

Program EVALPLOT  
(Version 2021-1)

by

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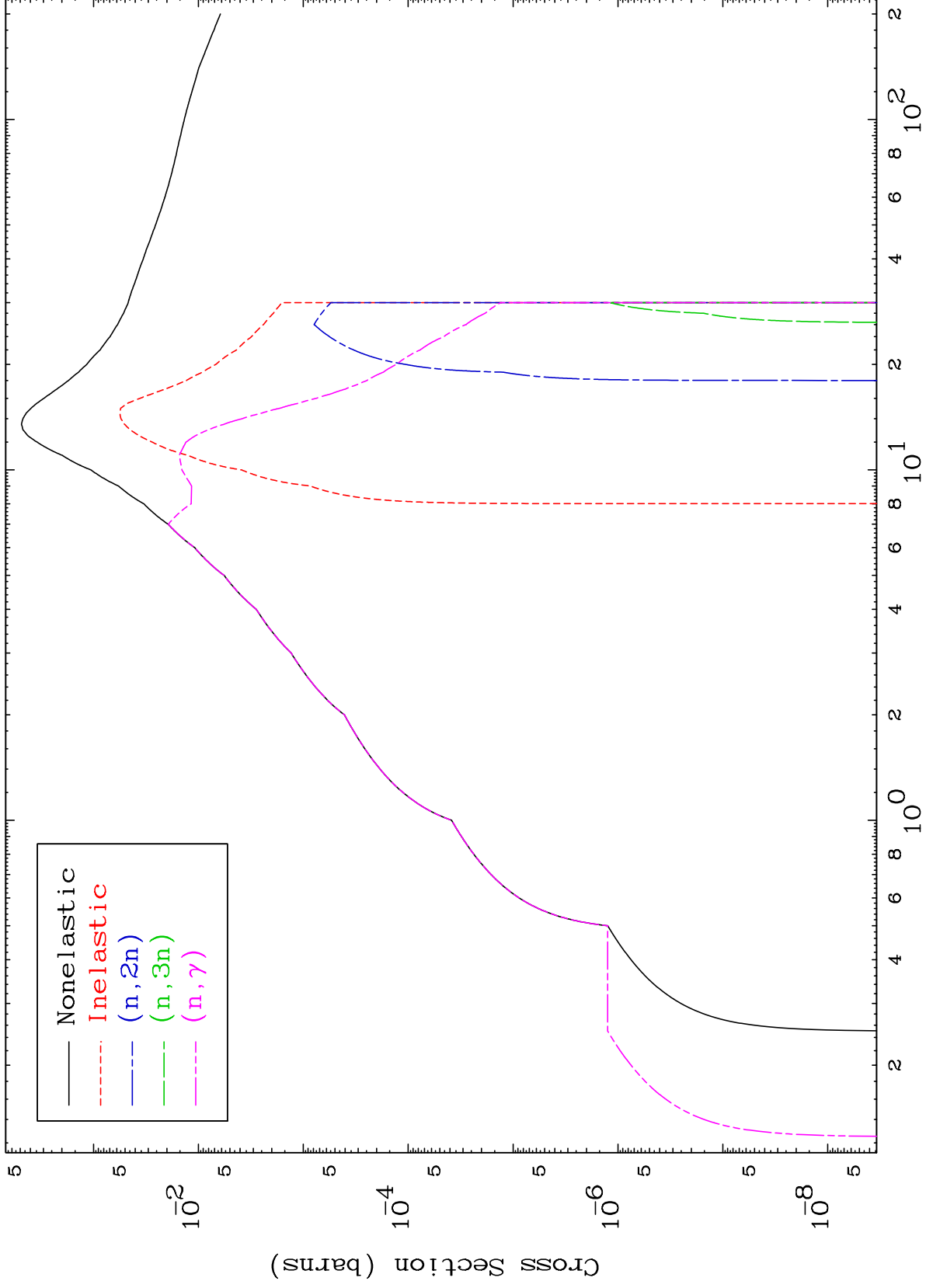
Press Mouse Button to Start

MAT 8707

Photon Major

87-Fr-206

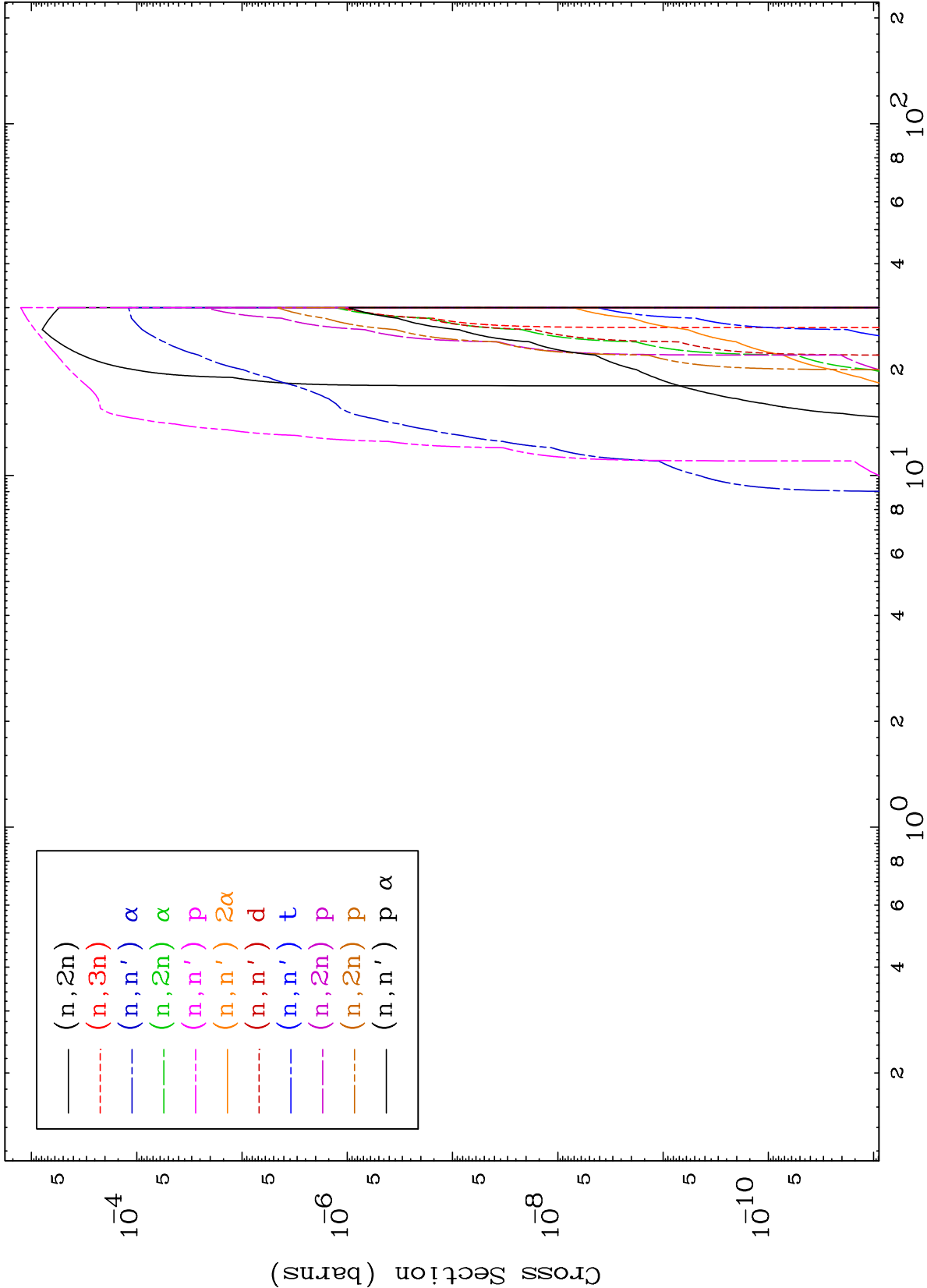
0 Kelvin Cross Sections



1

Incident Energy (MeV)

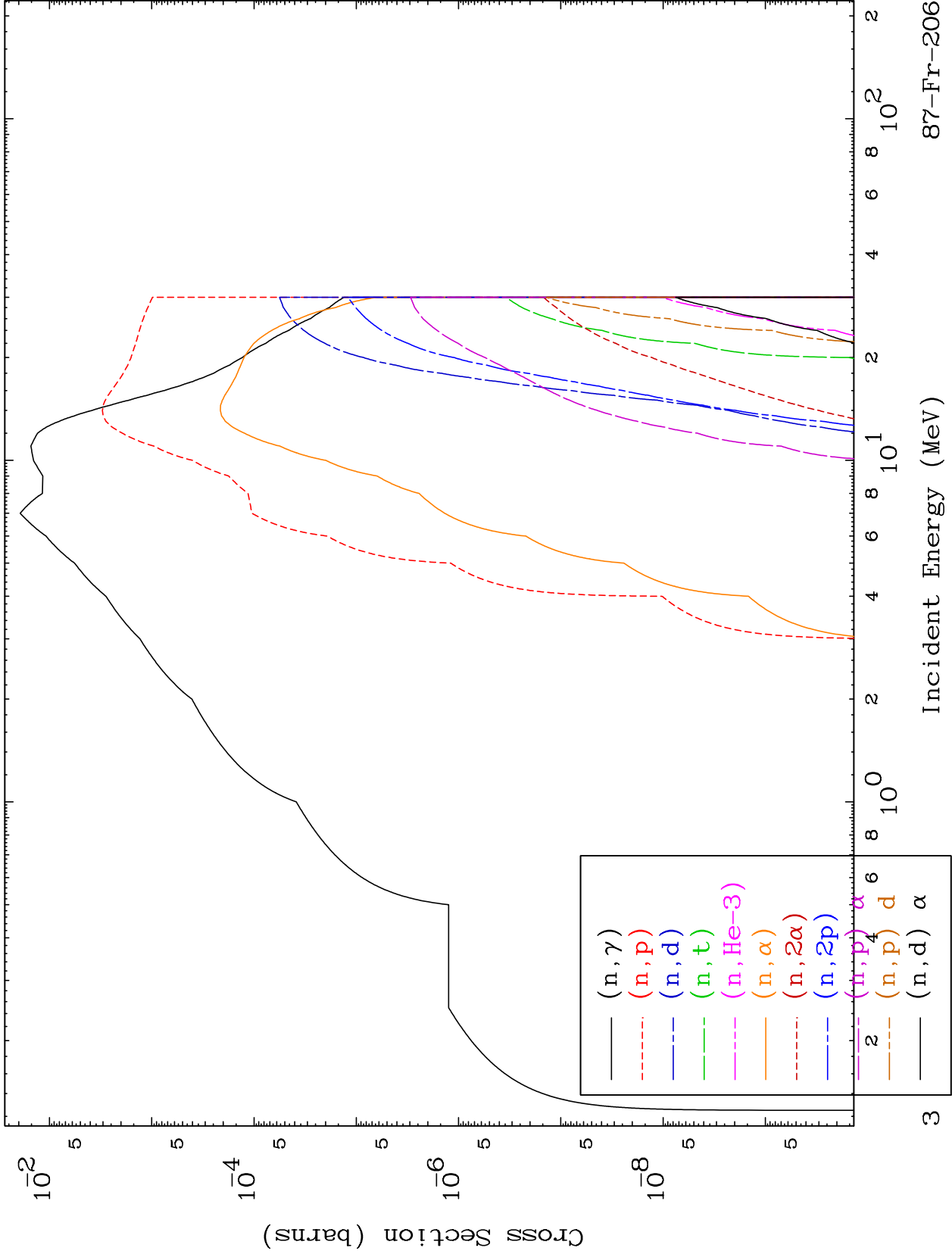
87-Fr-206

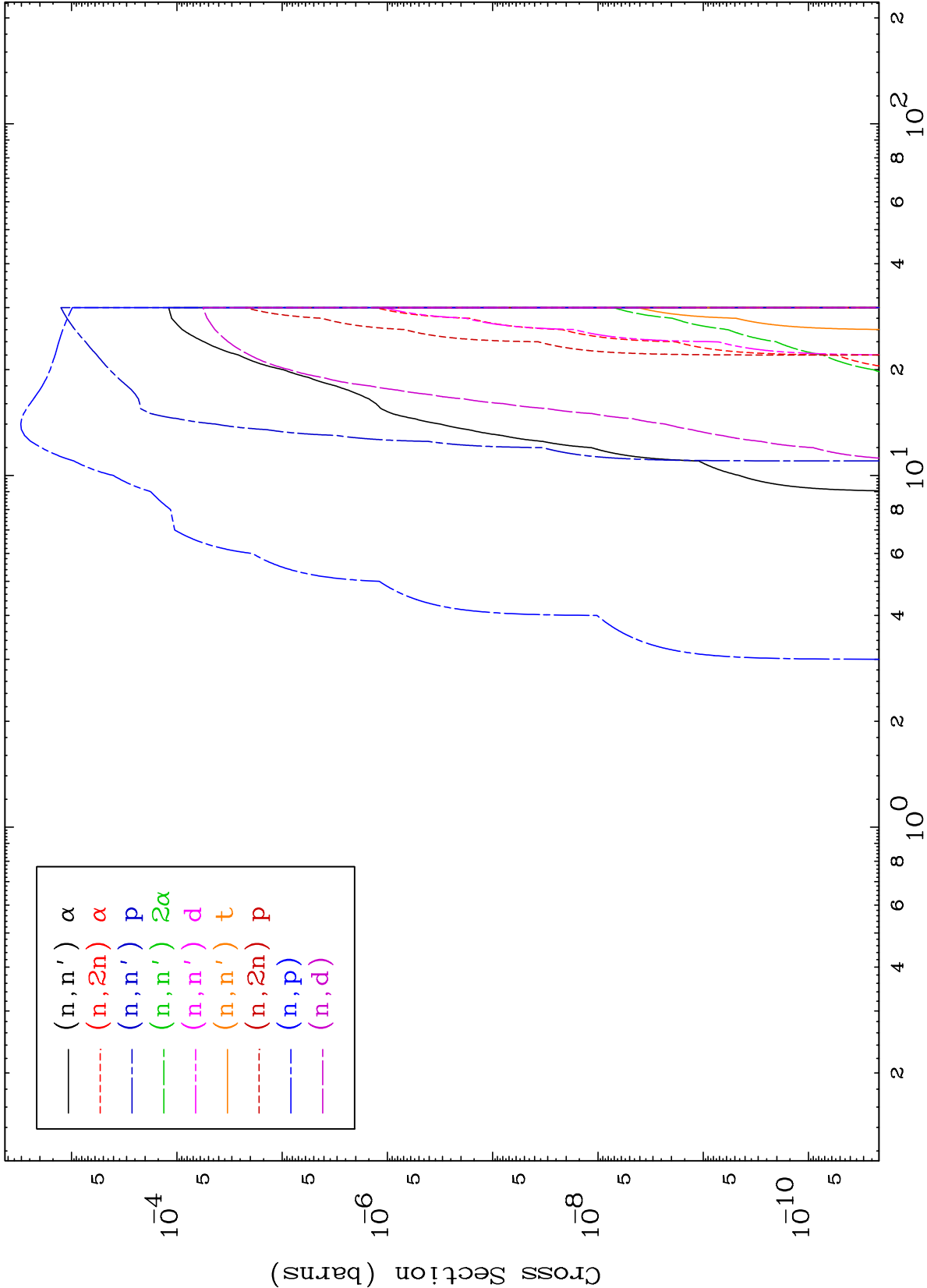


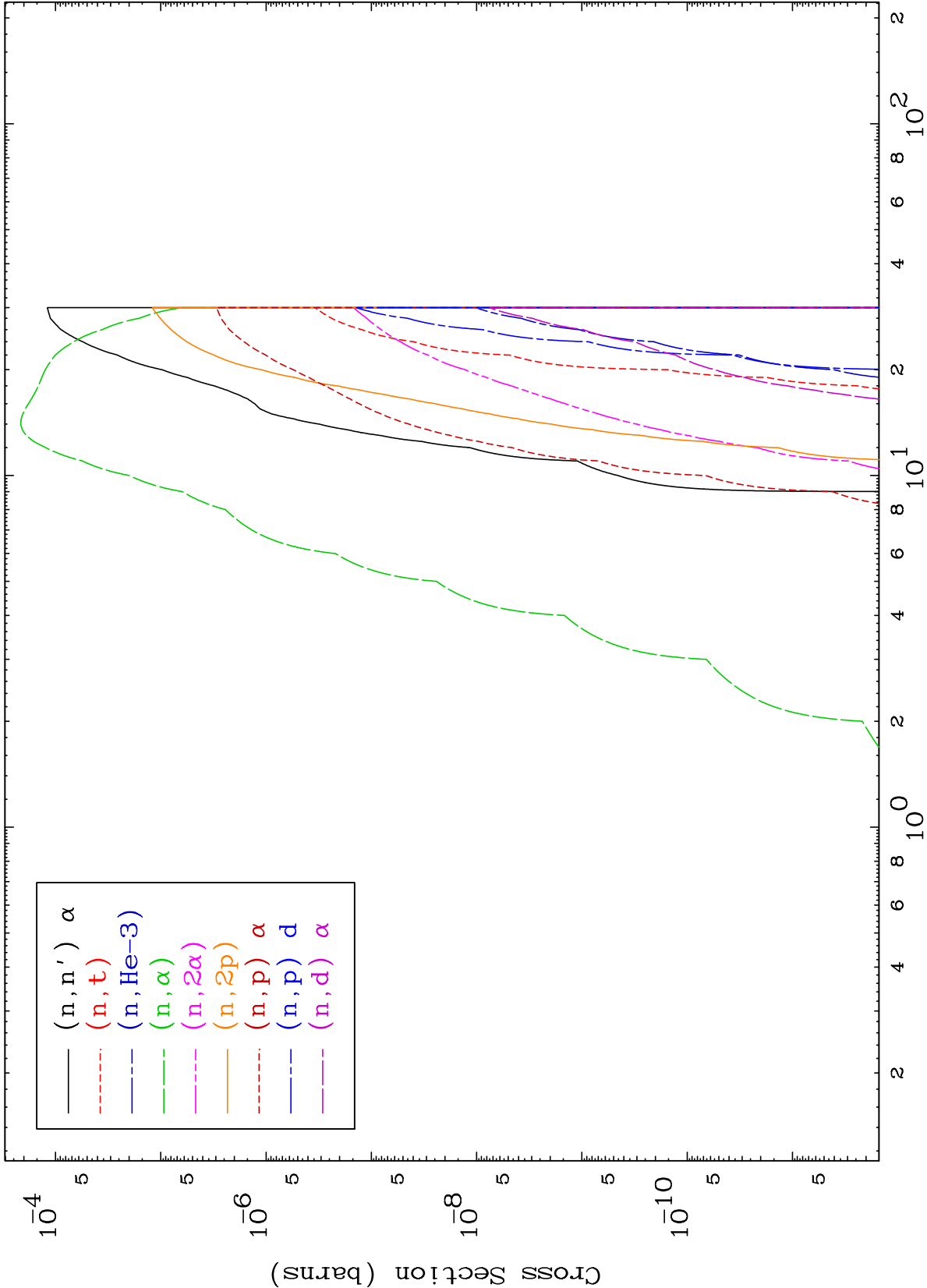
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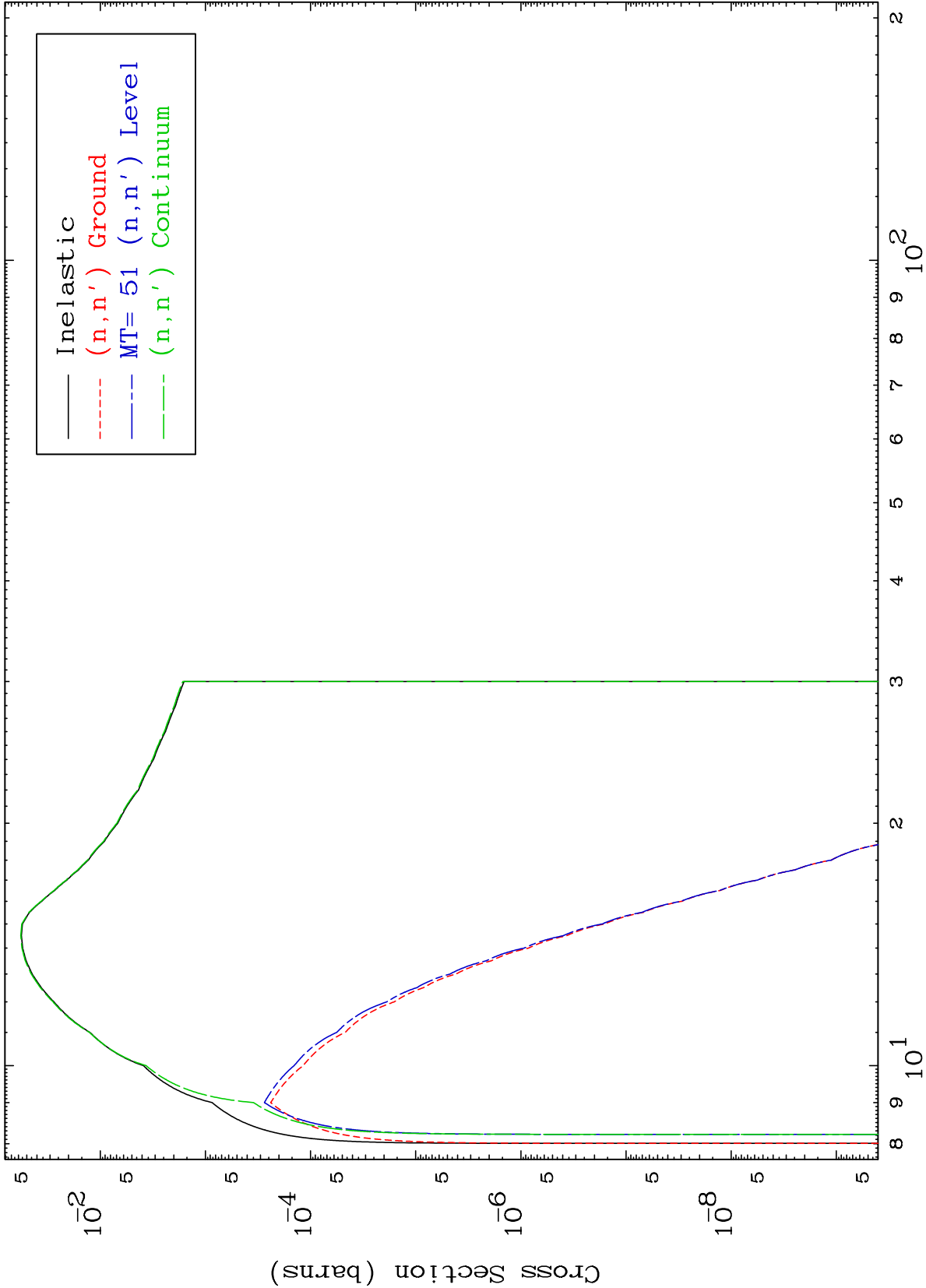
Photon Neutron Absorption  
0 Kelvin Cross Sections

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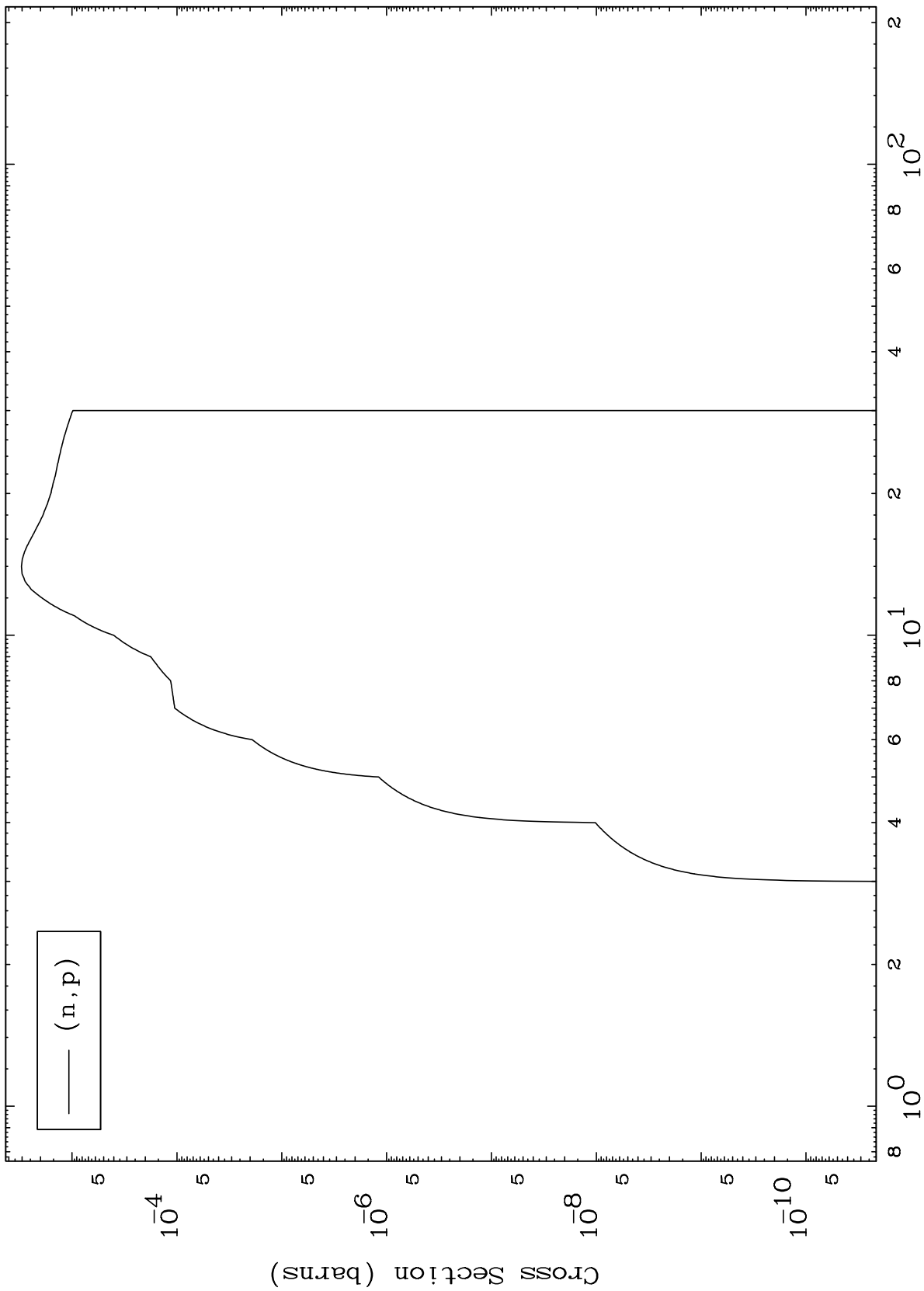




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( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections



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Incident Energy (MeV)

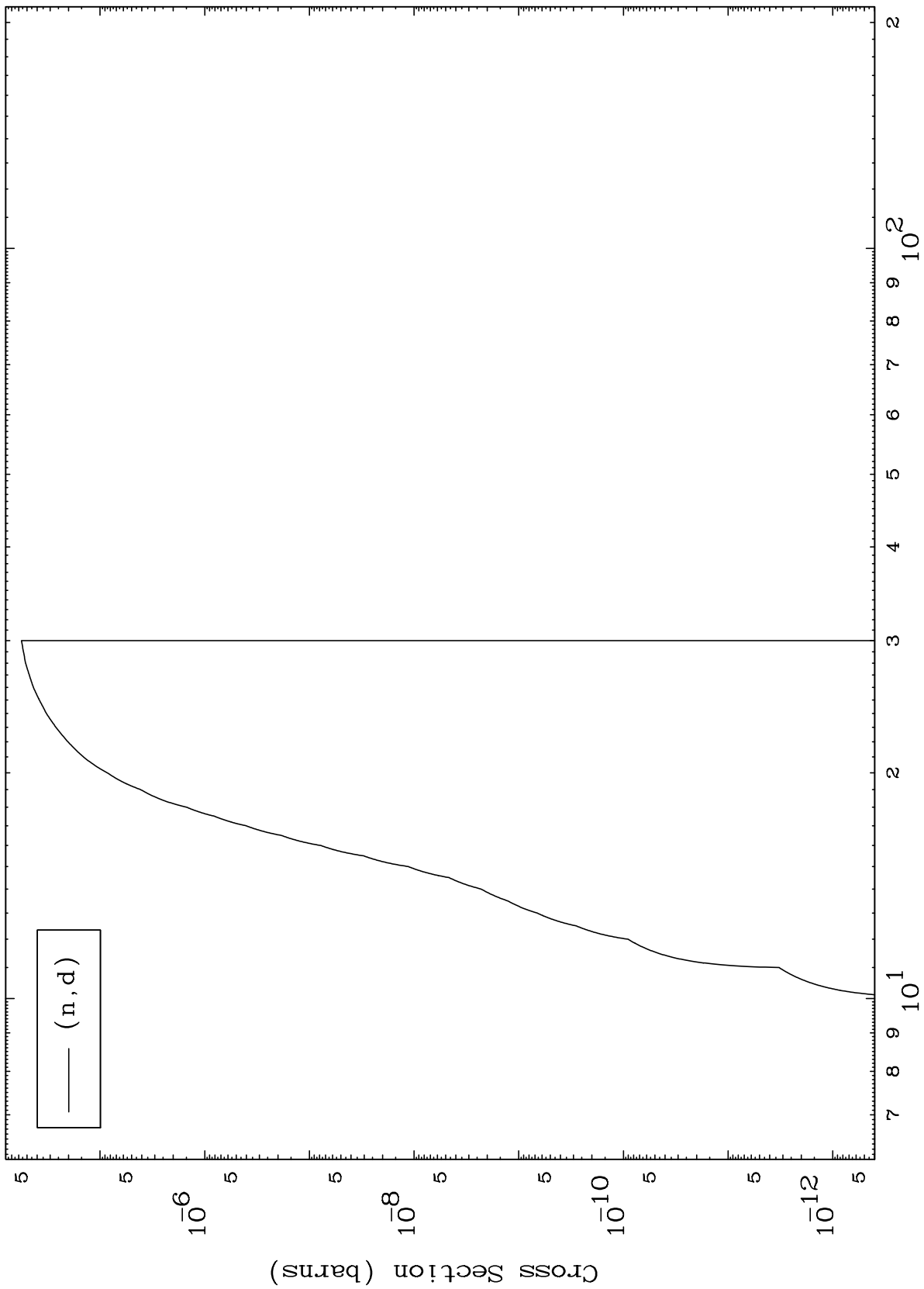
7



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( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections



8

Incident Energy (MeV)

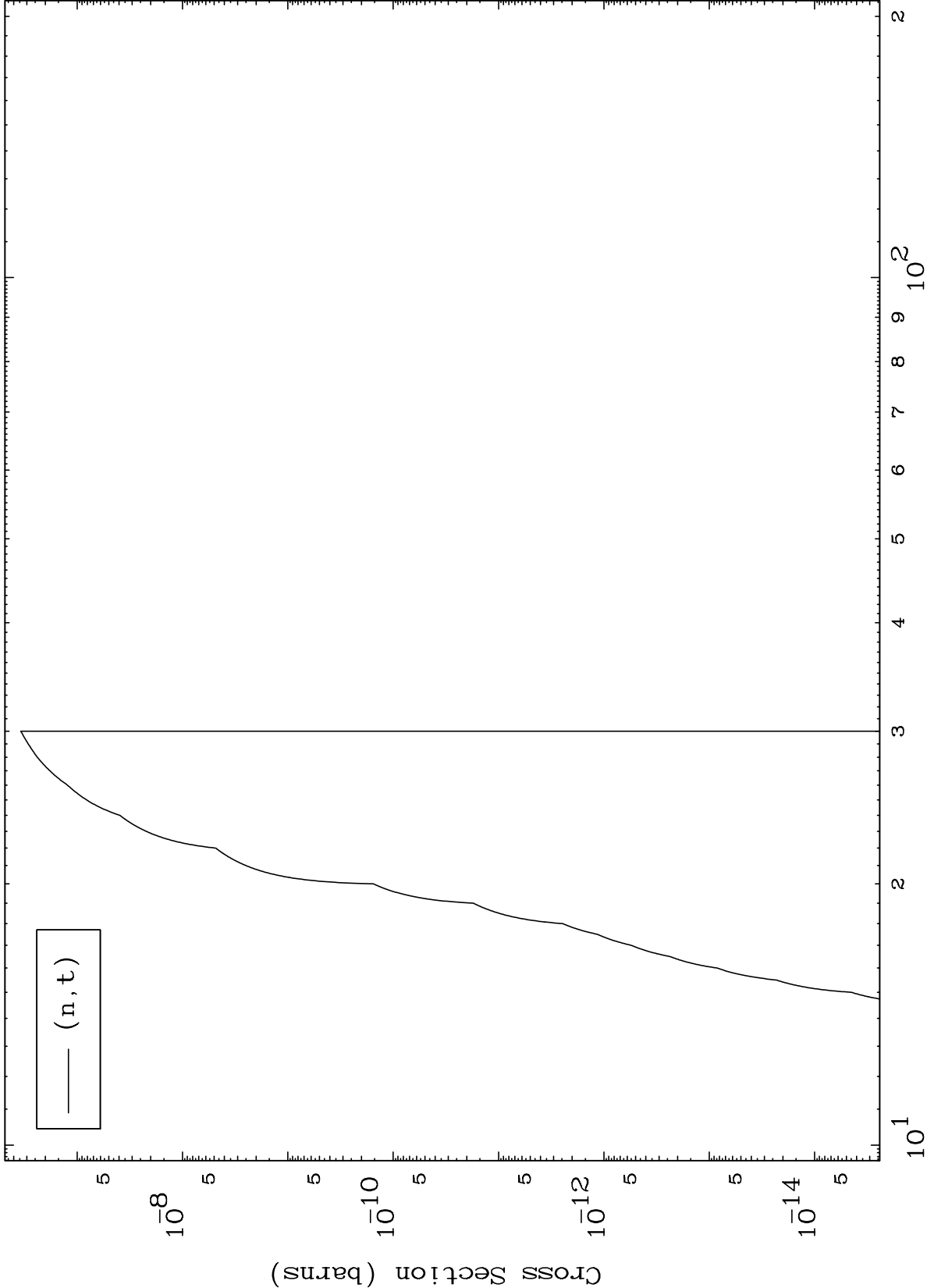
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( $\gamma, t$ ) Levels

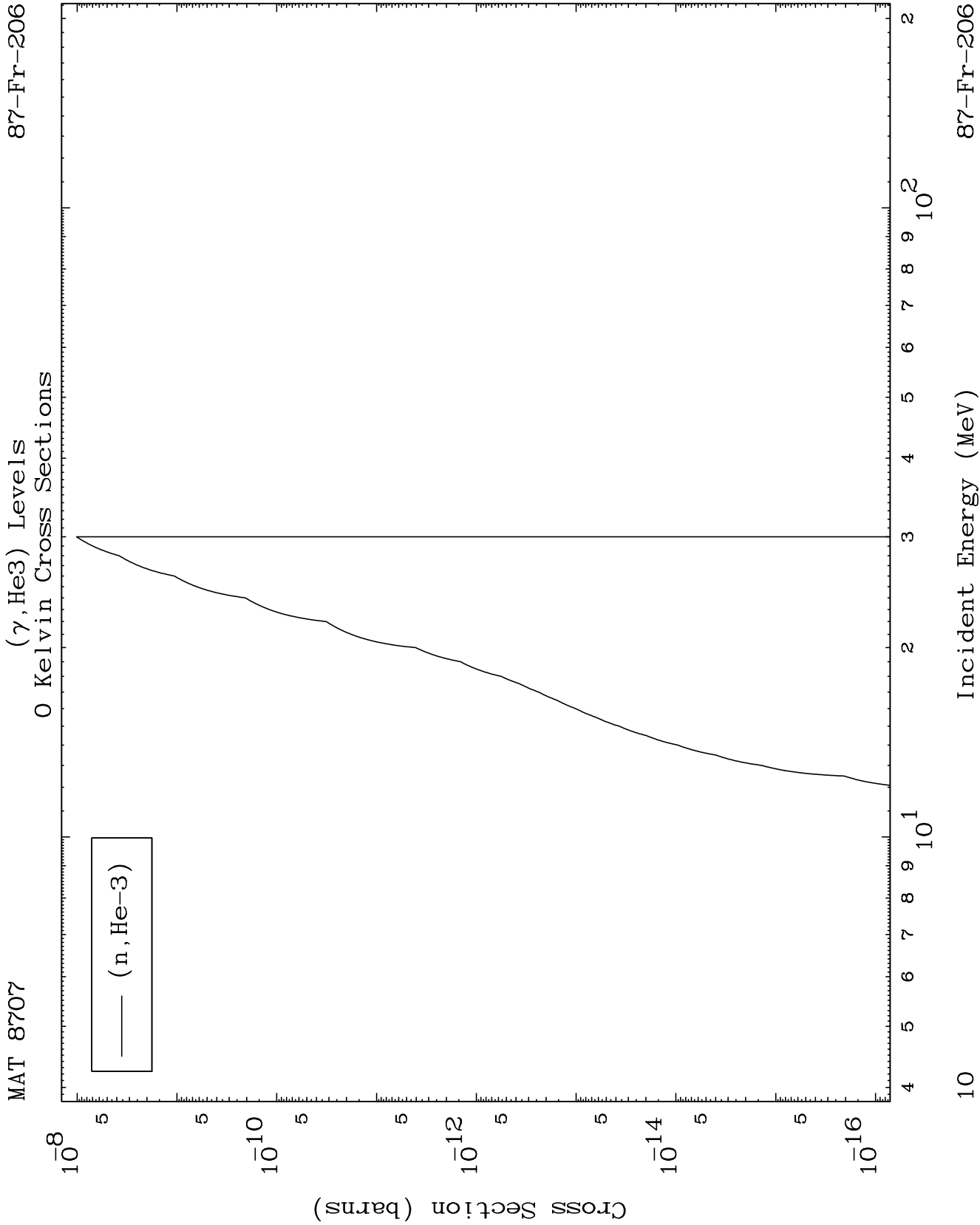
0 Kelvin Cross Sections

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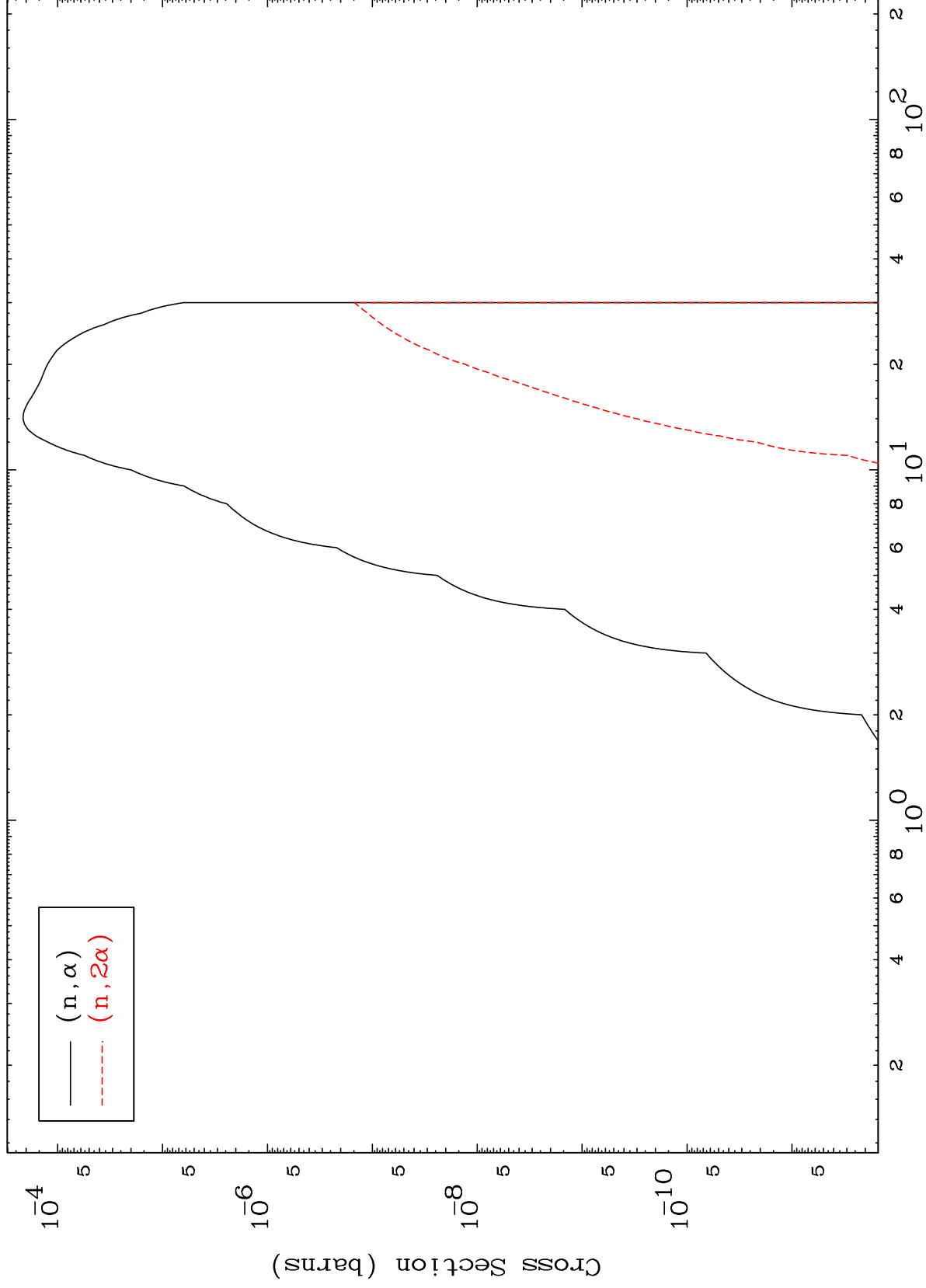


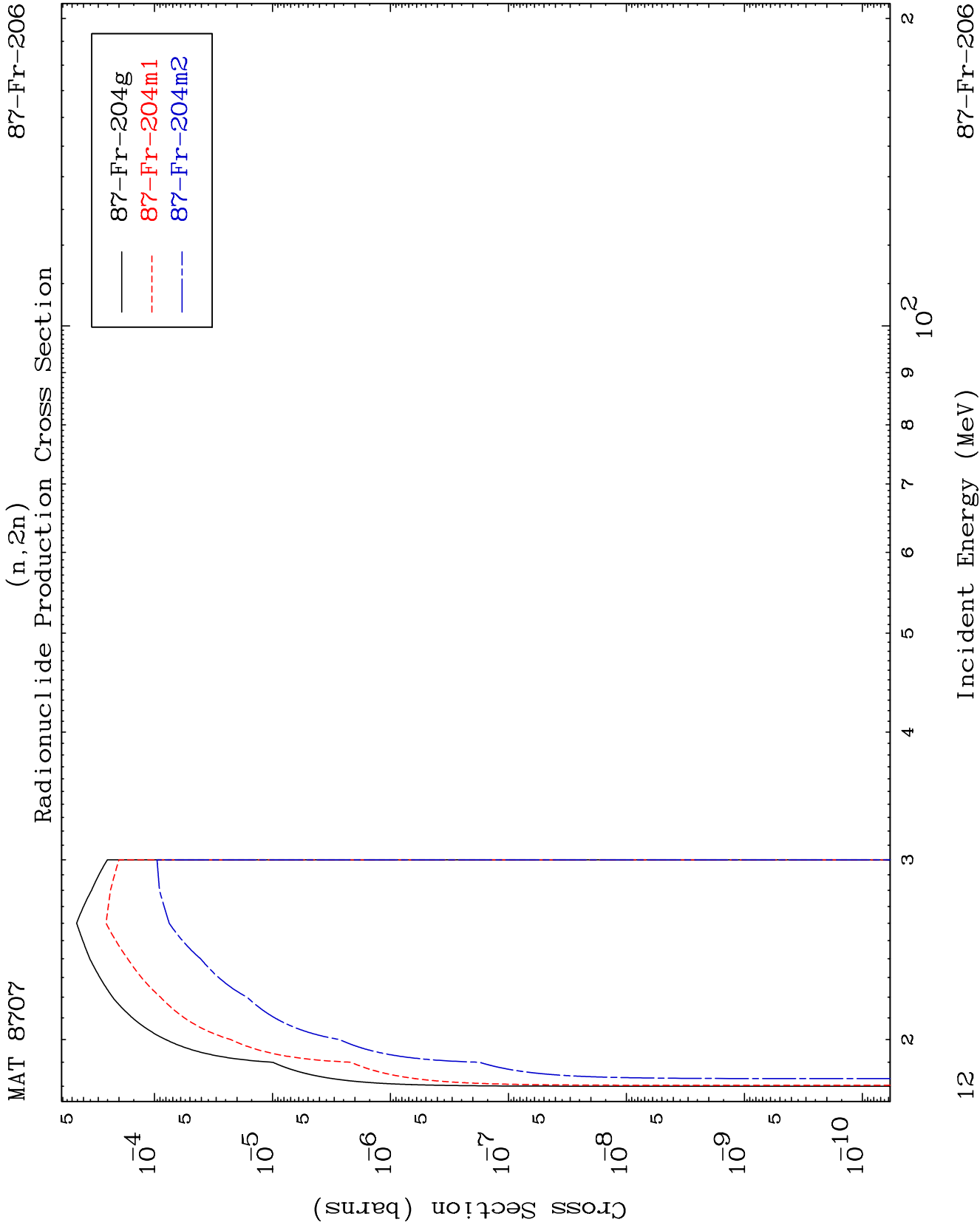
Incident Energy (MeV)

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( $\gamma, \alpha$ ) Levels  
0 Kelvin Cross Sections



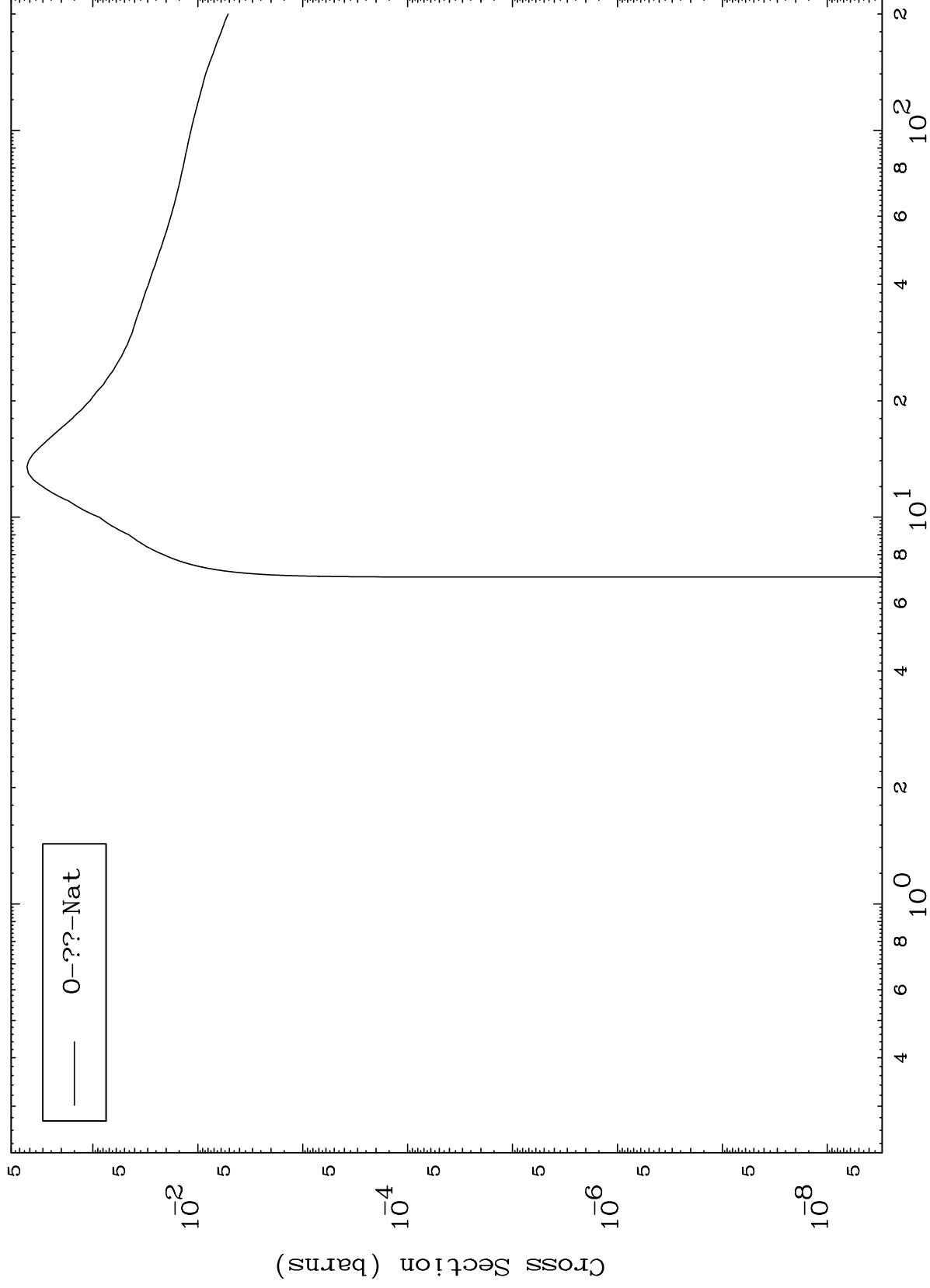


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Fission

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Radionuclide Production Cross Section



13

Incident Energy (MeV)

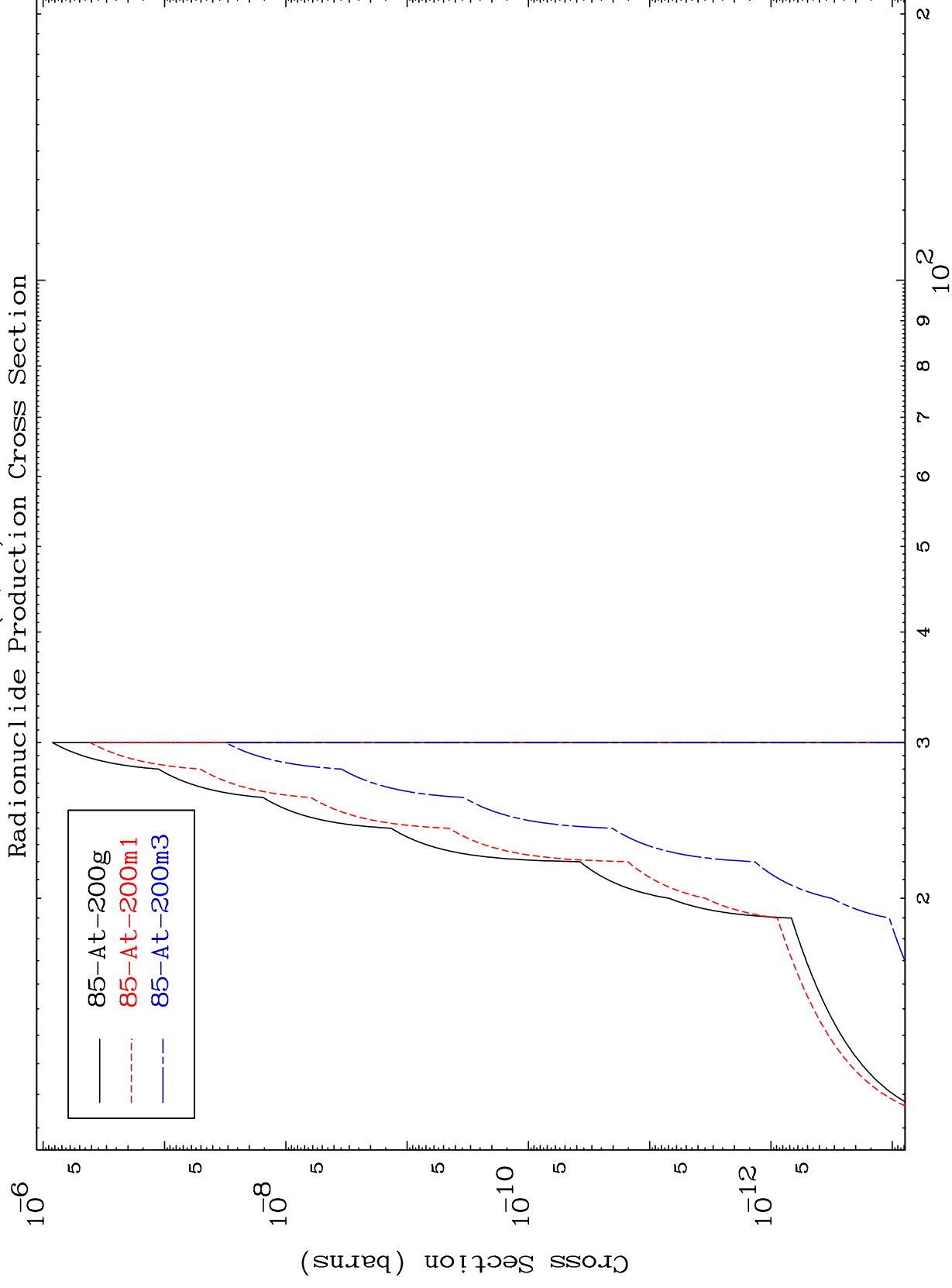
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$$(n, 2n) \propto$$

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Radionuclide Production Cross Section



14

Incident Energy (MeV)

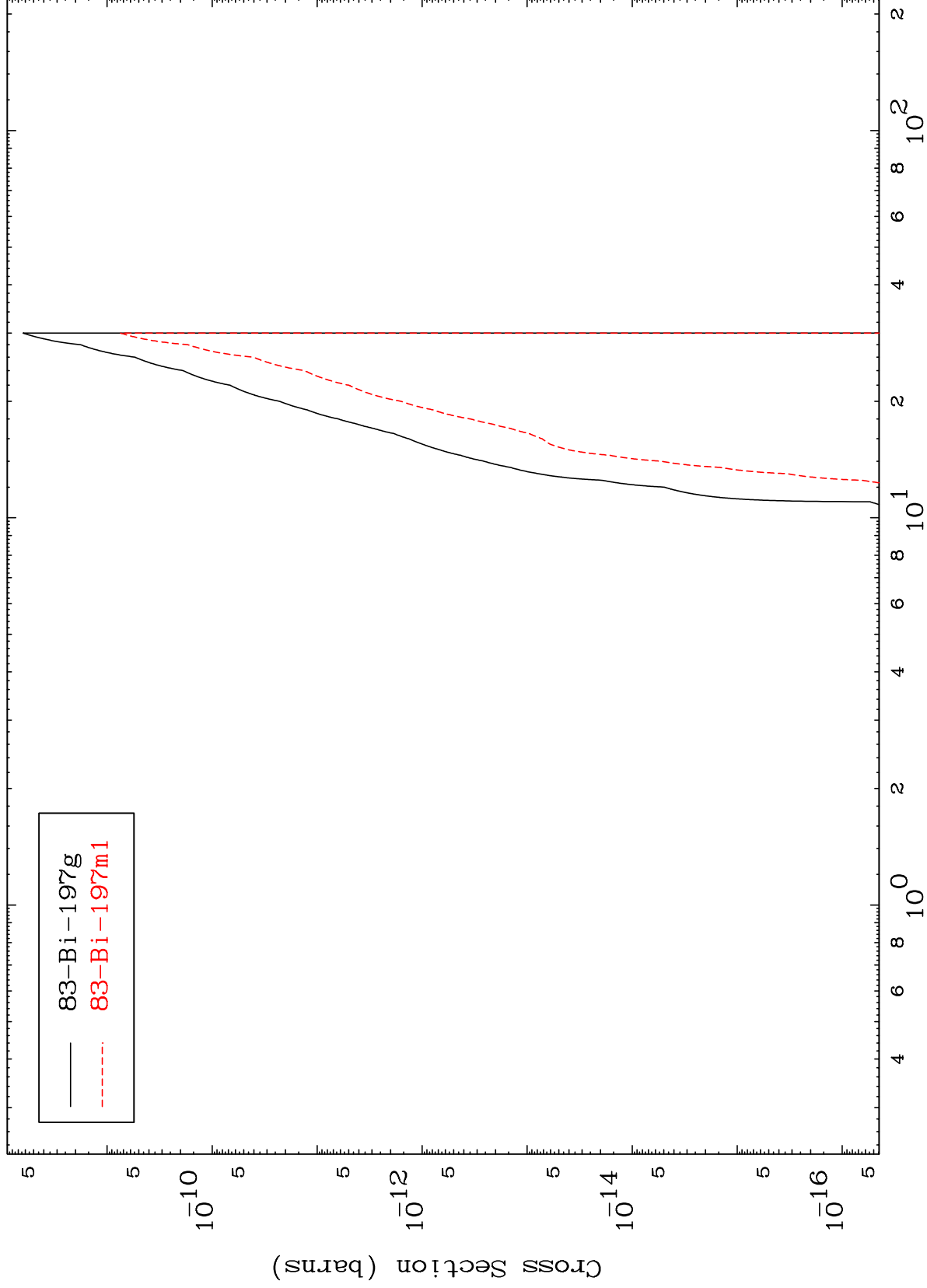
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(n,n') 2 $\alpha$

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Radionuclide Production Cross Section



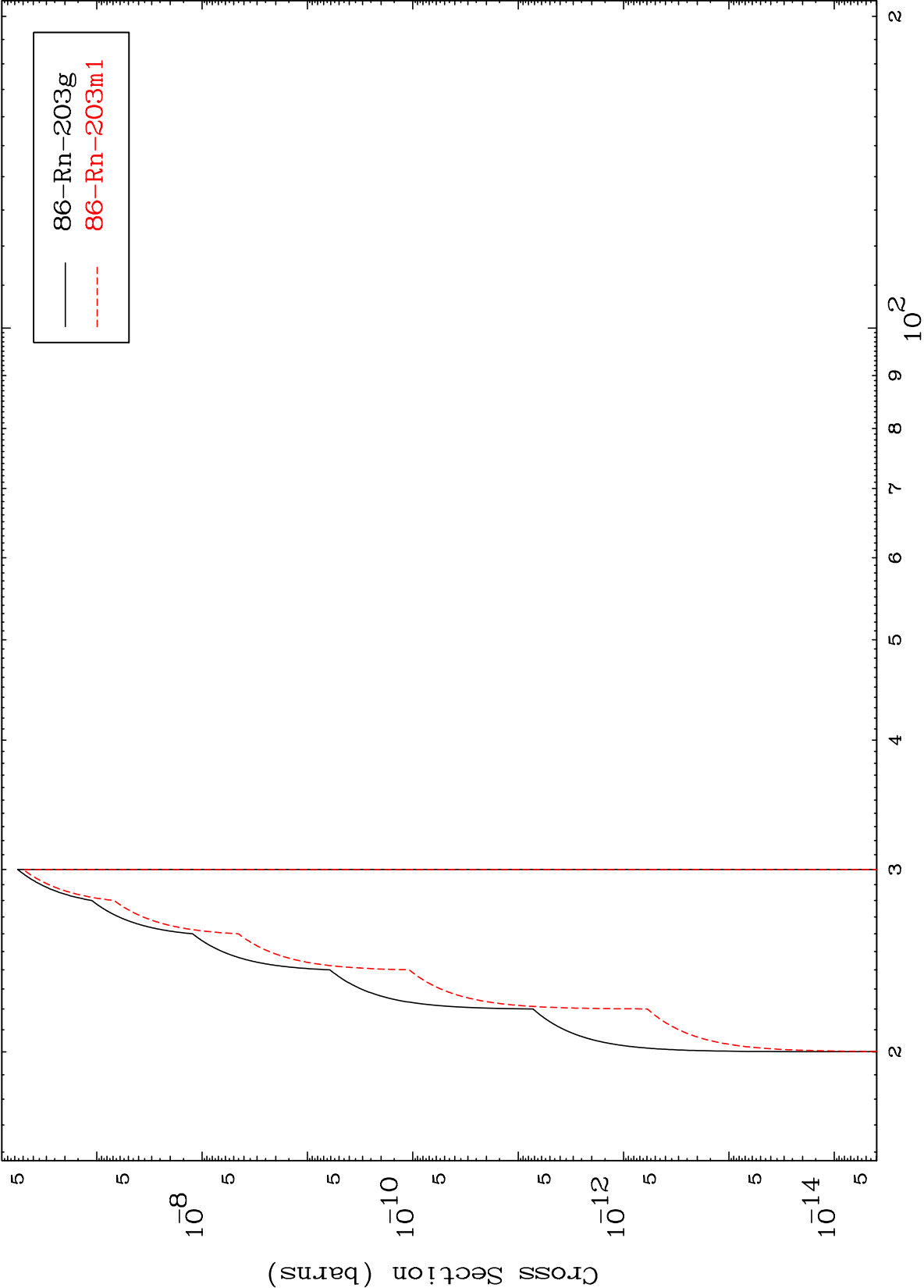
15

Incident Energy (MeV)

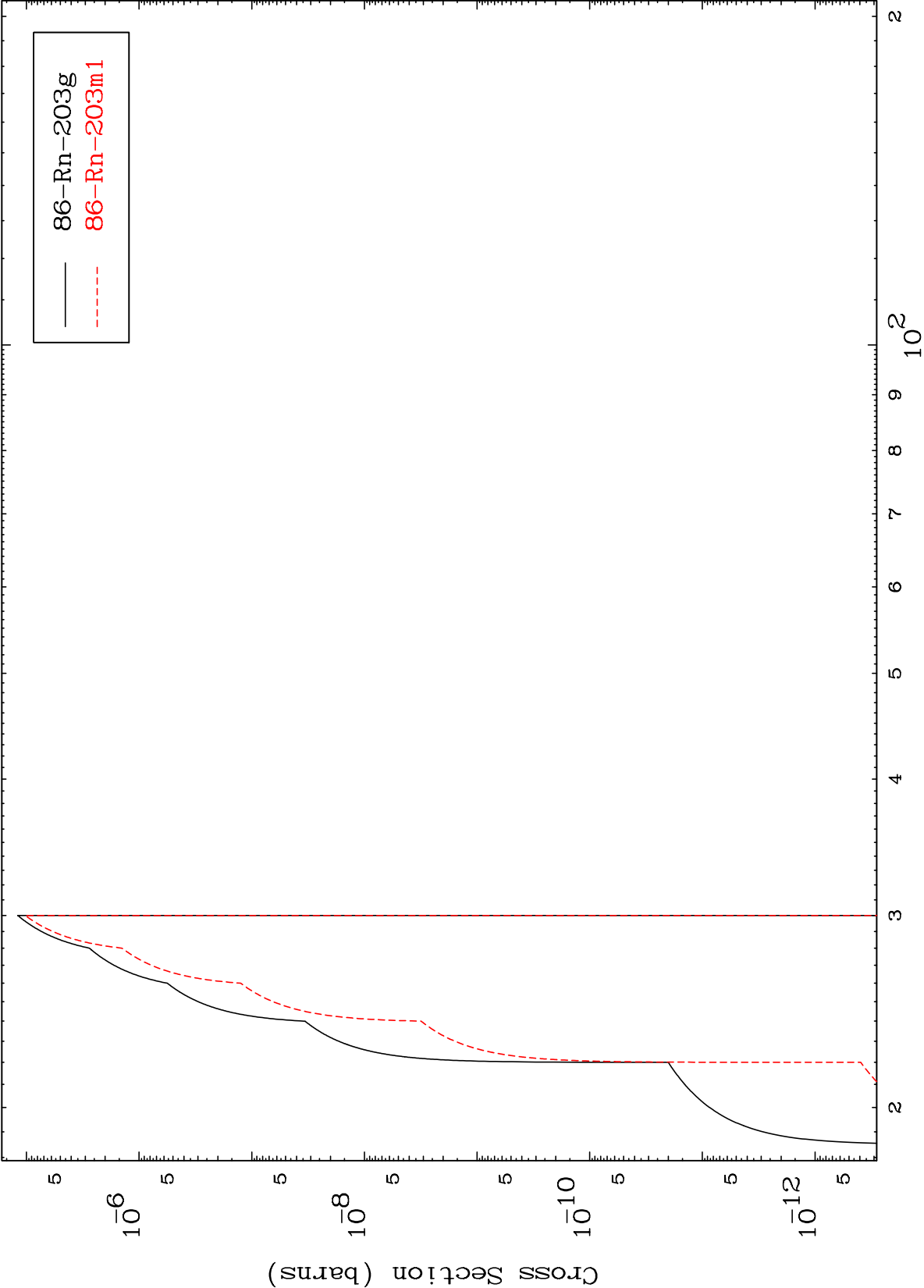
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(n,n') d  
Radionuclide Production Cross Section



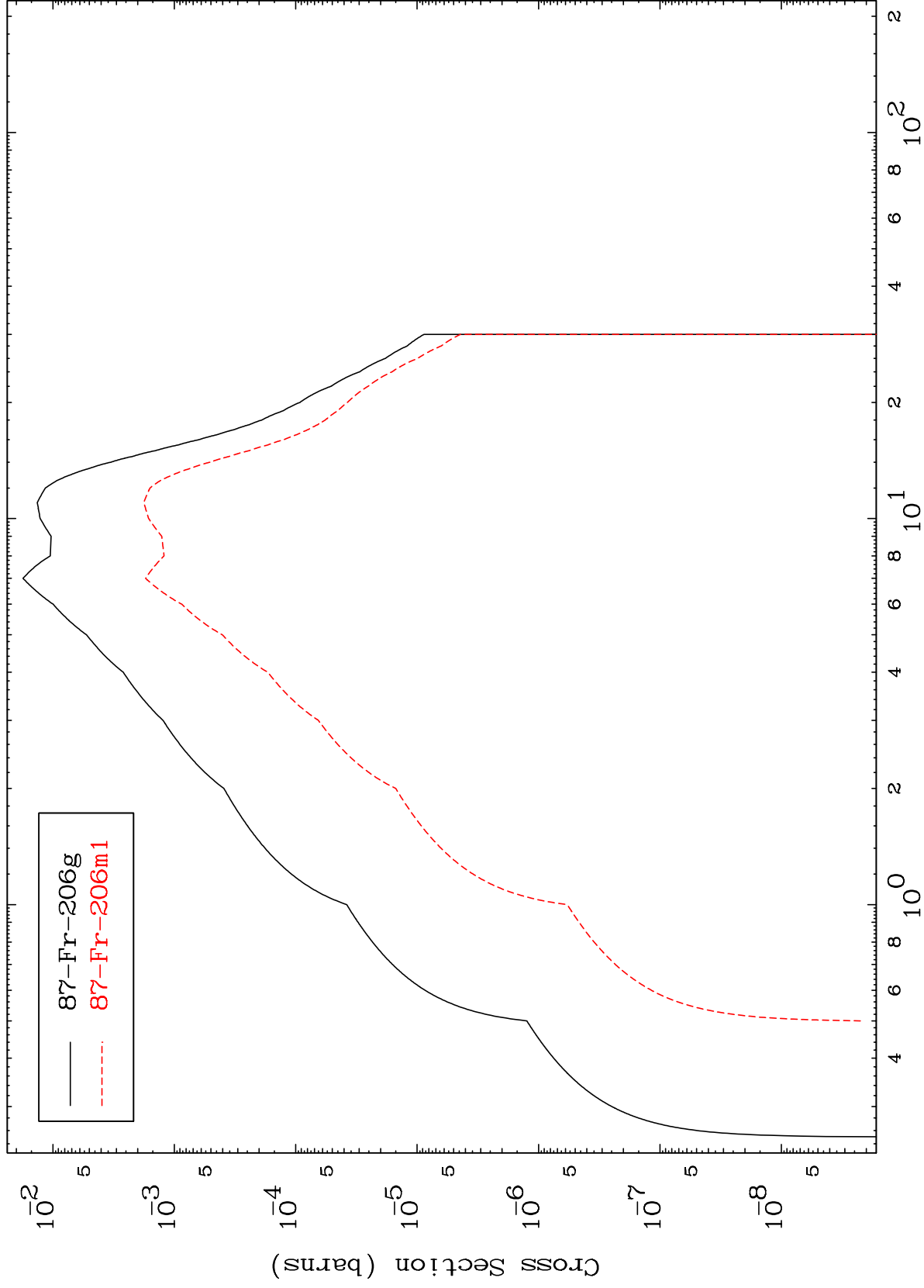
(n,2n) p  
Radionuclide Production Cross Section



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Radionuclide Production Cross Section



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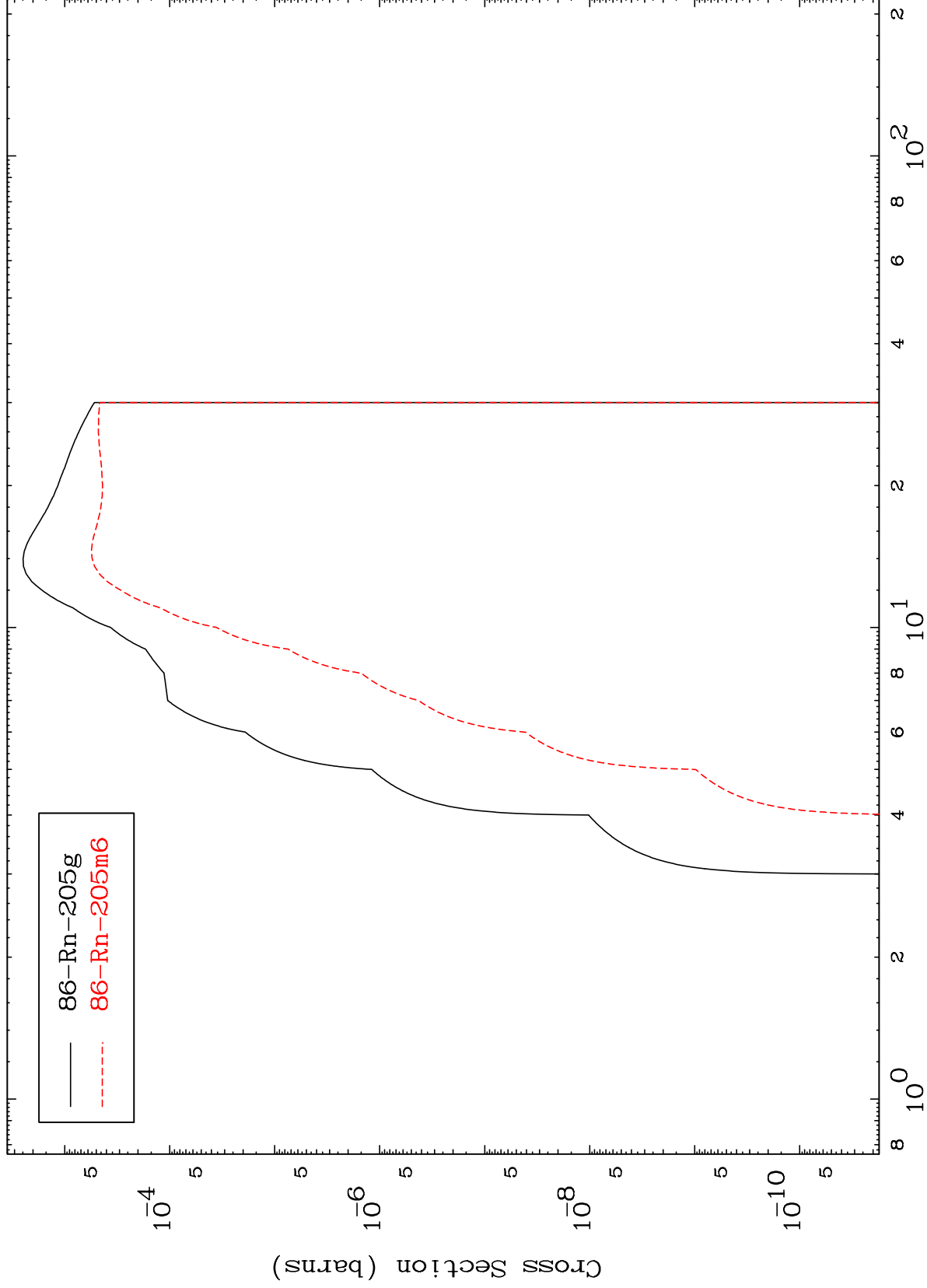
Incident Energy (MeV)

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Radionuclide Production Cross Section



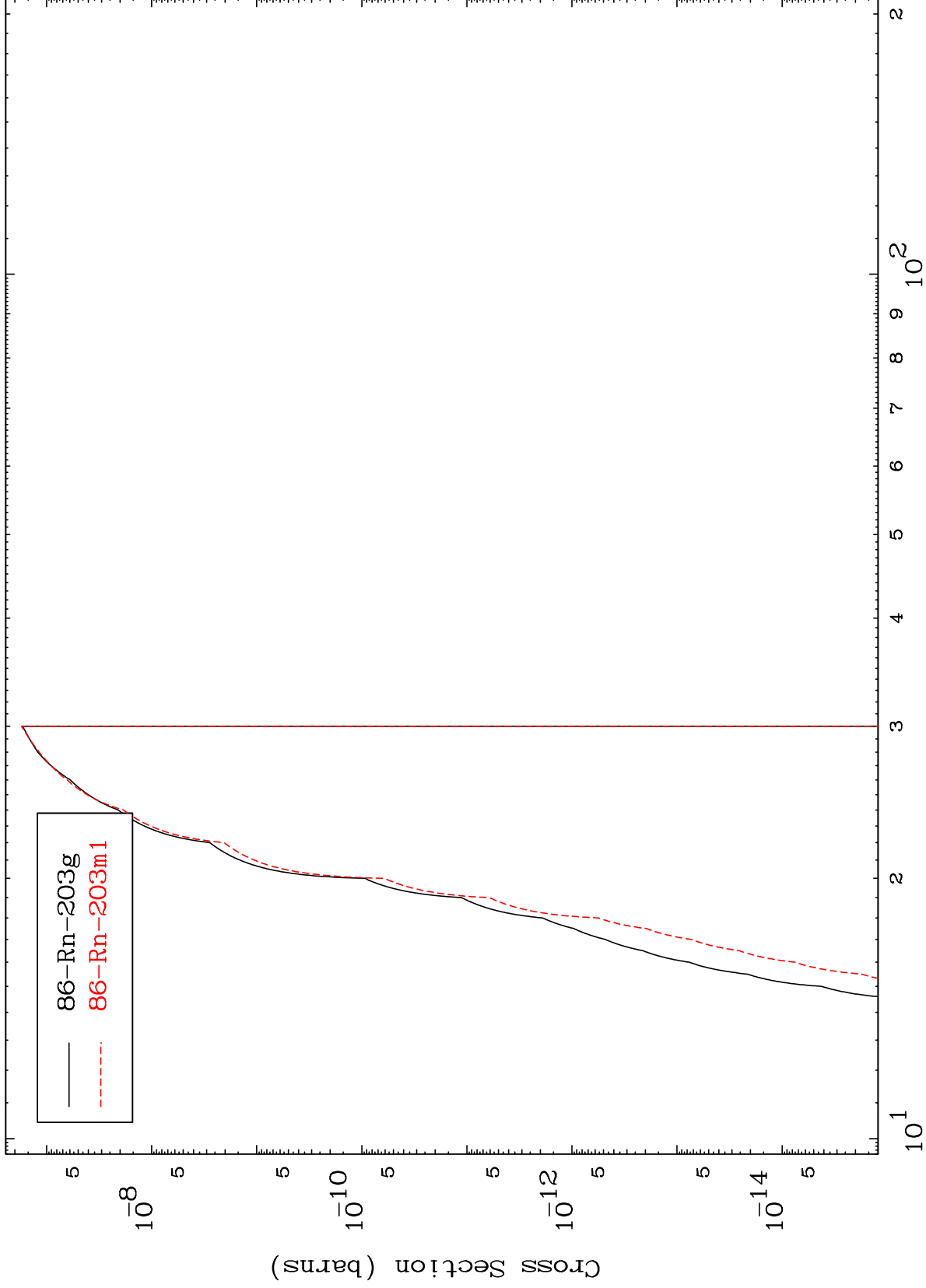
Incident Energy (MeV)

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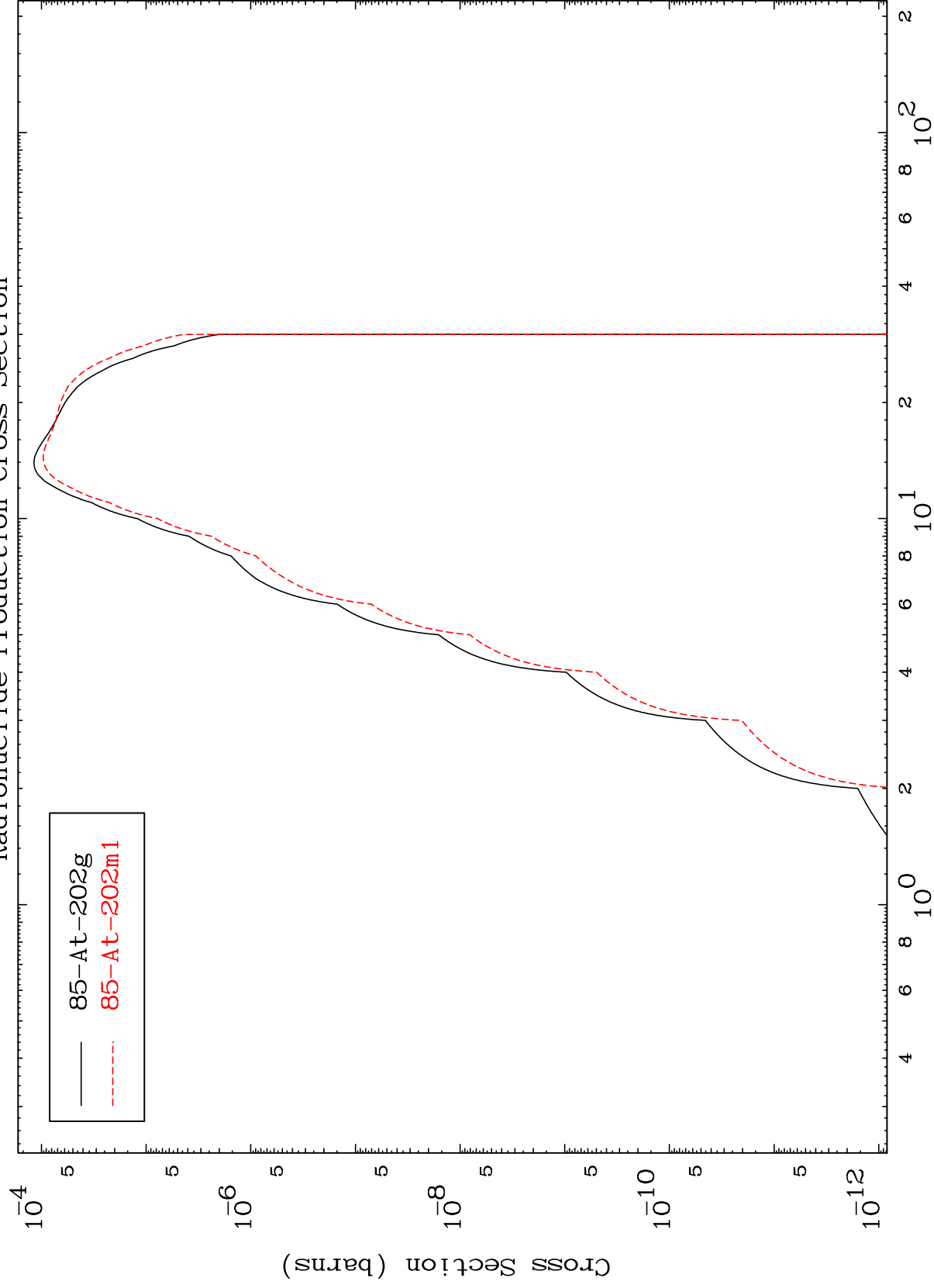
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Radionuclide Production Cross Section

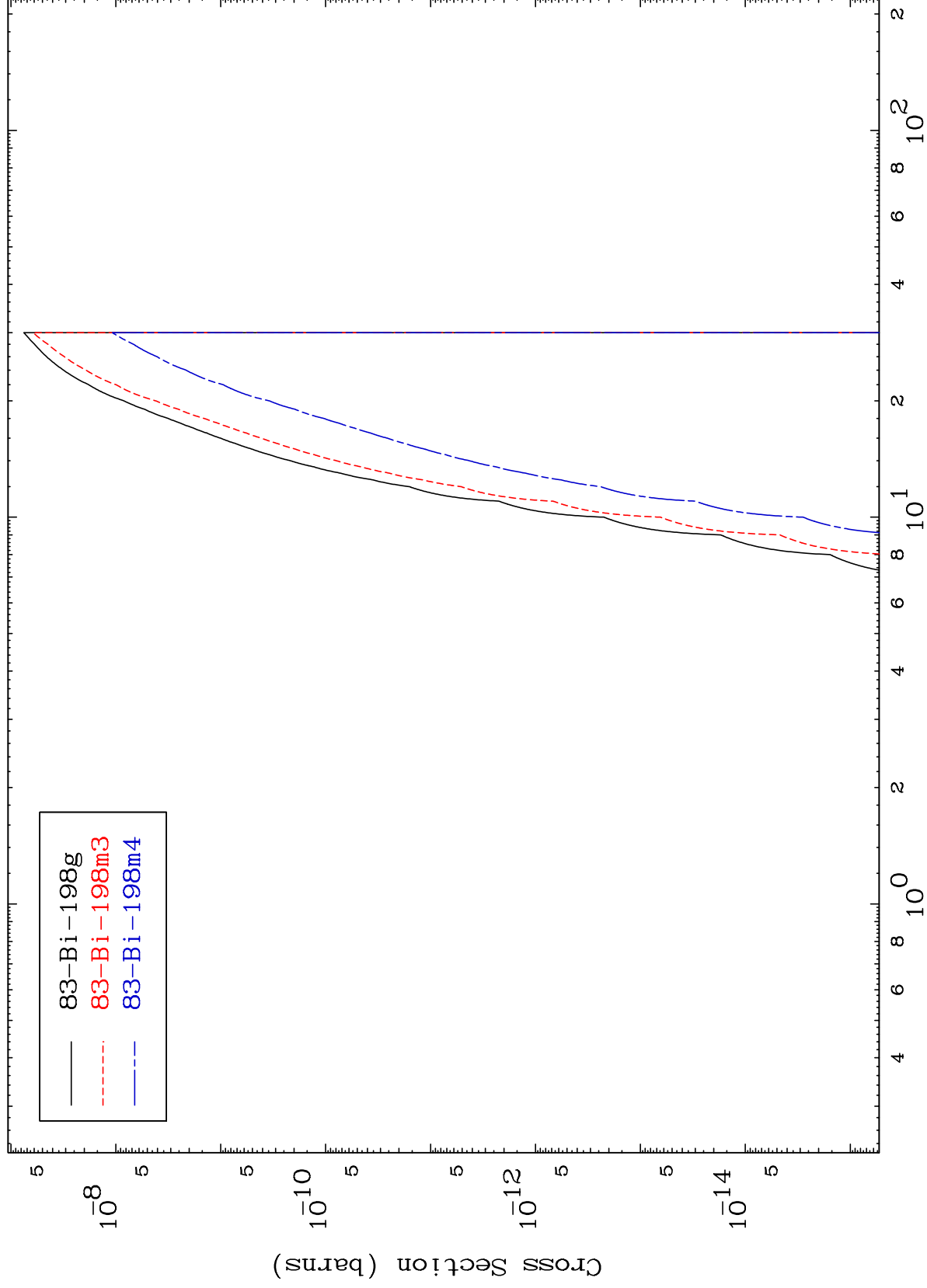


87-Fr-206

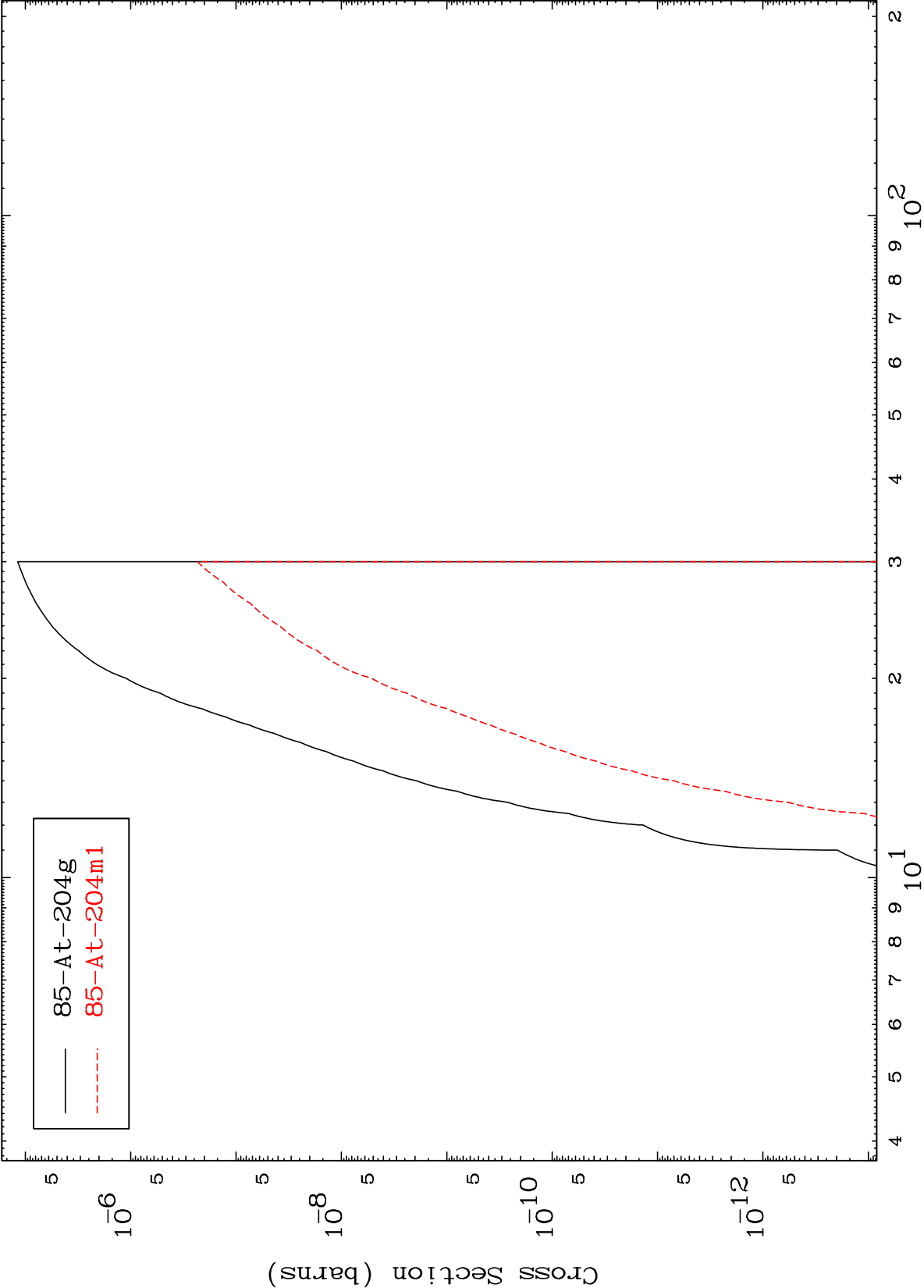
Incident Energy (MeV)

(n,  $\alpha$ )  
Radionuclide Production Cross Section

Radionuclide Production Cross Section  
(n,2α)



Radionuclide Production Cross Section  
(n,2p)





(n,p)  $\alpha$   
Radionuclide Production Cross Section

