

Program EVALPLOT  
(Version 2021-1)

by

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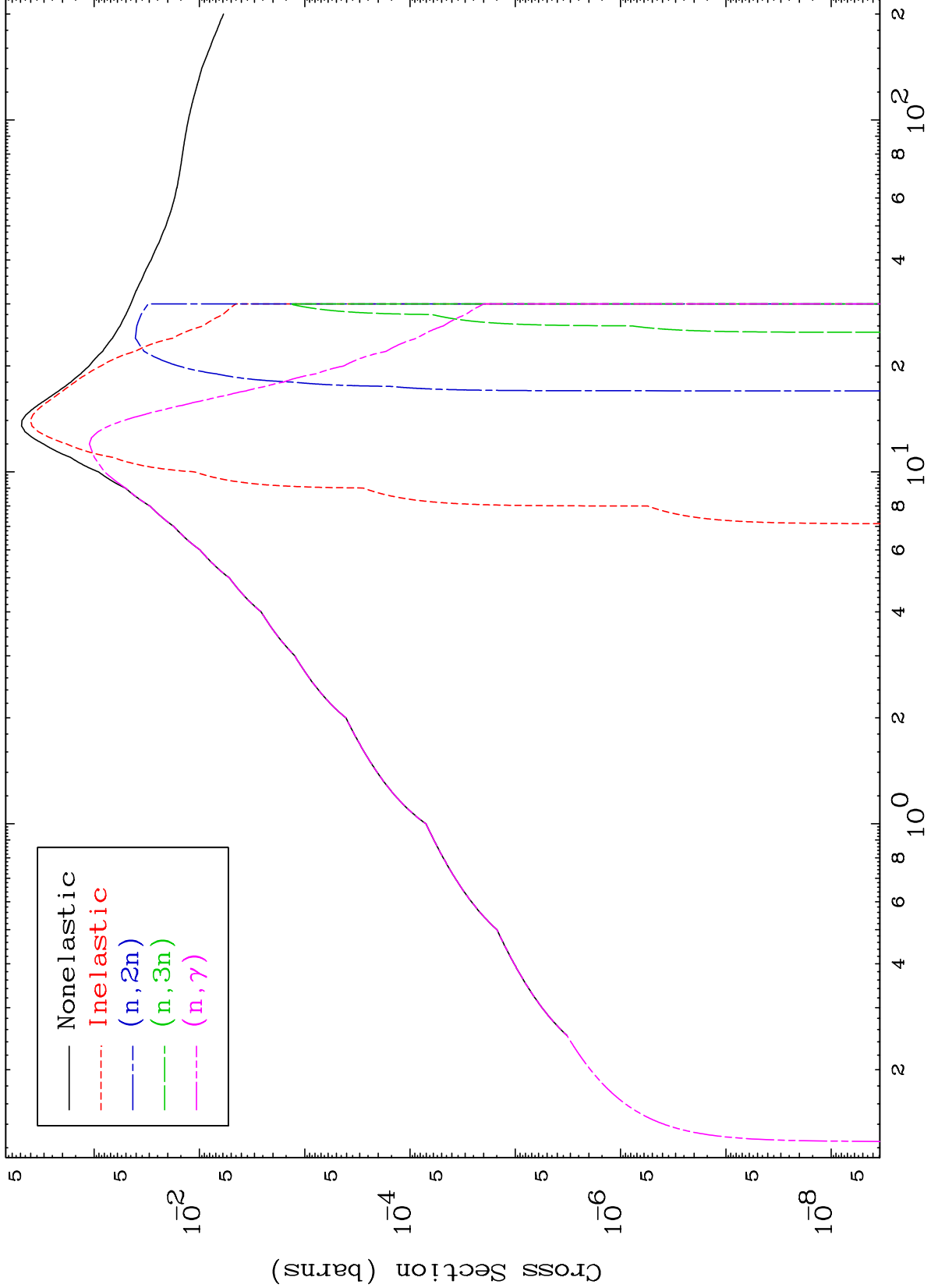
Press Mouse Button to Start

MAT 8294

Photon Major

83-Bi-198n

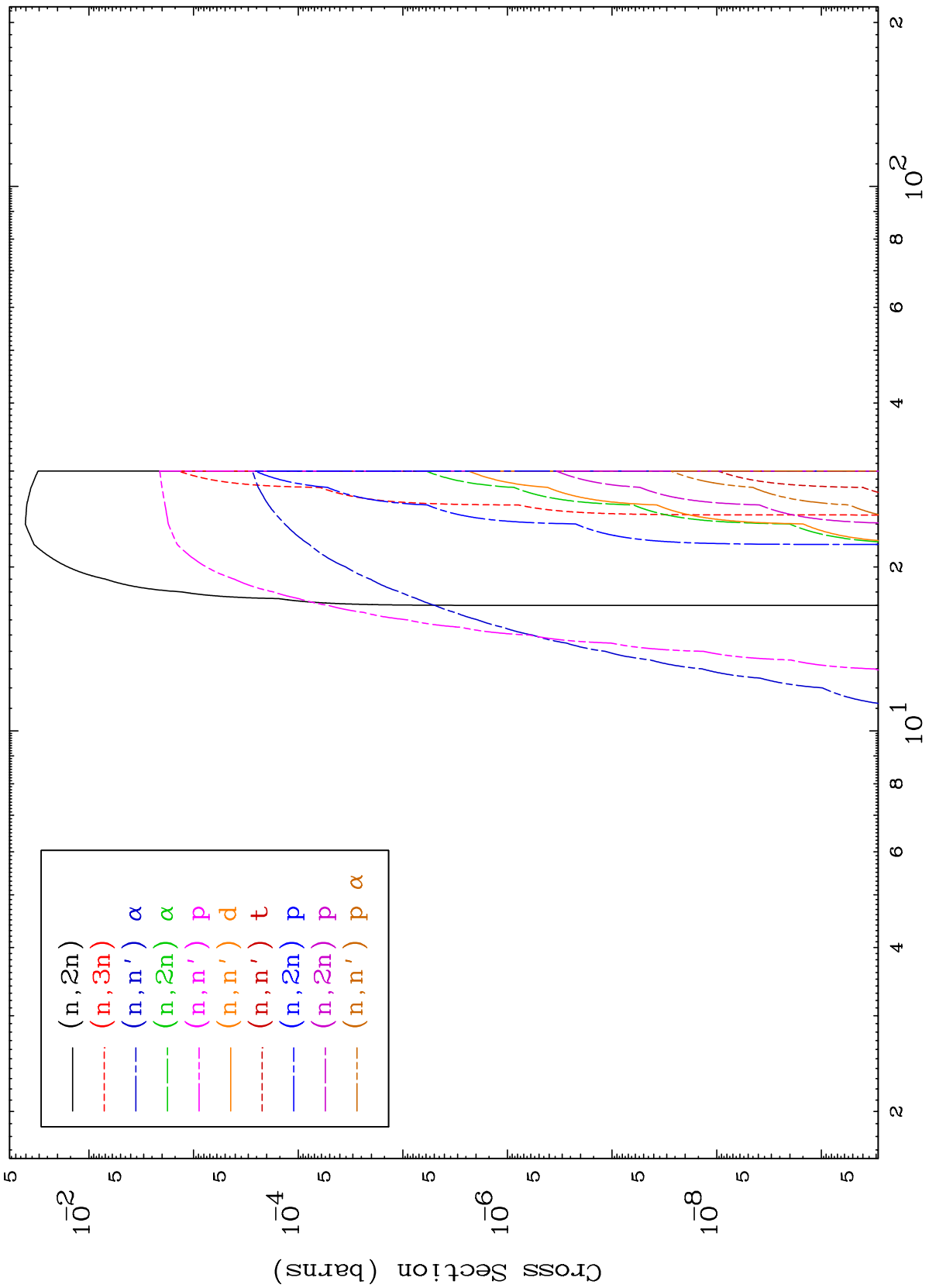
0 Kelvin Cross Sections



MAT 8294

Photon Neutron Absorption  
0 Kelvin Cross Sections

83-Bi-198n



83-Bi-198n

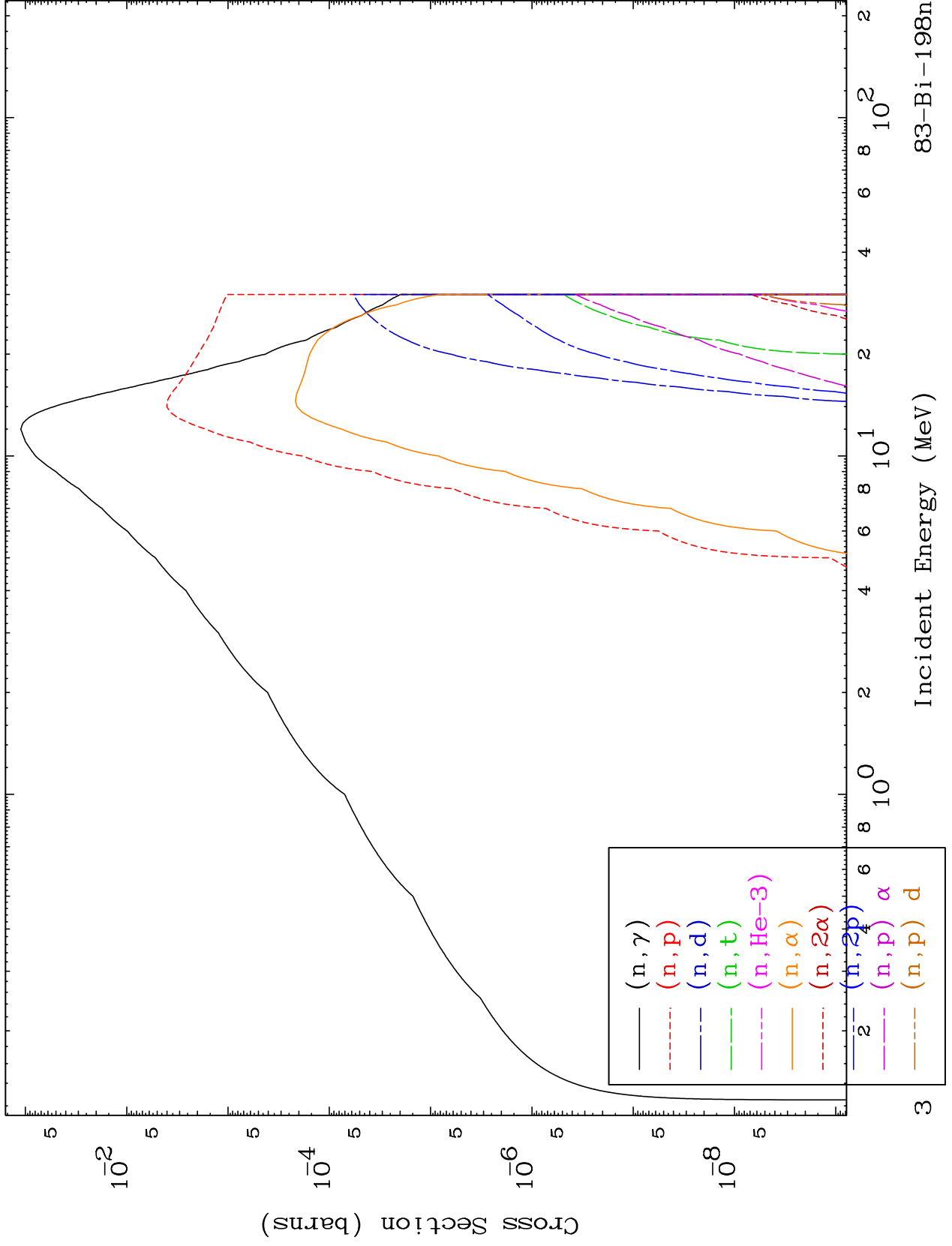
Incident Energy (MeV)

2

MAT 8294

Photon Neutron Absorption  
0 Kelvin Cross Sections

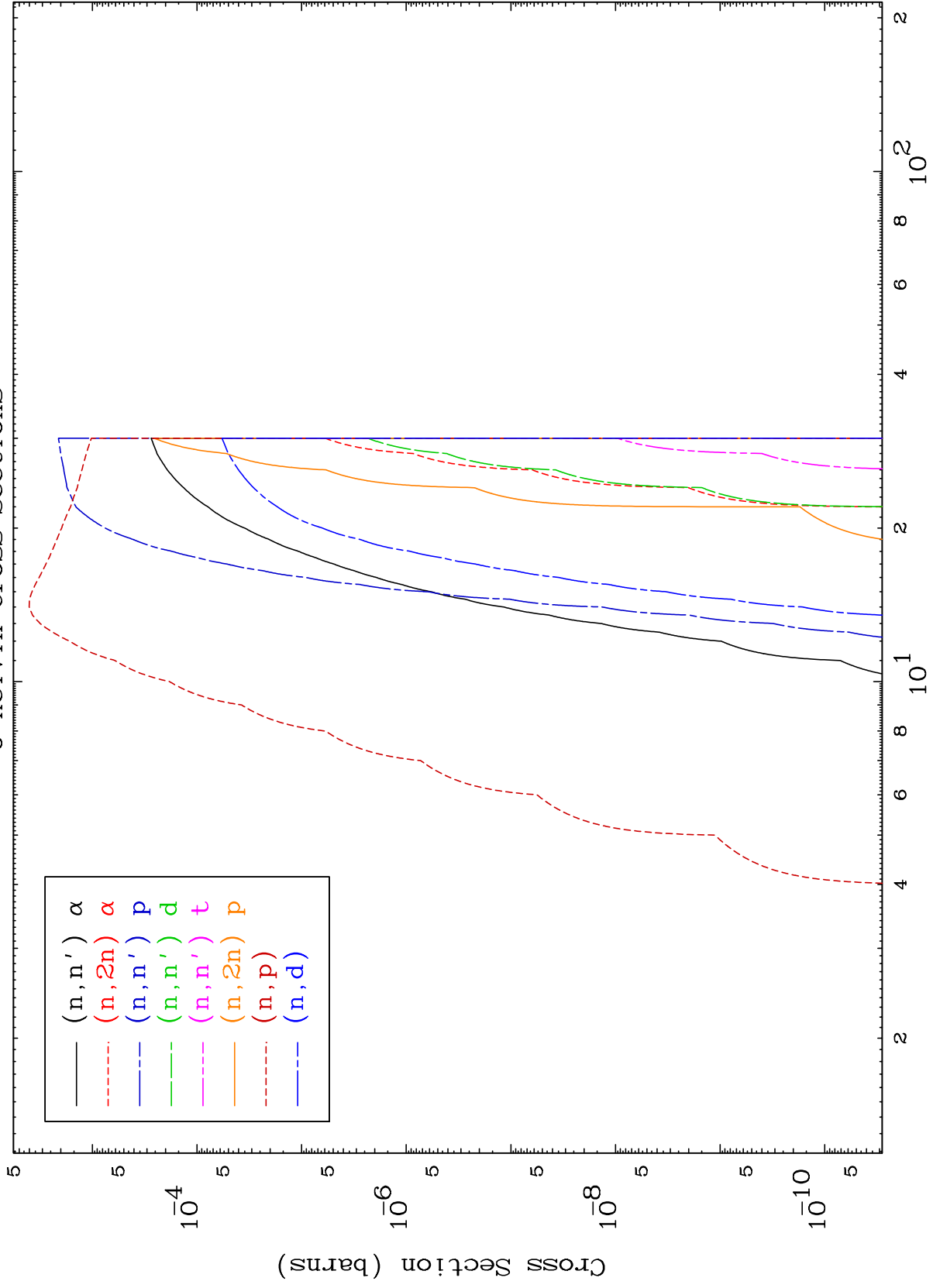
83-Bi-198n



MAT 8294

Photon Charged Particle  
0 Kelvin Cross Sections

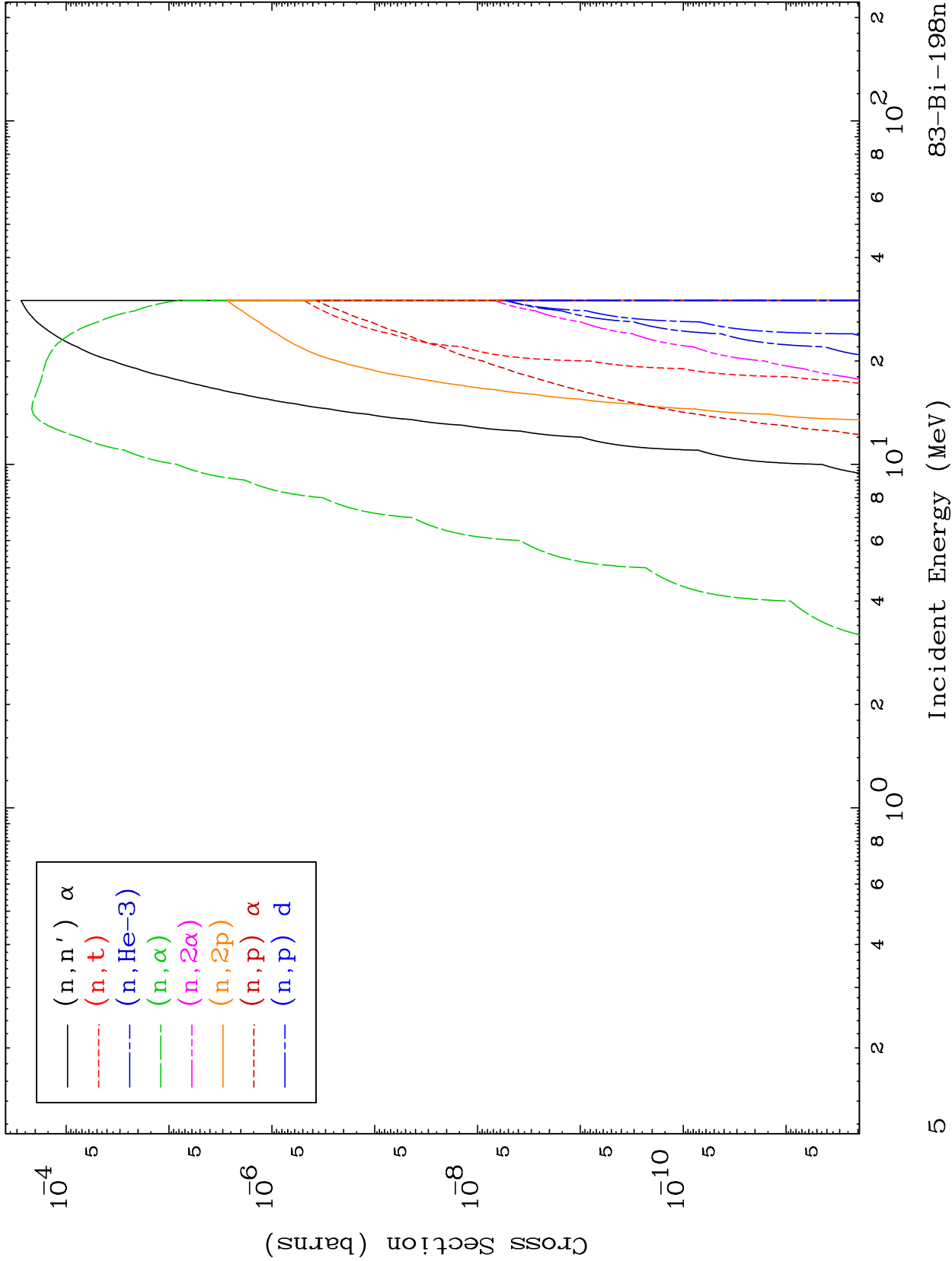
83-Bi-198n



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Photon Charged Particle  
0 Kelvin Cross Sections

83-Bi-198n

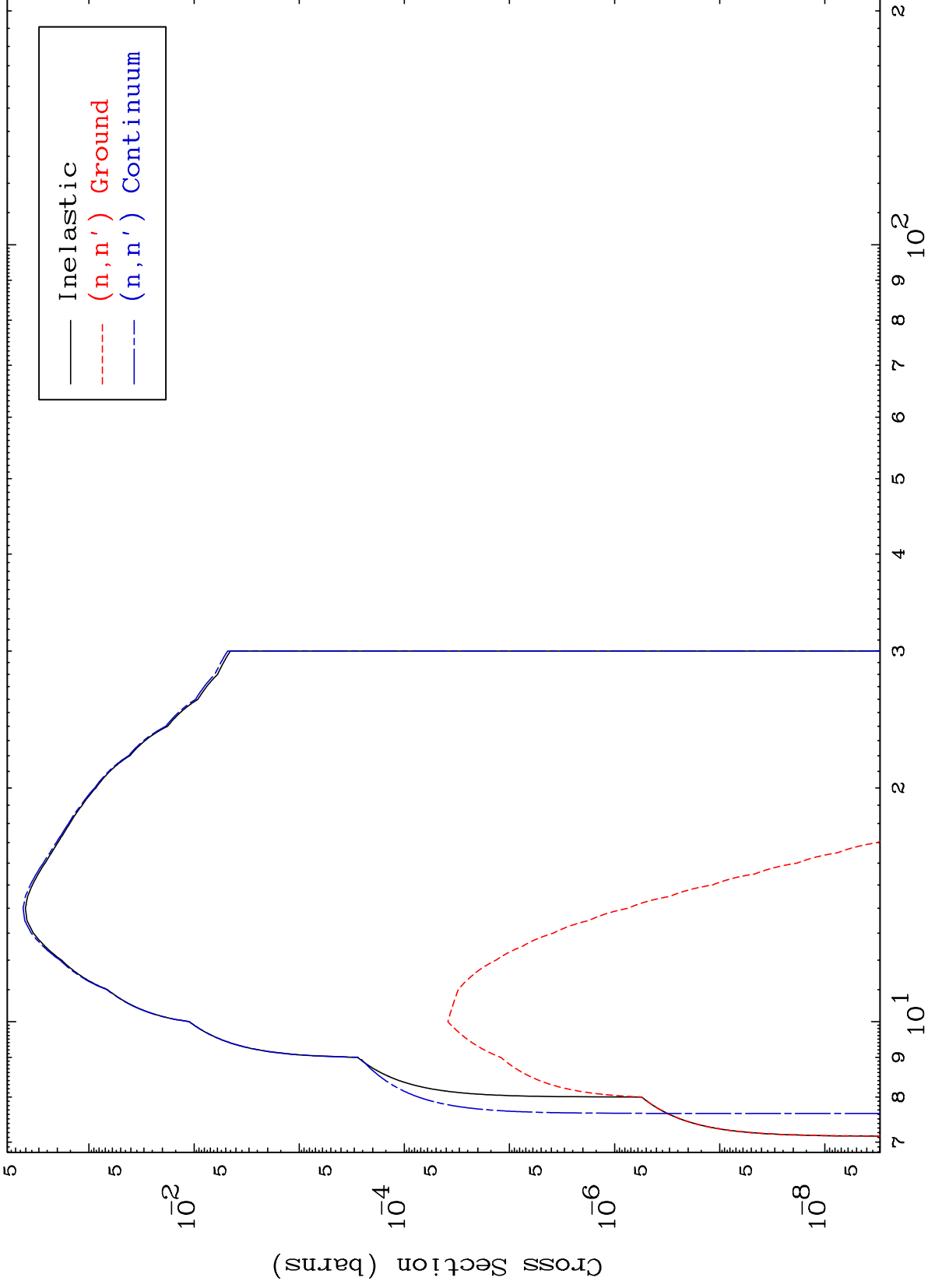


MAT 8294

( $\gamma, n'$ ) Levels

83-Bi-198n

0 Kelvin Cross Sections



Incident Energy (MeV)

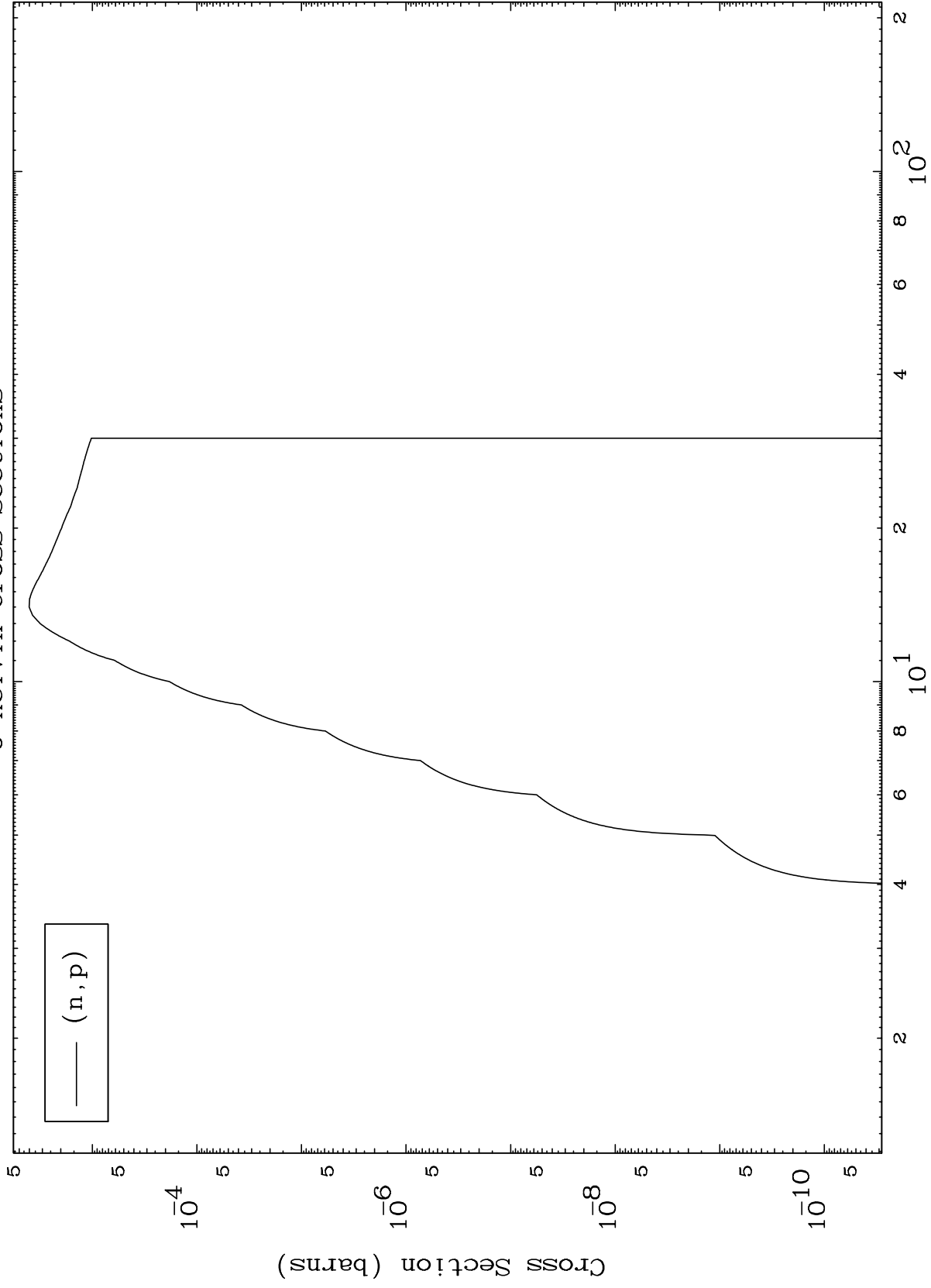
83-Bi-198n

6

MAT 8294

( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

83-Bi-198n

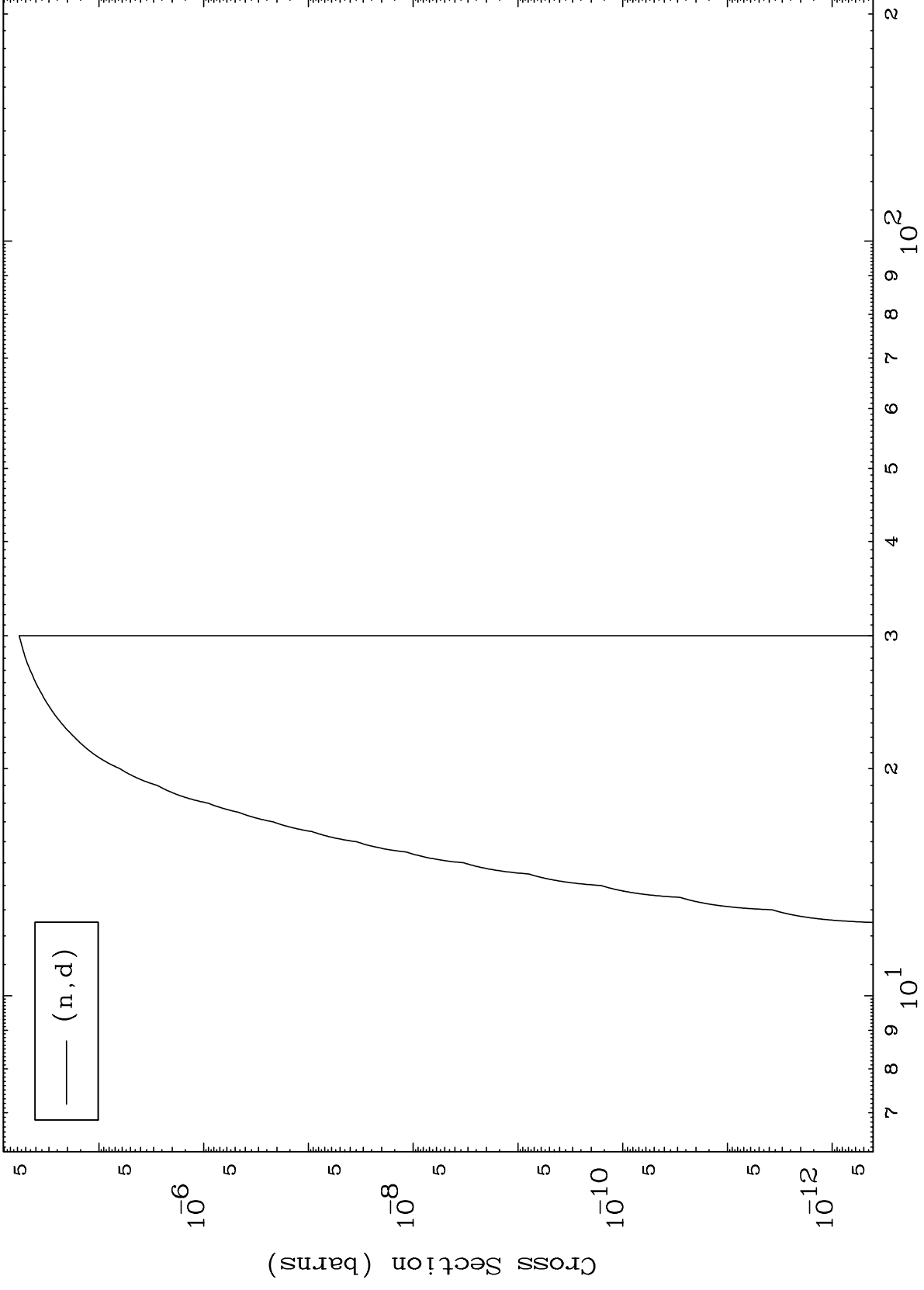




MAT 8294

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

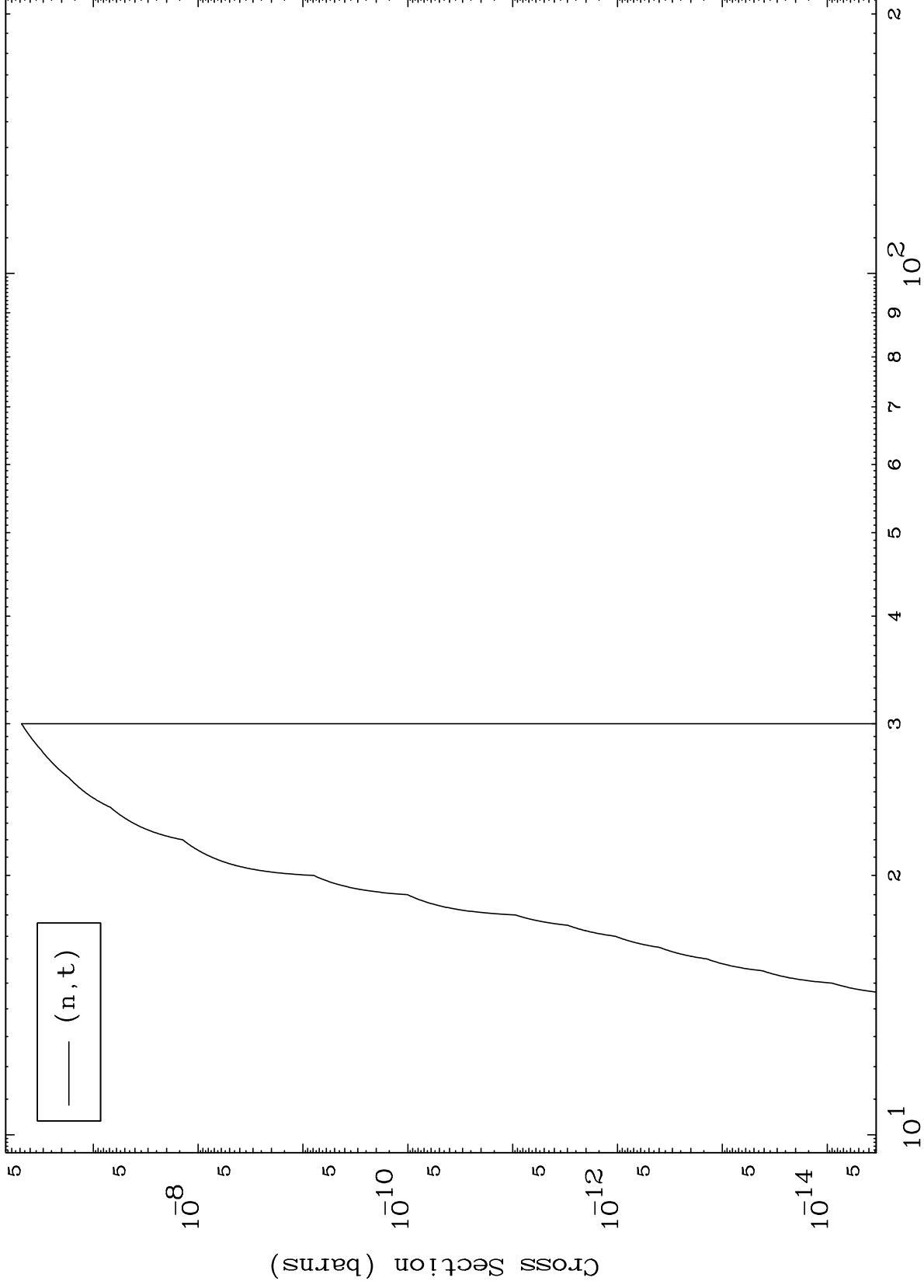
$^{83}\text{Bi}-198\text{n}$



MAT 8294

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

83-Bi-198n



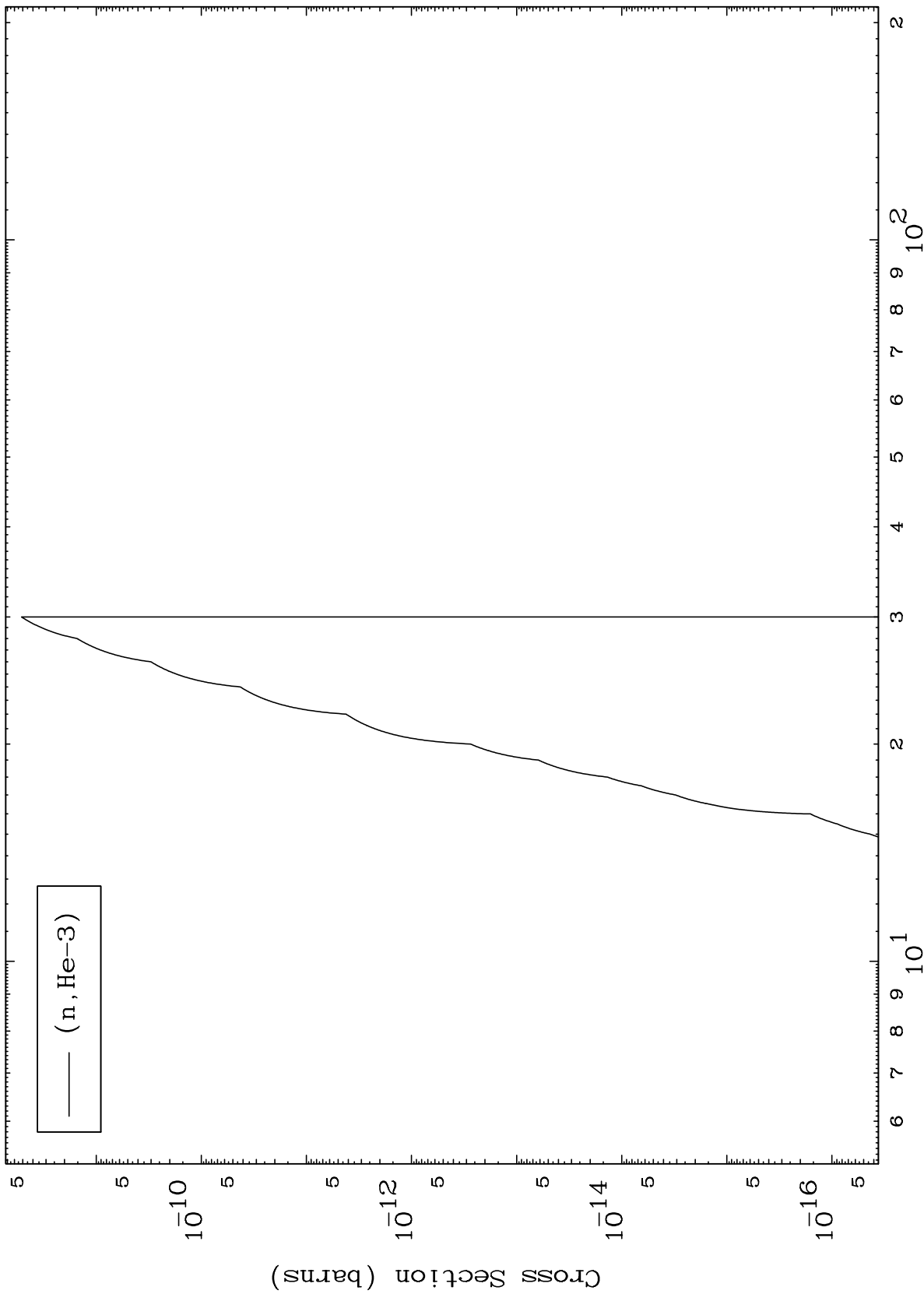
Incident Energy (MeV)

83-Bi-198n

MAT 8294

83-Bi-198n

( $\gamma, \text{He}3$ ) Levels  
0 Kelvin Cross Sections



10

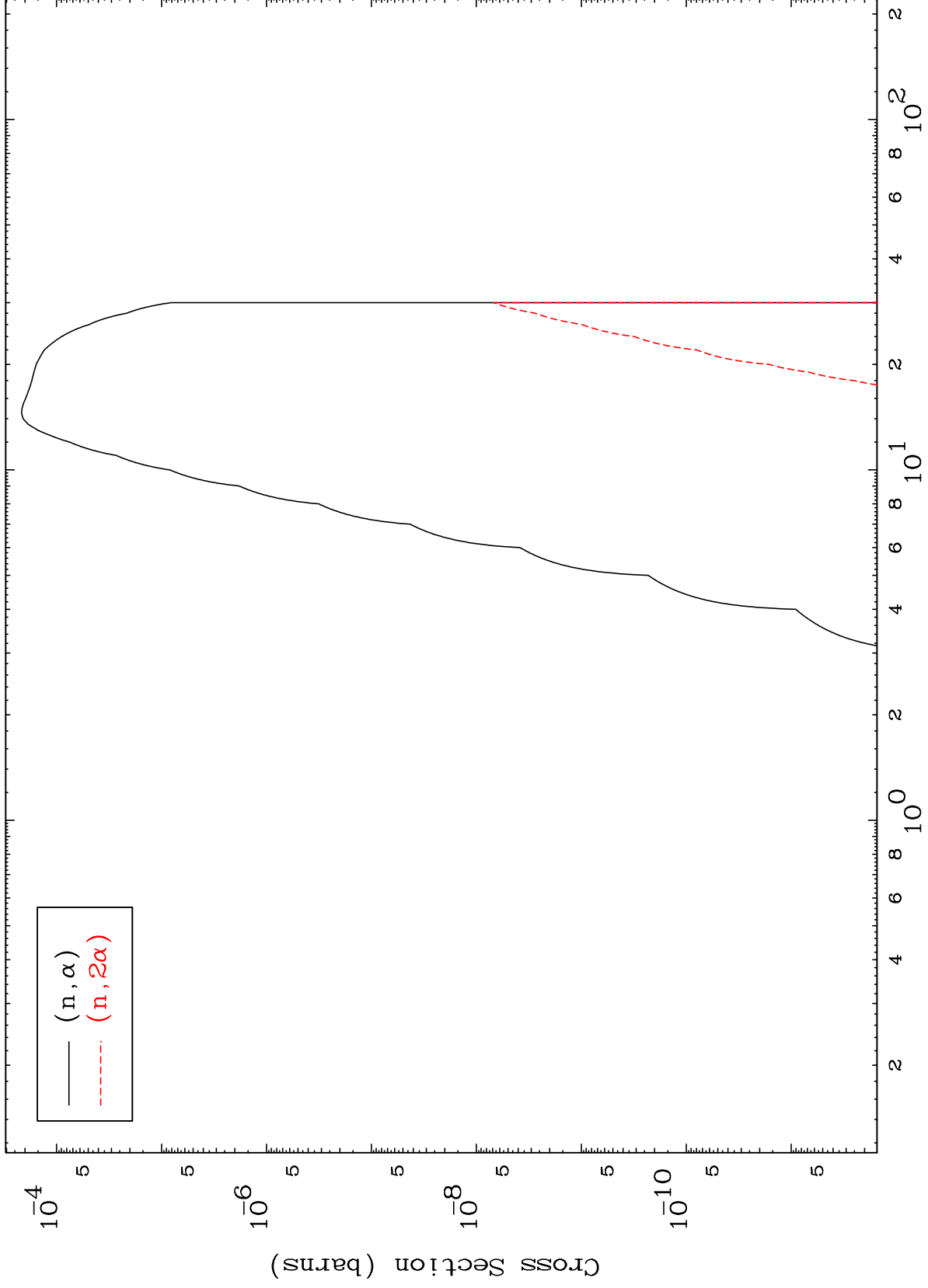
Incident Energy (MeV)

83-Bi-198n

MAT 8294

( $\gamma, \alpha$ ) Levels  
0 Kelvin Cross Sections

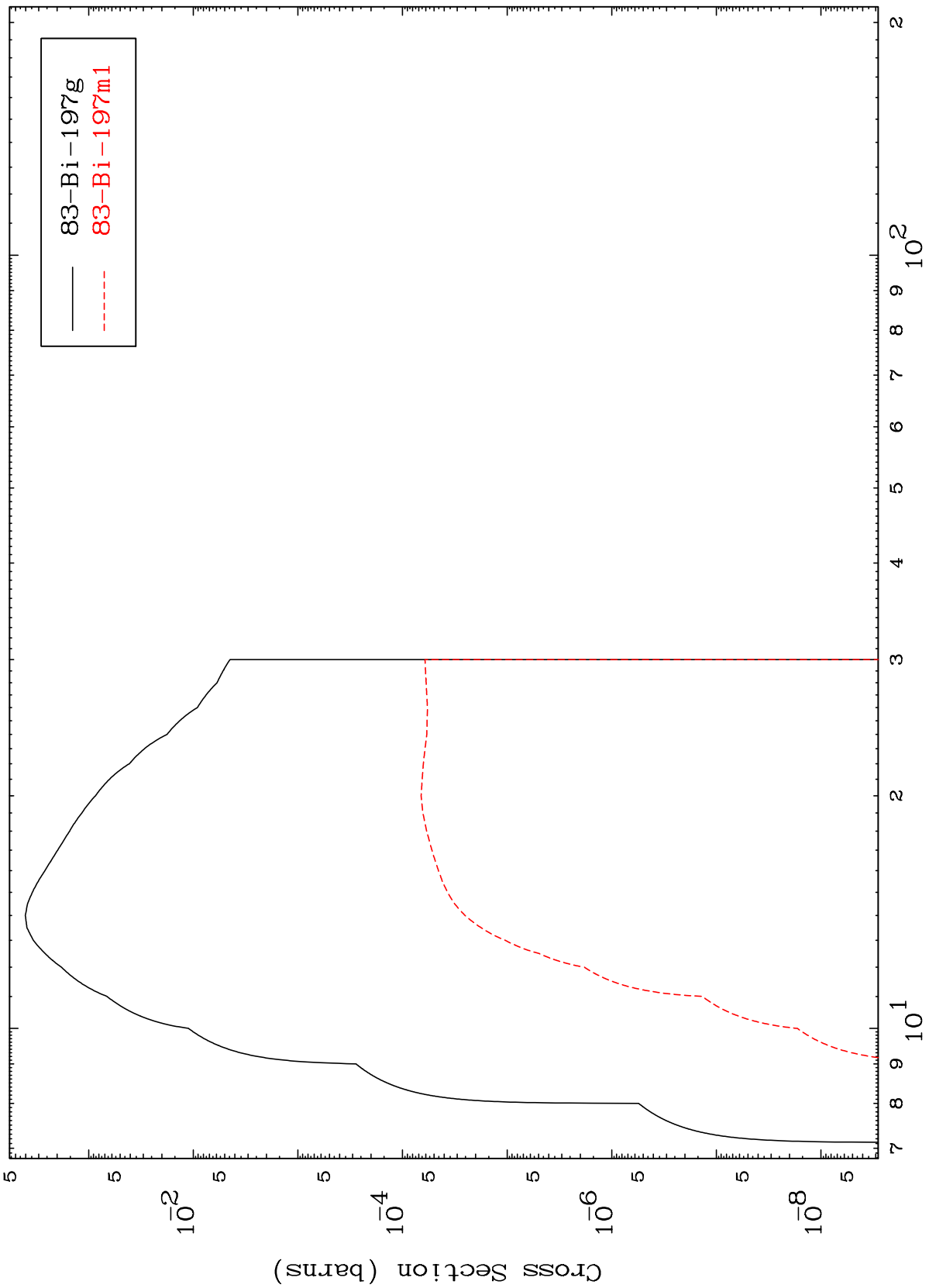
83-Bi-198n



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83-Bi-198n

Inelastic  
Radionuclide Production Cross Section



83-Bi-198n

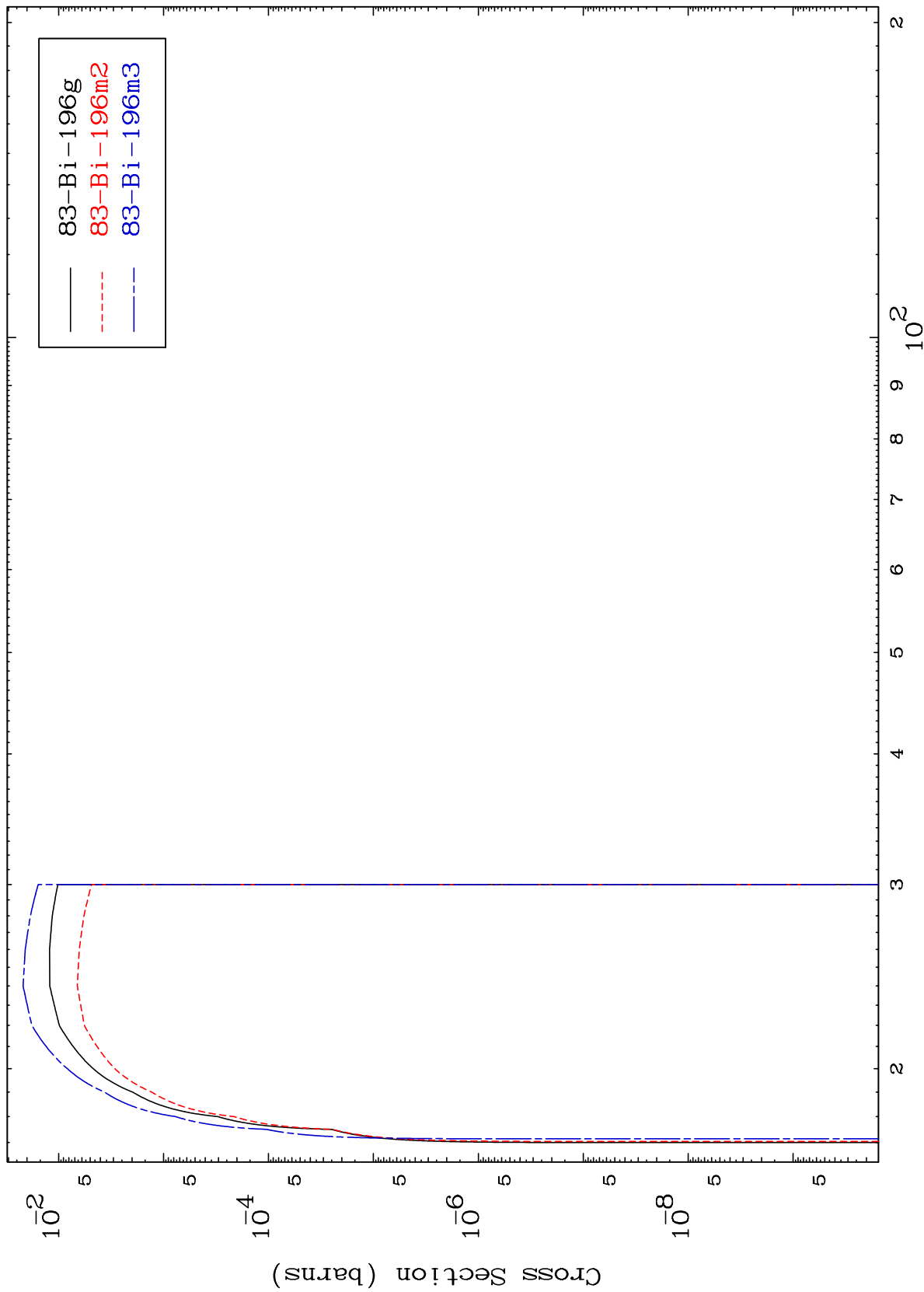
Incident Energy (MeV)

12

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<sup>83</sup>Bi-198n

(n,2n)  
Radionuclide Production Cross Section



13

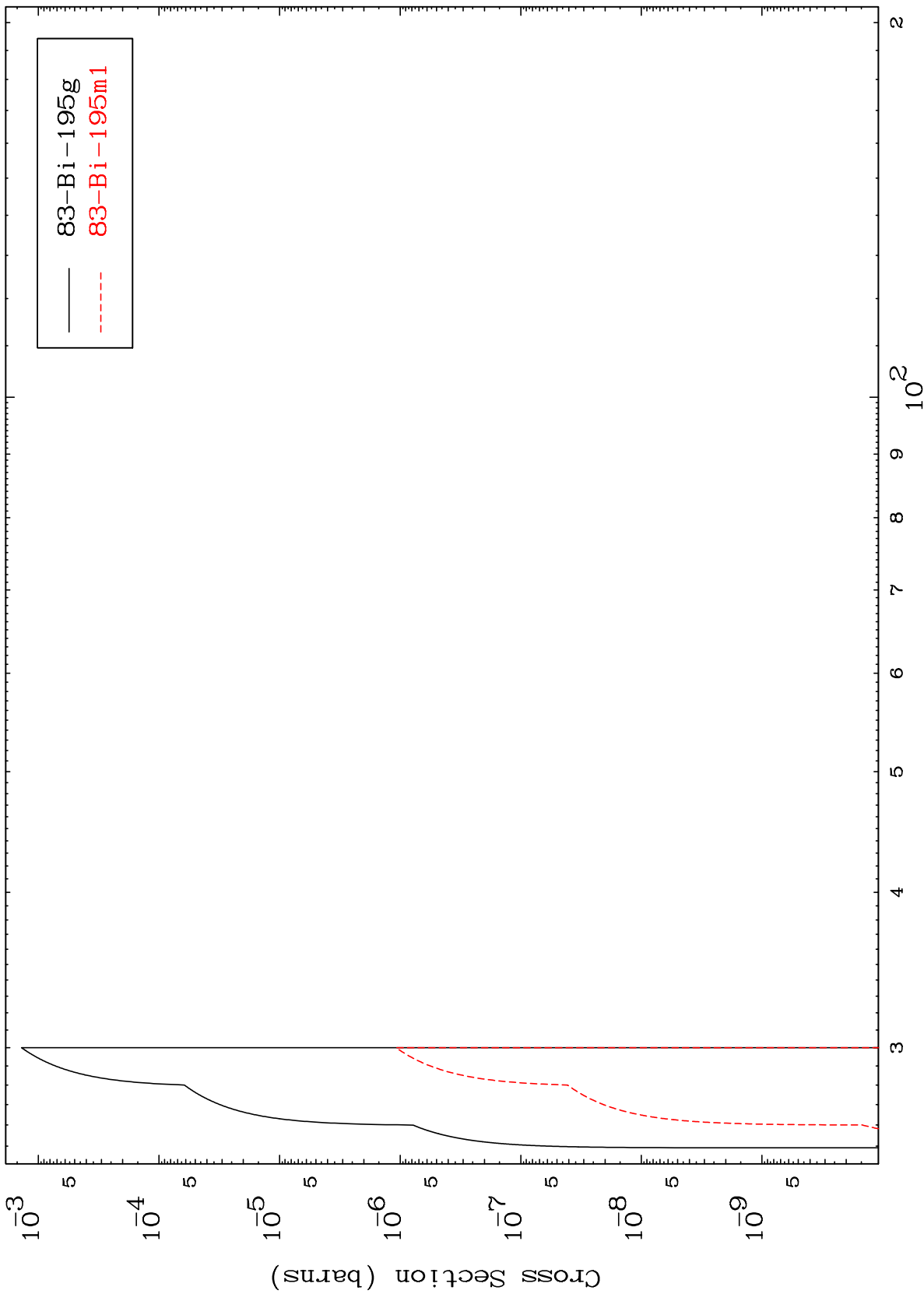
Incident Energy (MeV)

<sup>83</sup>Bi-198n

MAT 8294

83-Bi-198n

(n,3n)  
Radionuclide Production Cross Section



14

Incident Energy (MeV)

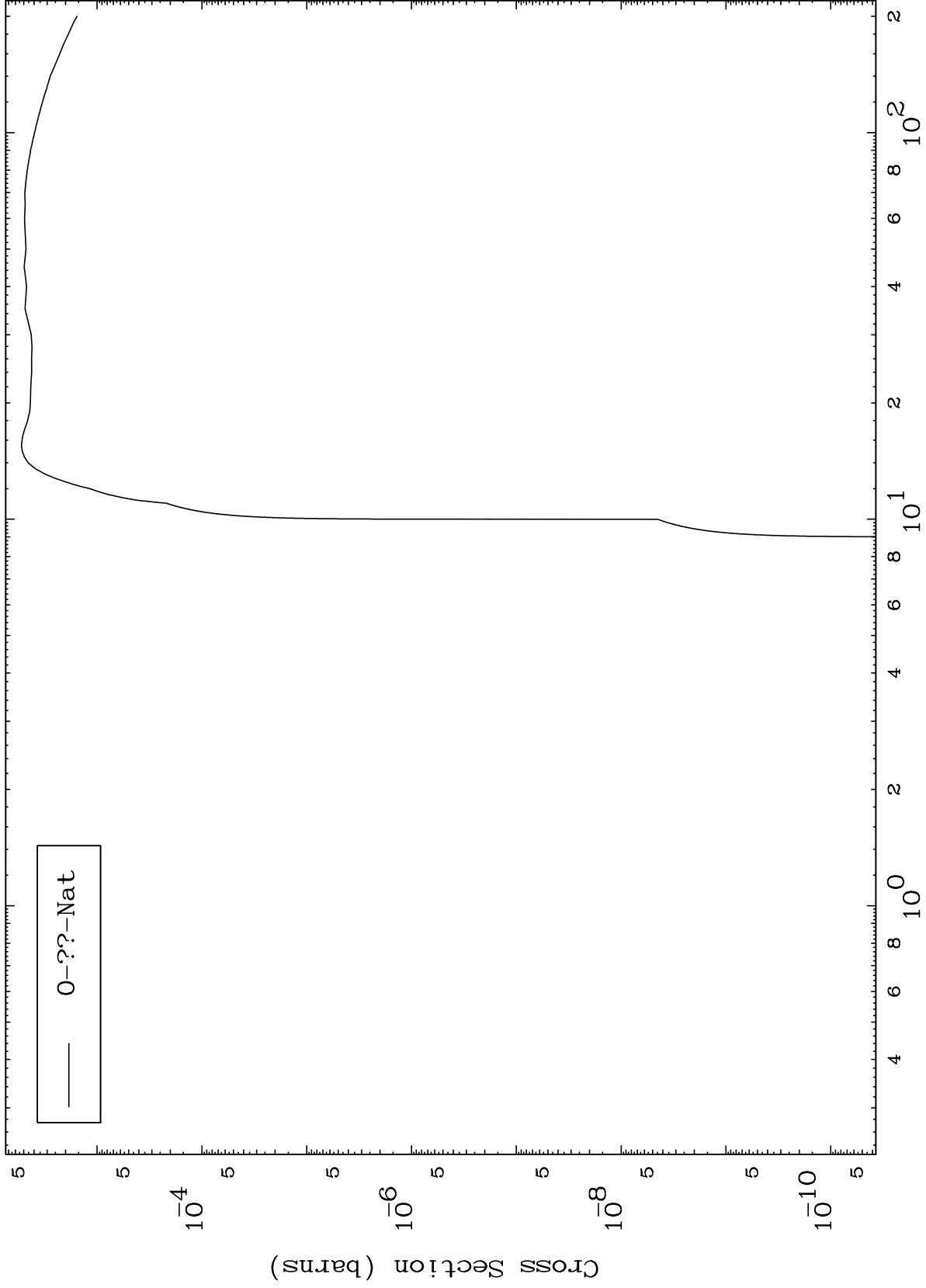
83-Bi-198n

MAT 8294

Fission

<sup>83</sup>Bi-198n

Radionuclide Production Cross Section



— 0-??-Nat

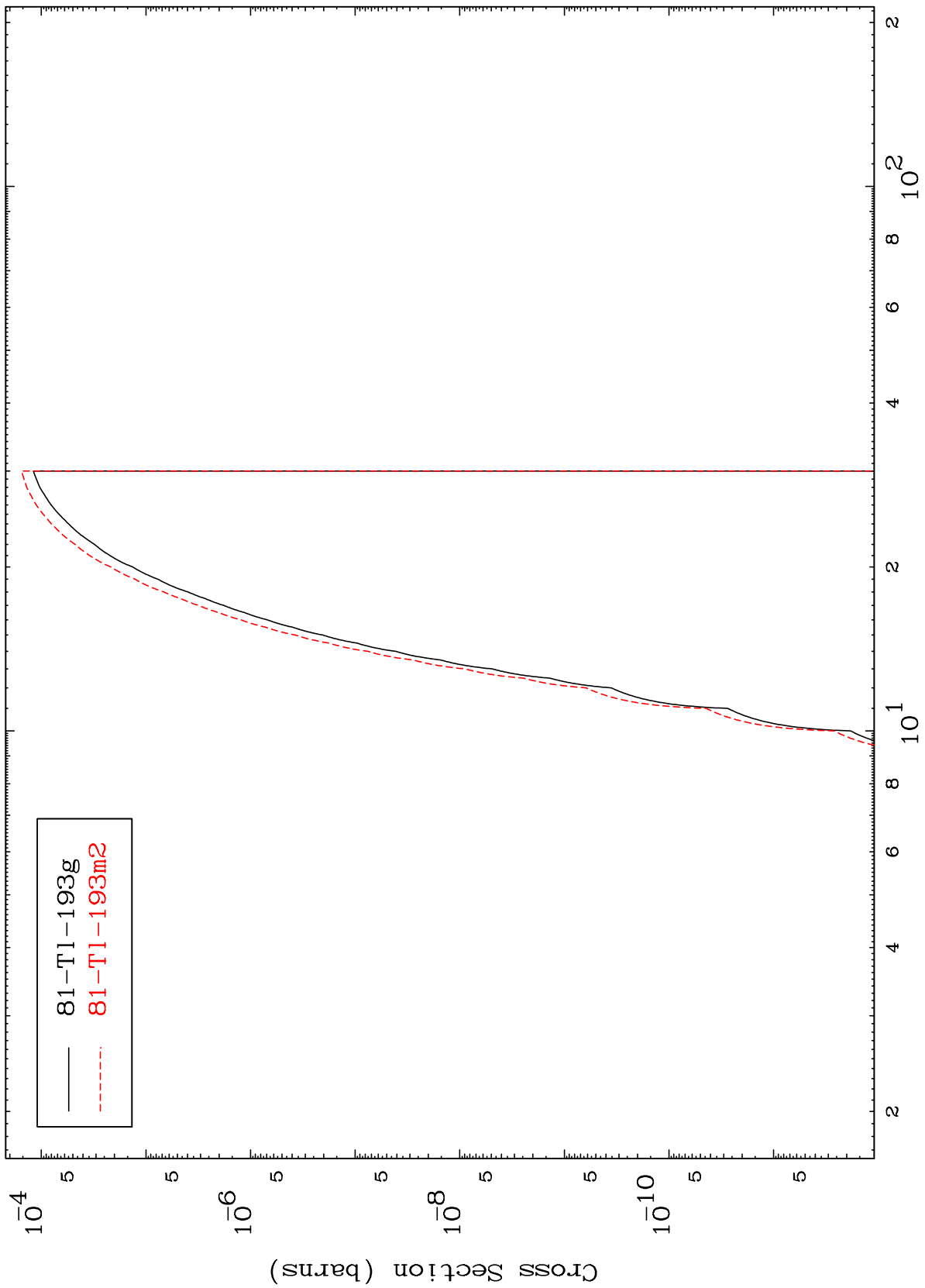


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83-Bi-198n

(n,n')  $\alpha$

Radionuclide Production Cross Section

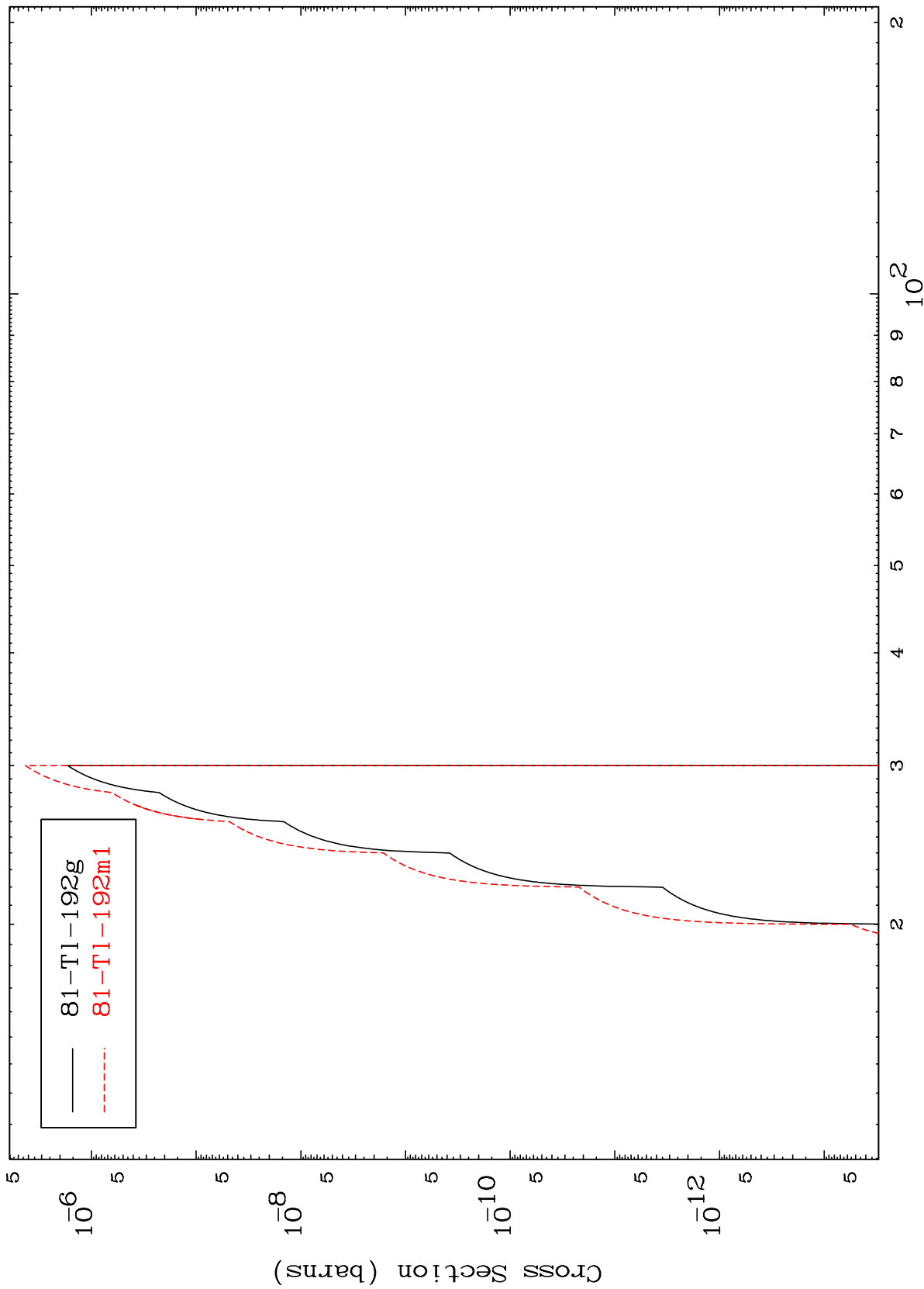


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(n,2n)  $\alpha$

83-Bi-198n

Radionuclide Production Cross Section

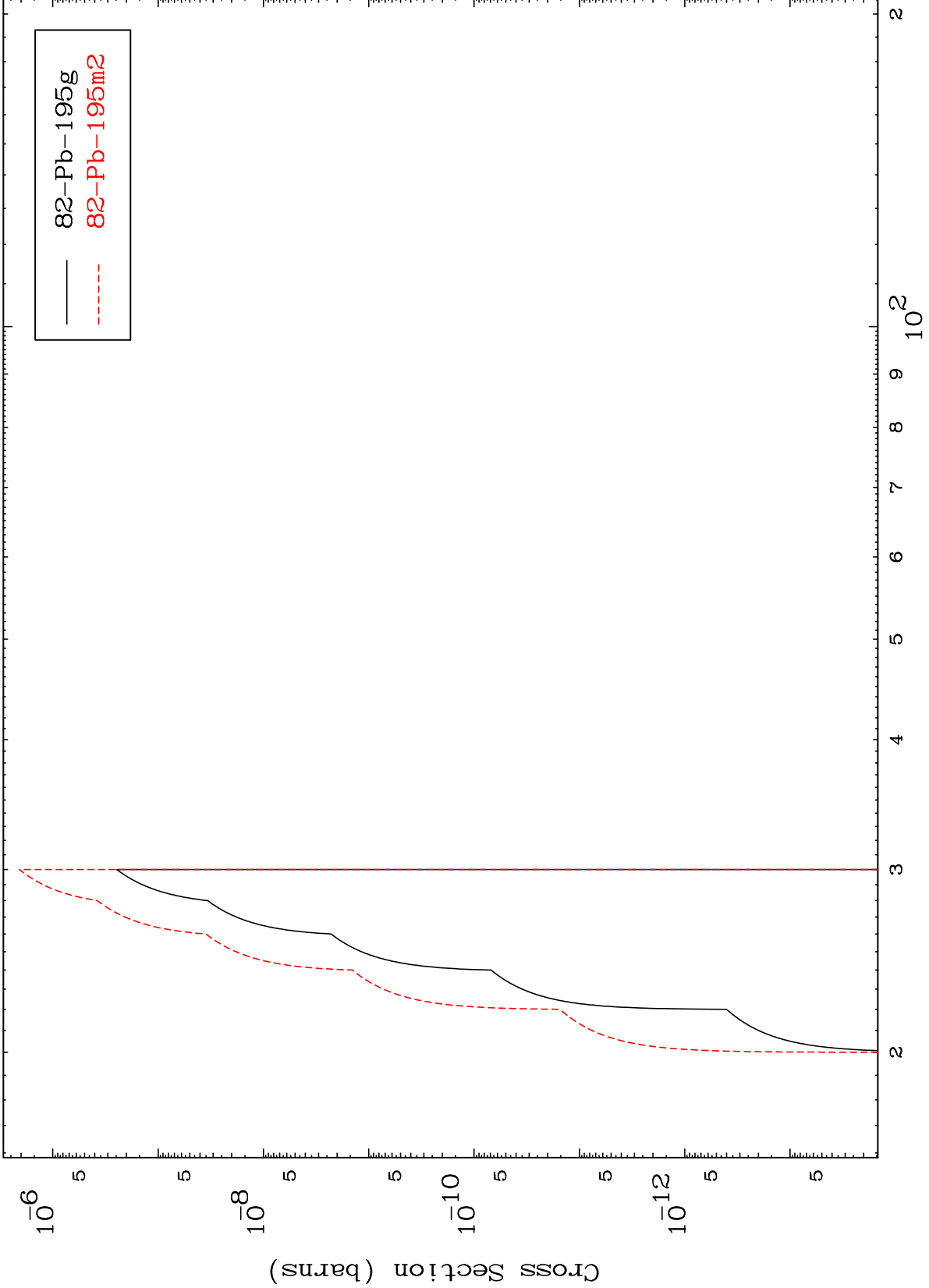


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(n,n') d

83-Bi-198n

Radionuclide Production Cross Section



18

Incident Energy (MeV)

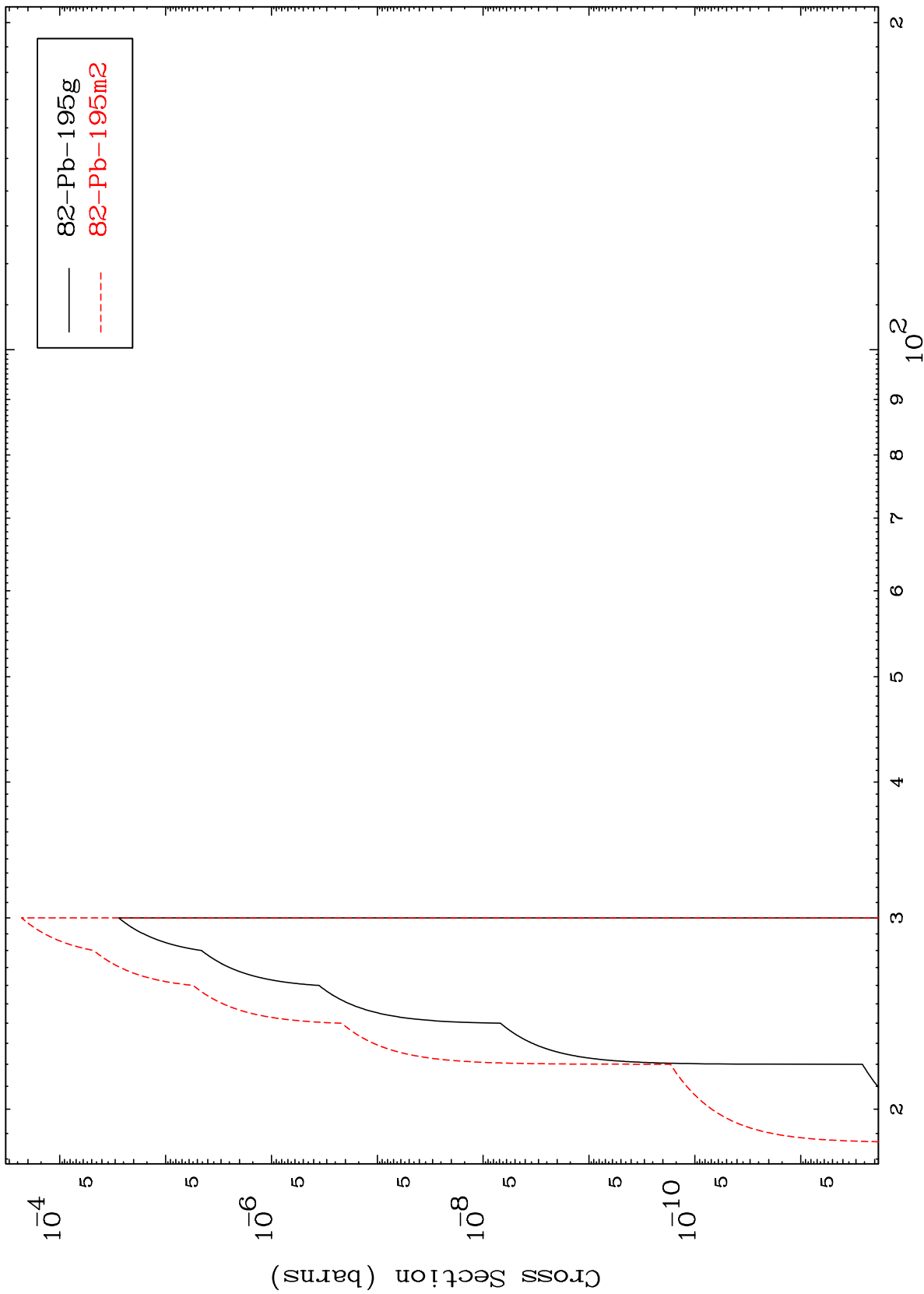
83-Bi-198n

MAT 8294

(n,2n) p

83-Bi-198n

Radionuclide Production Cross Section



19

Incident Energy (MeV)

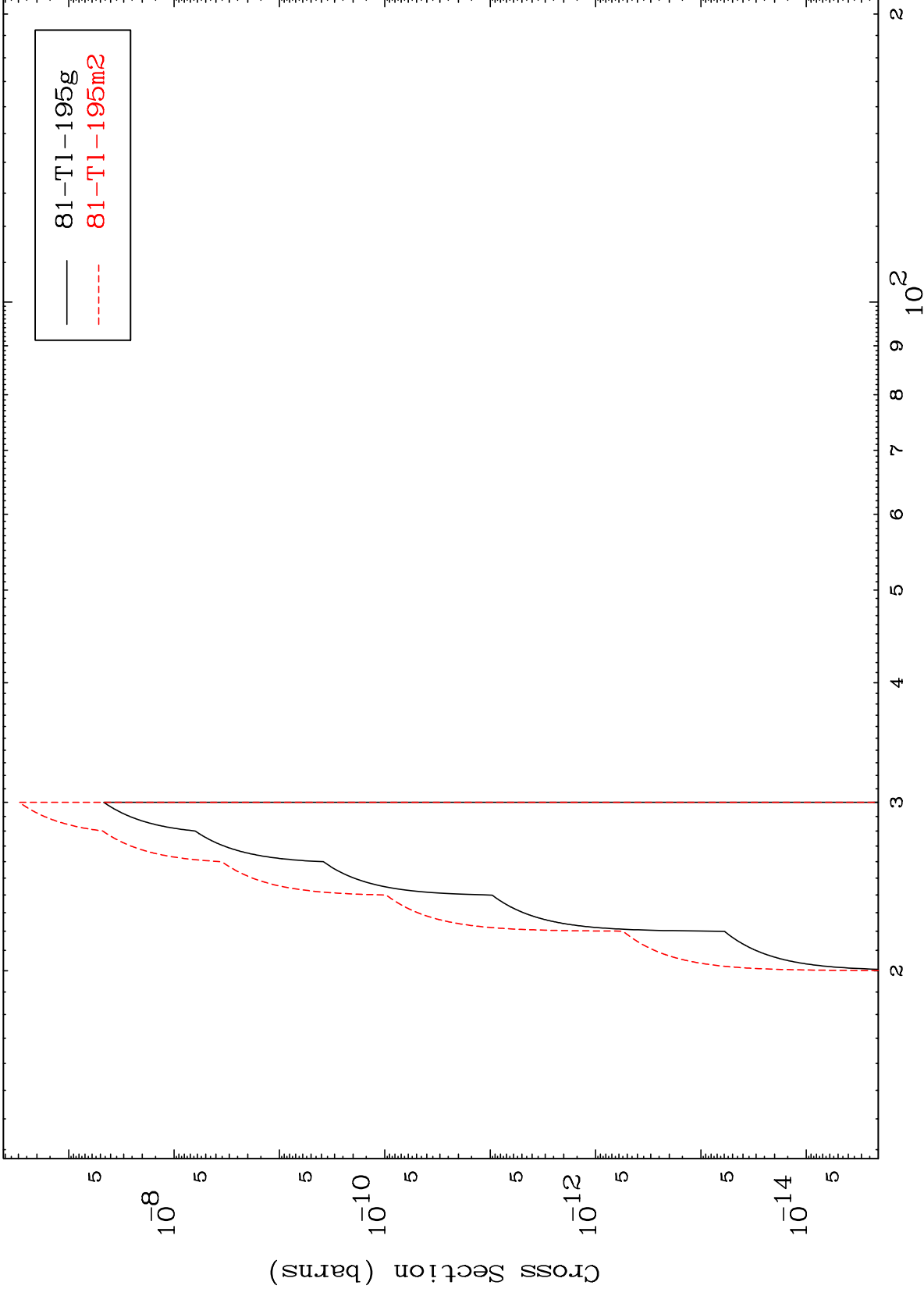
83-Bi-198n

MAT 8294

(n,2n) p

83-Bi-198n

Radionuclide Production Cross Section



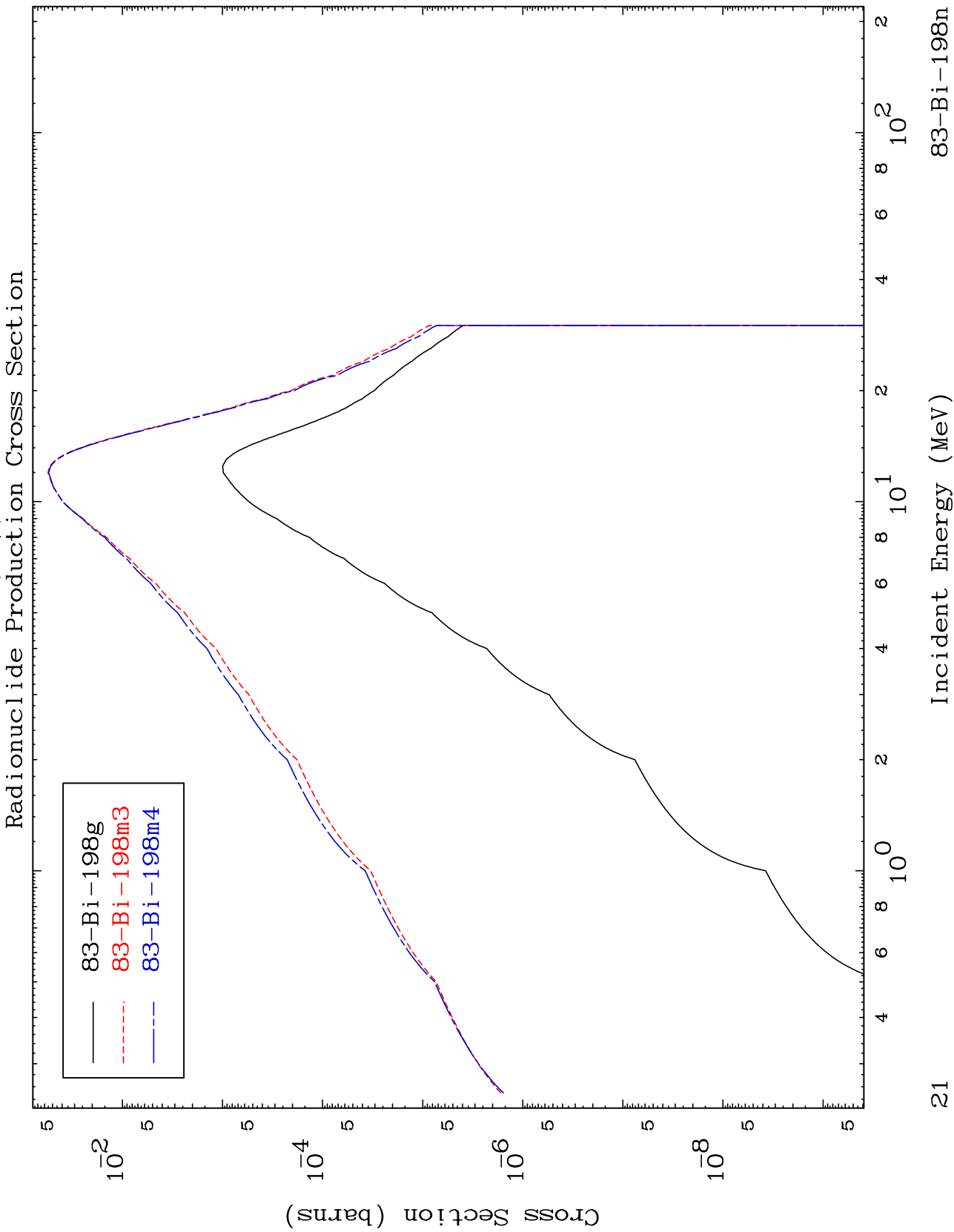
20

Incident Energy (MeV)

83-Bi-198n

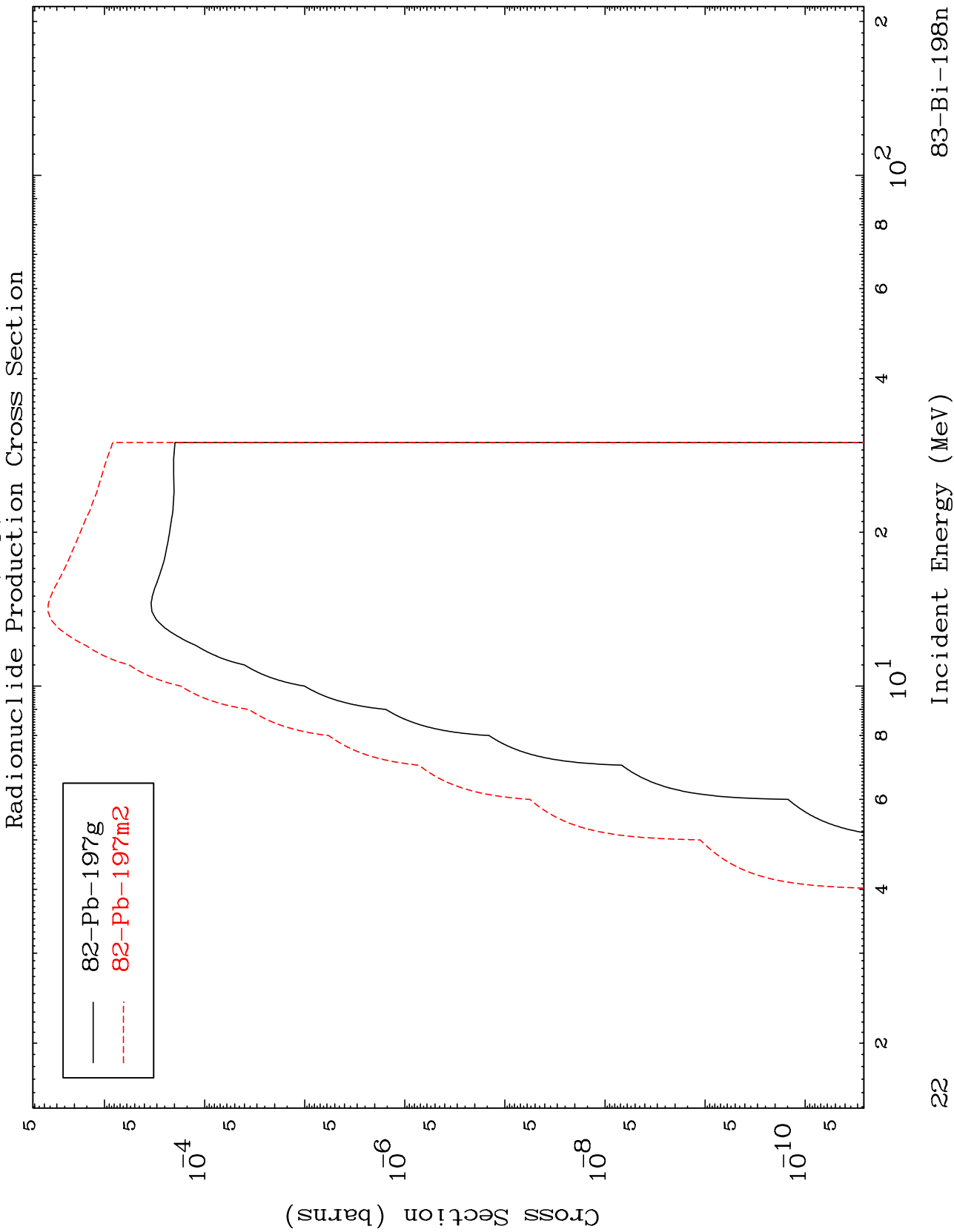
MAT 8294

$^{83}\text{Bi}-198\text{n}$



MAT 8294

83-Bi-198n

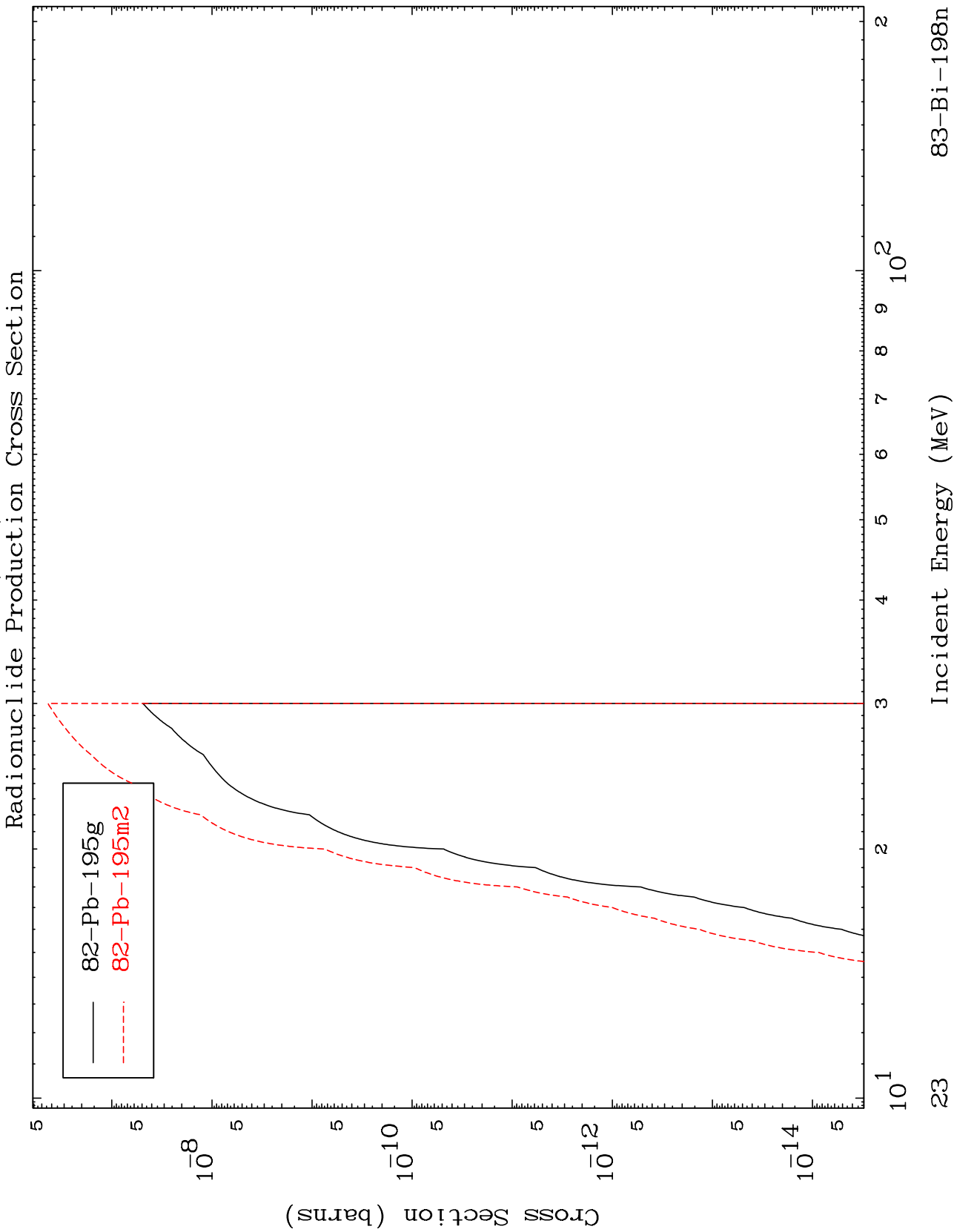


22

83-Bi-198n

MAT 8294

<sup>83</sup>Bi-198n

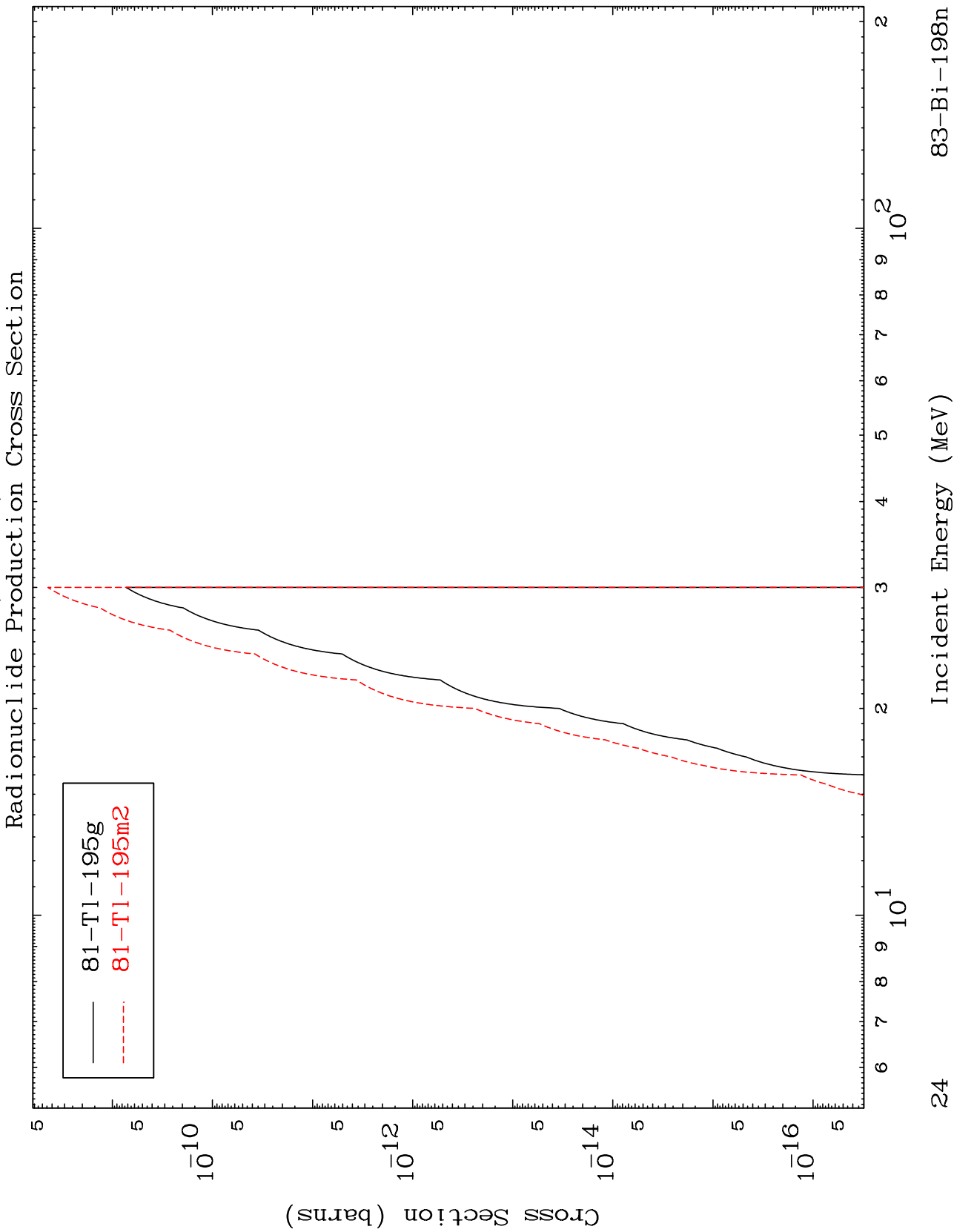


23



MAT 8294

$^{83}\text{Bi}-198\text{n}$

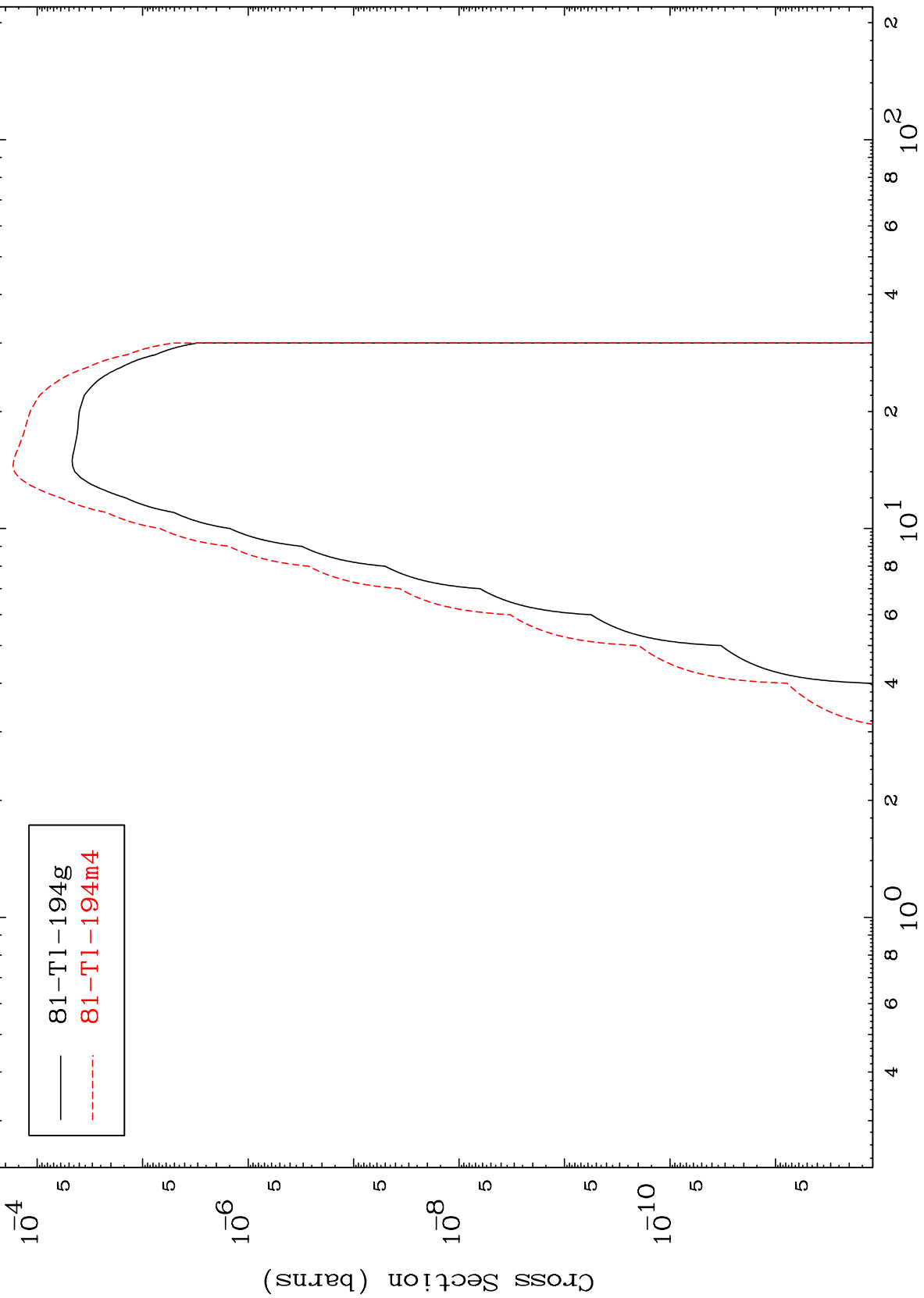


—  $^{81}\text{Tl}-195\text{g}$   
- - -  $^{81}\text{Tl}-195\text{m}2$

MAT 8294

83-Bi-198n

Radionuclide Production Cross Section



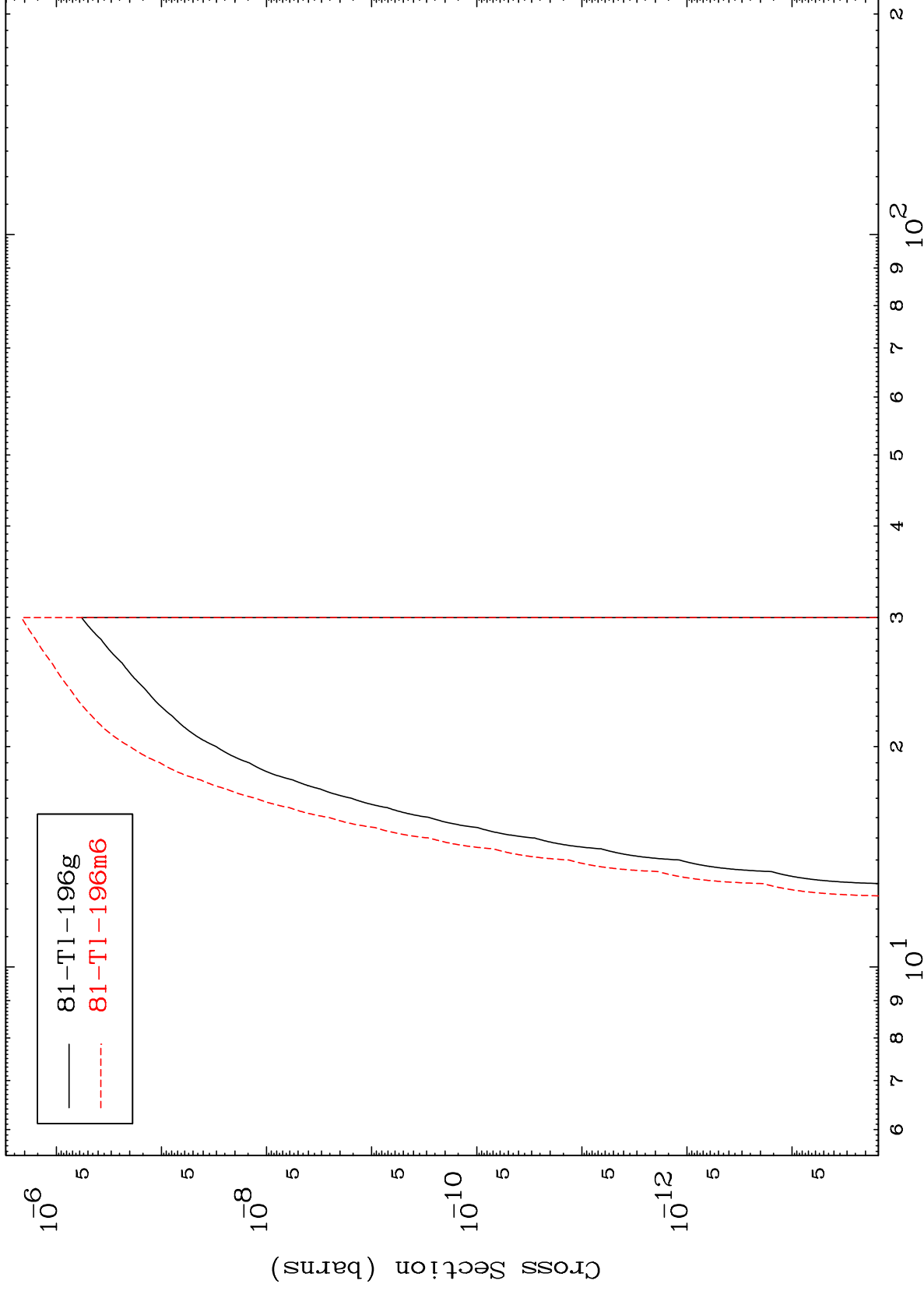
83-Bi-198n

Incident Energy (MeV)

MAT 8294

83-Bi-198n

(n,2p)  
Radionuclide Production Cross Section



26

Incident Energy (MeV)

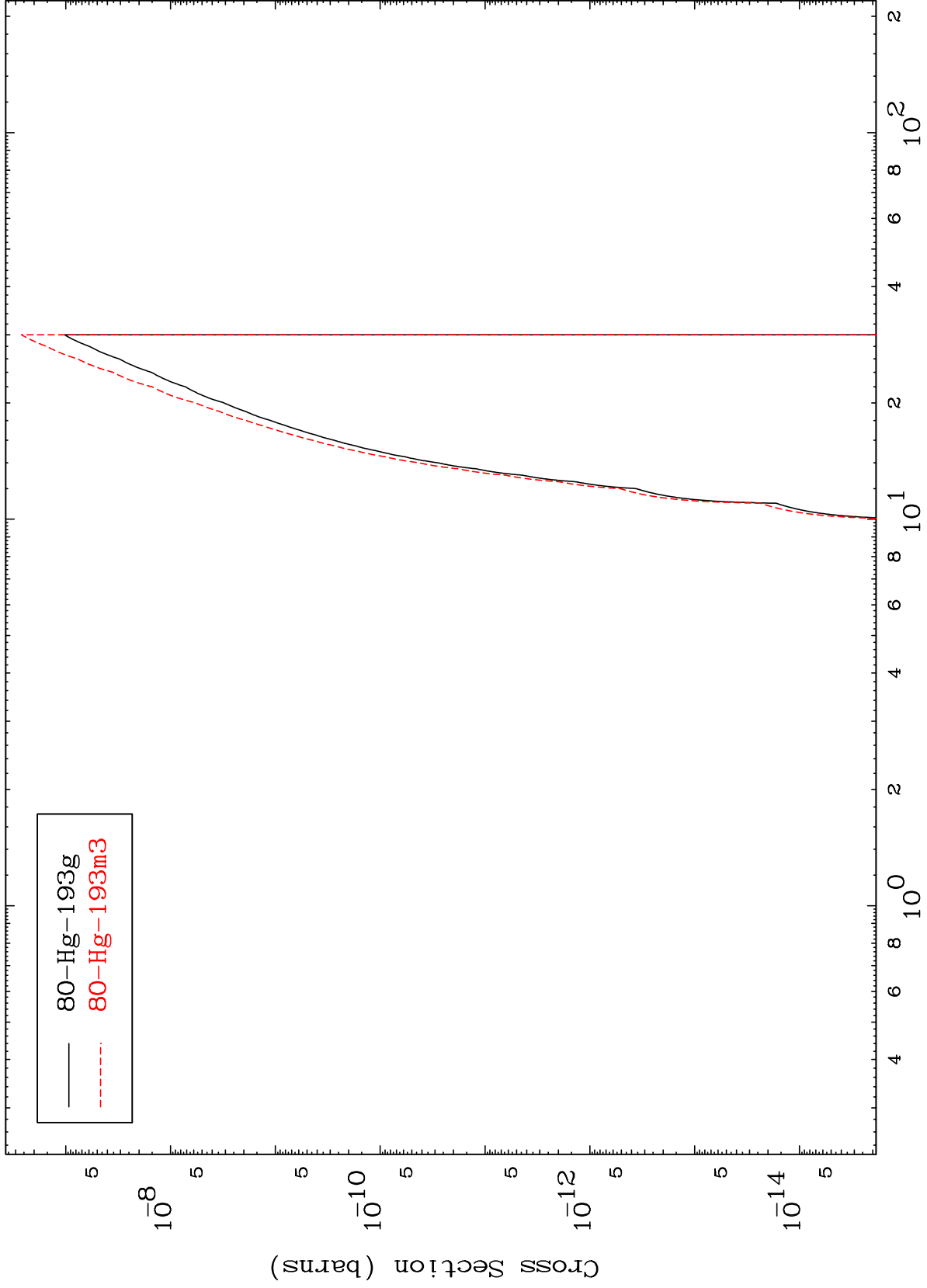
83-Bi-198n

MAT 8294

(n,p)  $\alpha$

$^{83}\text{Bi}-198\text{n}$

Radionuclide Production Cross Section



MAT 8294

(n,p) d

83-Bi-198n

Radionuclide Production Cross Section

