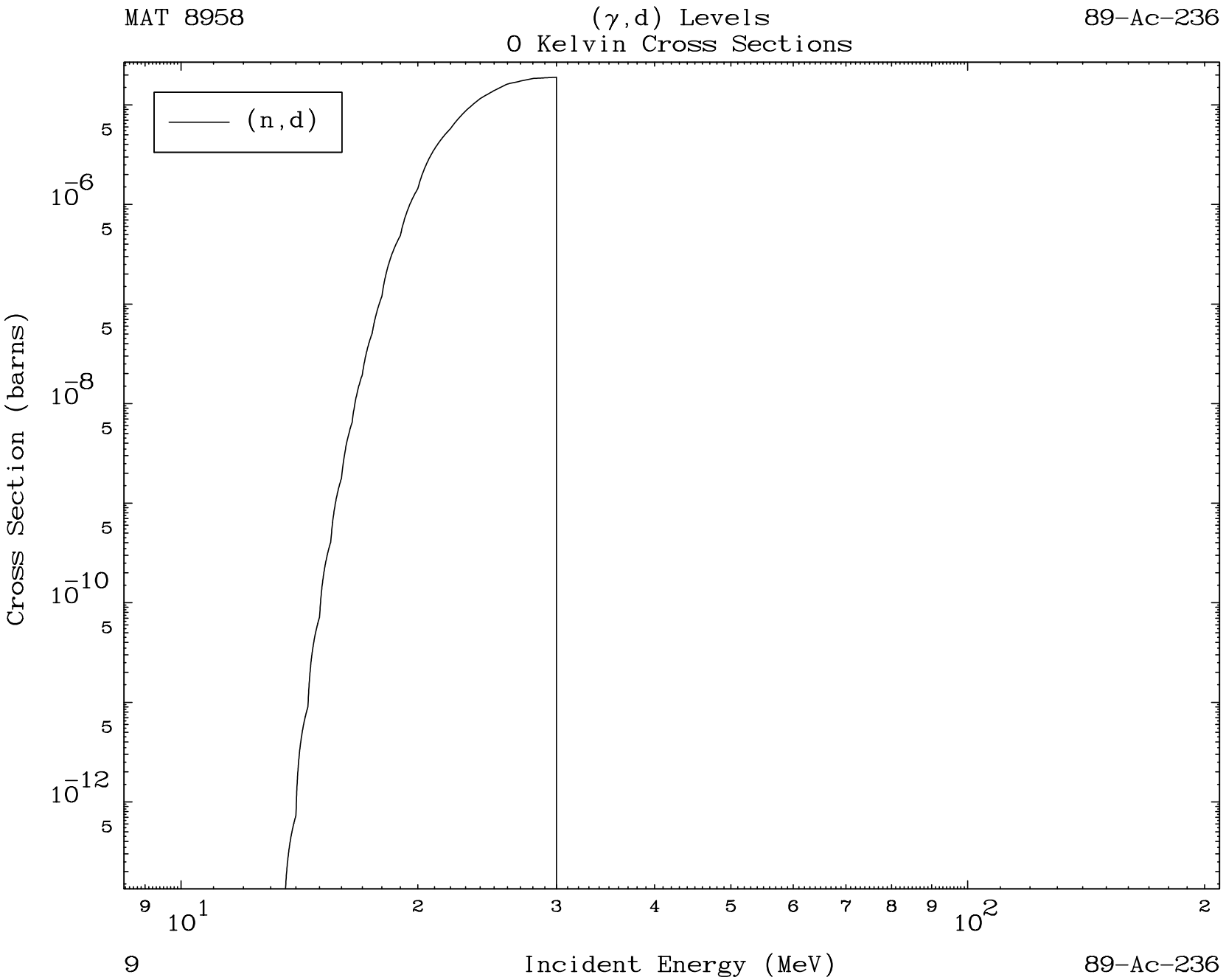

MAT 8958
Photon Charged Particle
0 Kelvin Cross Sections
89-Ac-236
(n,n') α
(n,2n) α
(n,3n) α
(n,n') p
(n,n') d
(n,n') t
(n,2n) p
Cross Section (barns)
10^-4
10^-6
10^-8
10^-10
Incident Energy (MeV)
10^1
10^2
4
89-Ac-236

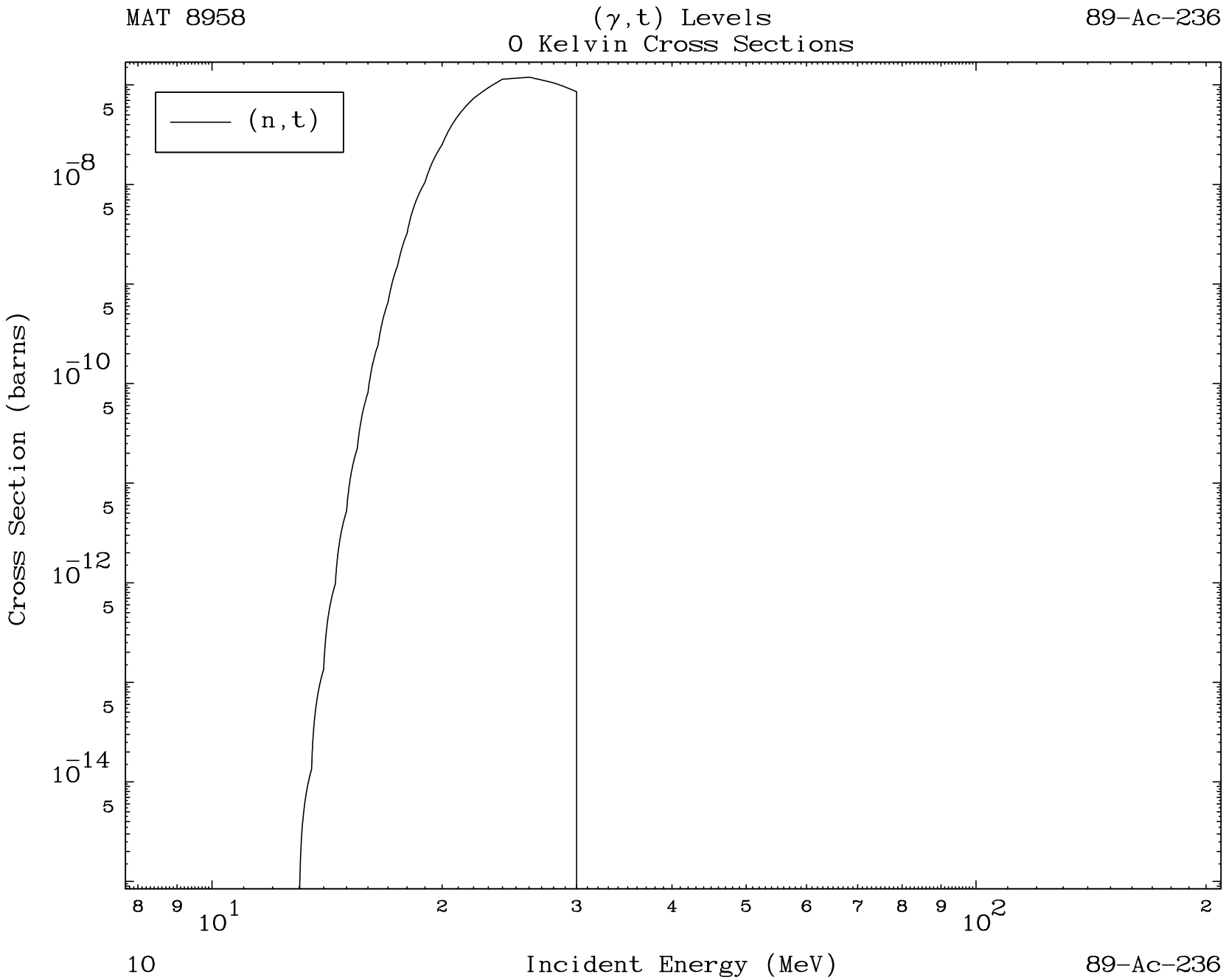

MAT 8958
Photon Charged Particle
0 Kelvin Cross Sections
89-Ac-236
(n,n') α
(n,3n) p
(n,p)
(n,d)
(n,t)
(n,α)
Cross Section (barns)
Incident Energy (MeV)
89-Ac-236
5

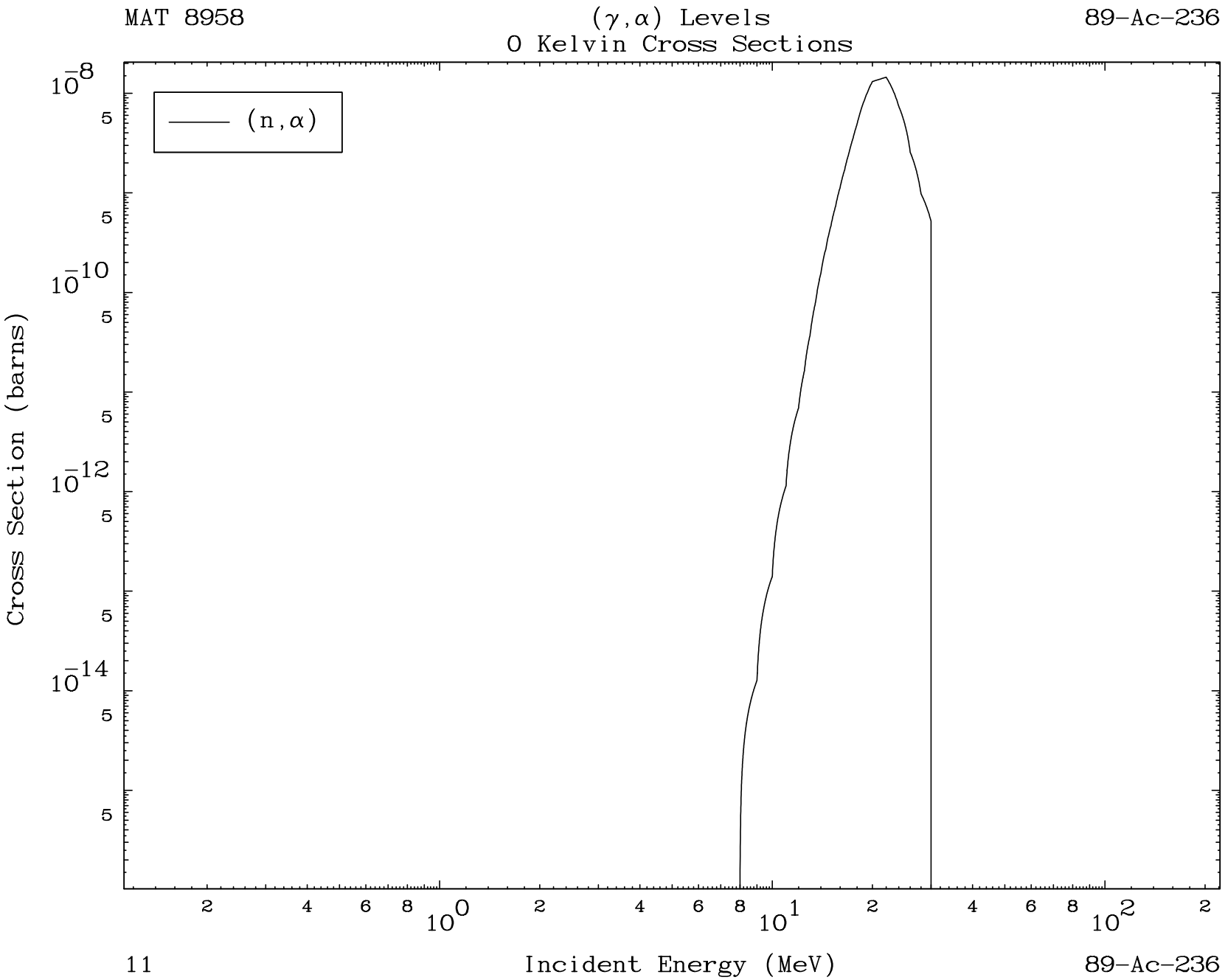

MAT 8958
Photon Fission
89-Ac-236
0 Kelvin Cross Sections
Fission
(n,f) First Chance
(n,nf) Second Chance
(n,2nf) Third Chance
(n,3nf) Fourth Chance
Cross Section (barns)
Incident Energy (MeV)
6
89-Ac-236

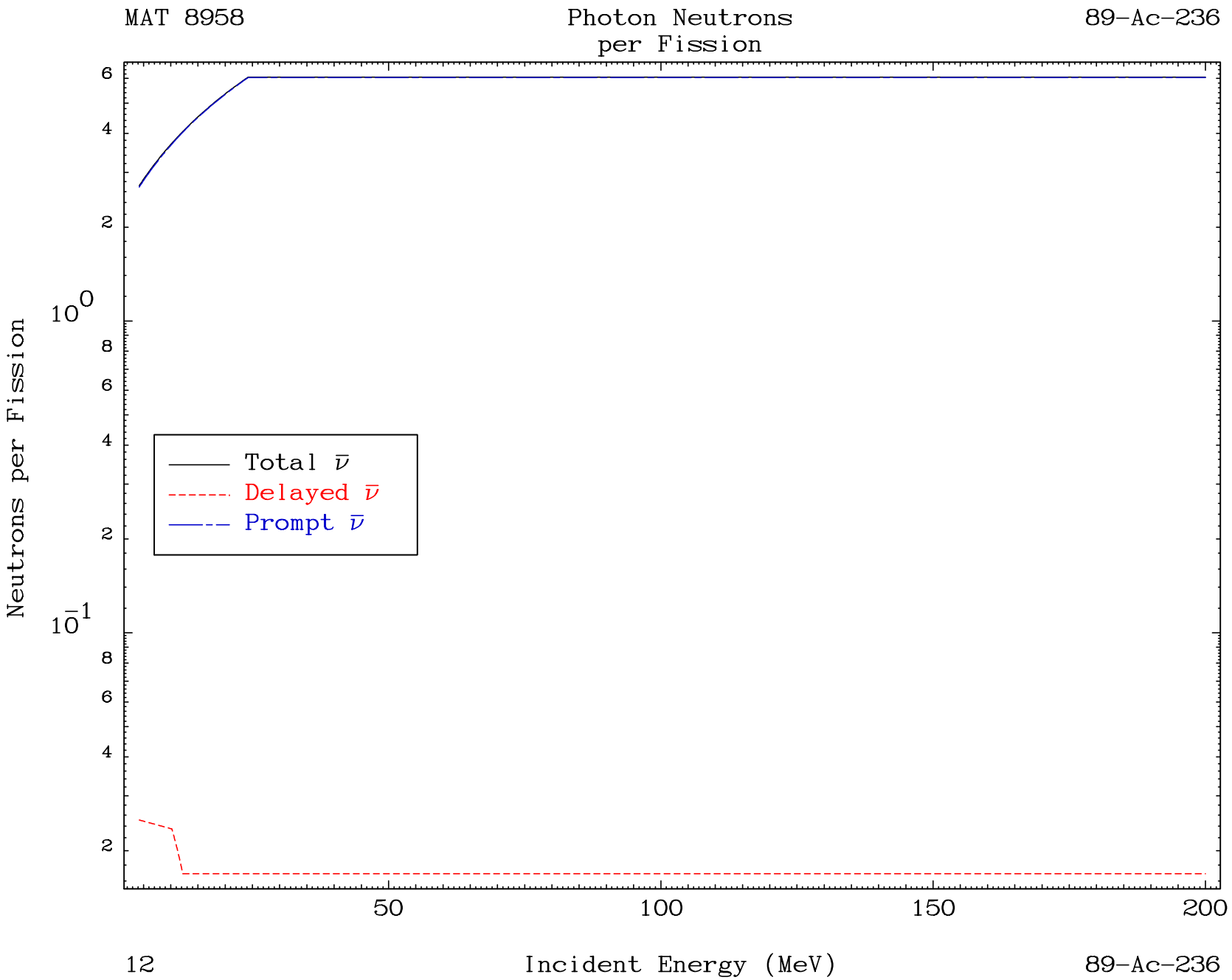

MAT 8958
(γ,n') Levels
89-Ac-236
0 Kelvin Cross Sections
Inelastic
(n,n') Ground
MT= 51 (n,n') Level
(n,n') Continuum
Cross Section (barns)
Incident Energy (MeV)
89-Ac-236

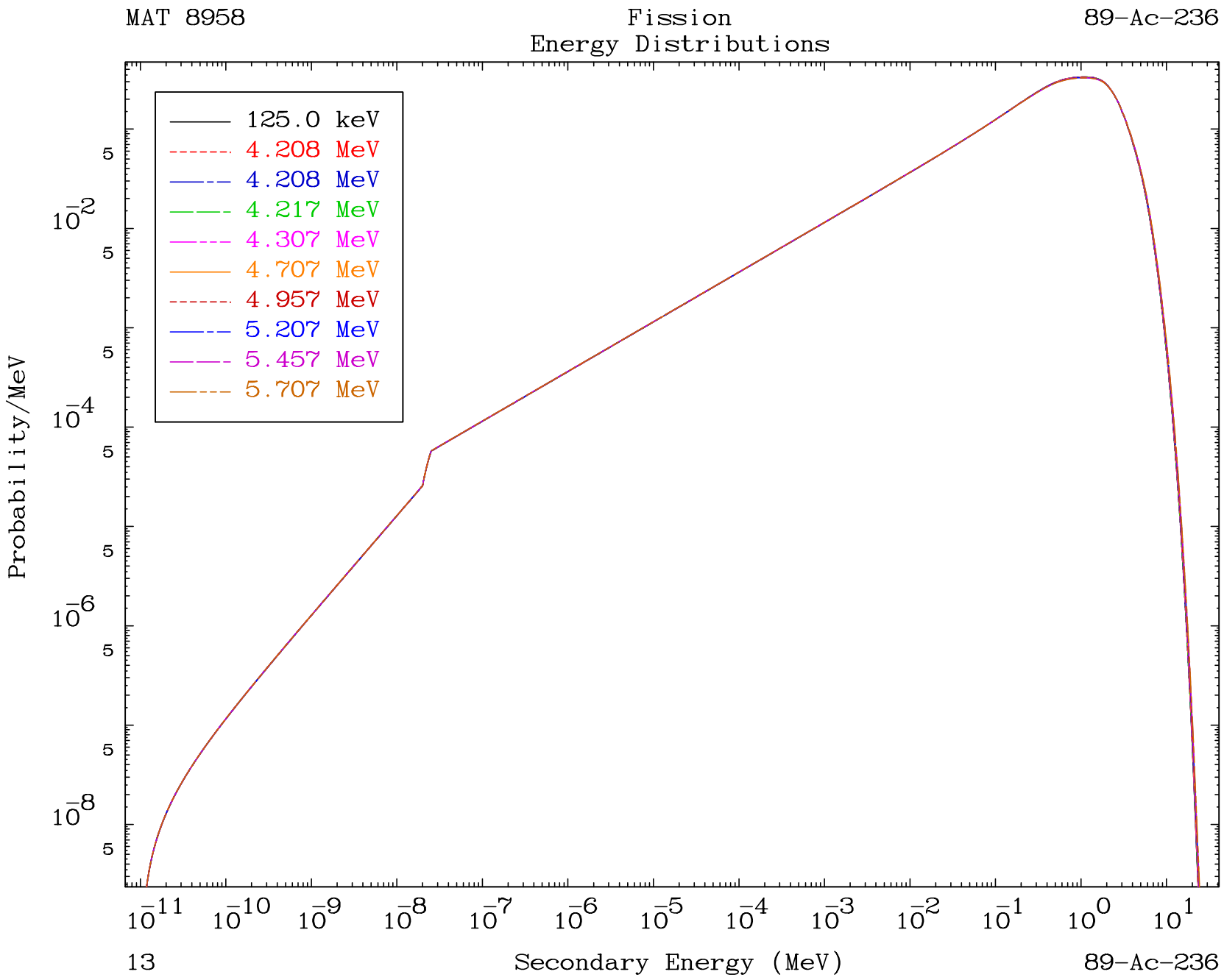

MAT 8958
(γ,p) Levels
89-Ac-236
0 Kelvin Cross Sections
(n,p)
Cross Section (barns)
Incident Energy (MeV)
8
89-Ac-236

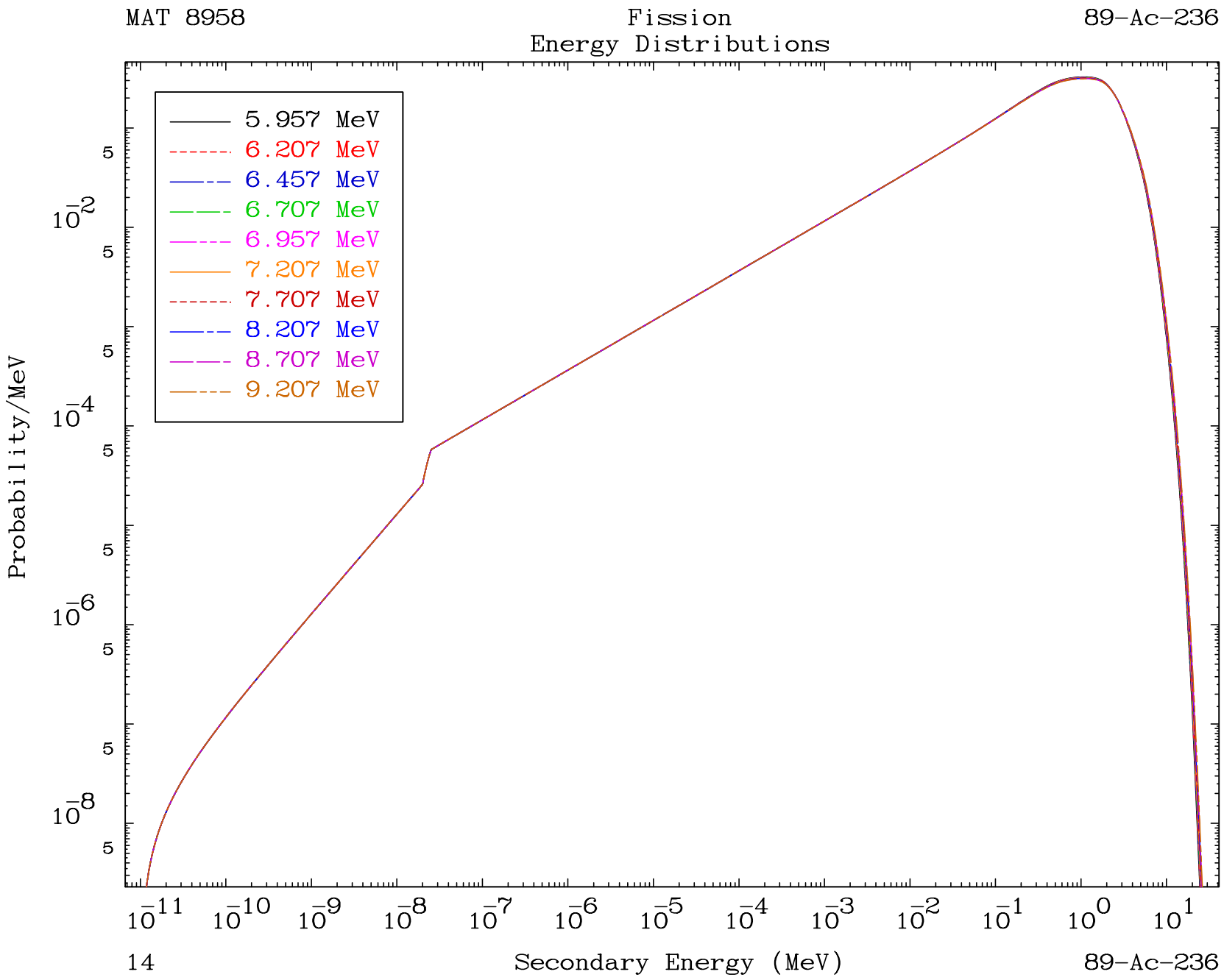

MAT 8958
(γ,d) Levels
89-Ac-236
0 Kelvin Cross Sections
(n,d)
Cross Section (barns)
10^-6
10^-8
10^-10
10^-12
5
9
10^1
2
3
4
5
6
7
8
9
10^2
2
9
Incident Energy (MeV)
89-Ac-236

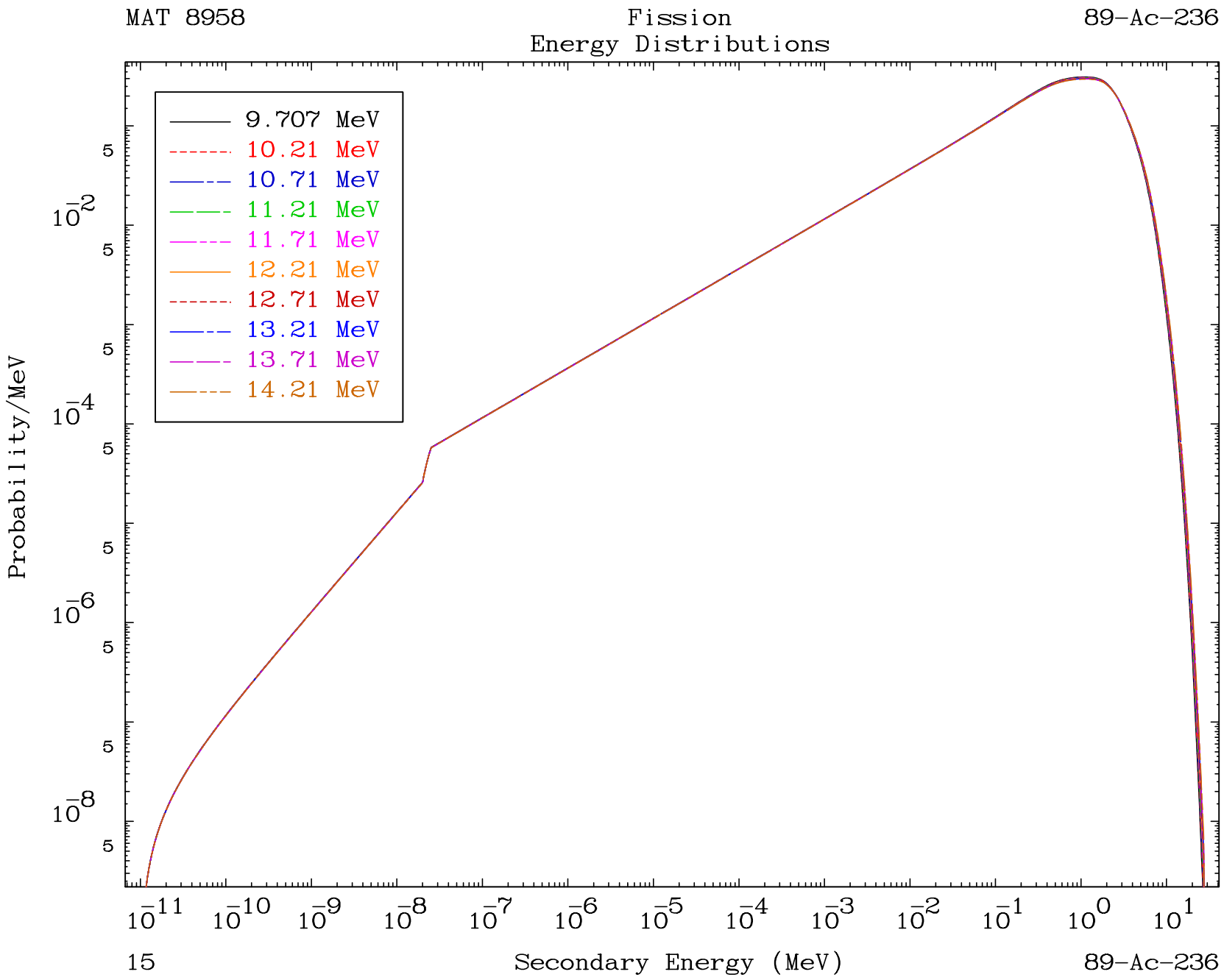

MAT 8958
(γ,t) Levels
89-Ac-236
0 Kelvin Cross Sections
(n,t)
Cross Section (barns)
10^-8
10^-10
10^-12
10^-14
5
8 9 10^1 2 3 4 5 6 7 8 9 10^2 2
10
Incident Energy (MeV)
89-Ac-236

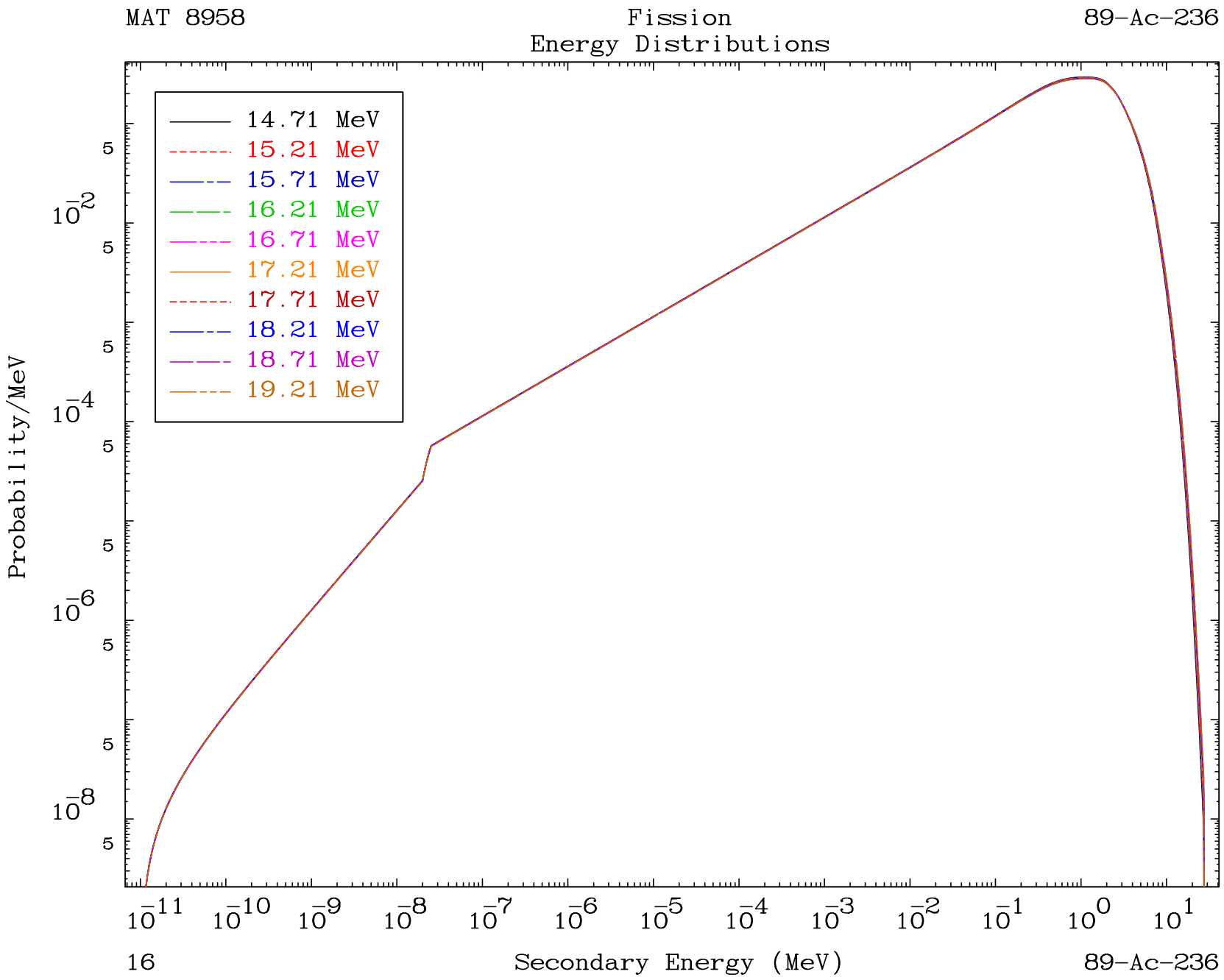

MAT 8958
(γ,α) Levels
89-Ac-236
0 Kelvin Cross Sections
(n,α)
Cross Section (barns)
Incident Energy (MeV)
11
89-Ac-236

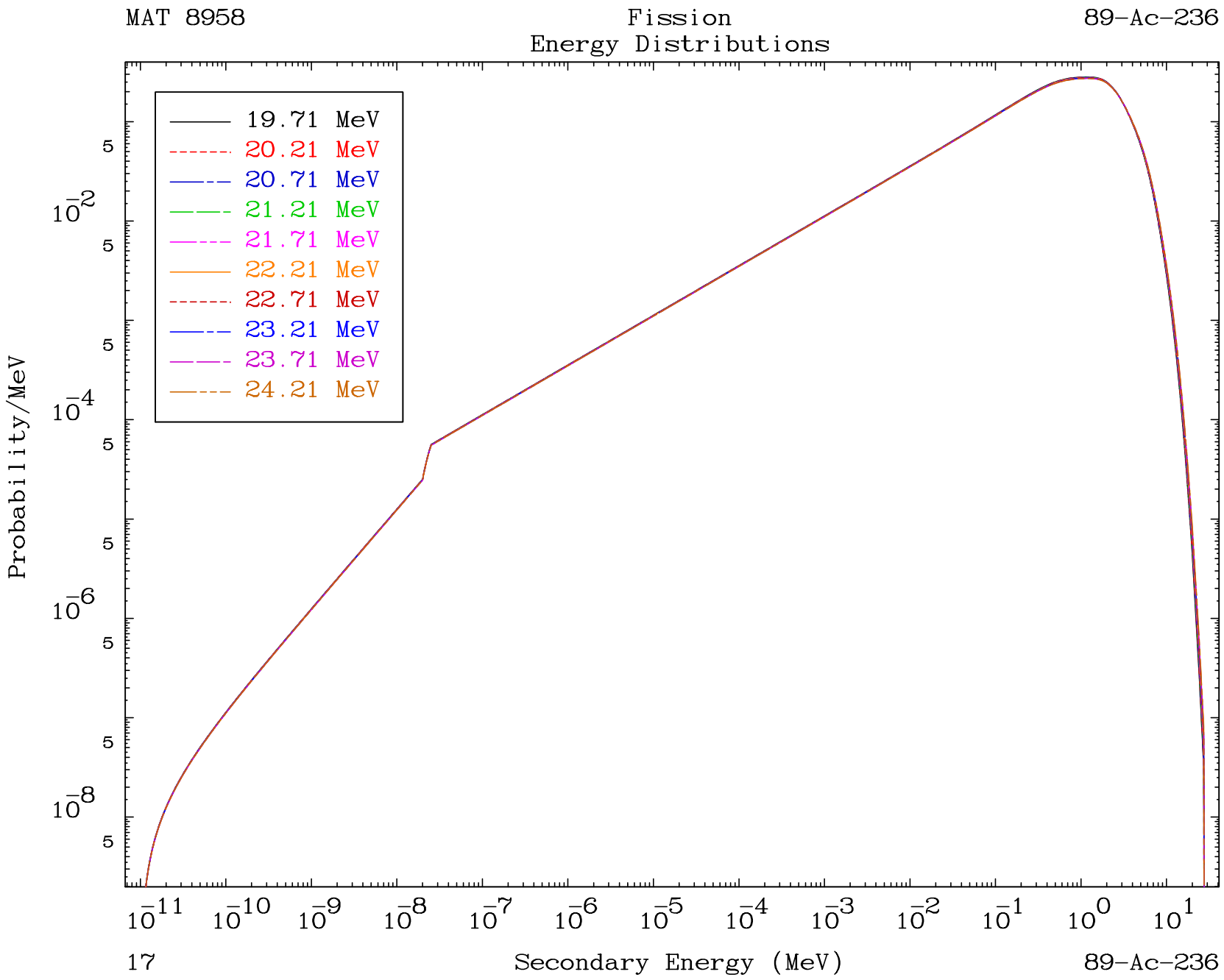

MAT 8958
Photon Neutrons per Fission
89-Ac-236
Neutrons per Fission
6
4
2
10^0
8
6
4
2
10^-1
8
6
4
2
Total ν̄
Delayed ν̄
Prompt ν̄
50
100
150
200
12
Incident Energy (MeV)
89-Ac-236


MAT 8958
Fission
Energy Distributions
89-Ac-236
125.0 keV
4.208 MeV
4.208 MeV
4.217 MeV
4.307 MeV
4.707 MeV
4.957 MeV
5.207 MeV
5.457 MeV
5.707 MeV
Probability/MeV
Secondary Energy (MeV)
13
89-Ac-236


MAT 8958
Fission
Energy Distributions
89-Ac-236
5.957 MeV
6.207 MeV
6.457 MeV
6.707 MeV
6.957 MeV
7.207 MeV
7.707 MeV
8.207 MeV
8.707 MeV
9.207 MeV
Probability/MeV
Secondary Energy (MeV)
14
89-Ac-236


MAT 8958
Fission
Energy Distributions
89-Ac-236
9.707 MeV
10.21 MeV
10.71 MeV
11.21 MeV
11.71 MeV
12.21 MeV
12.71 MeV
13.21 MeV
13.71 MeV
14.21 MeV
Probability/MeV
Secondary Energy (MeV)
15
89-Ac-236


MAT 8958
Fission
Energy Distributions
89-Ac-236
14.71 MeV
15.21 MeV
15.71 MeV
16.21 MeV
16.71 MeV
17.21 MeV
17.71 MeV
18.21 MeV
18.71 MeV
19.21 MeV
Probability/MeV
Secondary Energy (MeV)
16
89-Ac-236


MAT 8958
Fission
Energy Distributions
89-Ac-236
19.71 MeV
20.21 MeV
20.71 MeV
21.21 MeV
21.71 MeV
22.21 MeV
22.71 MeV
23.21 MeV
23.71 MeV
24.21 MeV
Probability/MeV
Secondary Energy (MeV)
17
89-Ac-236