

Program EVALPLOT  
(Version 2021-1)

by

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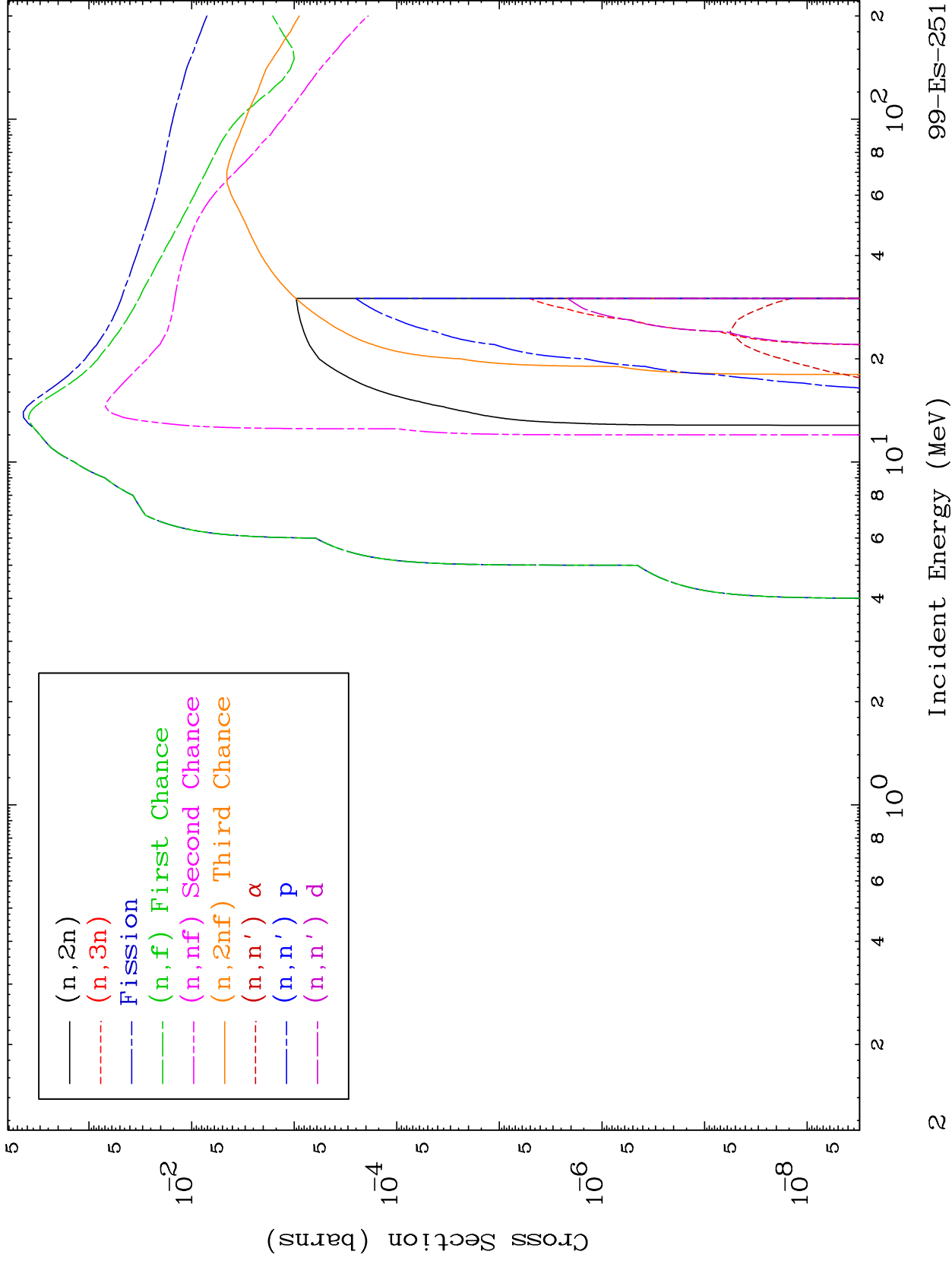
Press Mouse Button to Start

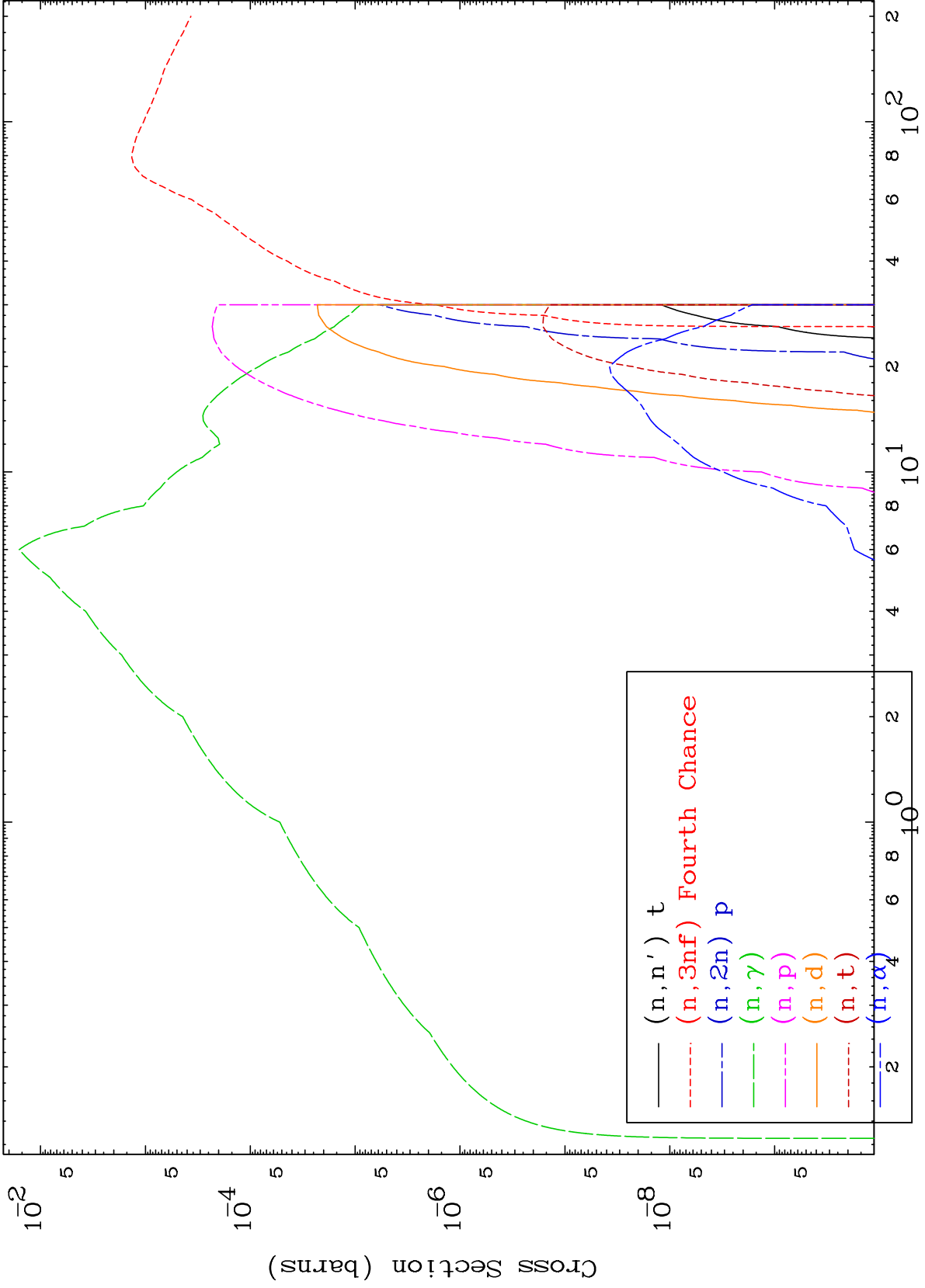


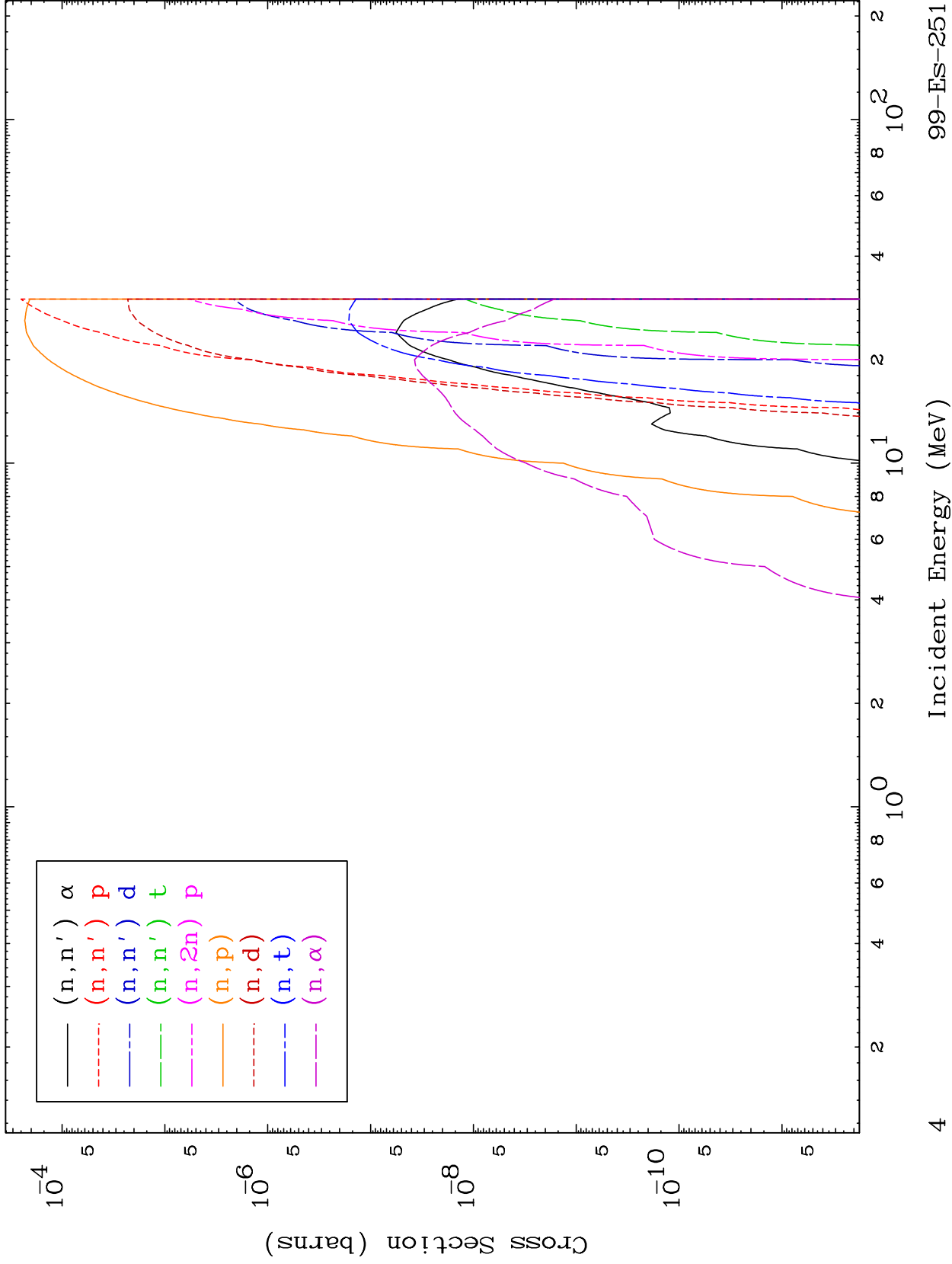
MAT 9911

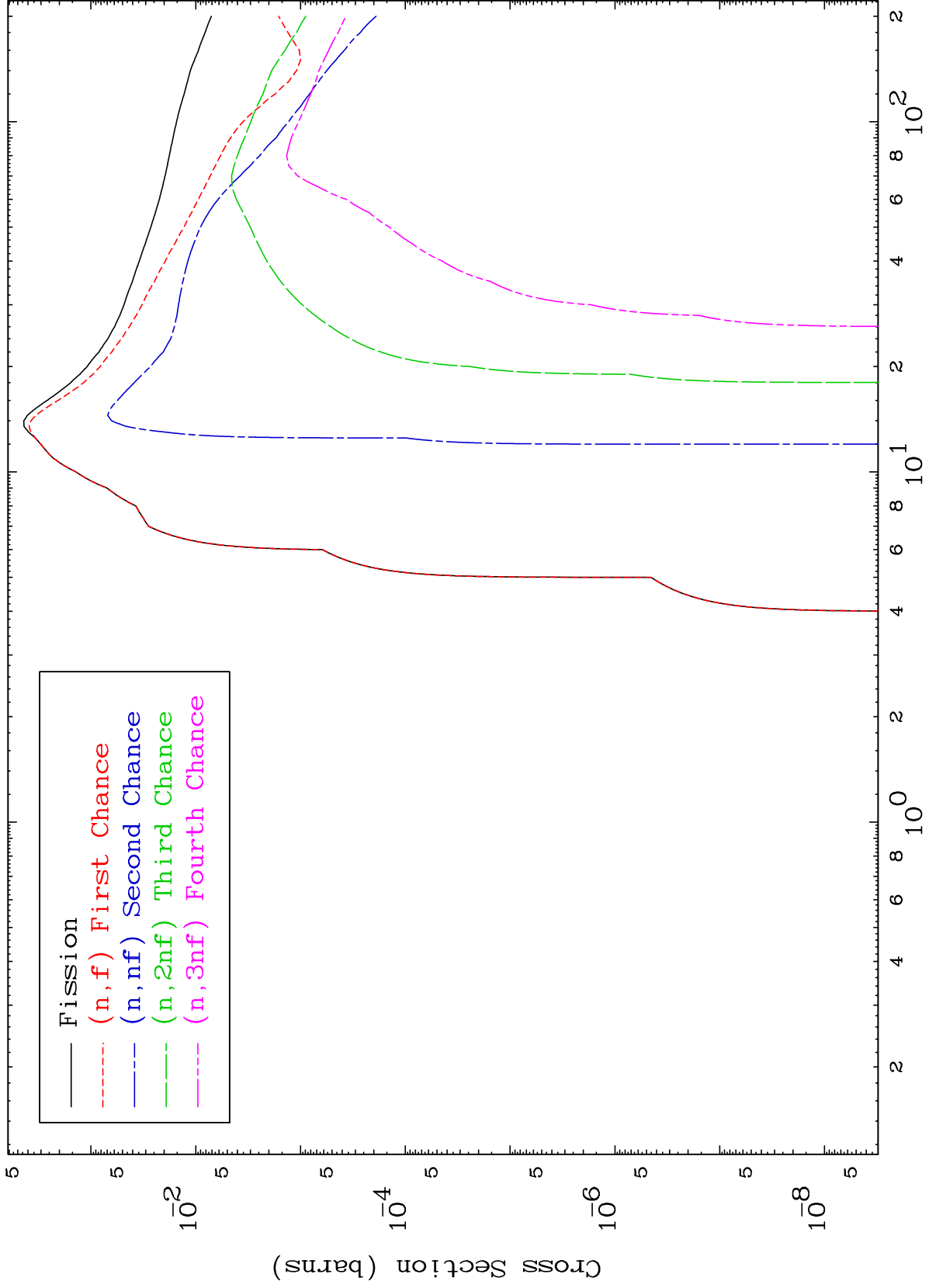
Photon Neutron Absorption  
0 Kelvin Cross Sections

99-Es-251





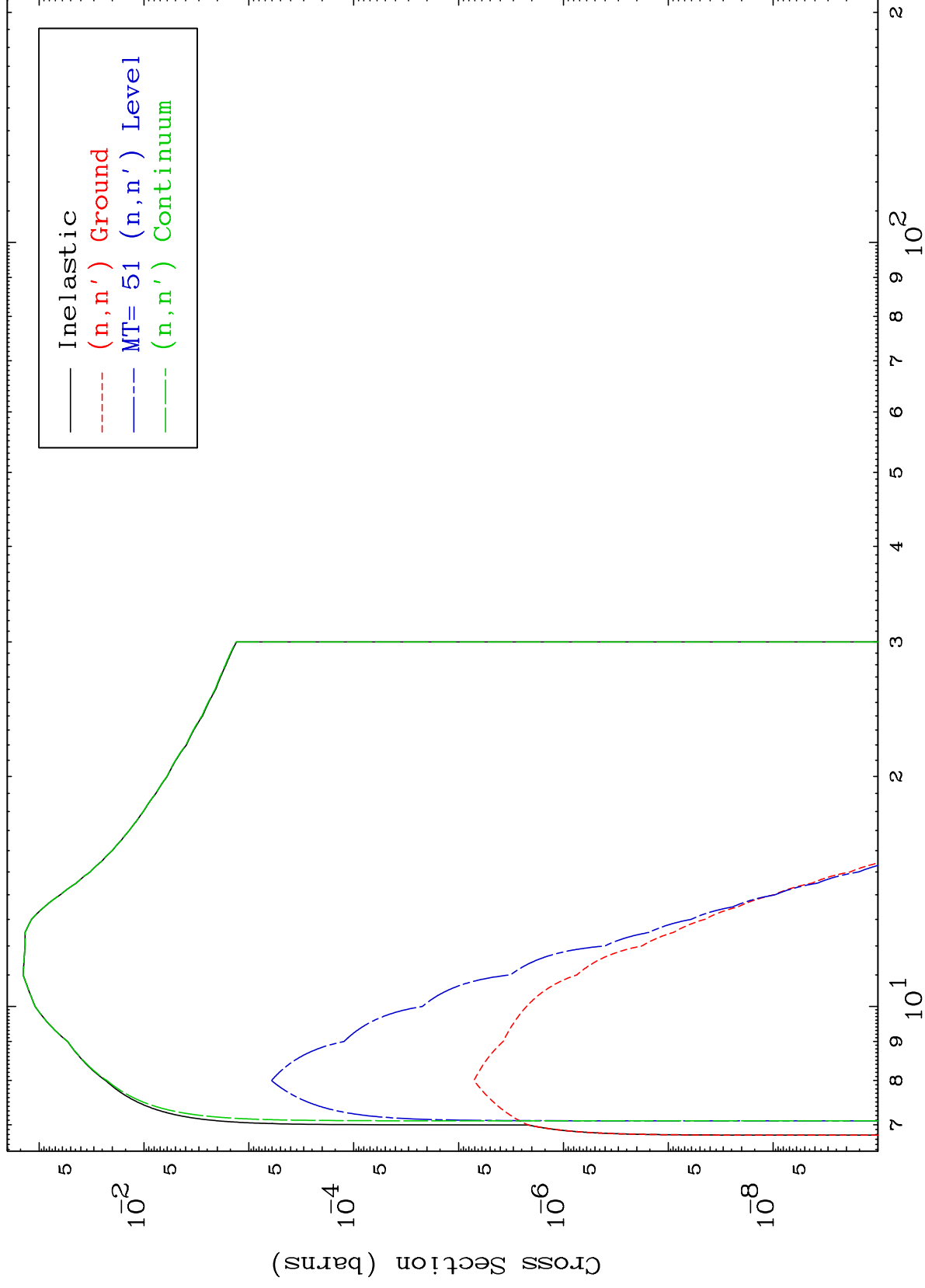




MAT 9911

99-Es-251

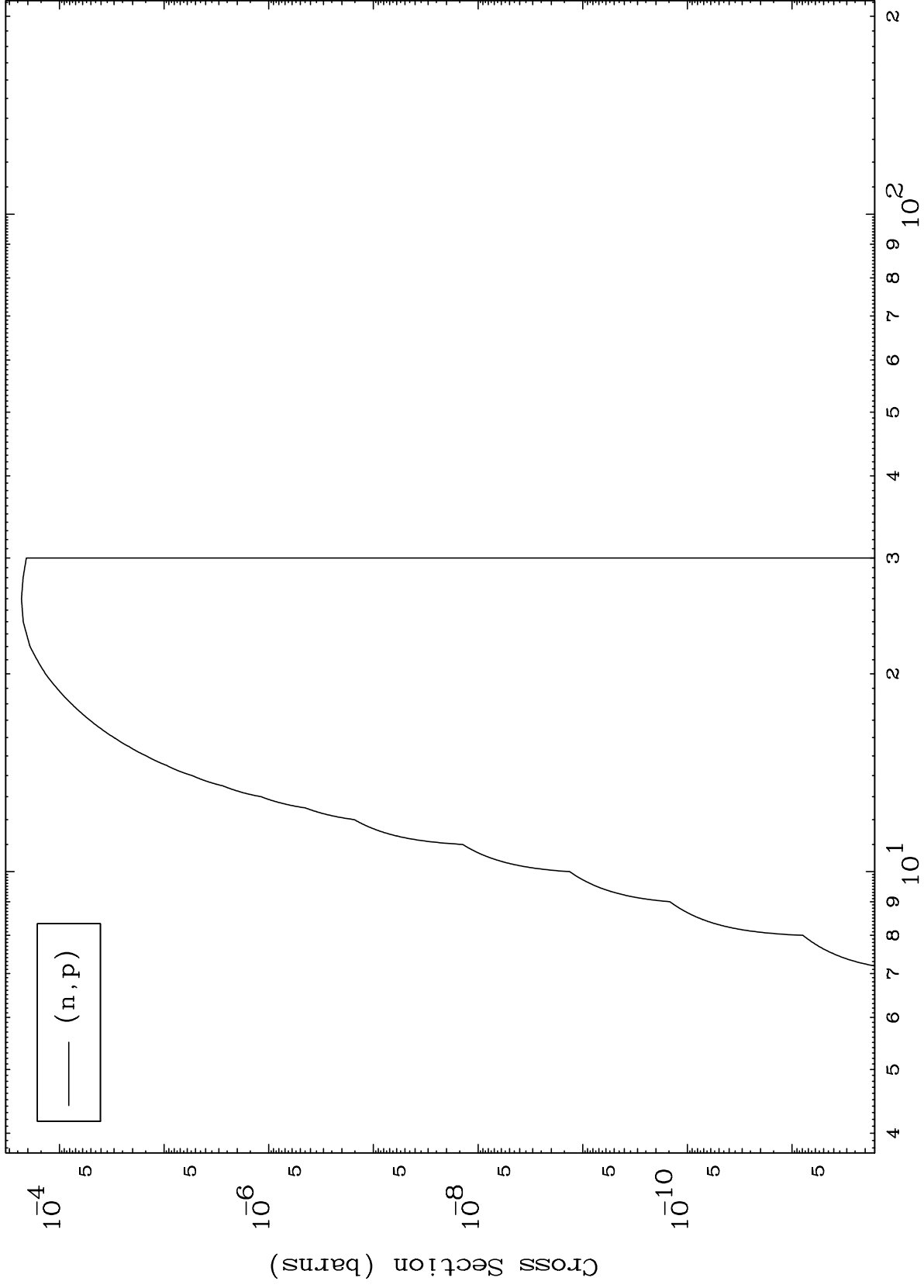
( $\gamma, n'$ ) Levels  
0 Kelvin Cross Sections



99-Es-251

Incident Energy (MeV)

6

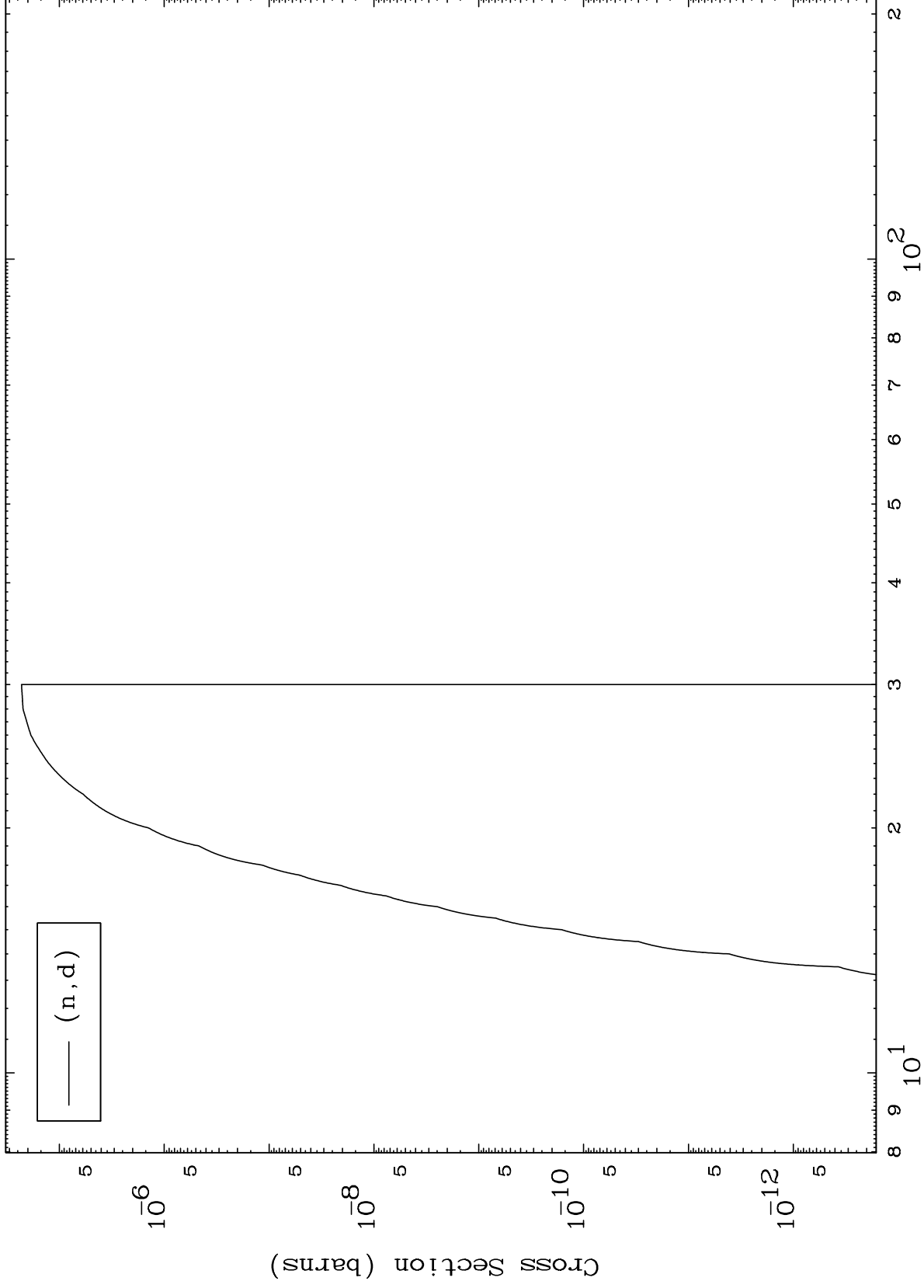




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( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

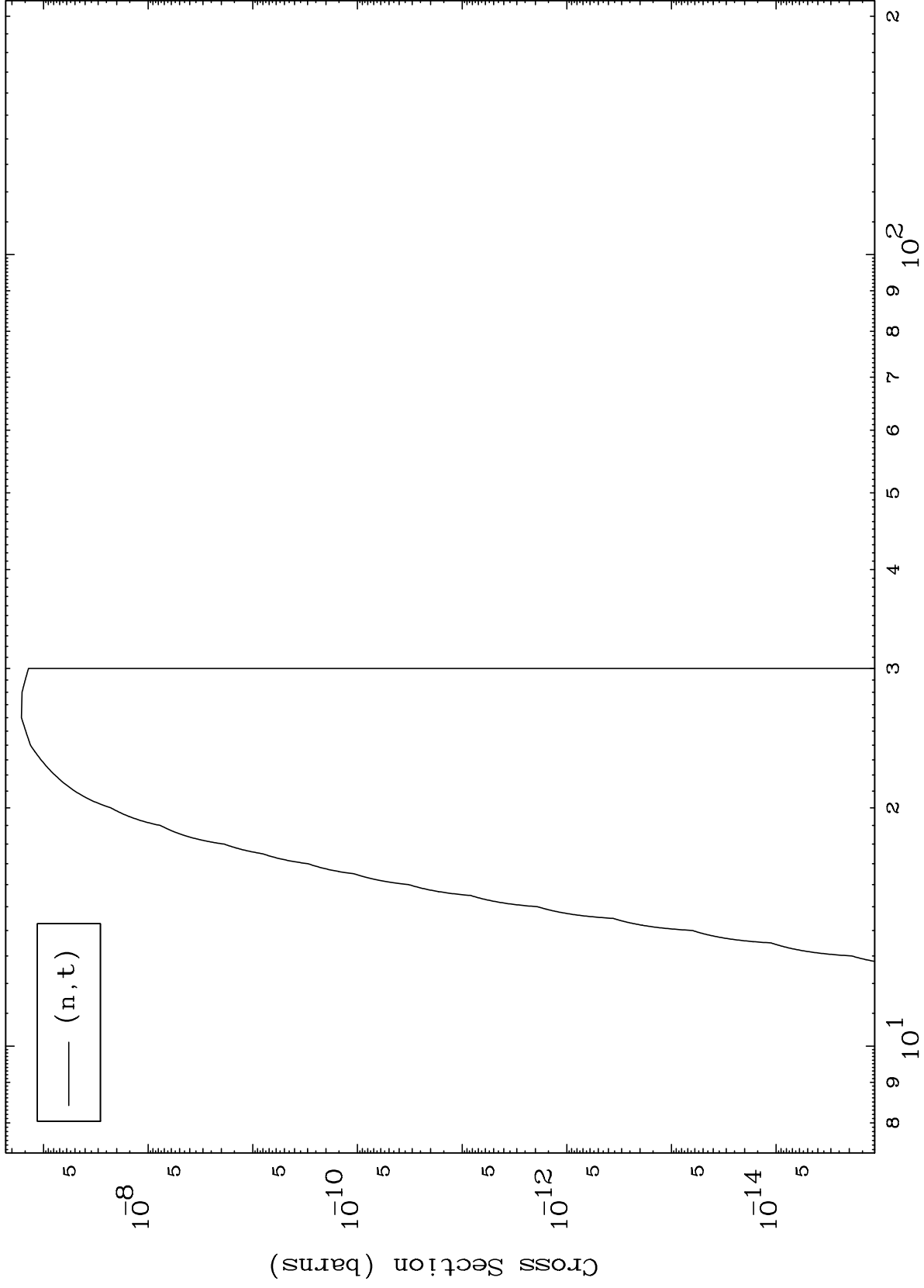
99-Es-251



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Incident Energy (MeV)

99-Es-251

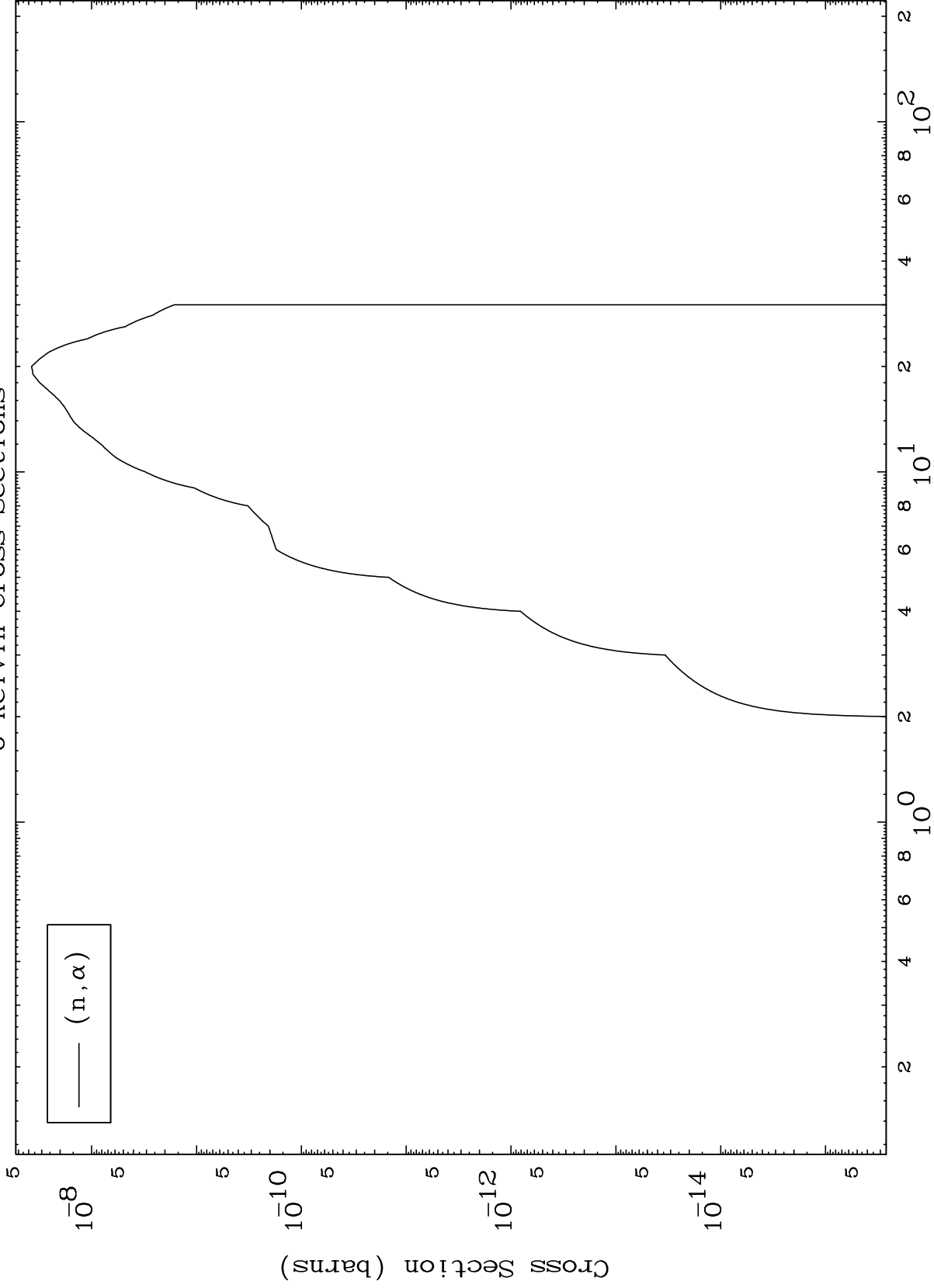


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( $\gamma, \alpha$ ) Levels

99-Es-251

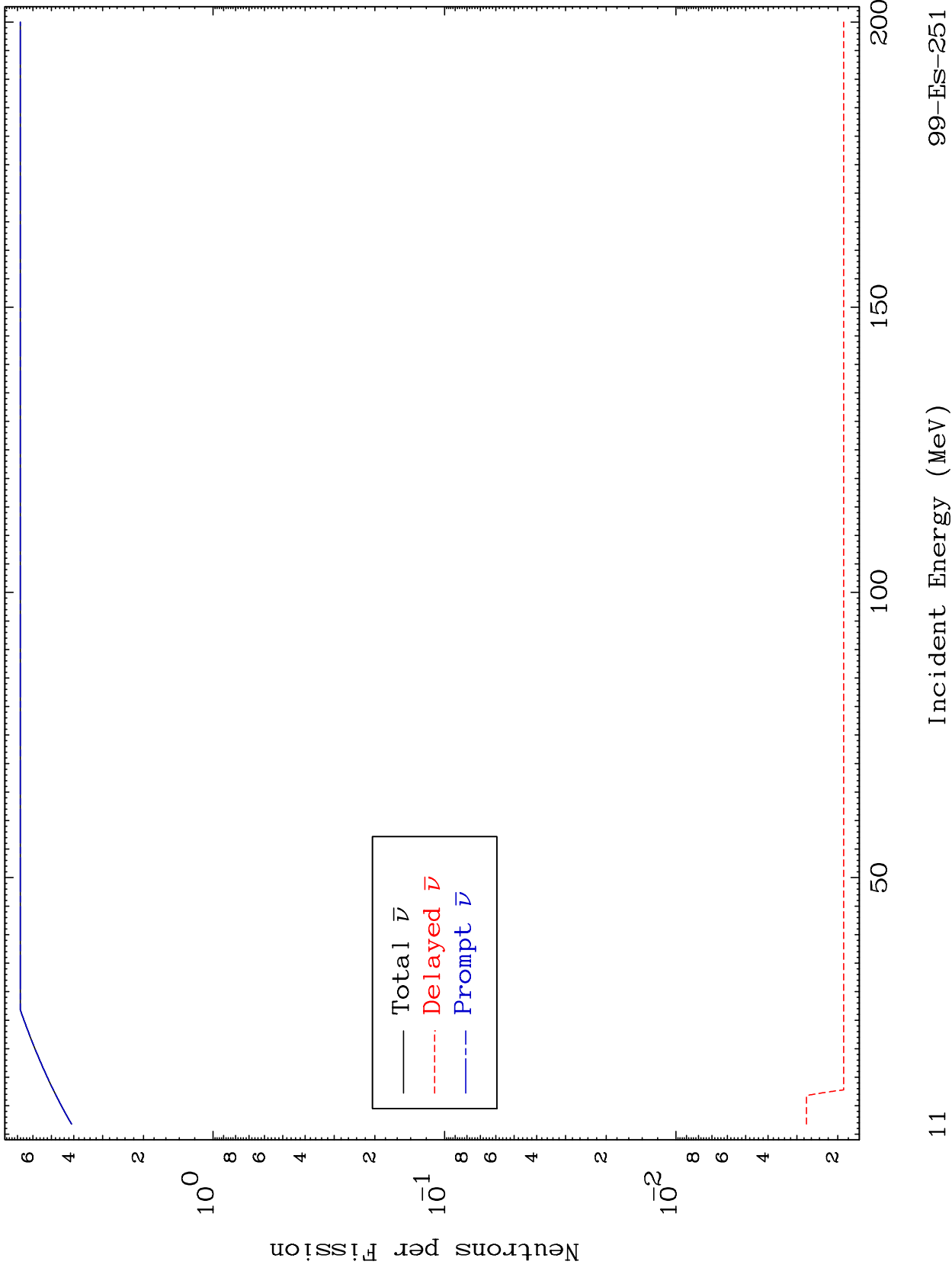
0 Kelvin Cross Sections



10

Incident Energy (MeV)

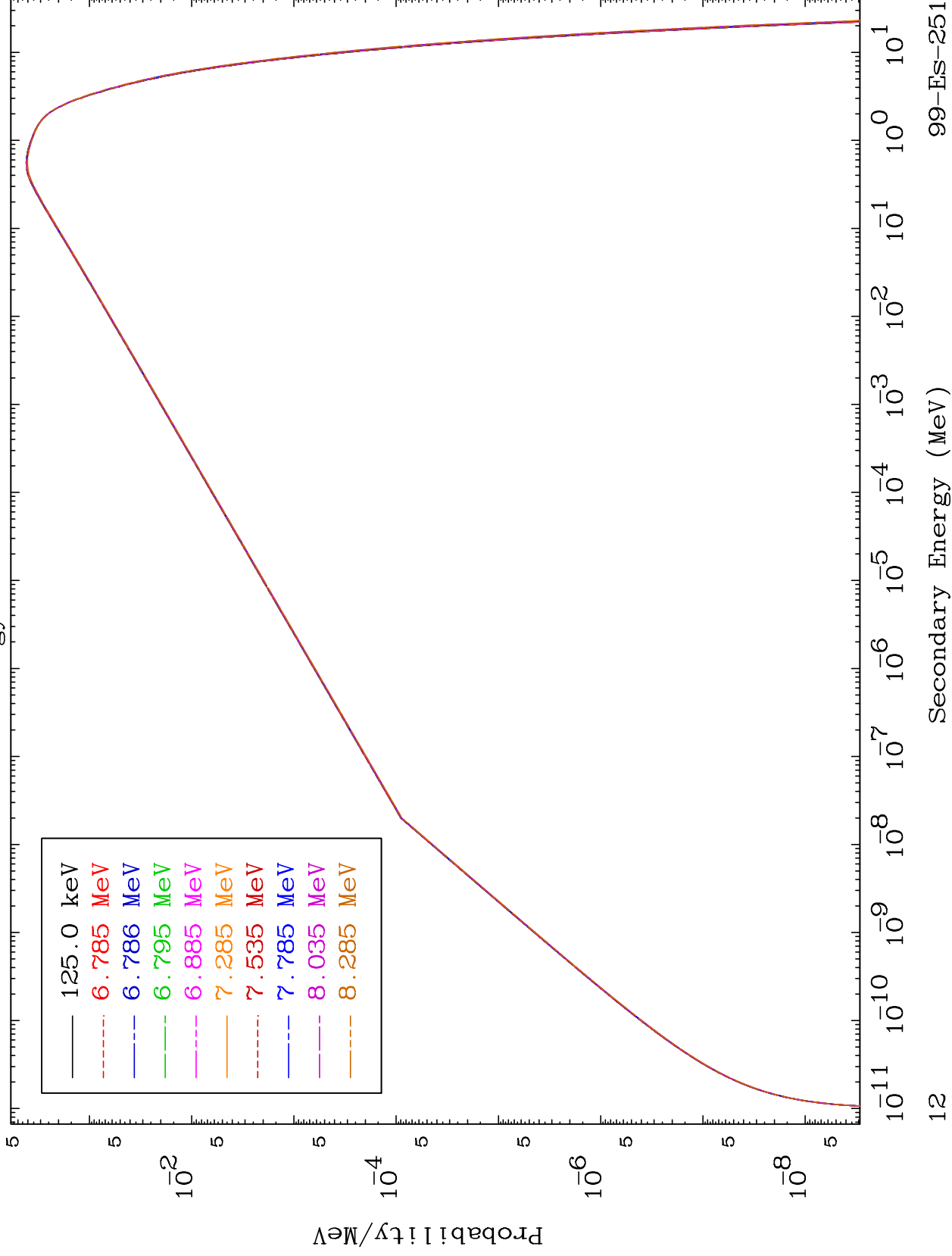
99-Es-251



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### Fission Energy Distributions

99-Es-251

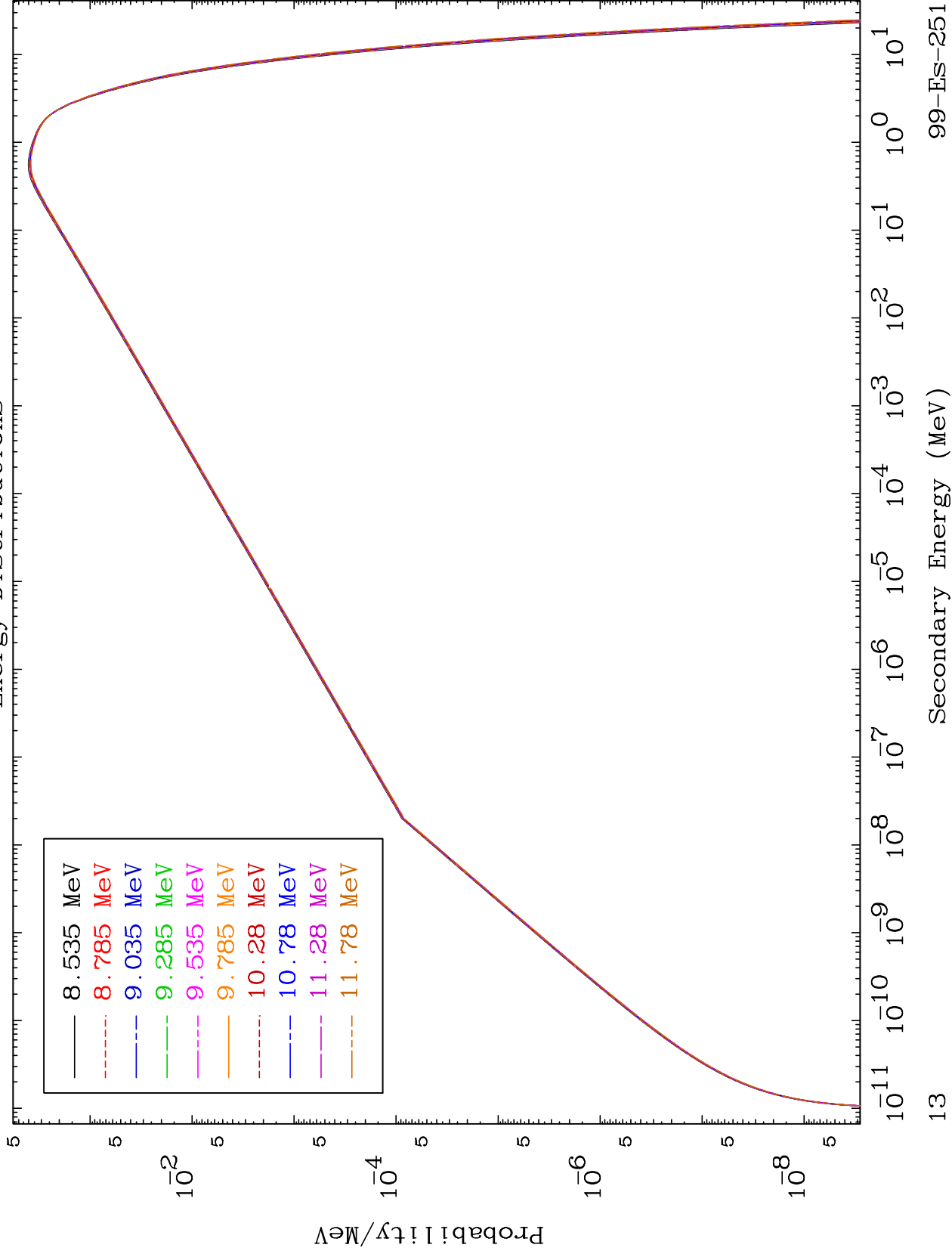


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### Fission Energy Distributions

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99-Es-251

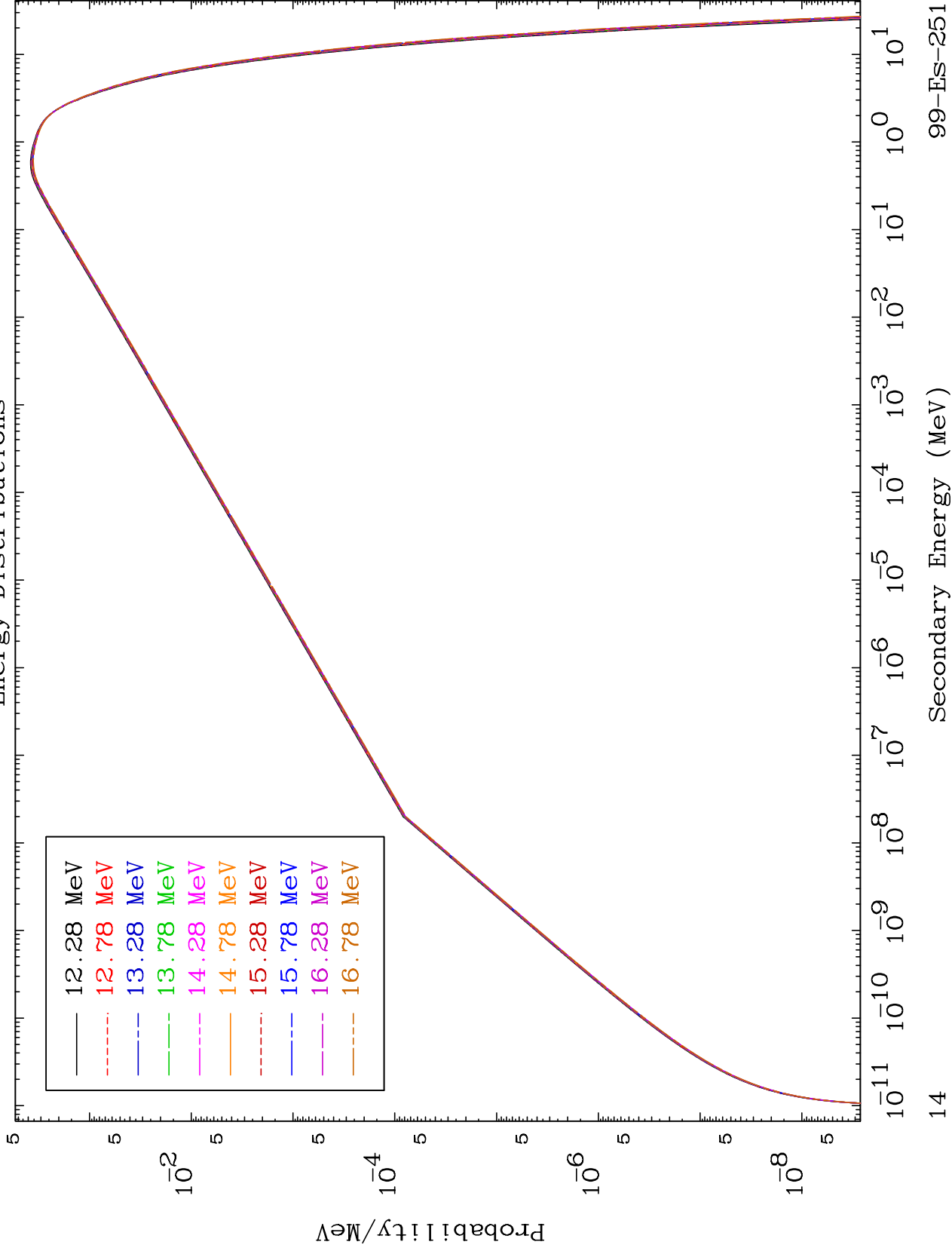
Secondary Energy (MeV)

13

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### Fission Energy Distributions

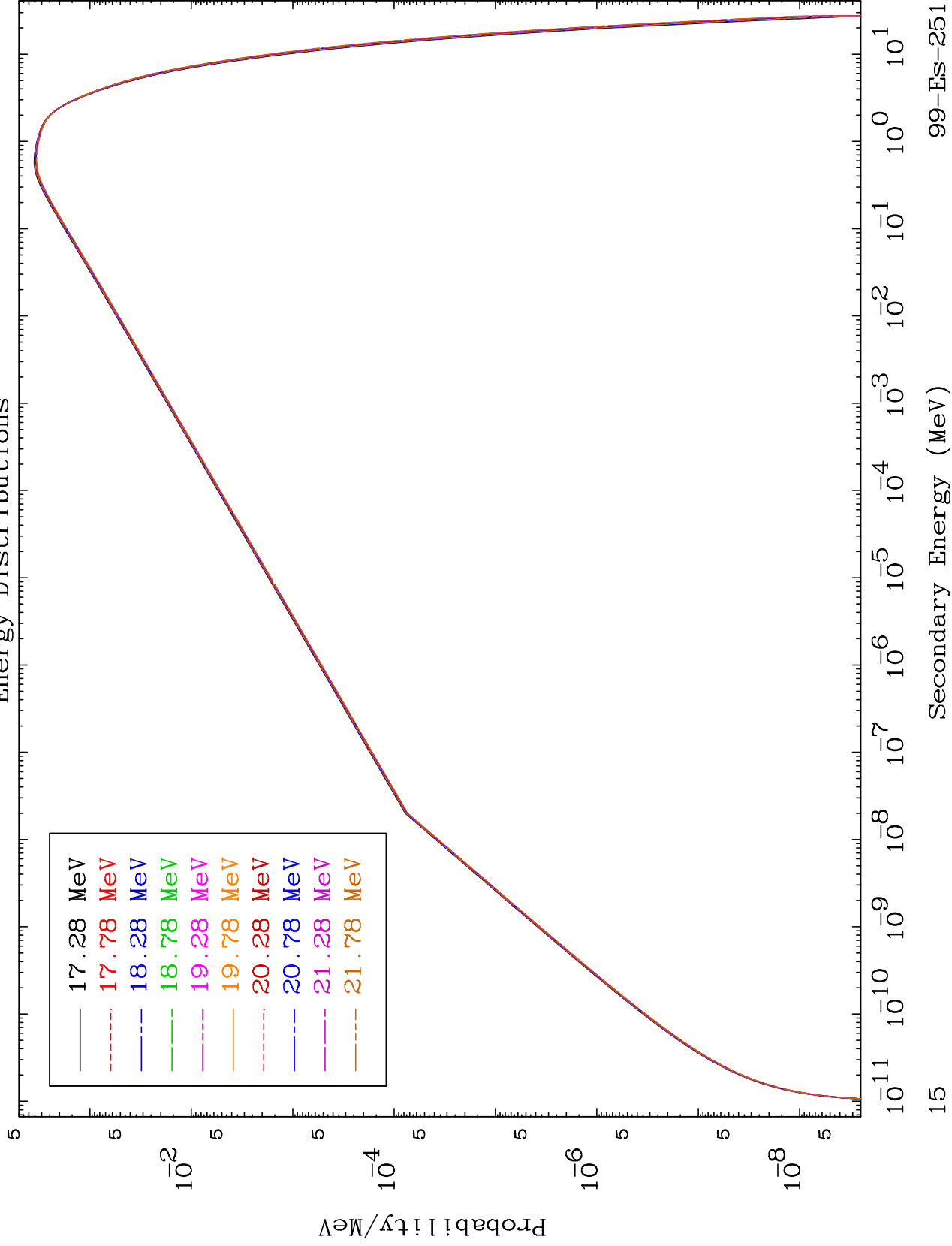
99-Es-251



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### Fission Energy Distributions

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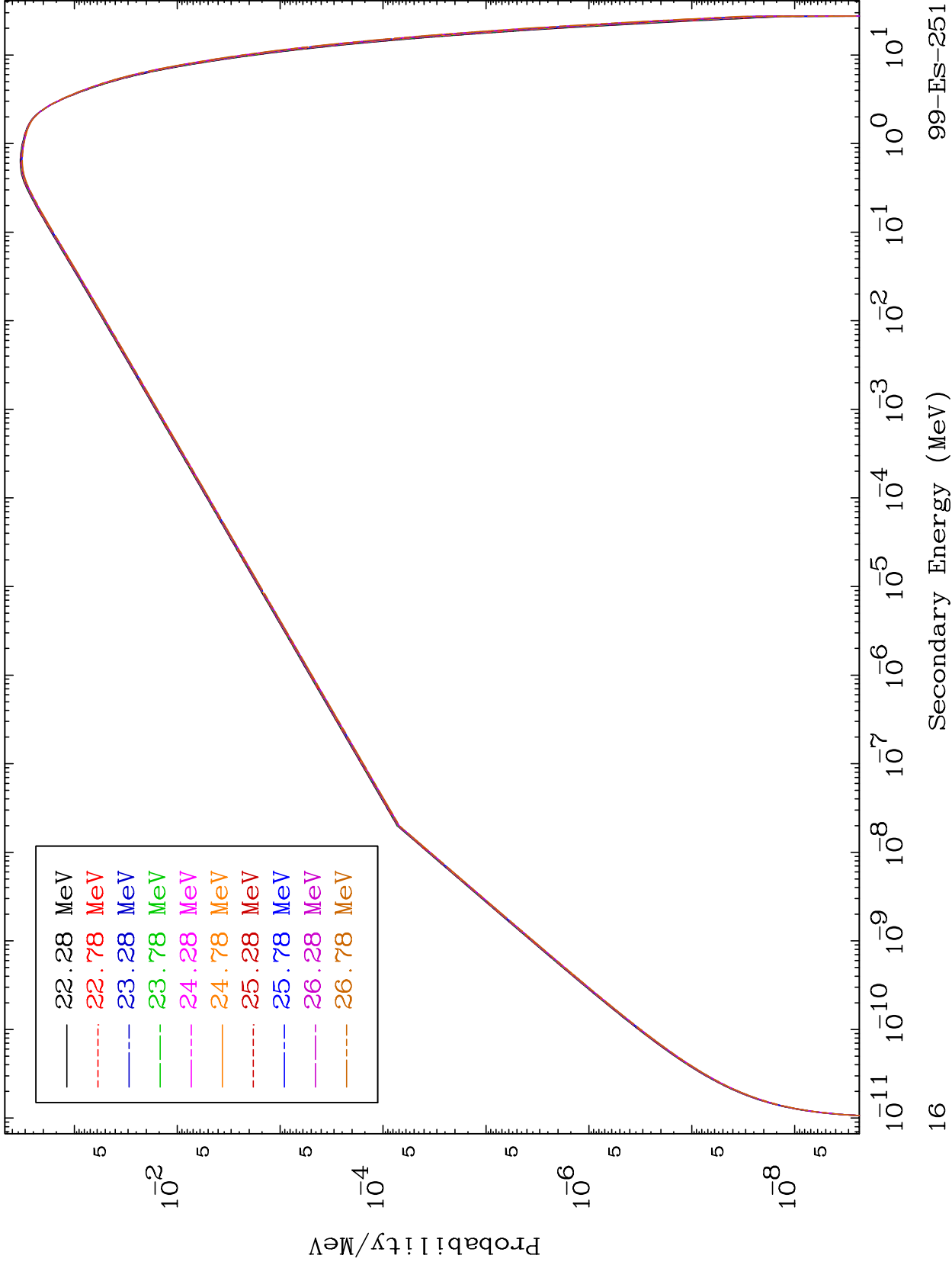
99-Es-251

Secondary Energy (MeV)

15



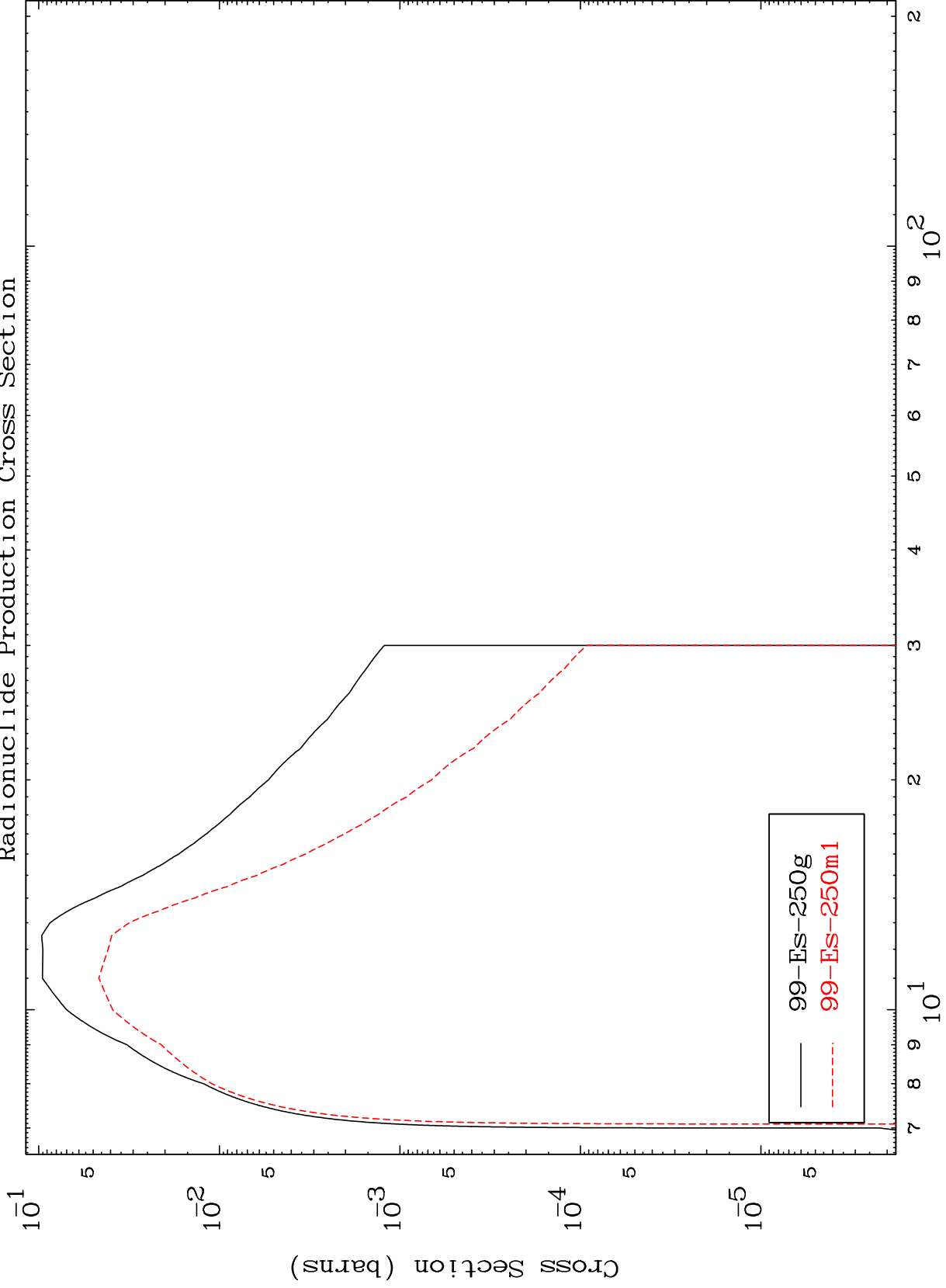
Fission Energy Distributions



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Inelastic  
Radionuclide Production Cross Section

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Incident Energy (MeV)

99-Es-251