

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

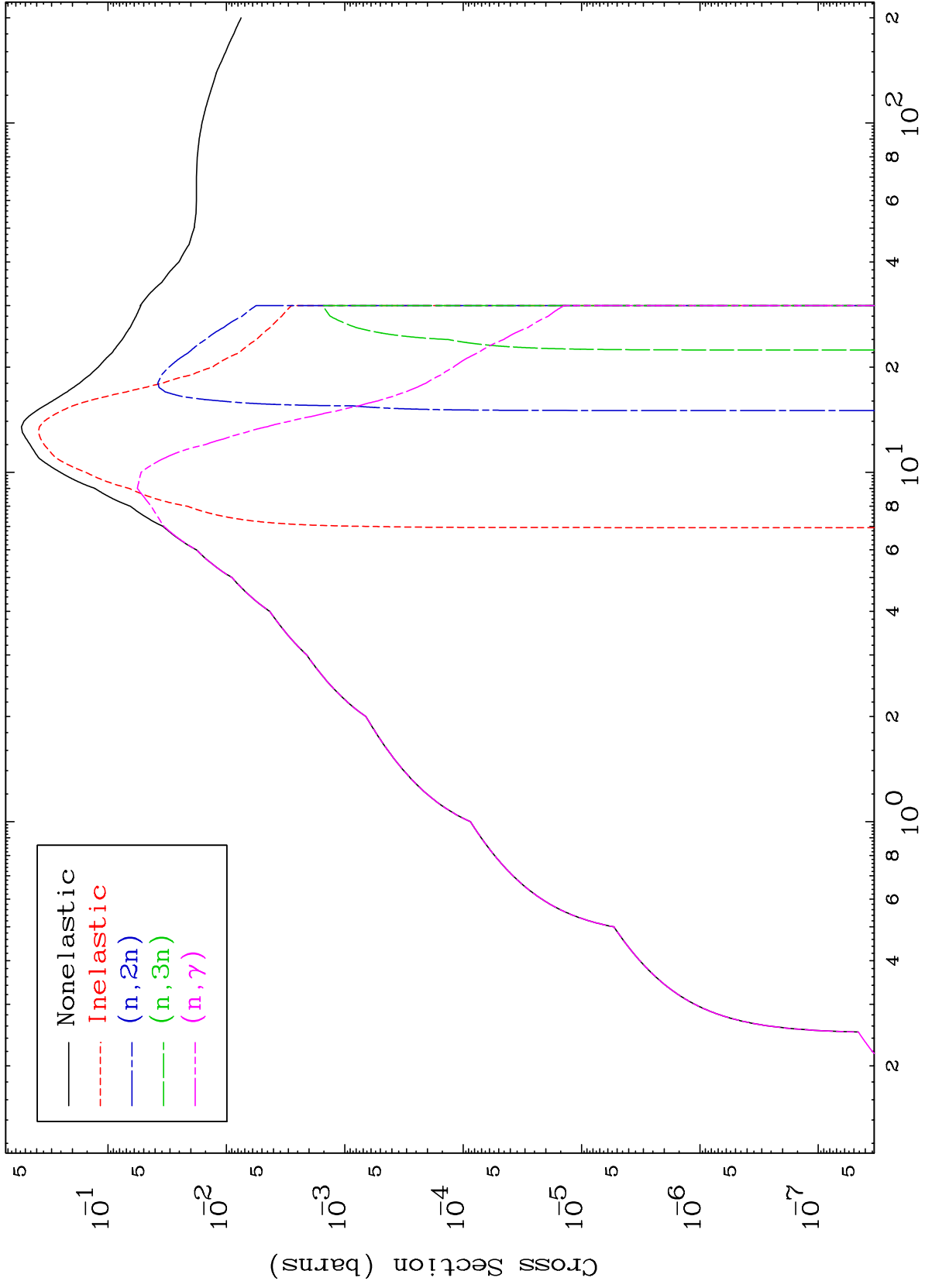
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9983

Photon Major
0 Kelvin Cross Sections

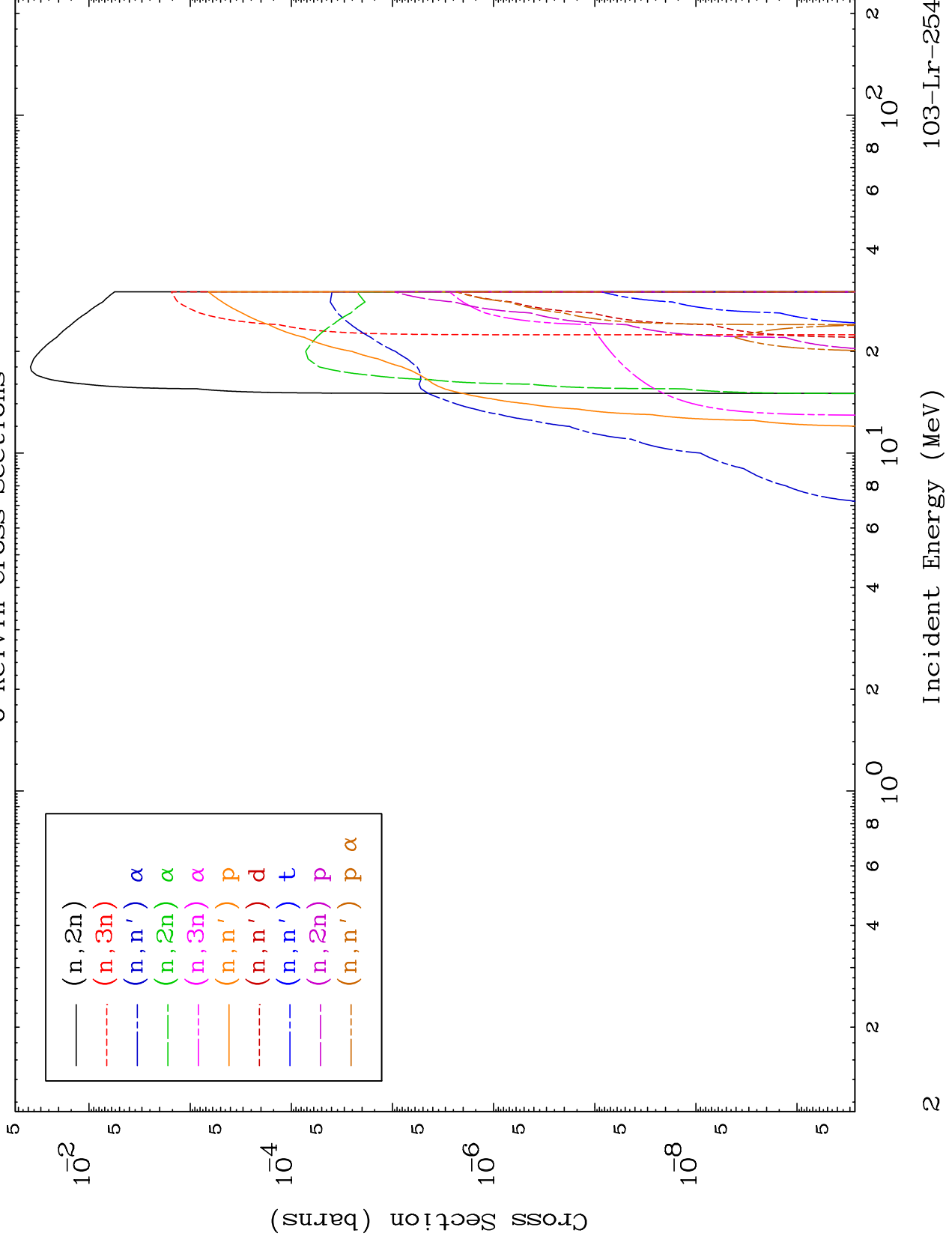
103-Lr-254

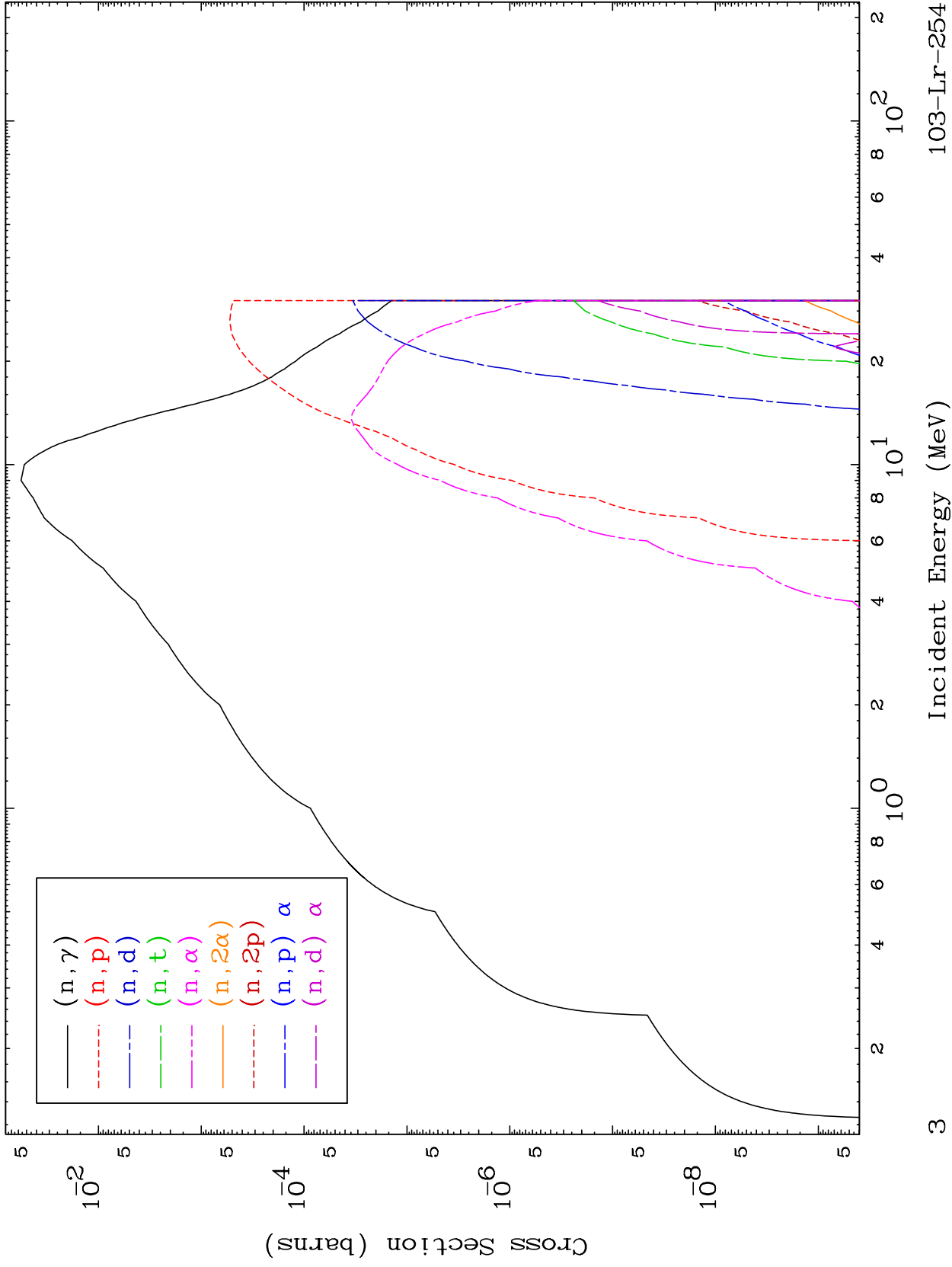


MAT 9983

Photon Neutron Absorption
0 Kelvin Cross Sections

103-Lr-254

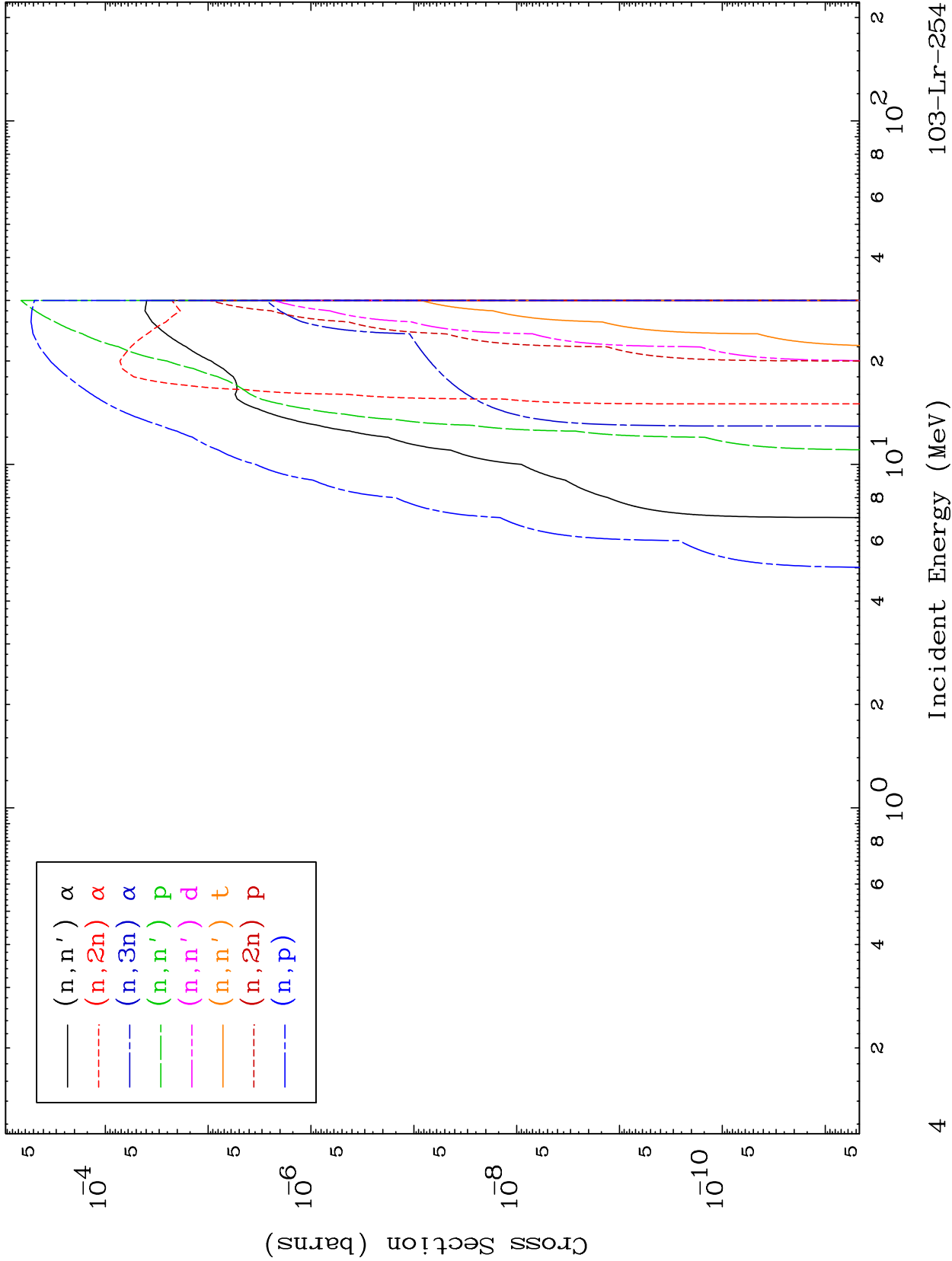




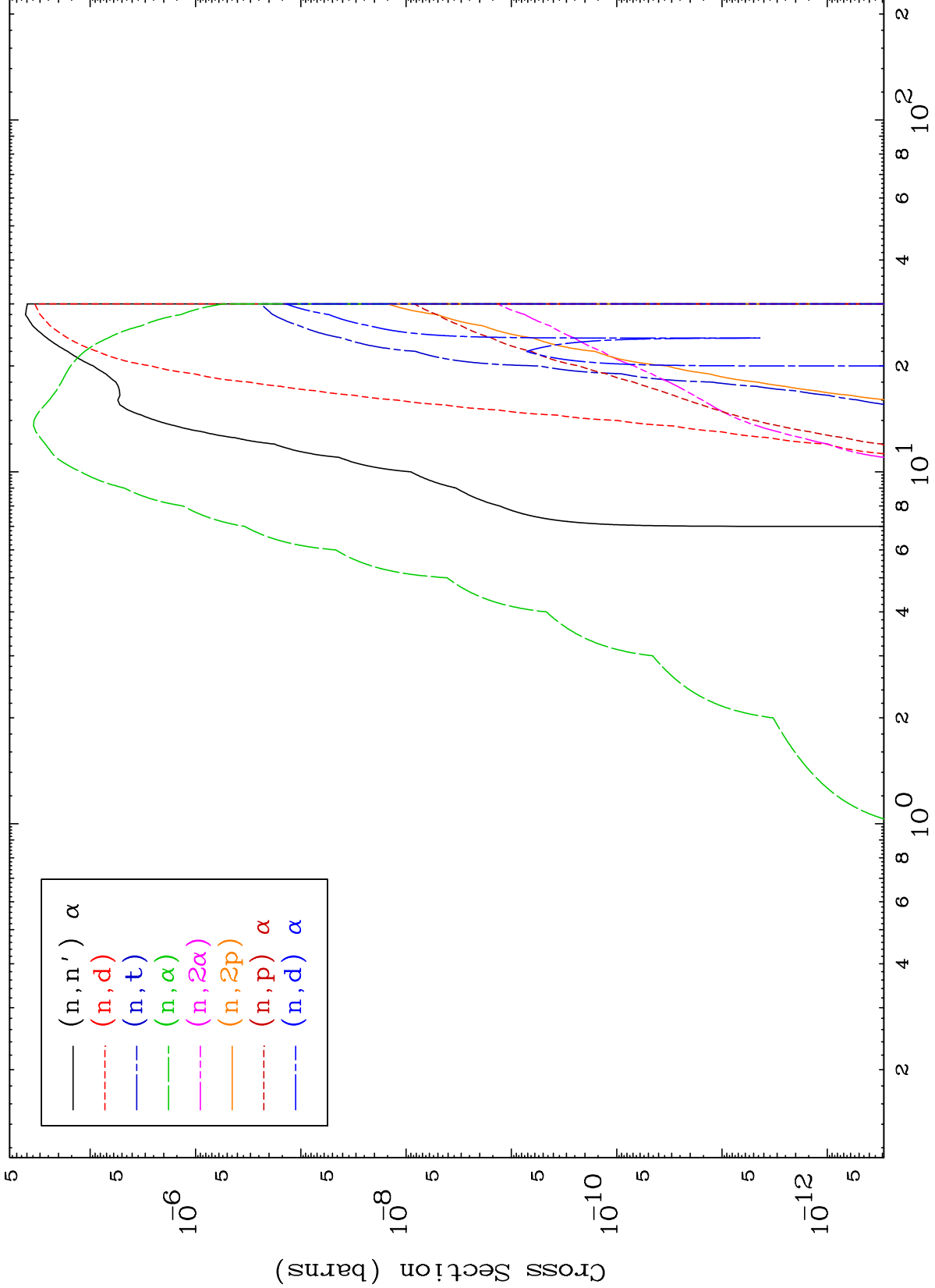
MAT 9983

Photon Charged Particle
0 Kelvin Cross Sections

103-Lr-254



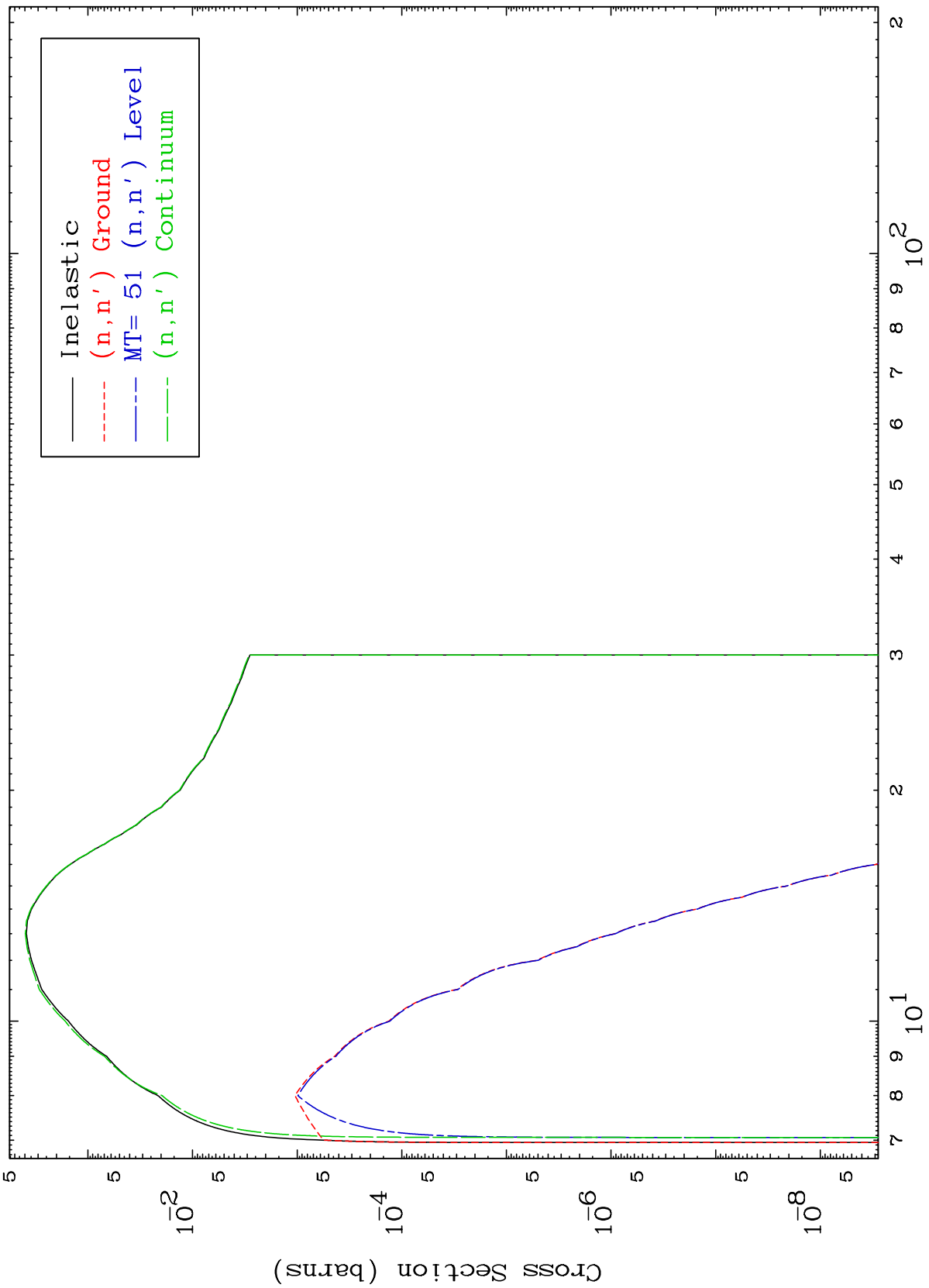
103-Lr-254



MAT 9983

103-Lr-254

(γ, n') Levels
0 Kelvin Cross Sections



103-Lr-254

Incident Energy (MeV)

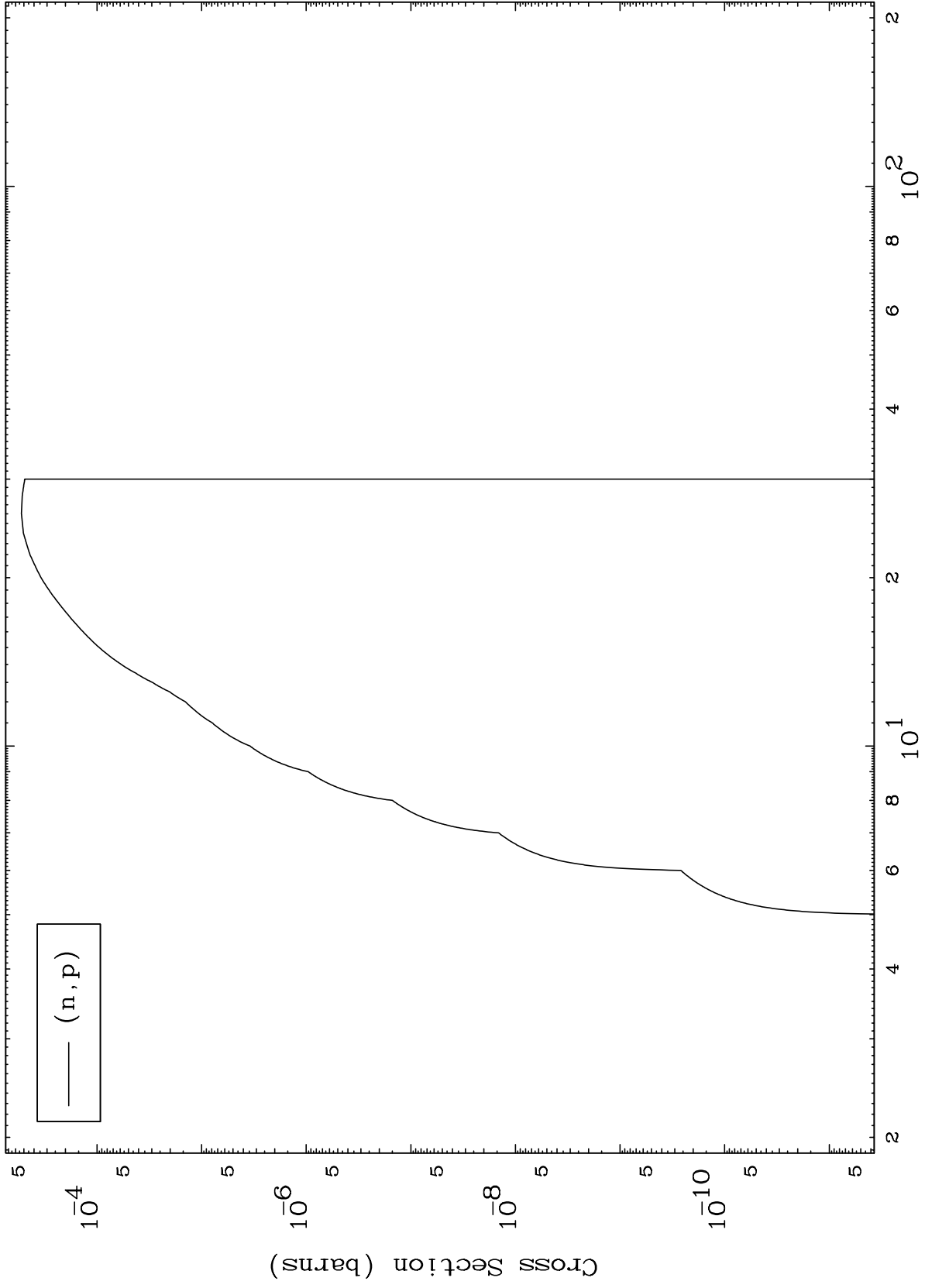
6

MAT 9983

(γ, p) Levels

103-Lr-254

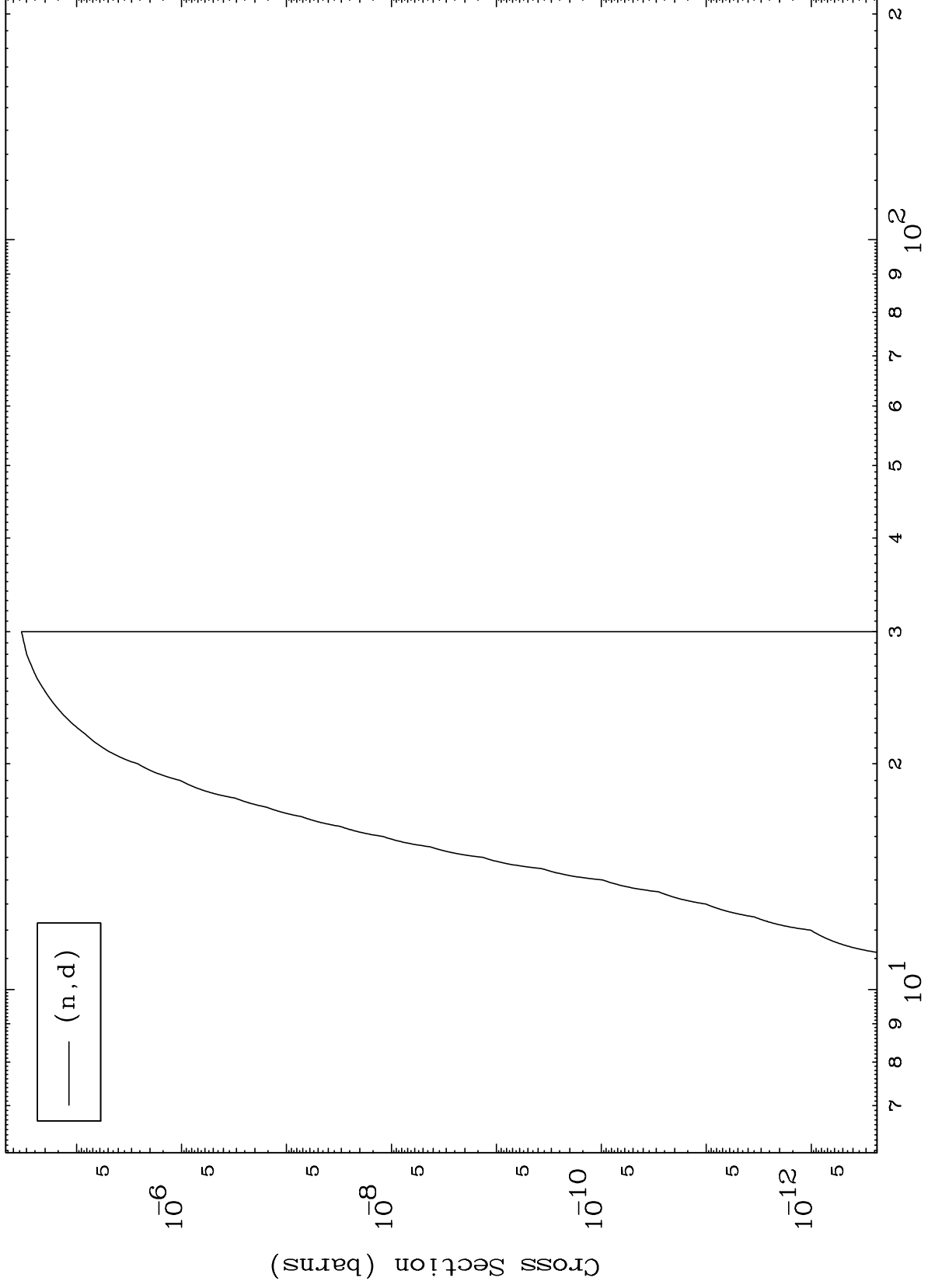
0 Kelvin Cross Sections



7

Incident Energy (MeV)

103-Lr-254

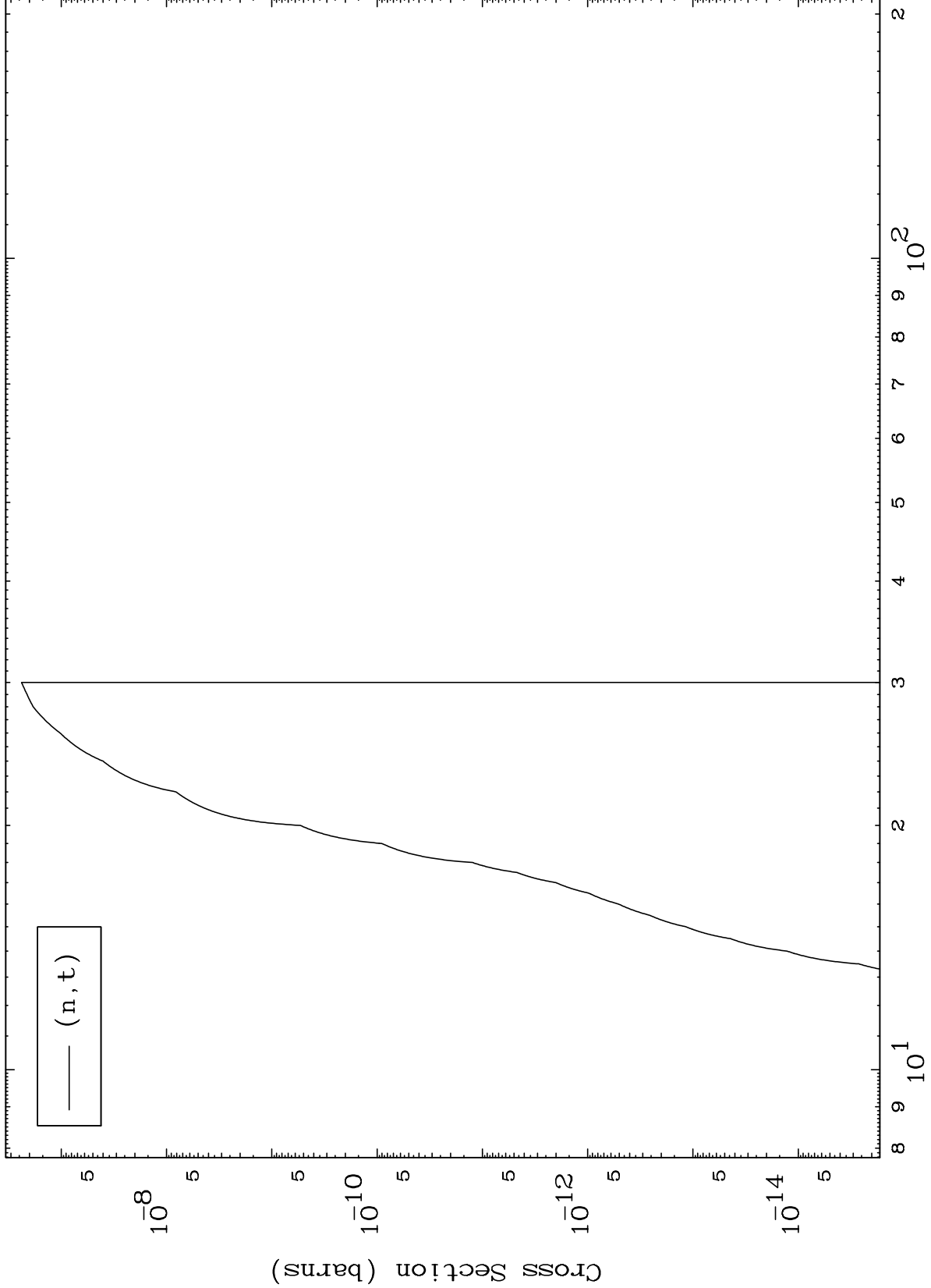


MAT 9983

(γ, t) Levels

103-Lr-254

0 Kelvin Cross Sections



9

Incident Energy (MeV)

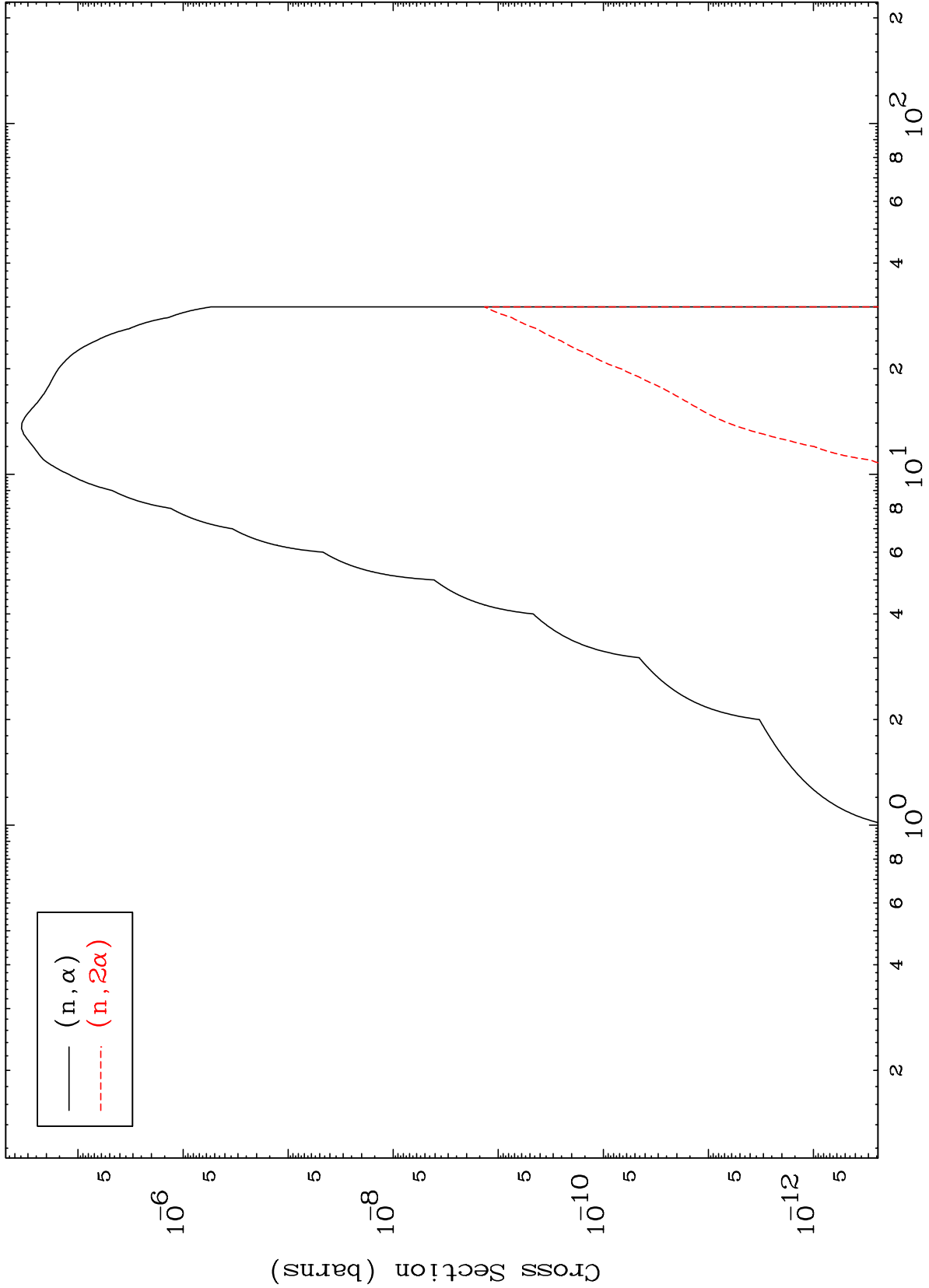
103-Lr-254

MAT 9983

(γ, α) Levels

103-Lr-254

0 Kelvin Cross Sections



— (n, α)
- - - ($n, 2\alpha$)

10

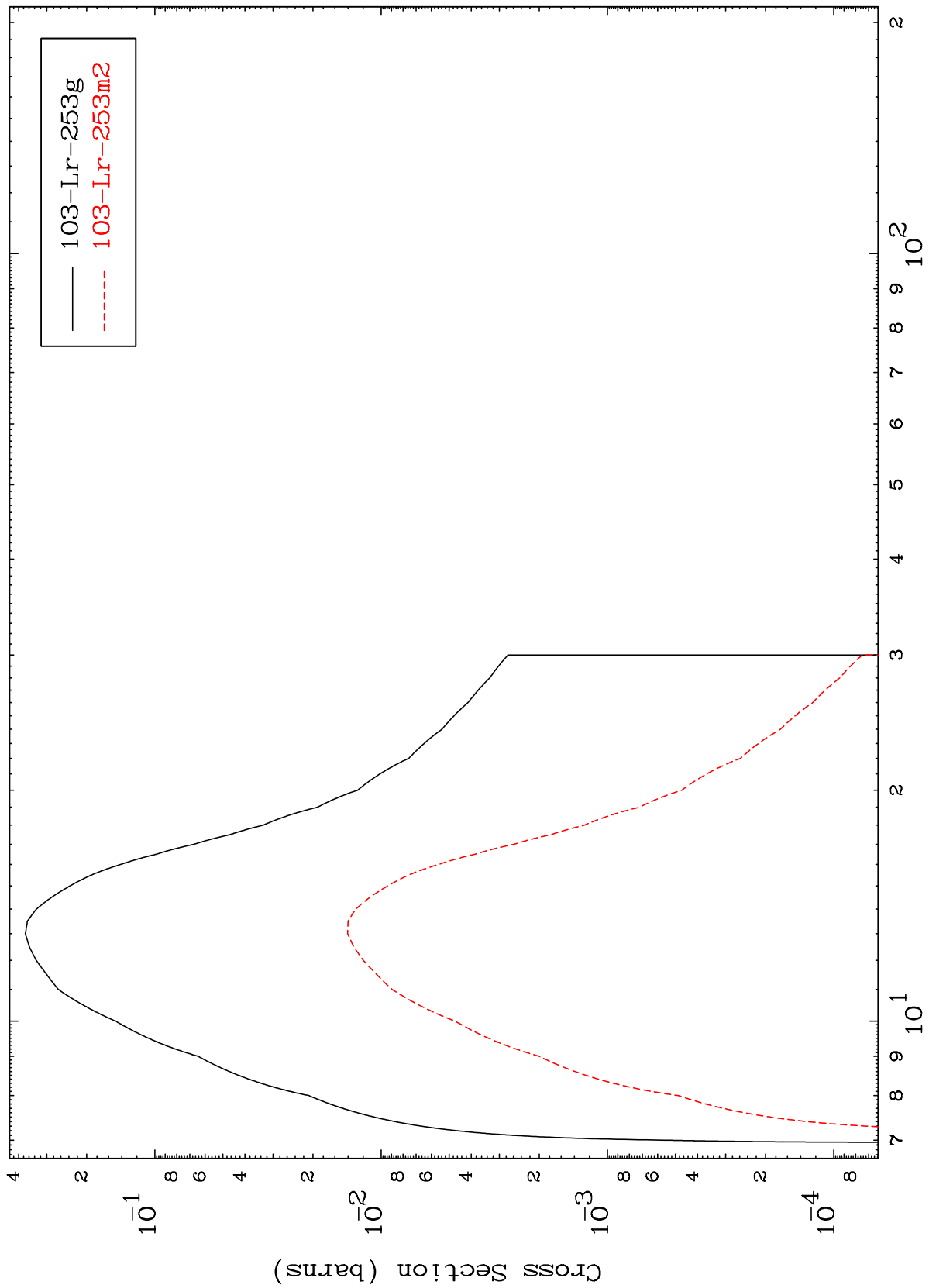
Incident Energy (MeV)

103-Lr-254

MAT 9983

103-Lr-254

Inelastic
Radionuclide Production Cross Section



103-Lr-253g
103-Lr-253m2

103-Lr-254

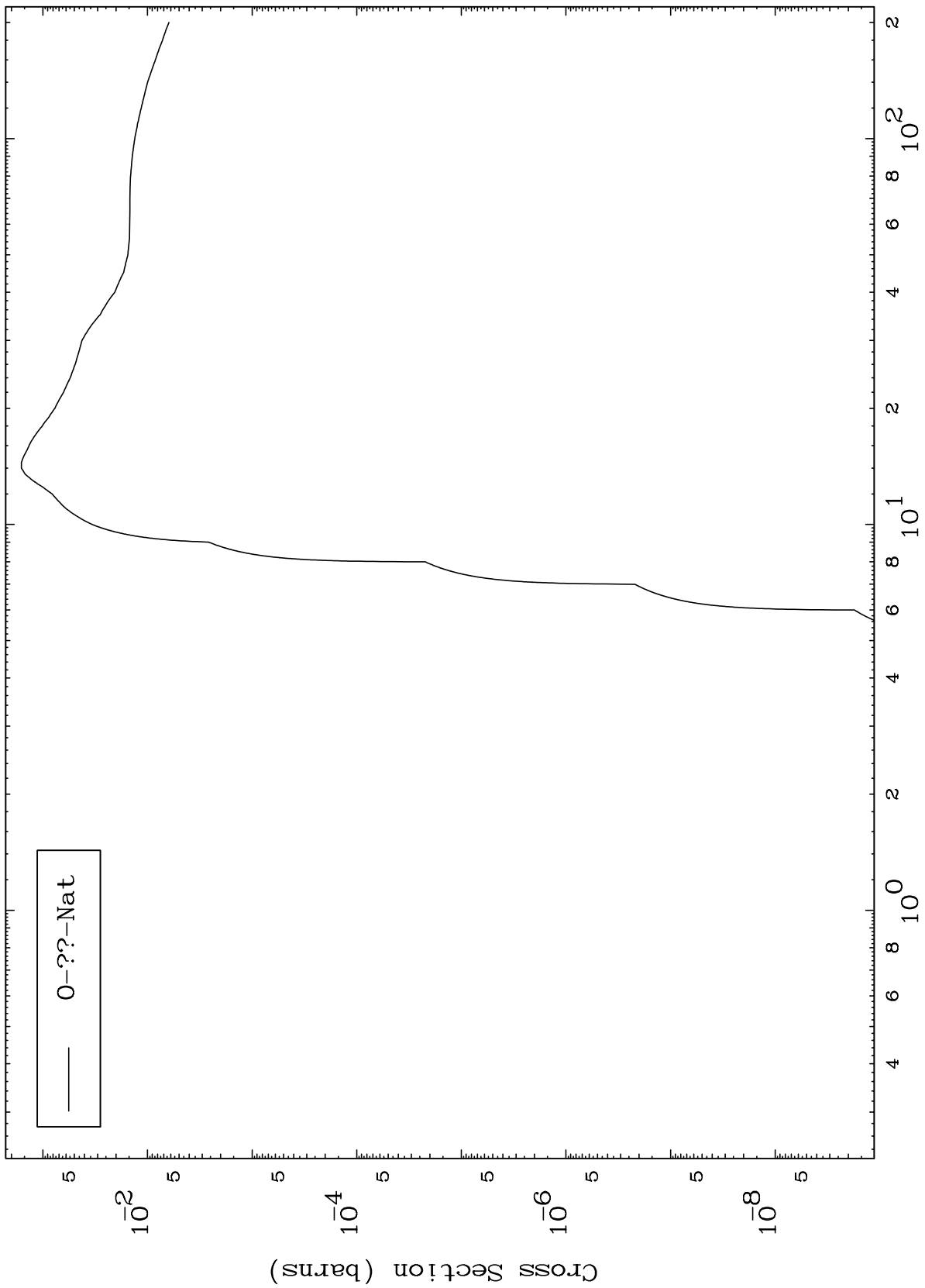
Incident Energy (MeV)

11

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103-Lr-254

Fission
Radionuclide Production Cross Section



103-Lr-254

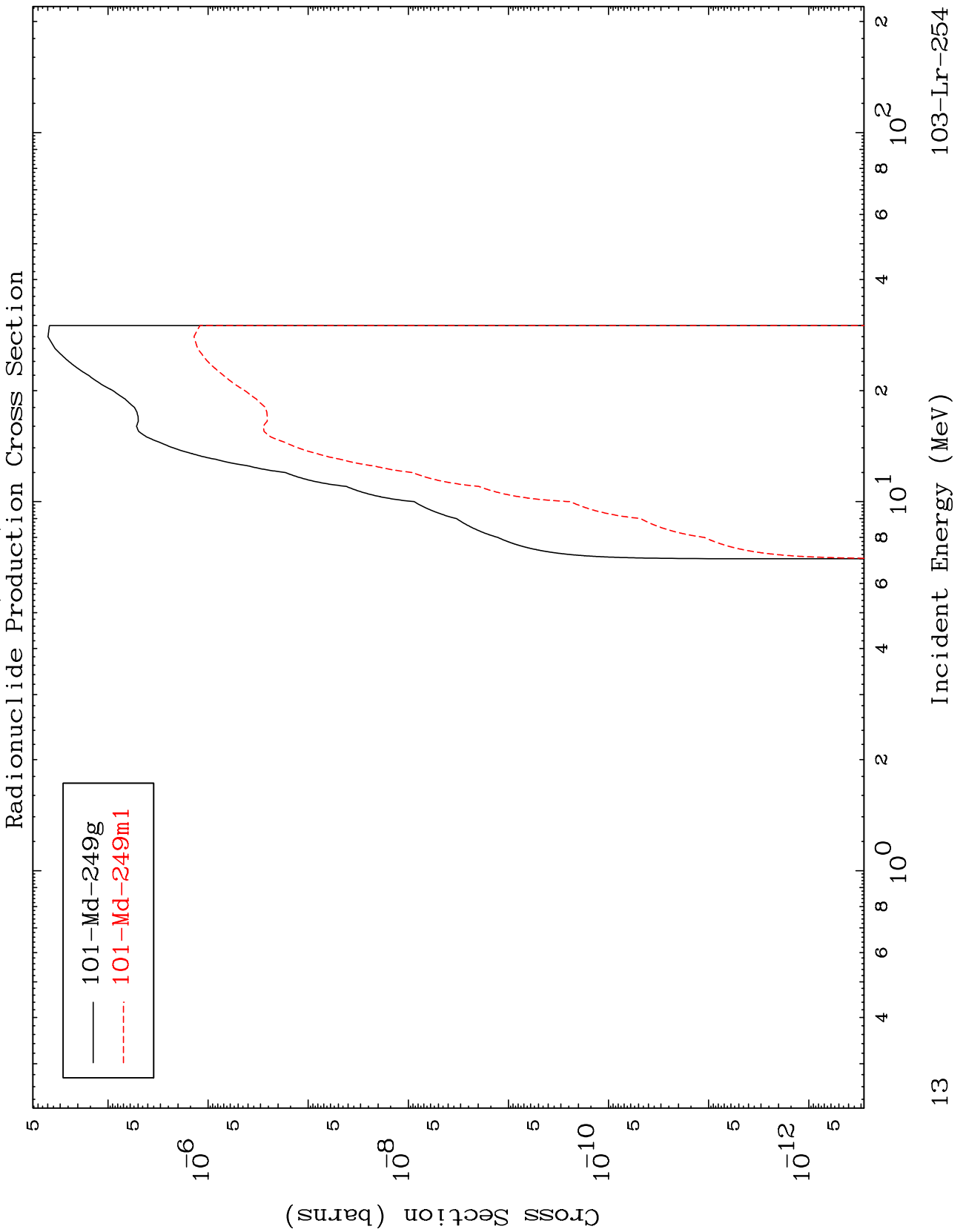
Incident Energy (MeV)

12

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$(n, n') \alpha$

103-Lr-254



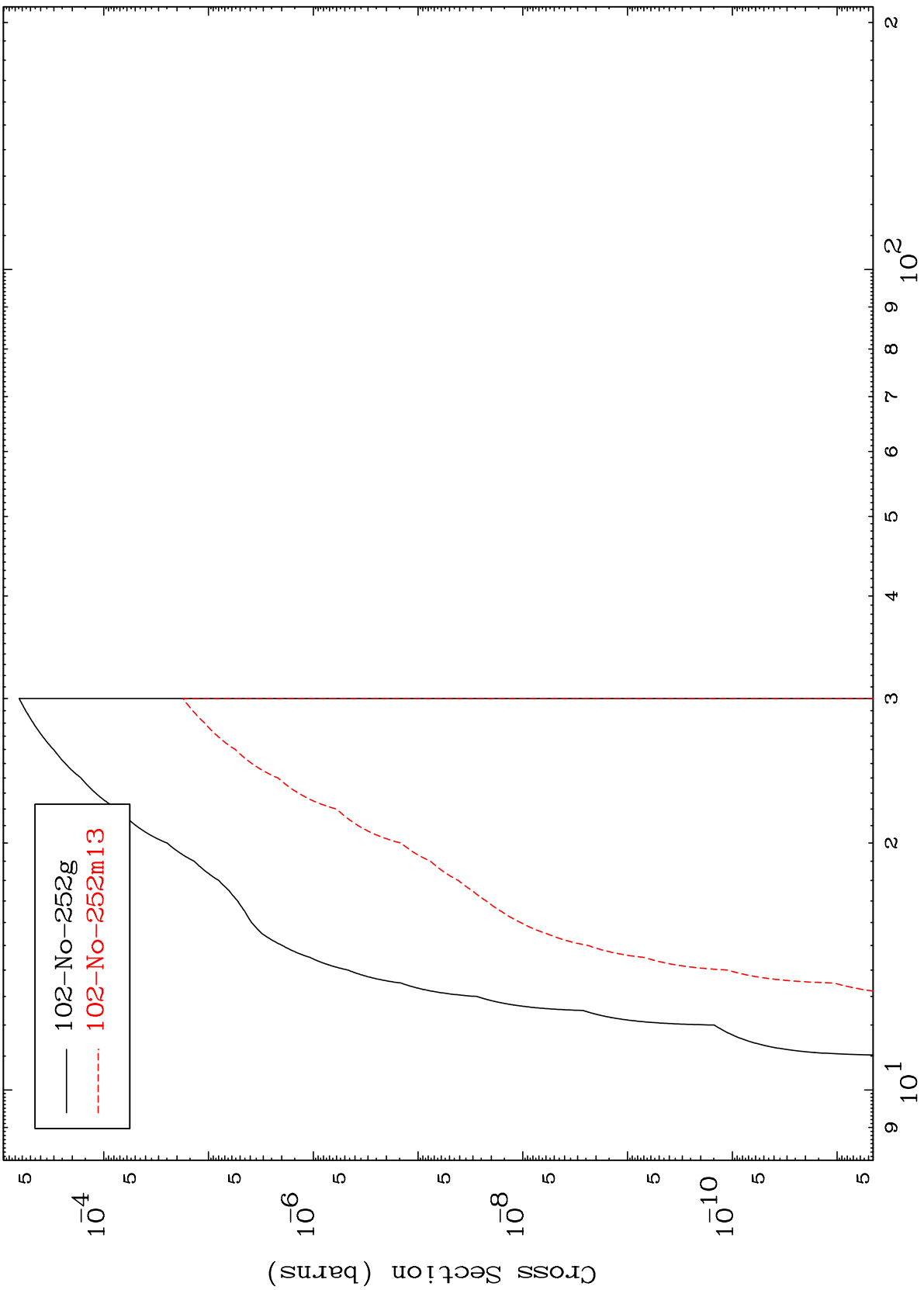
— 101-Md-249g
- - - 101-Md-249m1

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(n,n') p

103-Lr-254

Radionuclide Production Cross Section



14

Incident Energy (MeV)

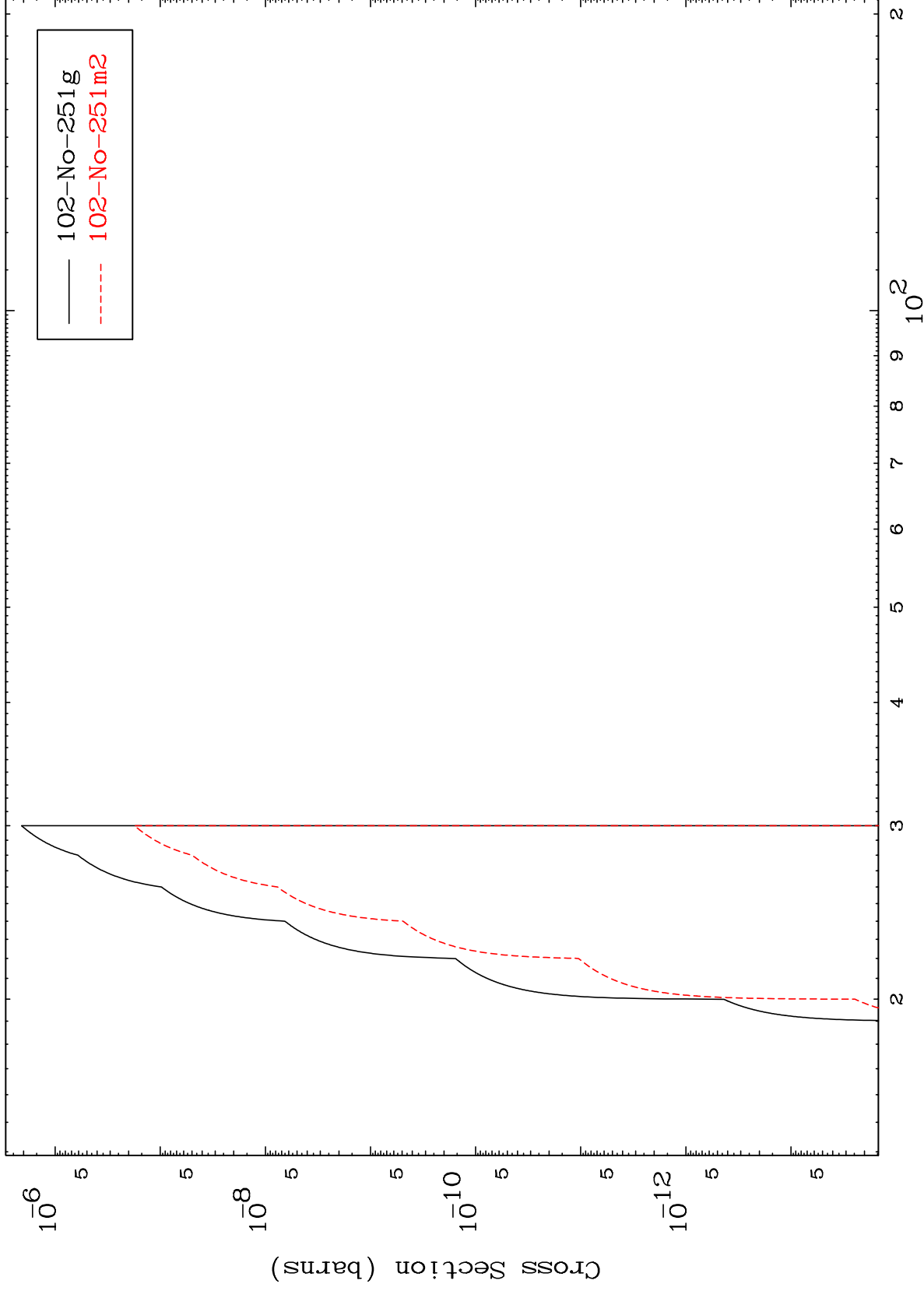
103-Lr-254

MAT 9983

(n,n') d

103-Lr-254

Radionuclide Production Cross Section



15

Incident Energy (MeV)

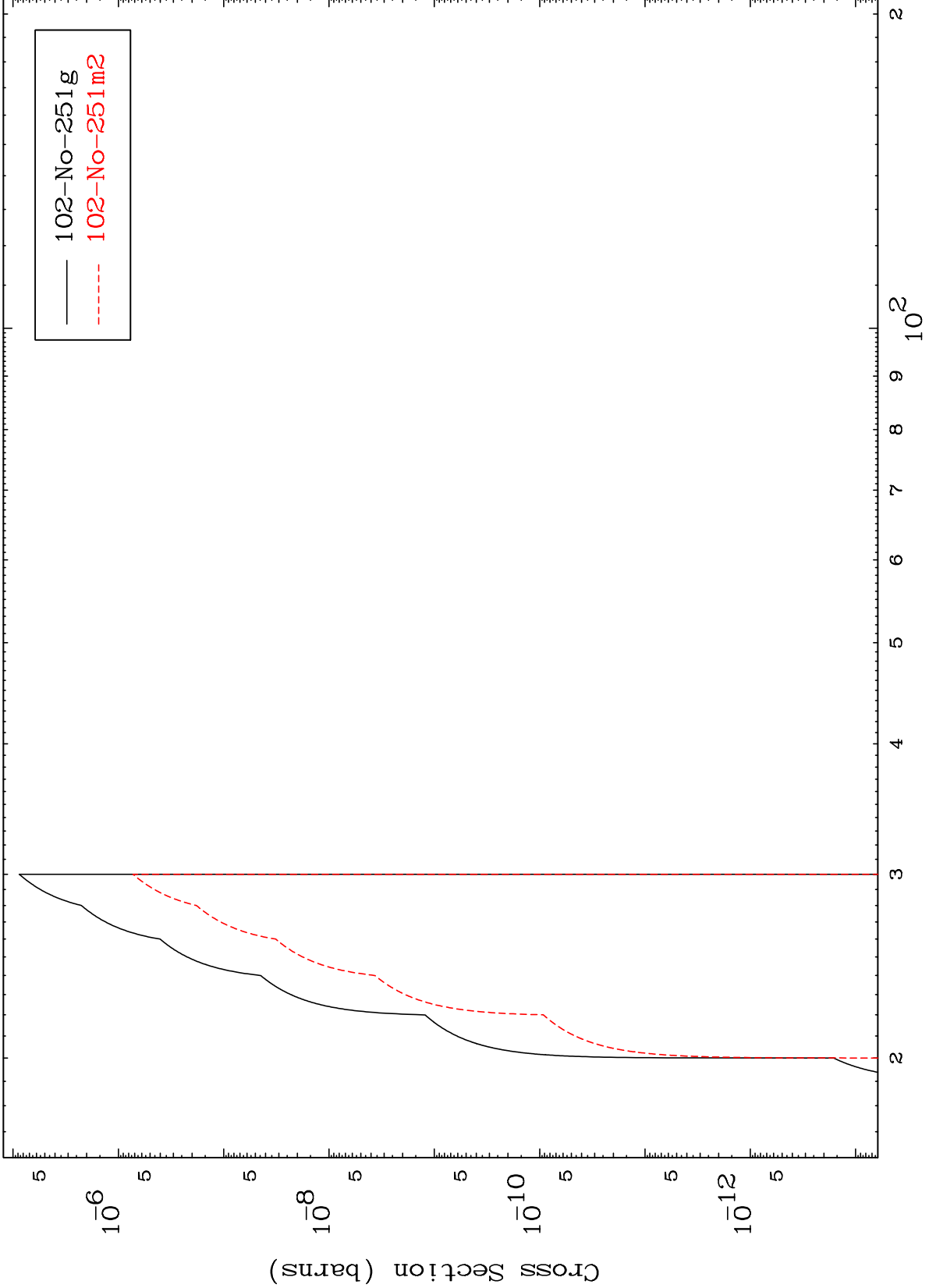
103-Lr-254

MAT 9983

(n,2n) p

103-Lr-254

Radionuclide Production Cross Section



16

Incident Energy (MeV)

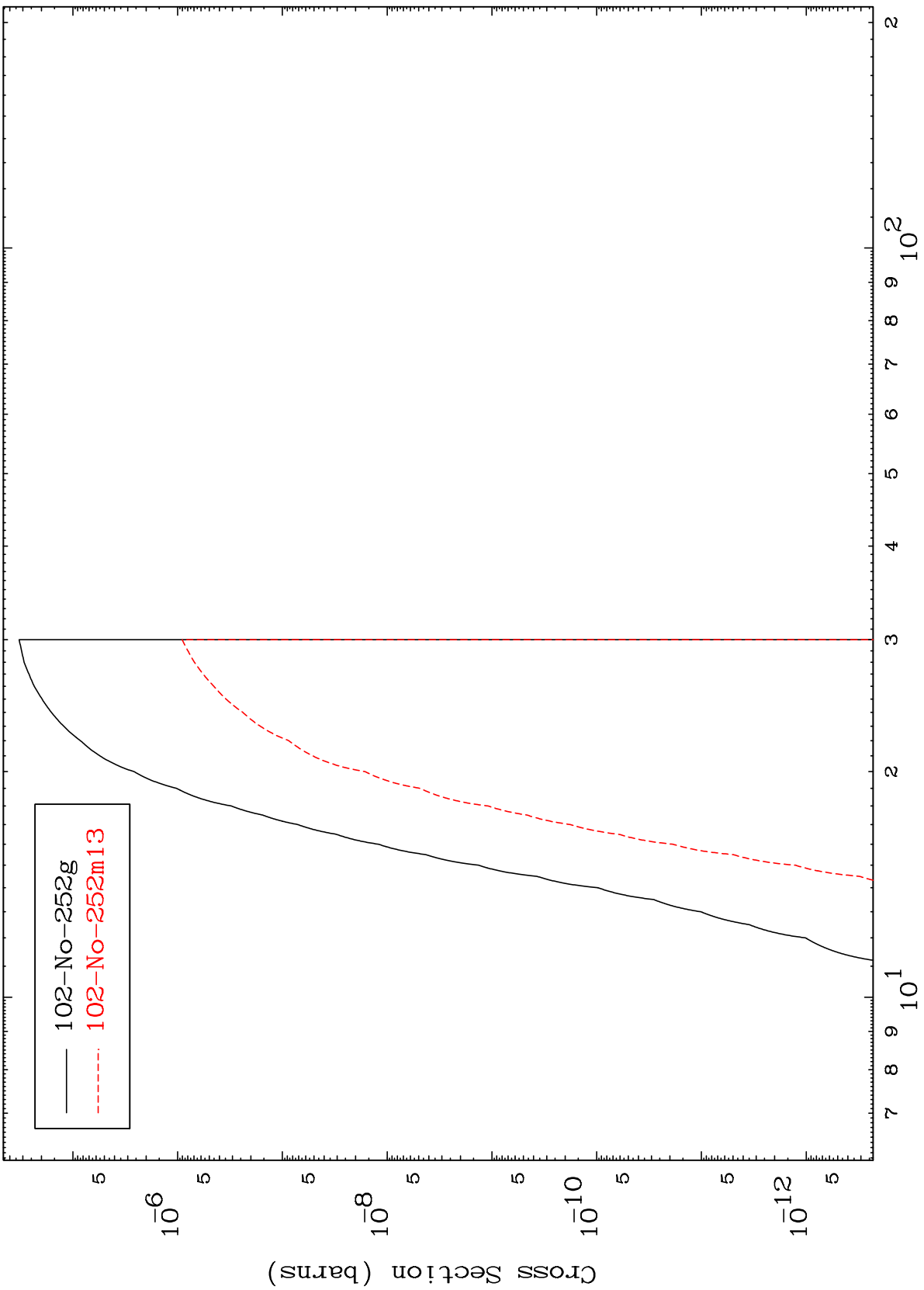
103-Lr-254

MAT 9983

(n,d)

103-Lr-254

Radionuclide Production Cross Section



17

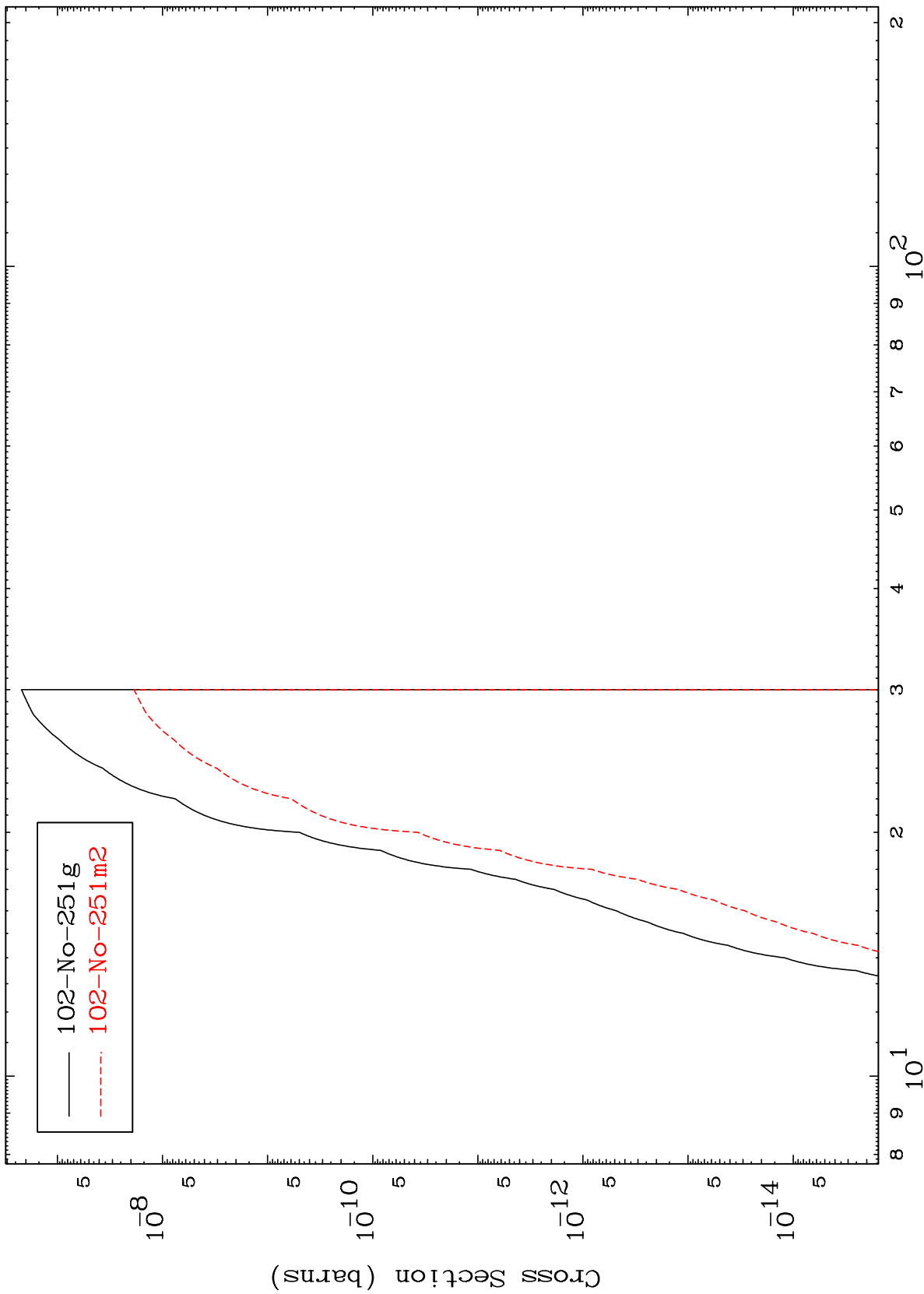
Incident Energy (MeV)

103-Lr-254

MAT 9983

103-Lr-254

(n,t)
Radionuclide Production Cross Section



103-Lr-254

Incident Energy (MeV)

18