

Program EVALPLOT
(Version 2021-1)

by

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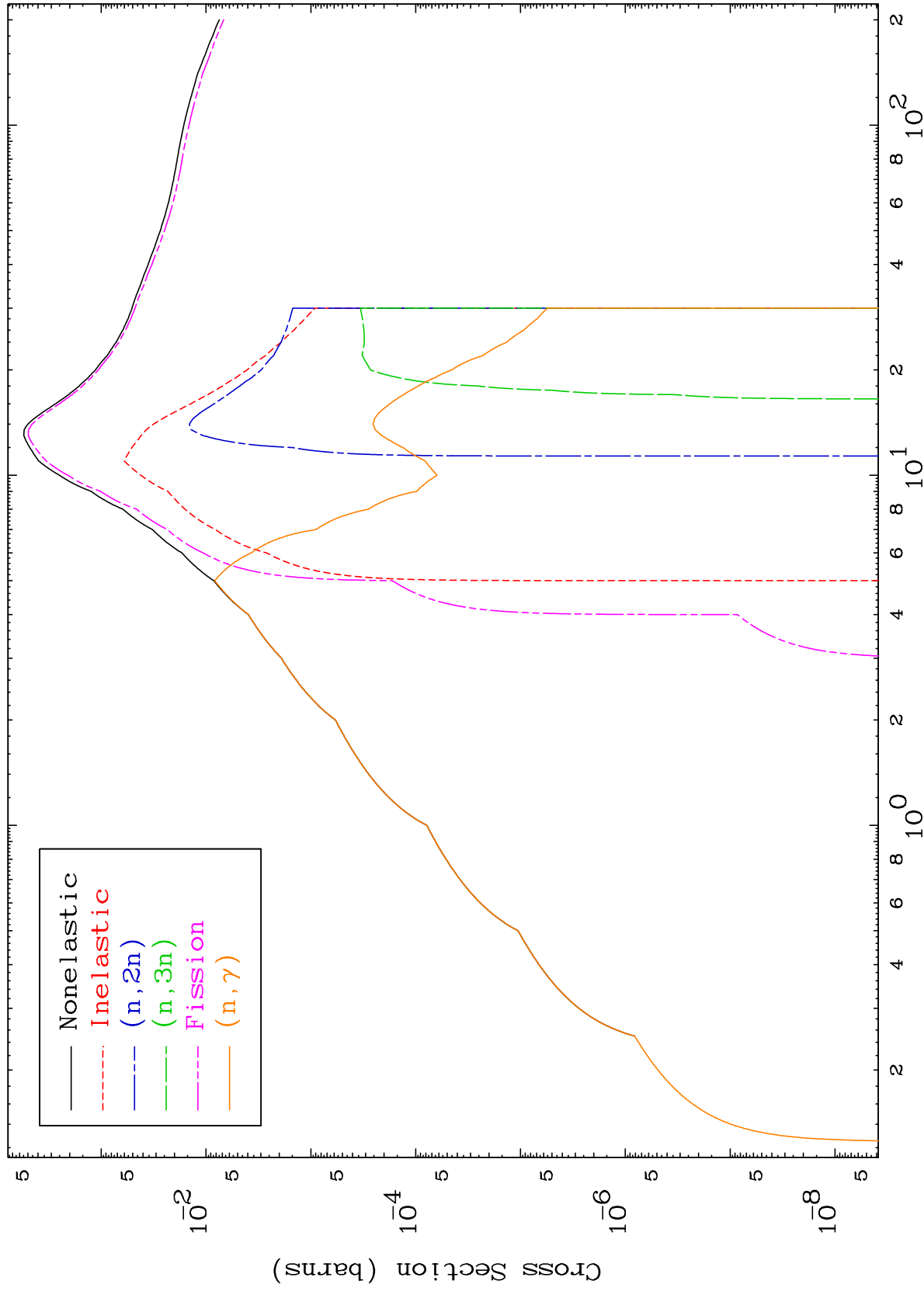
Press Mouse Button to Start

MAT 9938

Photon Major

100-Fm-257

0 Kelvin Cross Sections



1

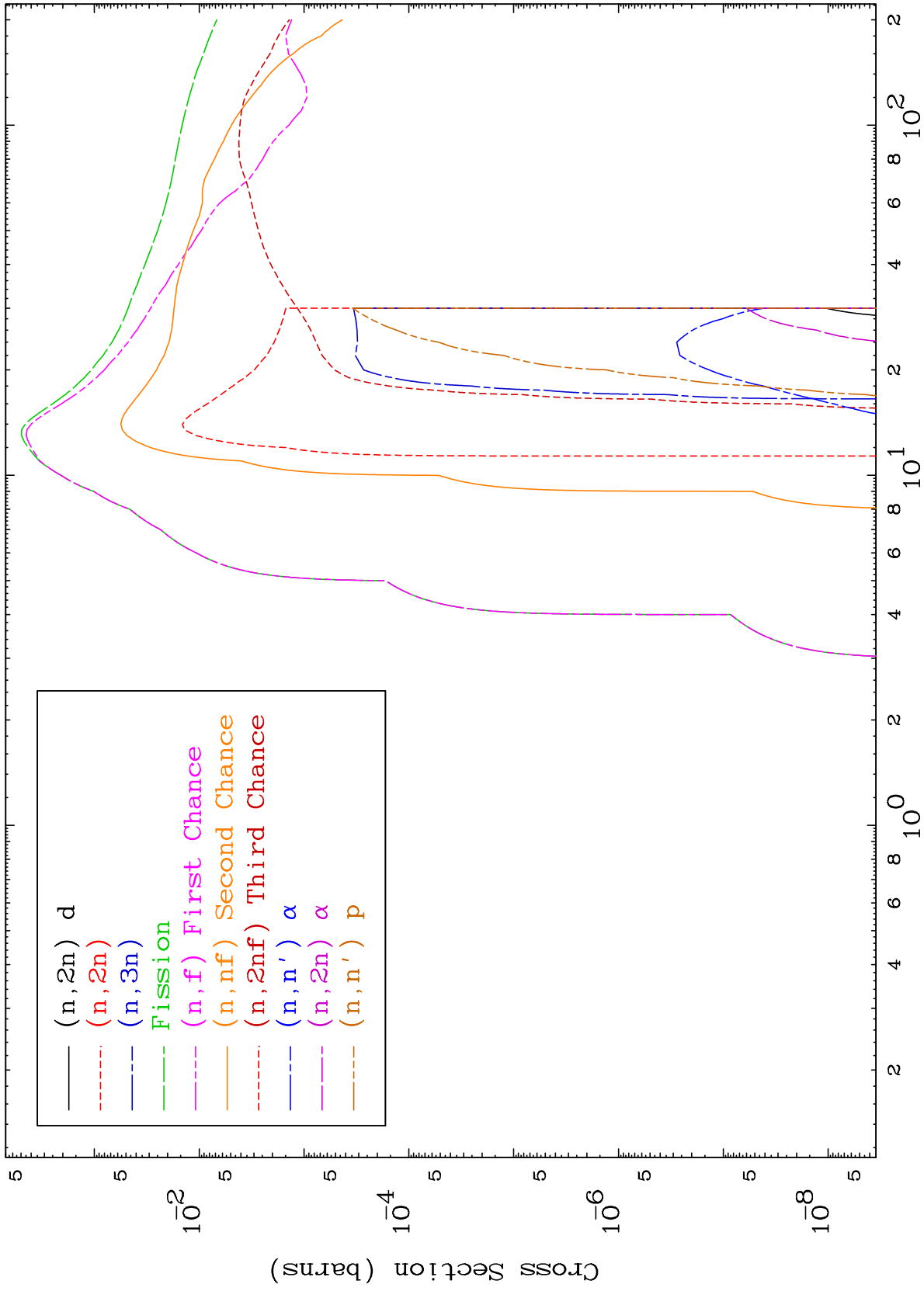
Incident Energy (MeV)

100-Fm-257

MAT 9938

Photon Neutron Absorption
0 Kelvin Cross Sections

100-Fm-257



2

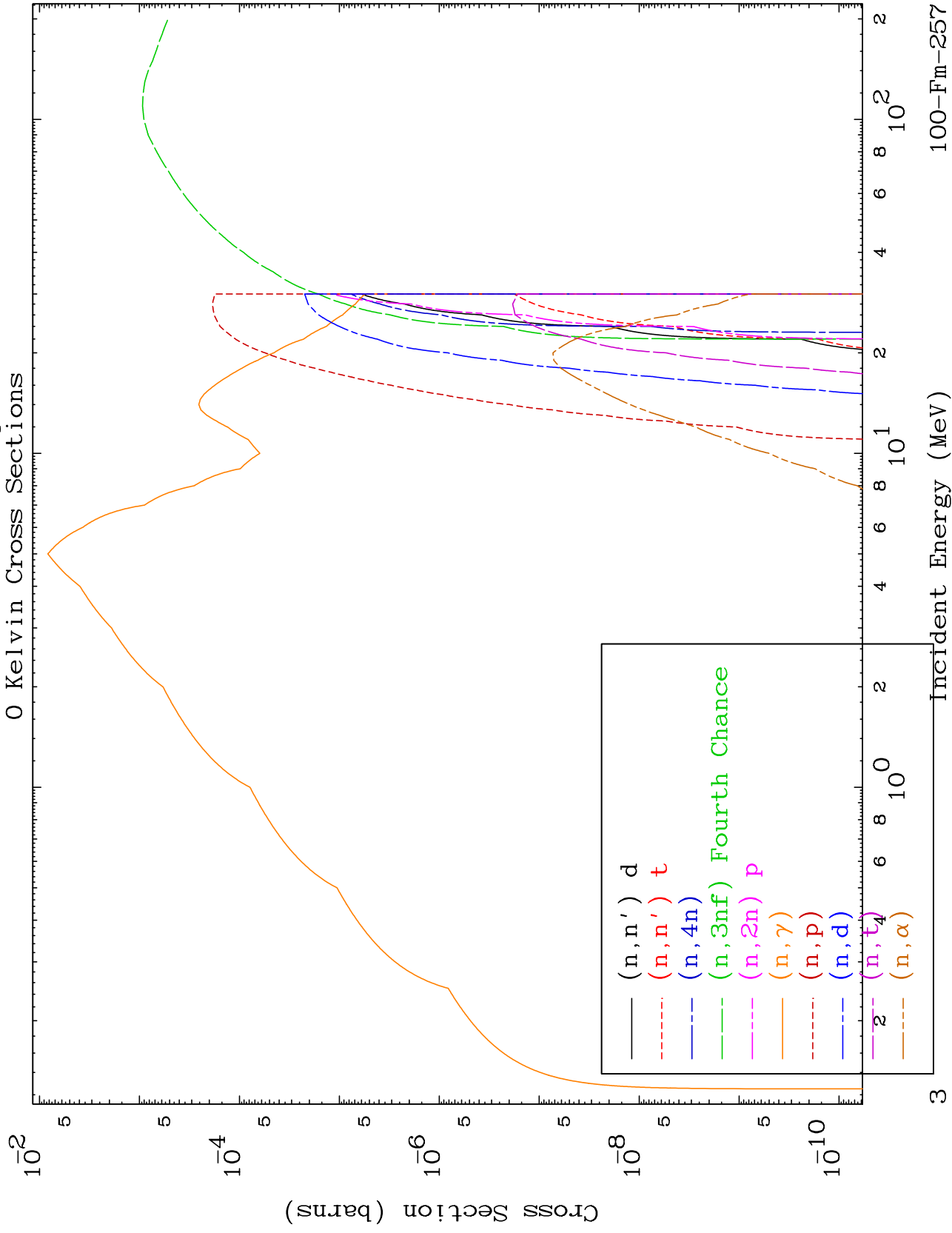
Incident Energy (MeV)

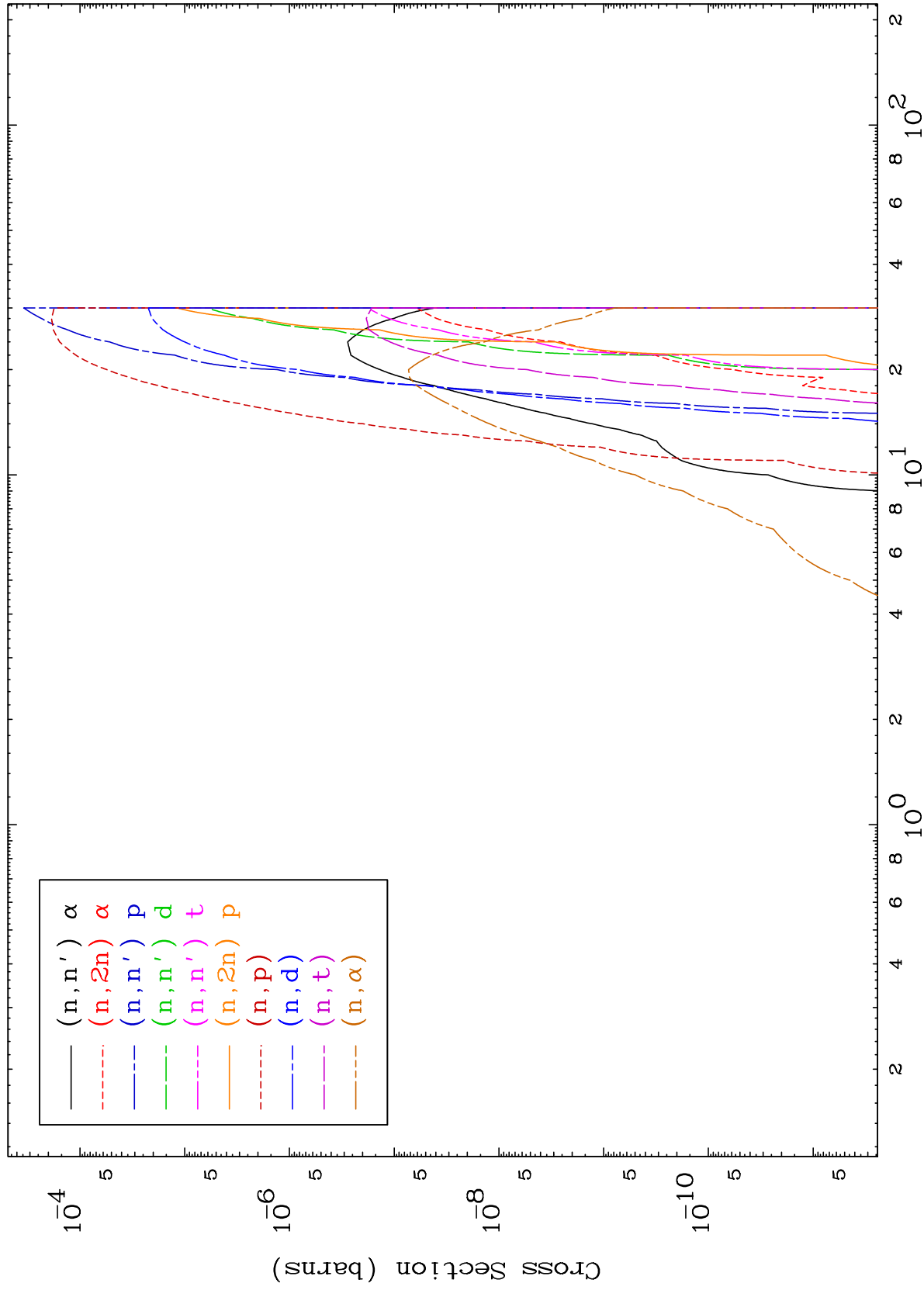
100-Fm-257

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Photon Neutron Absorption
0 Kelvin Cross Sections

100-Fm-257

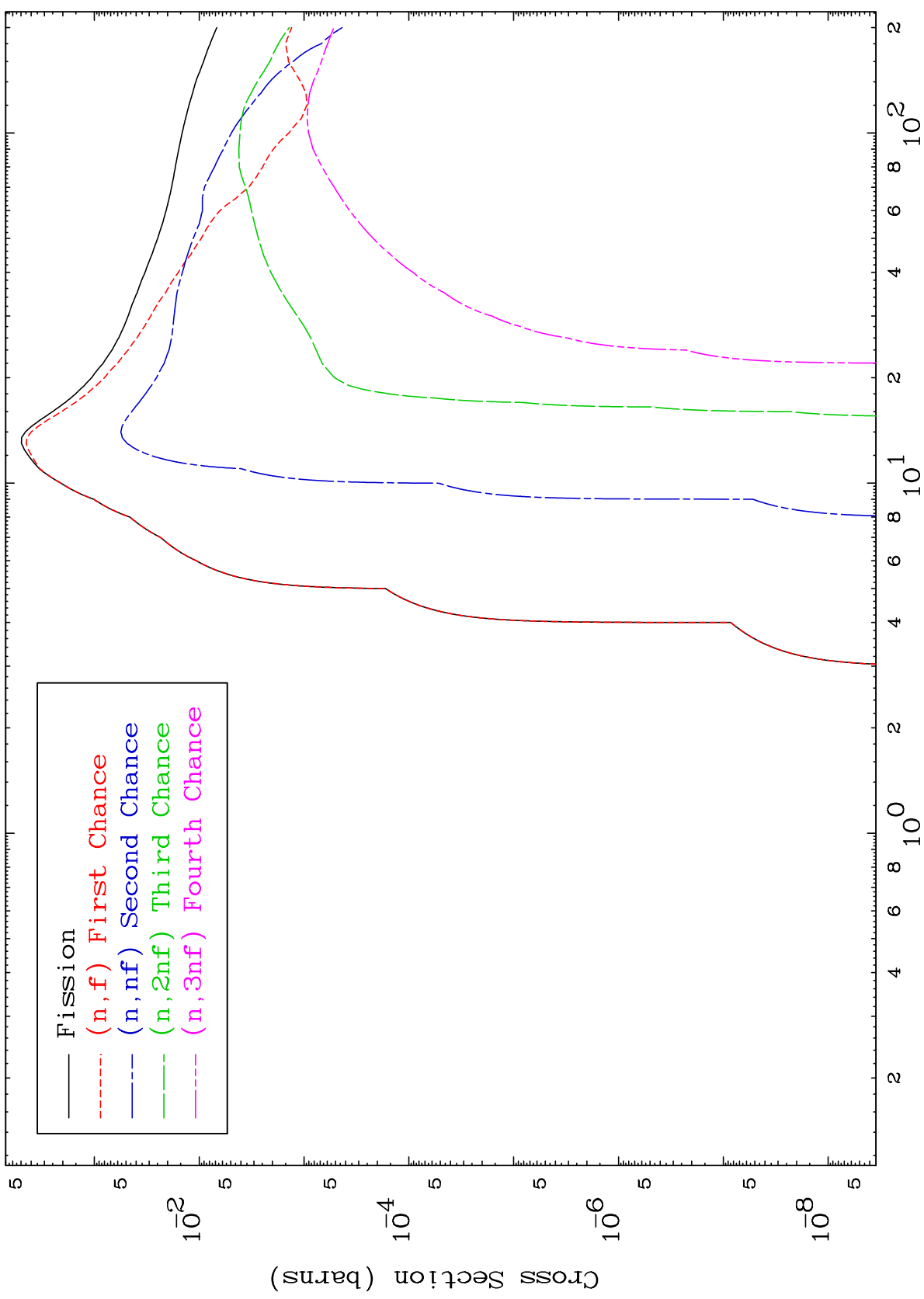




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Photon Fission
0 Kelvin Cross Sections

100-Fm-257



5

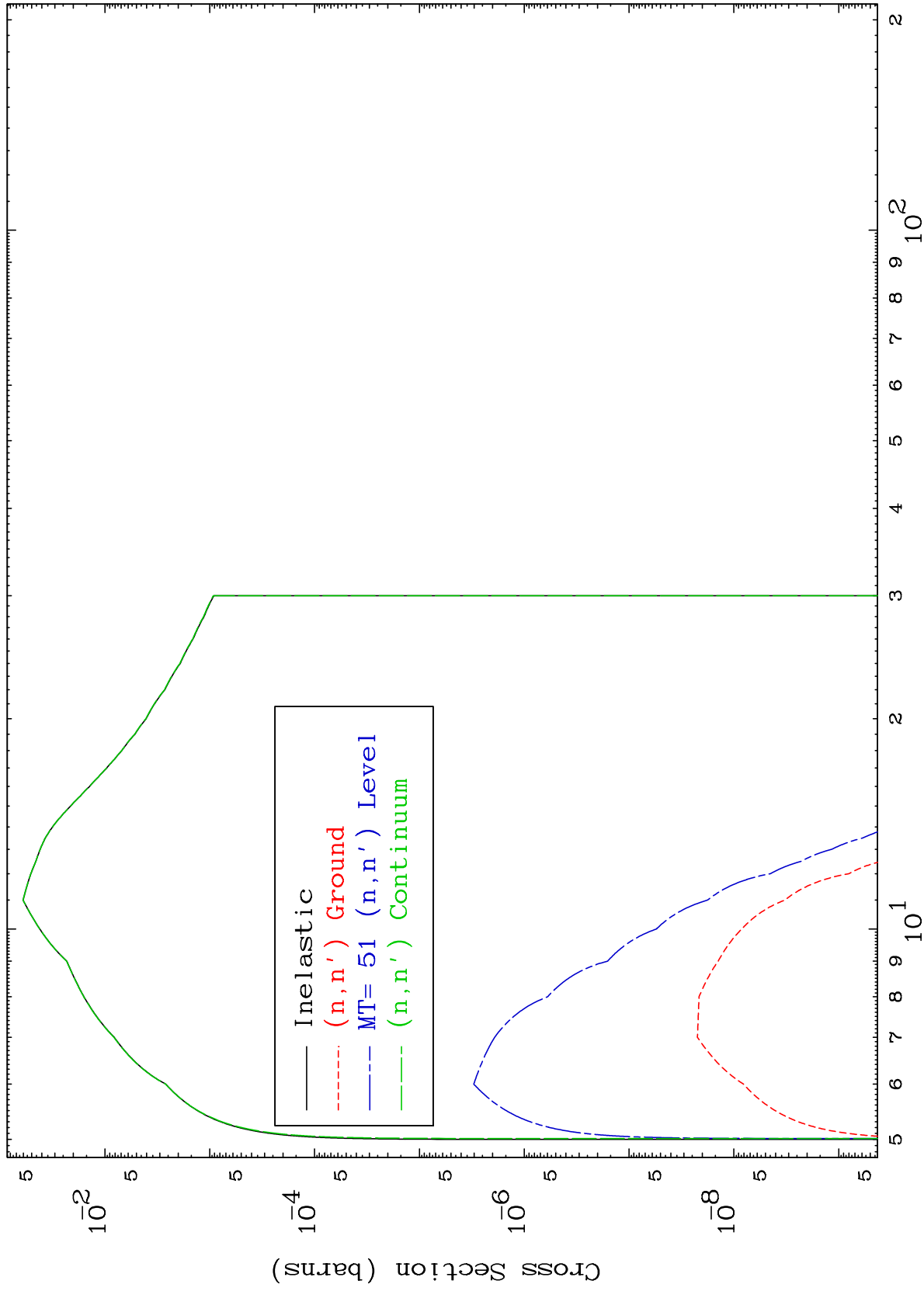
Incident Energy (MeV)

100-Fm-257

MAT 9938

(γ, n') Levels
0 Kelvin Cross Sections

100-Fm-257



6

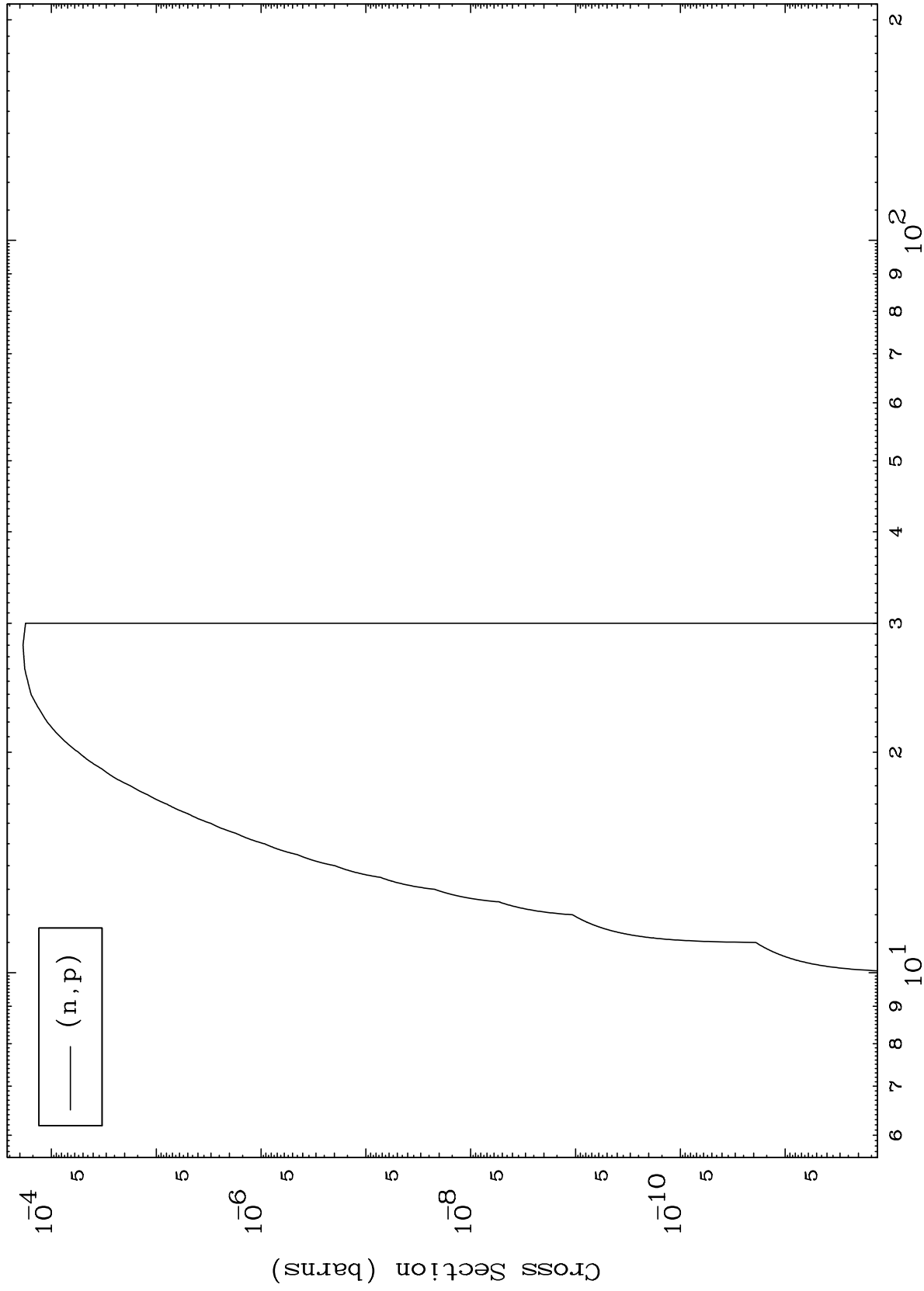
Incident Energy (MeV)

100-Fm-257

MAT 9938

100-Fm-257

(γ ,p) Levels
0 Kelvin Cross Sections



7

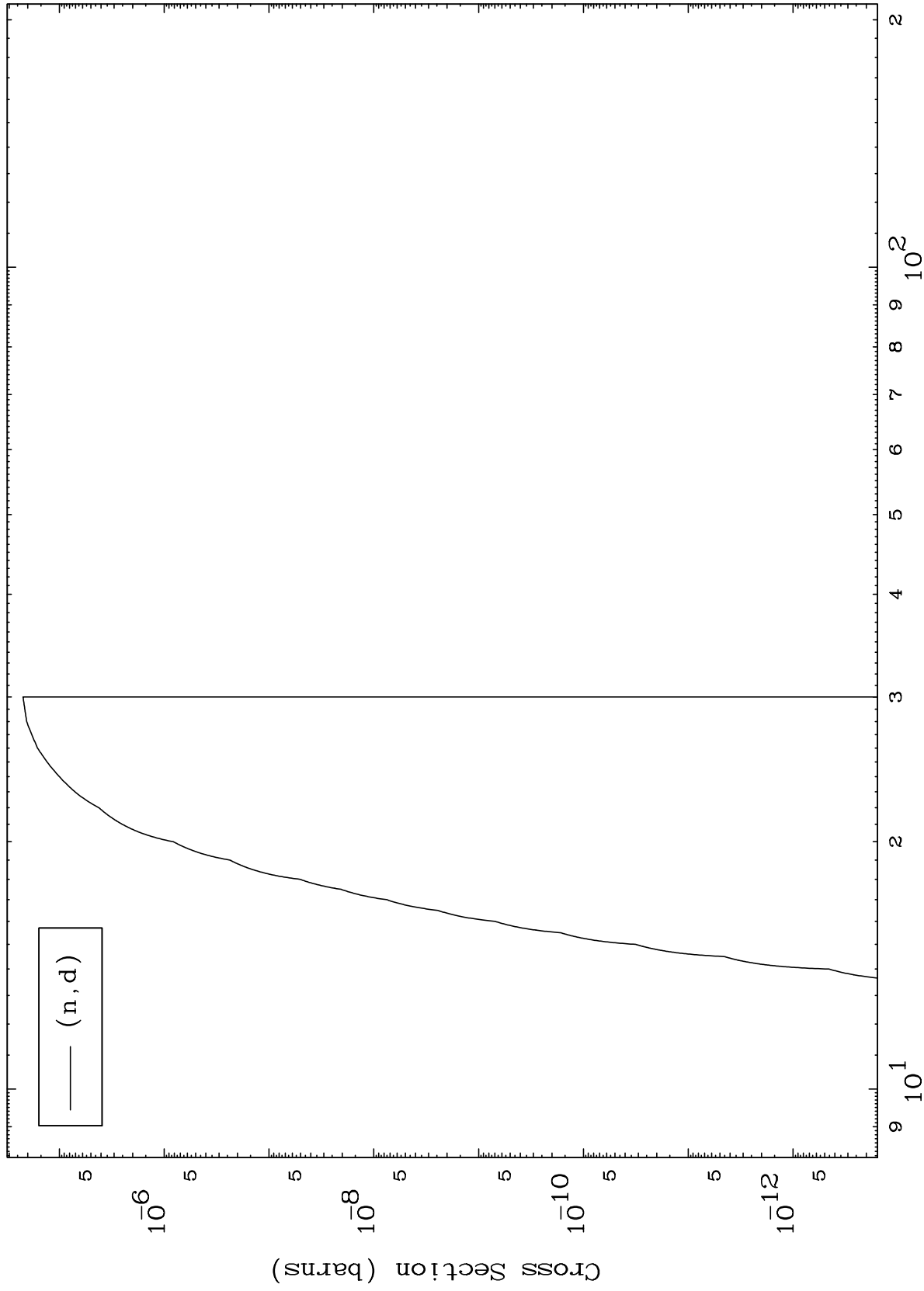
Incident Energy (MeV)

100-Fm-257

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(γ ,d) Levels
0 Kelvin Cross Sections

100-Fm-257



8

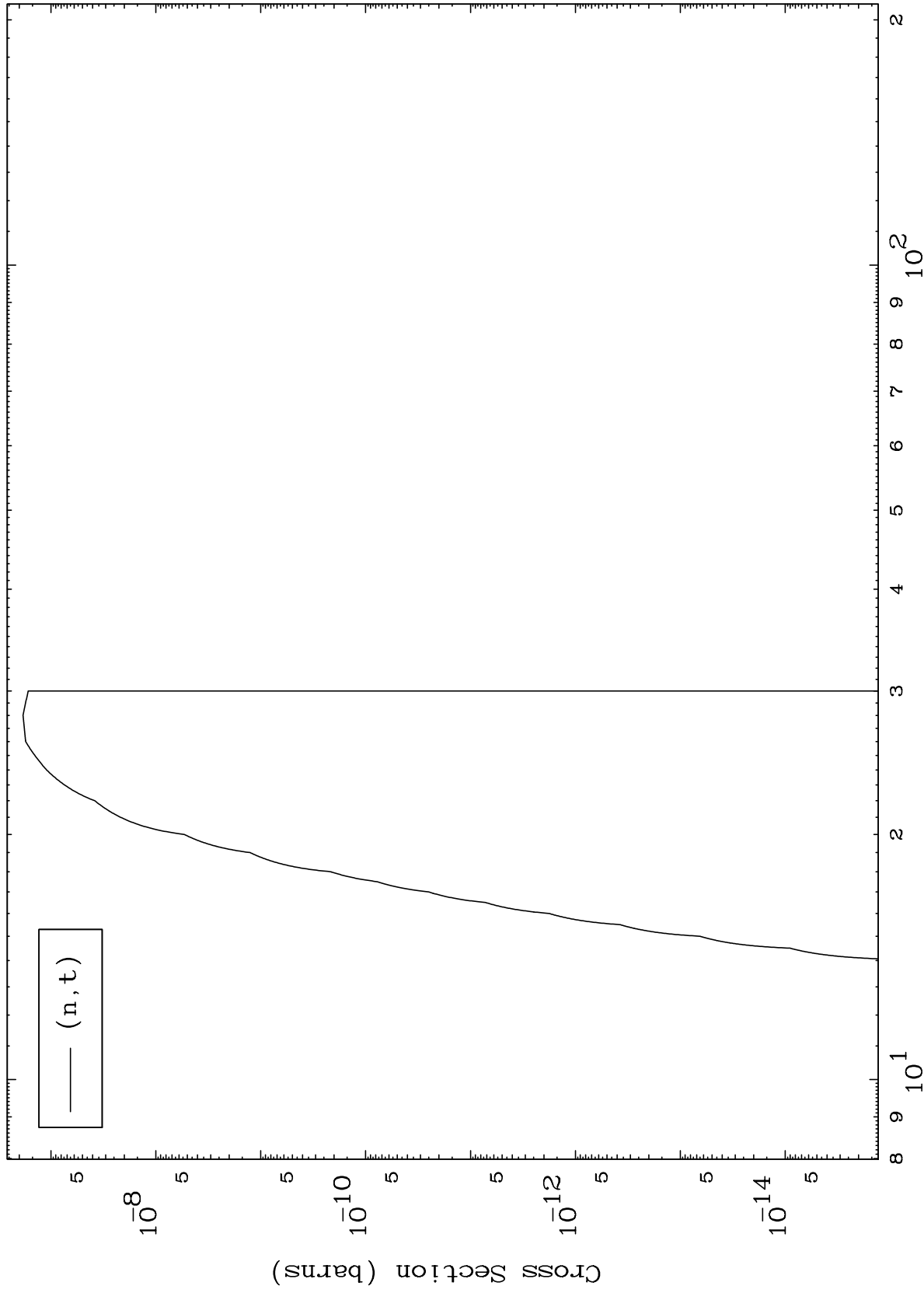
Incident Energy (MeV)

100-Fm-257

MAT 9938

(γ, t) Levels
0 Kelvin Cross Sections

100-Fm-257



9

Incident Energy (MeV)

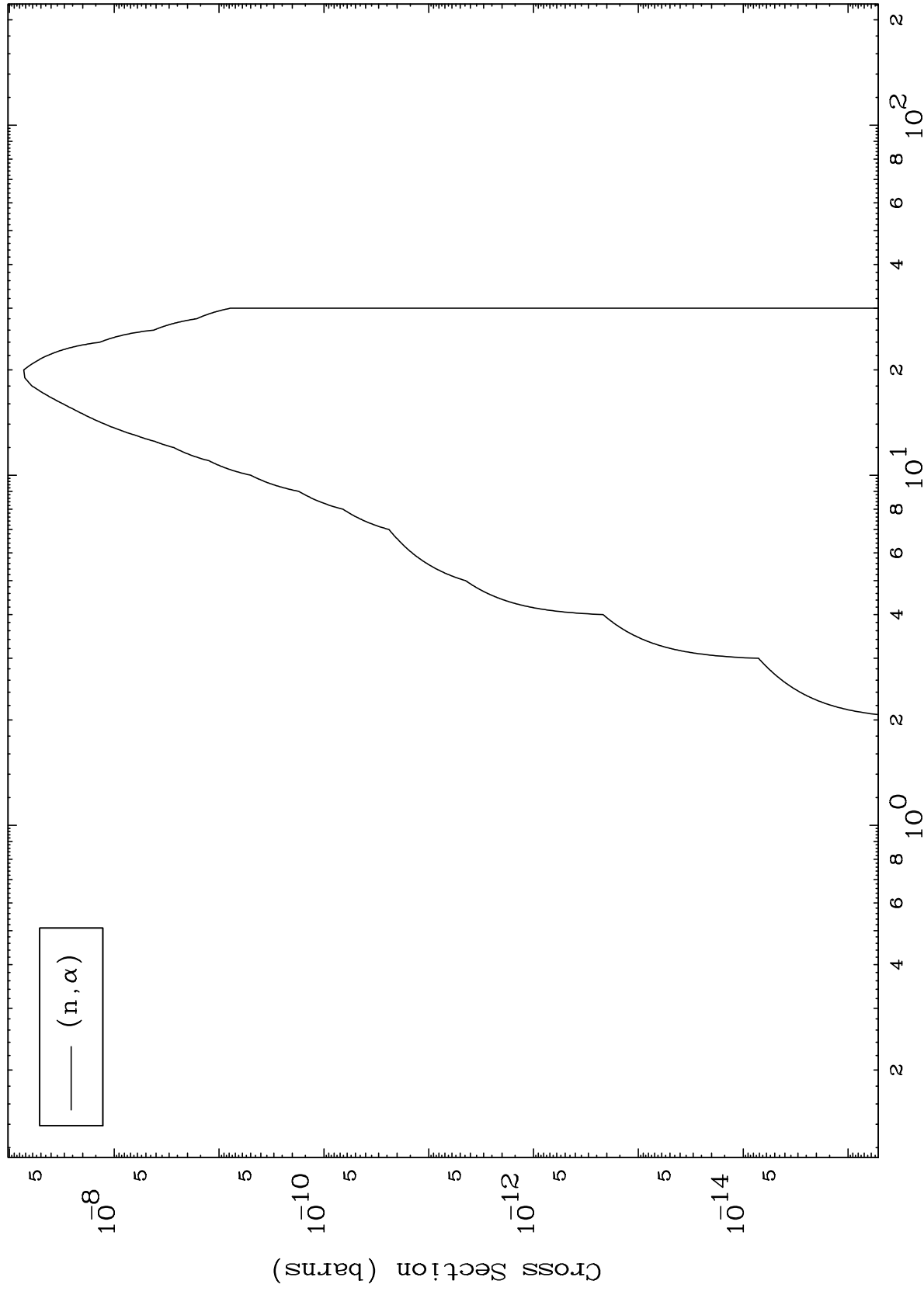
100-Fm-257

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(γ, α) Levels

100-Fm-257

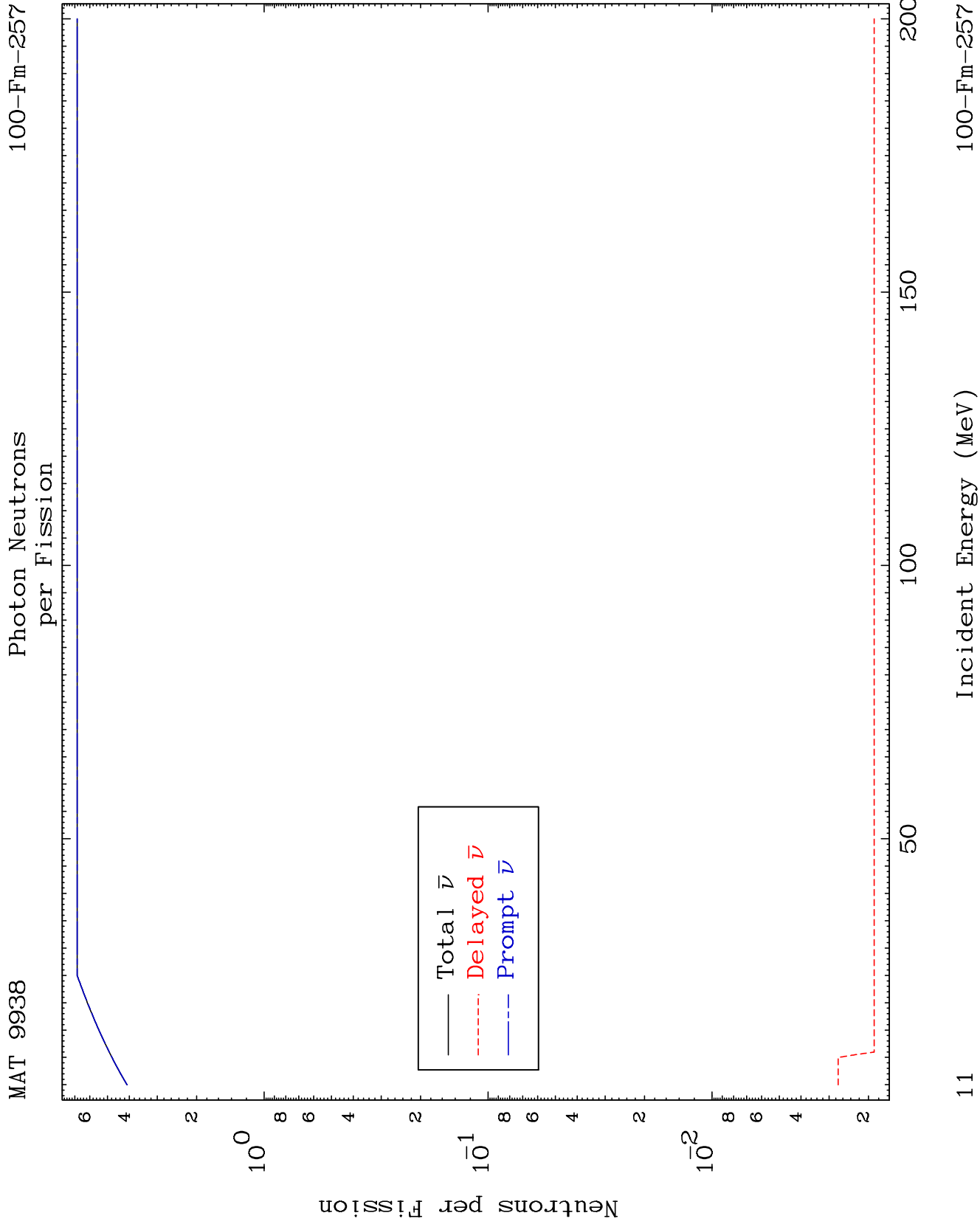
0 Kelvin Cross Sections



10

Incident Energy (MeV)

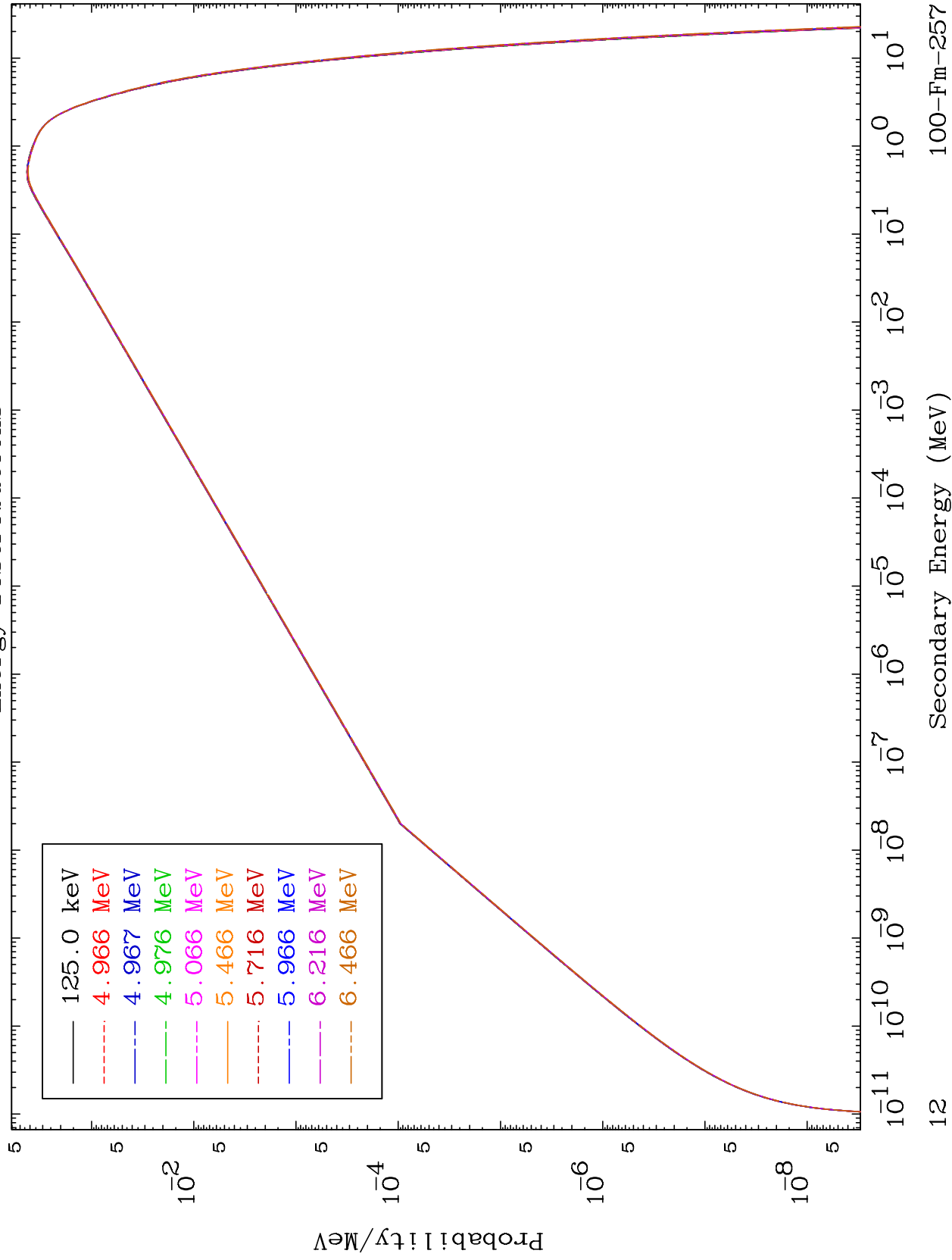
100-Fm-257



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Fission
Energy Distributions

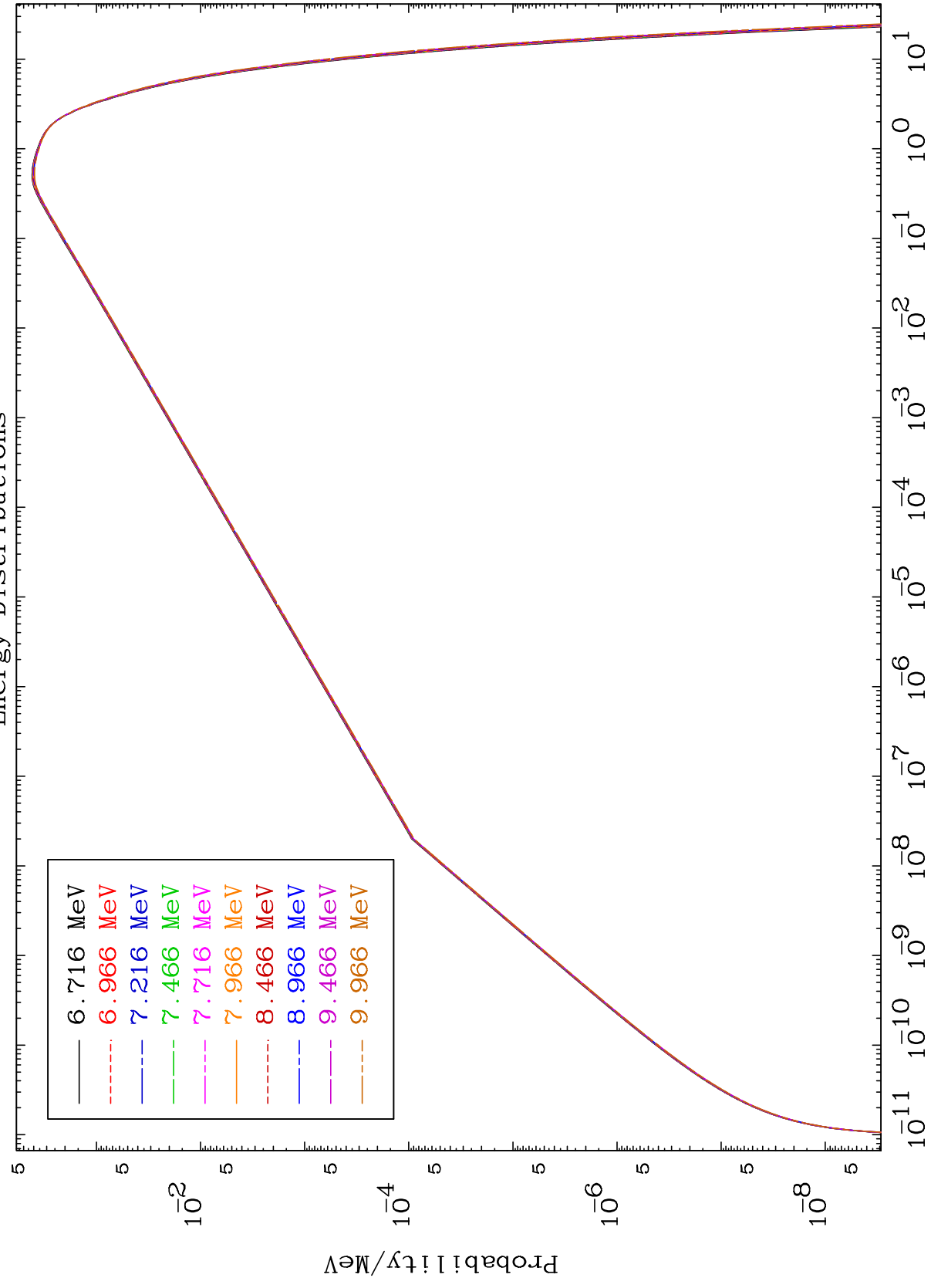
100-Fm-257



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Fission
Energy Distributions

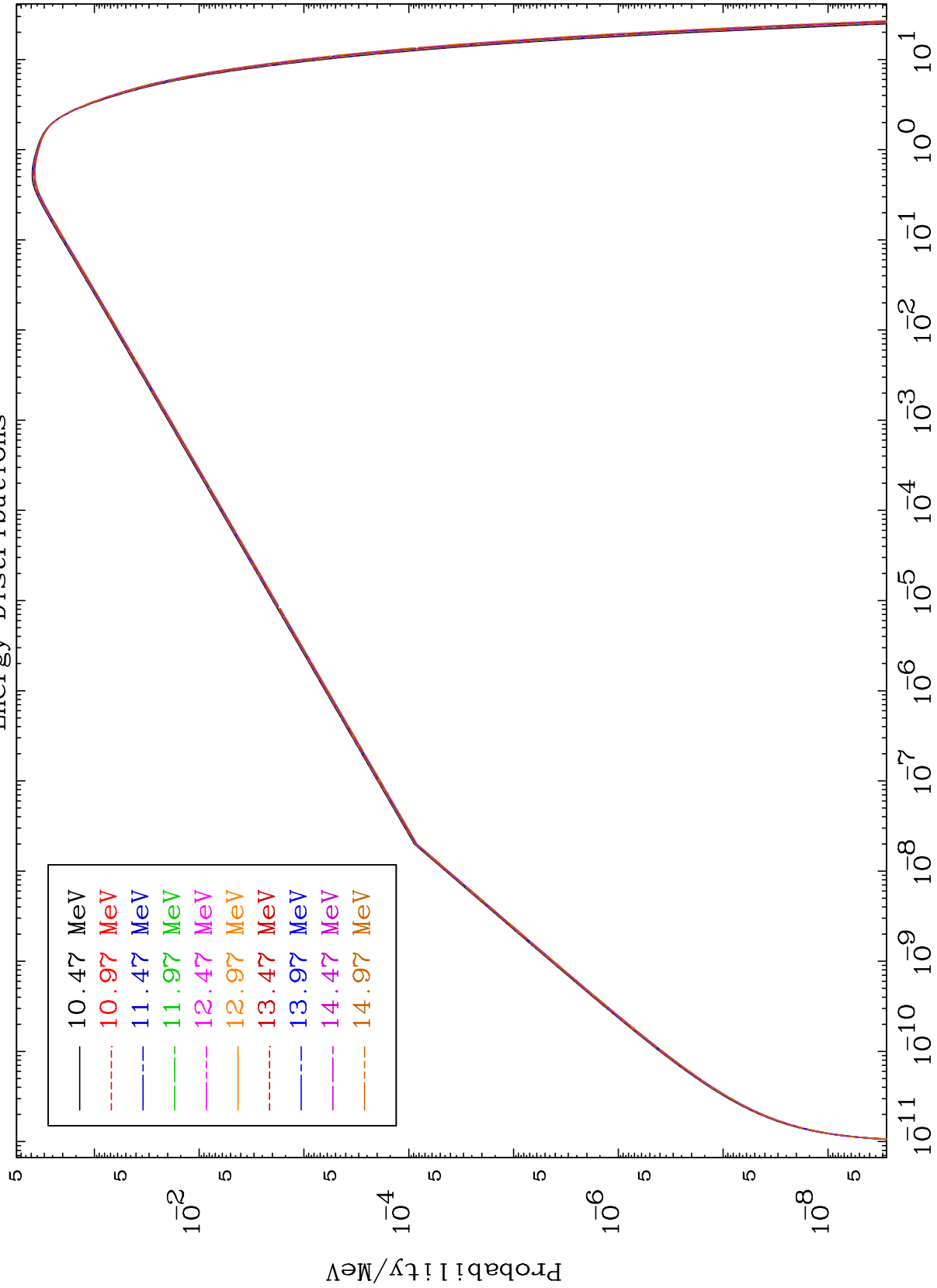
100-Fm-257



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Fission
Energy Distributions

100-Fm-257



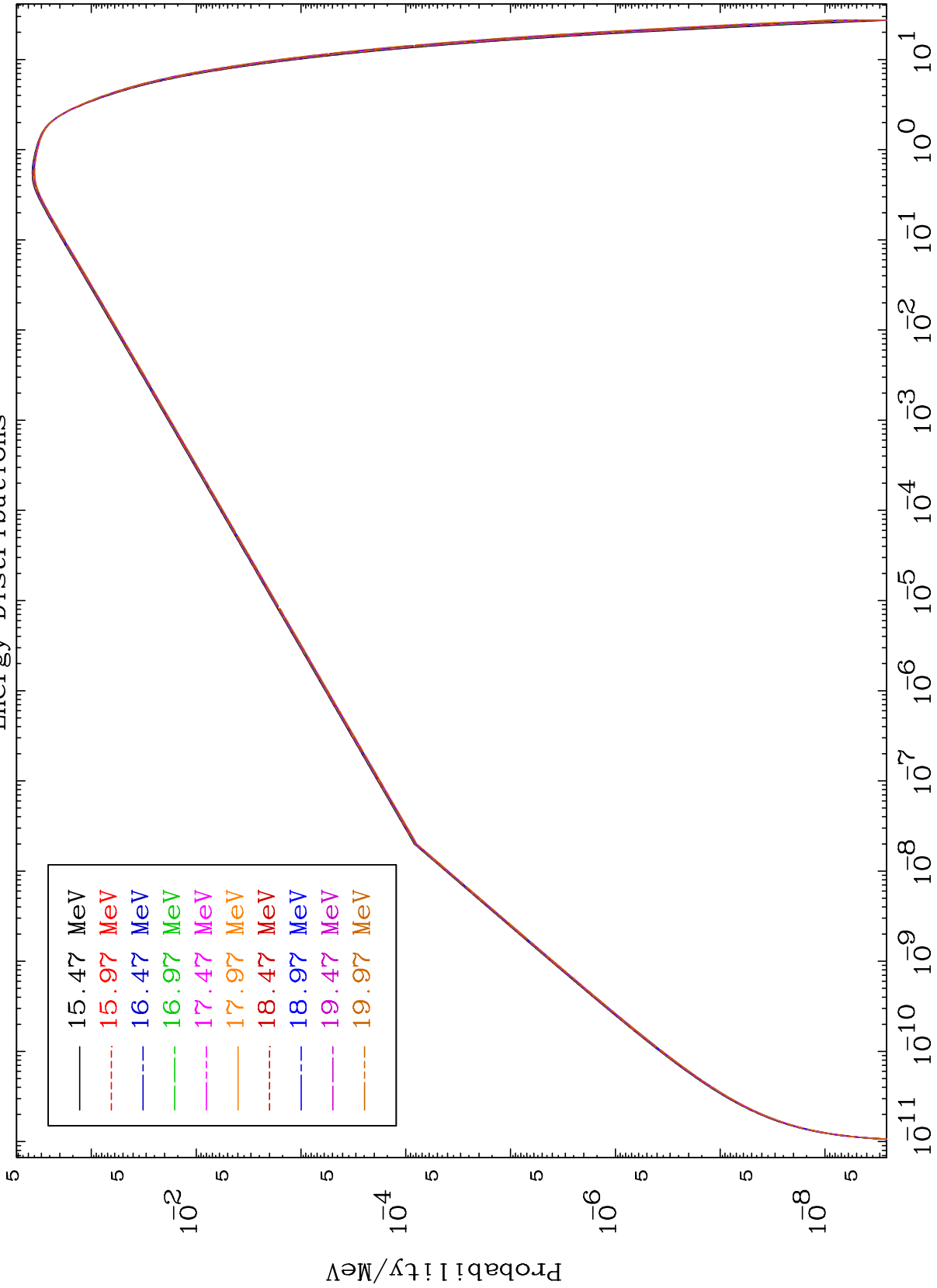
14

100-Fm-257

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Fission
Energy Distributions

100-Fm-257



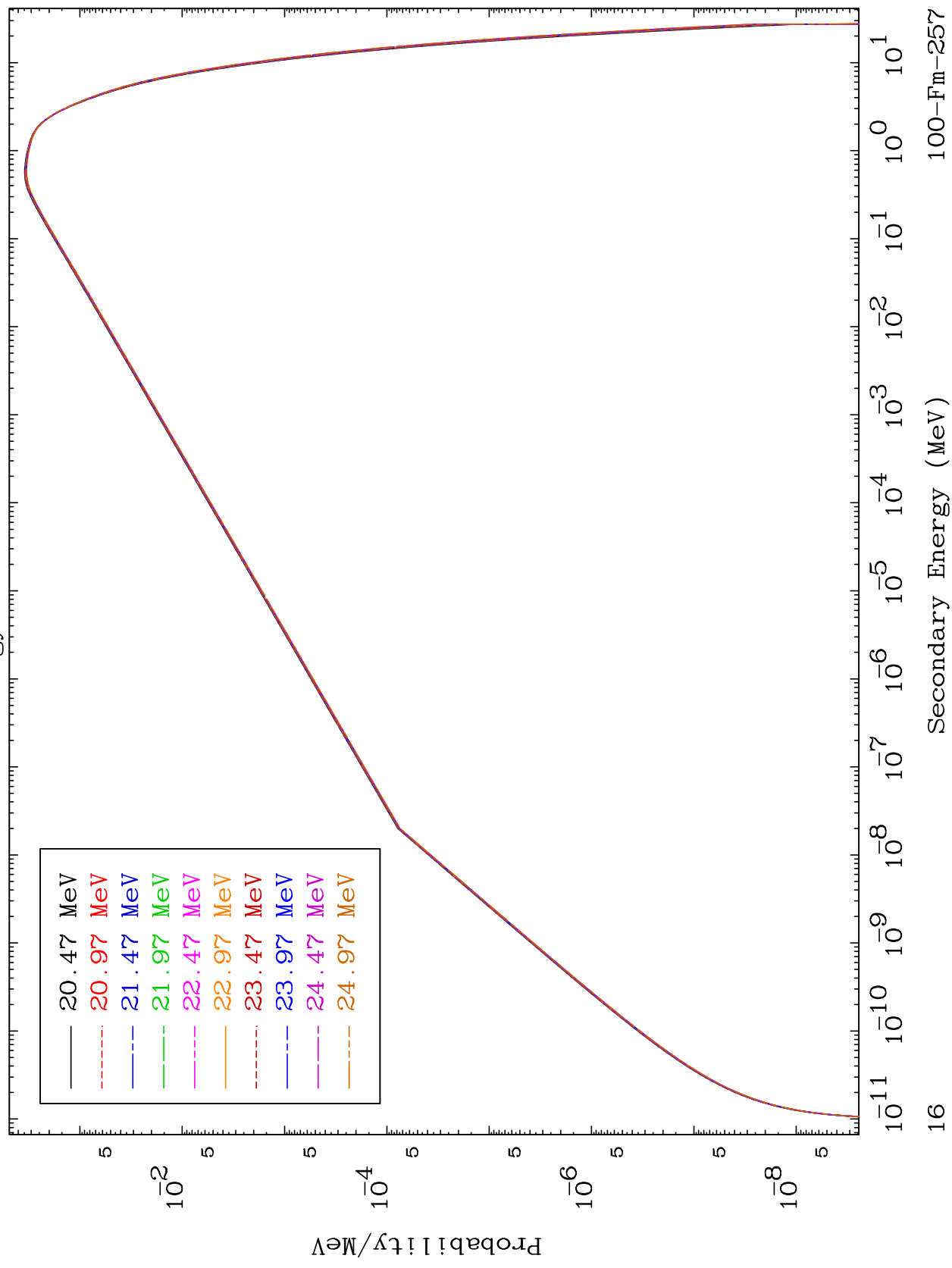
15

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Fission Energy Distributions

100-Fm-257

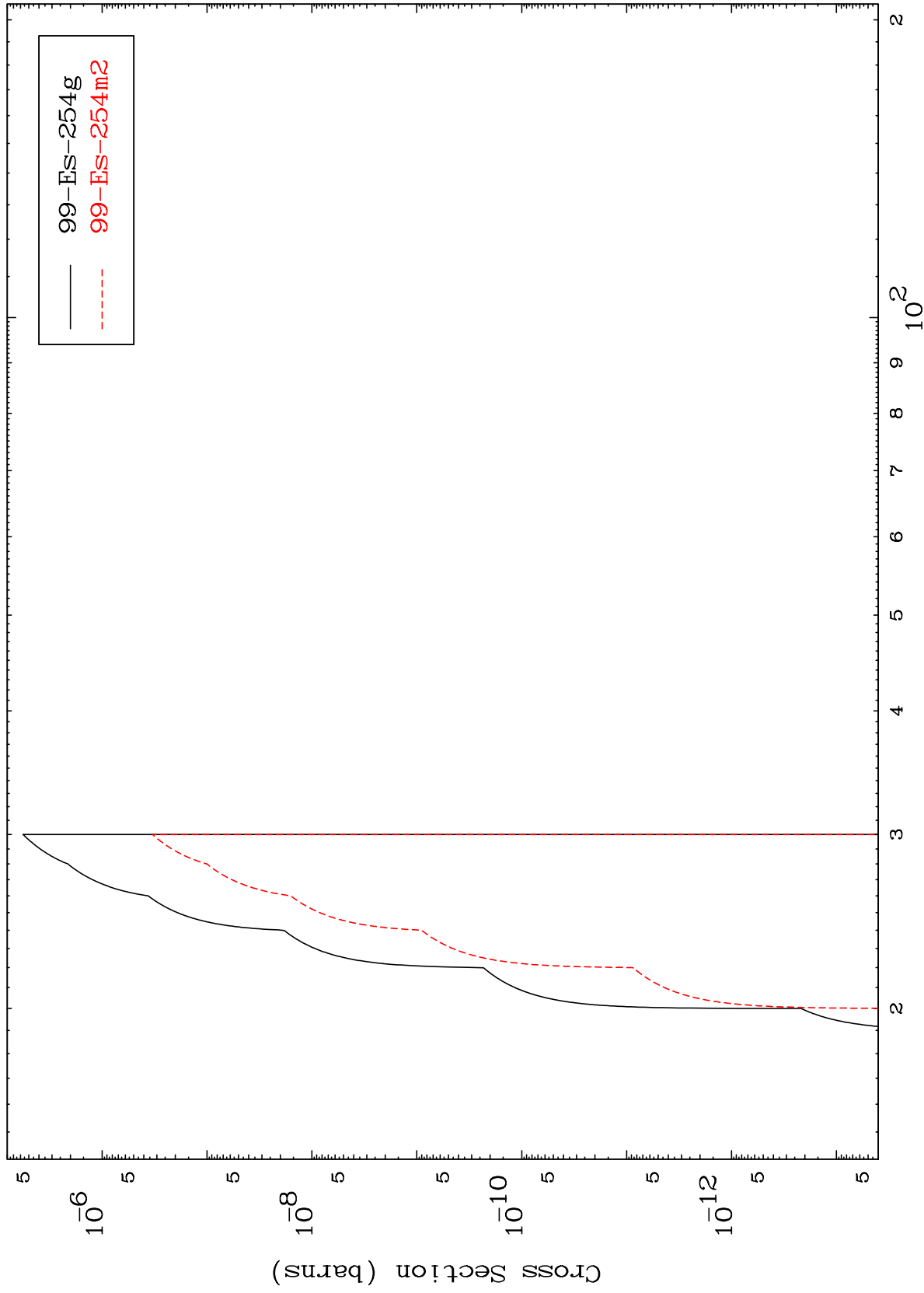


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(n,n') d

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Radionuclide Production Cross Section



17

Incident Energy (MeV)

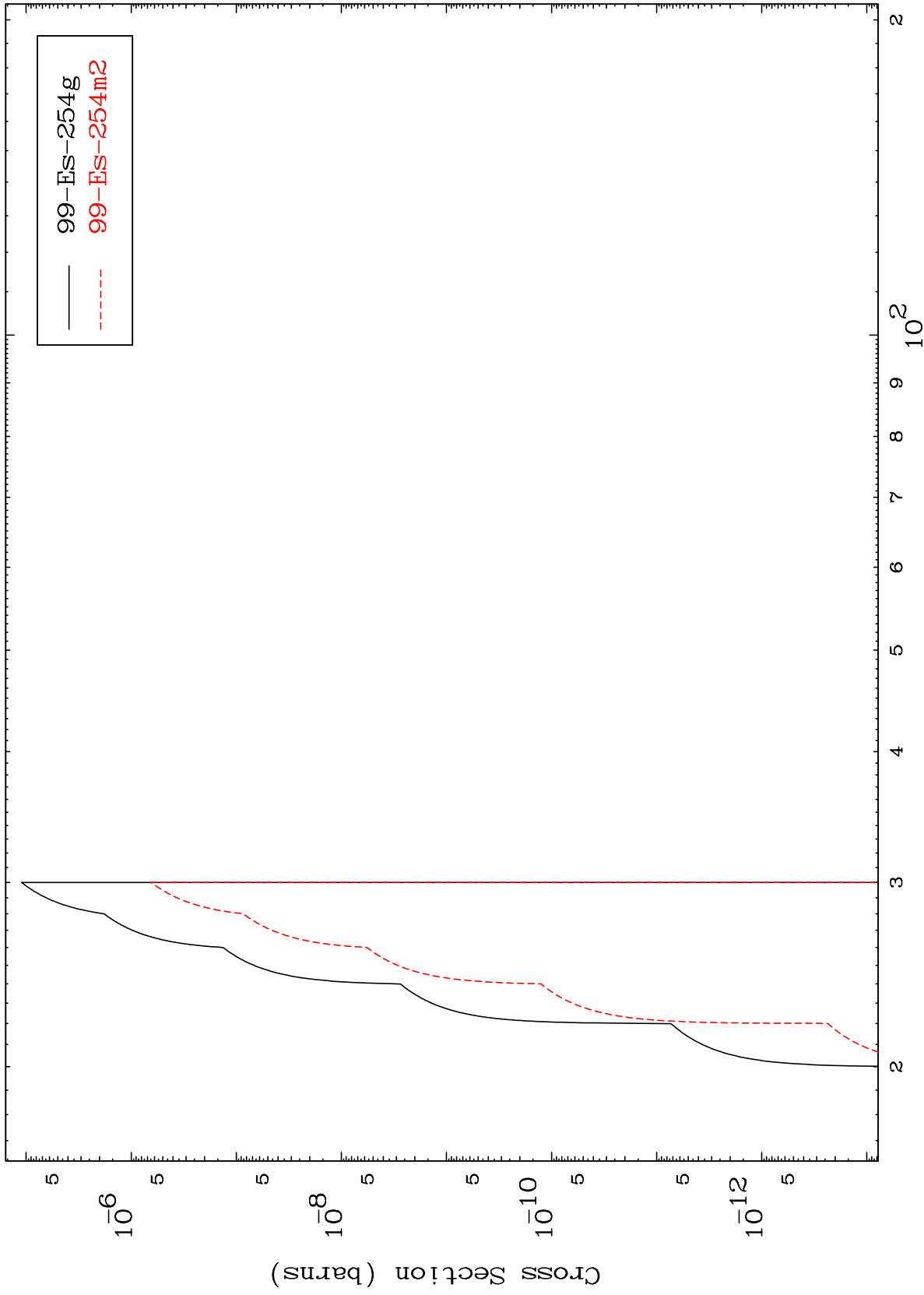
100-Fm-257

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(n,2n) p

100-Fm-257

Radionuclide Production Cross Section



18

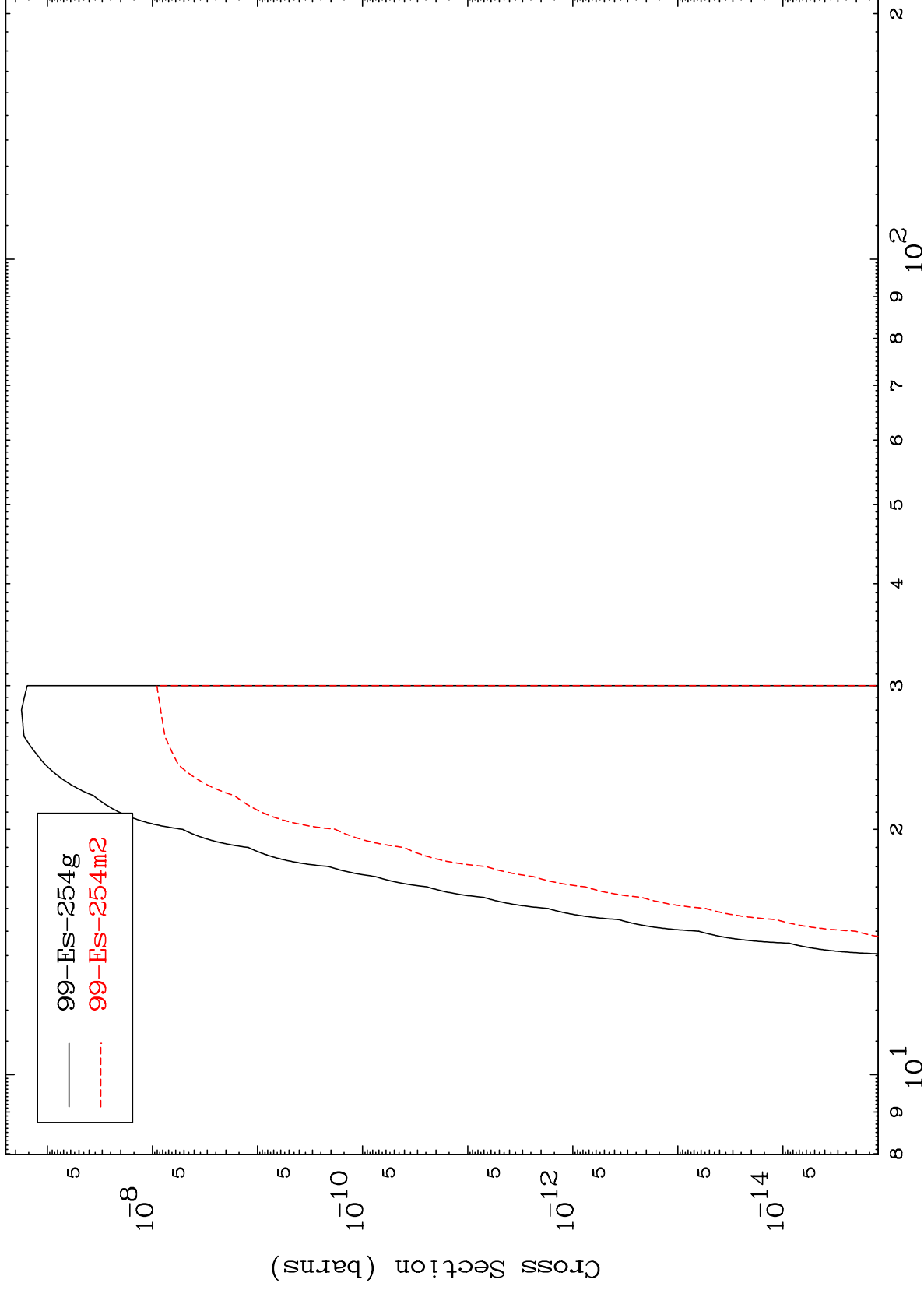
Incident Energy (MeV)

100-Fm-257

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Radionuclide Production Cross Section
(n,t)

100-Fm-257



19

Incident Energy (MeV)

100-Fm-257