

Program EVALPLOT
(Version 2021-1)

by

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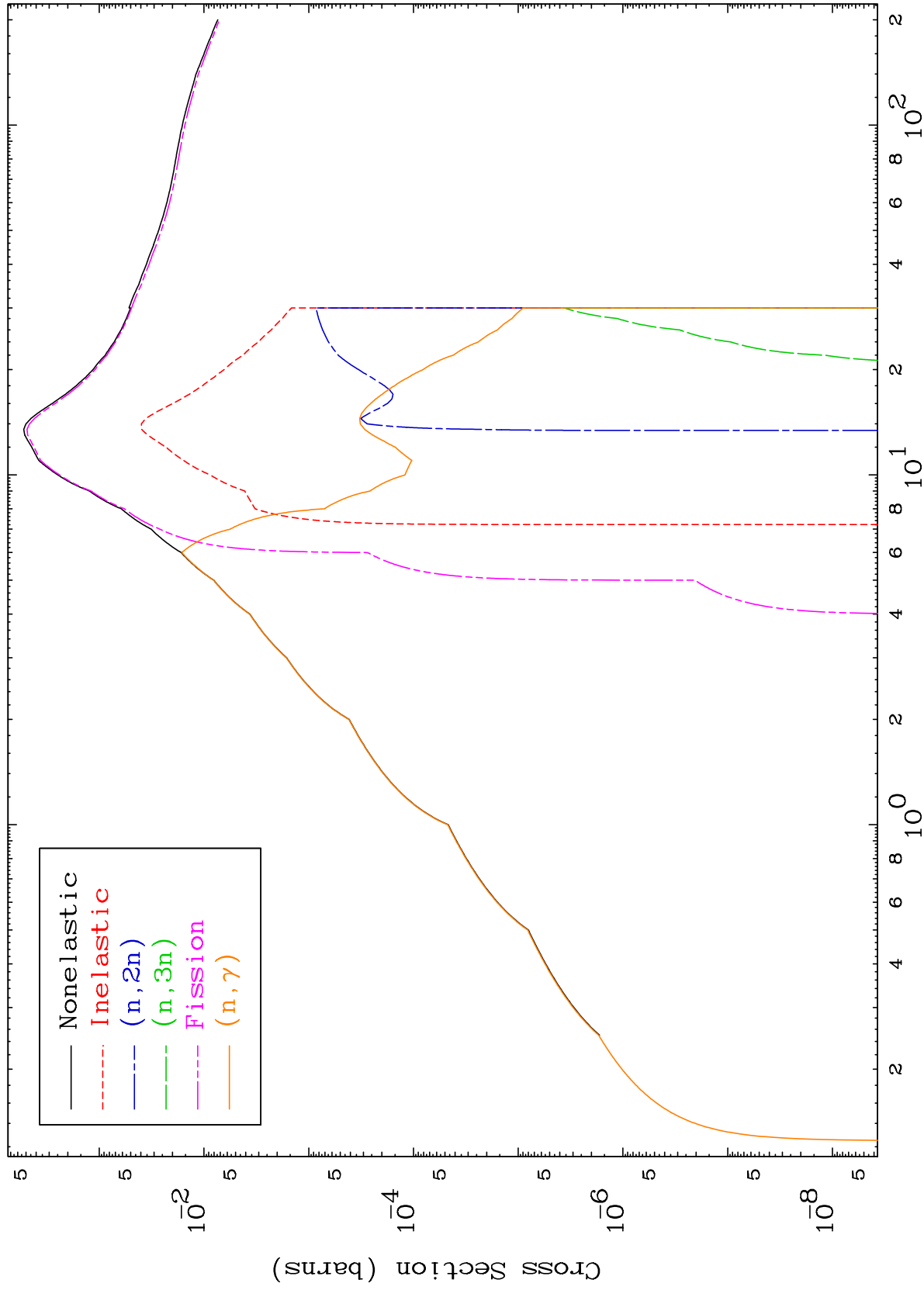
Press Mouse Button to Start

MAT 9933

Photon Major

100-Fm-252

0 Kelvin Cross Sections



1

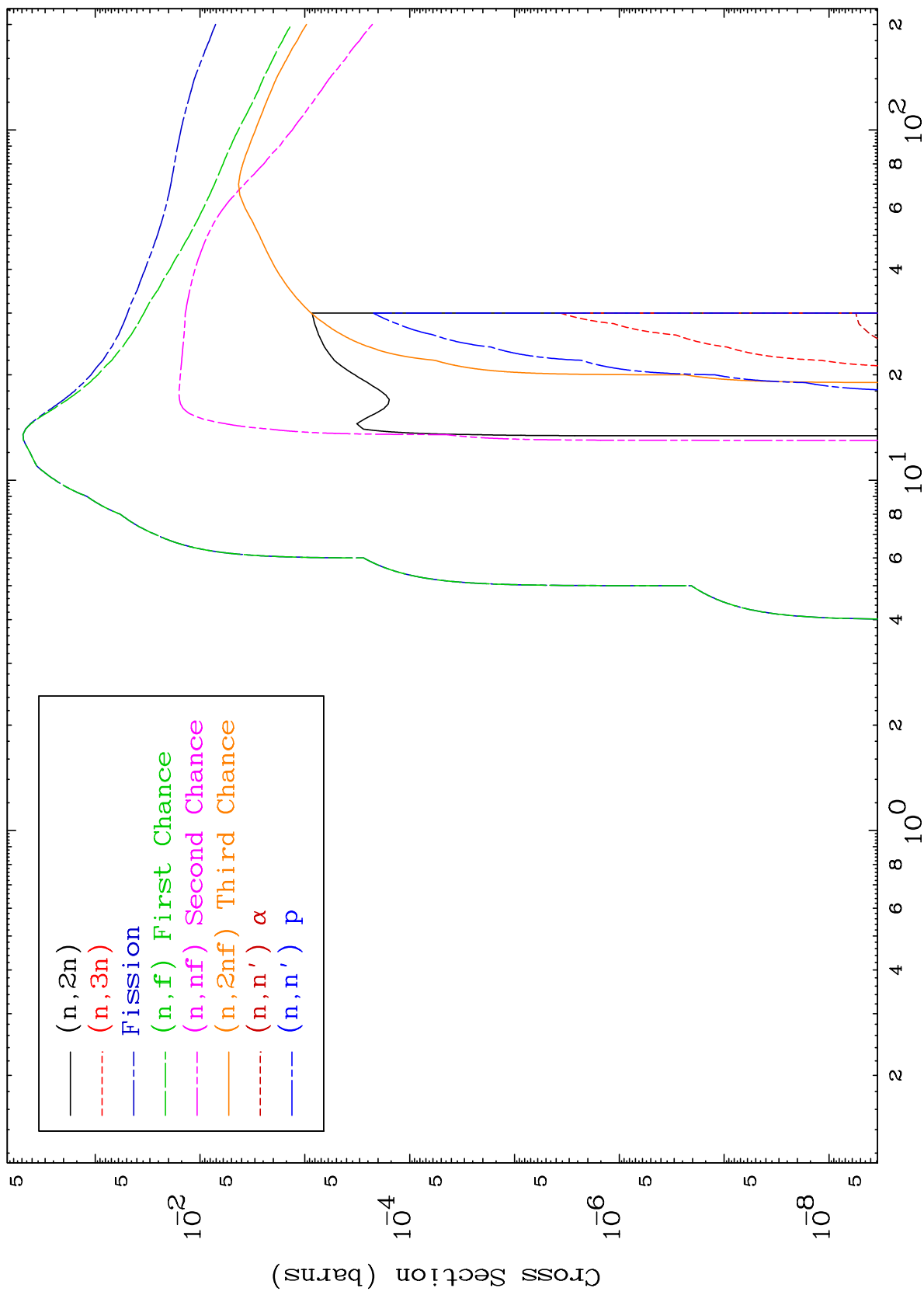
Incident Energy (MeV)

100-Fm-252

MAT 9933

Photon Neutron Absorption
0 Kelvin Cross Sections

100-Fm-252



2

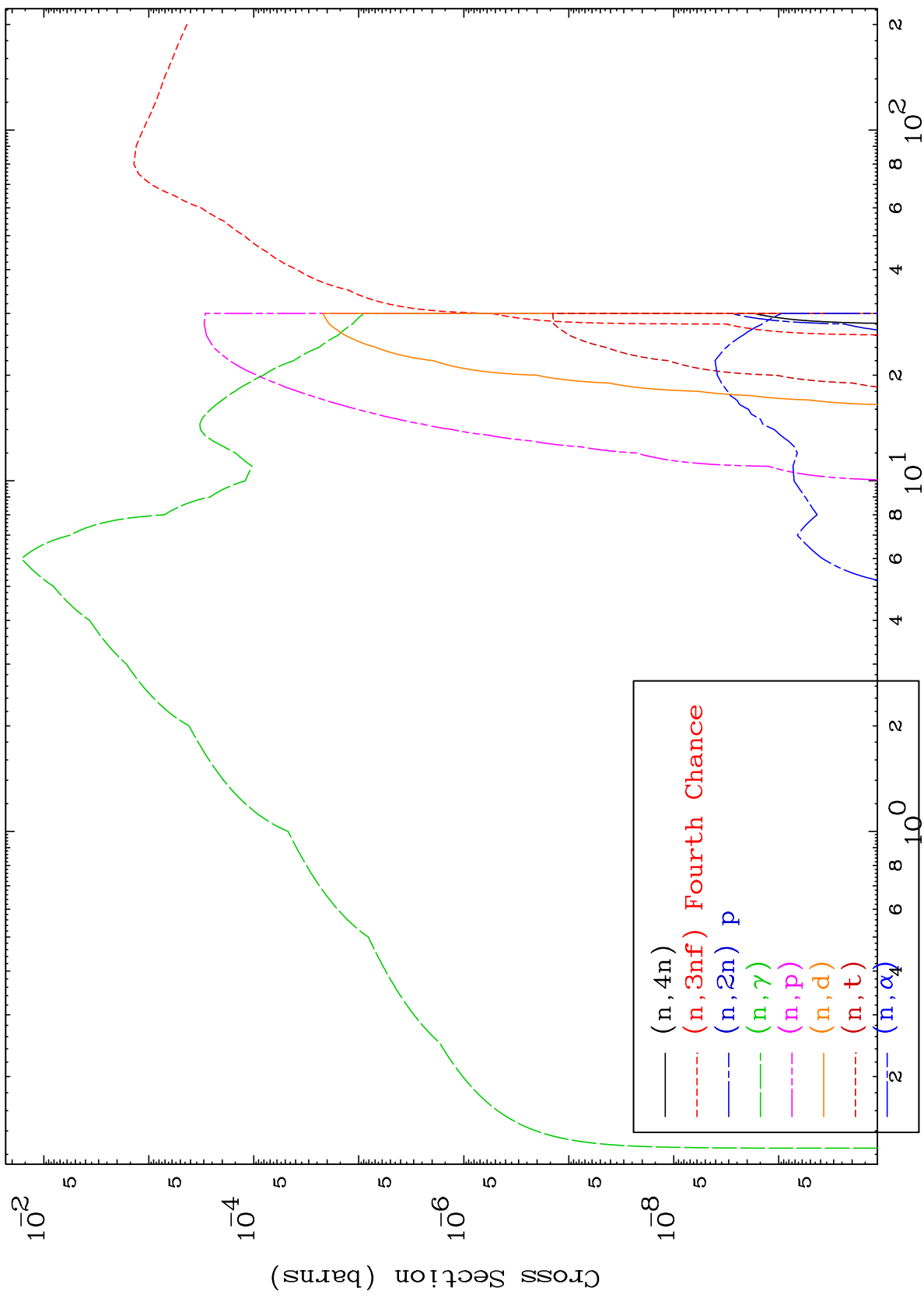
Incident Energy (MeV)

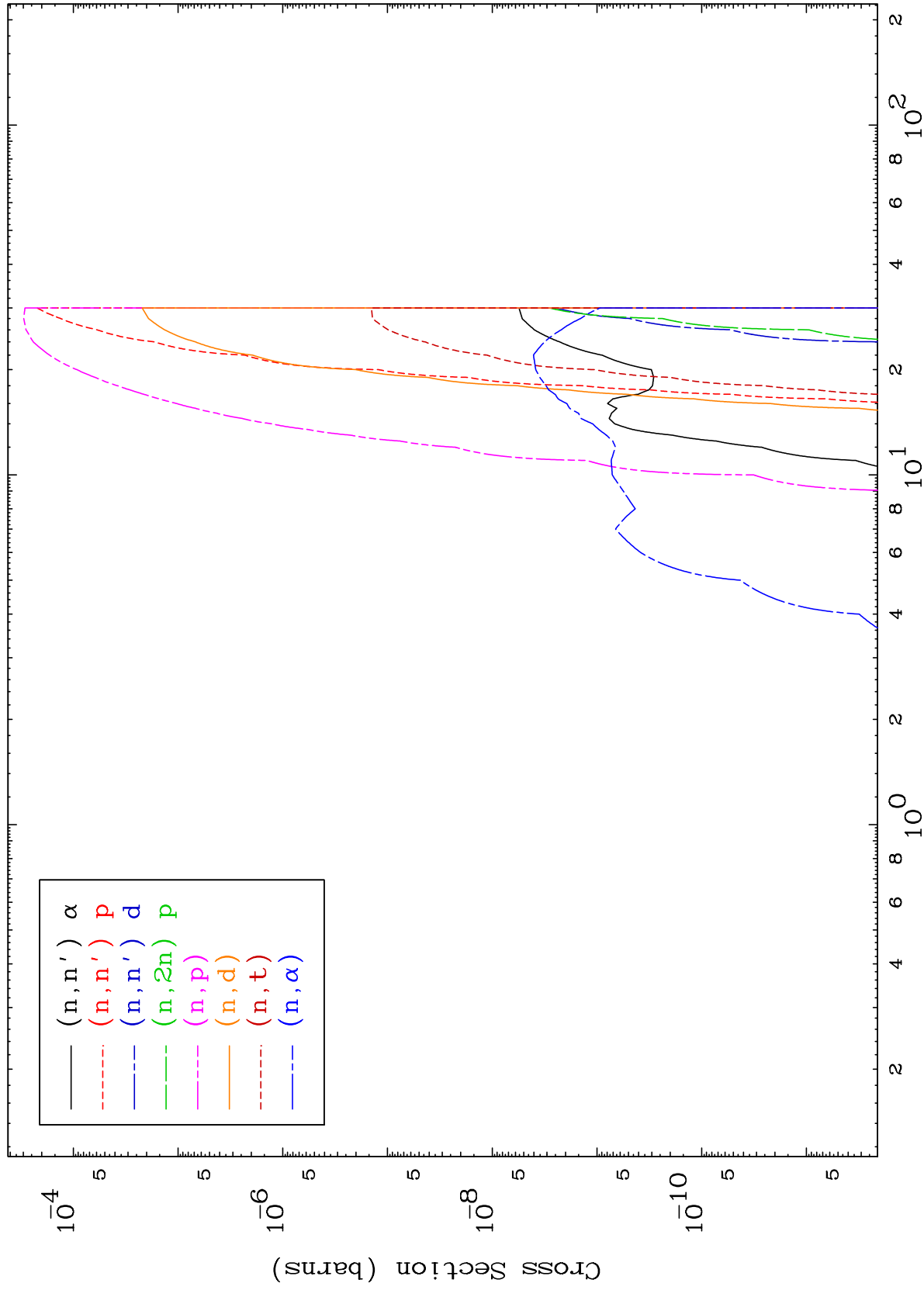
100-Fm-252

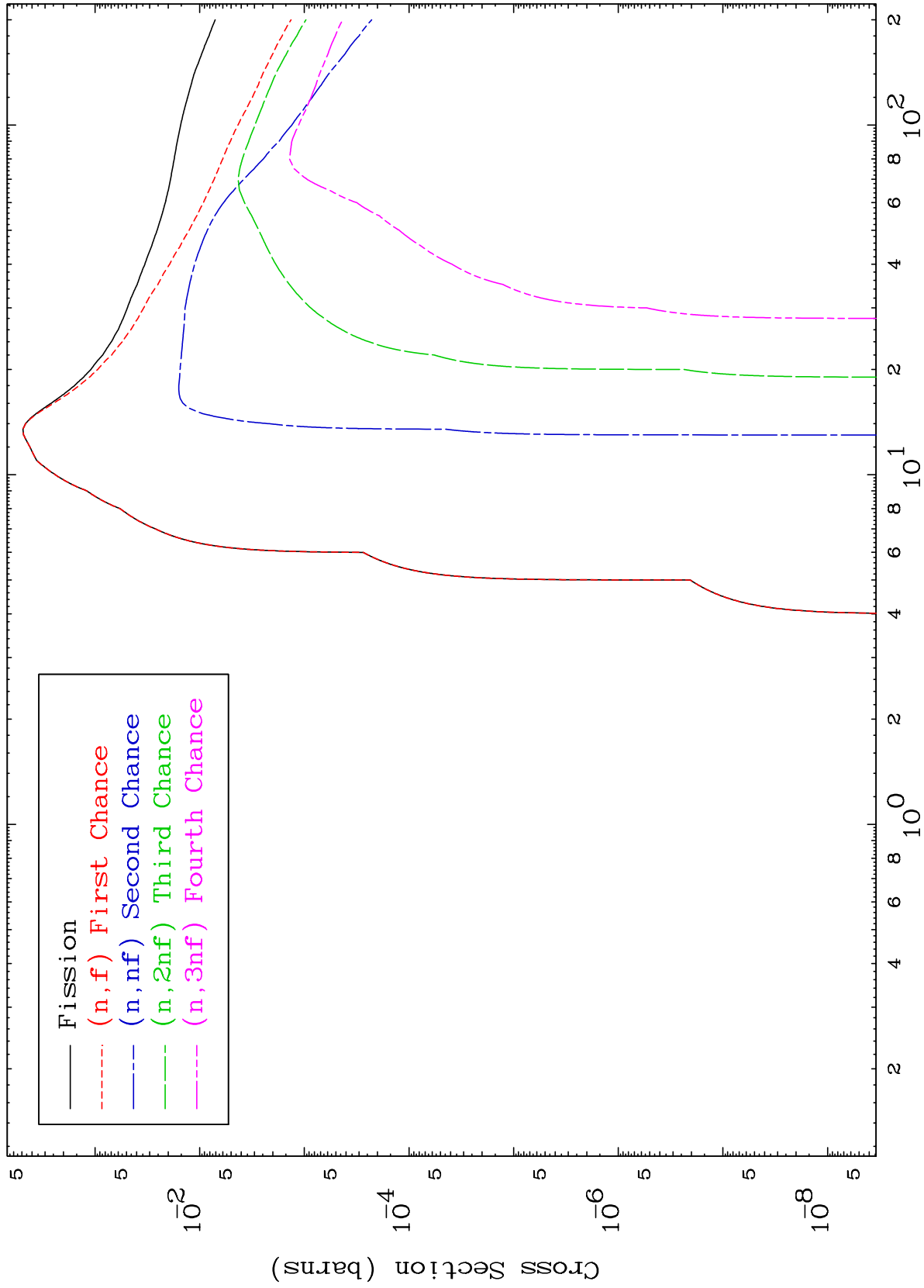
MAT 9933

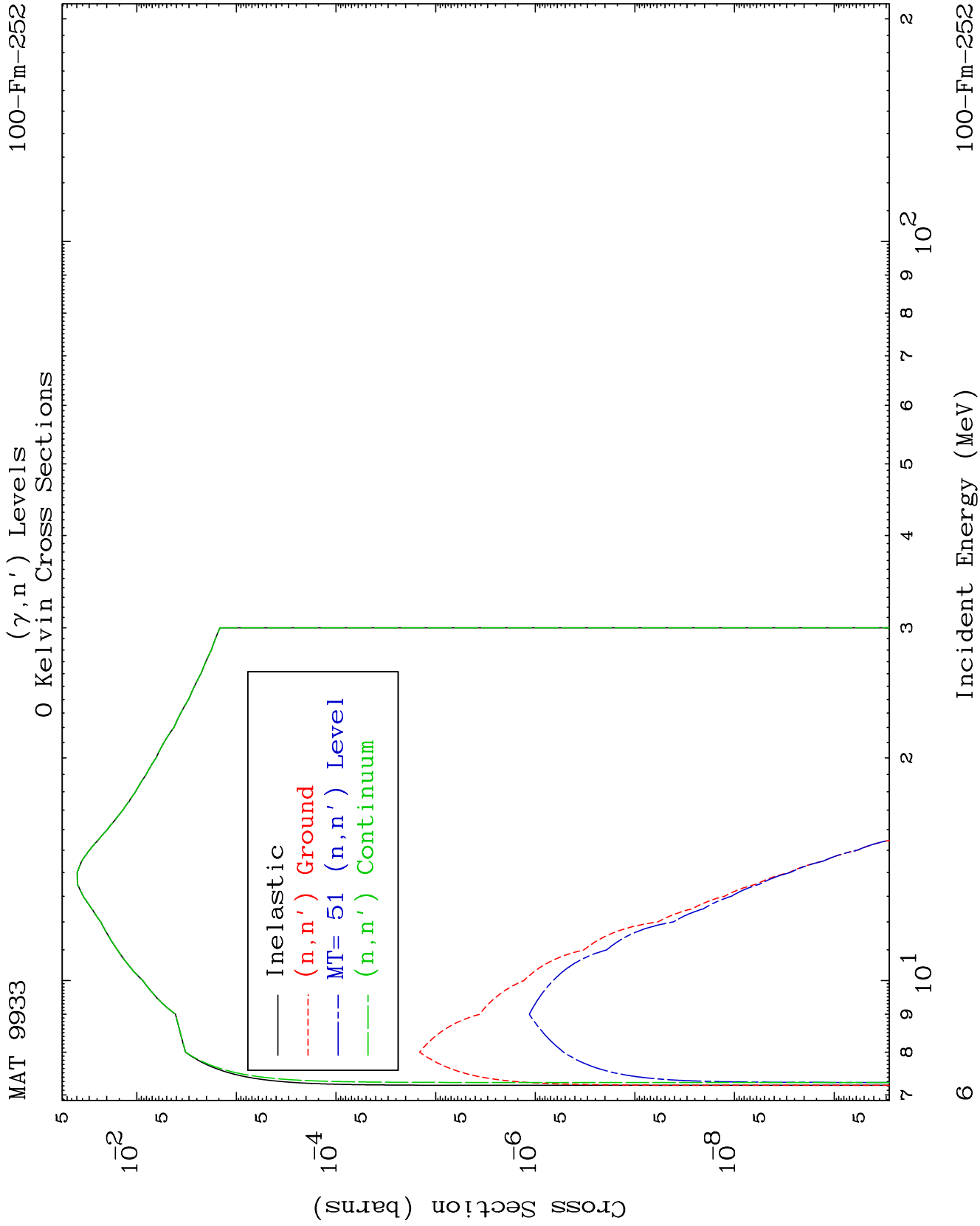
Photon Neutron Absorption
0 Kelvin Cross Sections

100-Fm-252





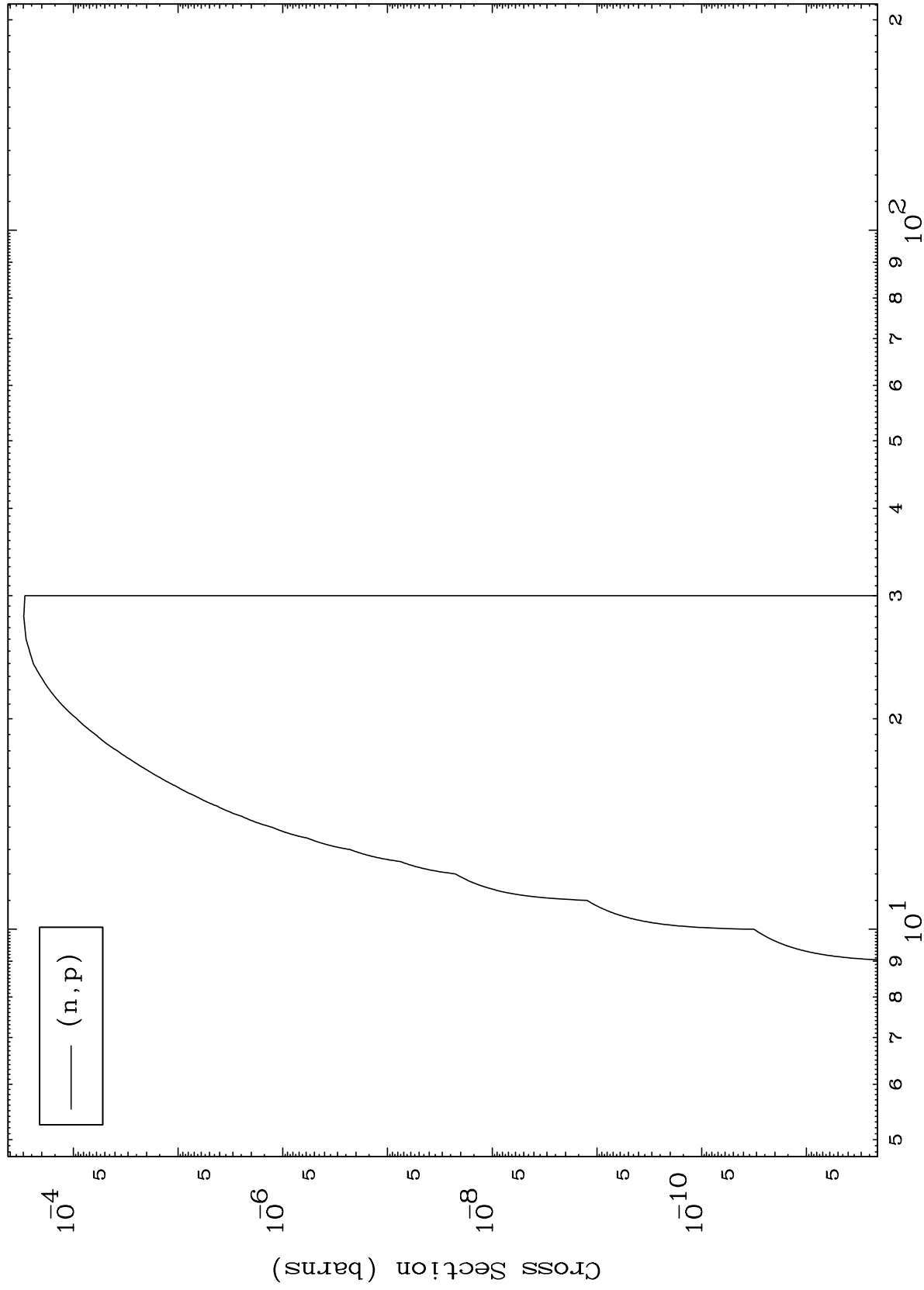




MAT 9933

100-Fm-252

(γ ,p) Levels
0 Kelvin Cross Sections



7

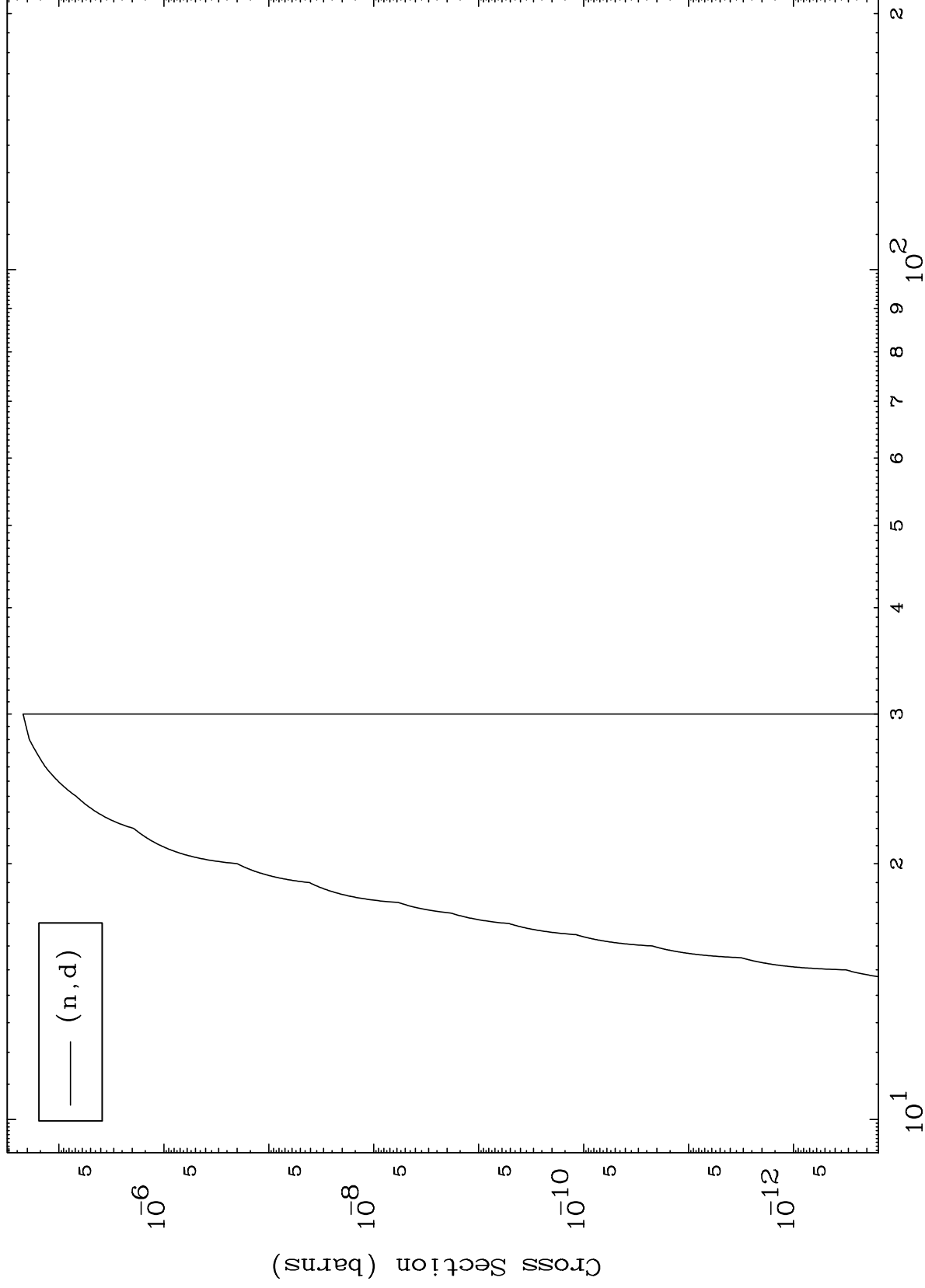
Incident Energy (MeV)

100-Fm-252

MAT 9933

(γ, d) Levels
0 Kelvin Cross Sections

100-Fm-252



8

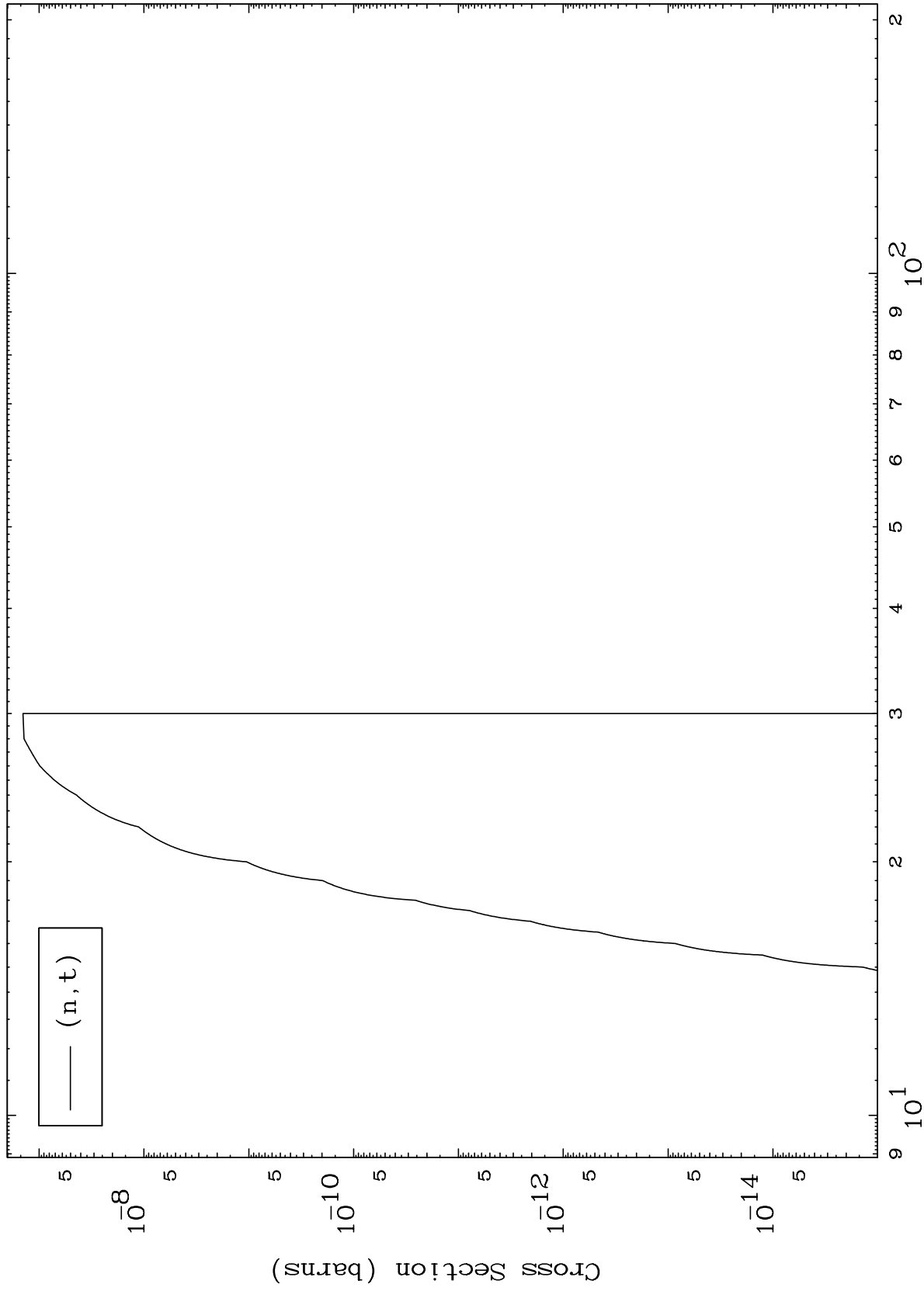
Incident Energy (MeV)

100-Fm-252

MAT 9933

(γ, t) Levels
0 Kelvin Cross Sections

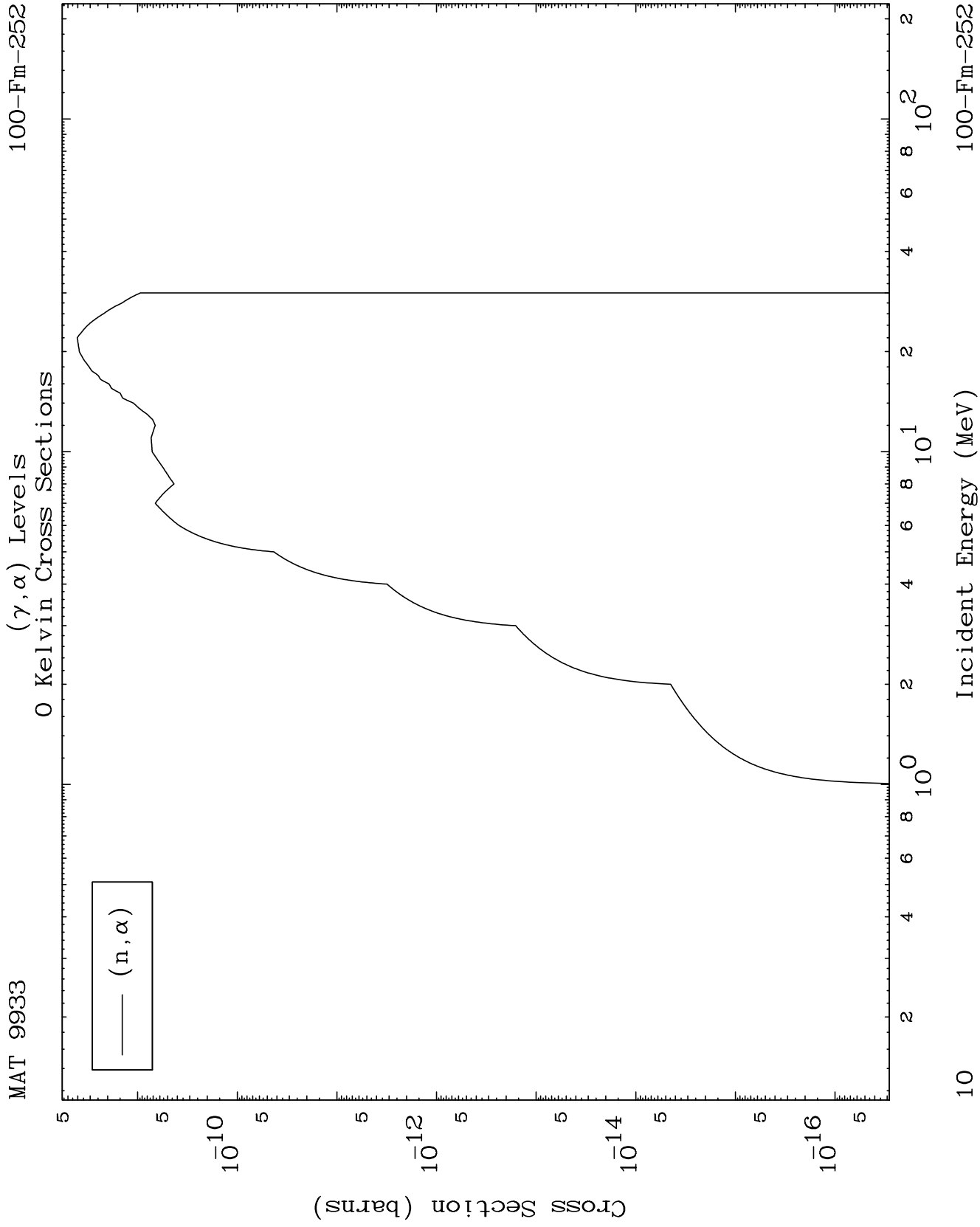
100-Fm-252



9

Incident Energy (MeV)

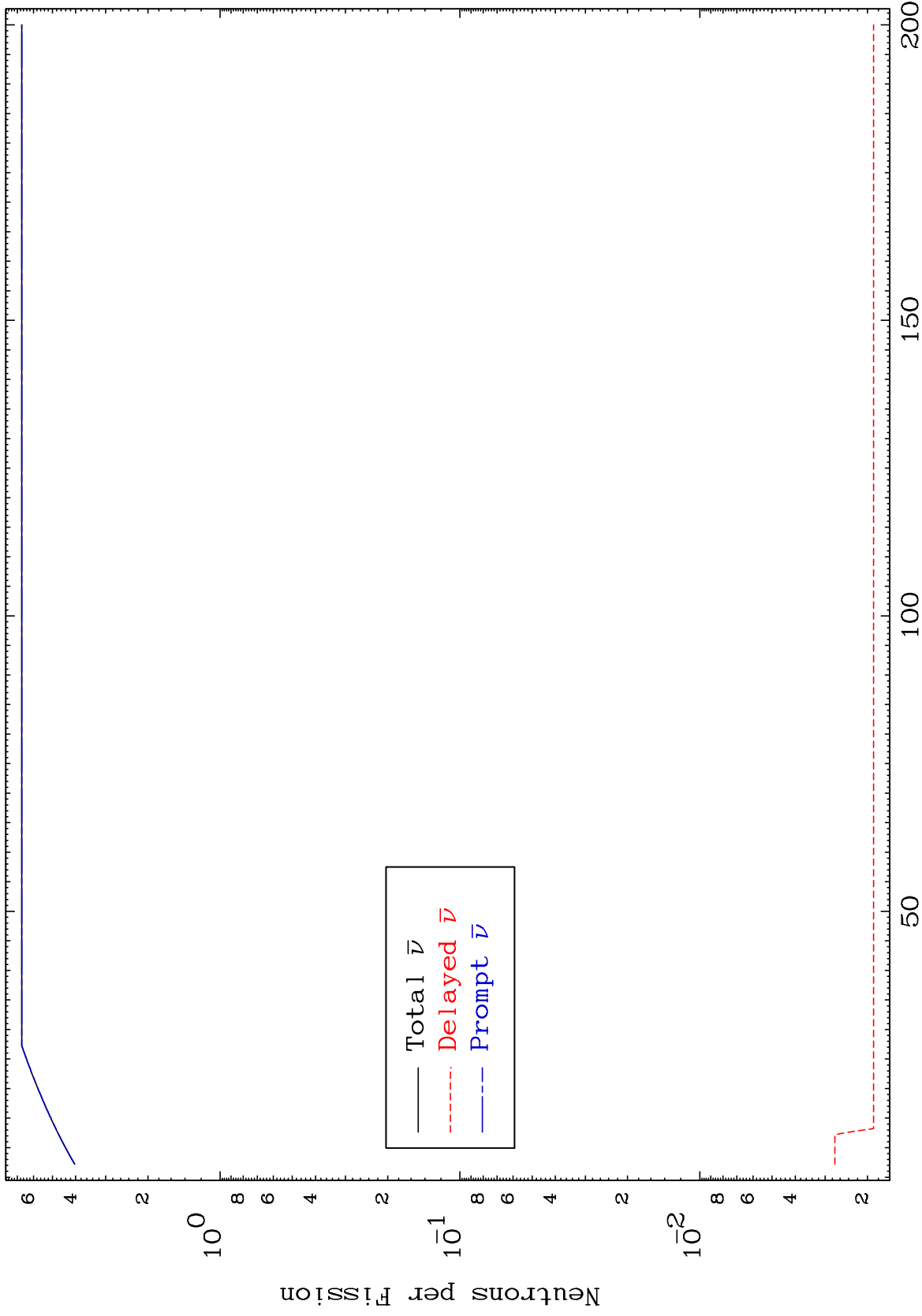
100-Fm-252



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Photon Neutrons
per Fission

100-Fm-252



11

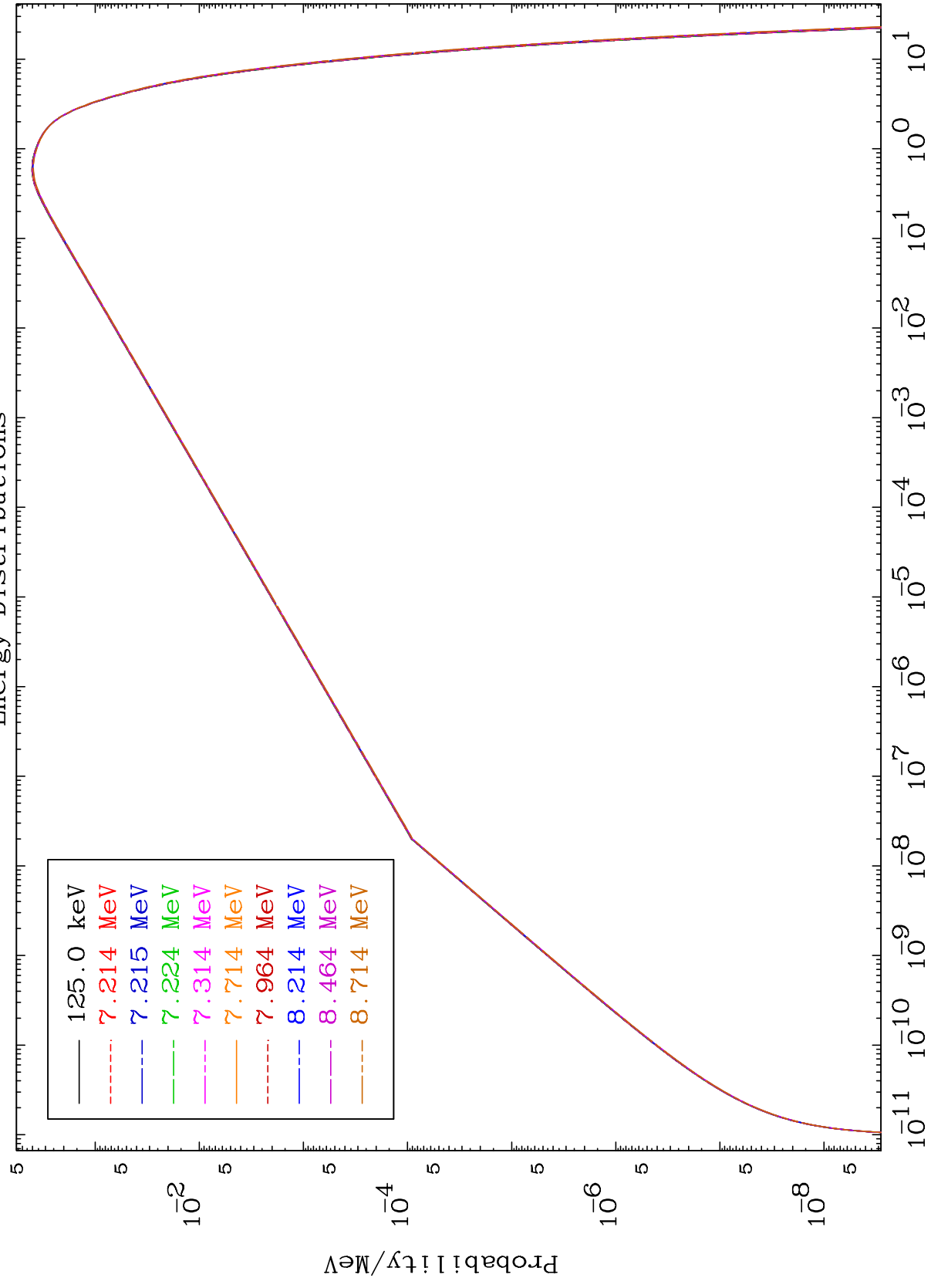
Incident Energy (MeV)

100-Fm-252

MAT 9933

Fission
Energy Distributions

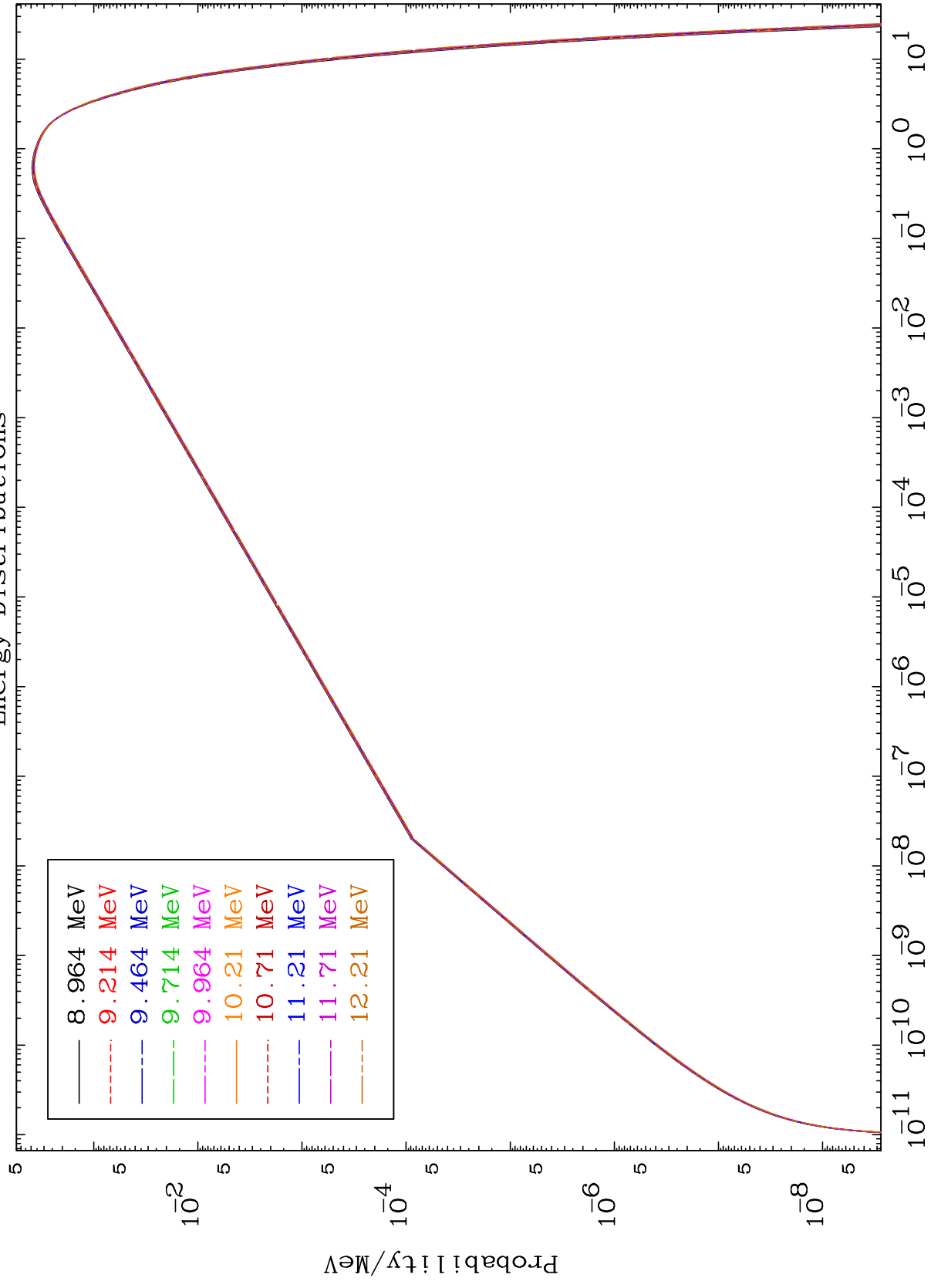
100-Fm-252



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Fission
Energy Distributions

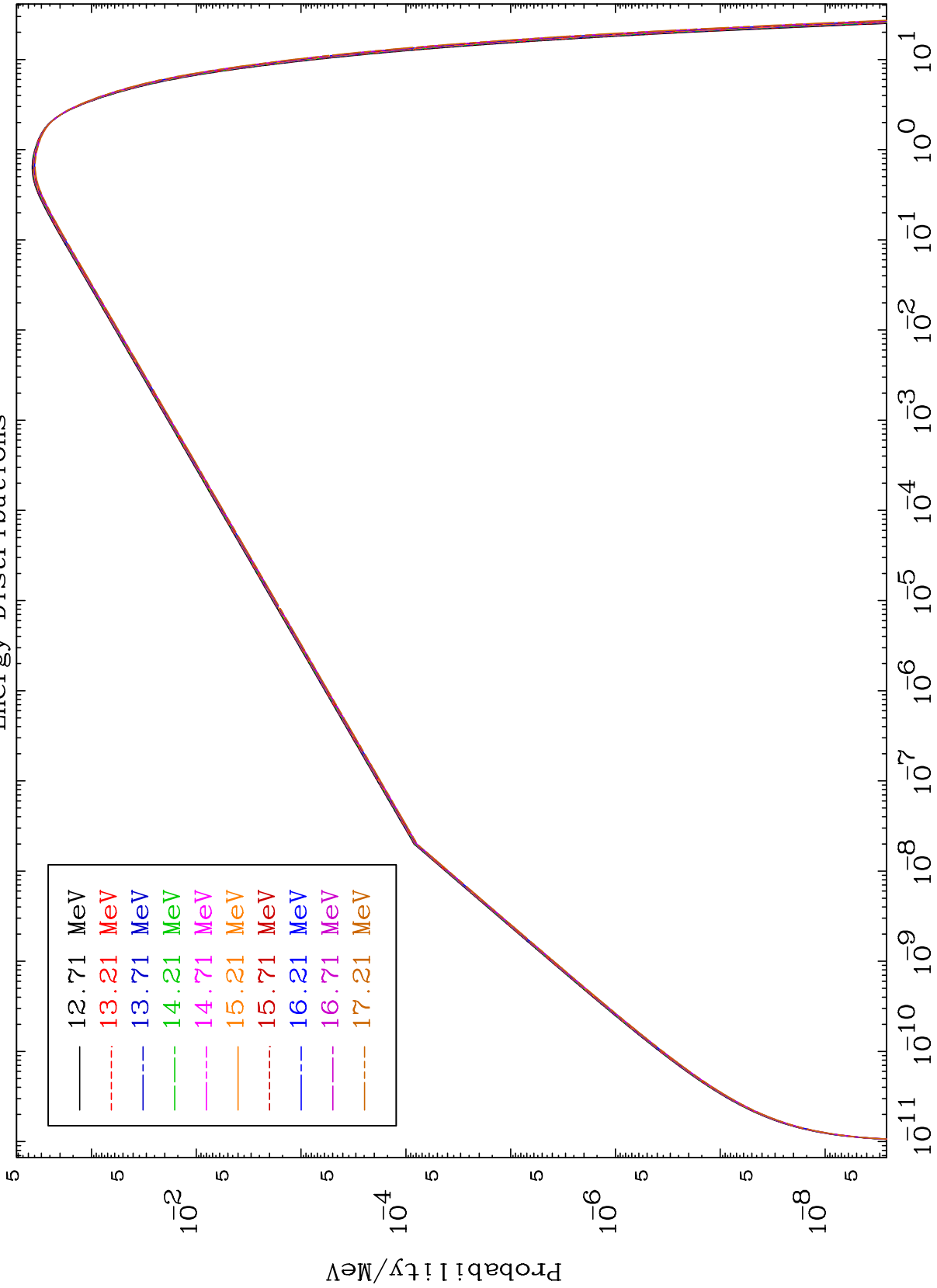
100-Fm-252



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Fission
Energy Distributions

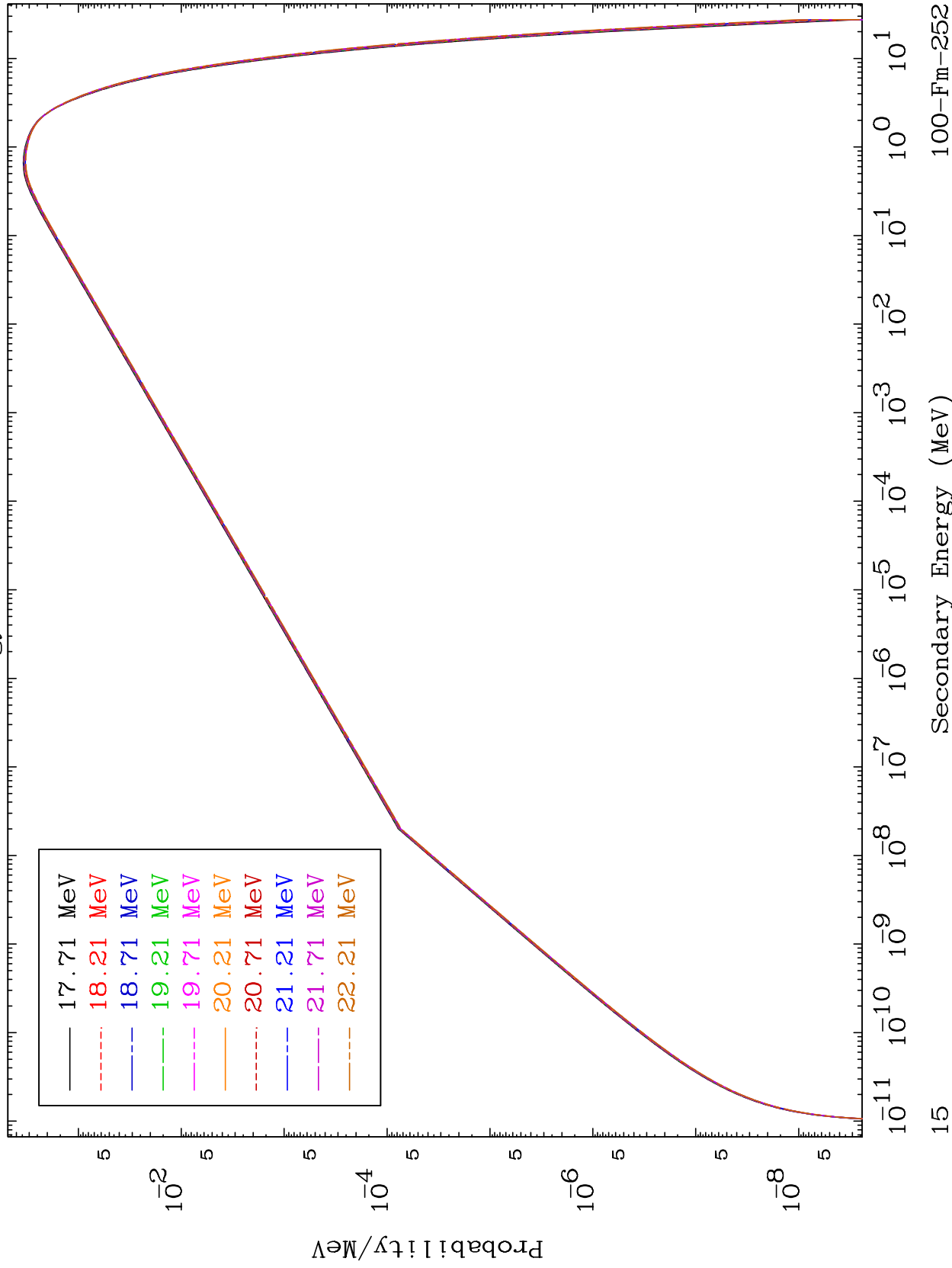
100-Fm-252



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Fission
Energy Distributions

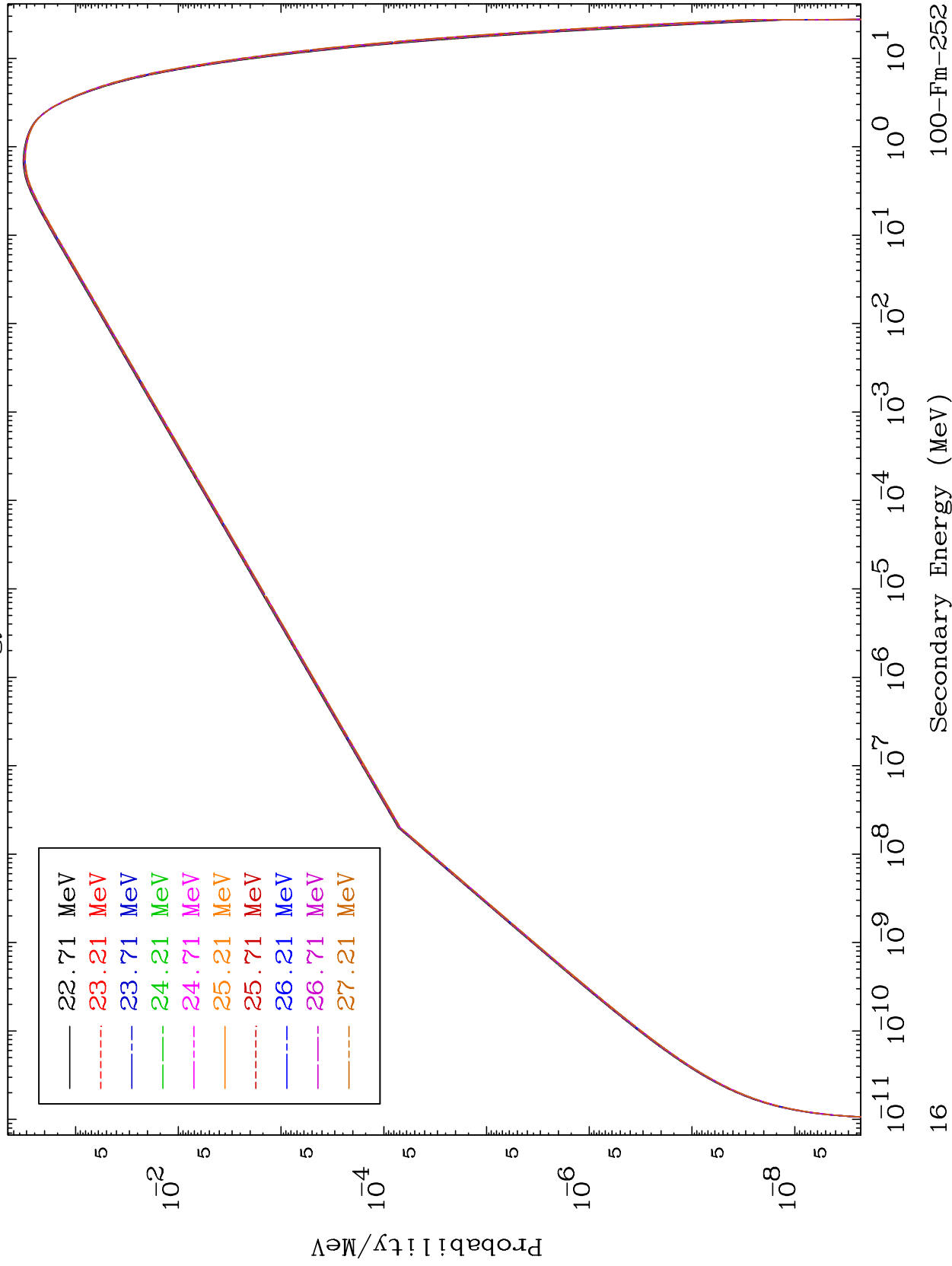
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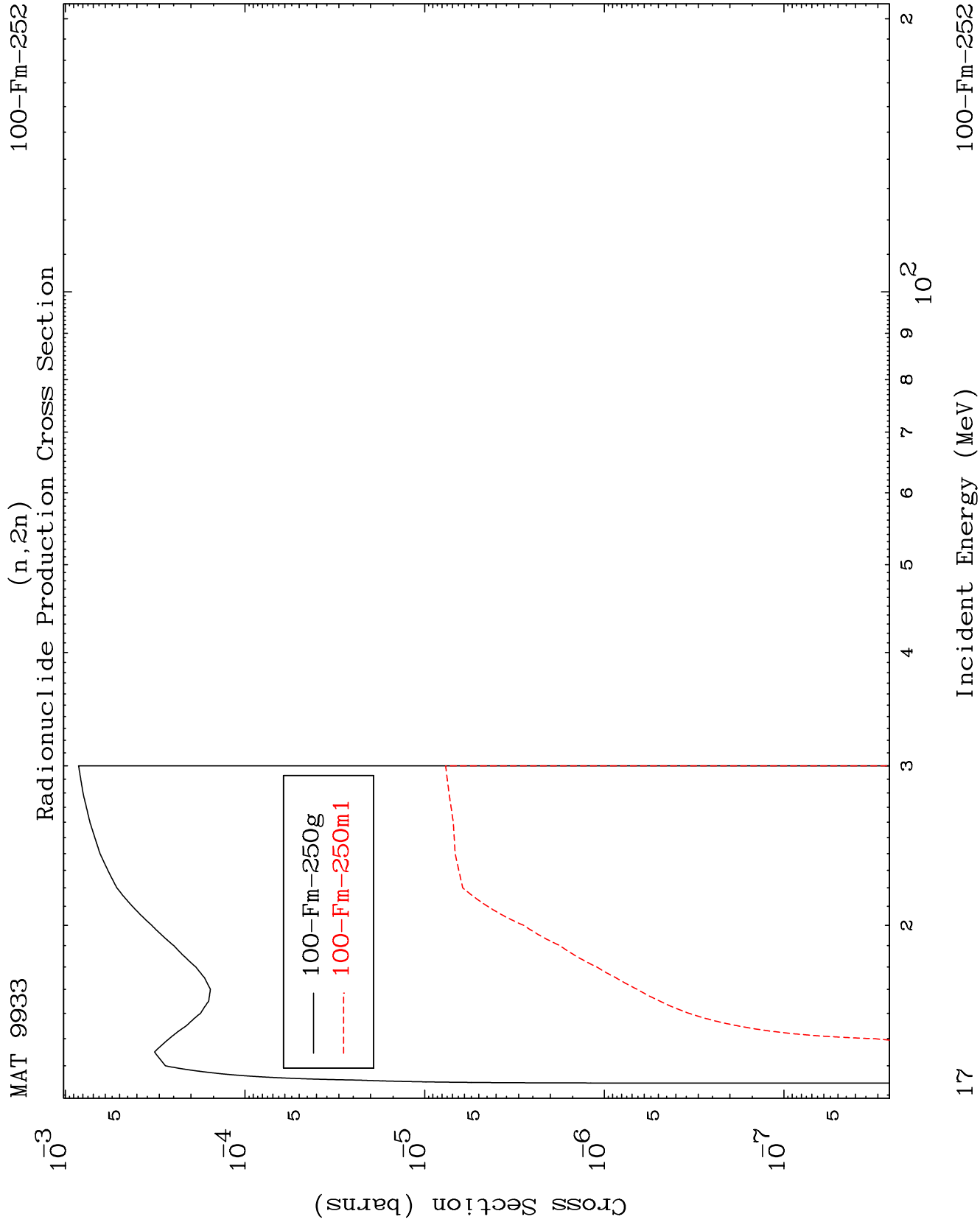


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Fission
Energy Distributions

100-Fm-252



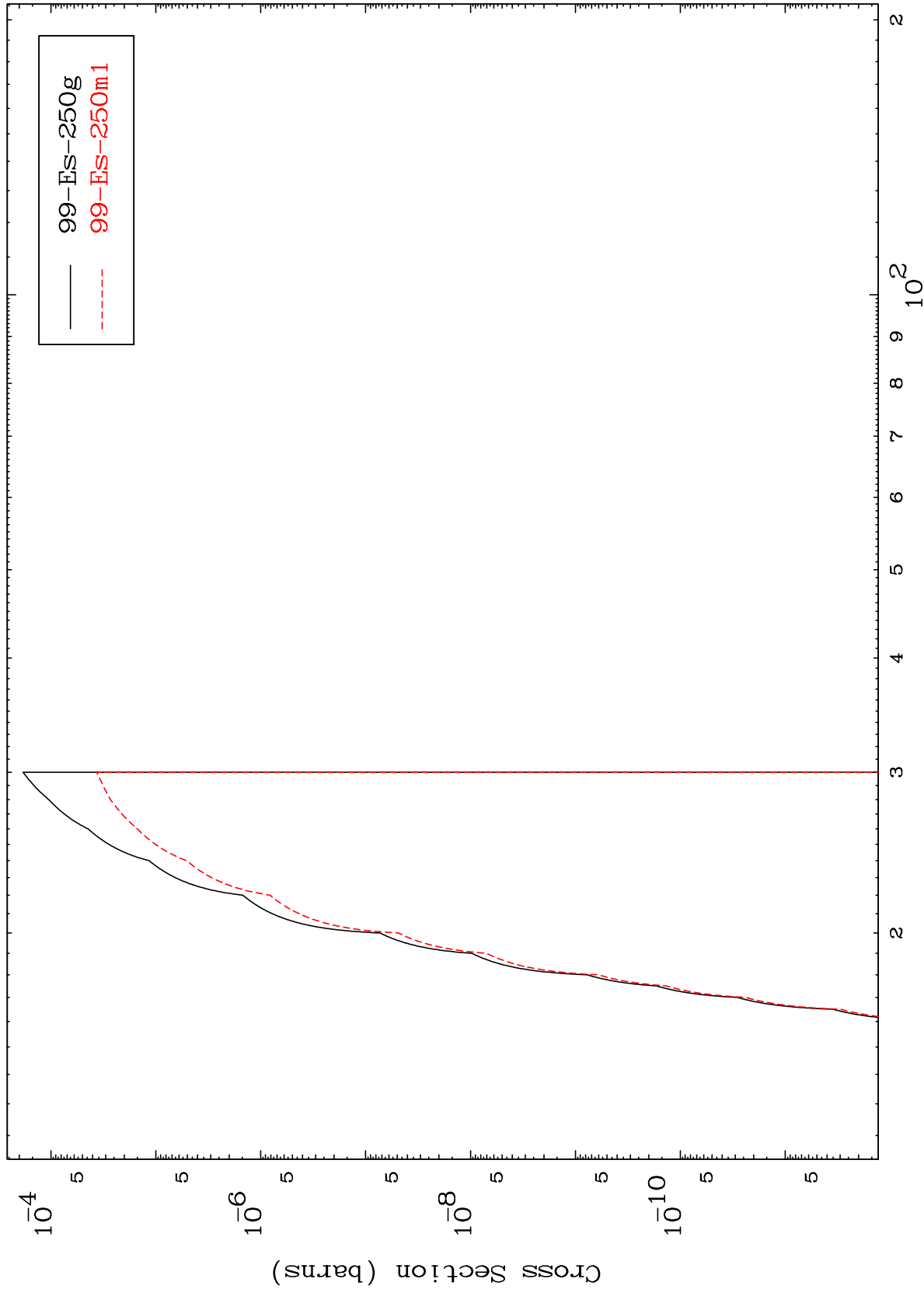


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(n,n') p

100-Fm-252

Radionuclide Production Cross Section



18

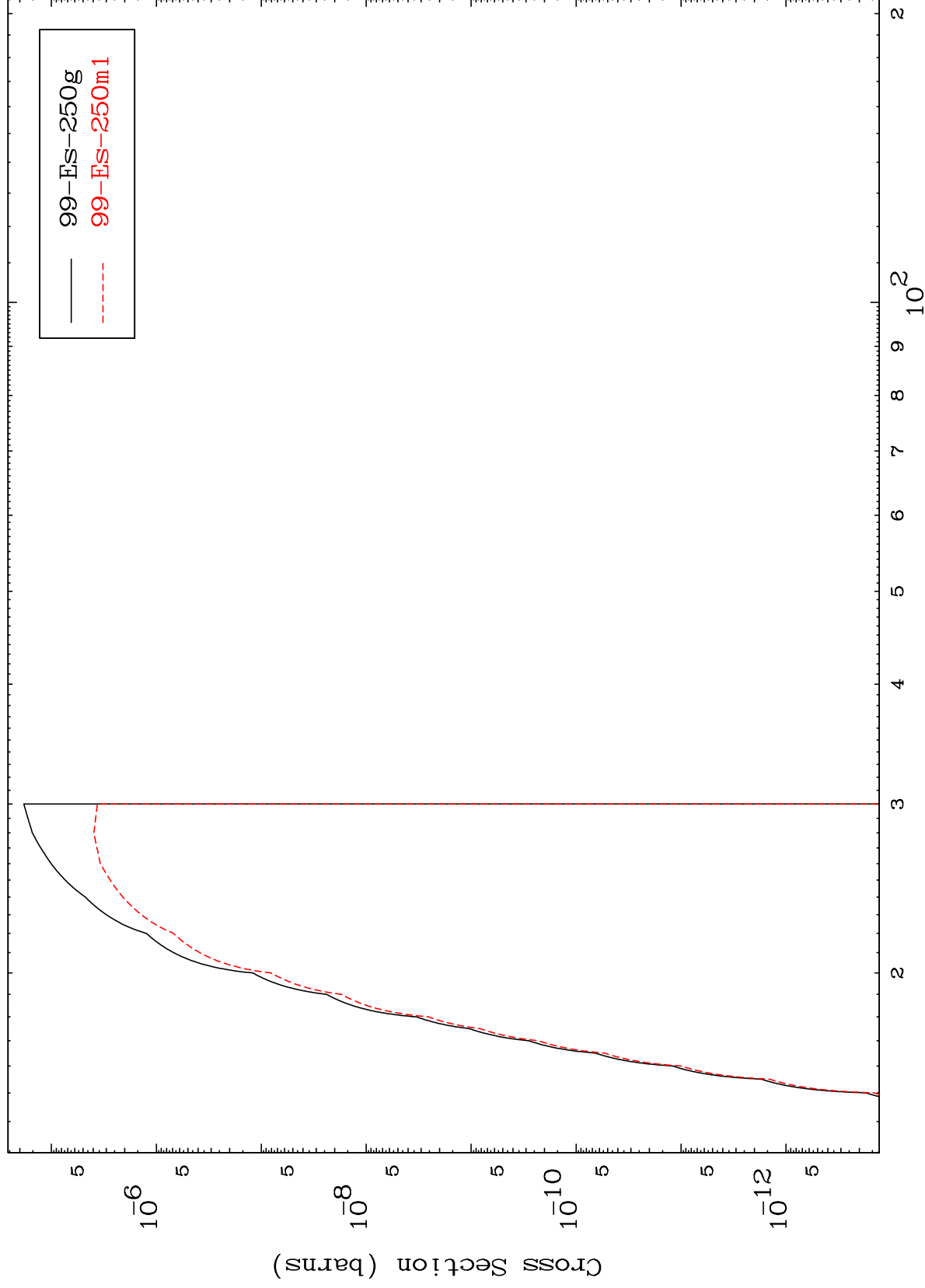
Incident Energy (MeV)

100-Fm-252

MAT 9933

(n,d)
Radionuclide Production Cross Section

100-Fm-252



19

Incident Energy (MeV)

100-Fm-252