

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

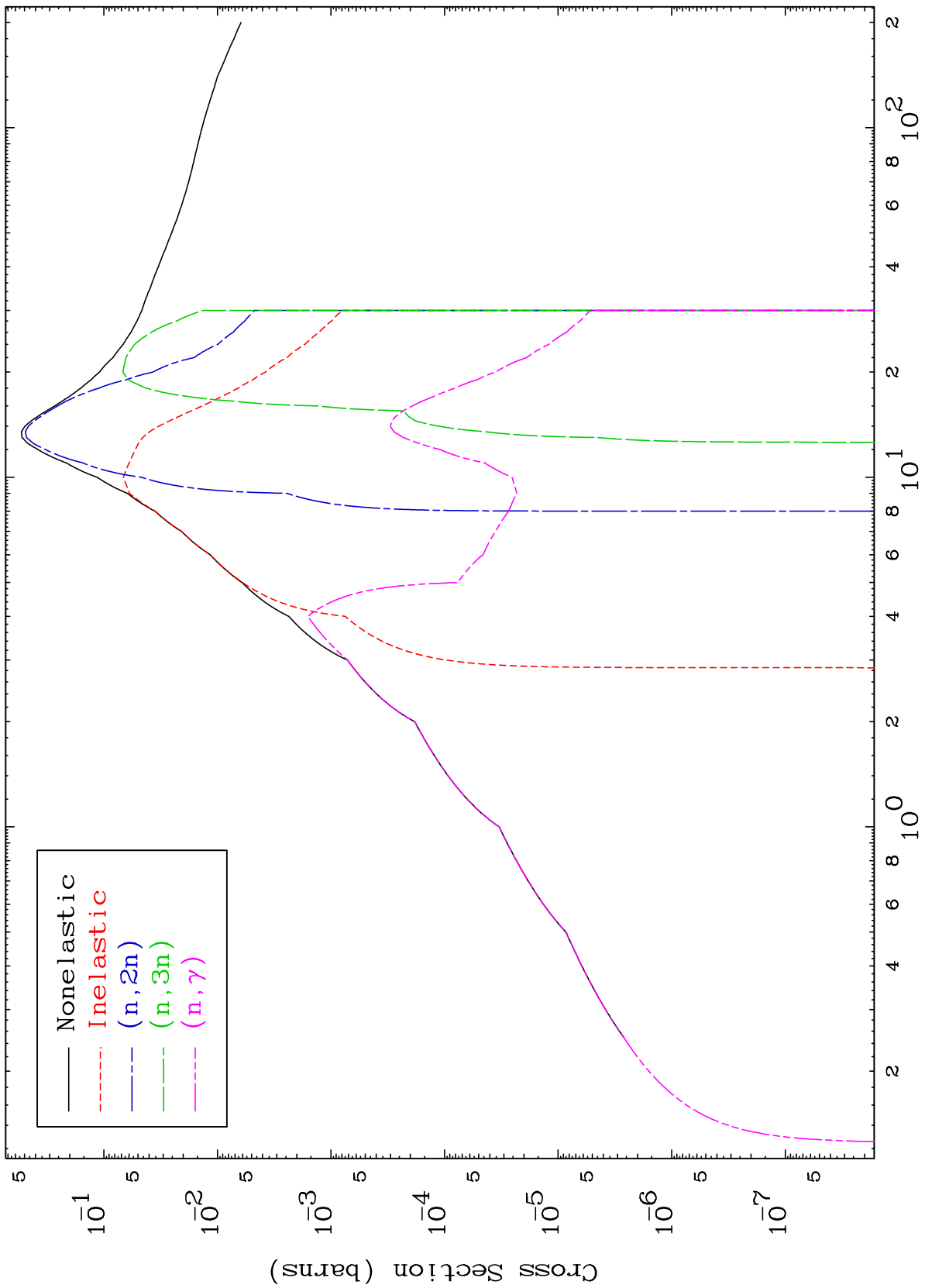
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8336

Photon Major
0 Kelvin Cross Sections

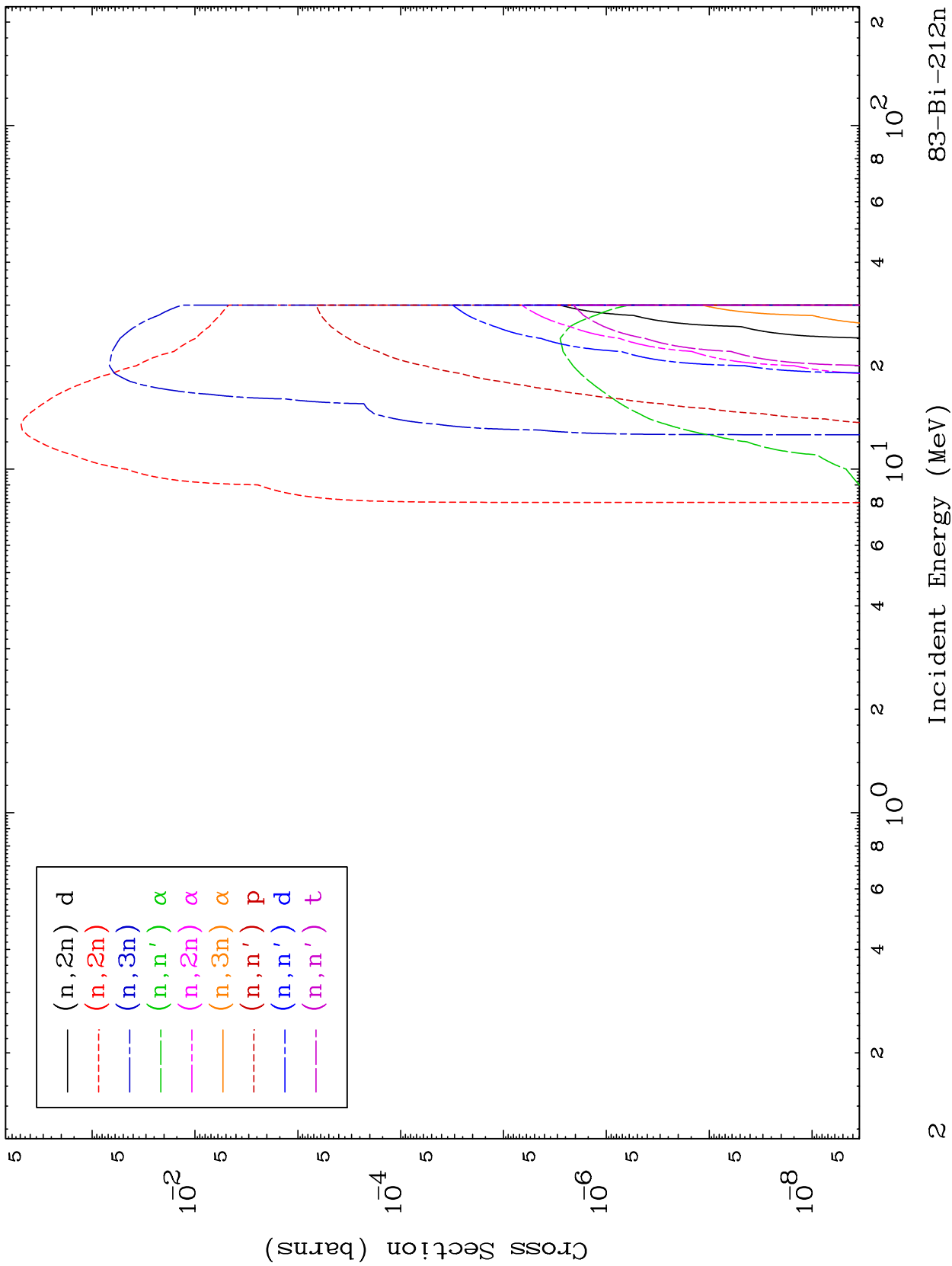
83-Bi-212n



MAT 8336

Photon Neutron Absorption
0 Kelvin Cross Sections

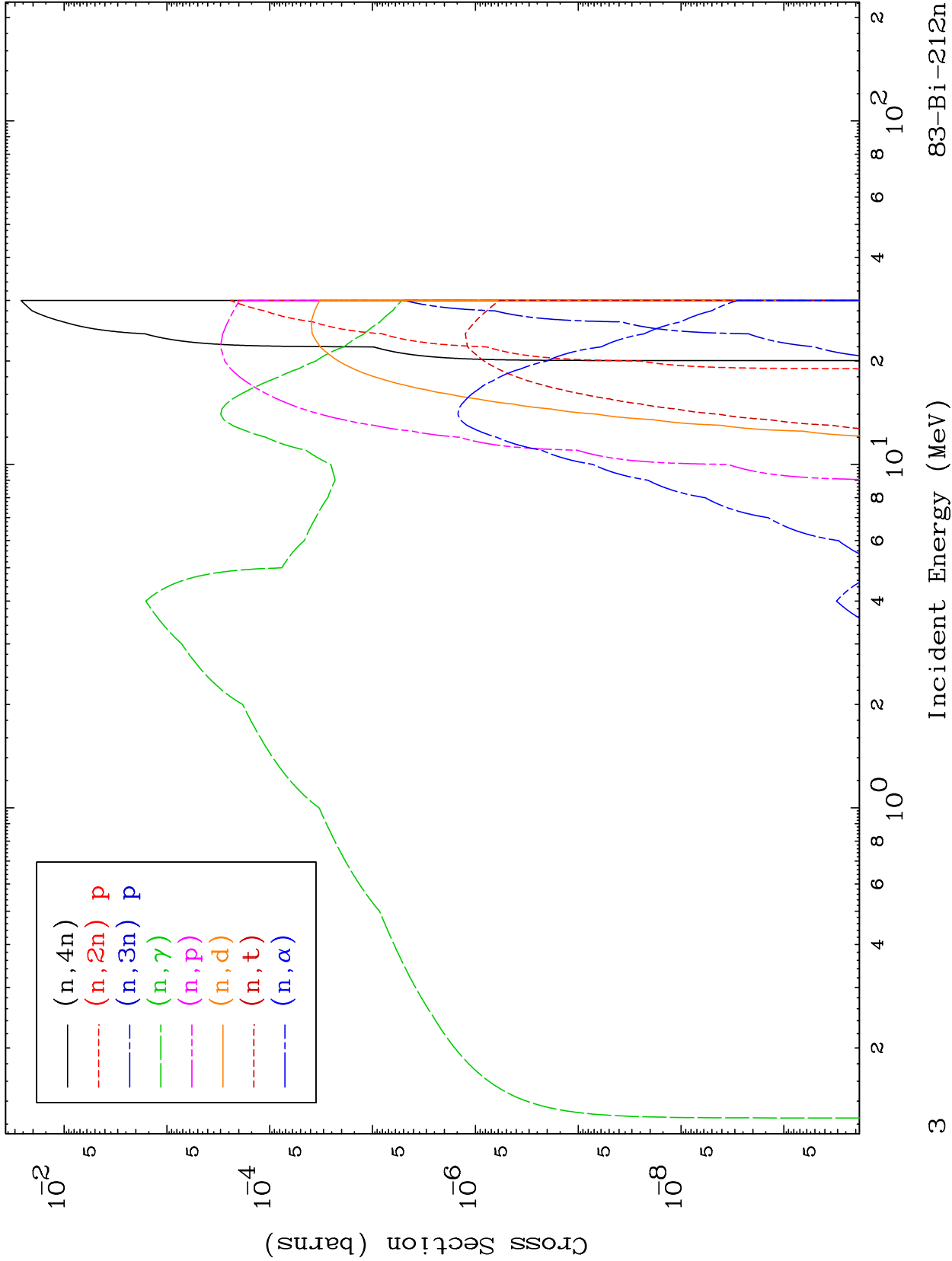
83-Bi-212n



MAT 8336

Photon Neutron Absorption
0 Kelvin Cross Sections

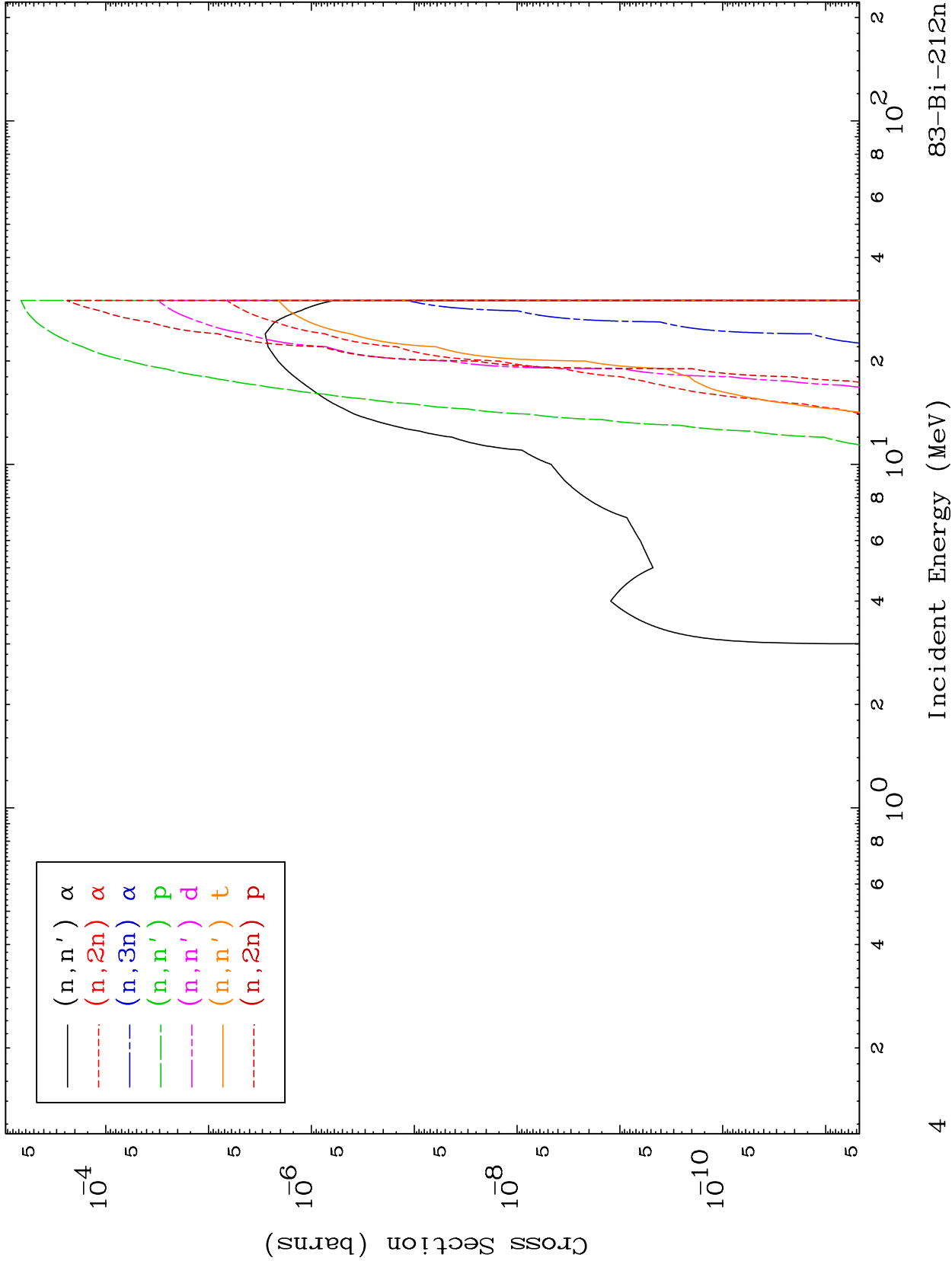
83-Bi-212n

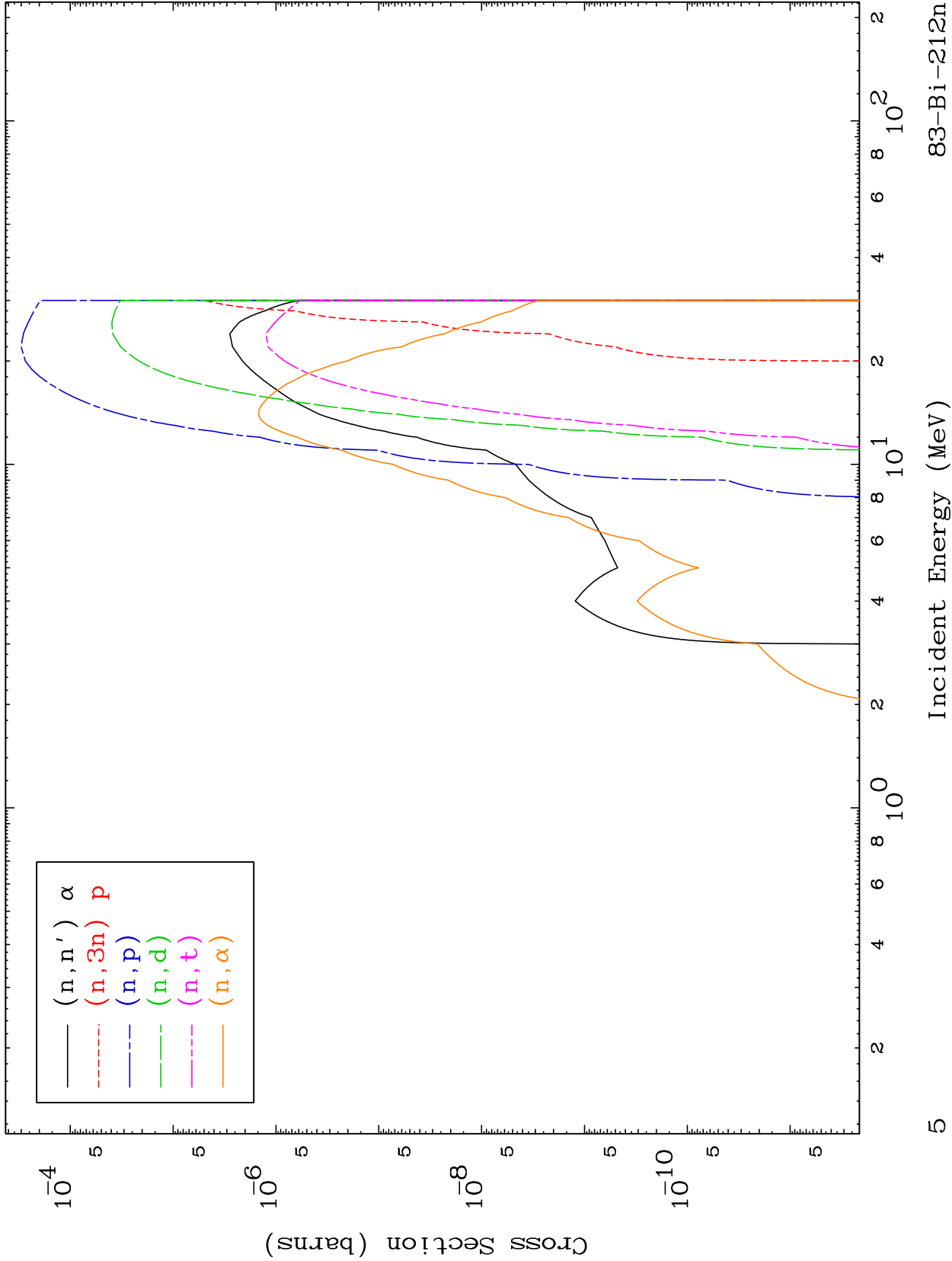


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Photon Charged Particle
0 Kelvin Cross Sections

83-Bi-212n

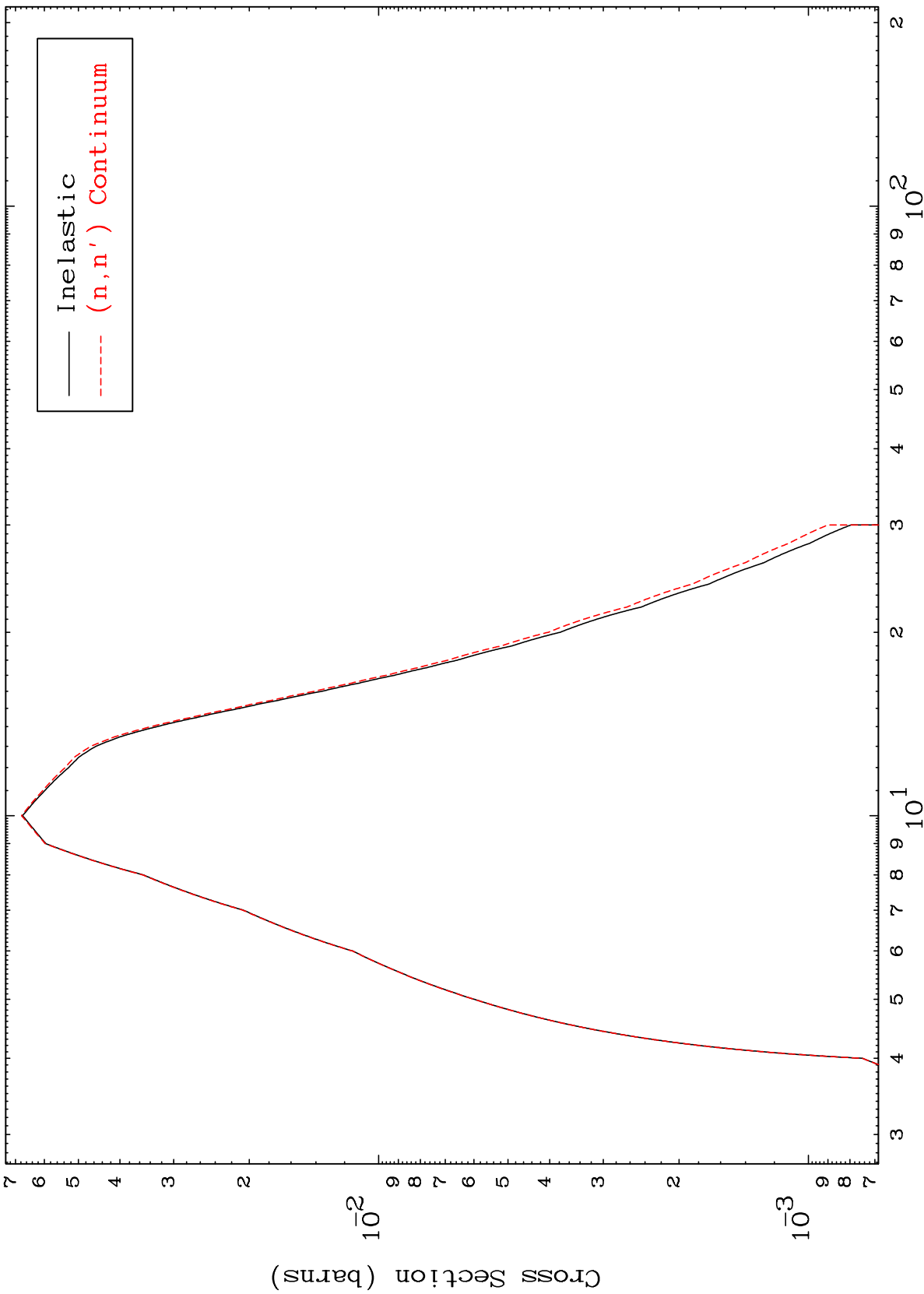




MAT 8336

83-Bi-212n

(γ, n') Levels
0 Kelvin Cross Sections



83-Bi-212n

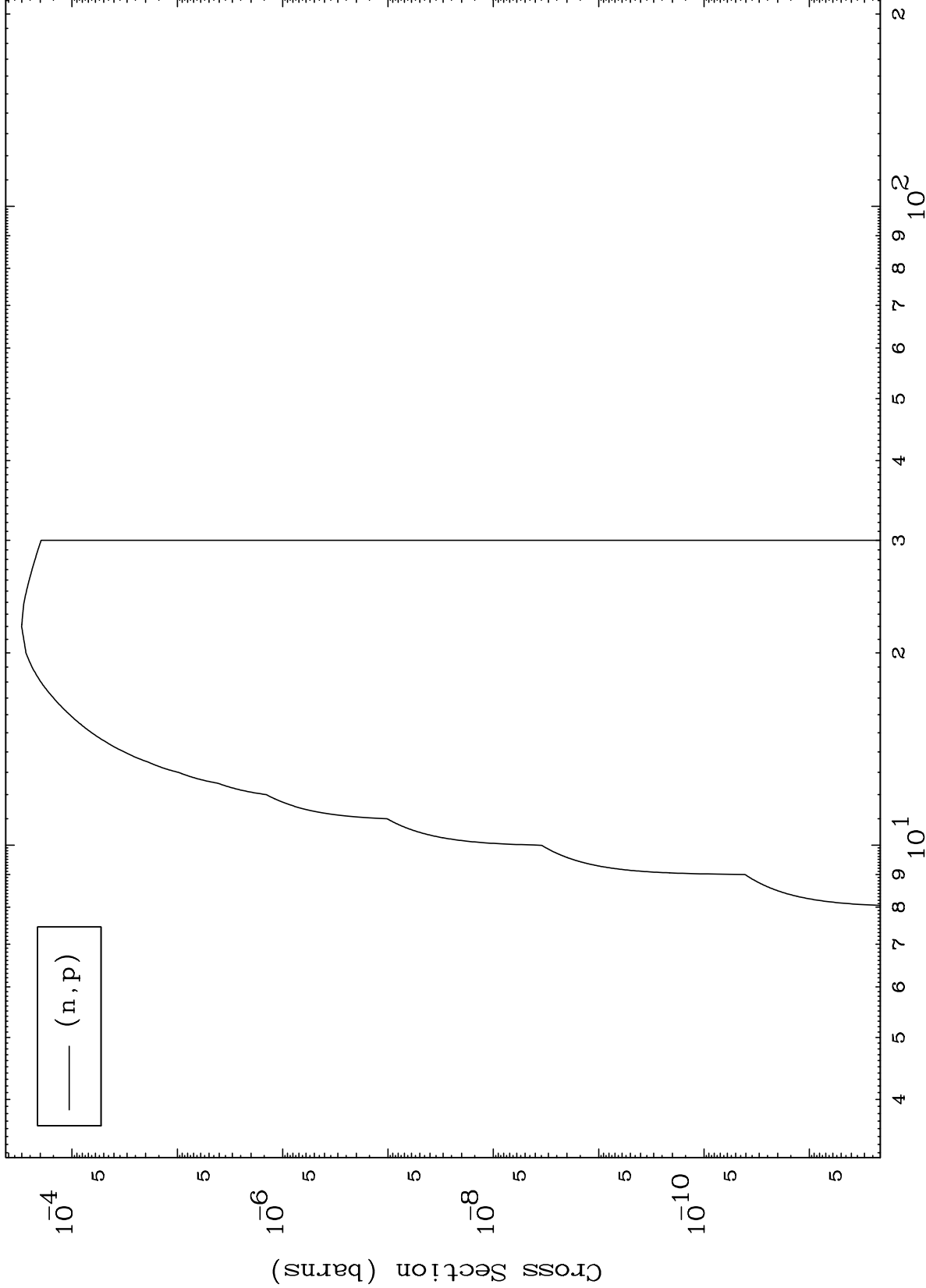
Incident Energy (MeV)

6

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(γ, p) Levels
0 Kelvin Cross Sections

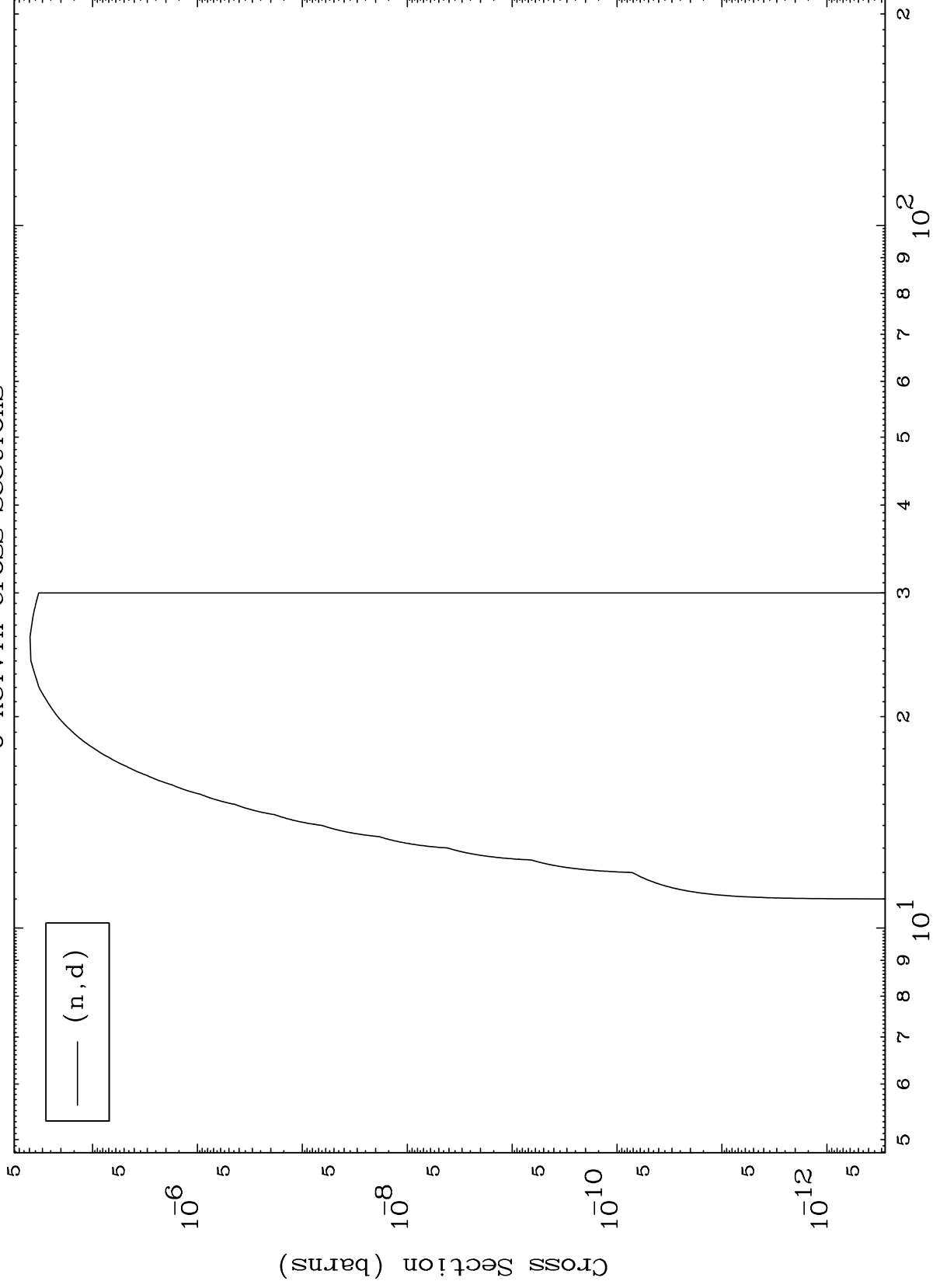
83-Bi-212n



MAT 8336

(γ, d) Levels
0 Kelvin Cross Sections

$^{83}\text{Bi}-212n$



8

Incident Energy (MeV)

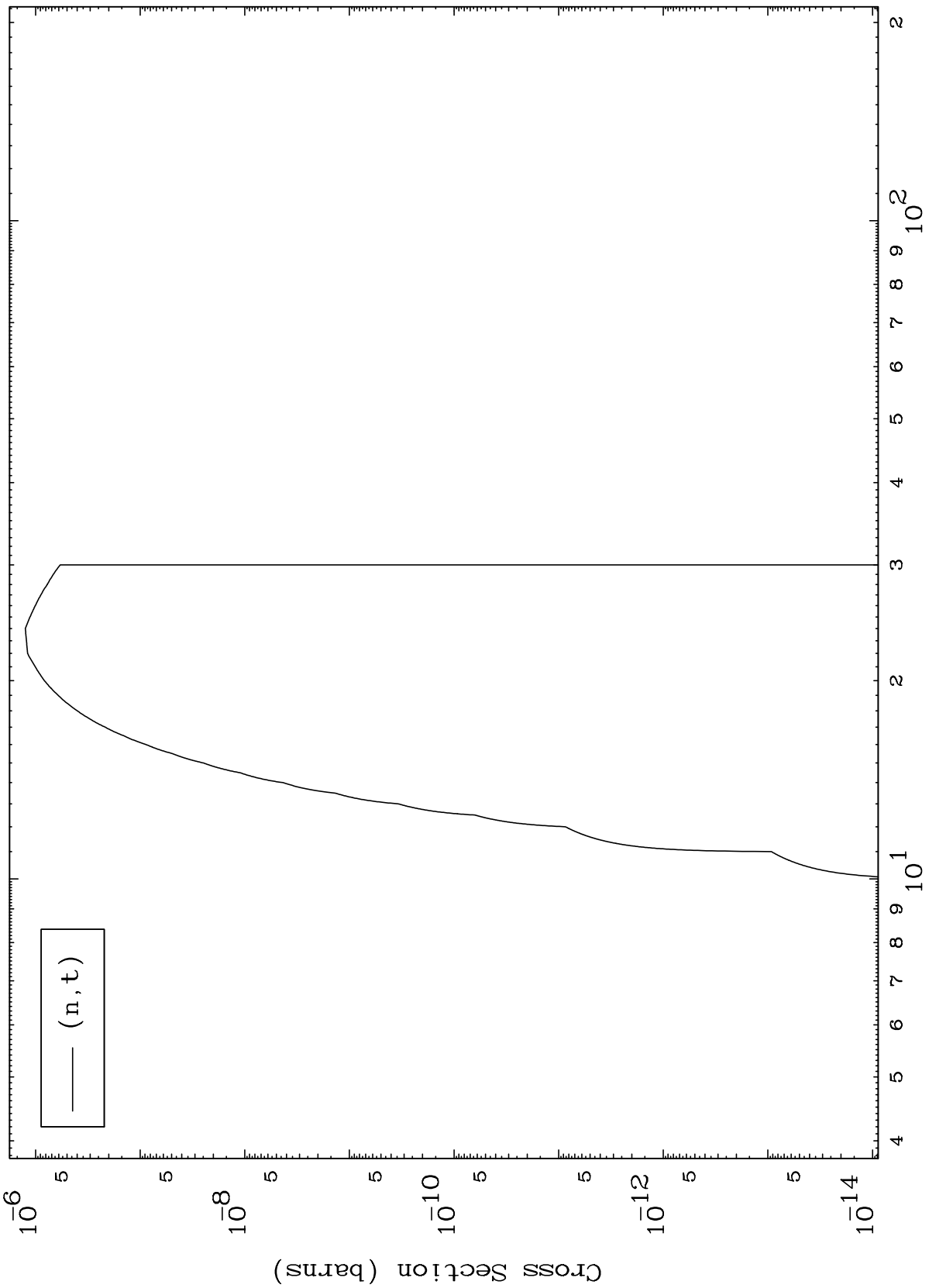
$^{83}\text{Bi}-212n$

MAT 8336

(γ, t) Levels

$^{83}\text{Bi}-^{212}\text{n}$

0 Kelvin Cross Sections

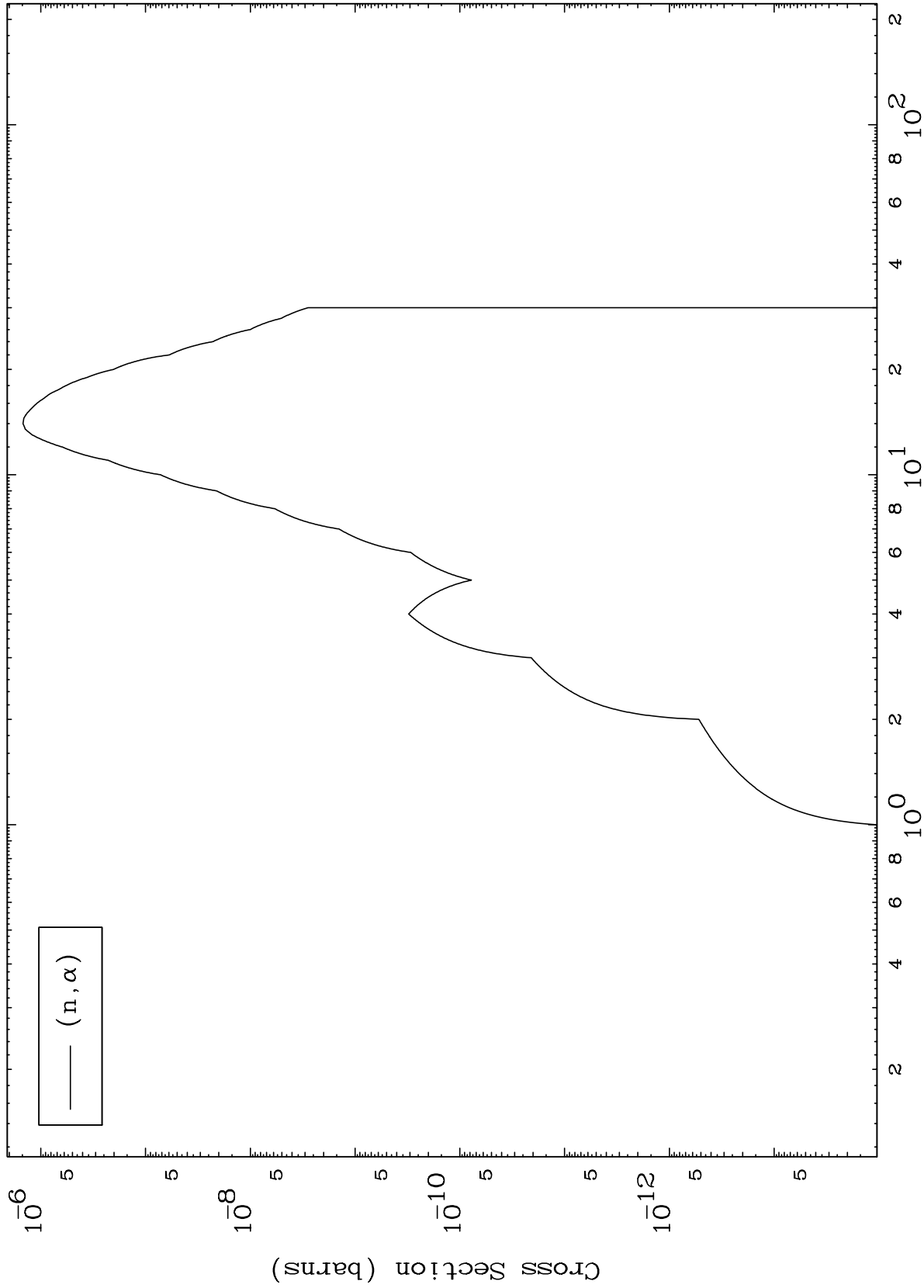


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(γ, α) Levels

83-Bi-212n

0 Kelvin Cross Sections



10

Incident Energy (MeV)

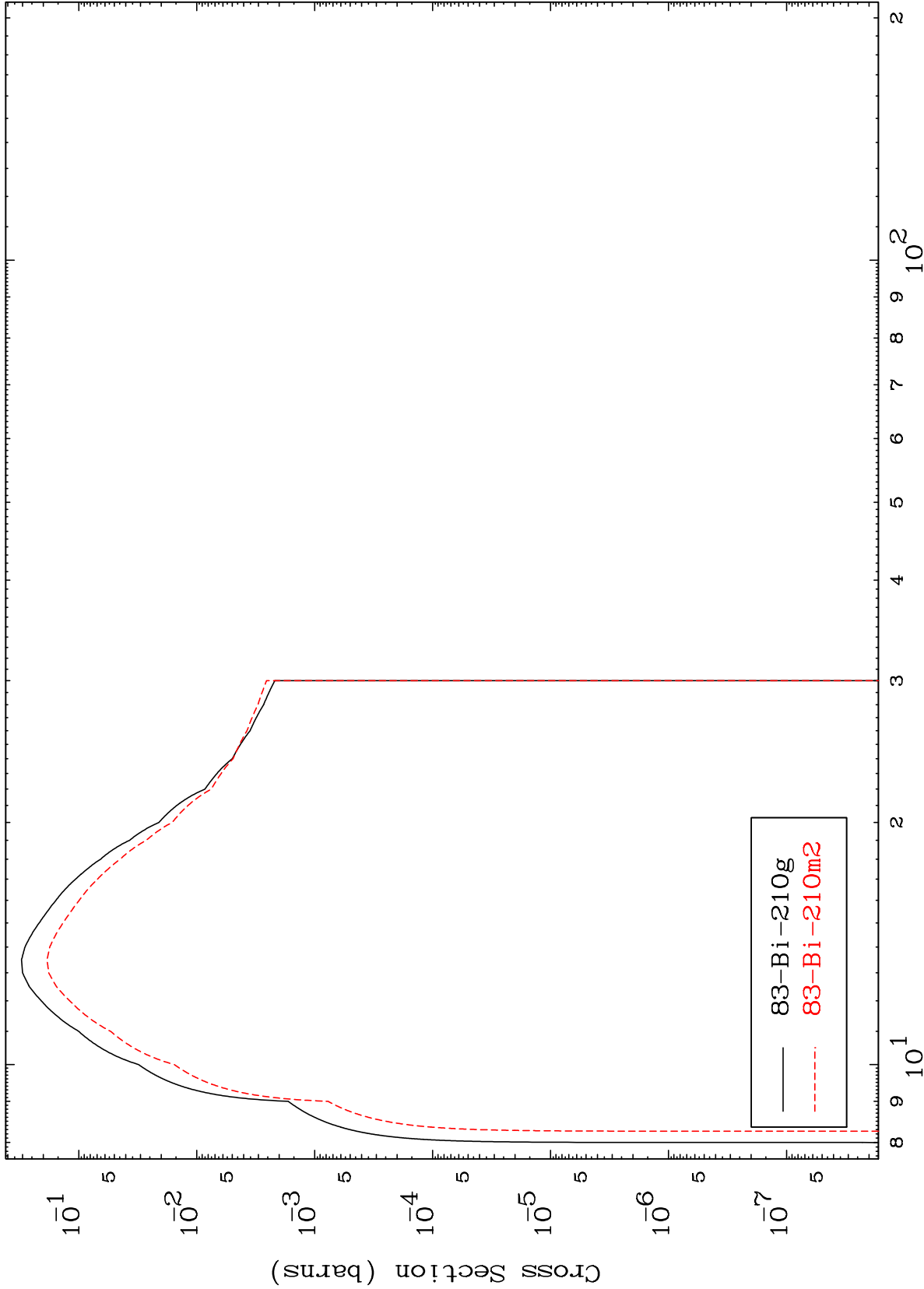
83-Bi-212n

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(n,2n)

83-Bi-212n

Radionuclide Production Cross Section



Incident Energy (MeV)

83-Bi-212n

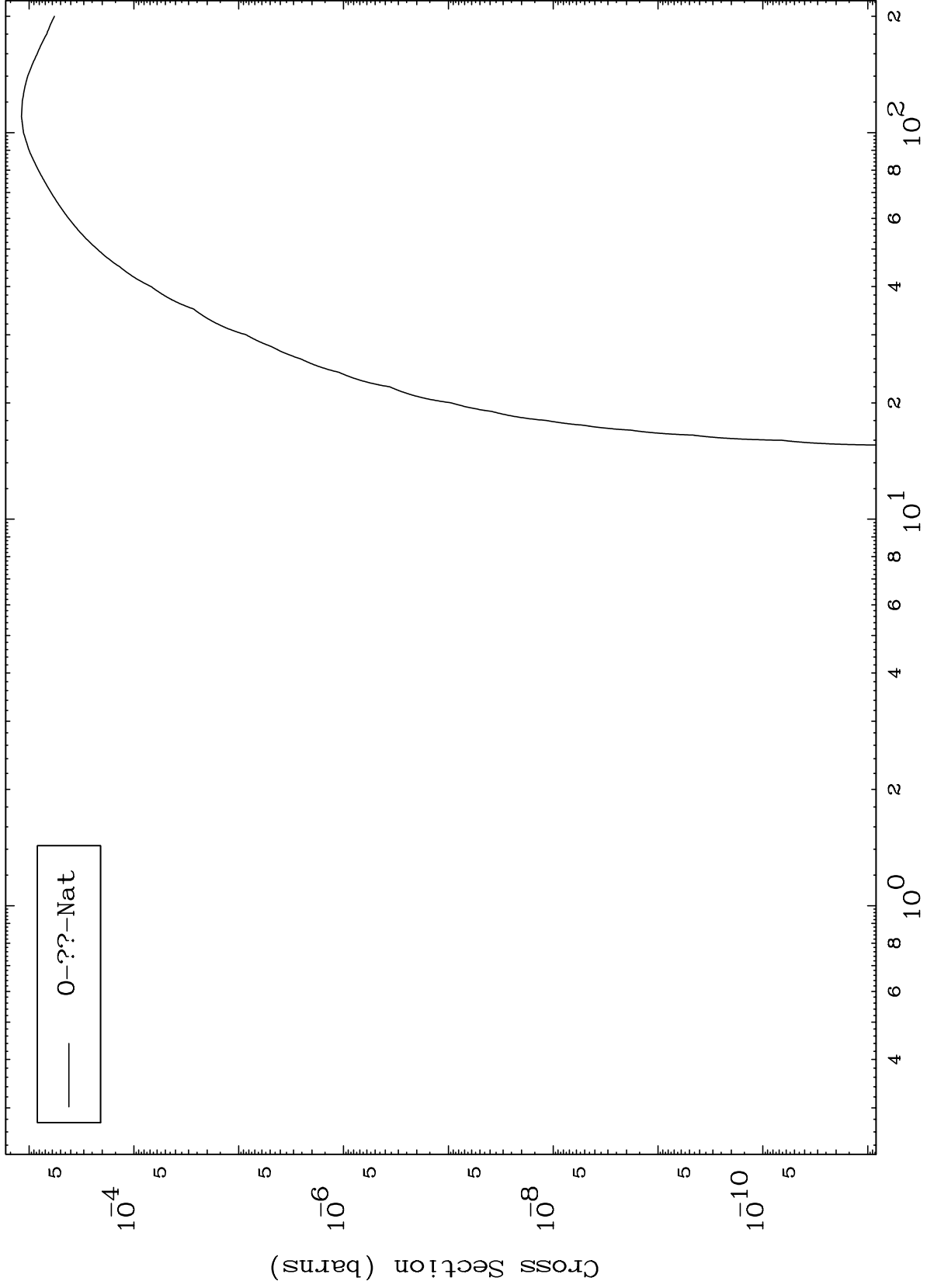
11

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Fission

⁸³Bi-212n

Radionuclide Production Cross Section

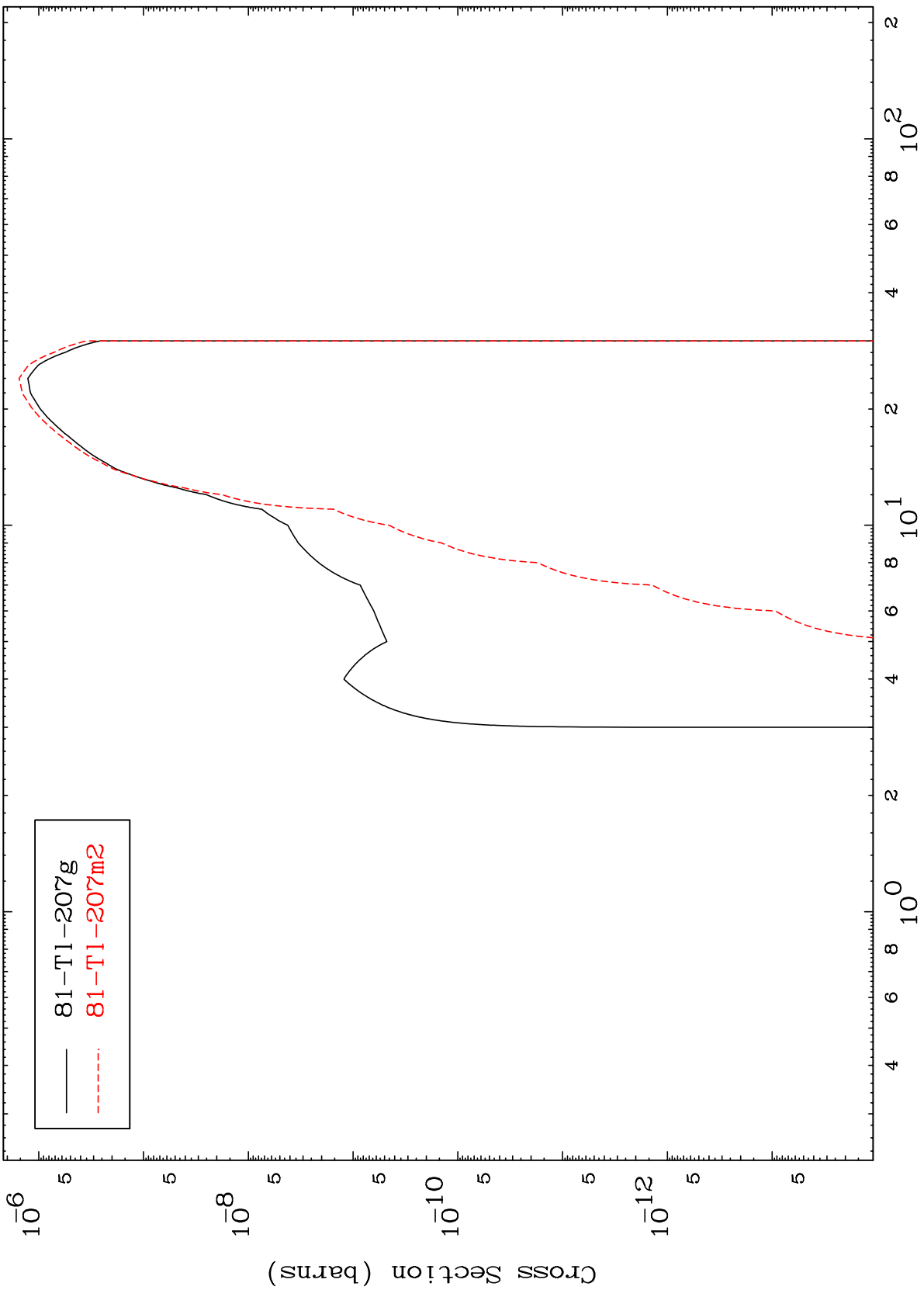


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$(n, n') \alpha$

$^{83}\text{Bi}-^{212}\text{n}$

Radionuclide Production Cross Section

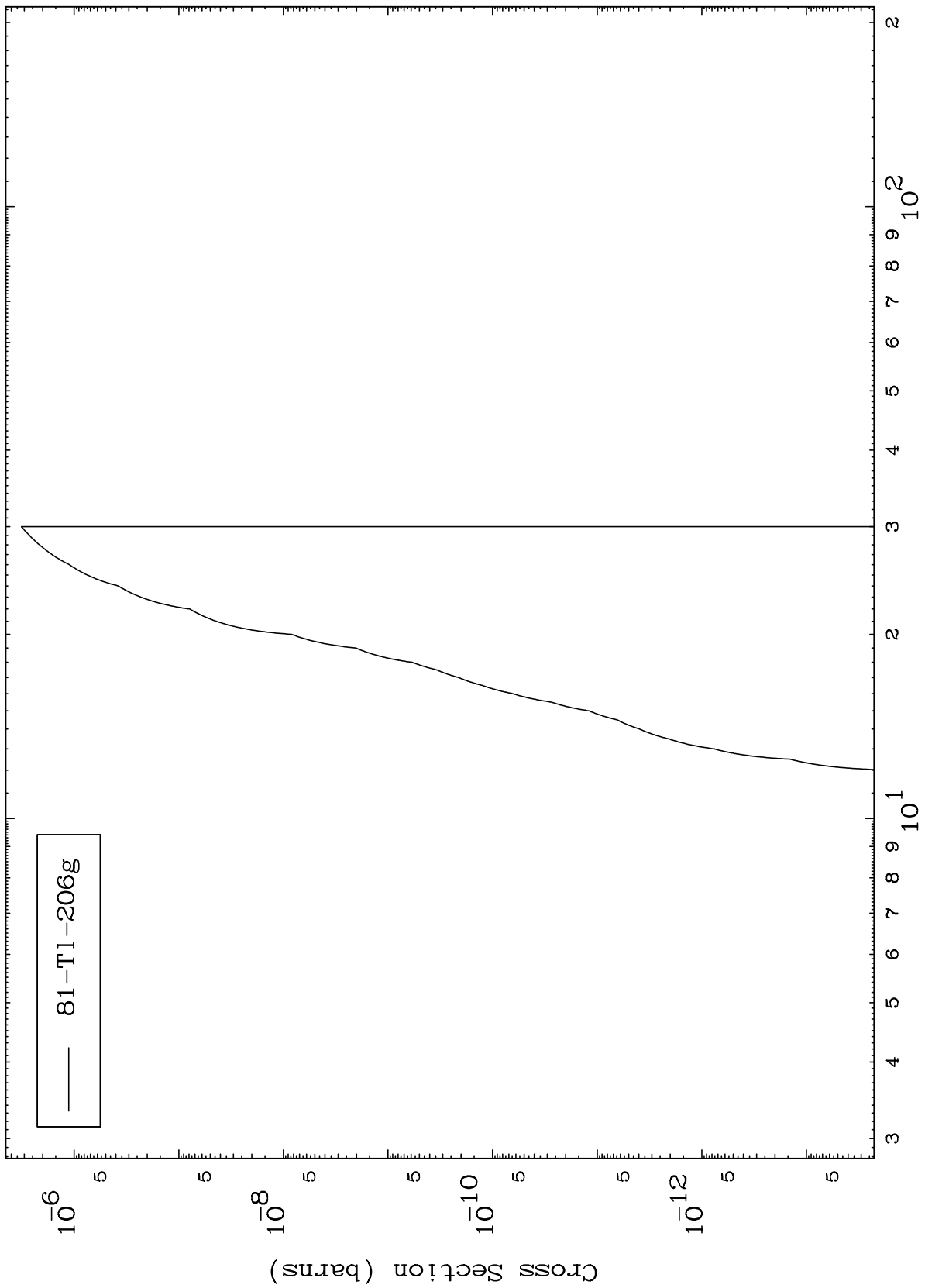


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(n,2n) α

83-Bi-212n

Radionuclide Production Cross Section



14

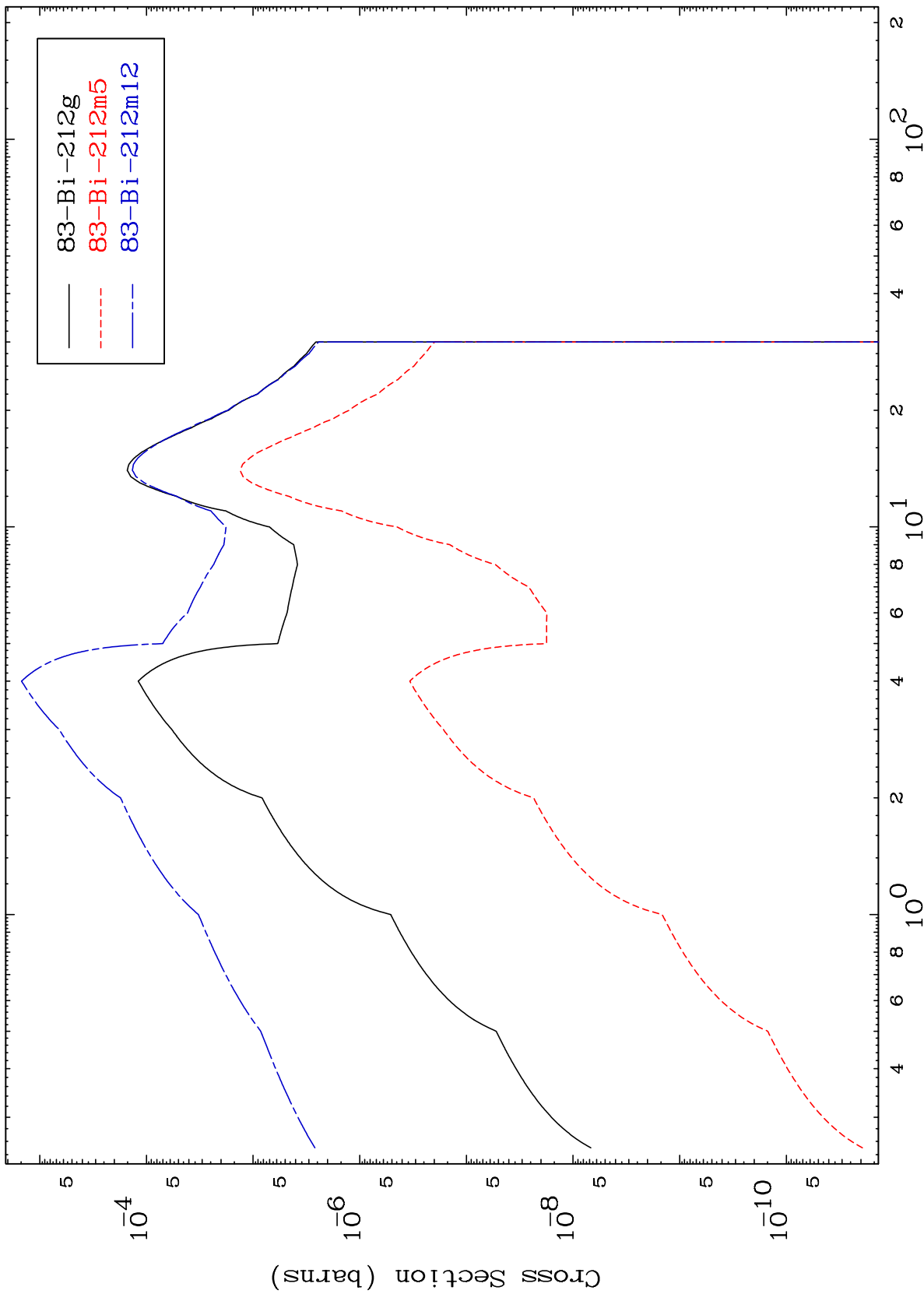
Incident Energy (MeV)

83-Bi-212n

MAT 8336

$^{83}\text{Bi}-^{212}\text{n}$

(n, γ)
Radionuclide Production Cross Section



15

Incident Energy (MeV)

$^{83}\text{Bi}-^{212}\text{n}$