

Program EVALPLOT  
(Version 2021-1)

by

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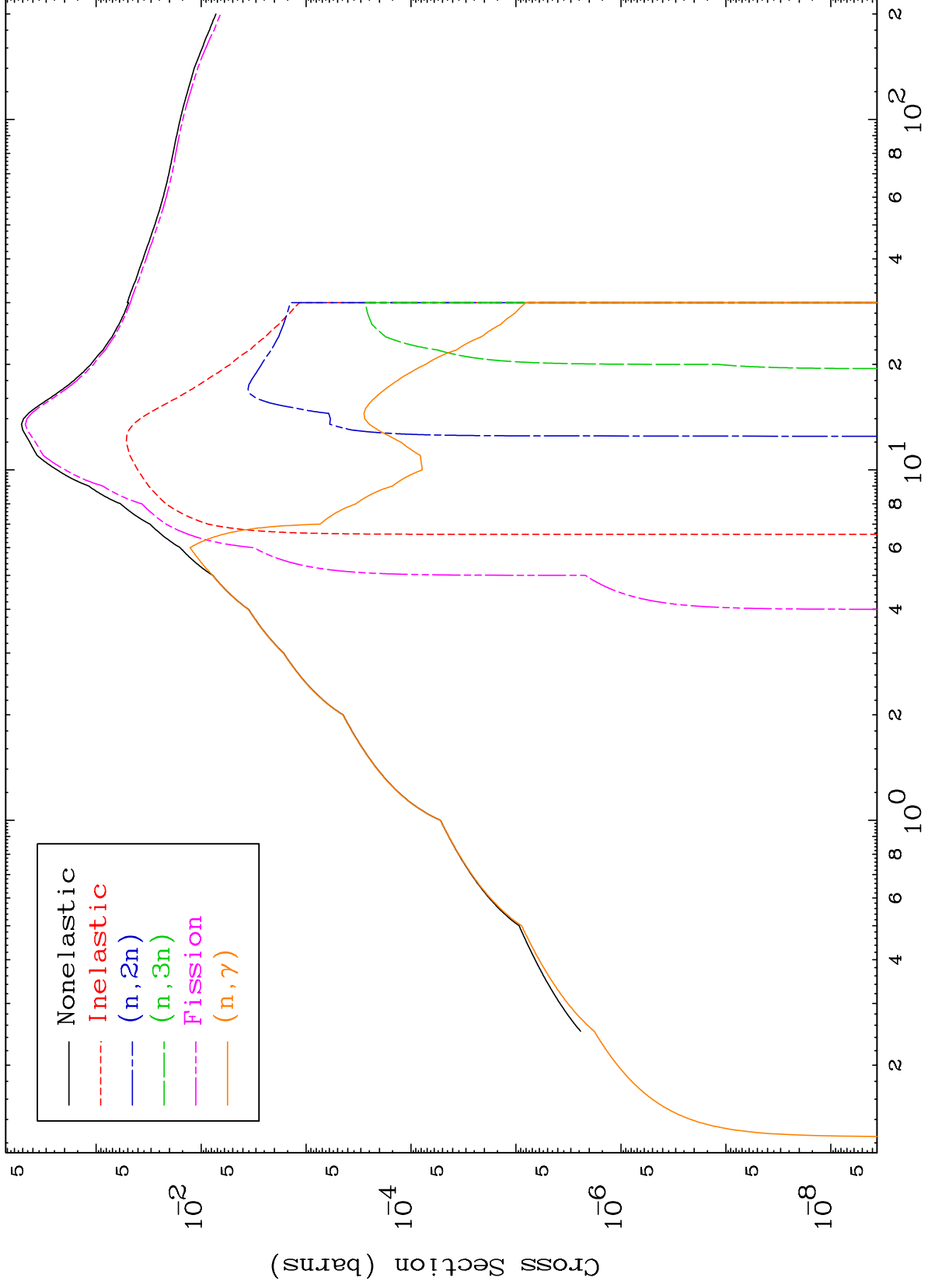
Press Mouse Button to Start

MAT 9746

Photon Major

97-Bk-247

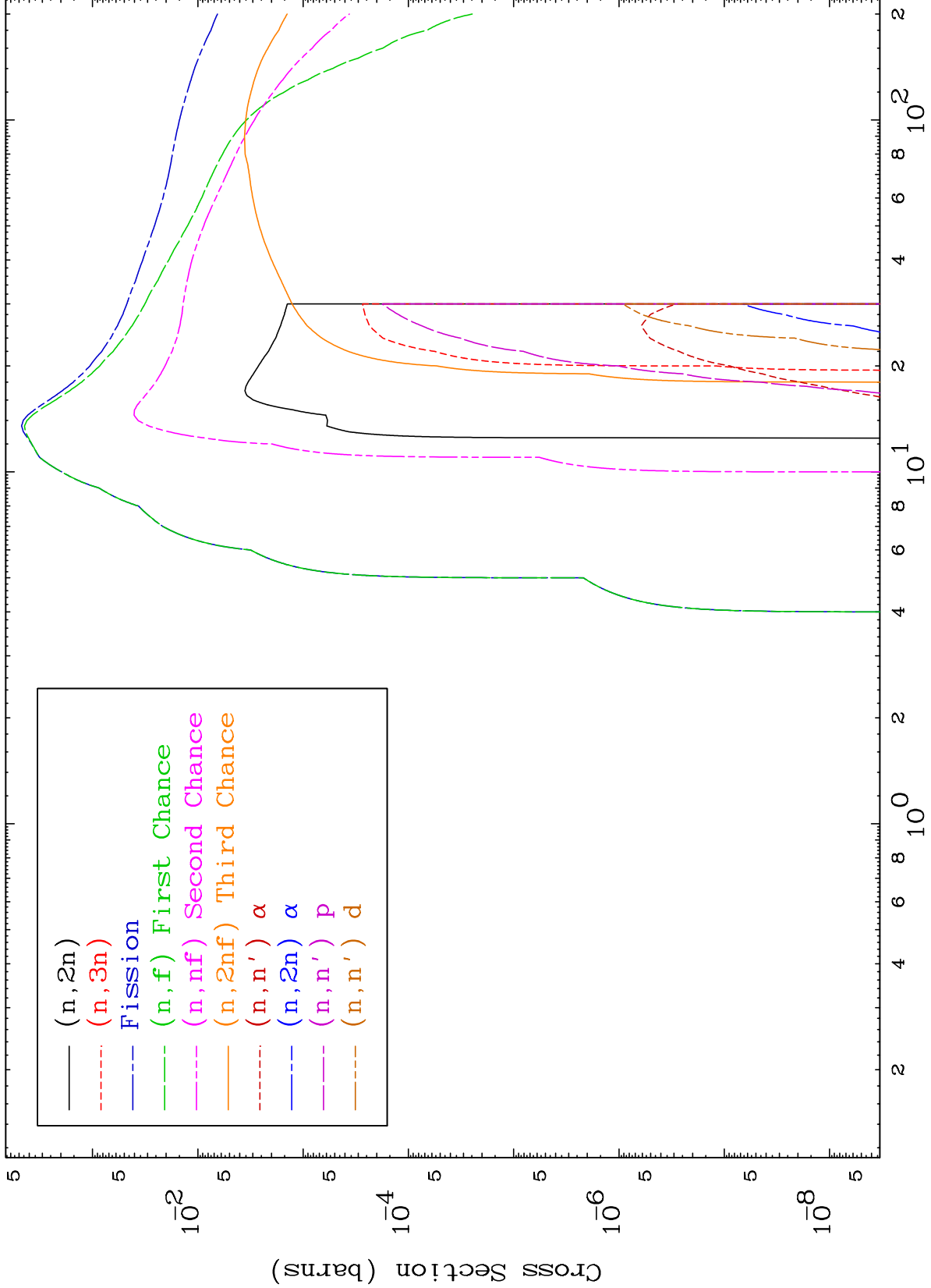
0 Kelvin Cross Sections



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Photon Neutron Absorption  
0 Kelvin Cross Sections

97-Bk-247



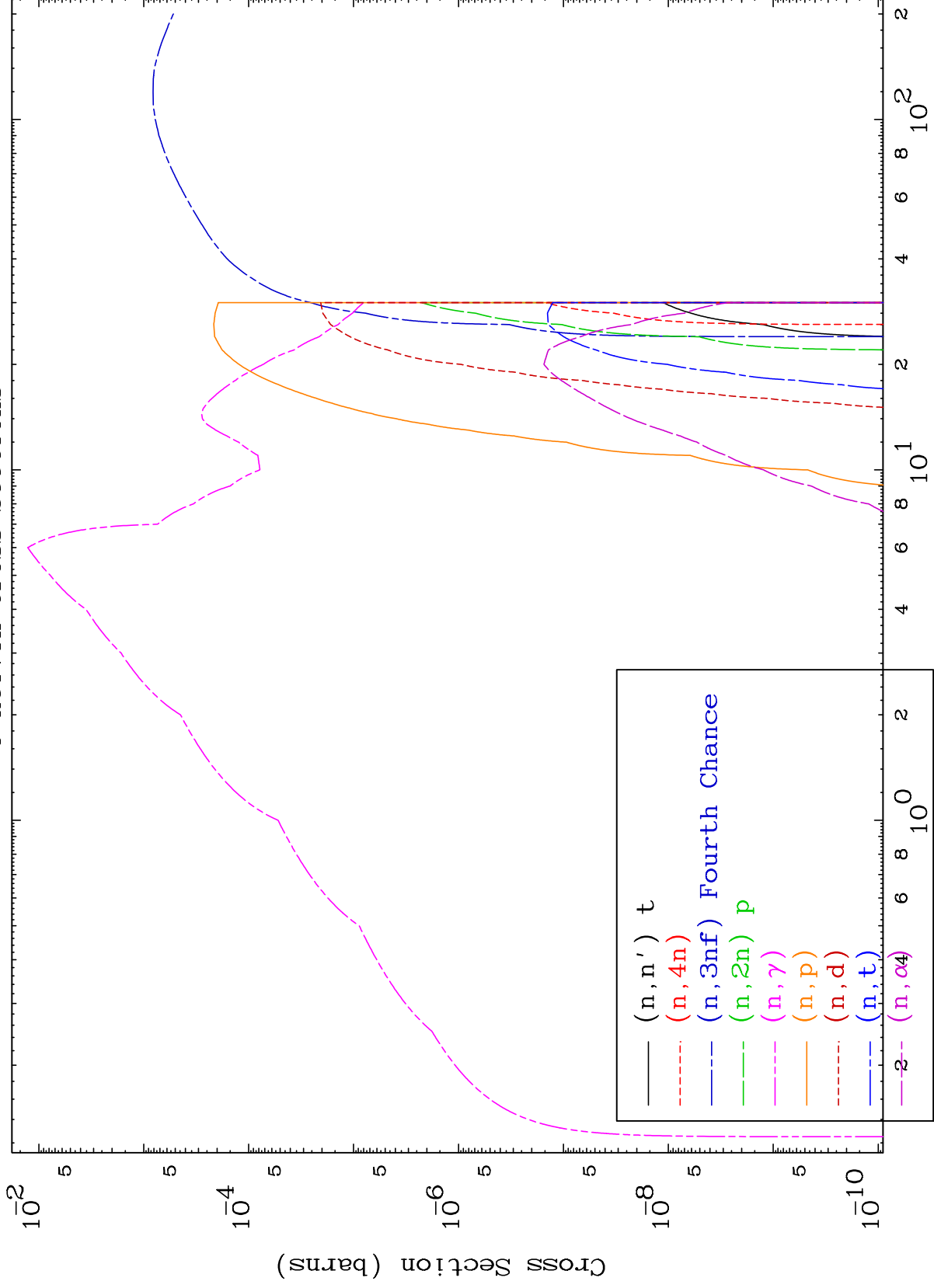
97-Bk-247

Incident Energy (MeV)

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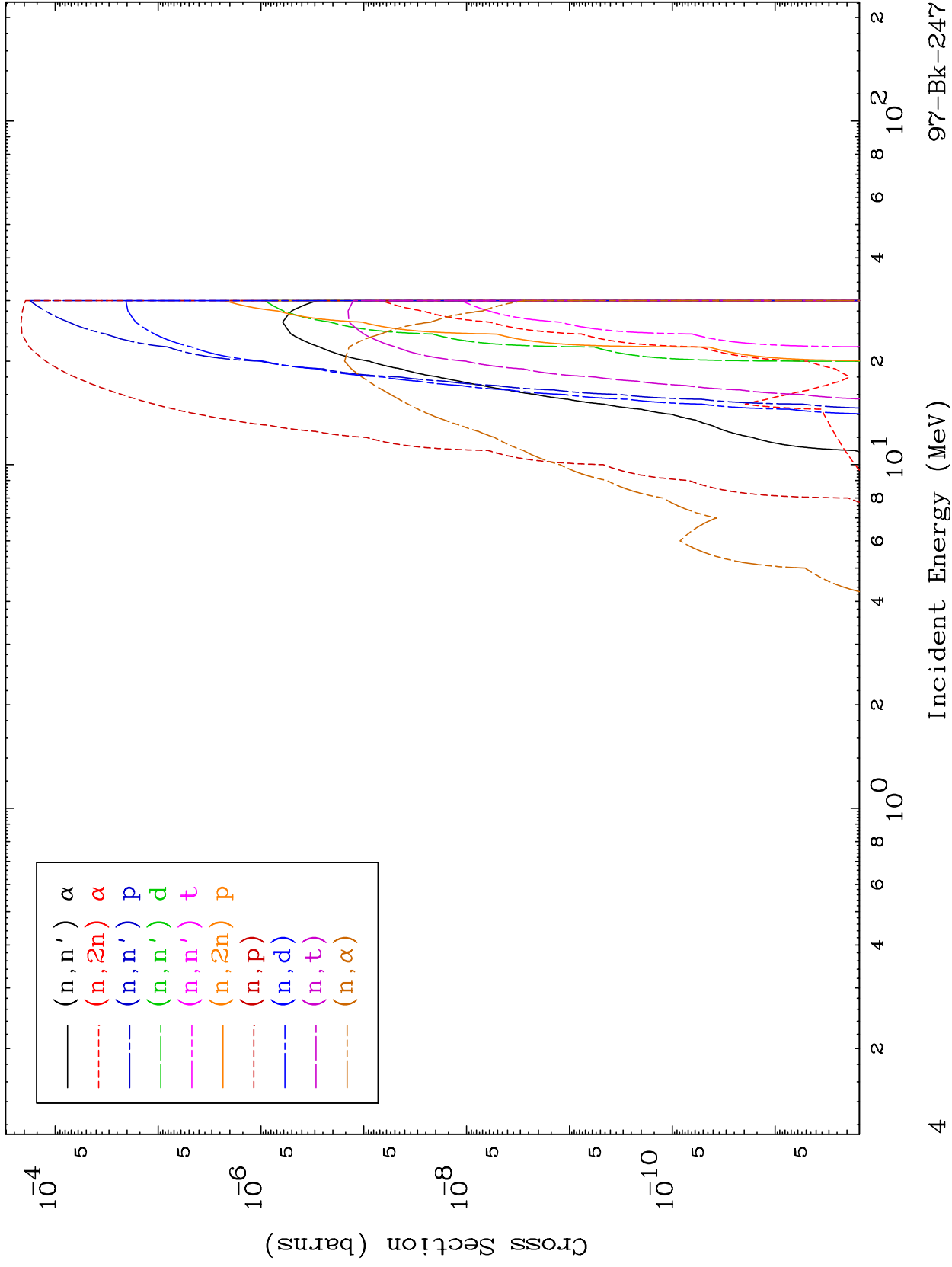
Photon Neutron Absorption  
0 Kelvin Cross Sections

97-Bk-247

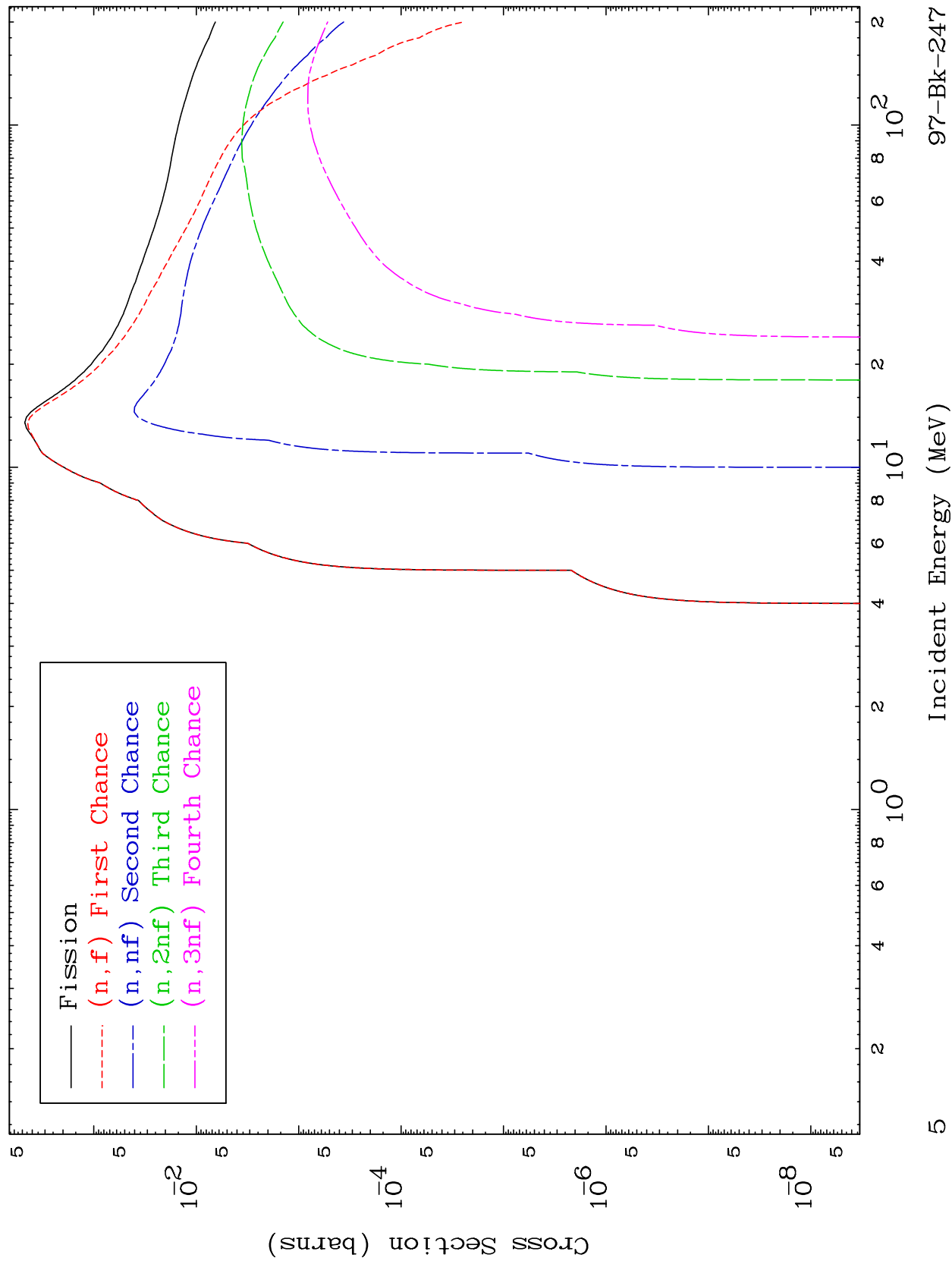


97-Bk-247

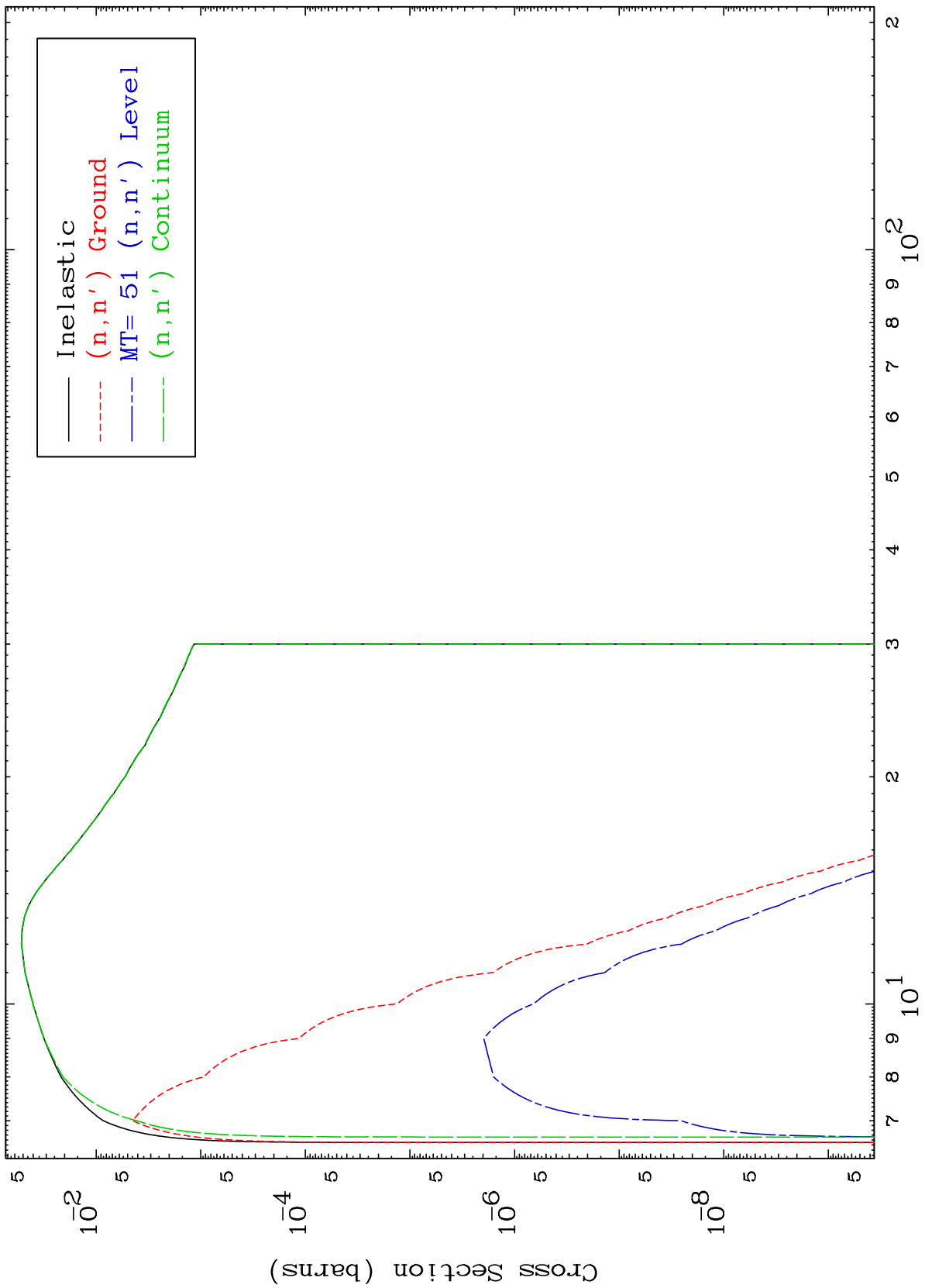
Incident Energy (MeV)



Photon Fission  
0 Kelvin Cross Sections



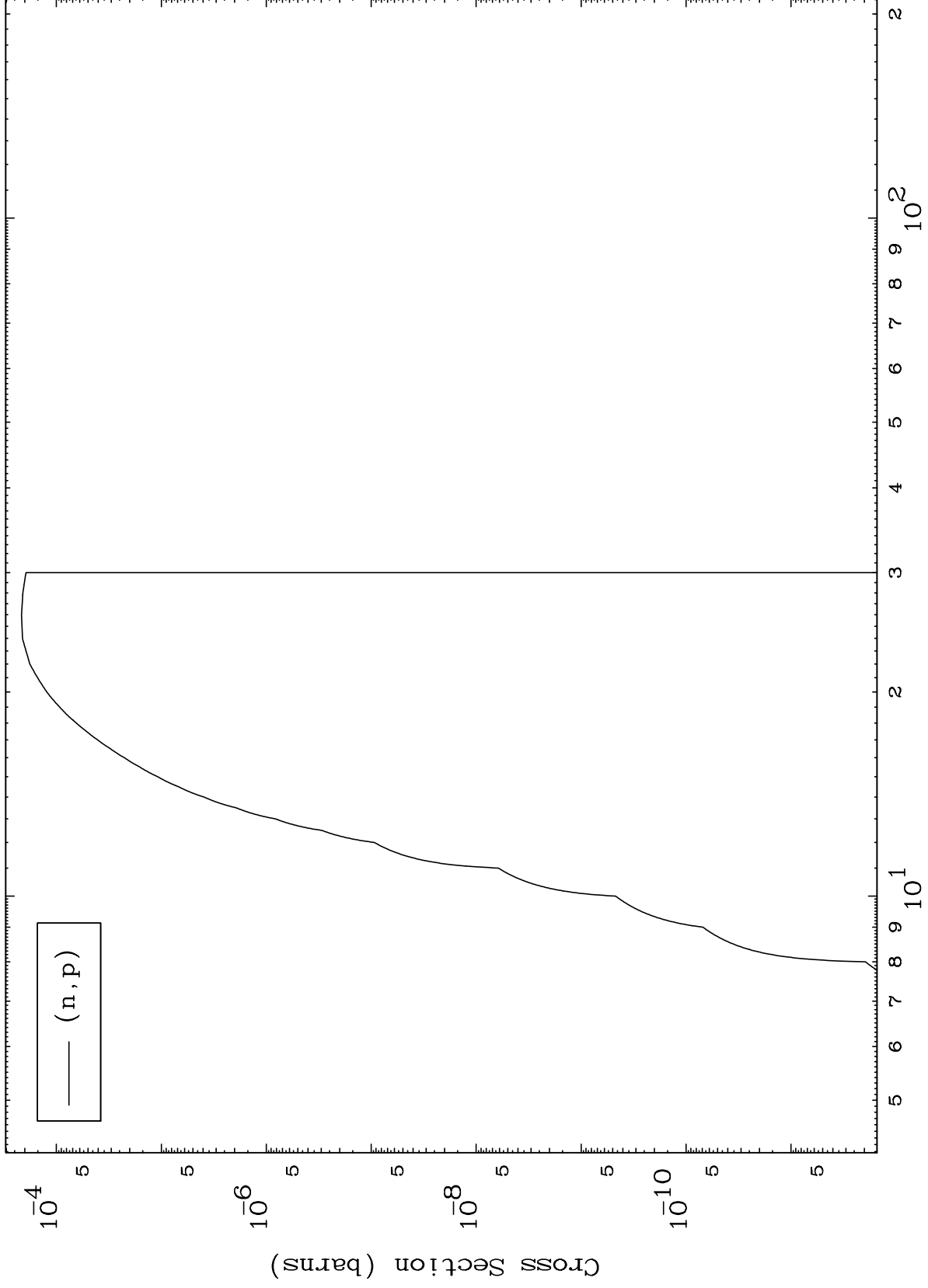
( $\gamma, n'$ ) Levels  
0 Kelvin Cross Sections



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( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

97-Bk-247



Incident Energy (MeV)

97-Bk-247

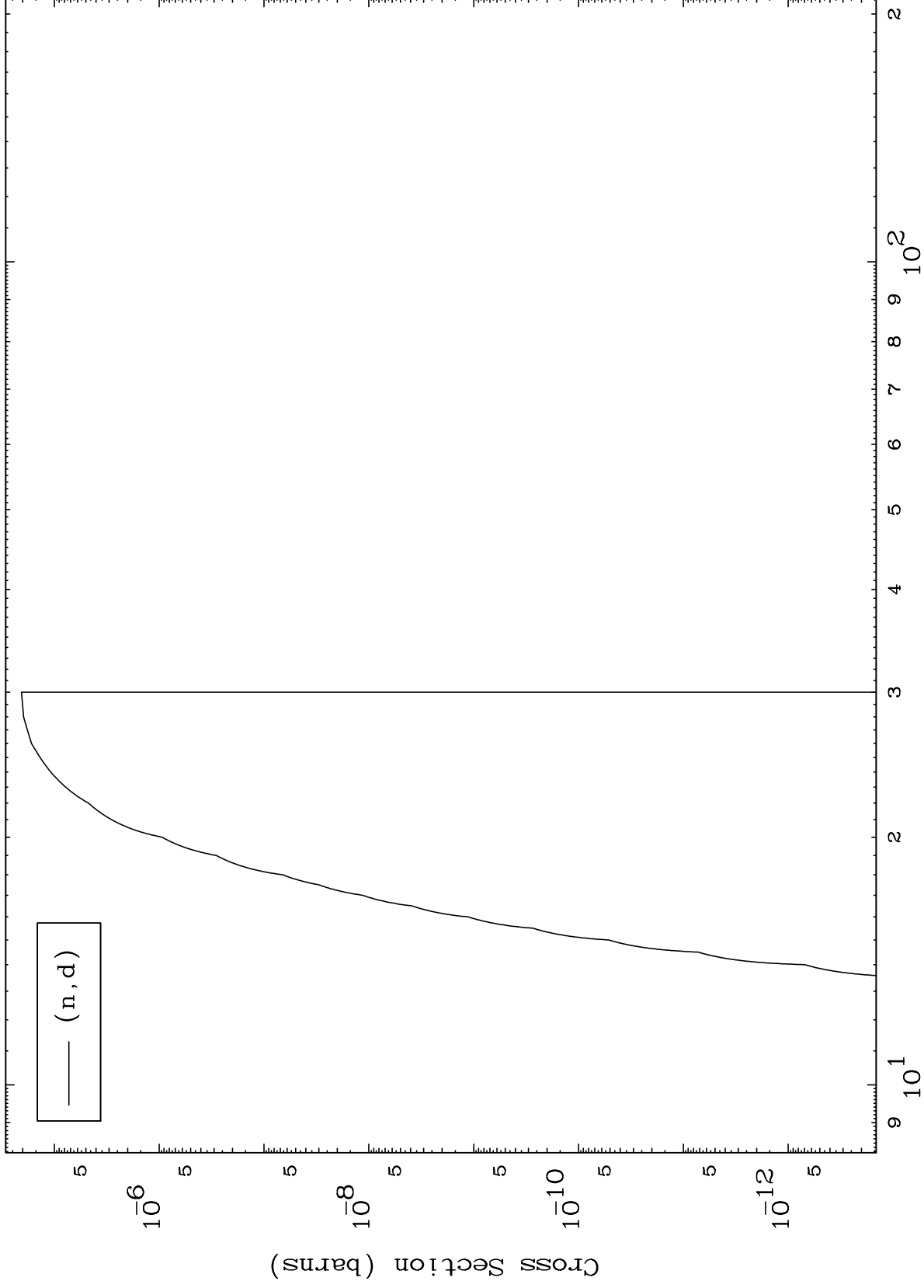
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( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

97-Bk-247



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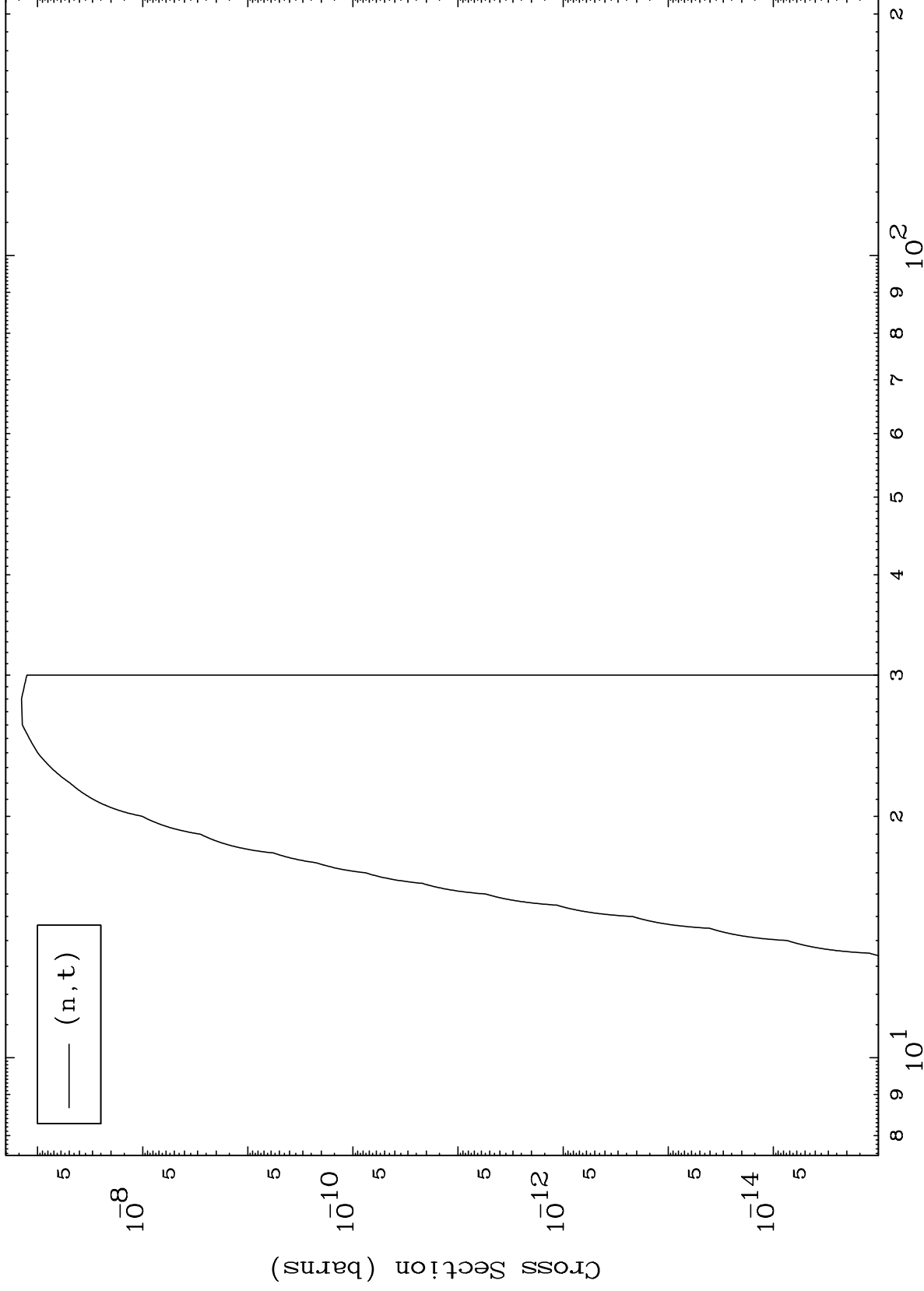
Incident Energy (MeV)

97-Bk-247

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( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

97-Bk-247



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Incident Energy (MeV)

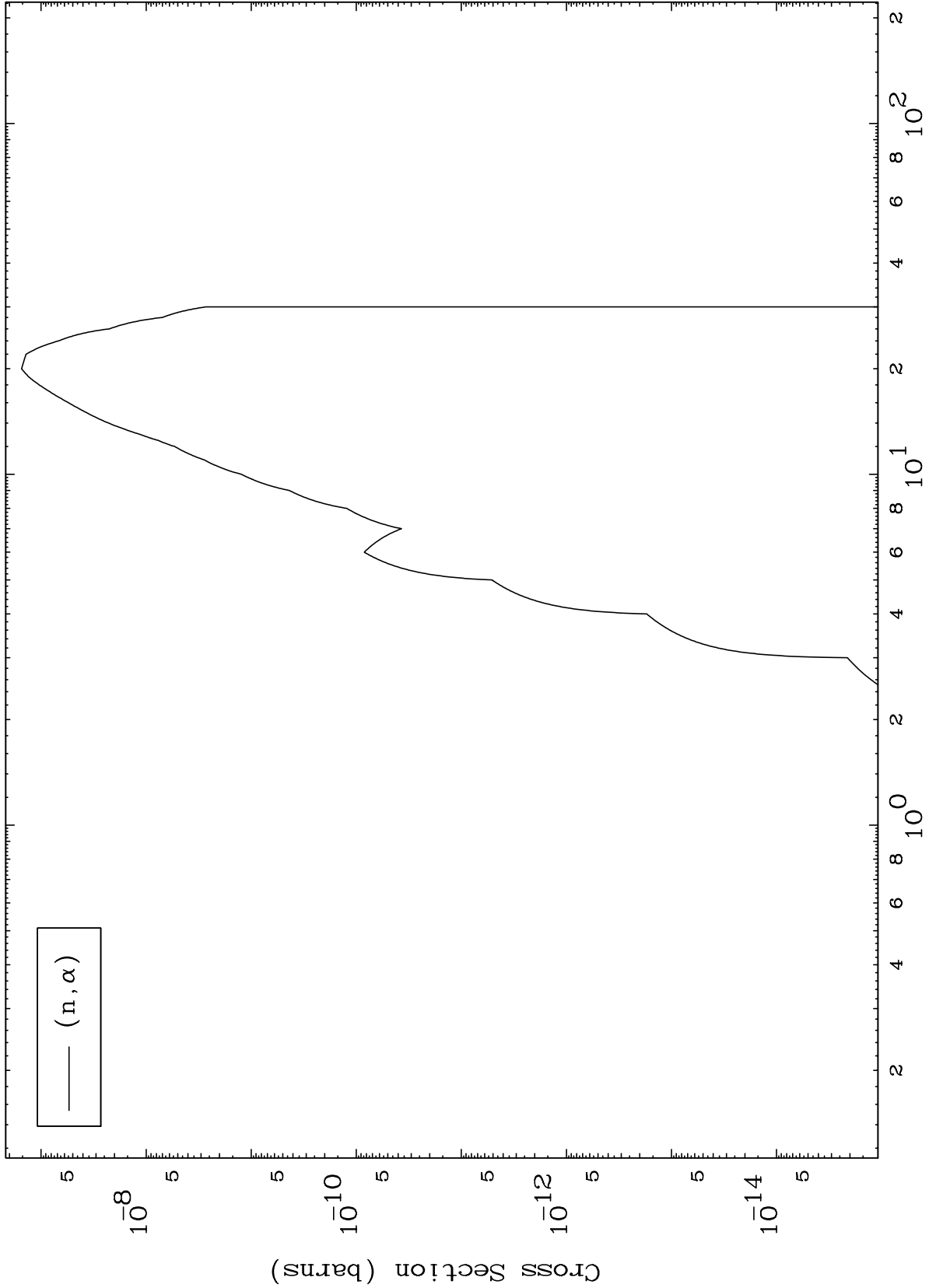
97-Bk-247

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( $\gamma, \alpha$ ) Levels

97-Bk-247

0 Kelvin Cross Sections

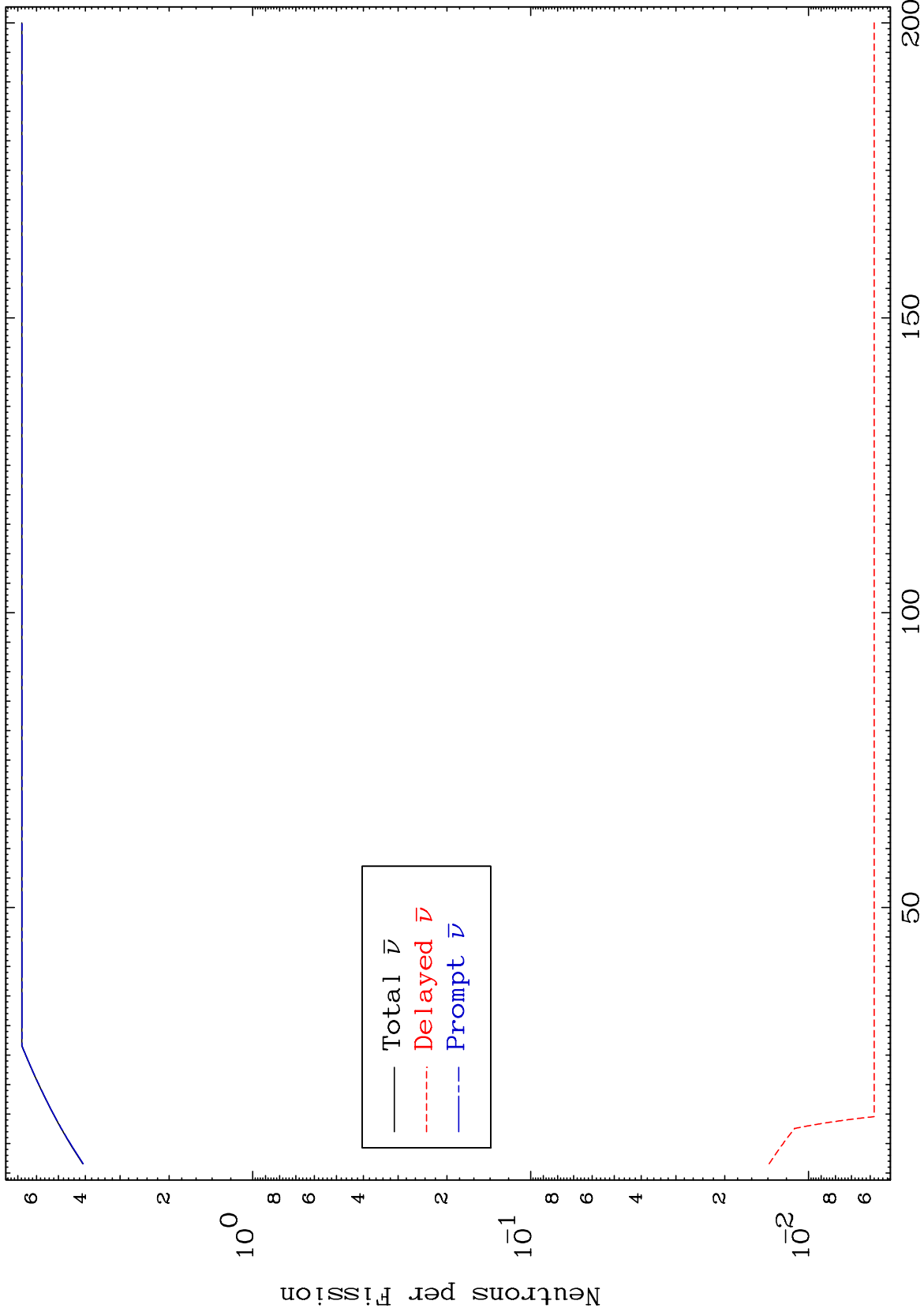


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Incident Energy (MeV)

97-Bk-247

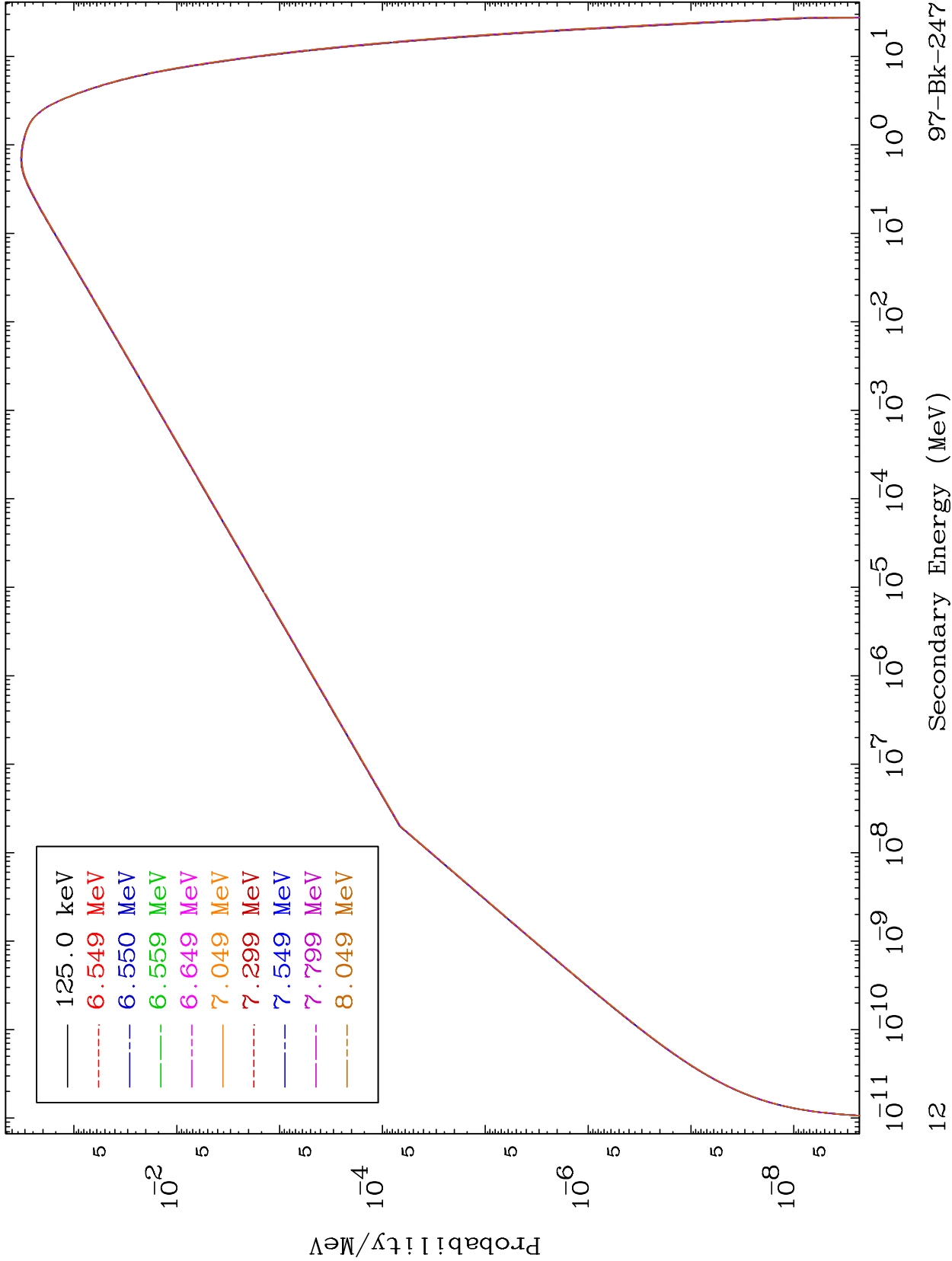
Photon Neutrons  
per Fission



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### Fission Energy Distributions

97-Bk-247

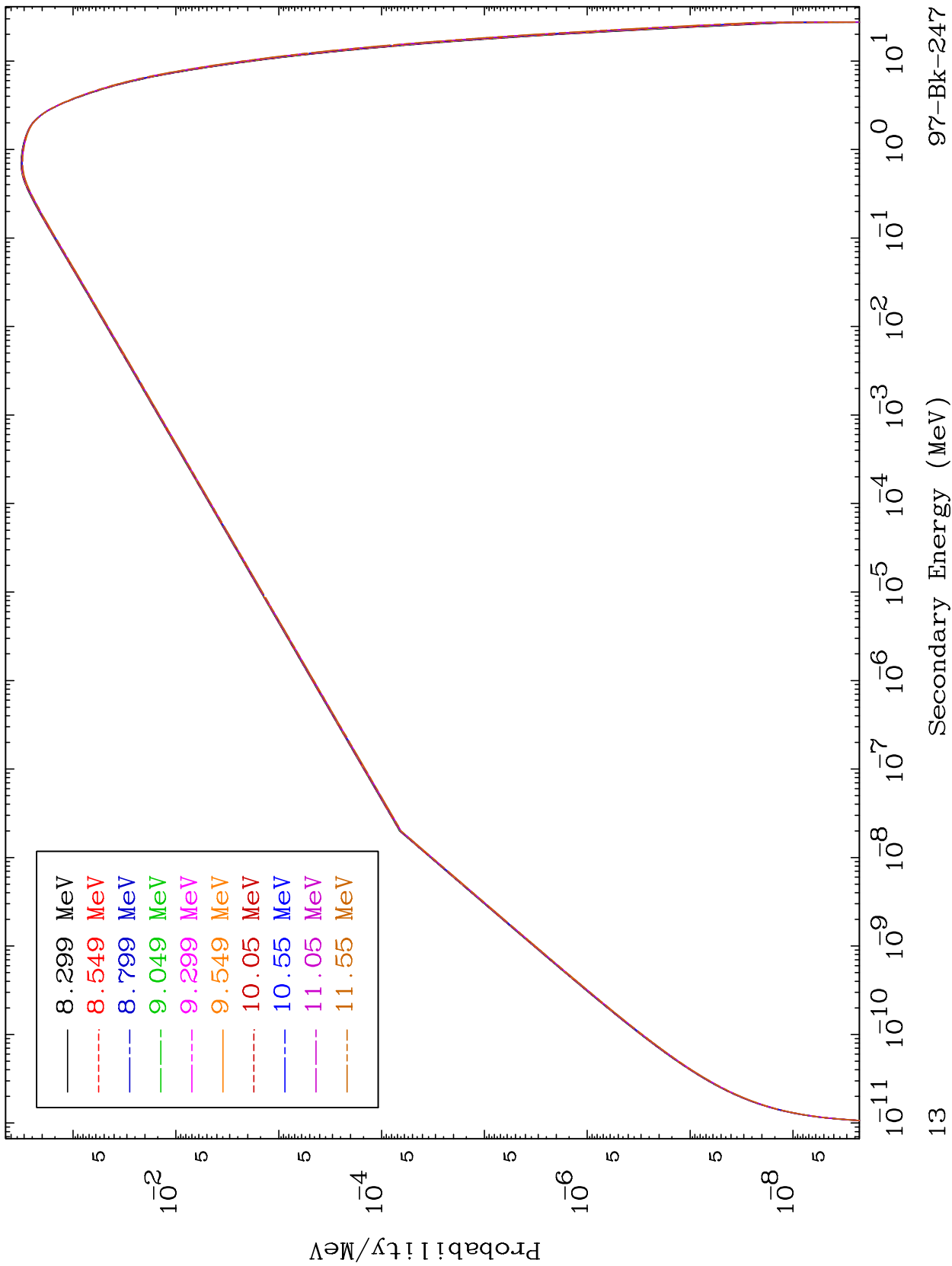


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### Fission Energy Distributions

97-Bk-247



97-Bk-247

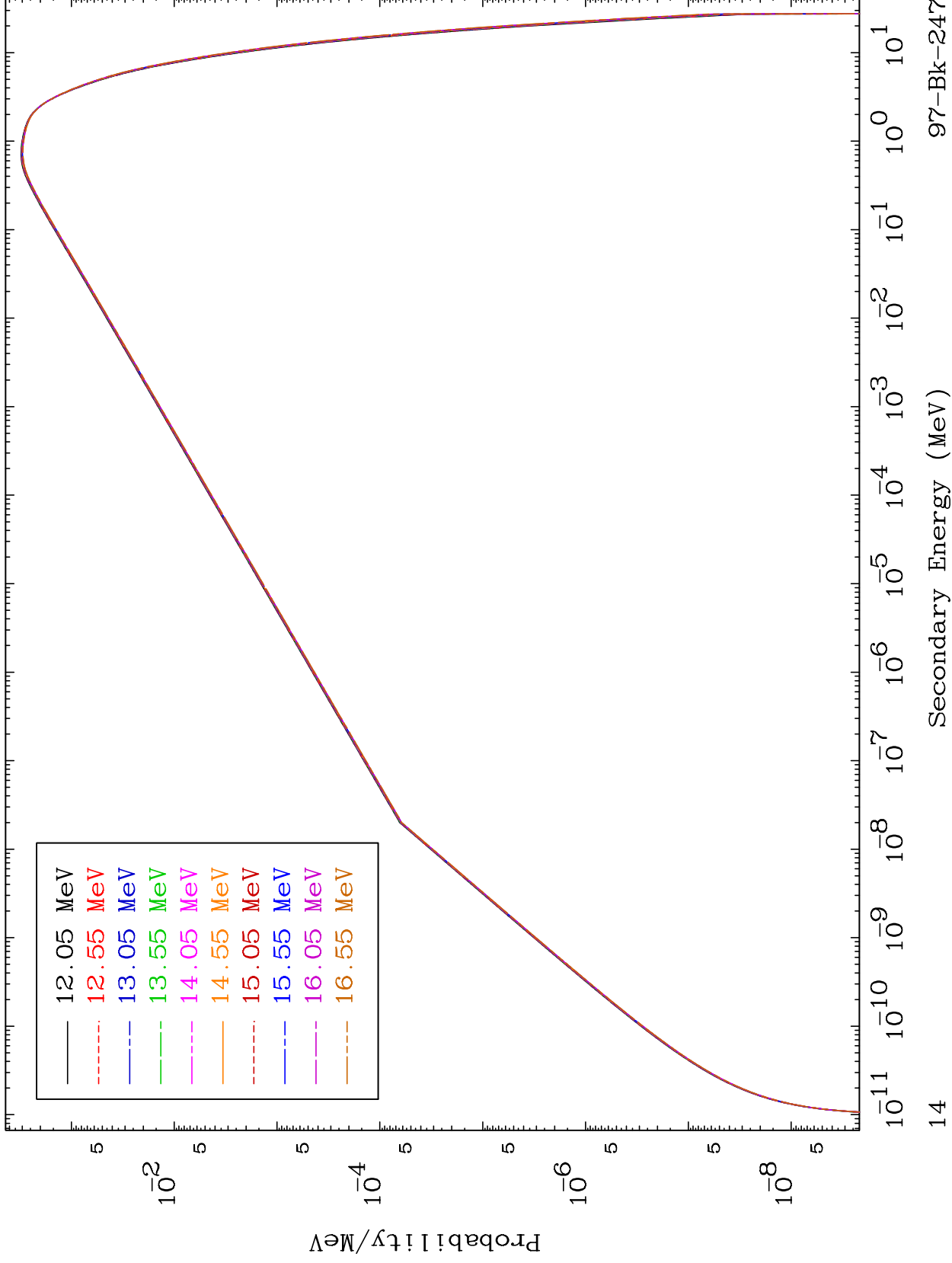
Secondary Energy (MeV)

13

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### Fission Energy Distributions

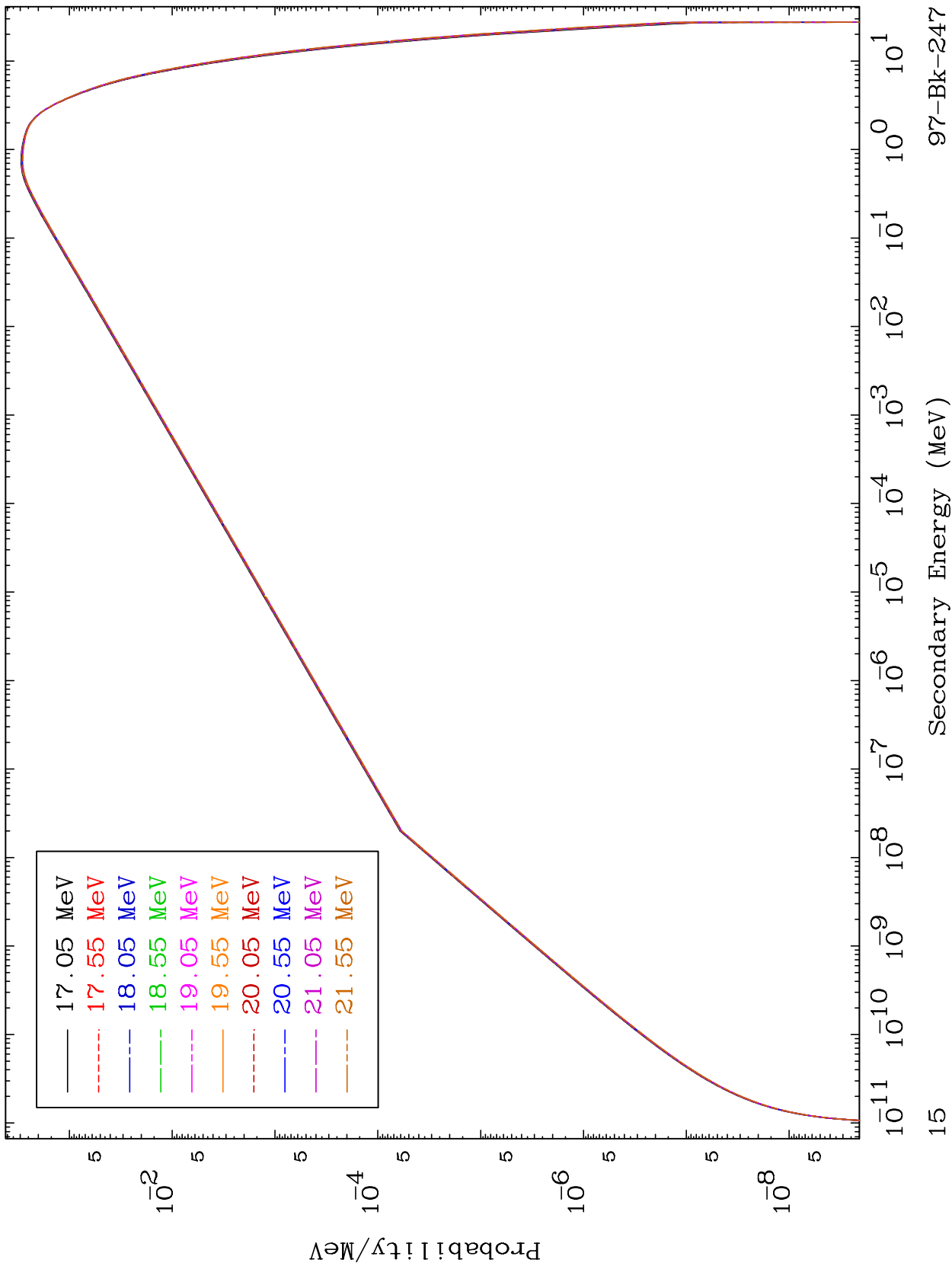
97-Bk-247



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### Fission Energy Distributions

97-Bk-247



97-Bk-247

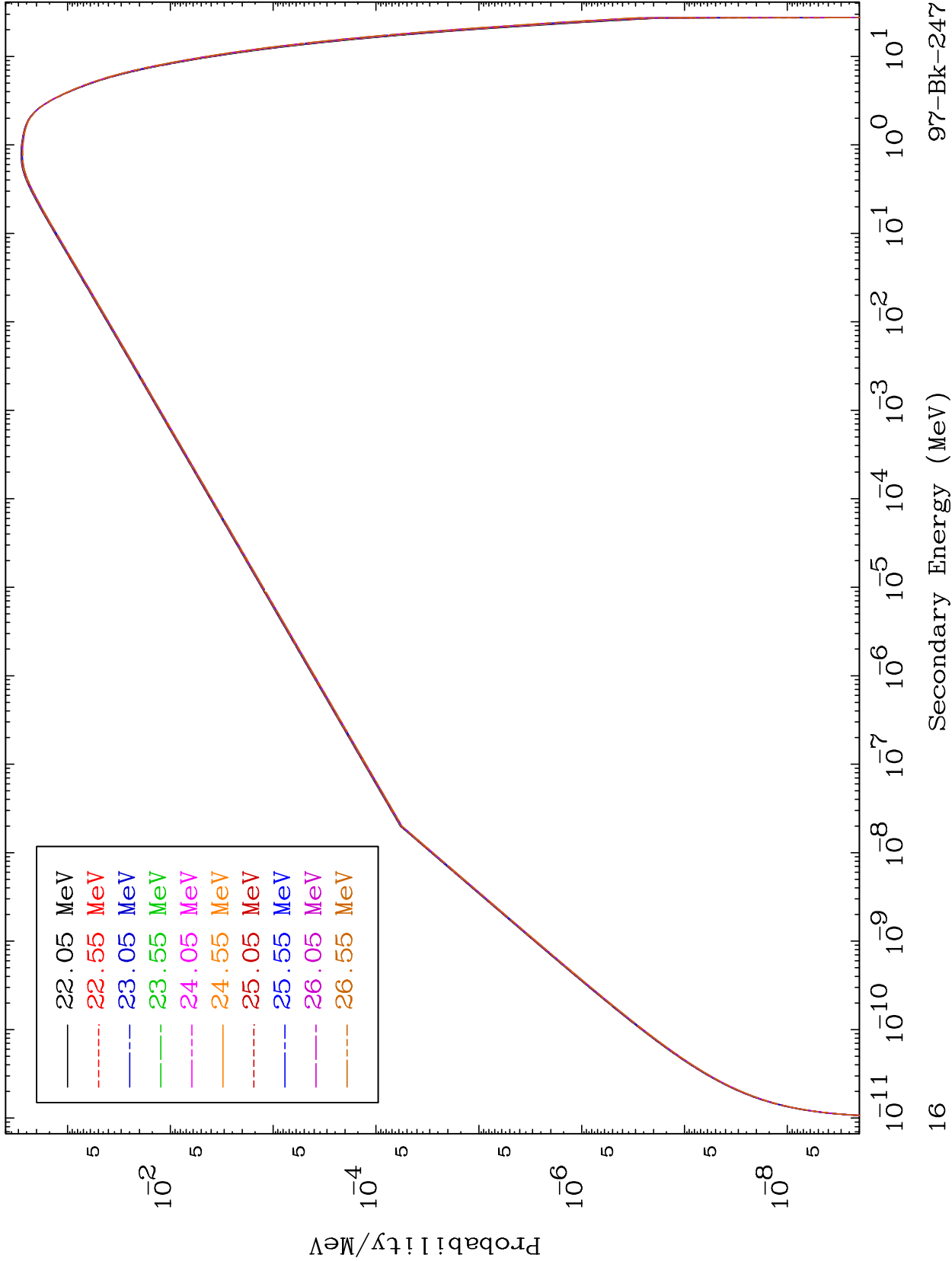
Secondary Energy (MeV)



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### Fission Energy Distributions

97-Bk-247



97-Bk-247

Secondary Energy (MeV)

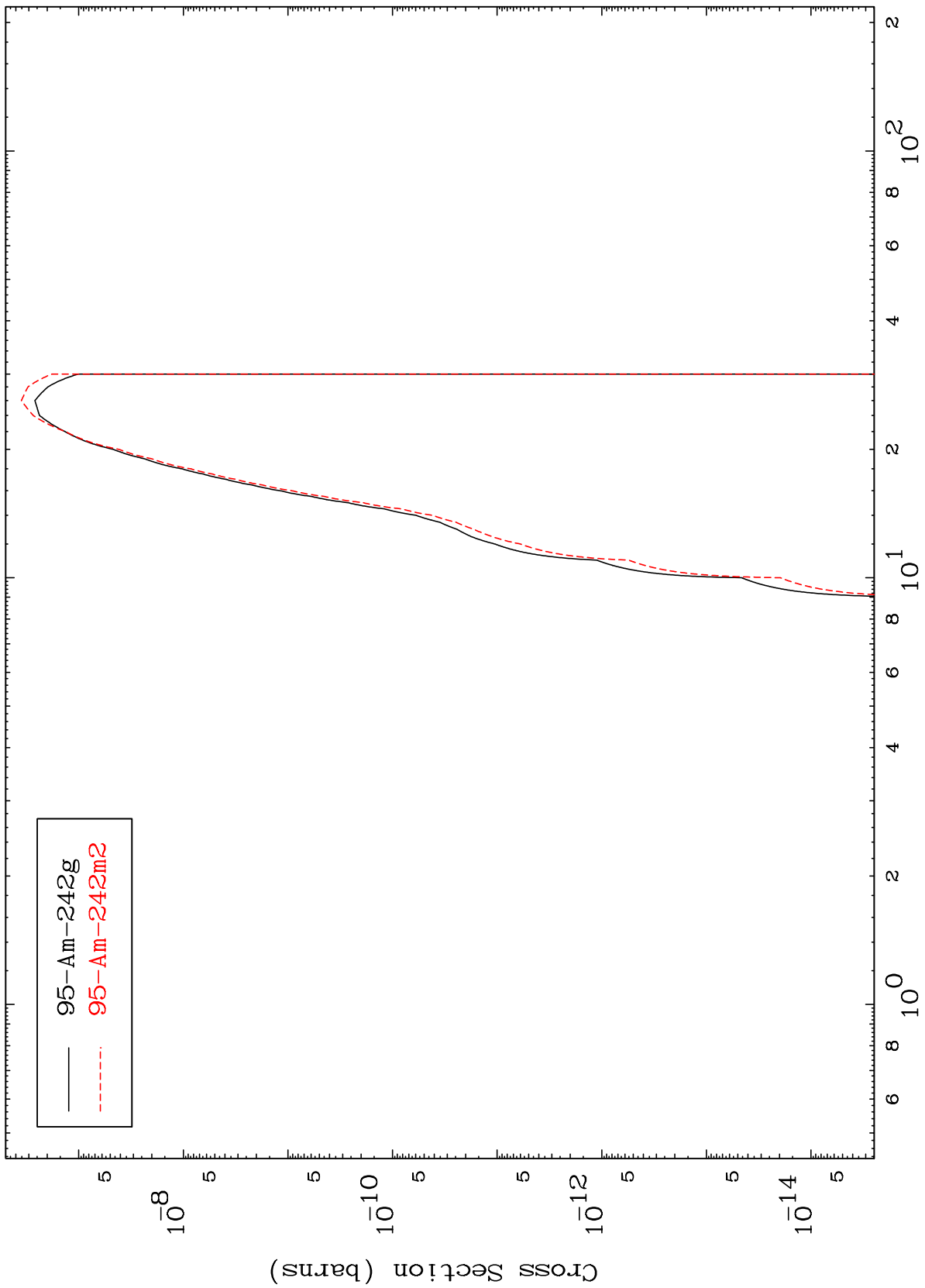
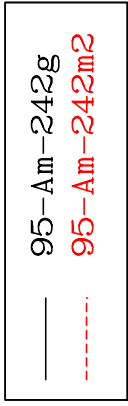
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(n,n')  $\alpha$

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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