

Program EVALPLOT
(Version 2021-1)

by

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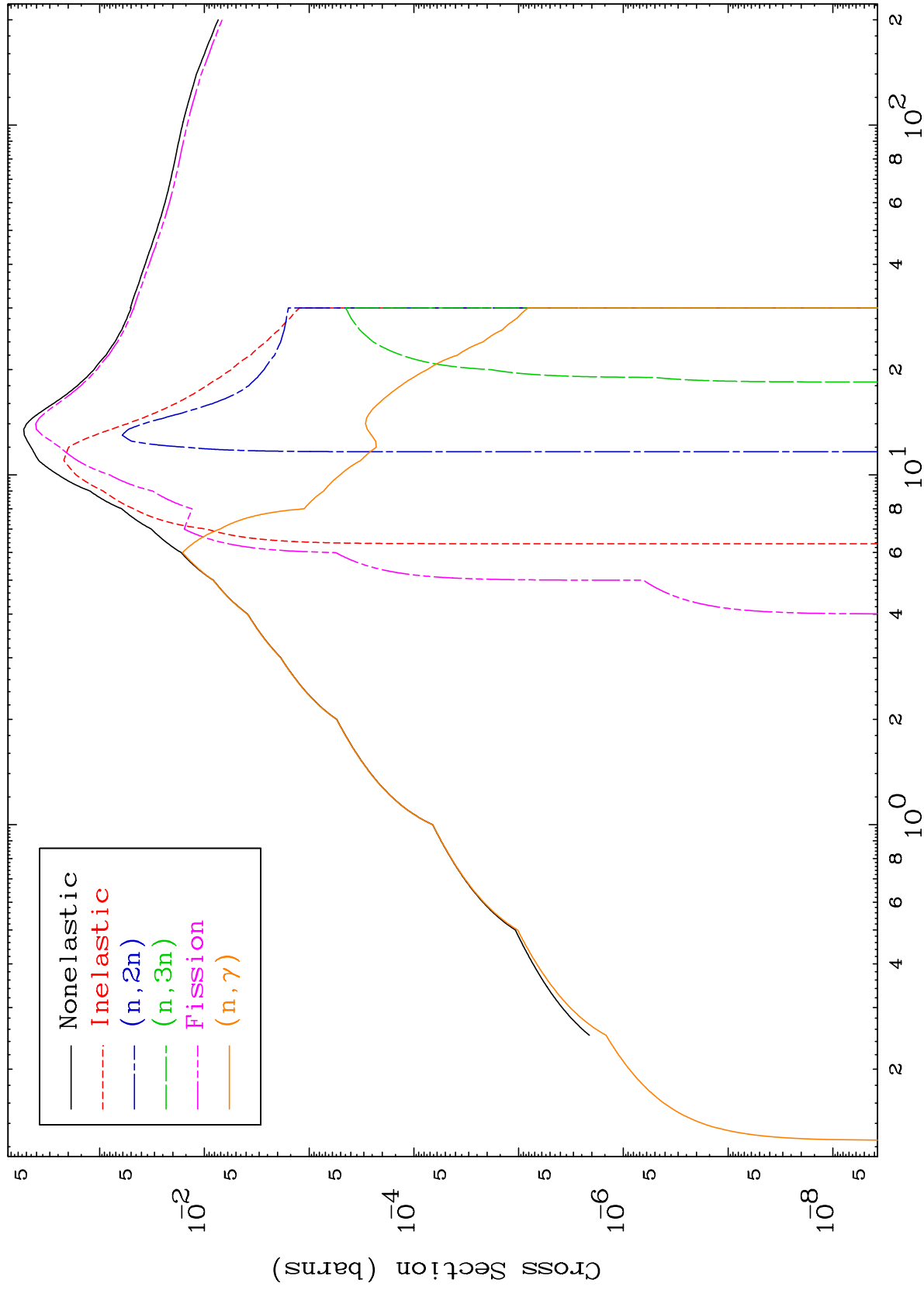
Press Mouse Button to Start

MAT 9913

Photon Major

99-Es-253

0 Kelvin Cross Sections



1

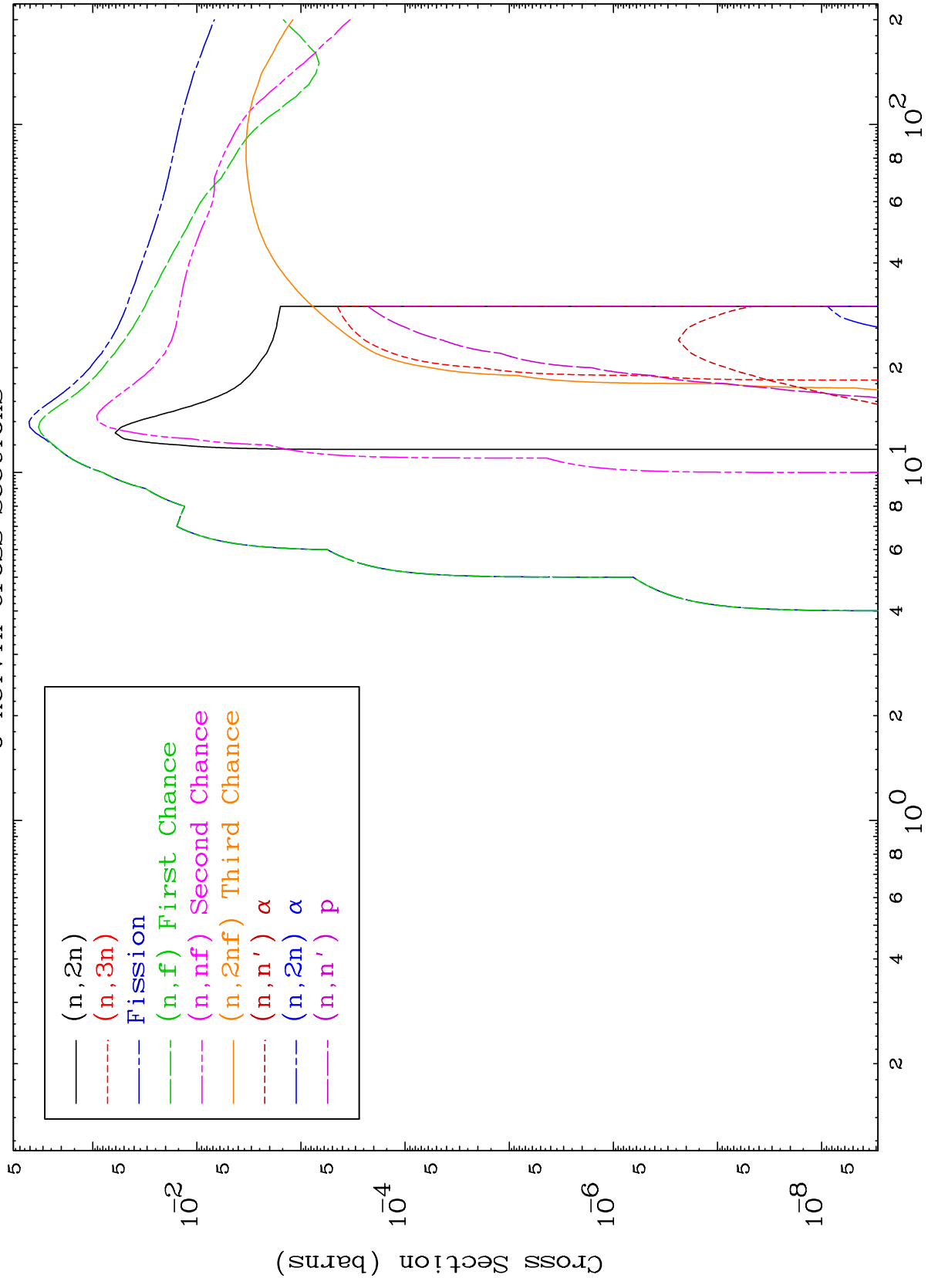
Incident Energy (MeV)

99-Es-253

MAT 9913

Photon Neutron Absorption
0 Kelvin Cross Sections

99-Es-253



2

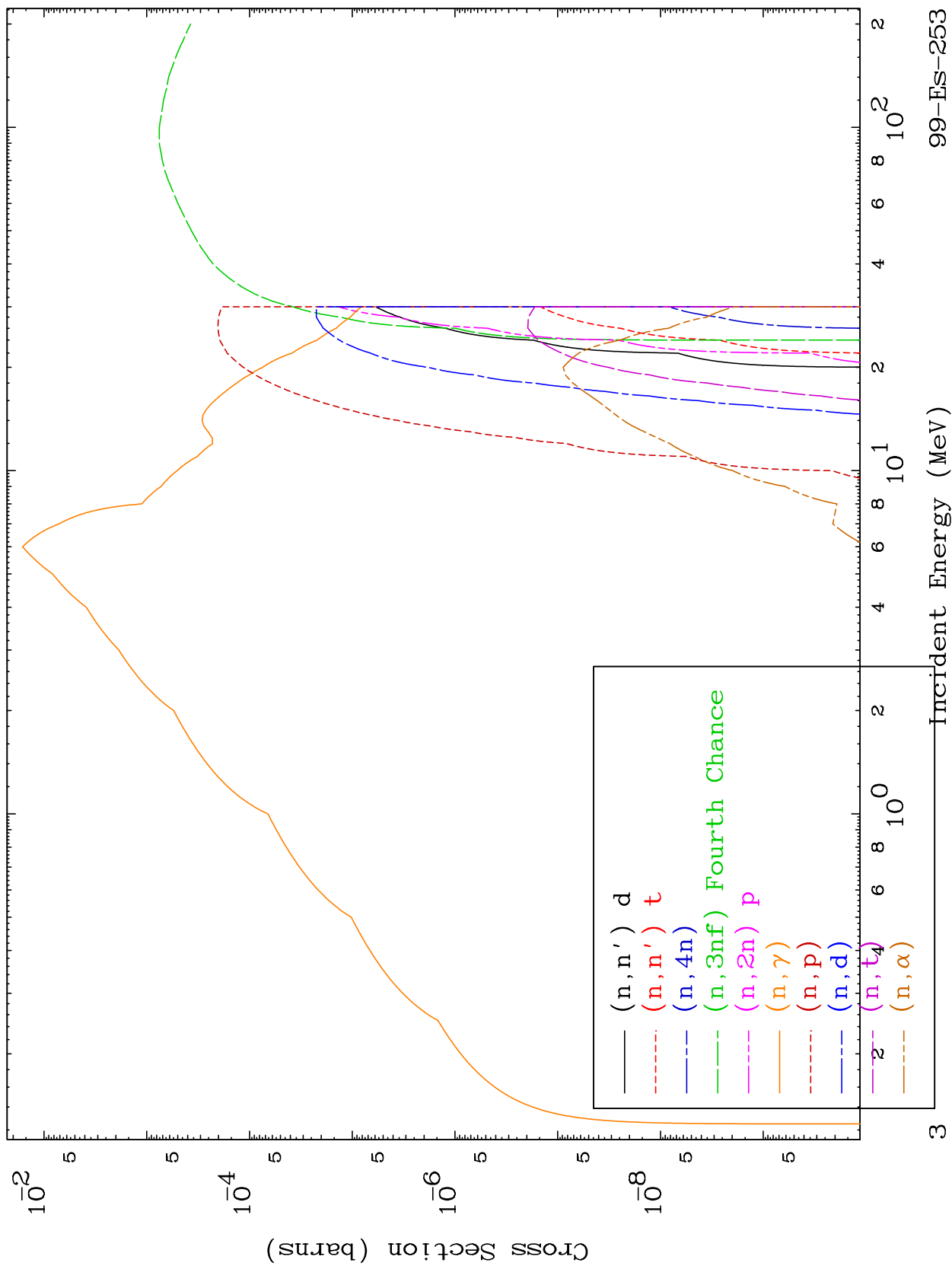
Incident Energy (MeV)

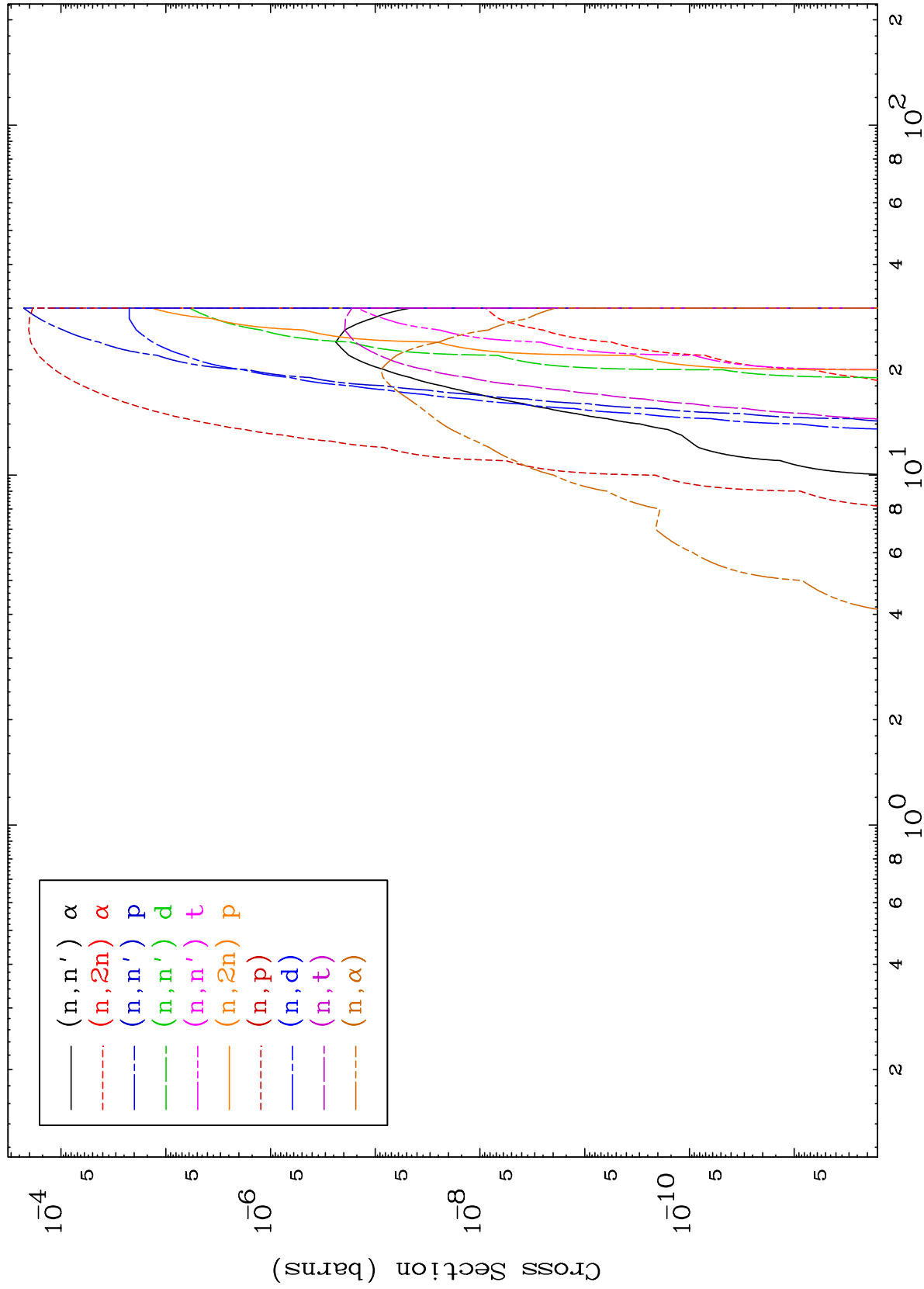
99-Es-253

MAT 9913

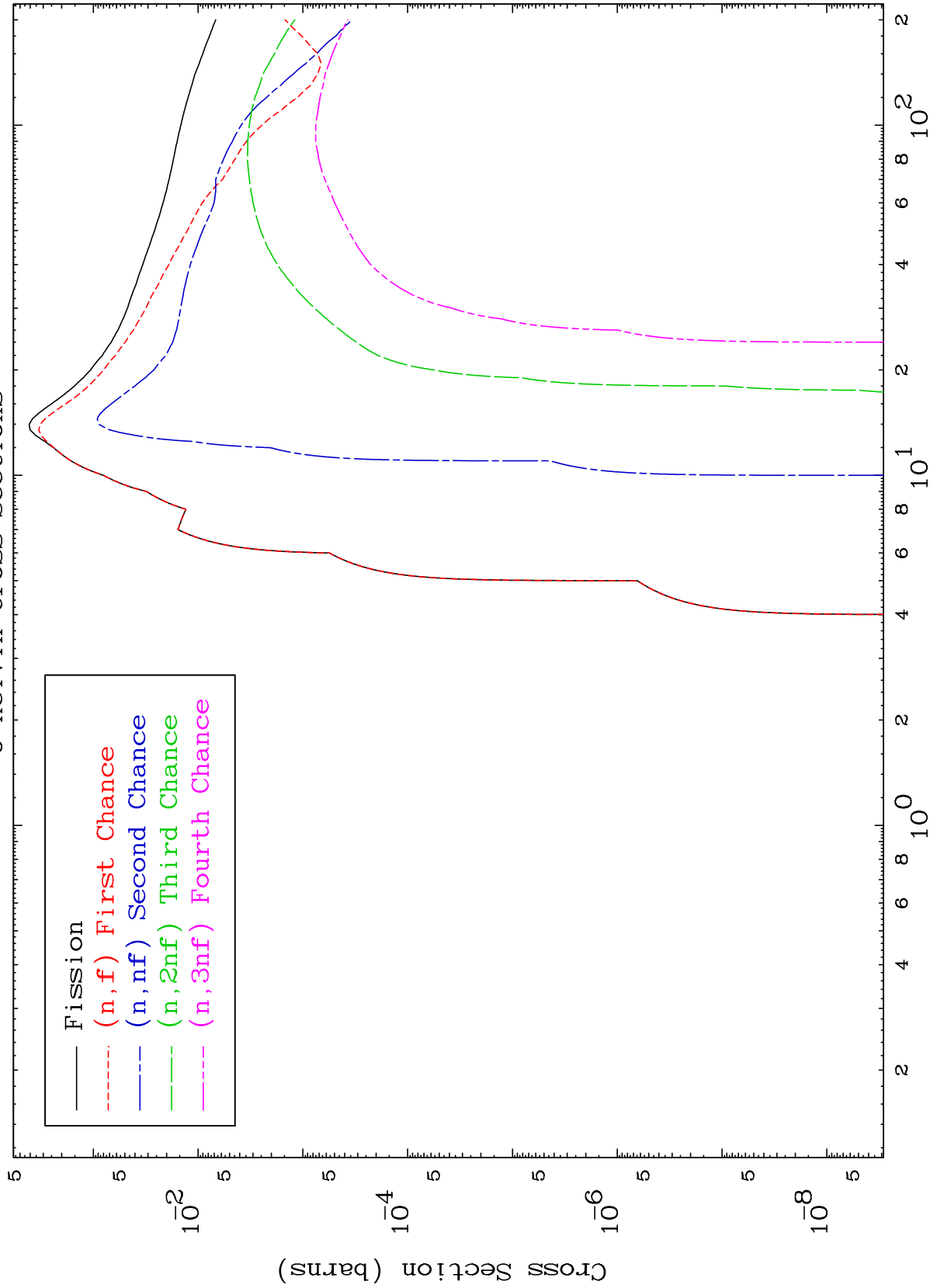
Photon Neutron Absorption
0 Kelvin Cross Sections

99-Es-253





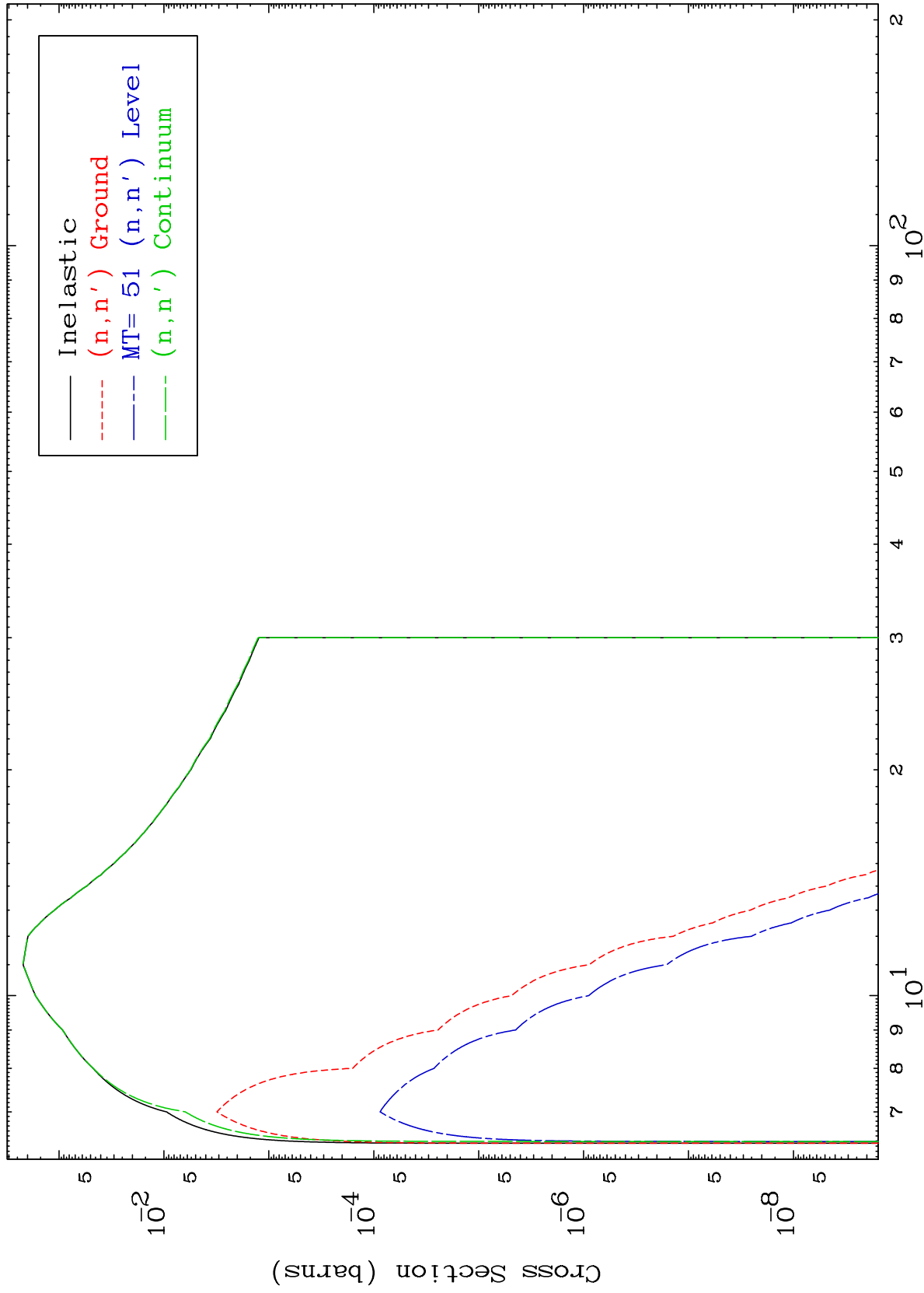
0 Kelvin Cross Sections



MAT 9913

99-Es-253

(γ, n') Levels
0 Kelvin Cross Sections



6

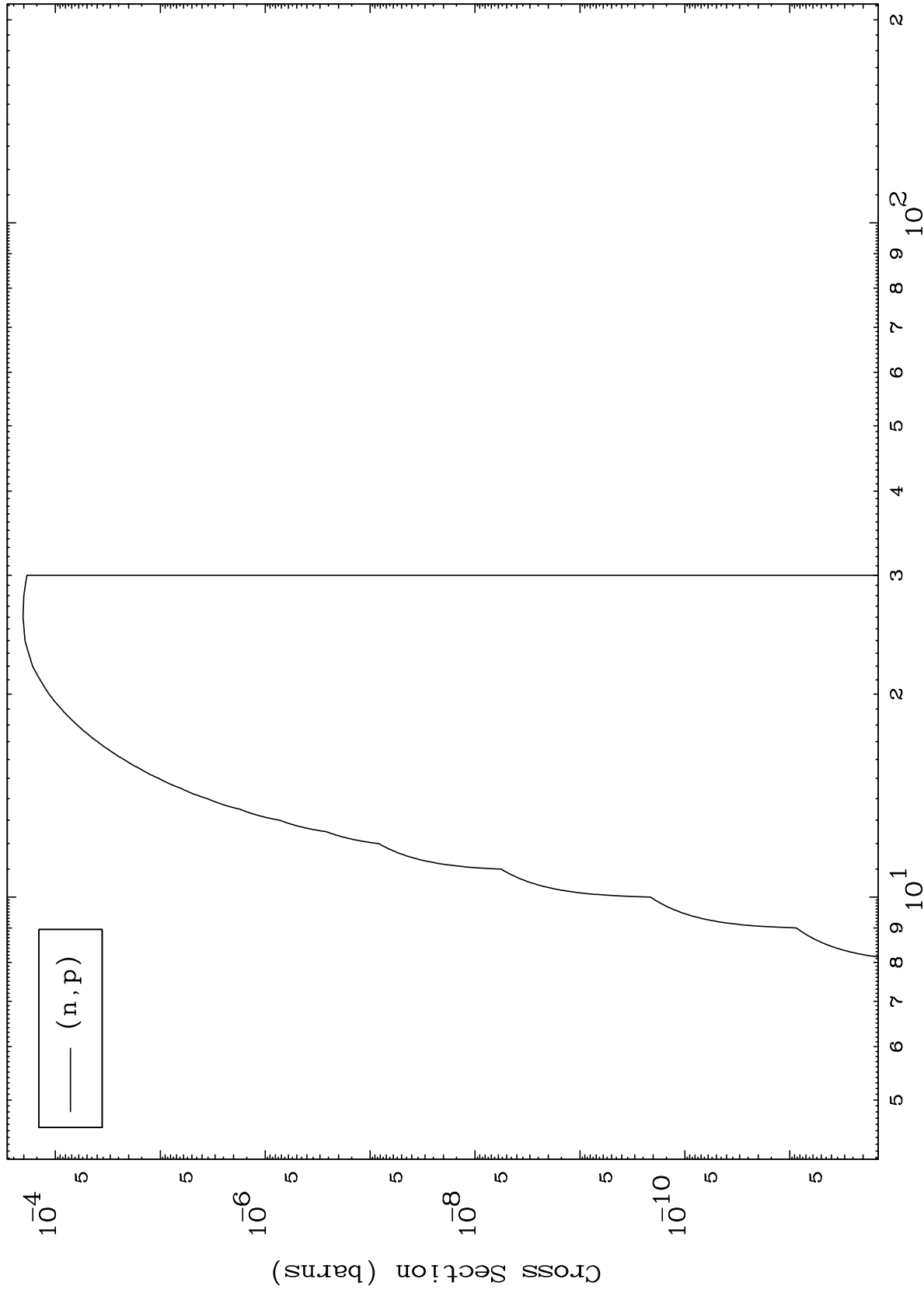
Incident Energy (MeV)

99-Es-253

MAT 9913

99-Es-253

(γ ,p) Levels
0 Kelvin Cross Sections



7

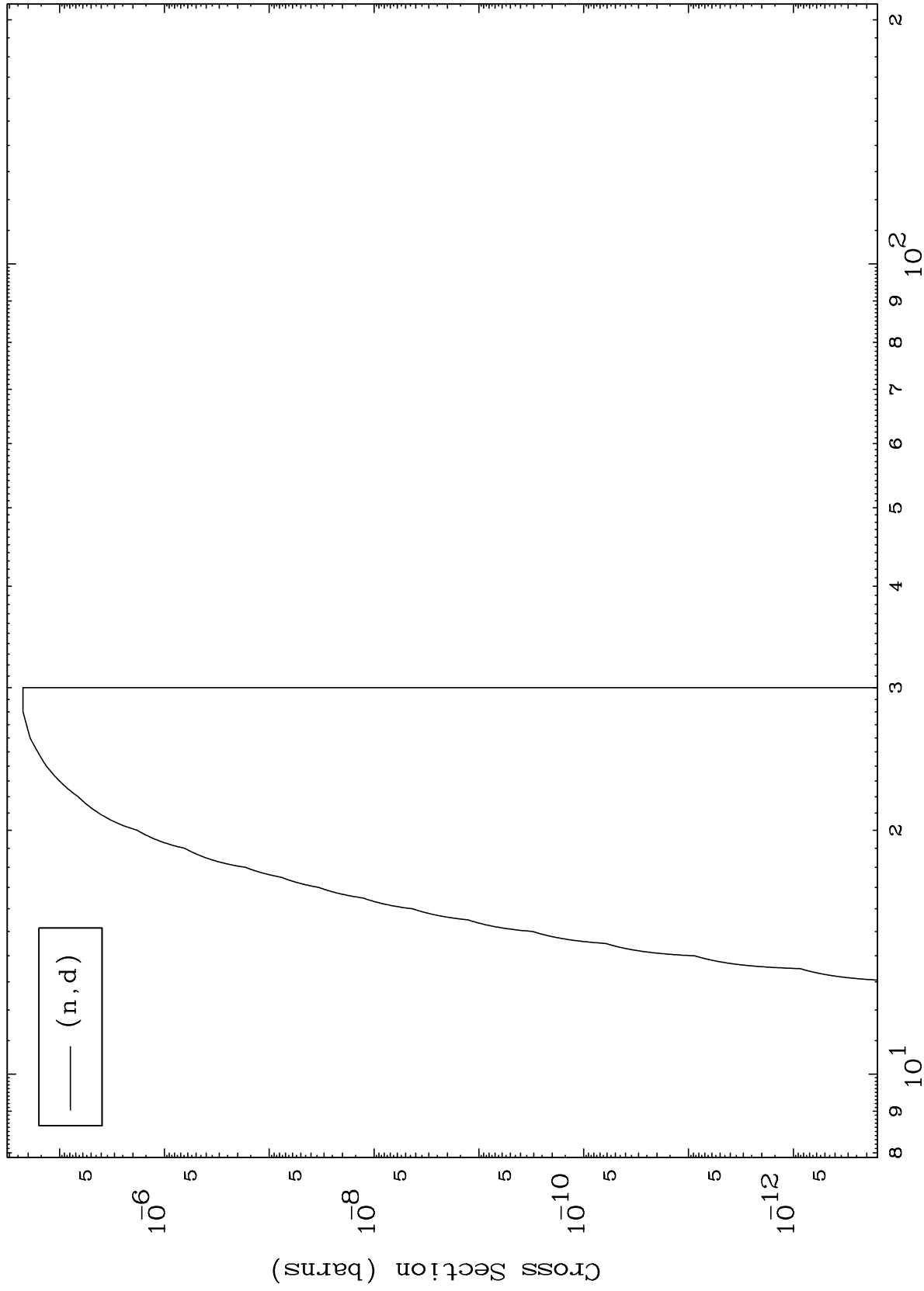
Incident Energy (MeV)

99-Es-253

MAT 9913

99-Es-253

(γ, d) Levels
0 Kelvin Cross Sections



8

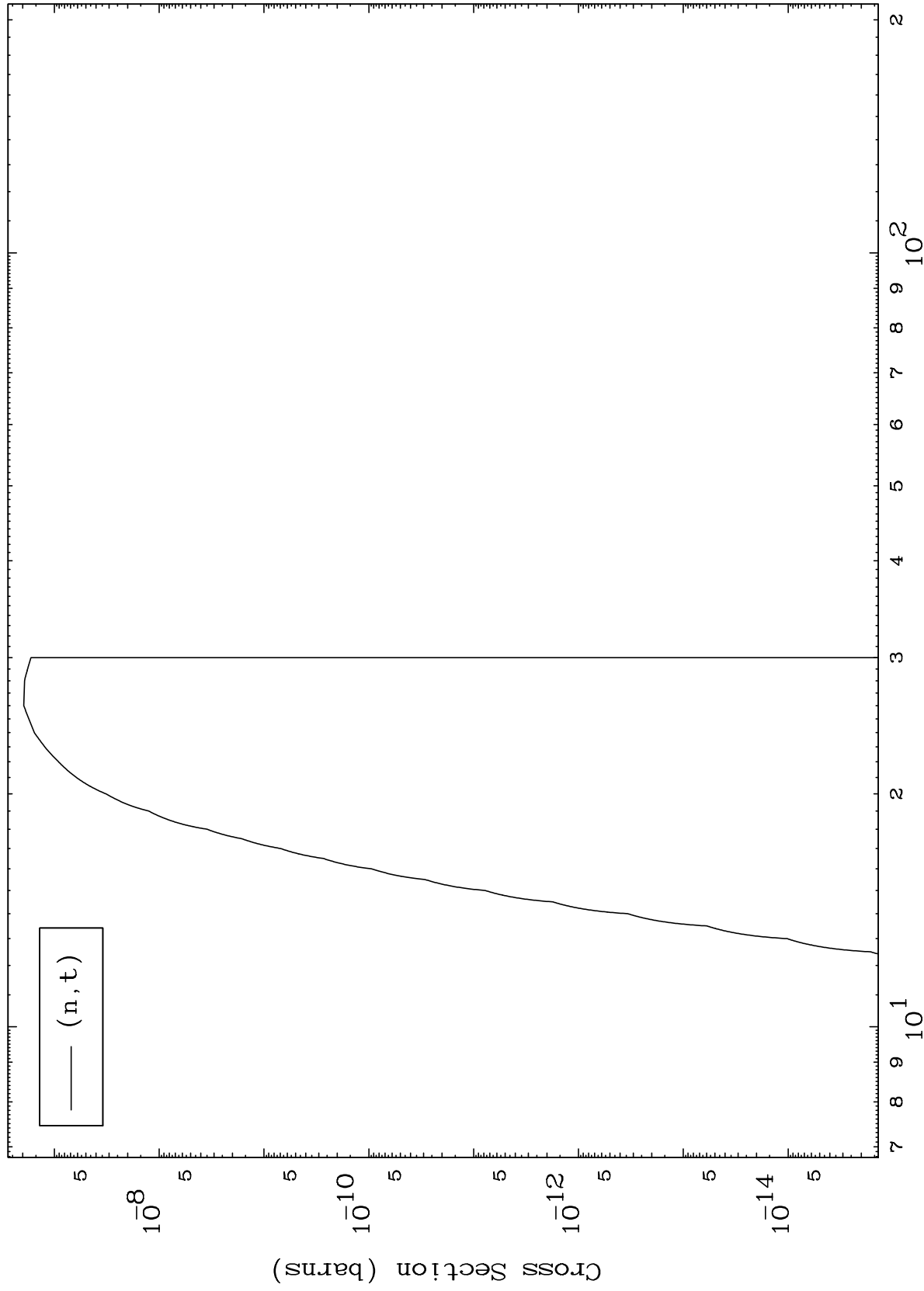
Incident Energy (MeV)

99-Es-253

MAT 9913

99-Es-253

(γ, t) Levels
0 Kelvin Cross Sections

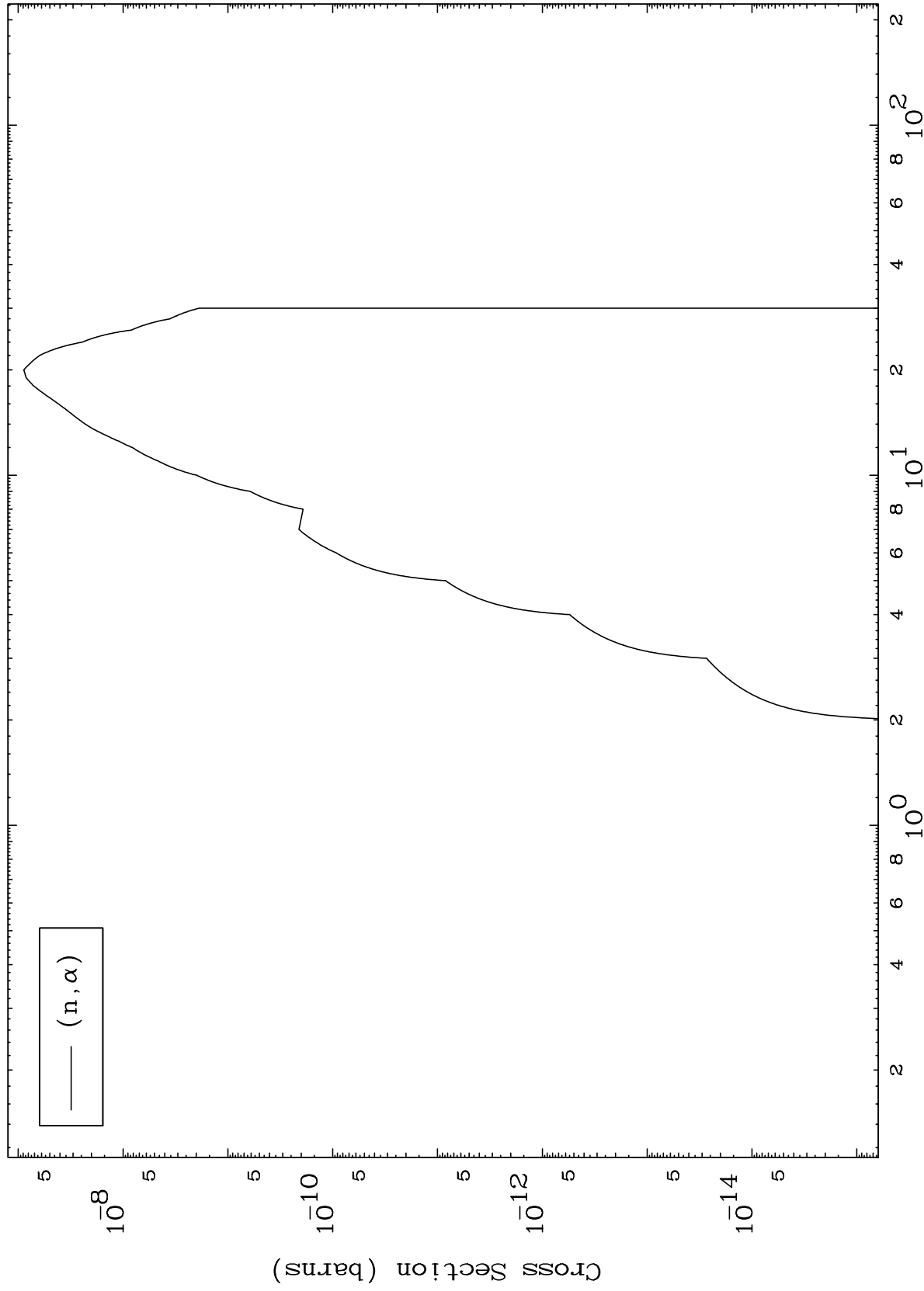


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(γ, α) Levels

99-Es-253

0 Kelvin Cross Sections



10

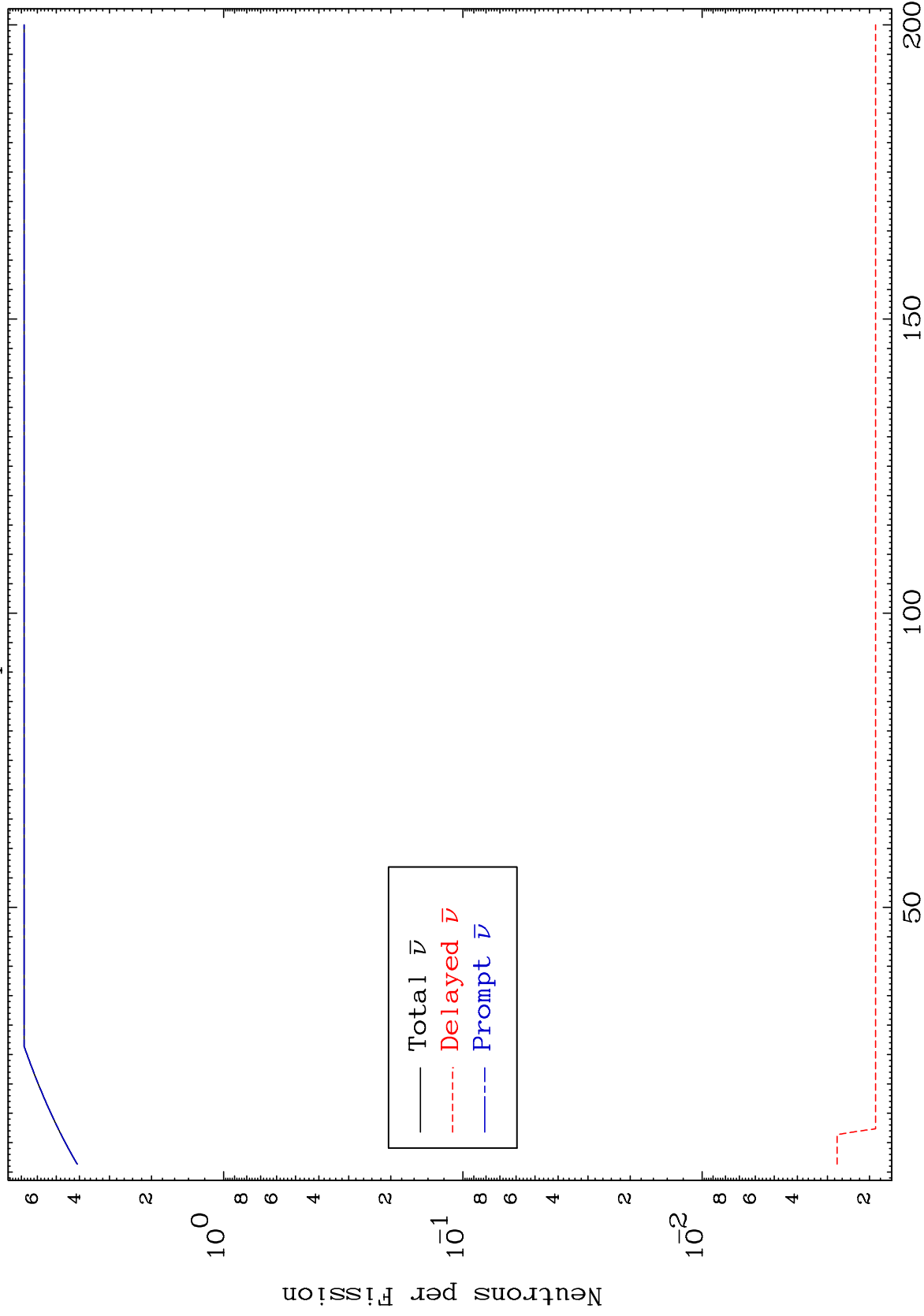
Incident Energy (MeV)

99-Es-253

MAT 9913

Photon Neutrons
per Fission

99-Es-253



11

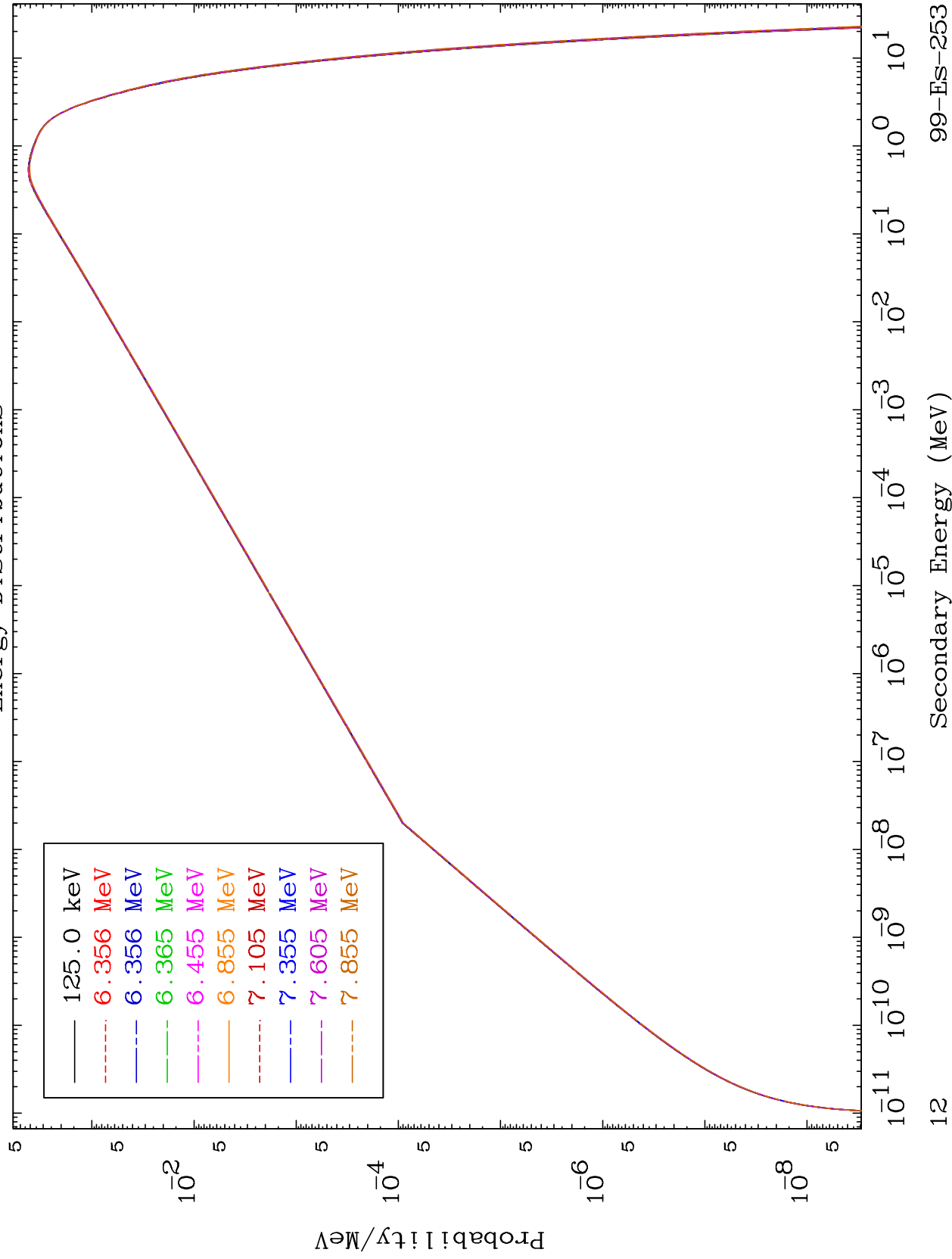
Incident Energy (MeV)

99-Es-253

MAT 9913

Fission
Energy Distributions

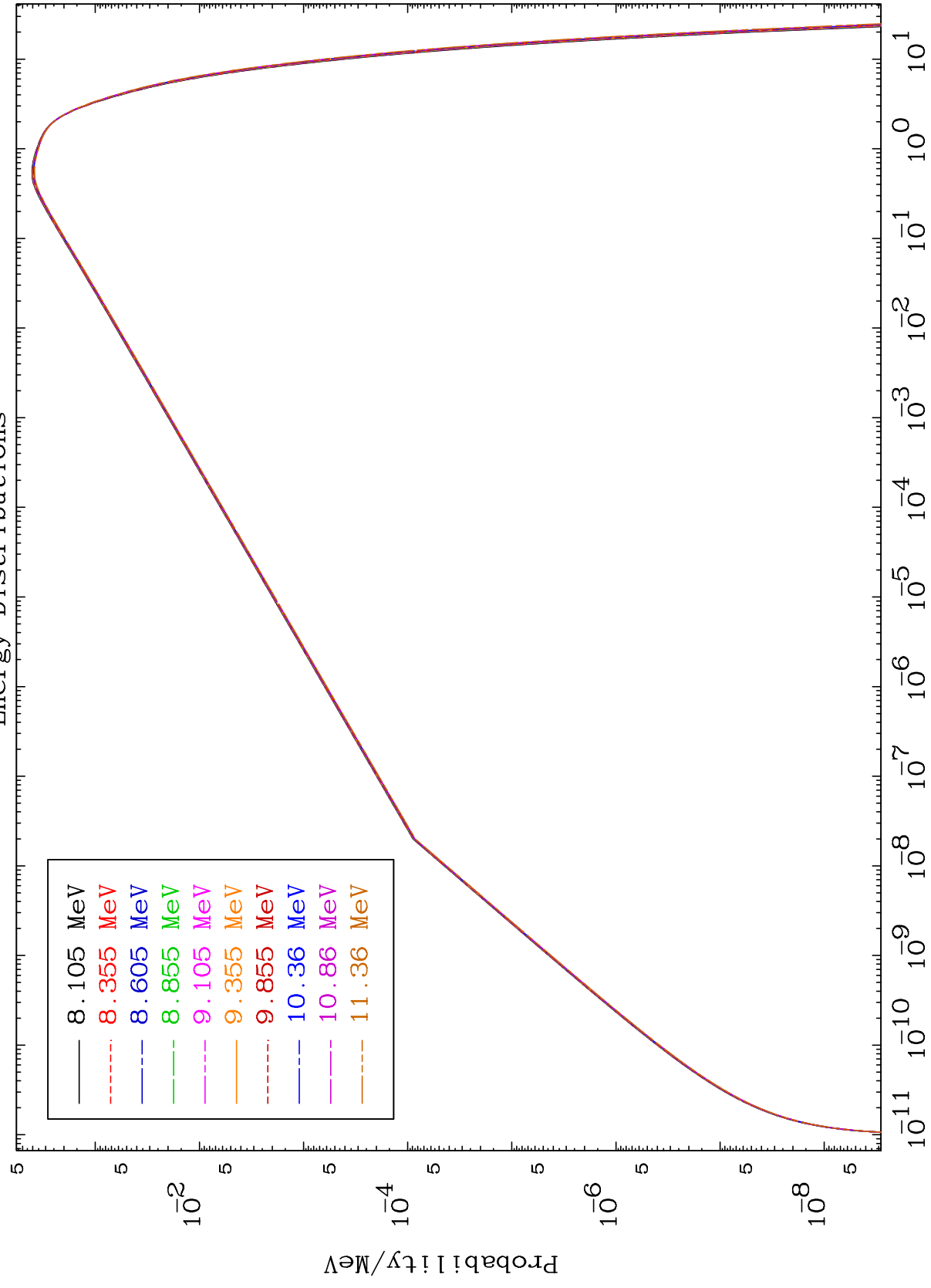
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MAT 9913

Fission
Energy Distributions

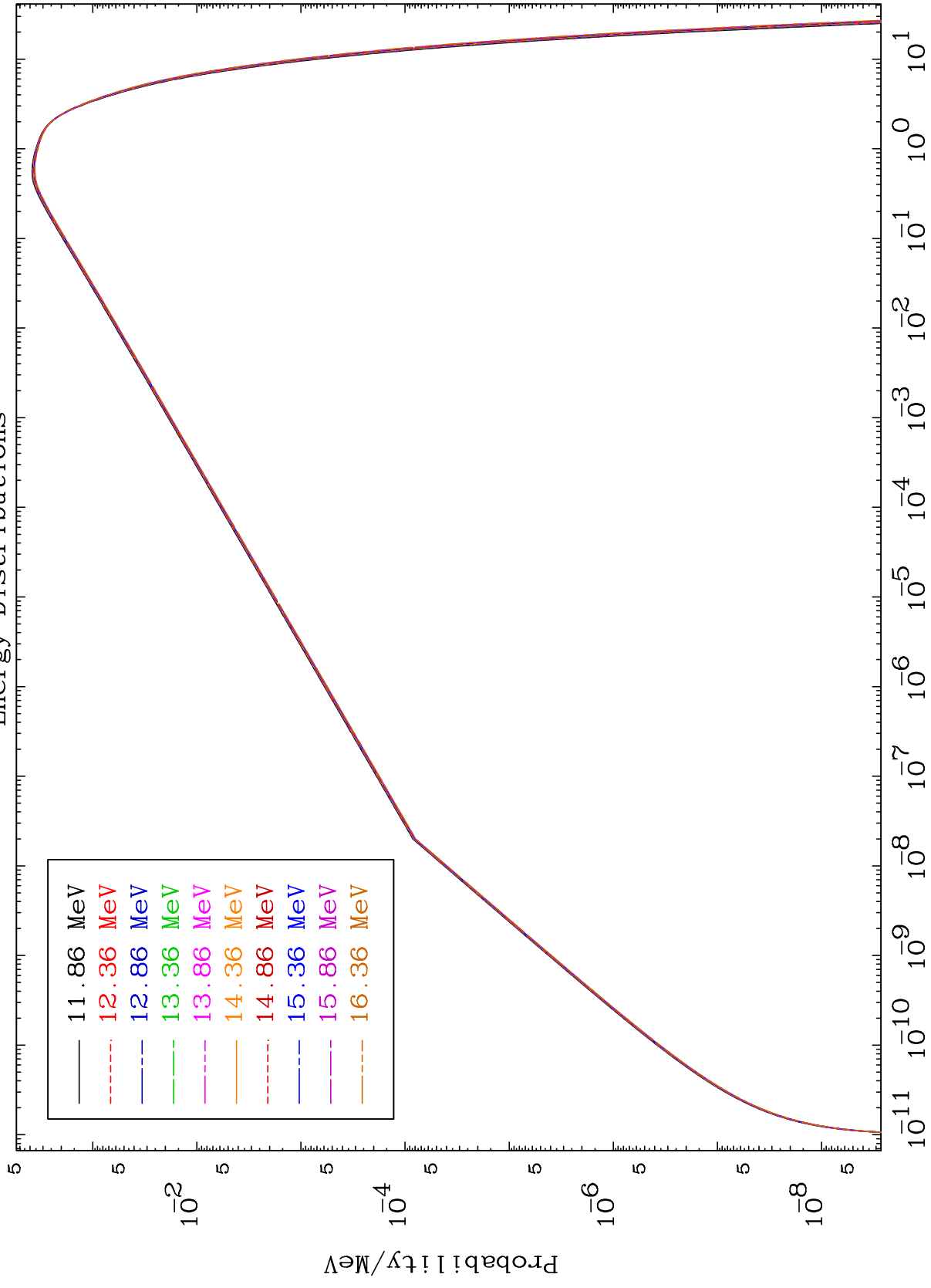
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MAT 9913

Fission
Energy Distributions

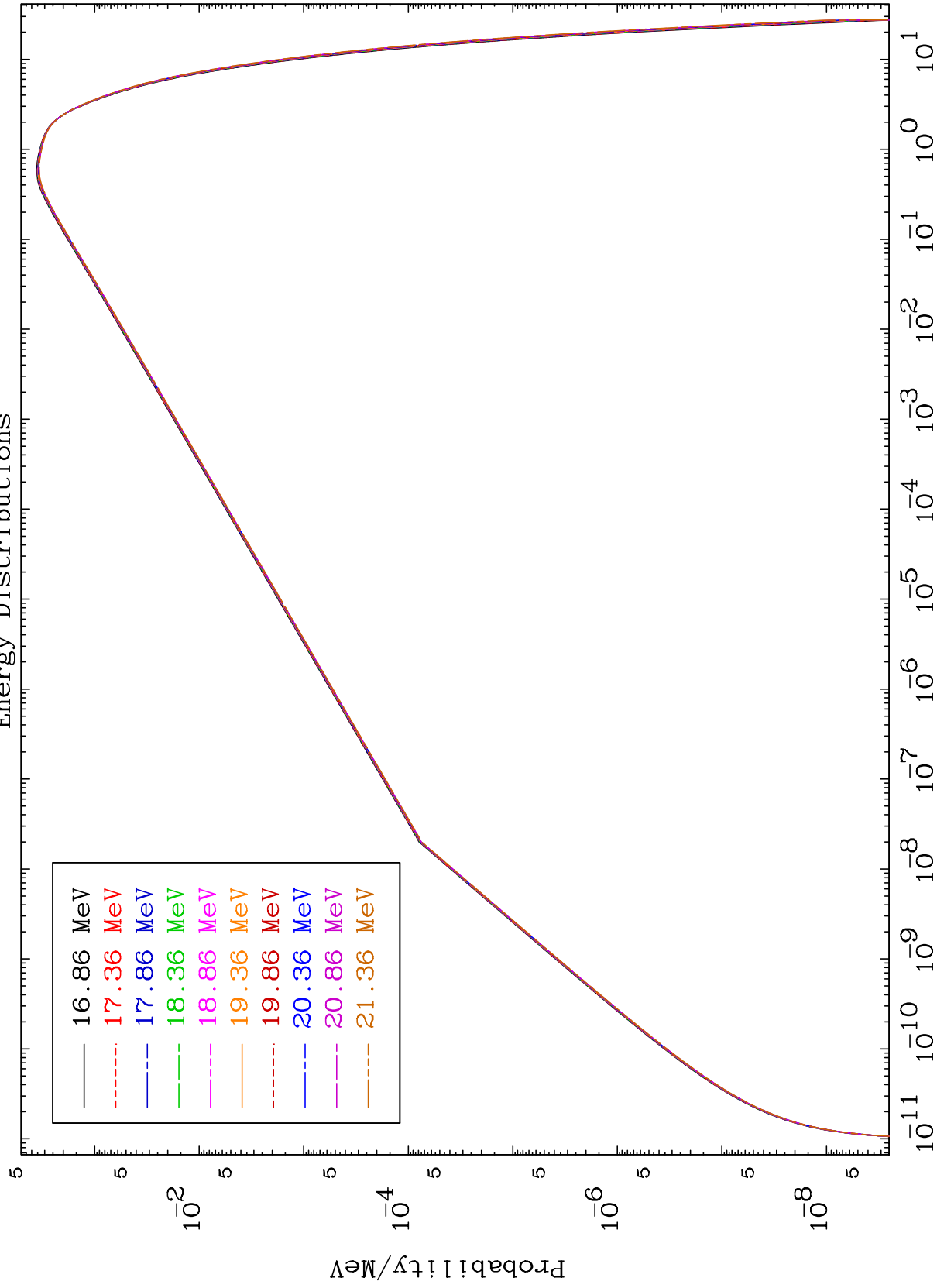
99-Es-253



MAT 9913

Fission
Energy Distributions

99-Es-253



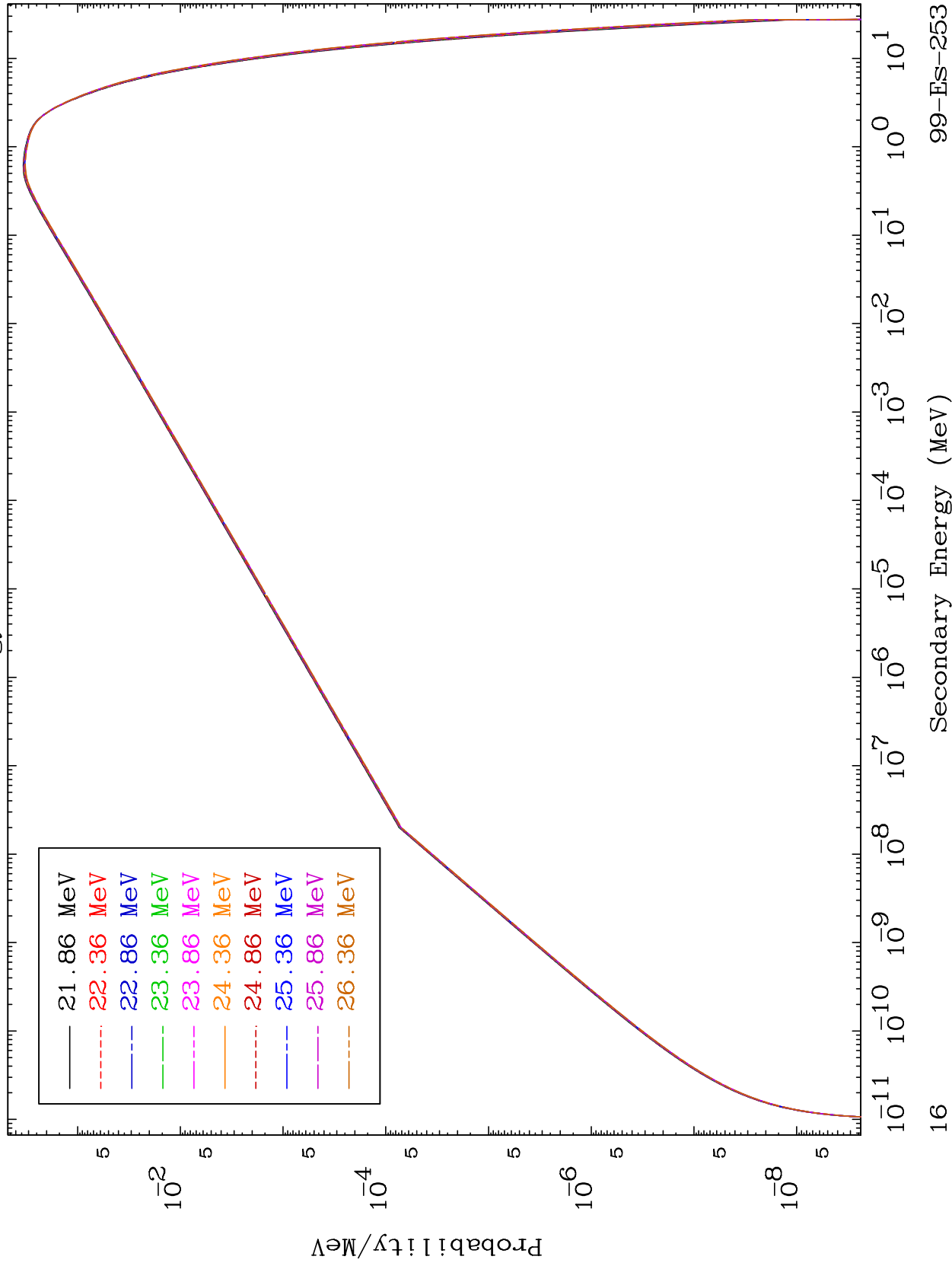
15

99-Es-253

MAT 9913

Fission
Energy Distributions

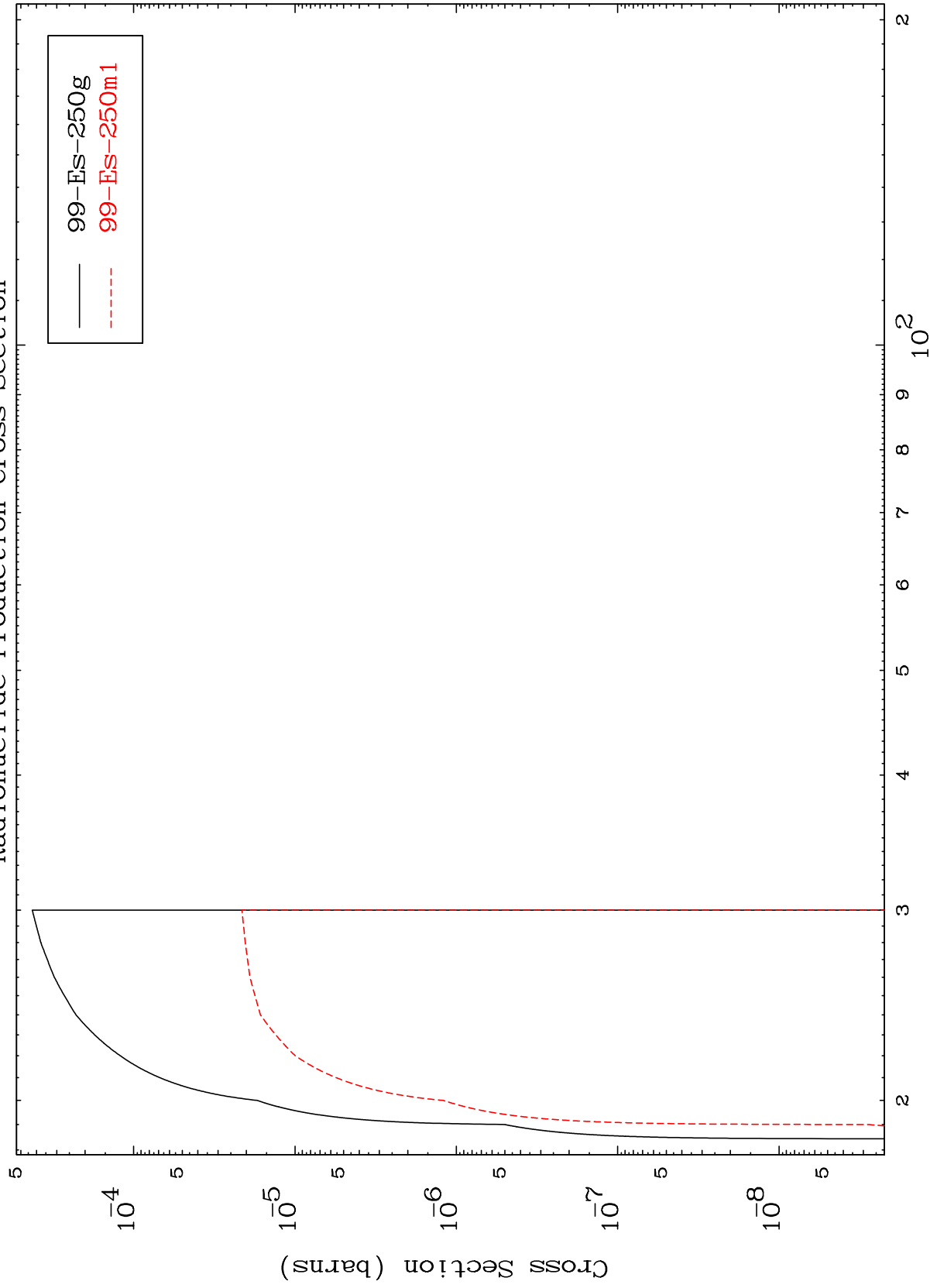
99-Es-253



MAT 9913

99-Es-253

(n,3n)
Radionuclide Production Cross Section



17

Incident Energy (MeV)

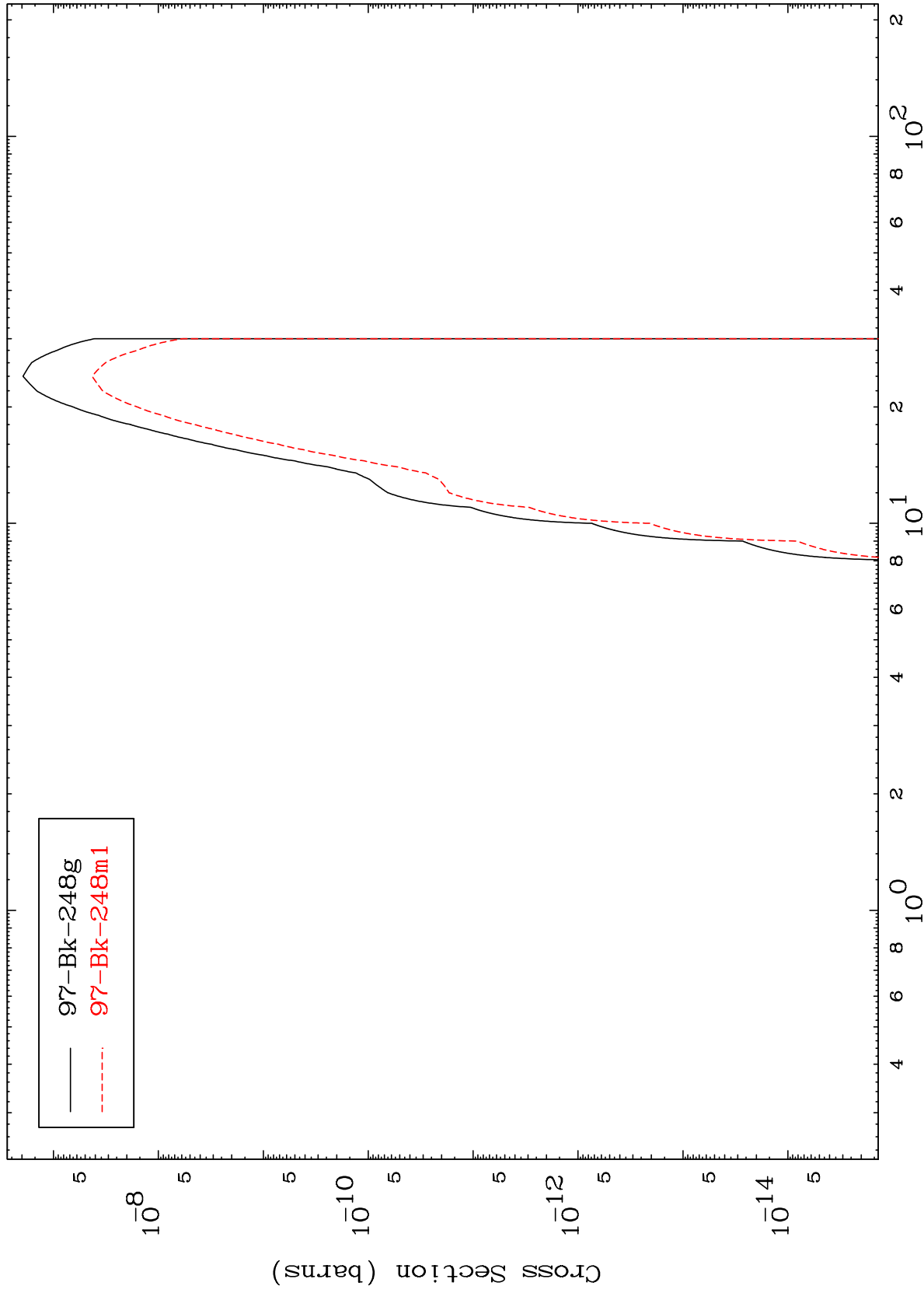
99-Es-253

MAT 9913

(n,n') α

99-Es-253

Radionuclide Production Cross Section



18

Incident Energy (MeV)

99-Es-253