

Program EVALPLOT
(Version 2021-1)

by

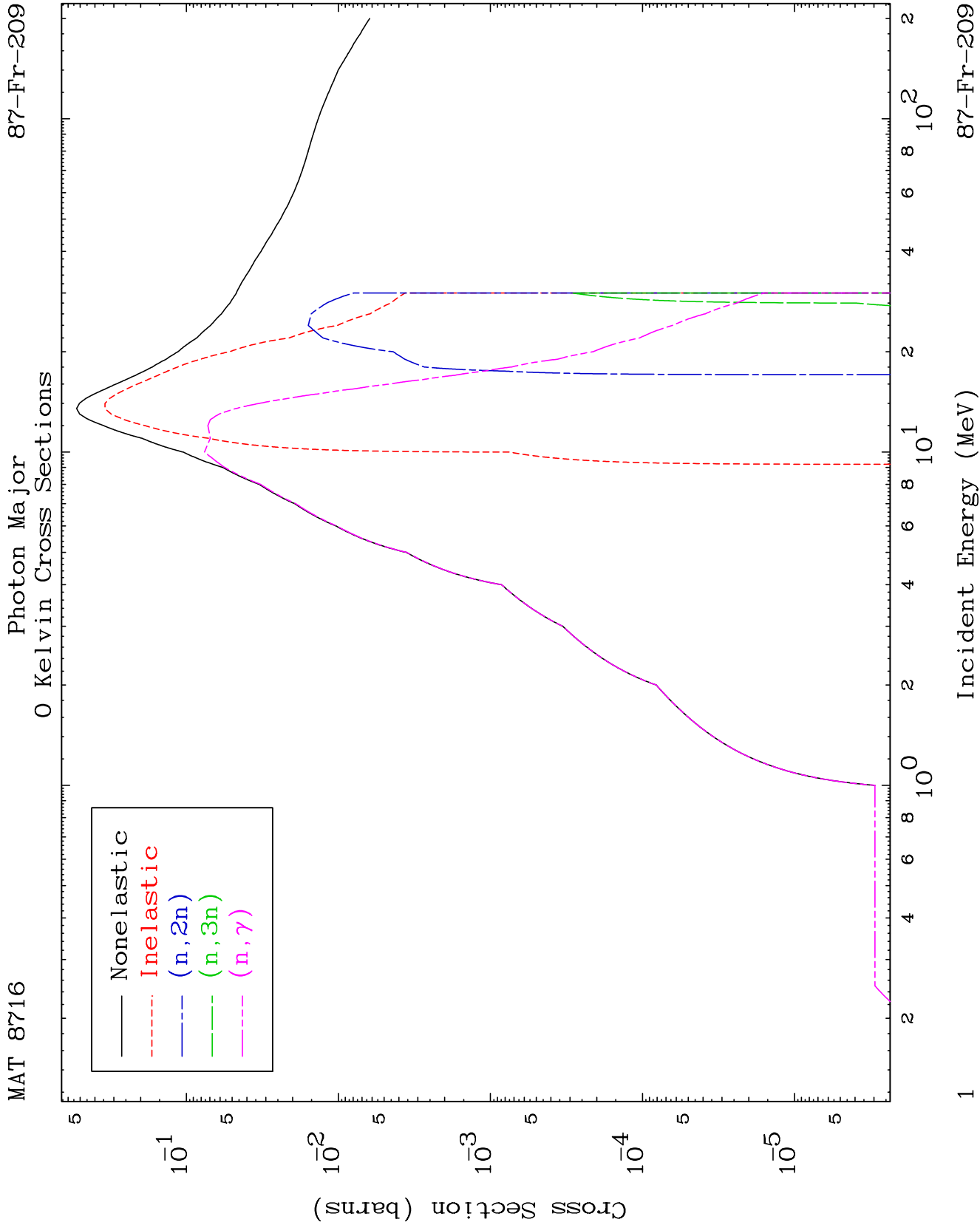
Dermott E. Cullen
(Present Contact Information)

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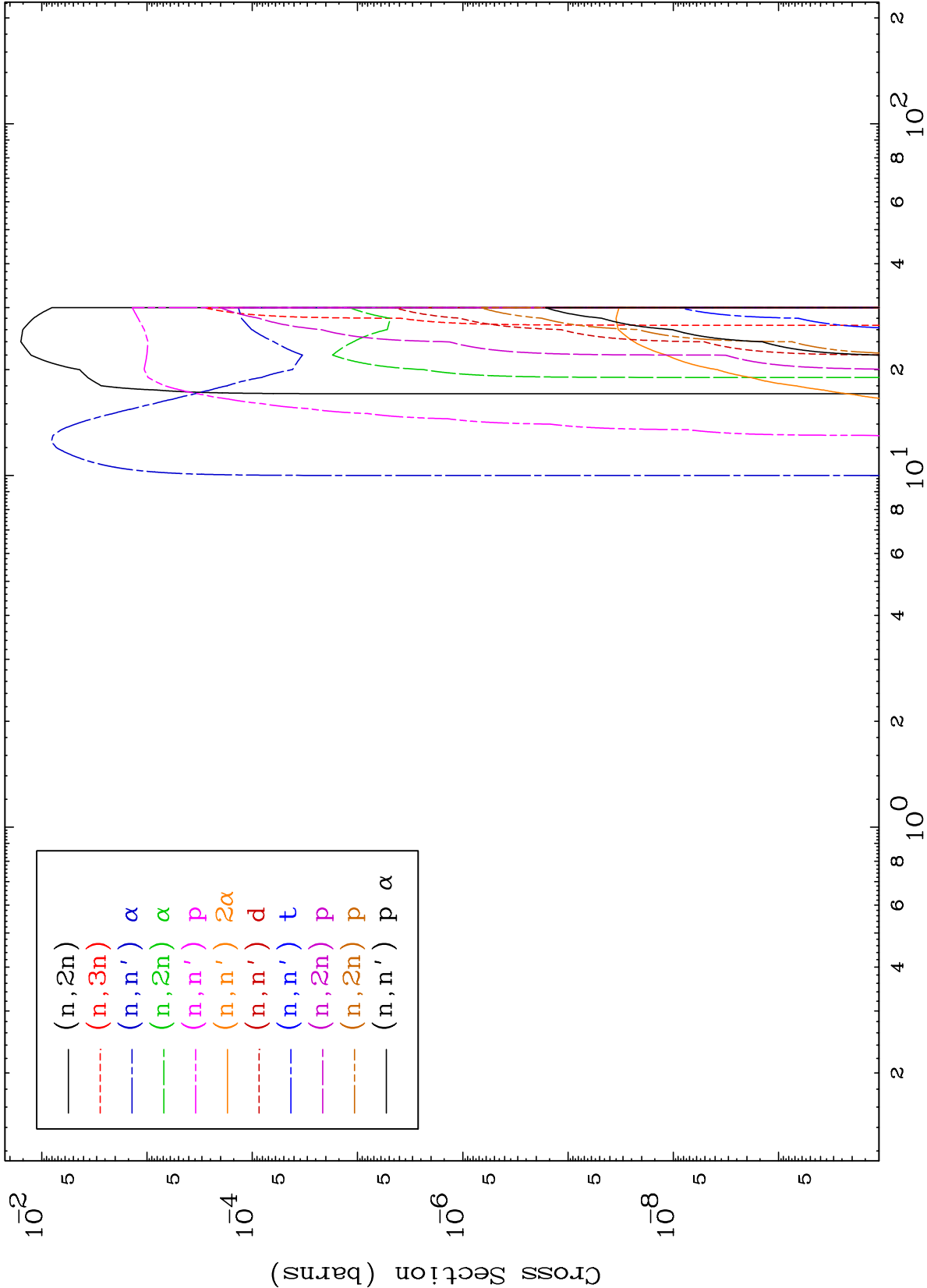
Press Mouse Button to Start

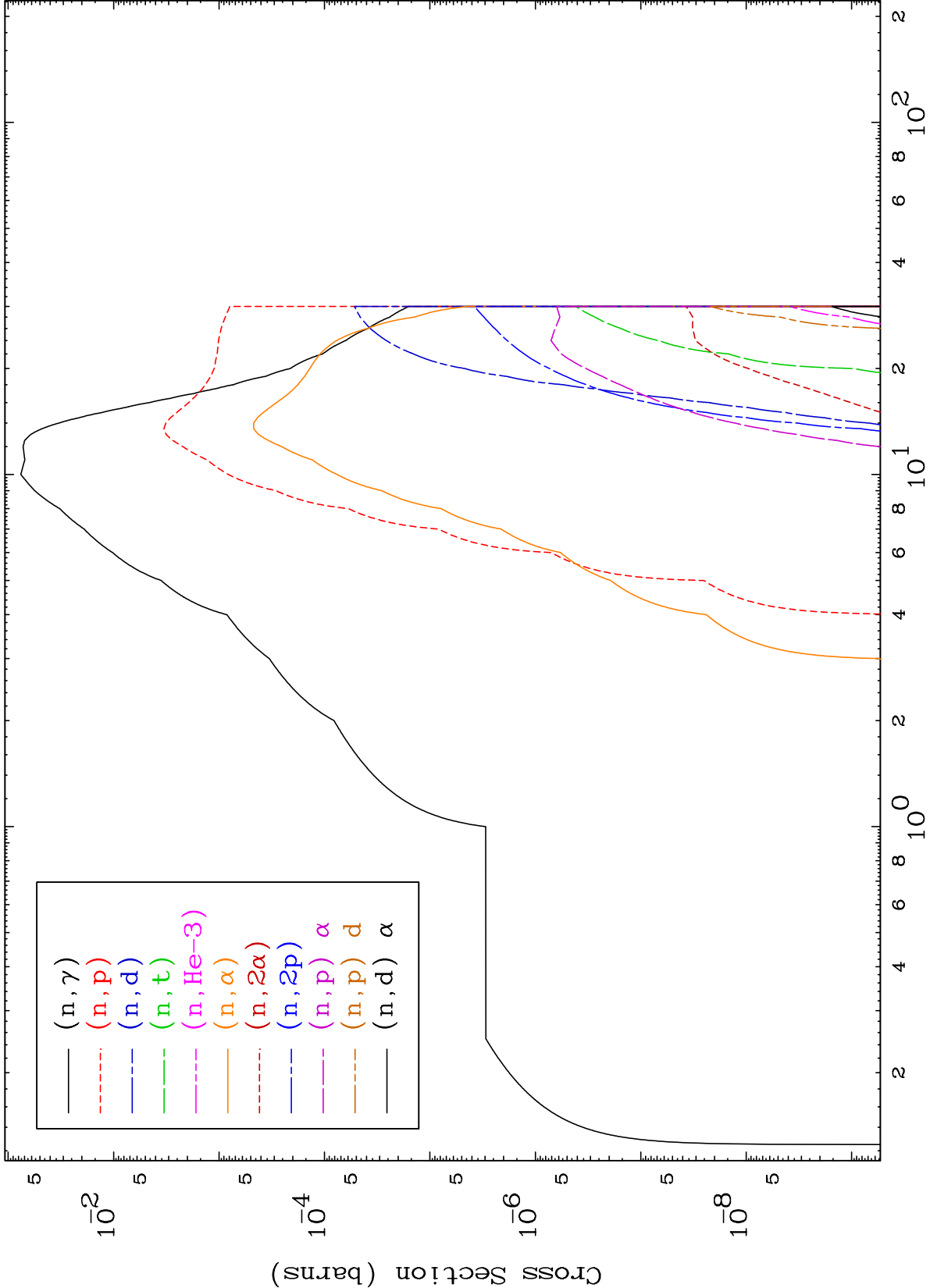


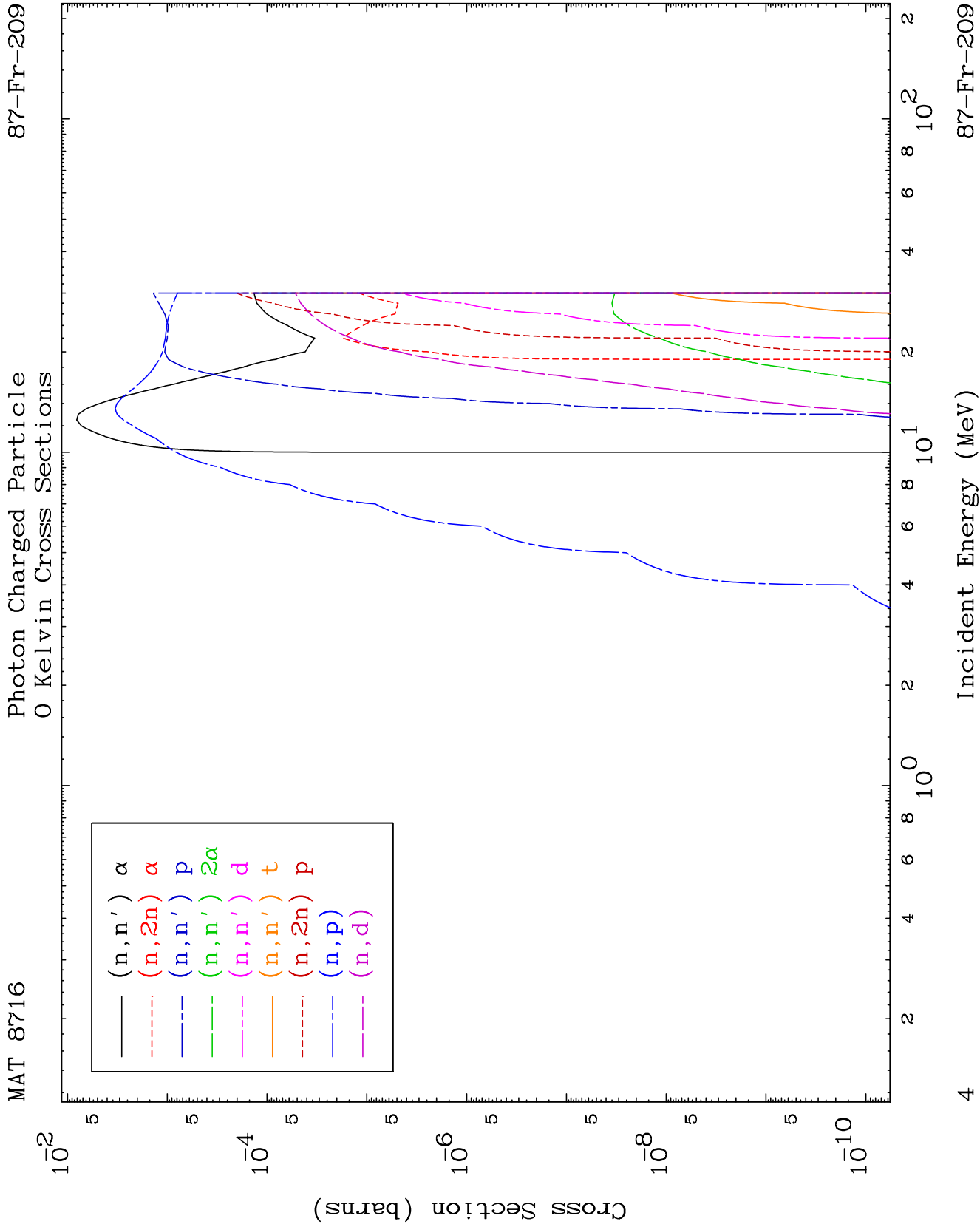
MAT 8716

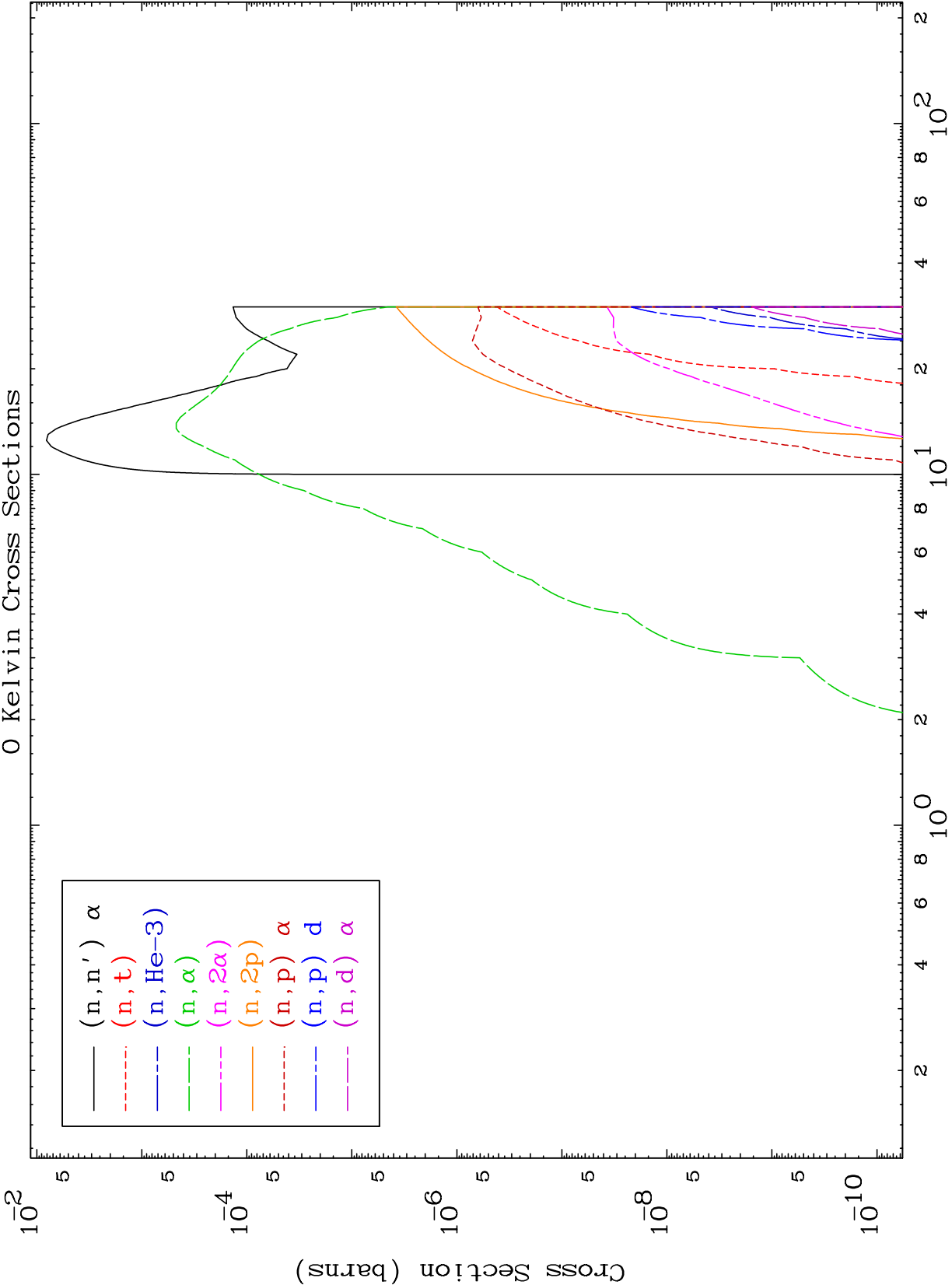
Photon Neutron Absorption
0 Kelvin Cross Sections

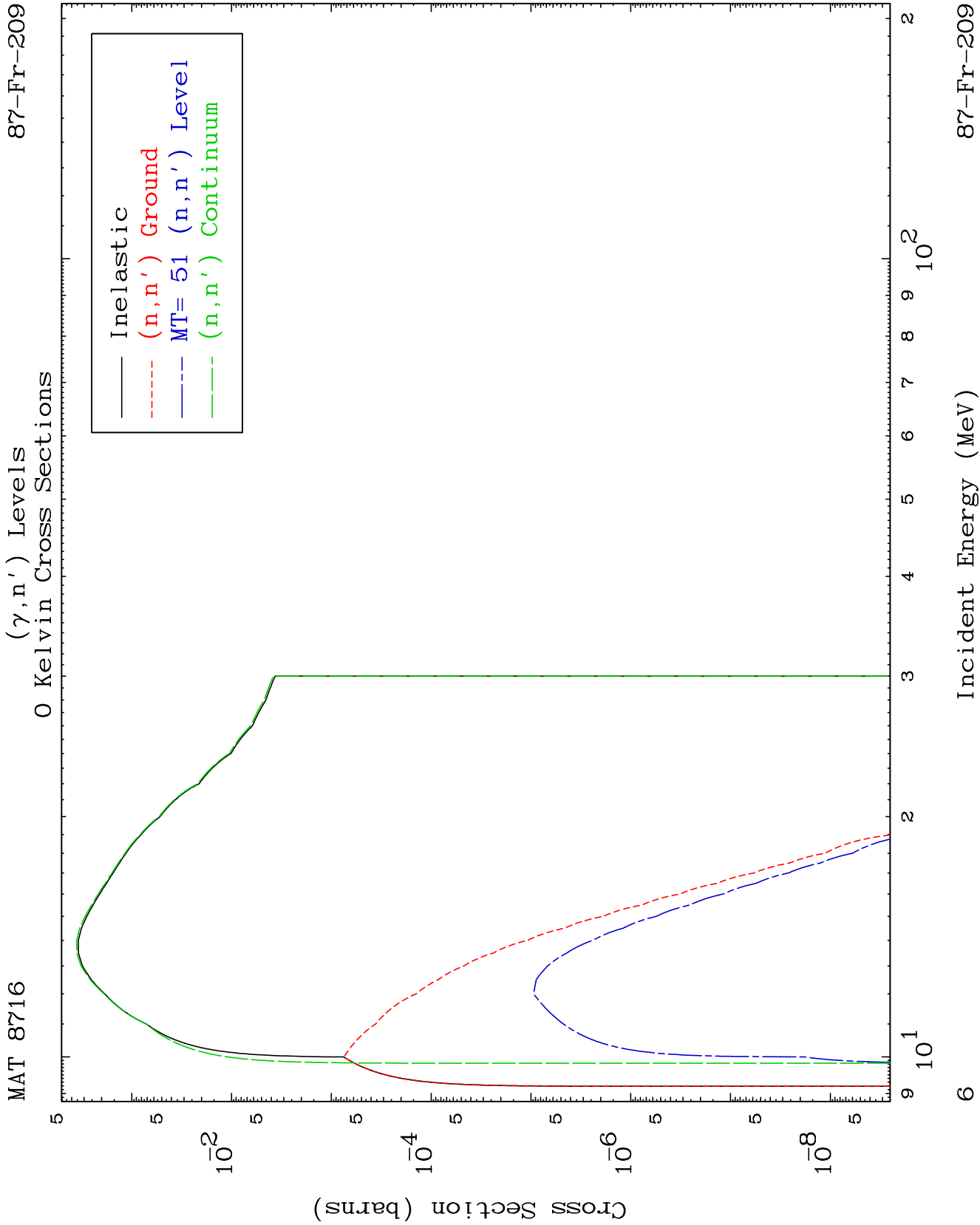
87-Fr-209







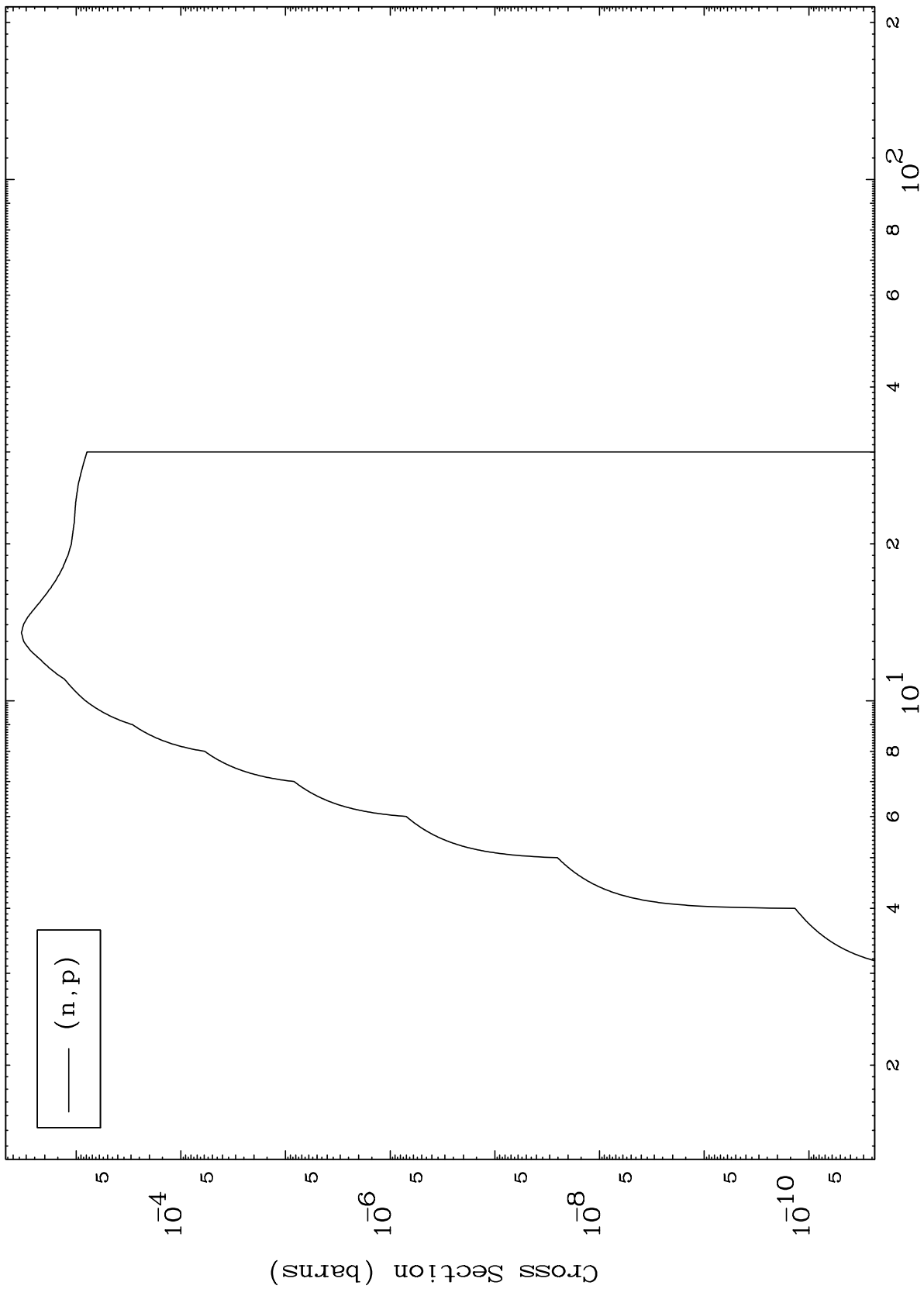




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(γ ,p) Levels
0 Kelvin Cross Sections



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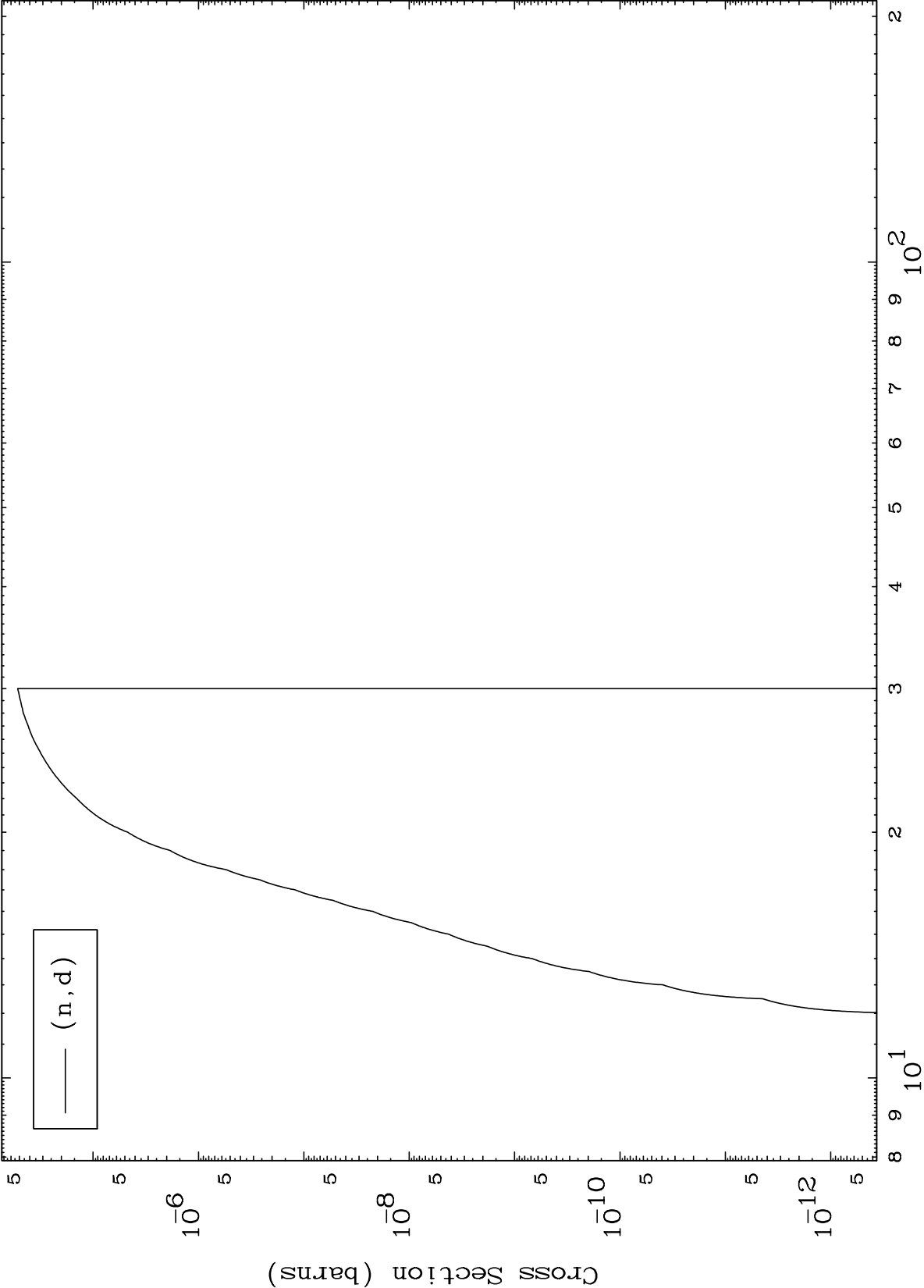
87-Fr-209

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(γ, d) Levels

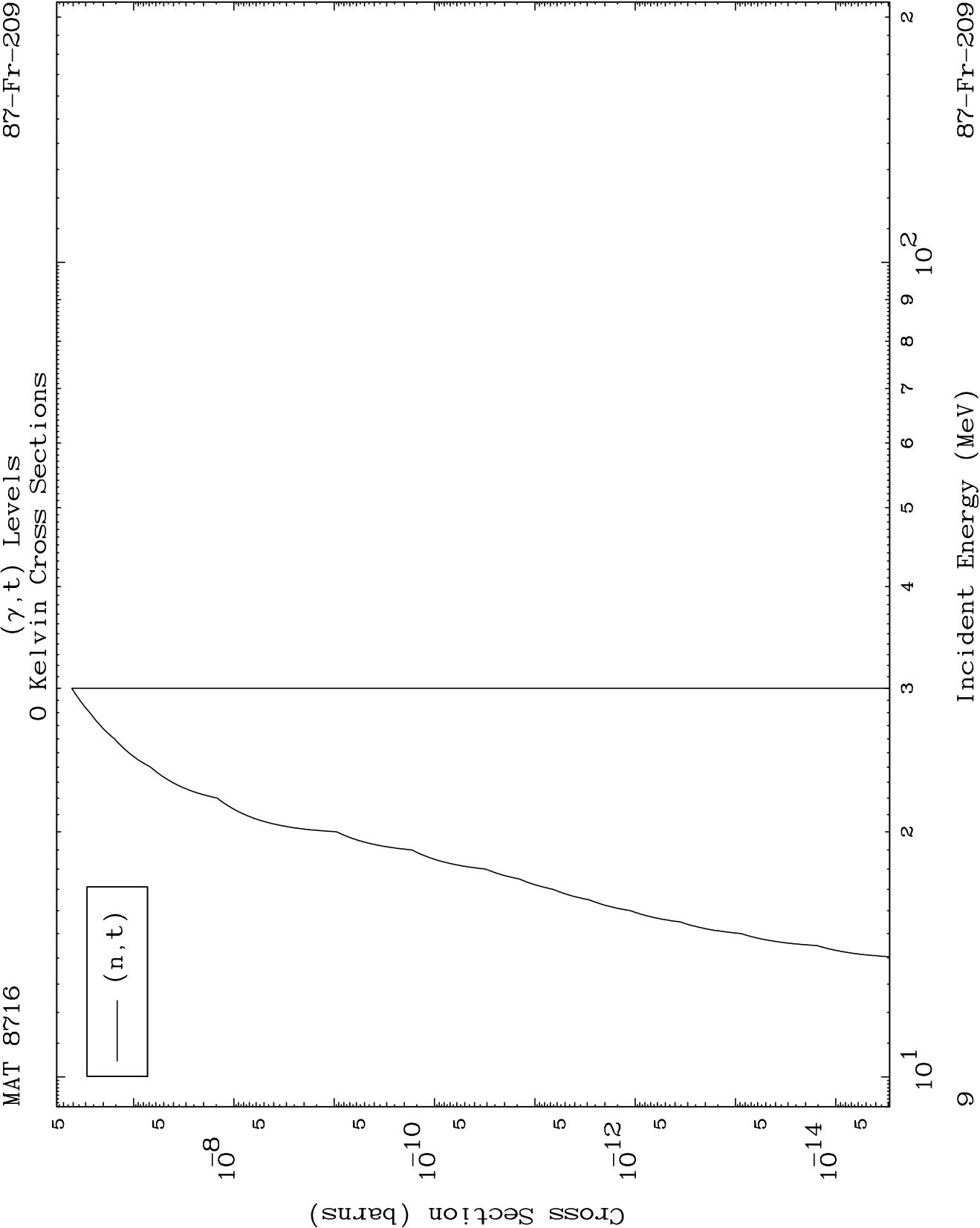
87-Fr-209

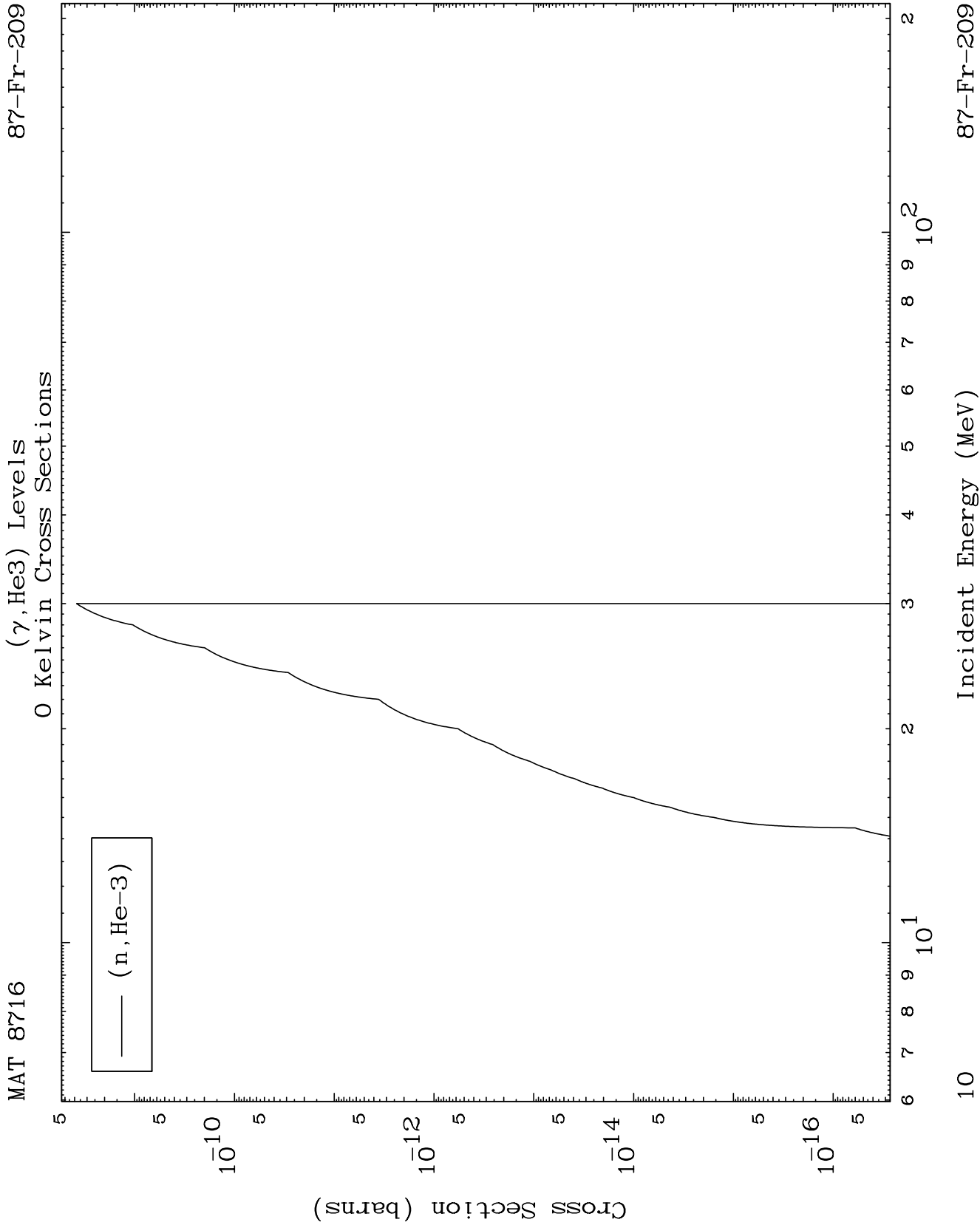
0 Kelvin Cross Sections



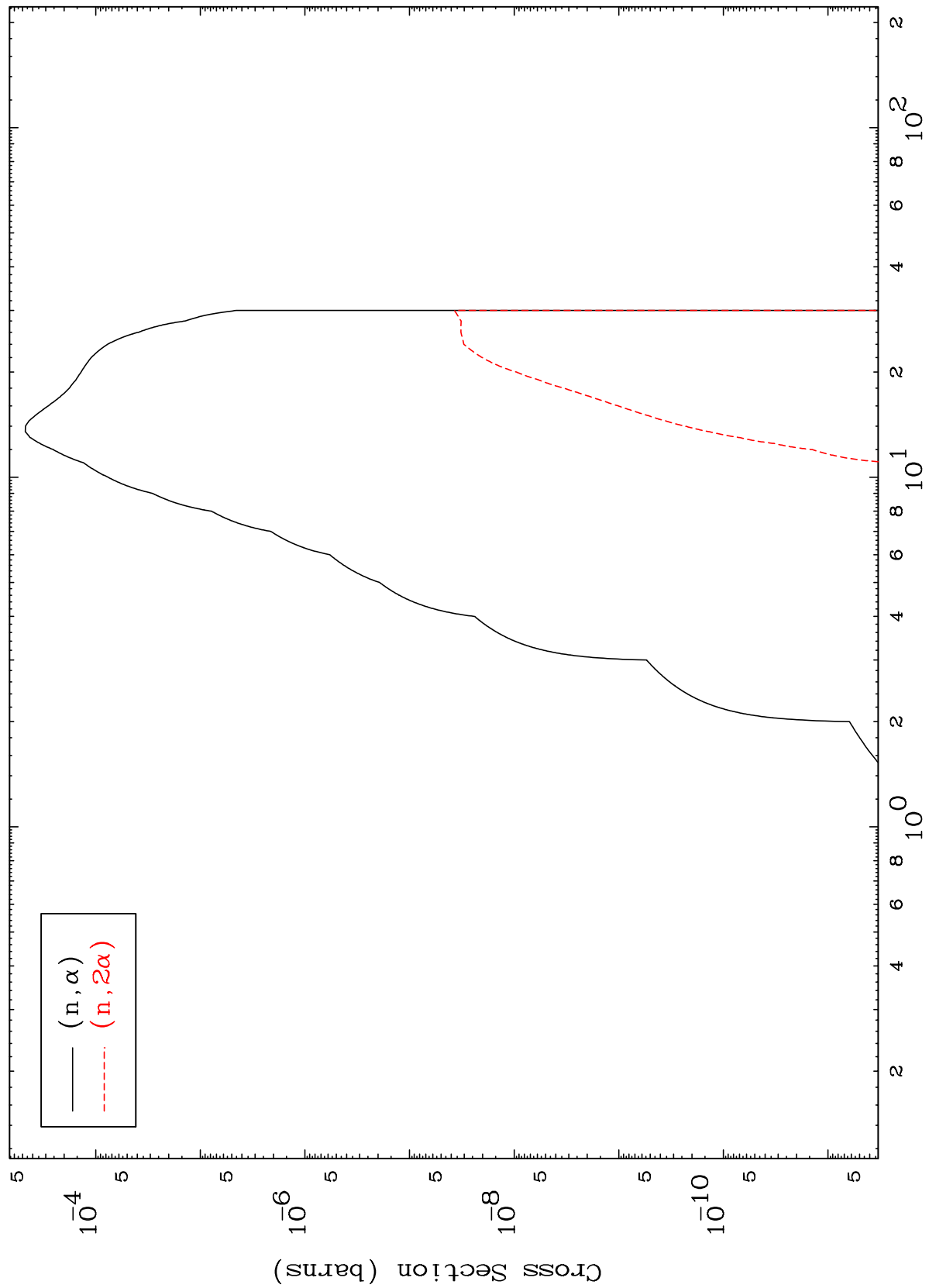
Incident Energy (MeV)

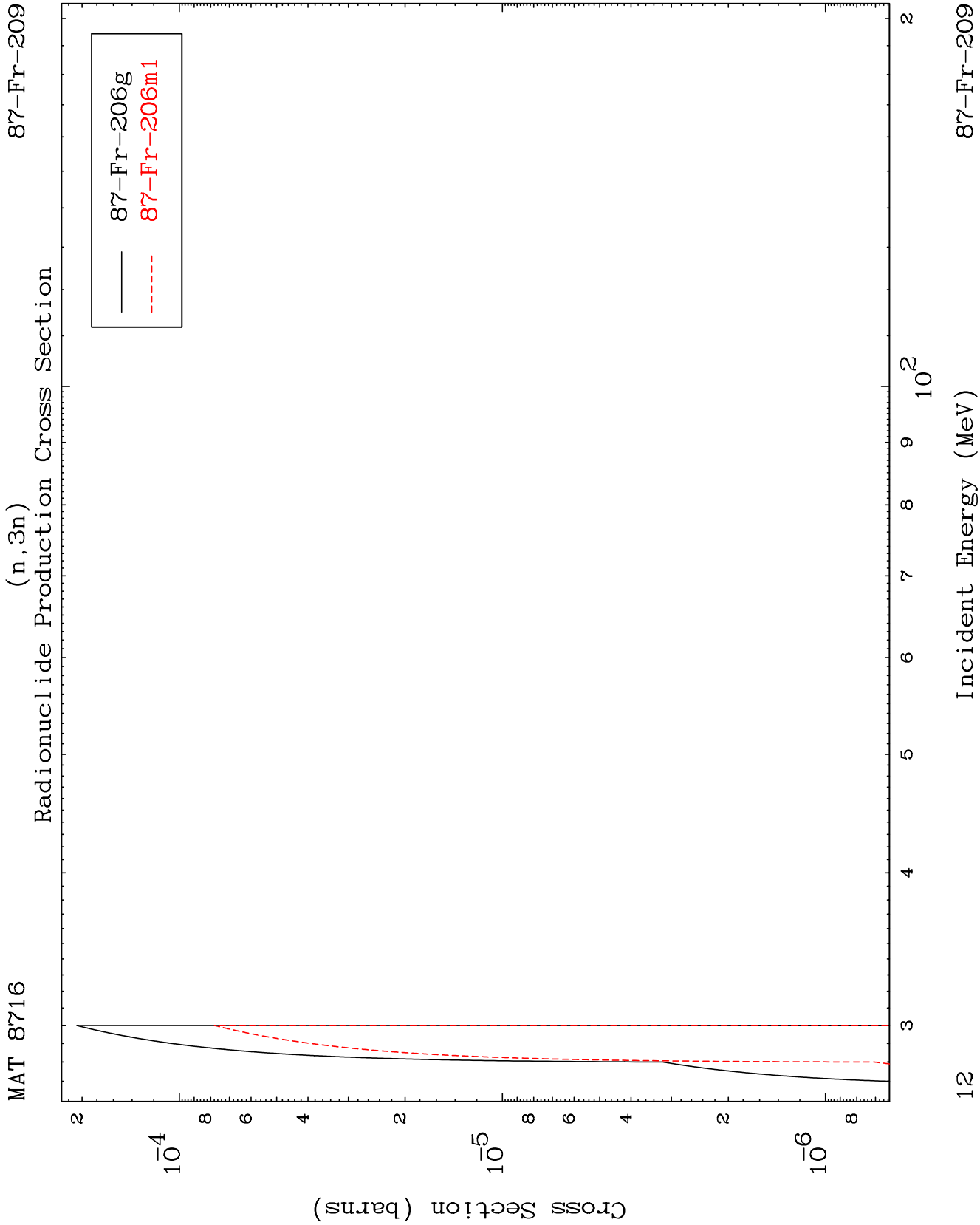
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(γ, α) Levels
0 Kelvin Cross Sections



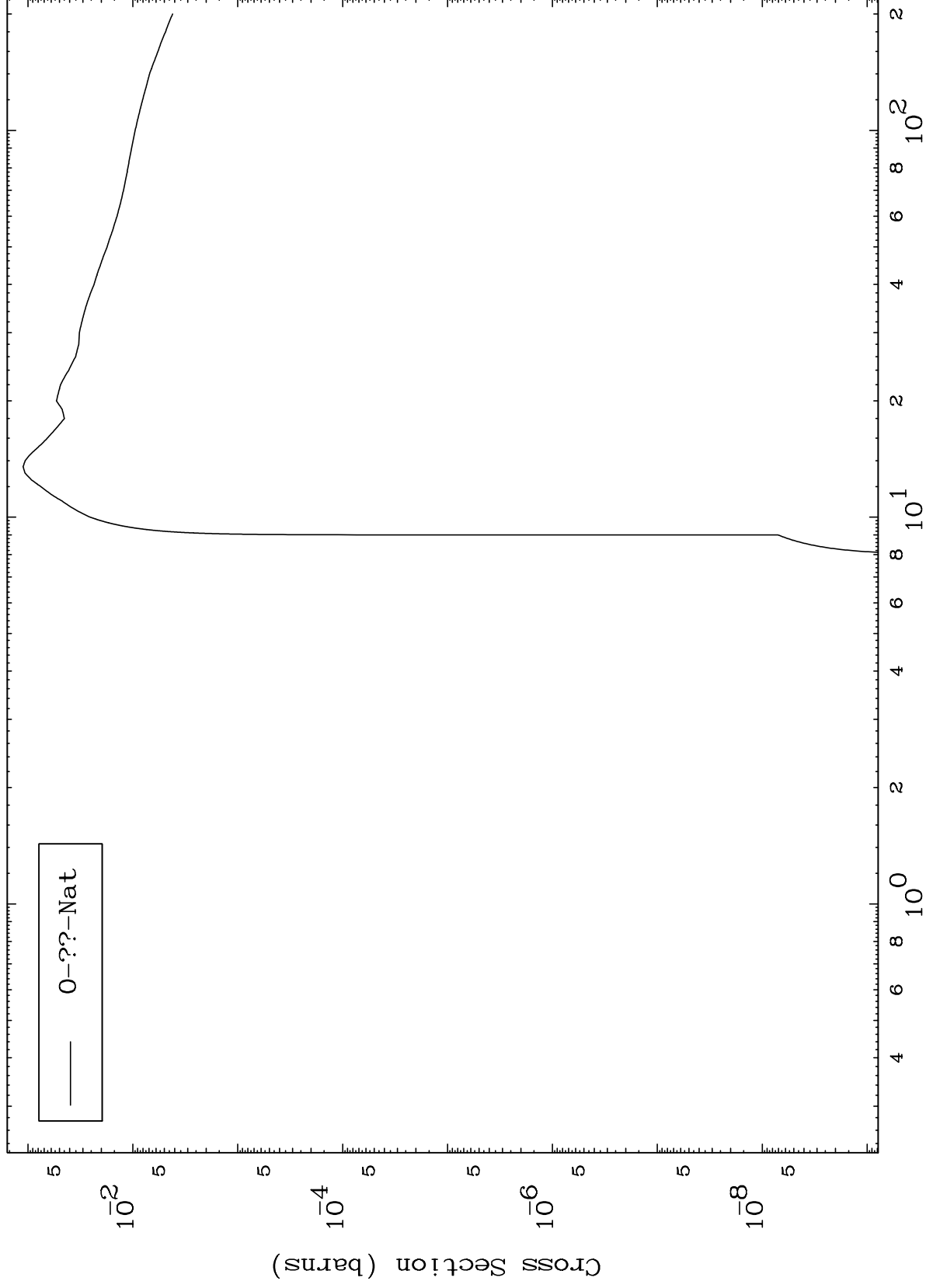


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Fission

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Radionuclide Production Cross Section



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Incident Energy (MeV)

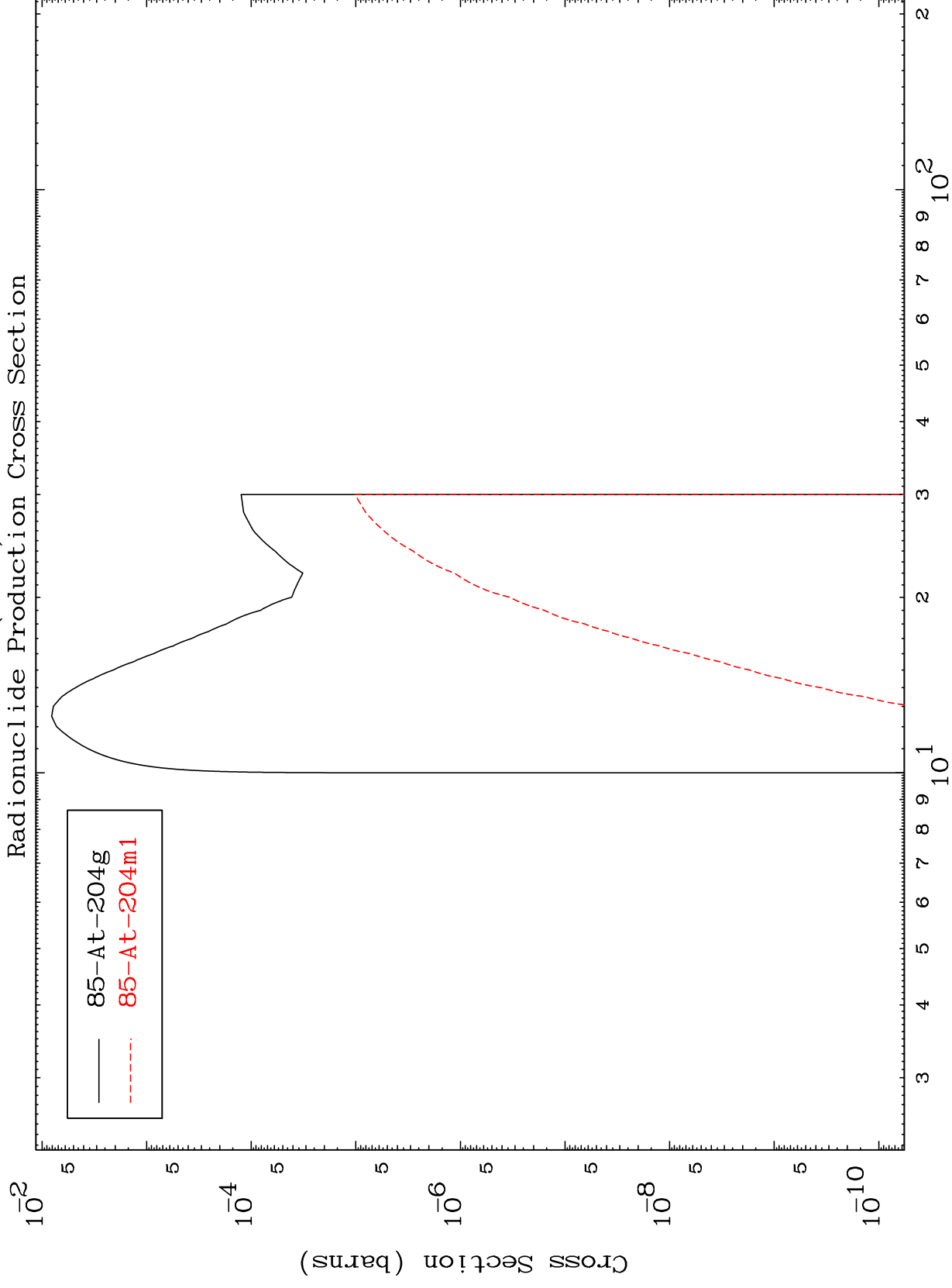
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$$(n, n') \propto$$

Radionuclide Production Cross Section

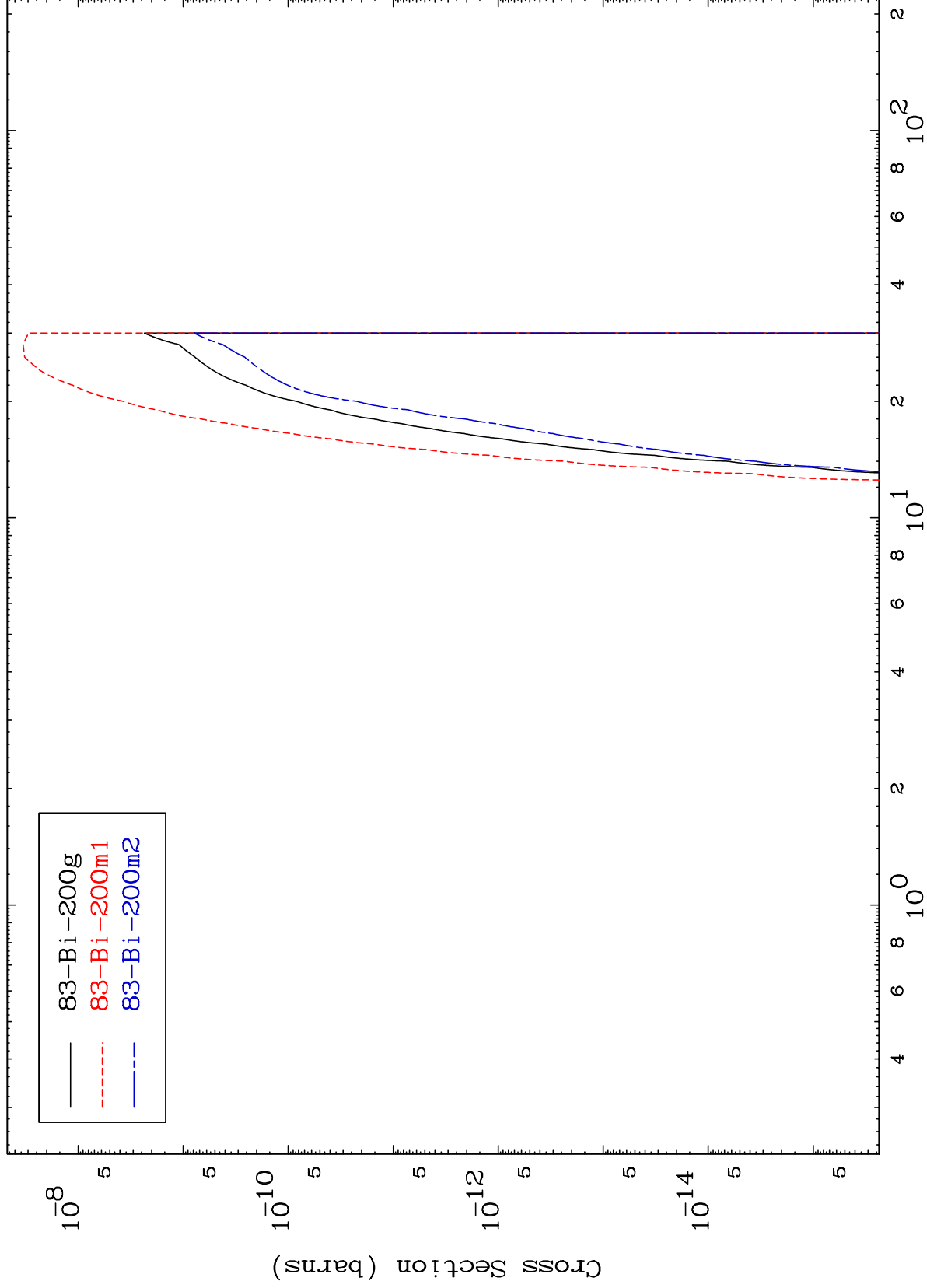


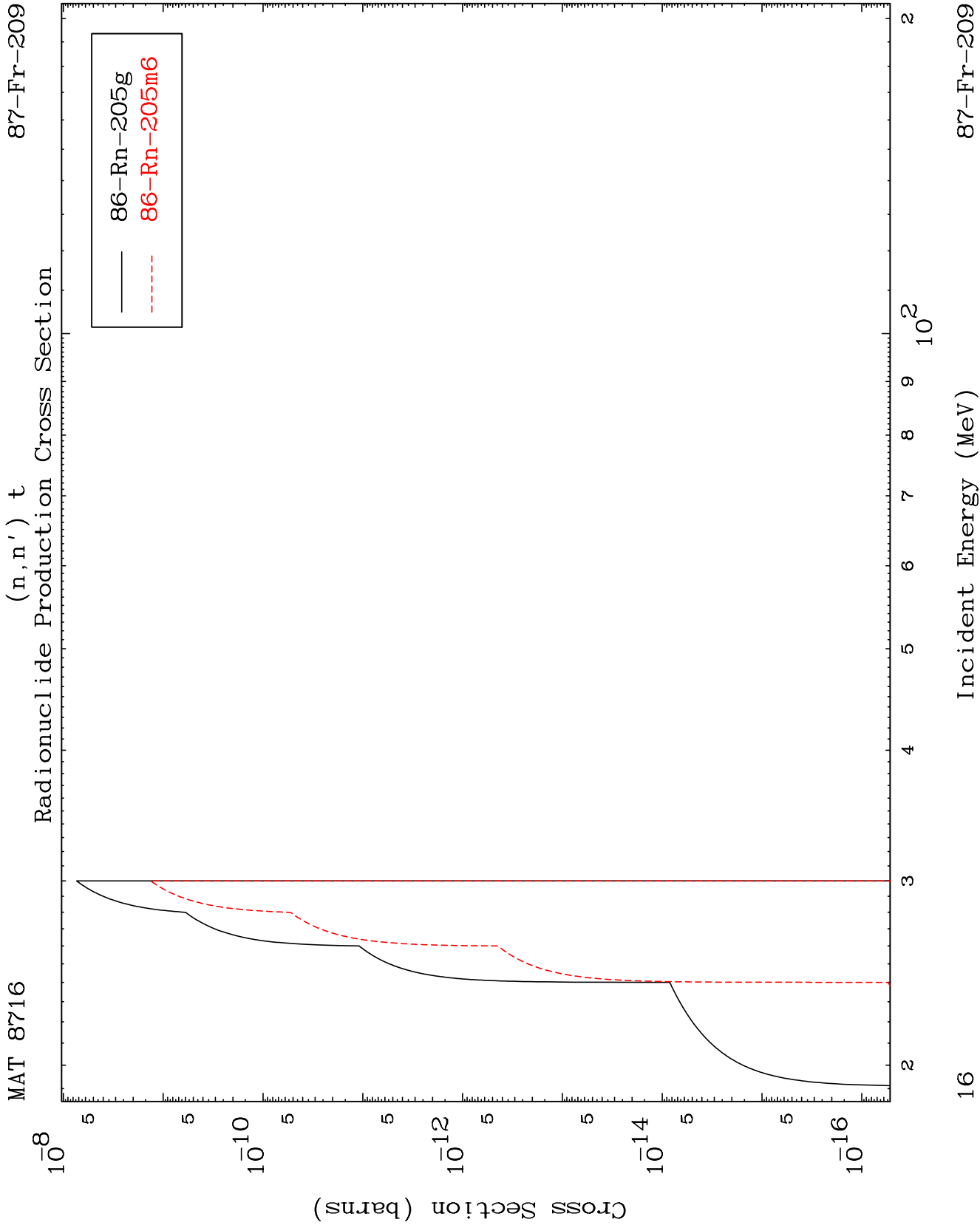
14

Incident Energy (MeV)

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Radionuclide Production Cross Section

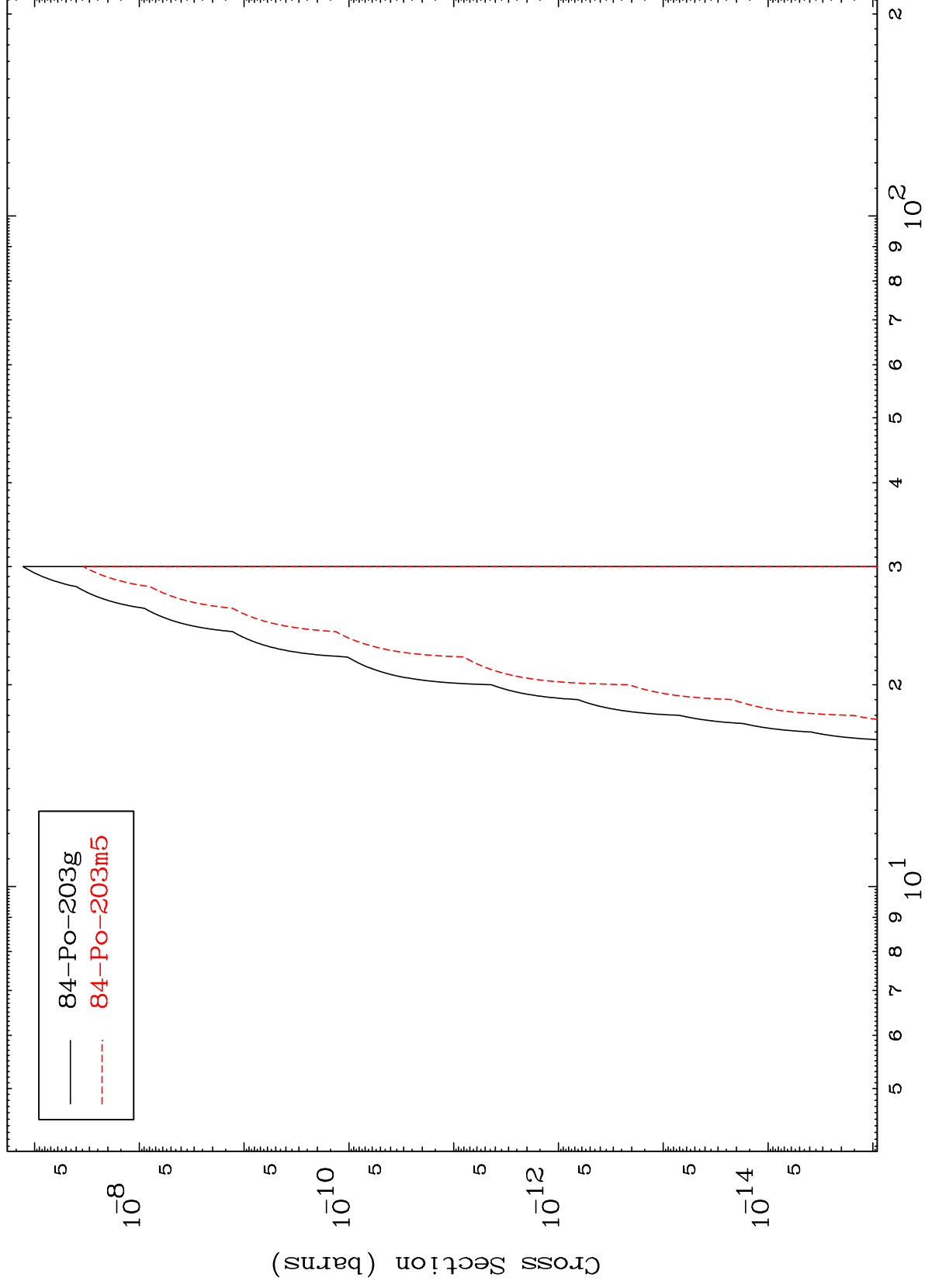




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(n,n') p α
Radionuclide Production Cross Section



17

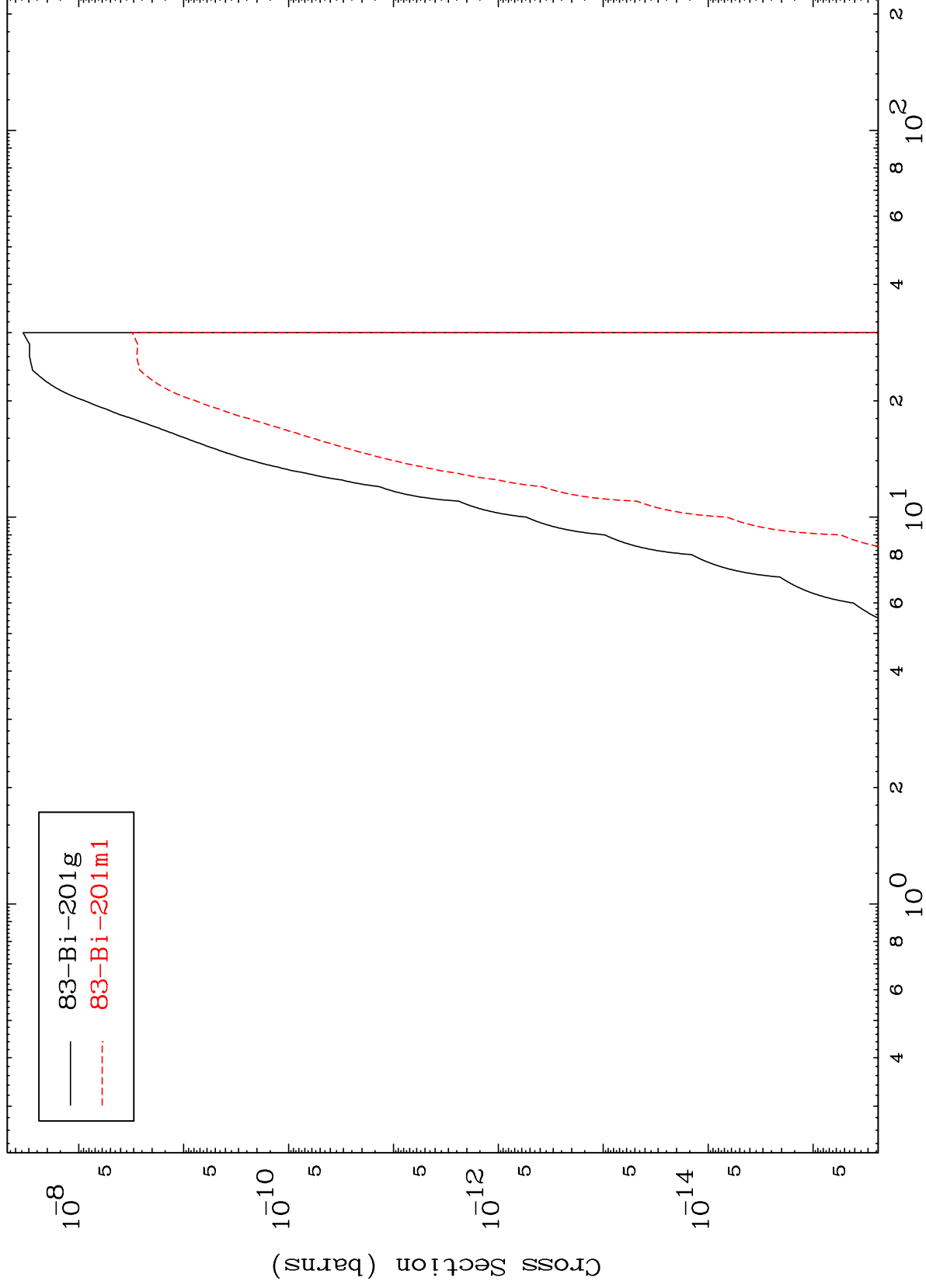
Incident Energy (MeV)

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Radionuclide Production Cross Section
(n,2α)



18

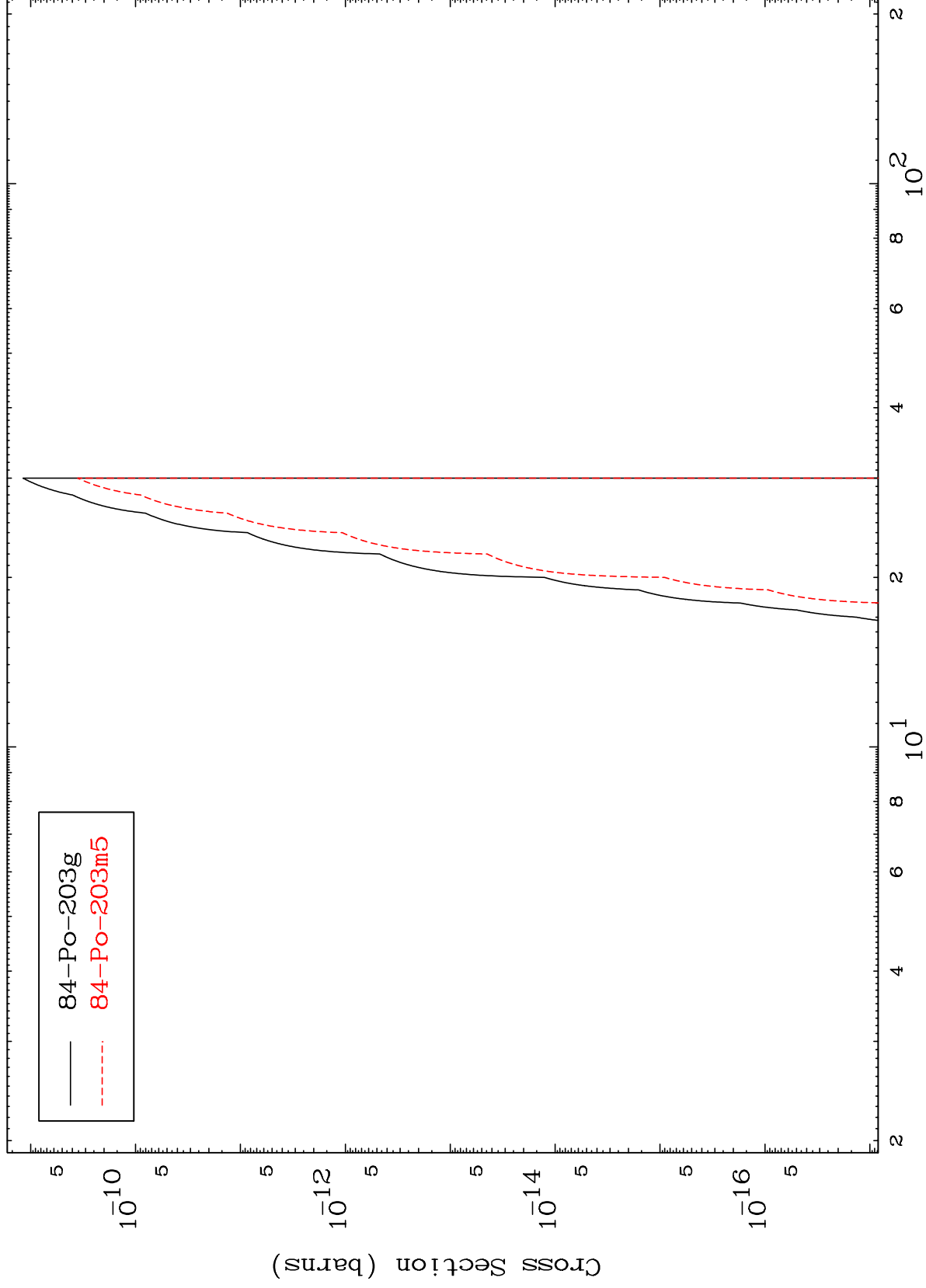
Incident Energy (MeV)

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(n,d) α
Radionuclide Production Cross Section



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Incident Energy (MeV)

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