

Program EVALPLOT  
(Version 2021-1)

by

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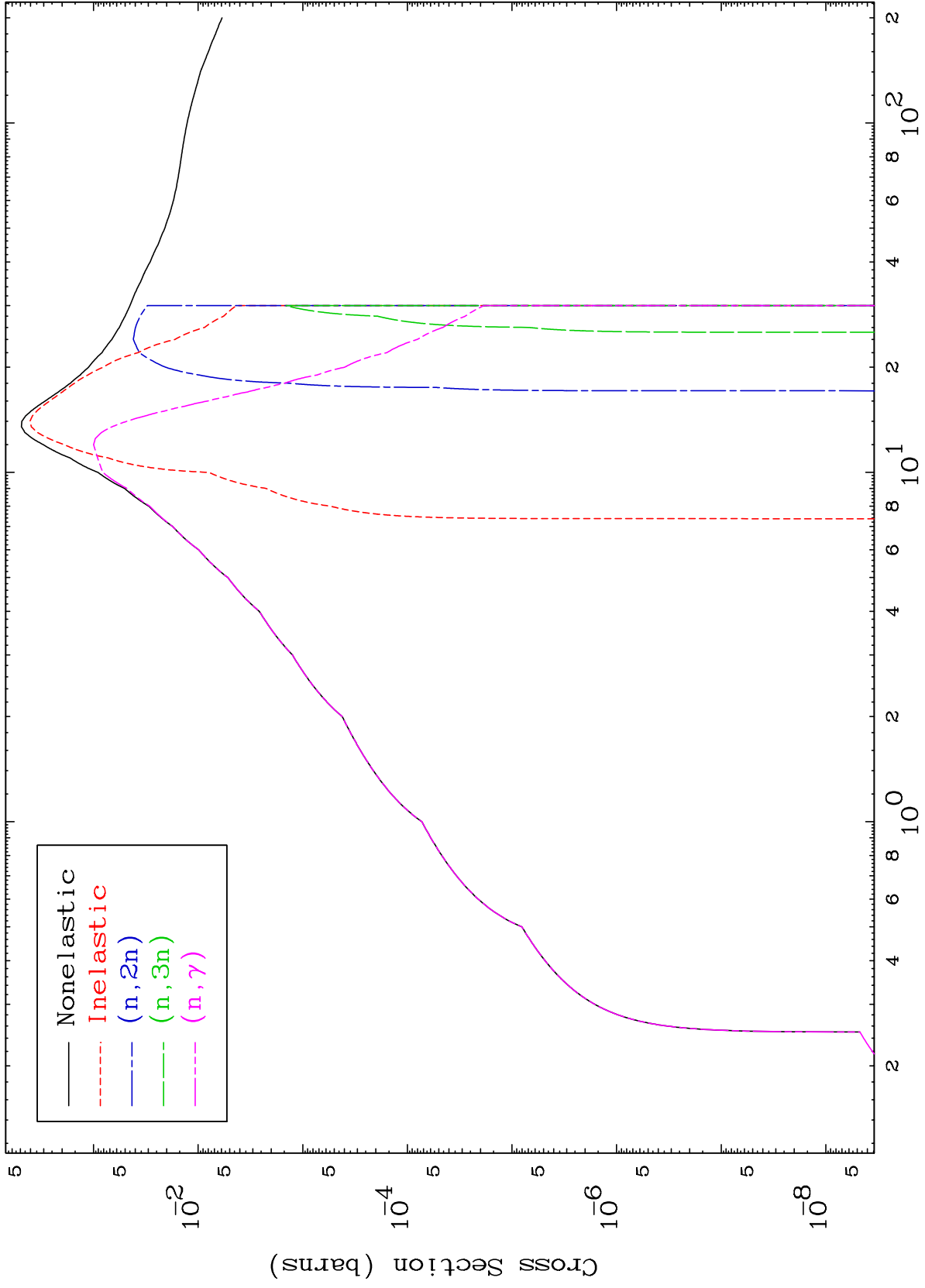
Press Mouse Button to Start

MAT 8293

Photon Major

83-Bi-198m

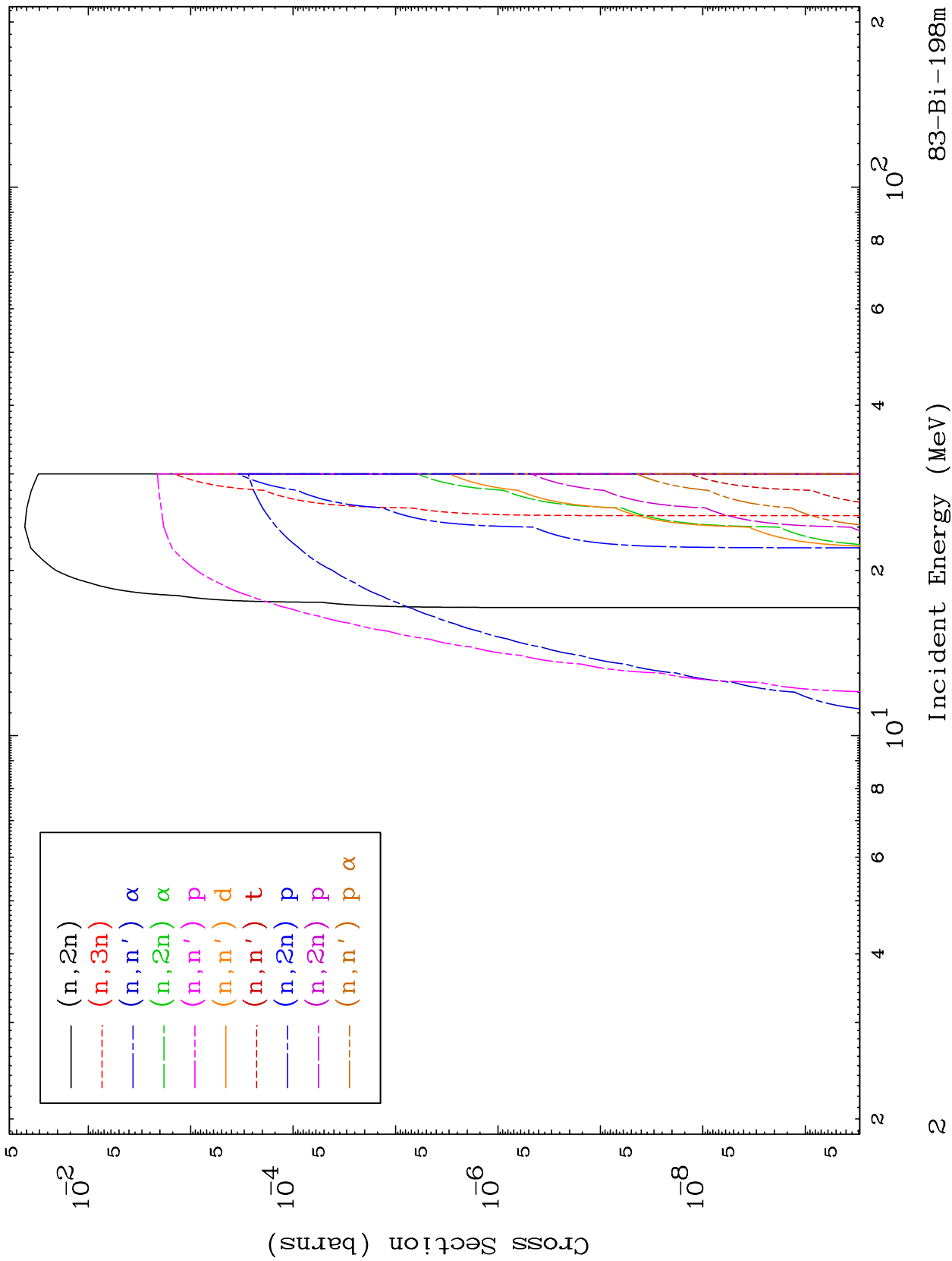
0 Kelvin Cross Sections



MAT 8293

Photon Neutron Absorption  
0 Kelvin Cross Sections

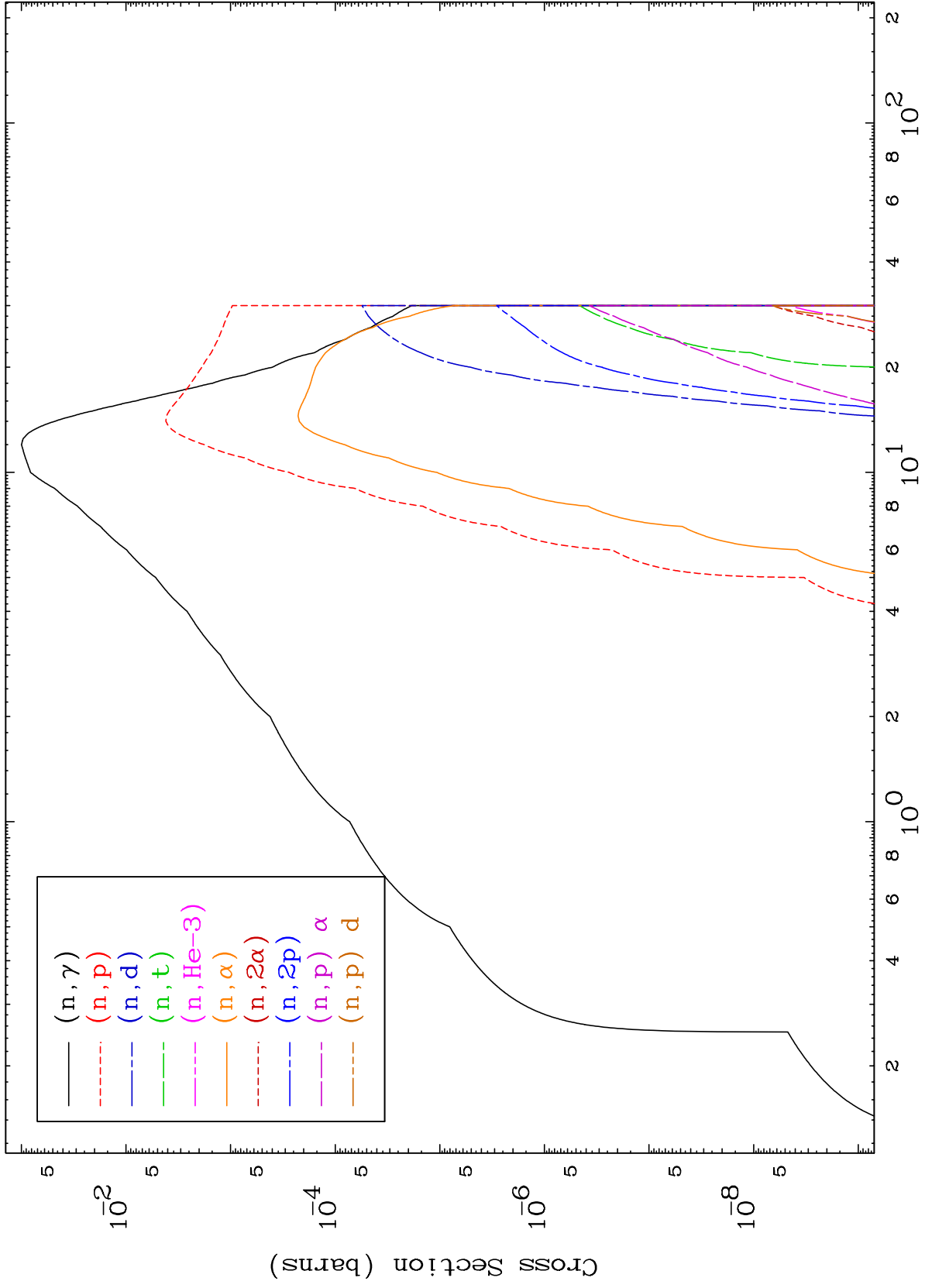
83-Bi-198m

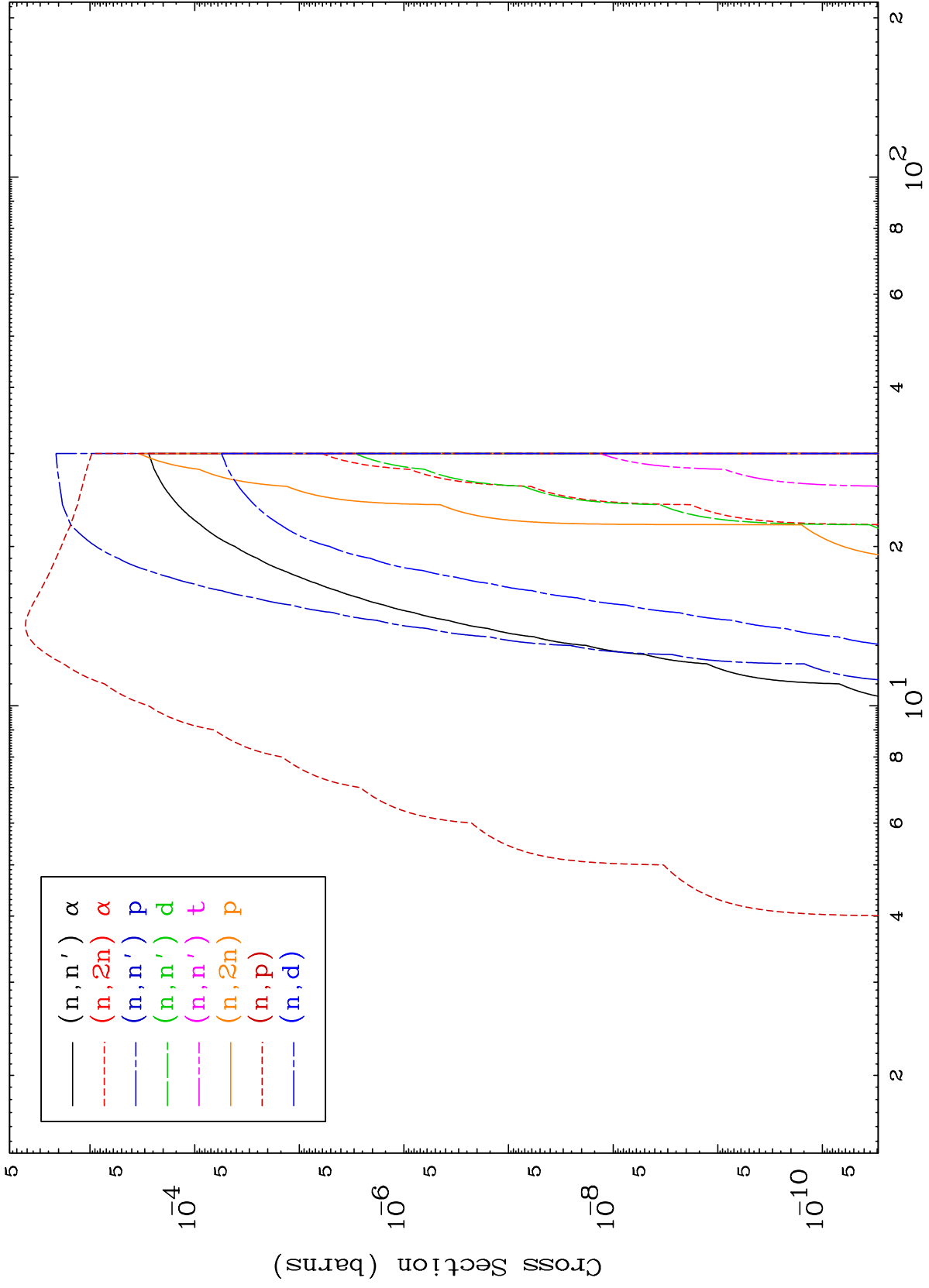


83-Bi-198m

Incident Energy (MeV)

2

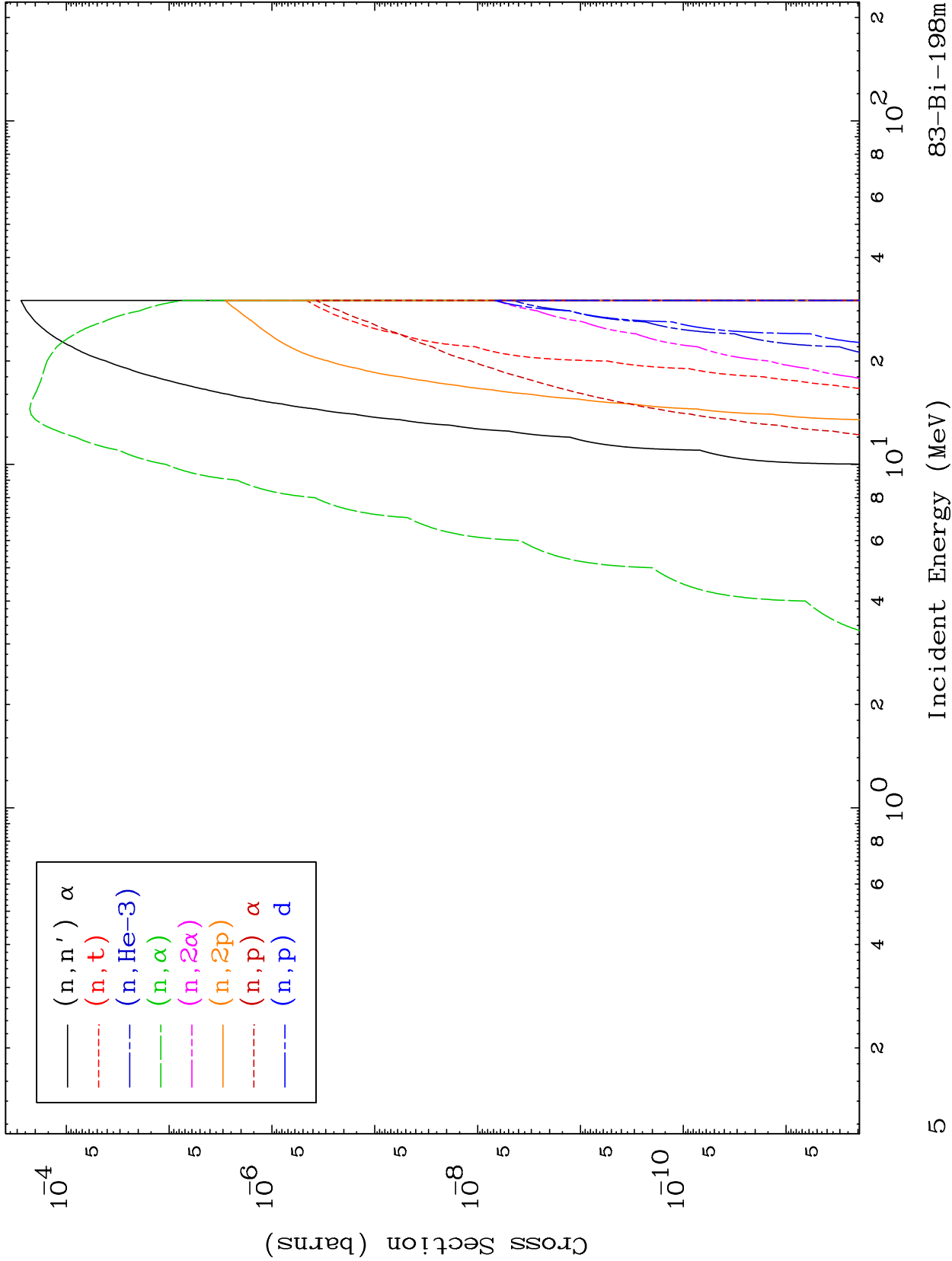




MAT 8293

Photon Charged Particle  
0 Kelvin Cross Sections

83-Bi-198m

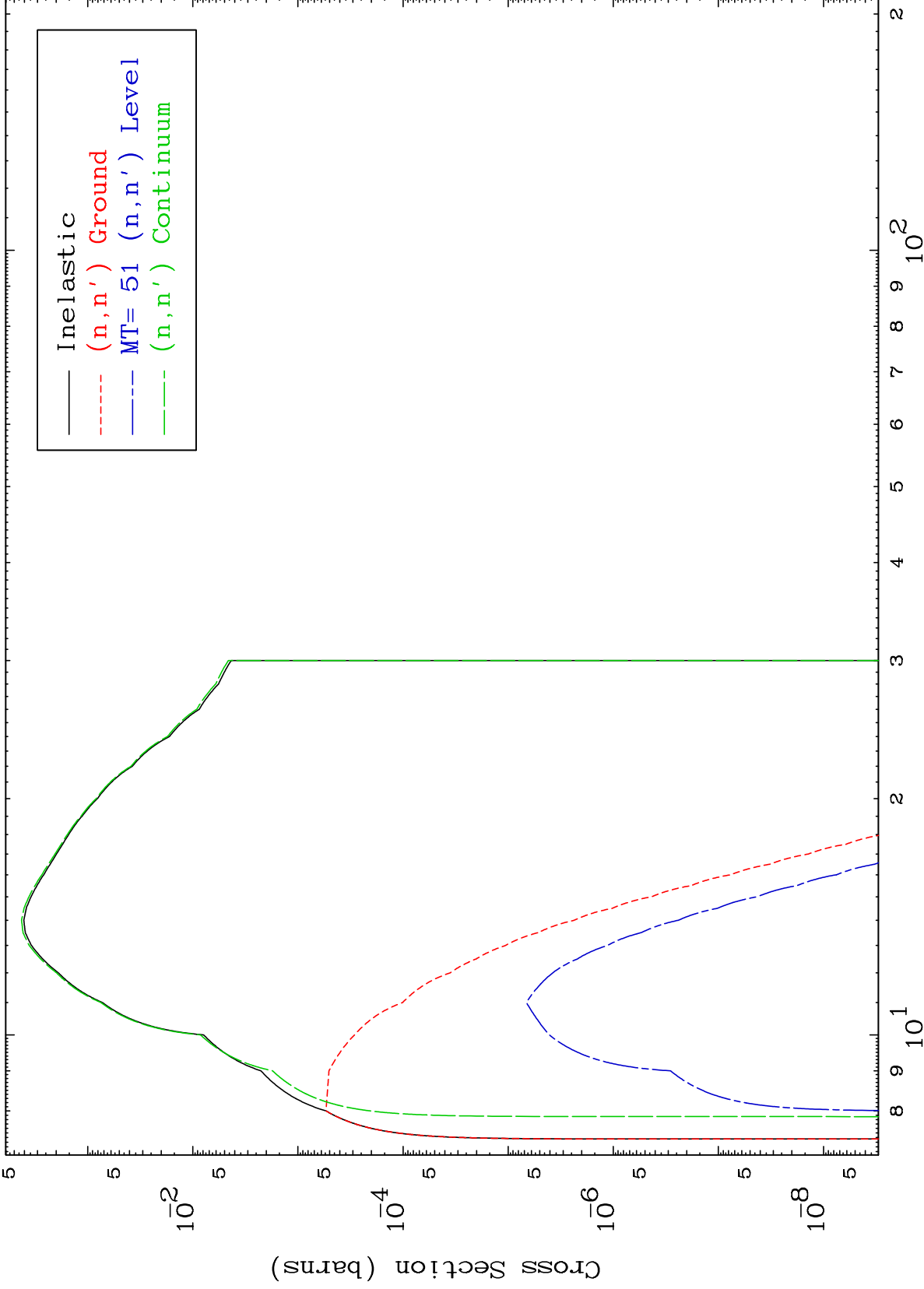


MAT 8293

( $\gamma, n'$ ) Levels

83-Bi-198m

0 Kelvin Cross Sections



Incident Energy (MeV)

83-Bi-198m

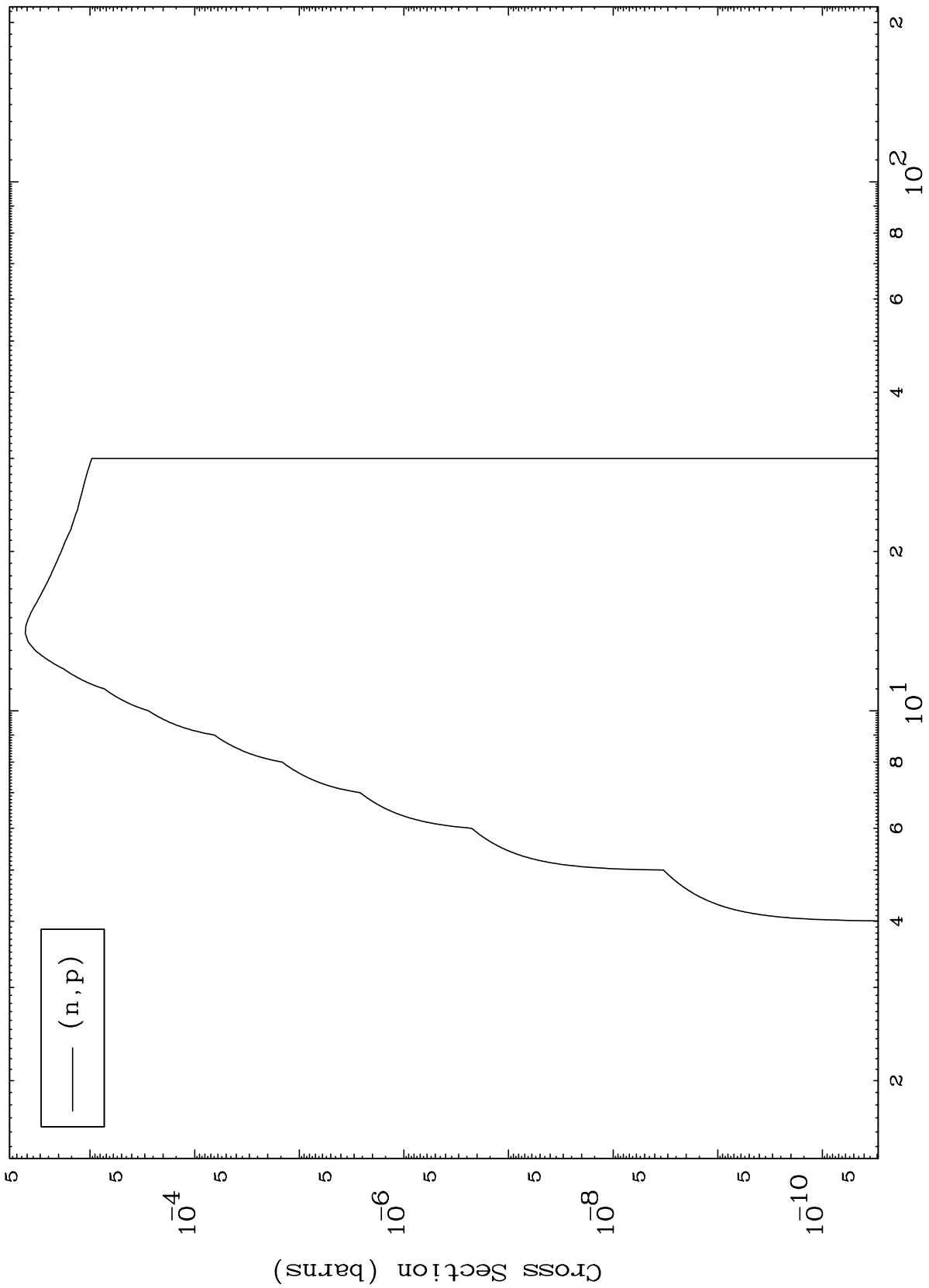
6

MAT 8293

( $\gamma, p$ ) Levels

83-Bi-198m

0 Kelvin Cross Sections



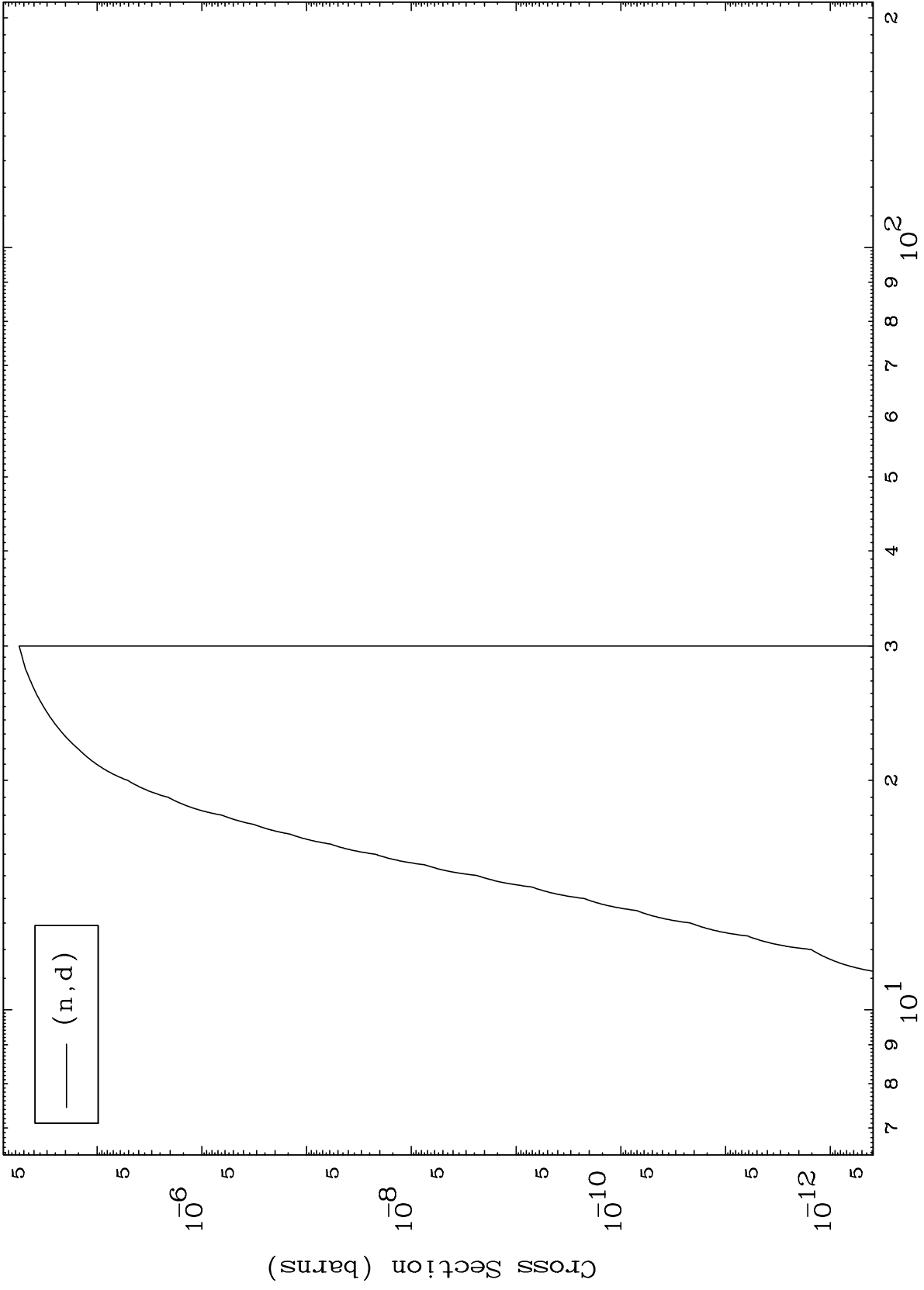
(n, p)



MAT 8293

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

$^{83}\text{Bi}-198\text{m}$



8

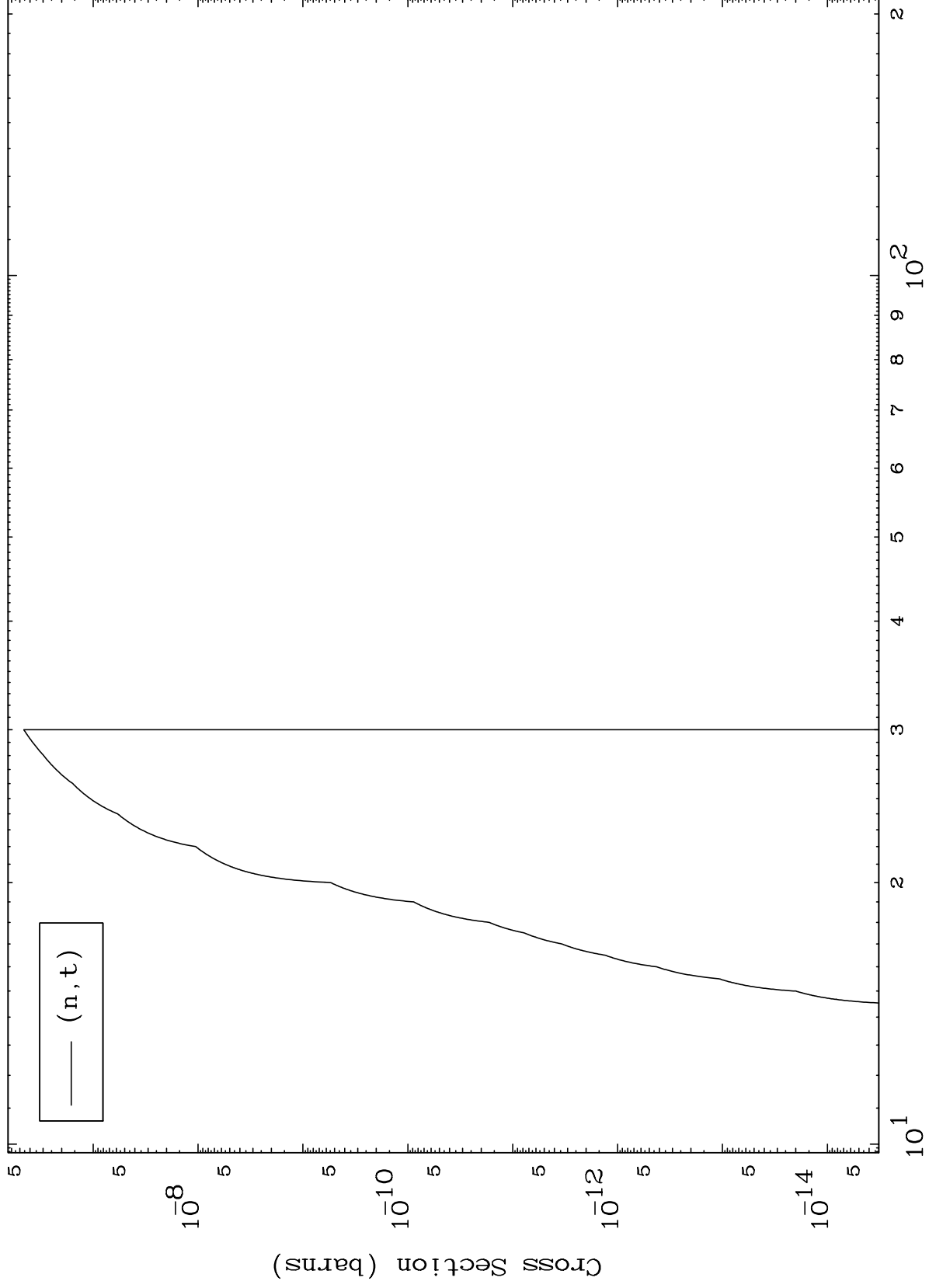
Incident Energy (MeV)

$^{83}\text{Bi}-198\text{m}$

MAT 8293

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

83-Bi-198m



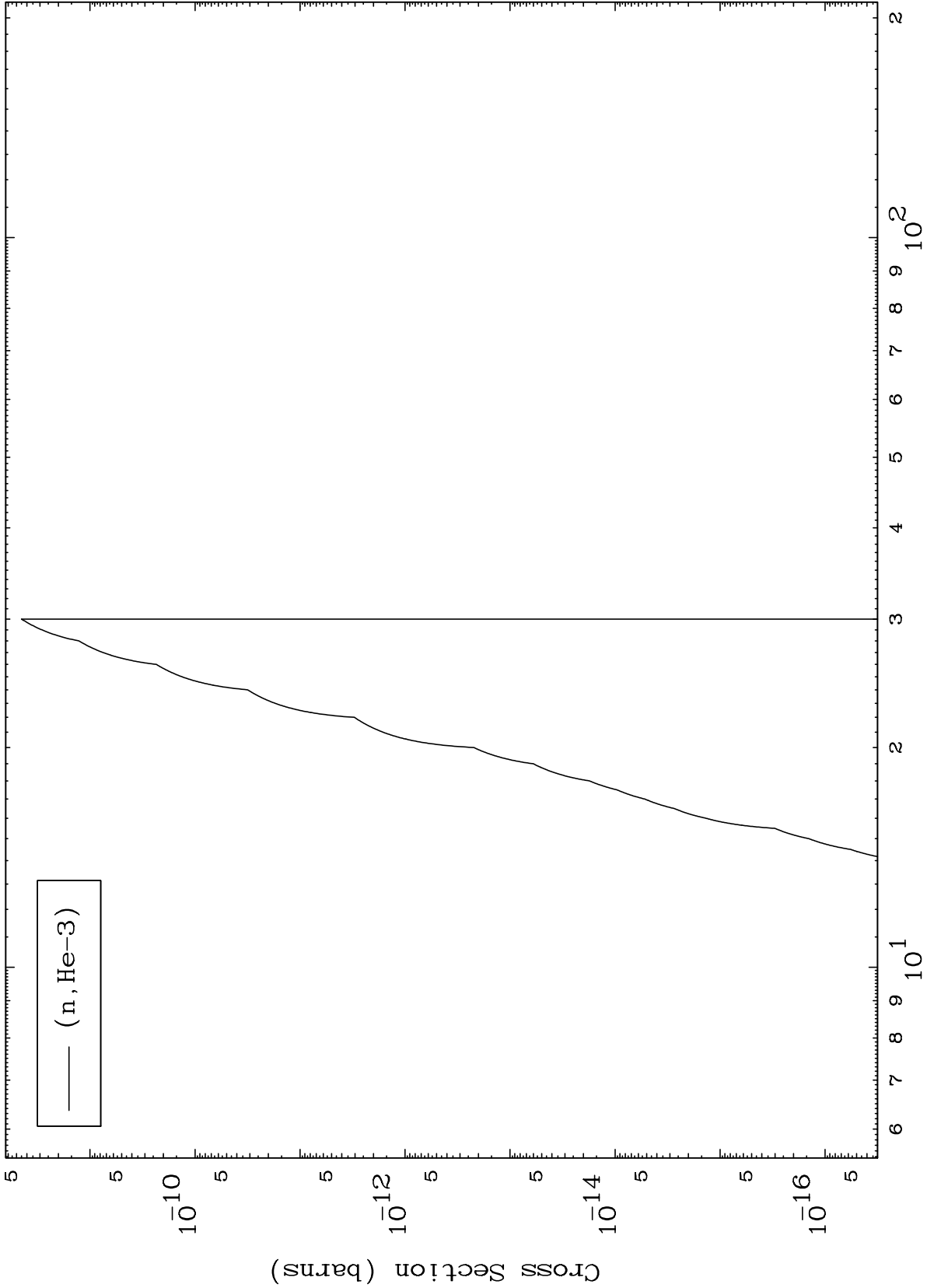
83-Bi-198m

Incident Energy (MeV)

MAT 8293

( $\gamma, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

83-Bi-198m



10

Incident Energy (MeV)

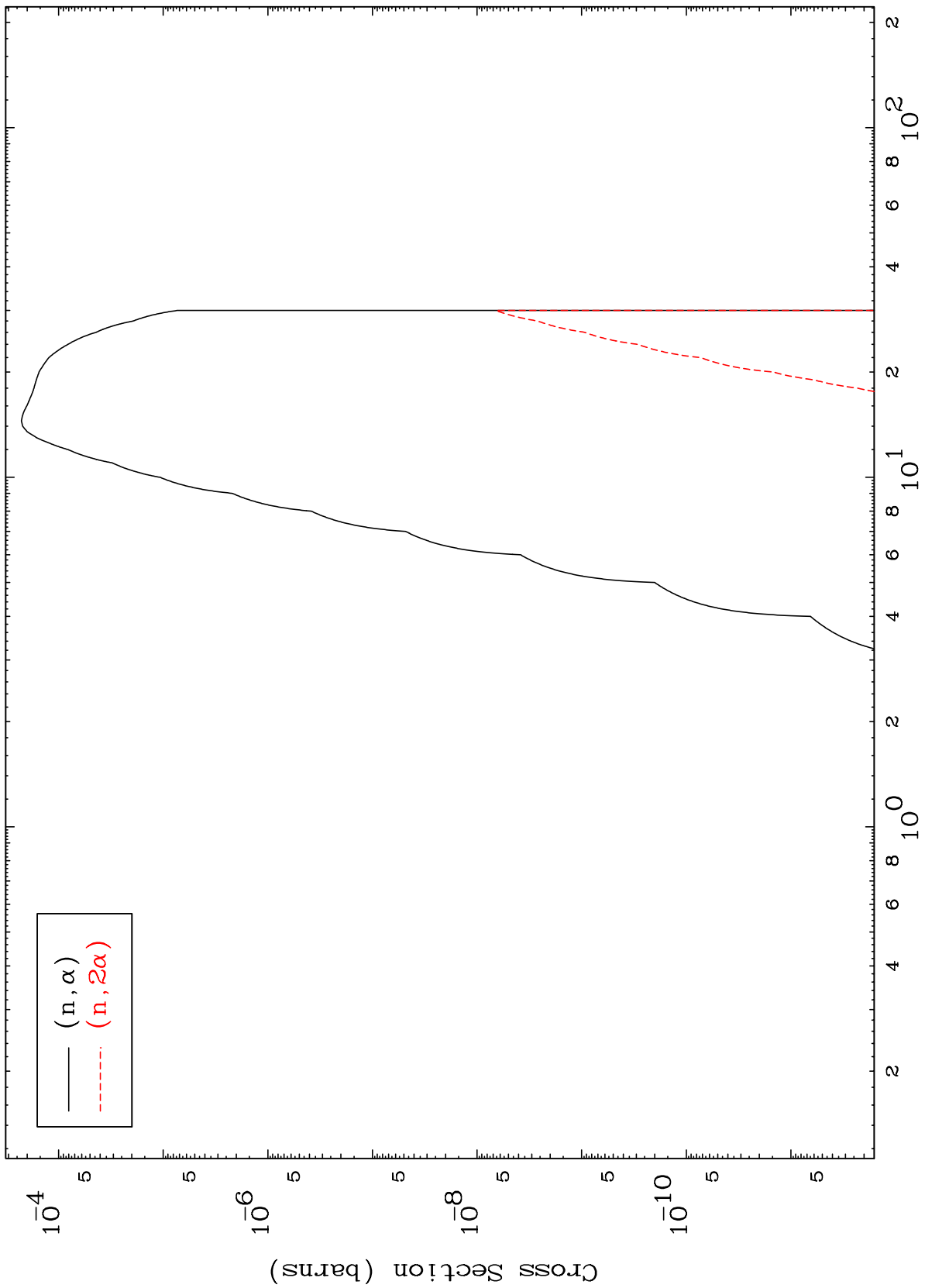
83-Bi-198m

MAT 8293

( $\gamma, \alpha$ ) Levels

83-Bi-198m

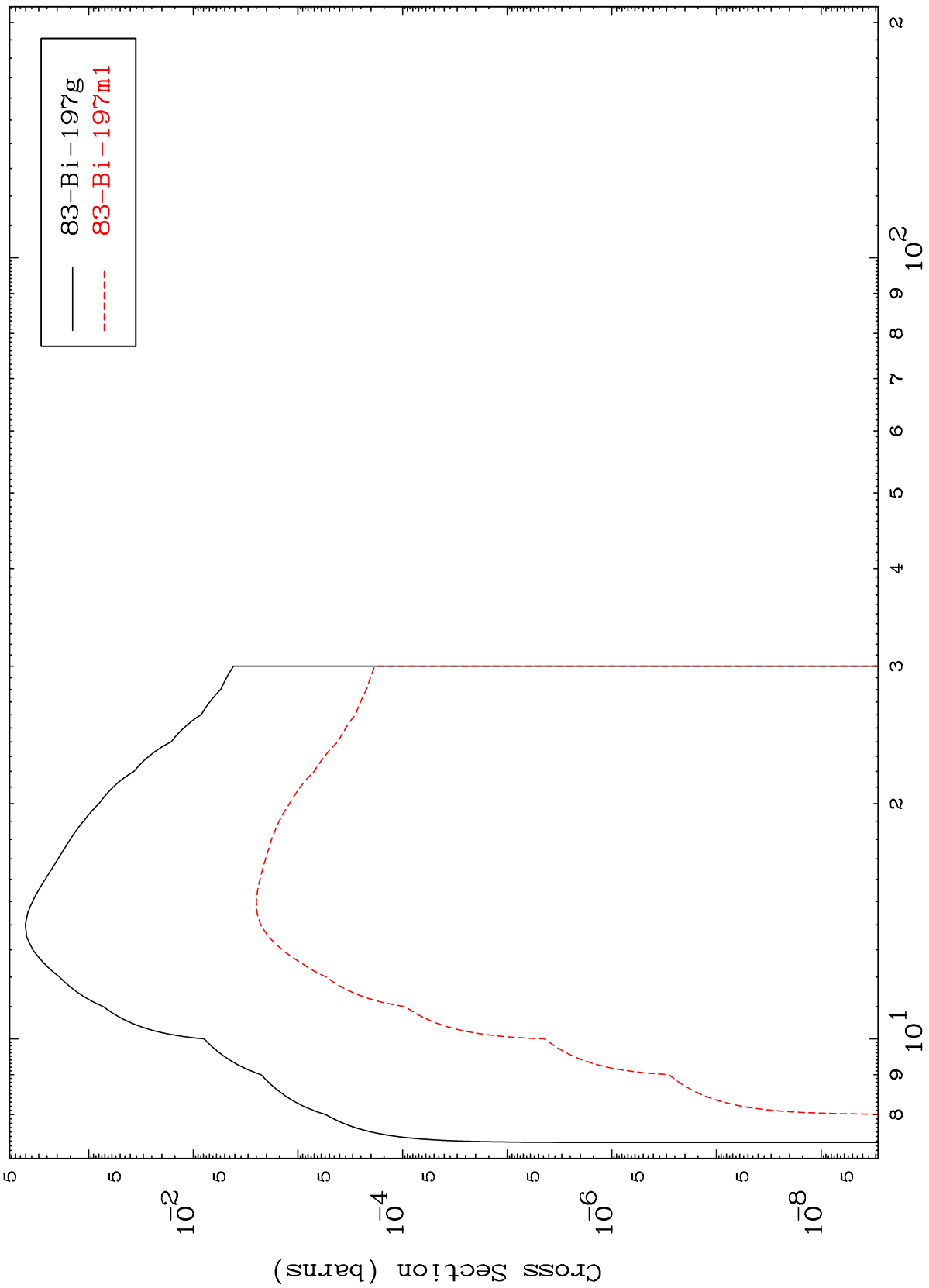
0 Kelvin Cross Sections



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83-Bi-198m

Inelastic  
Radionuclide Production Cross Section



83-Bi-198m

Incident Energy (MeV)

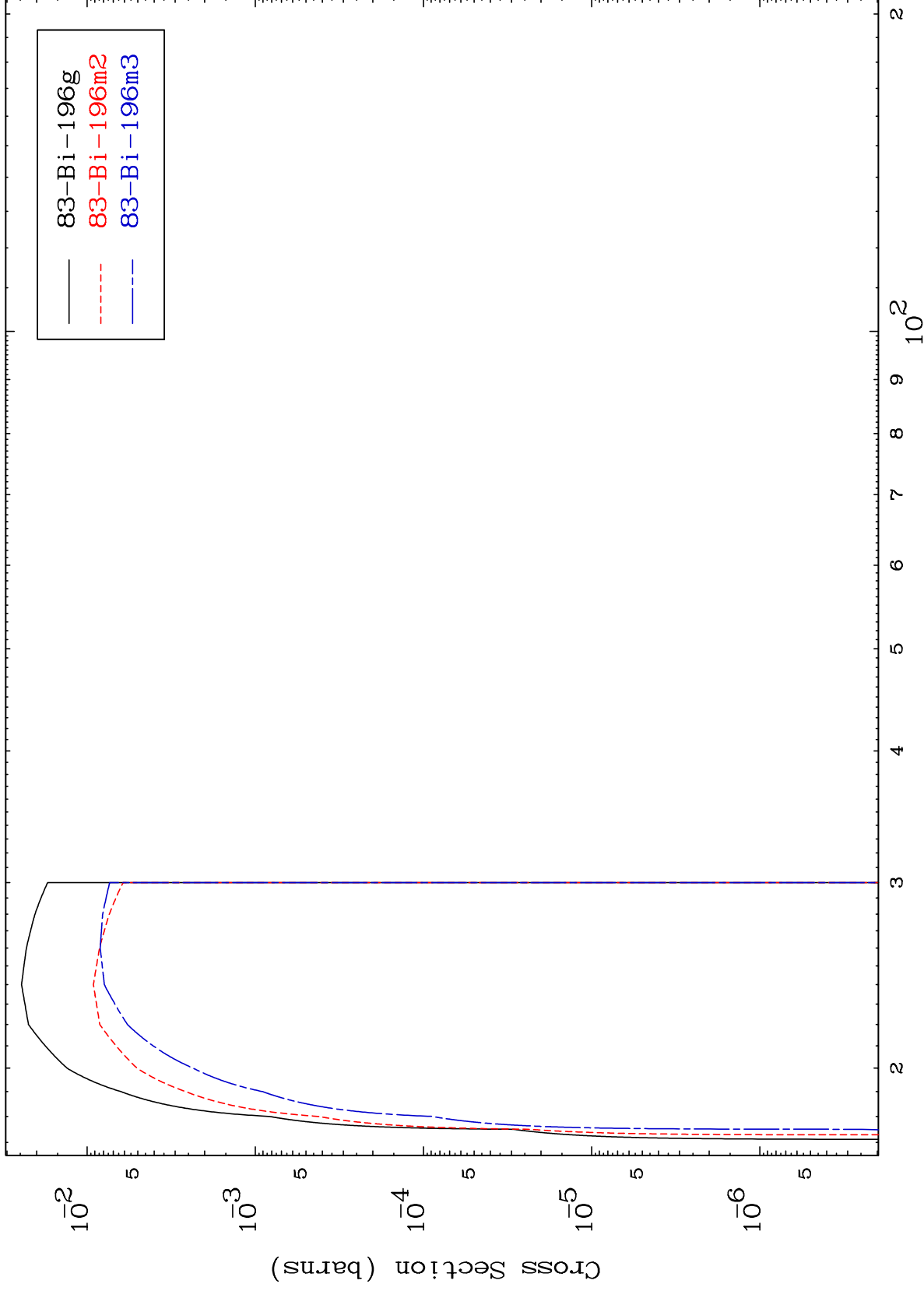
12

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(n,2n)

83-Bi-198m

Radionuclide Production Cross Section



13

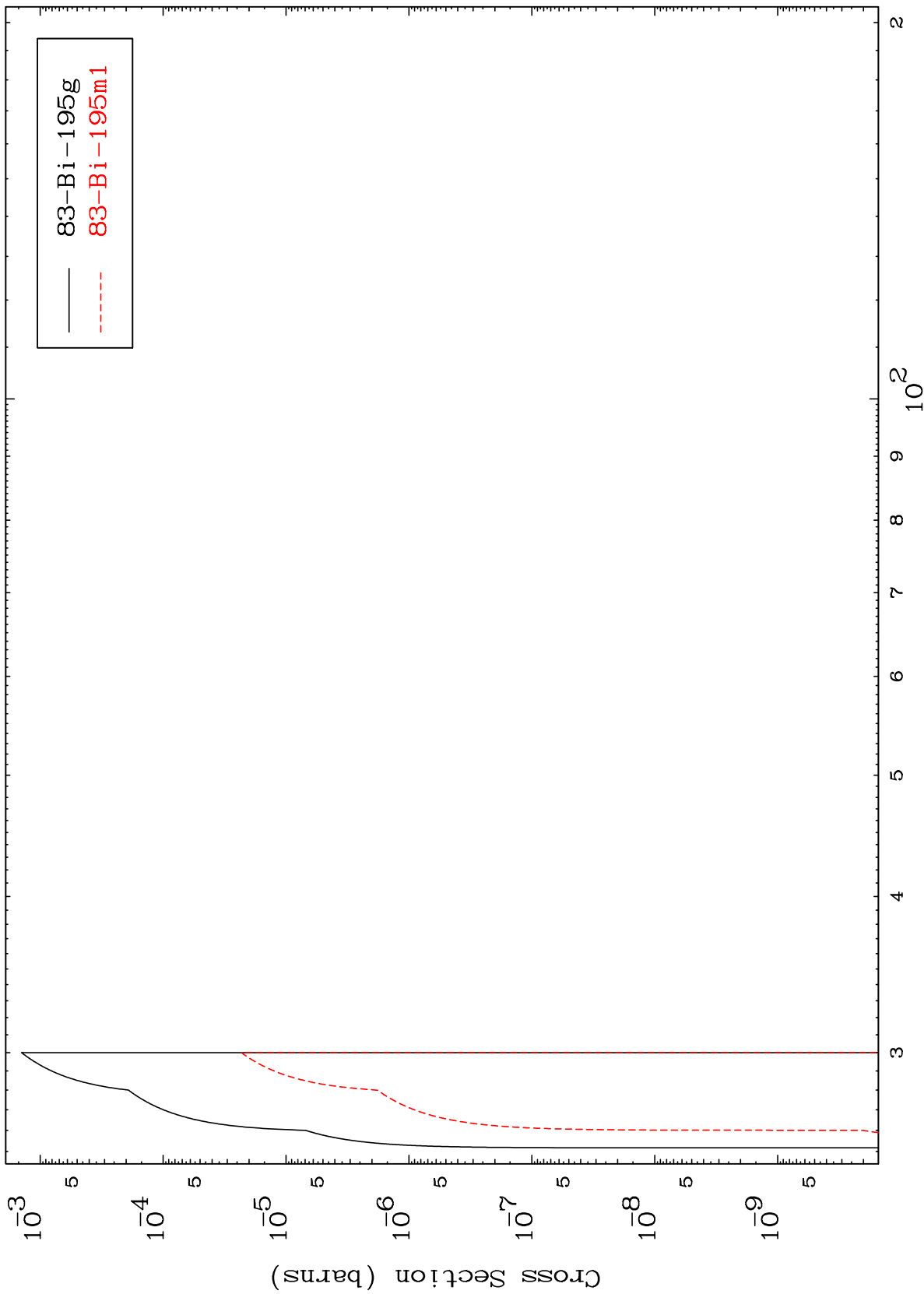
Incident Energy (MeV)

83-Bi-198m

MAT 8293

83-Bi-198m

(n,3n)  
Radionuclide Production Cross Section



83-Bi-198m

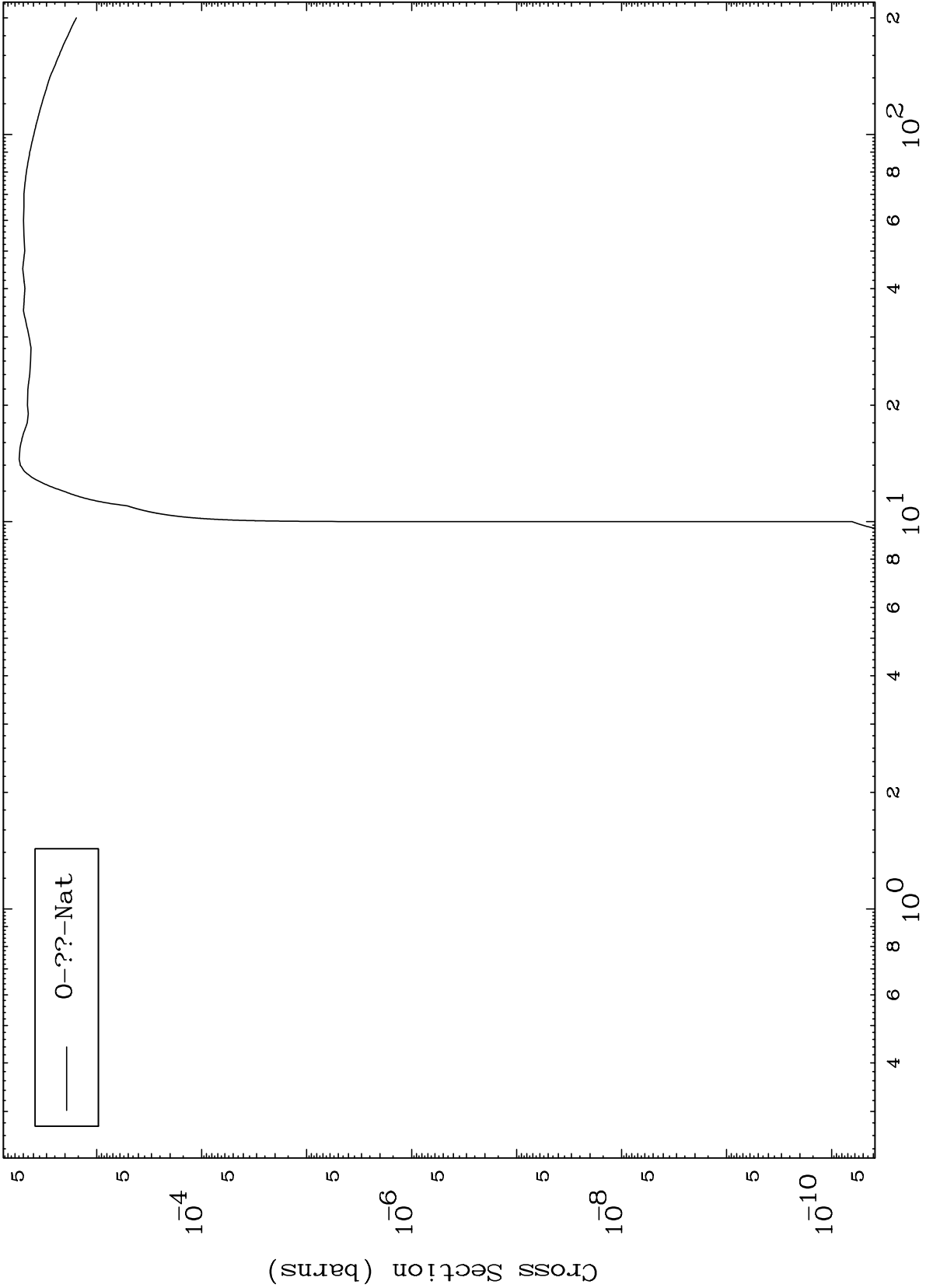
Incident Energy (MeV)

14

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83-Bi-198m

Fission  
Radionuclide Production Cross Section



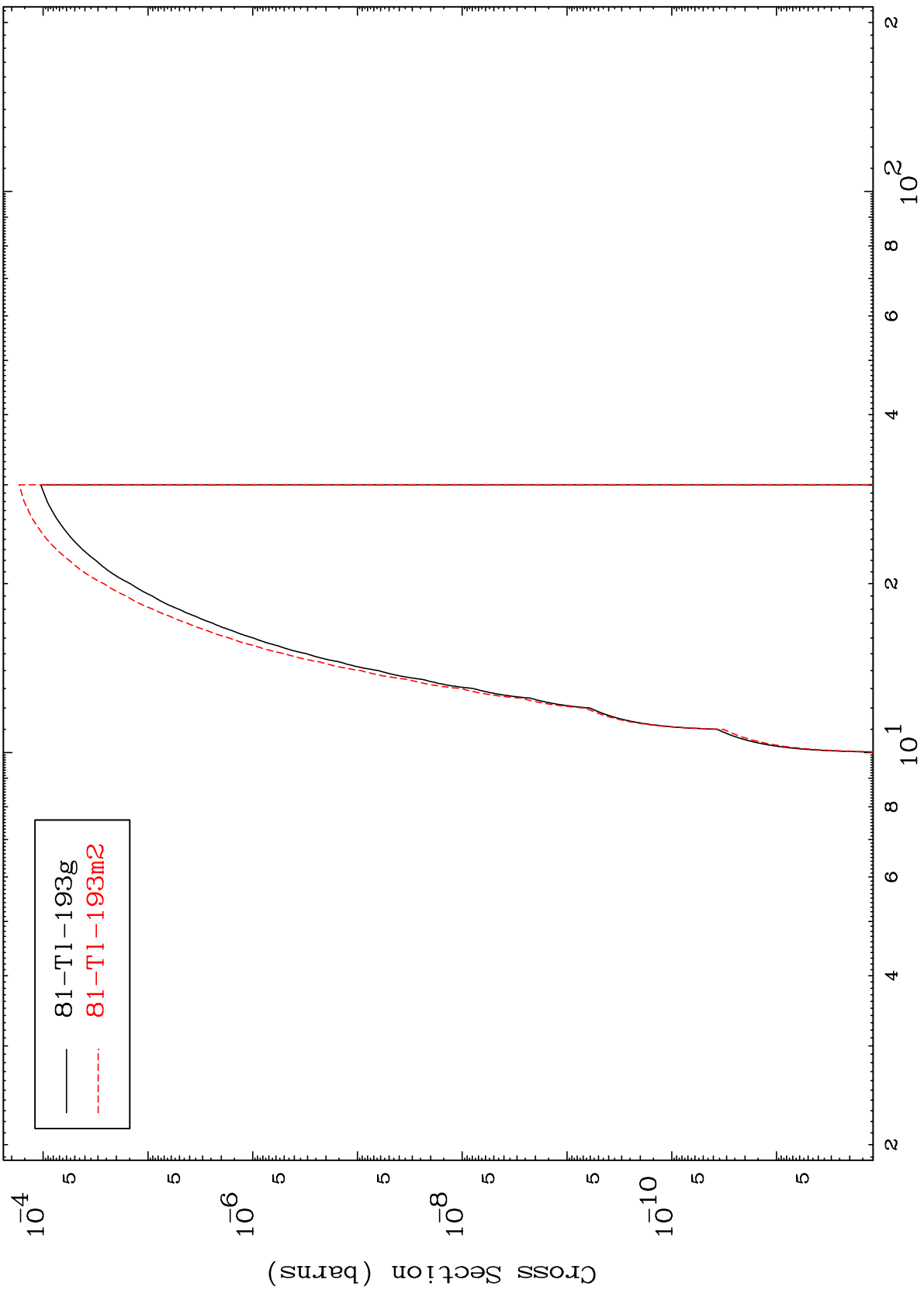


MAT 8293

$(n, n') \alpha$

$^{83}\text{Bi}-198\text{m}$

Radionuclide Production Cross Section



16

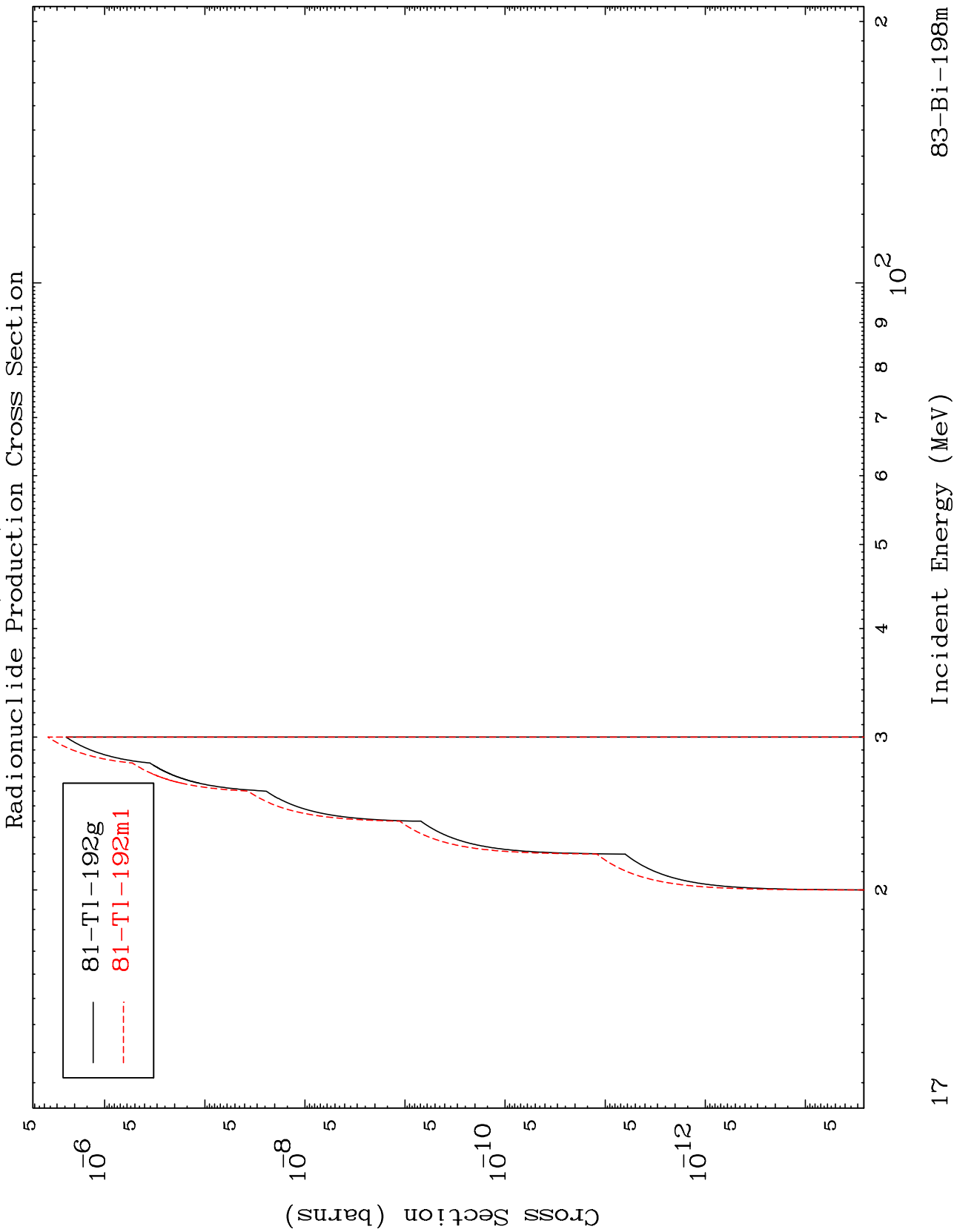
Incident Energy (MeV)

$^{83}\text{Bi}-198\text{m}$

MAT 8293

$(n,2n) \alpha$

$^{83}\text{Bi}-198\text{m}$

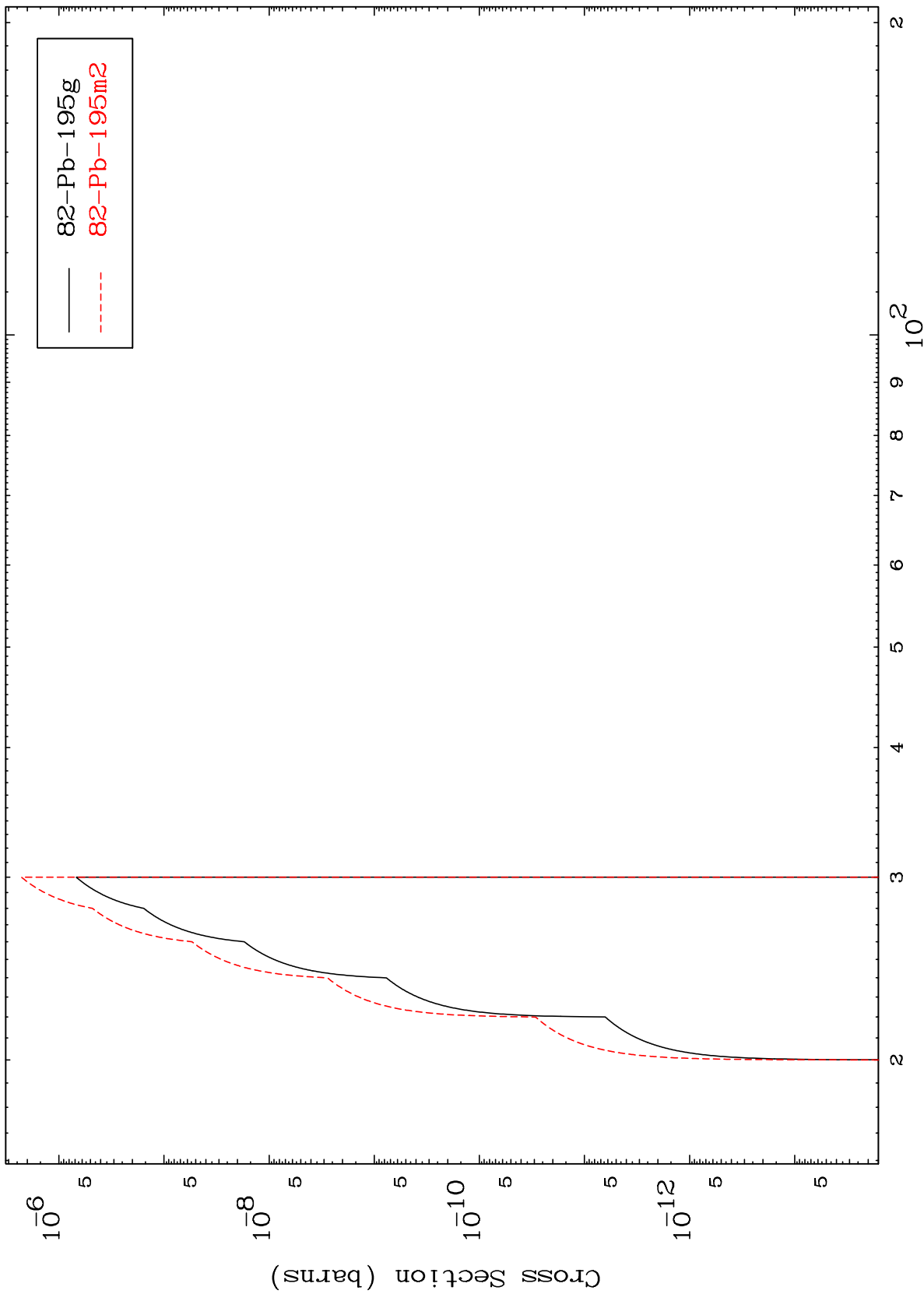


MAT 8293

(n,n') d

83-Bi-198m

Radionuclide Production Cross Section



18

Incident Energy (MeV)

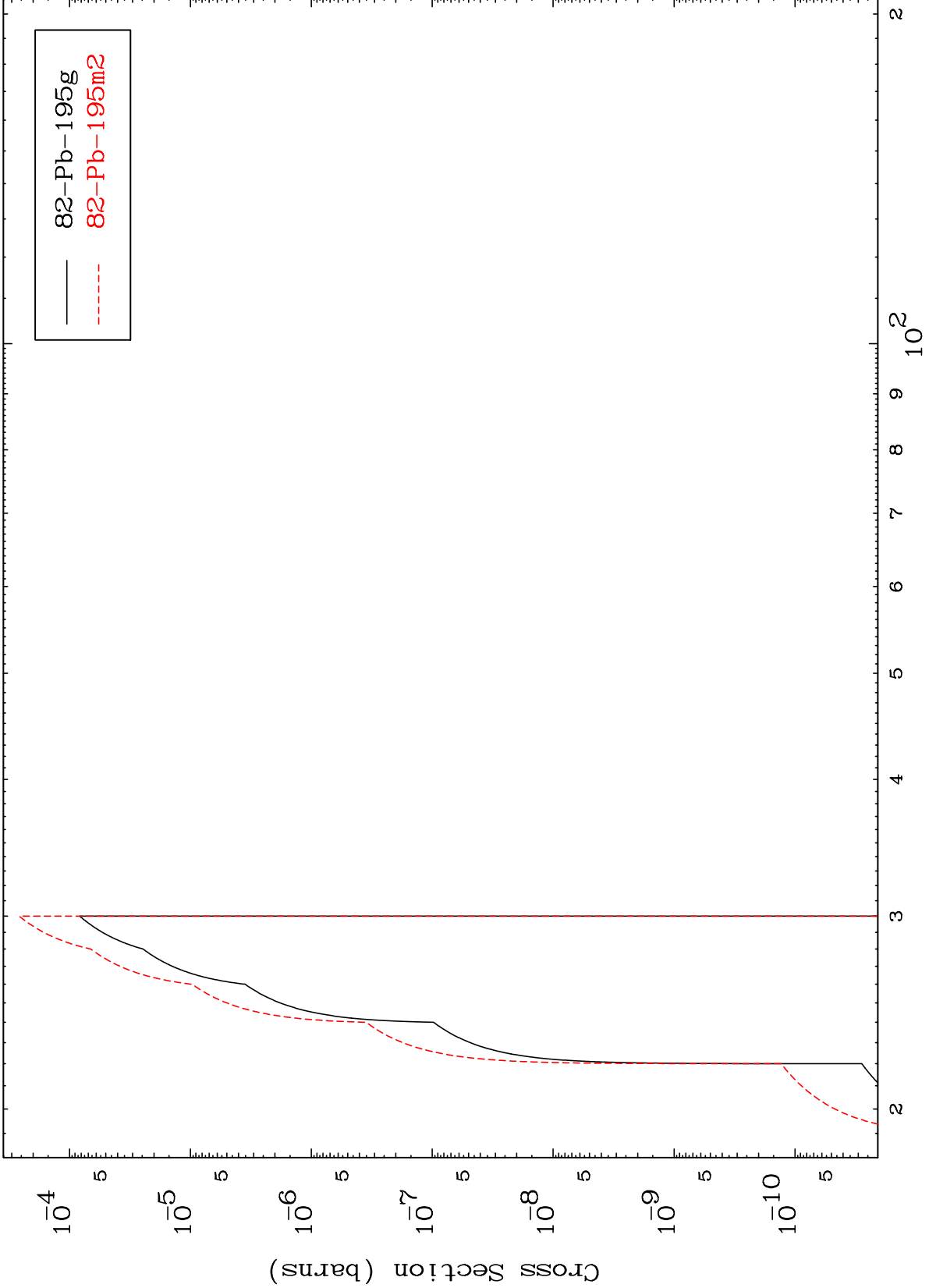
83-Bi-198m

MAT 8293

(n,2n) p

83-Bi-198m

Radionuclide Production Cross Section



19

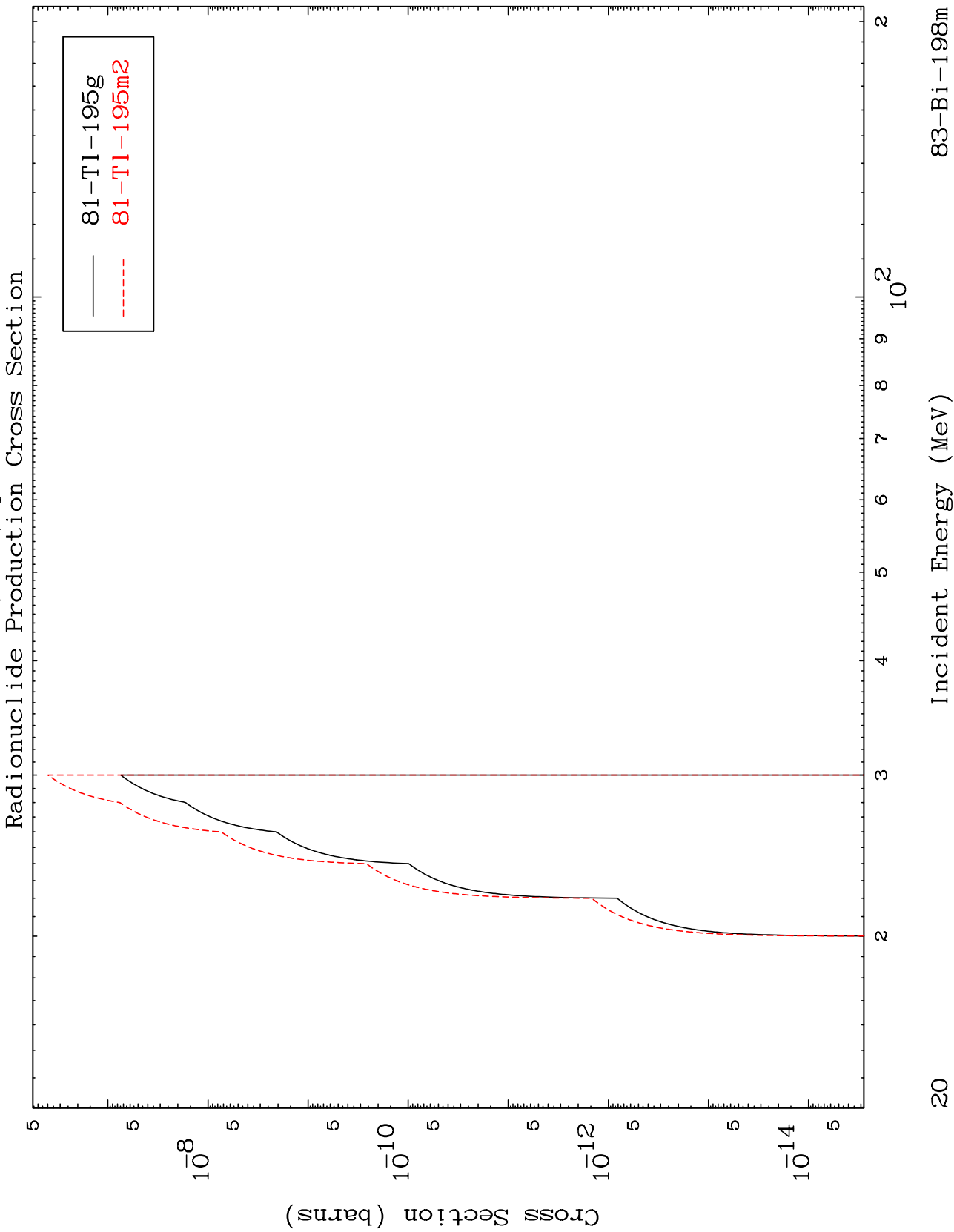
Incident Energy (MeV)

83-Bi-198m

MAT 8293

(n,2n) p

83-Bi-198m



20

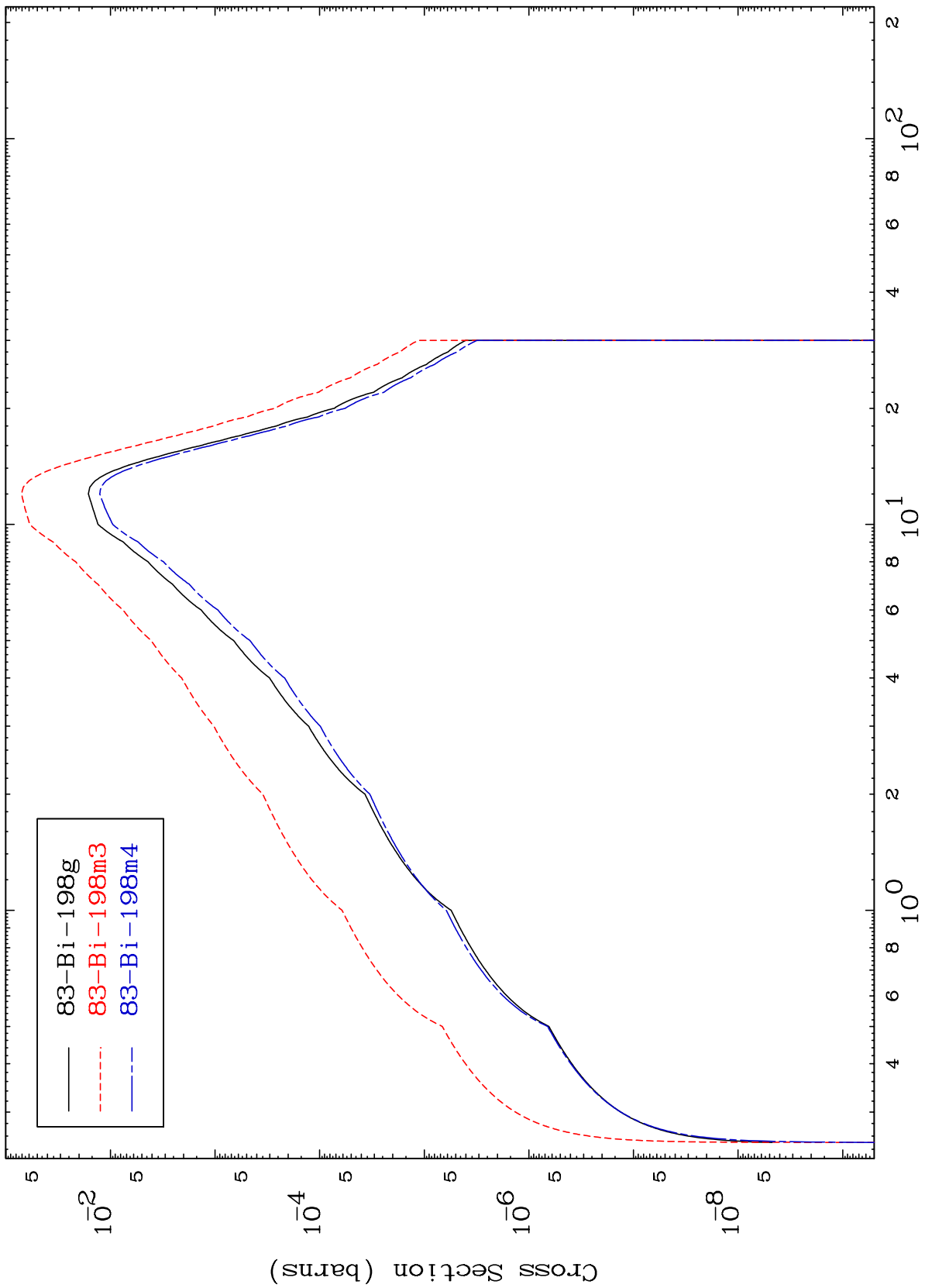
Incident Energy (MeV)

83-Bi-198m

MAT 8293

$^{83}\text{Bi-198m}$

Radionuclide Production Cross Section  
(n,  $\gamma$ )



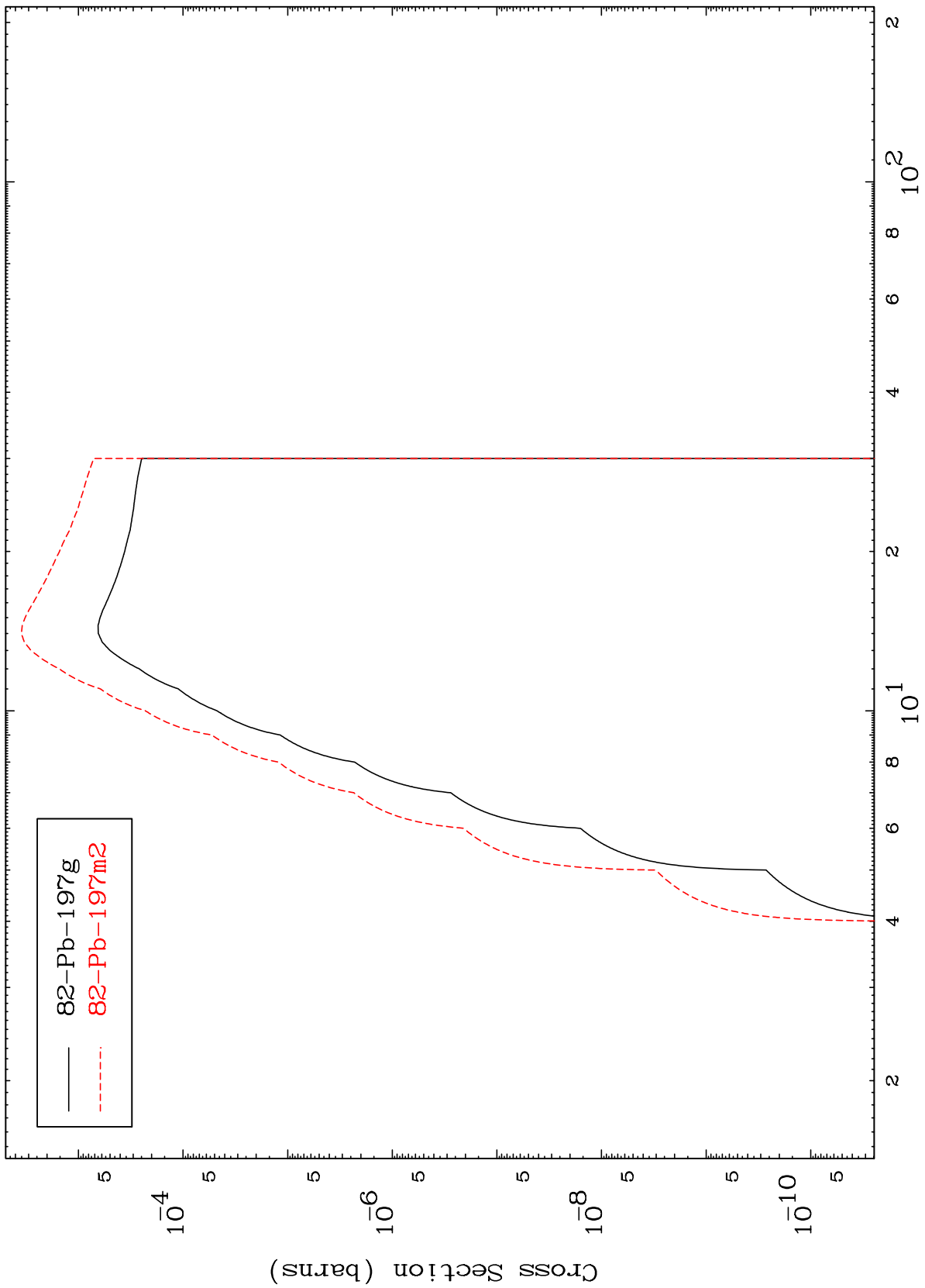
$^{83}\text{Bi-198m}$

Incident Energy (MeV)

MAT 8293

83-Bi-198m

(n,p)  
Radionuclide Production Cross Section



22

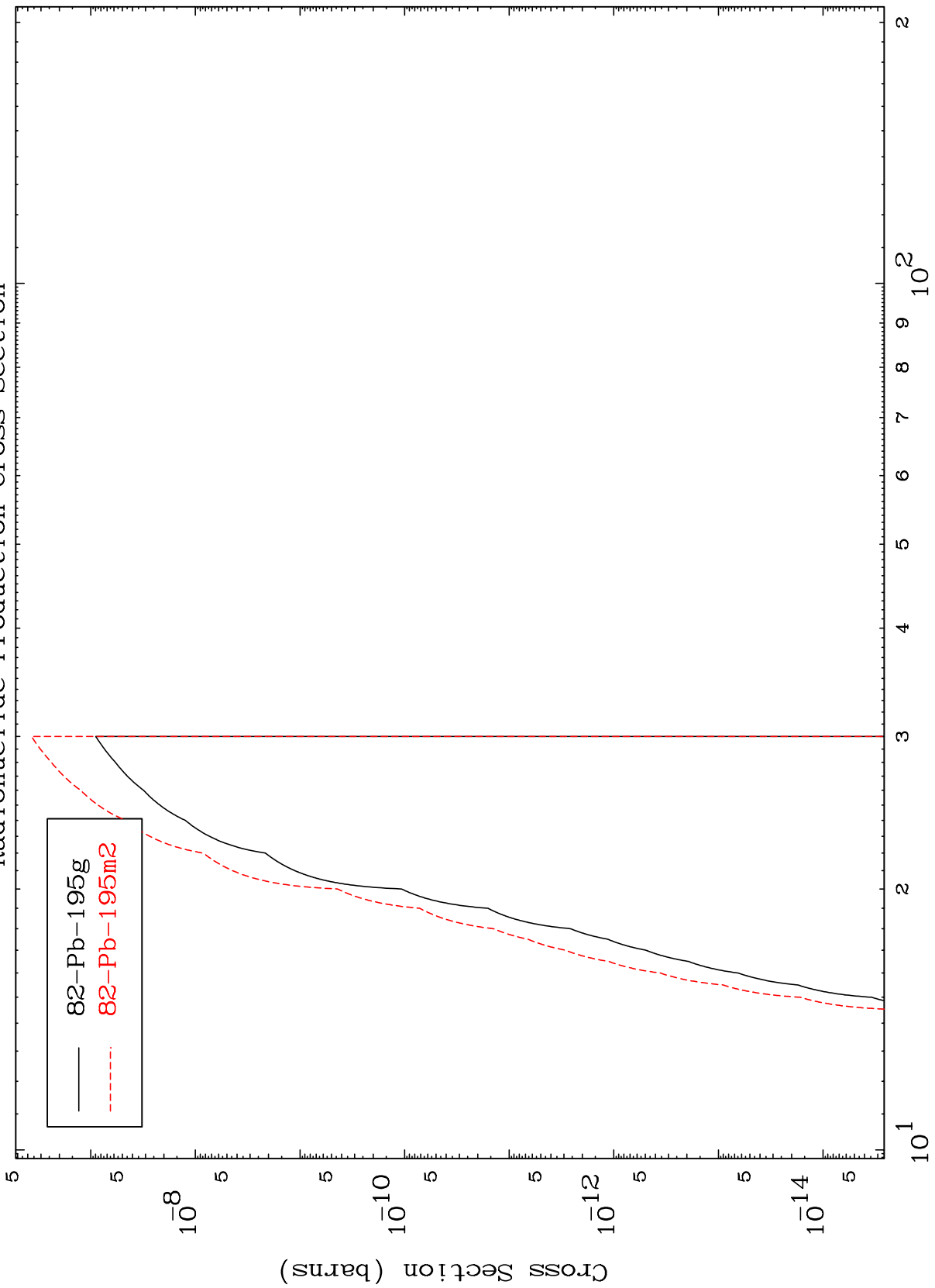
83-Bi-198m

Incident Energy (MeV)

MAT 8293

<sup>83</sup>Bi-198m

(n,t)  
Radionuclide Production Cross Section



<sup>83</sup>Bi-198m

Incident Energy (MeV)

23

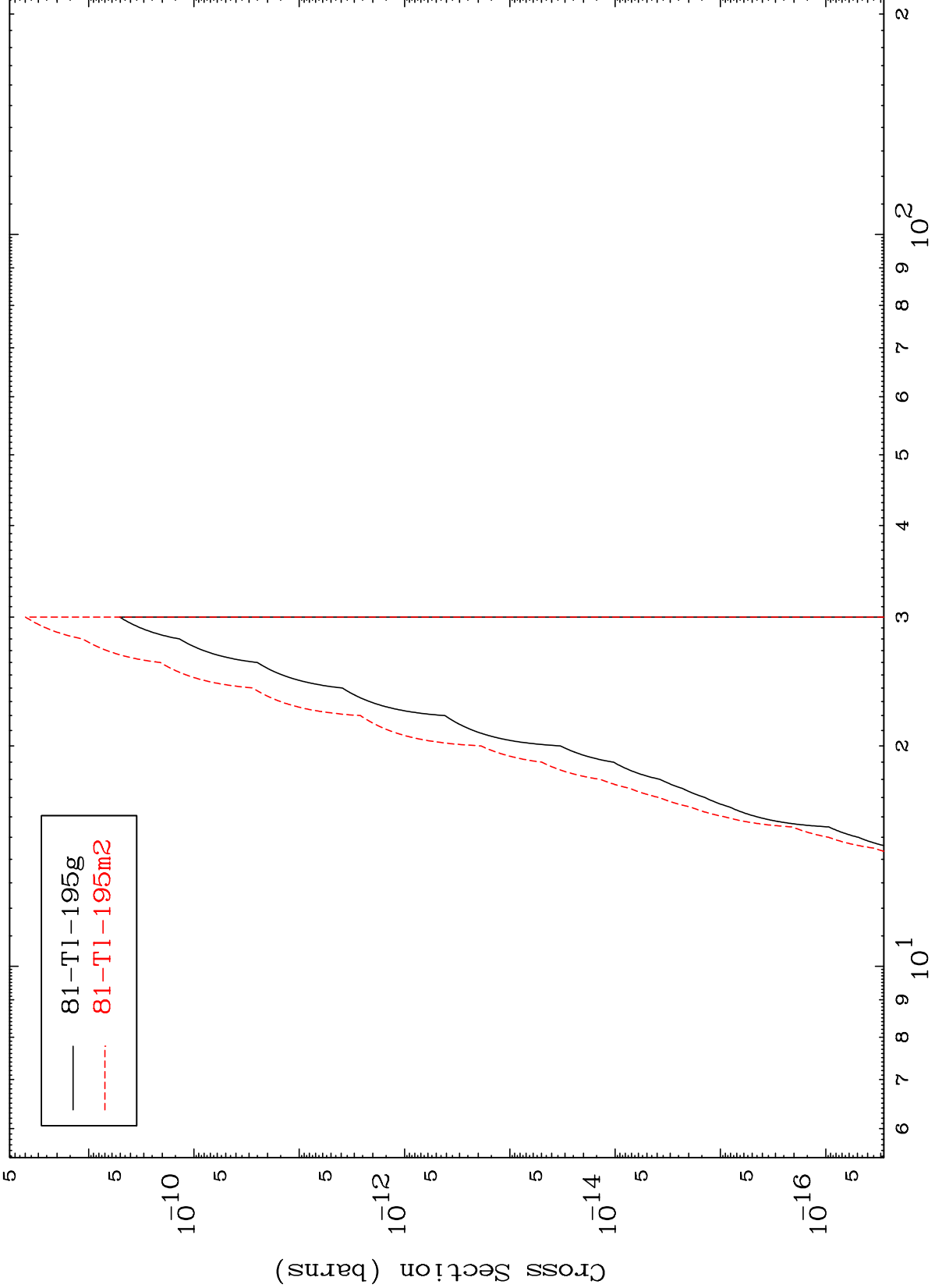


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(n,He-3)

83-Bi-198m

Radionuclide Production Cross Section



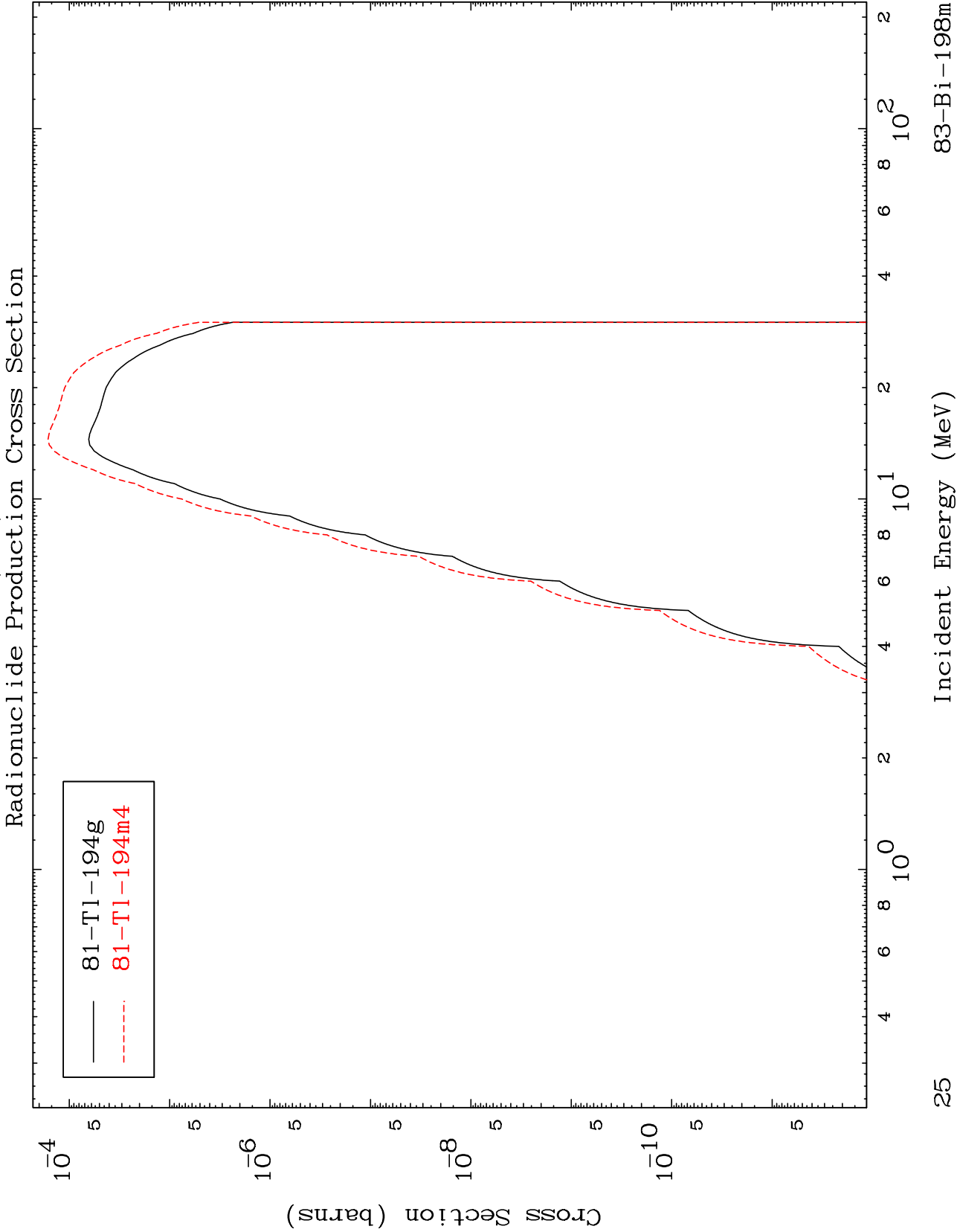
24

Incident Energy (MeV)

83-Bi-198m

MAT 8293

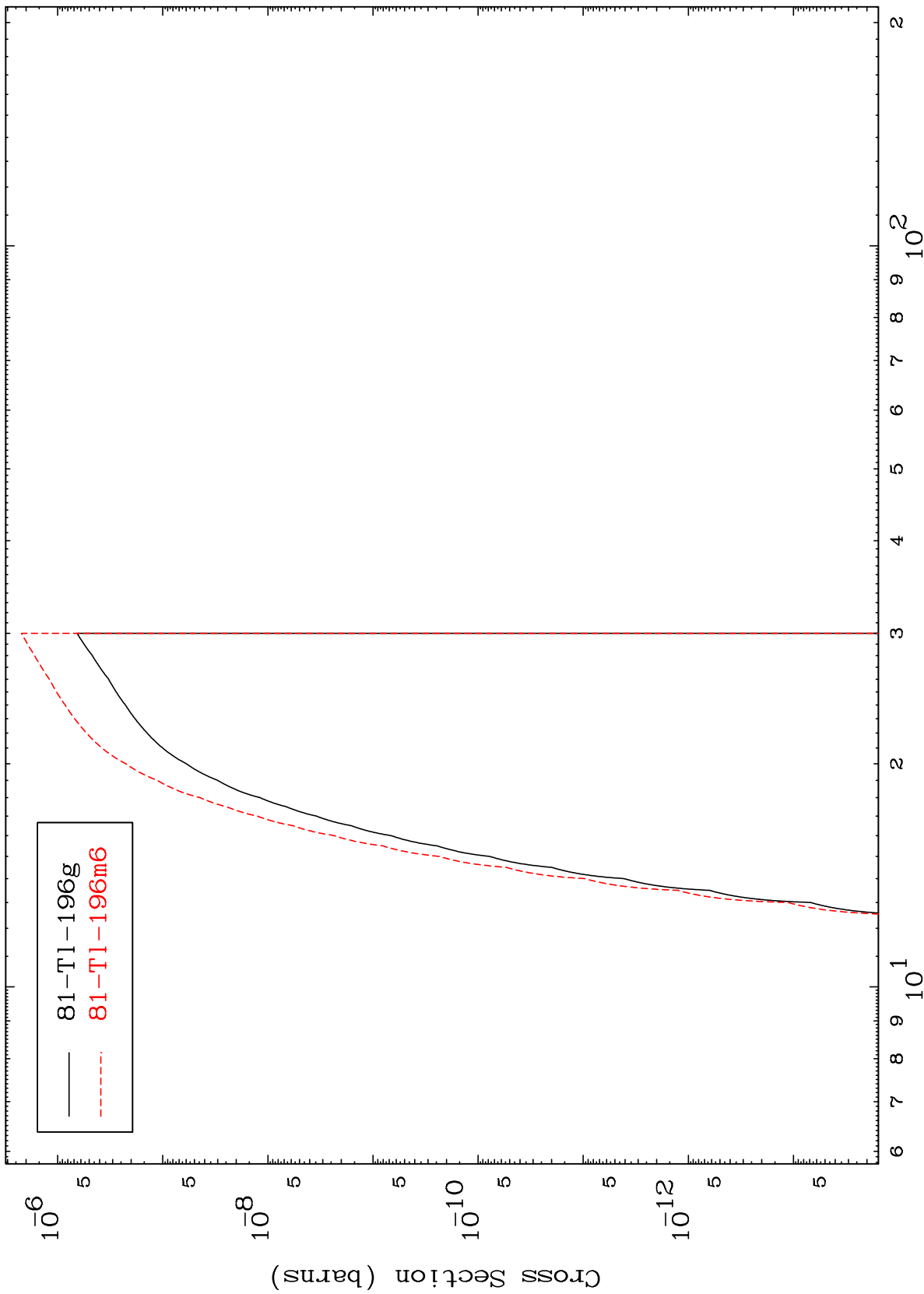
<sup>83</sup>Bi-198m



MAT 8293

83-Bi-198m

(n,2p)  
Radionuclide Production Cross Section



26

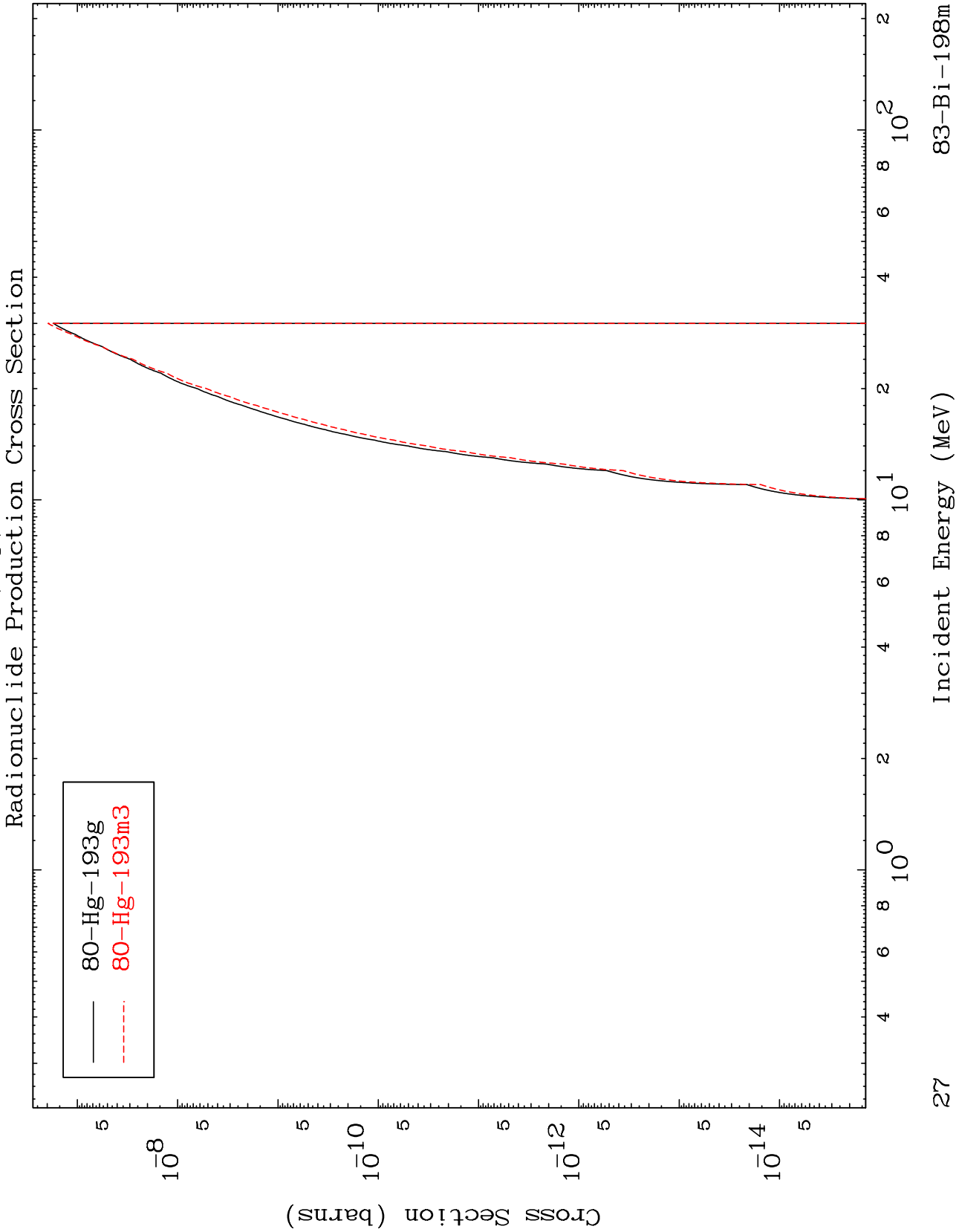
Incident Energy (MeV)

83-Bi-198m

MAT 8293

(n,p)  $\alpha$

$^{83}\text{Bi-198m}$



Radionuclide Production Cross Section

