

Program EVALPLOT
(Version 2021-1)

by

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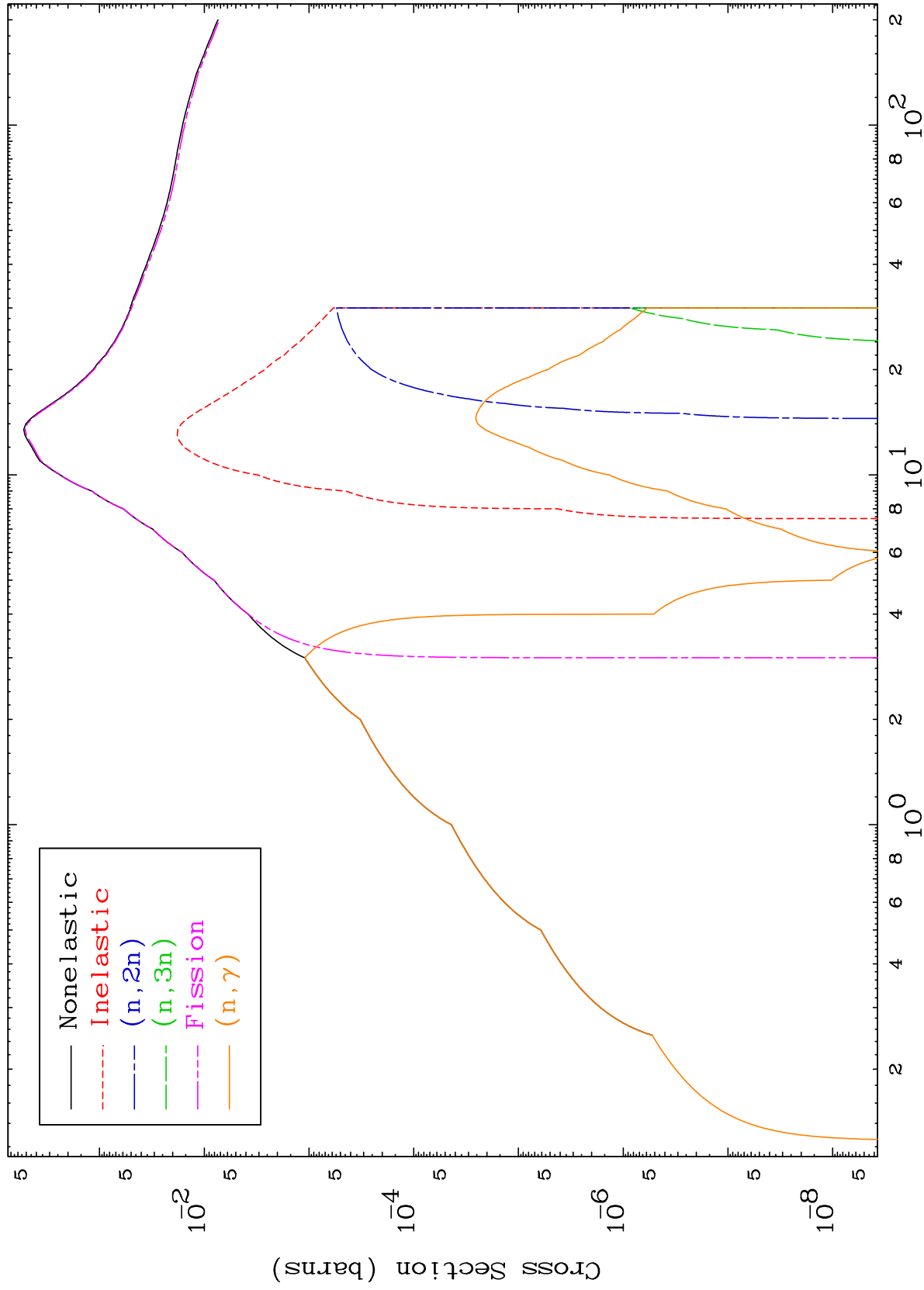
Press Mouse Button to Start

MAT 9952

Photon Major

101-Md-251

0 Kelvin Cross Sections



1

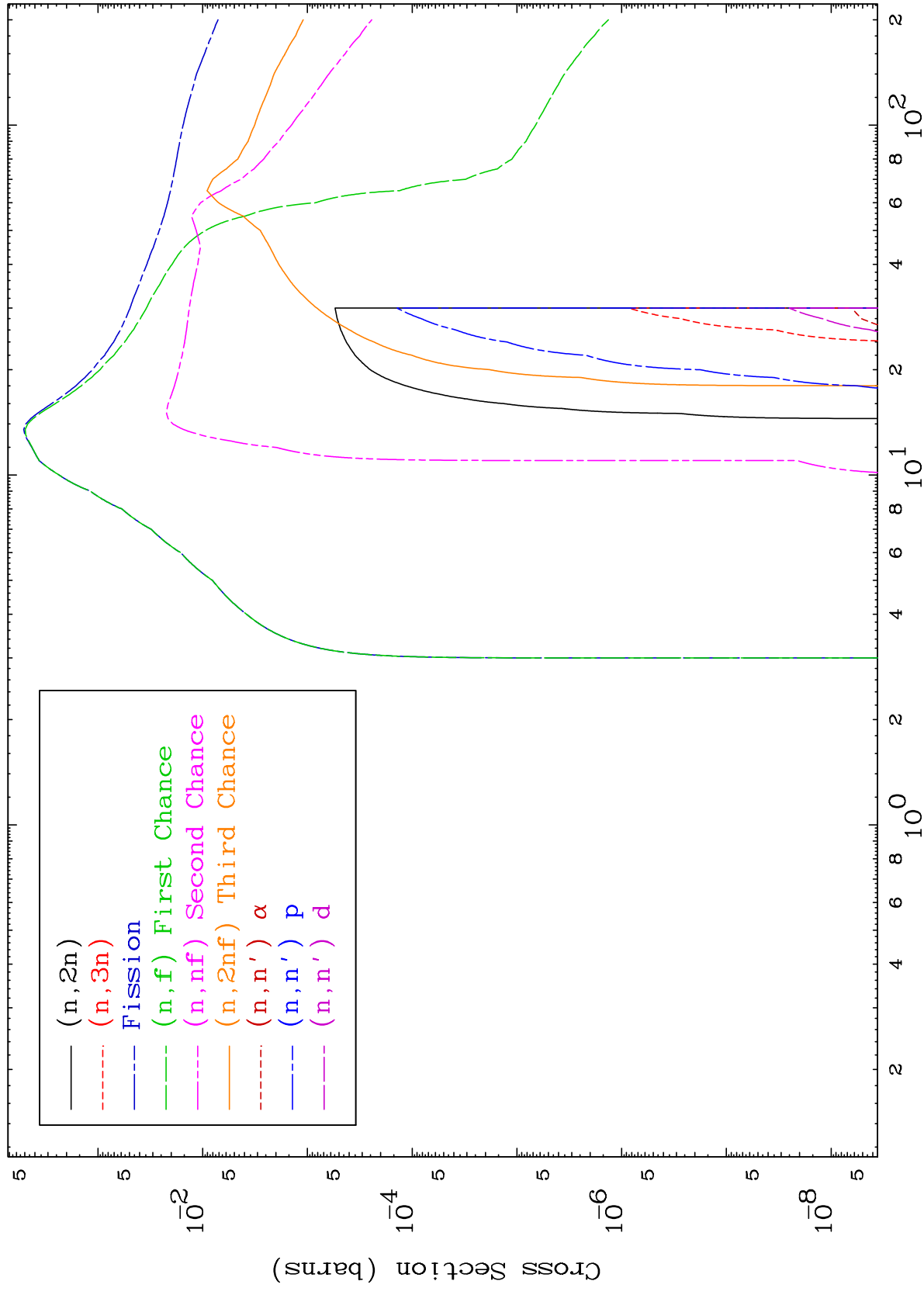
Incident Energy (MeV)

101-Md-251

MAT 9952

Photon Neutron Absorption
0 Kelvin Cross Sections

101-Md-251



2

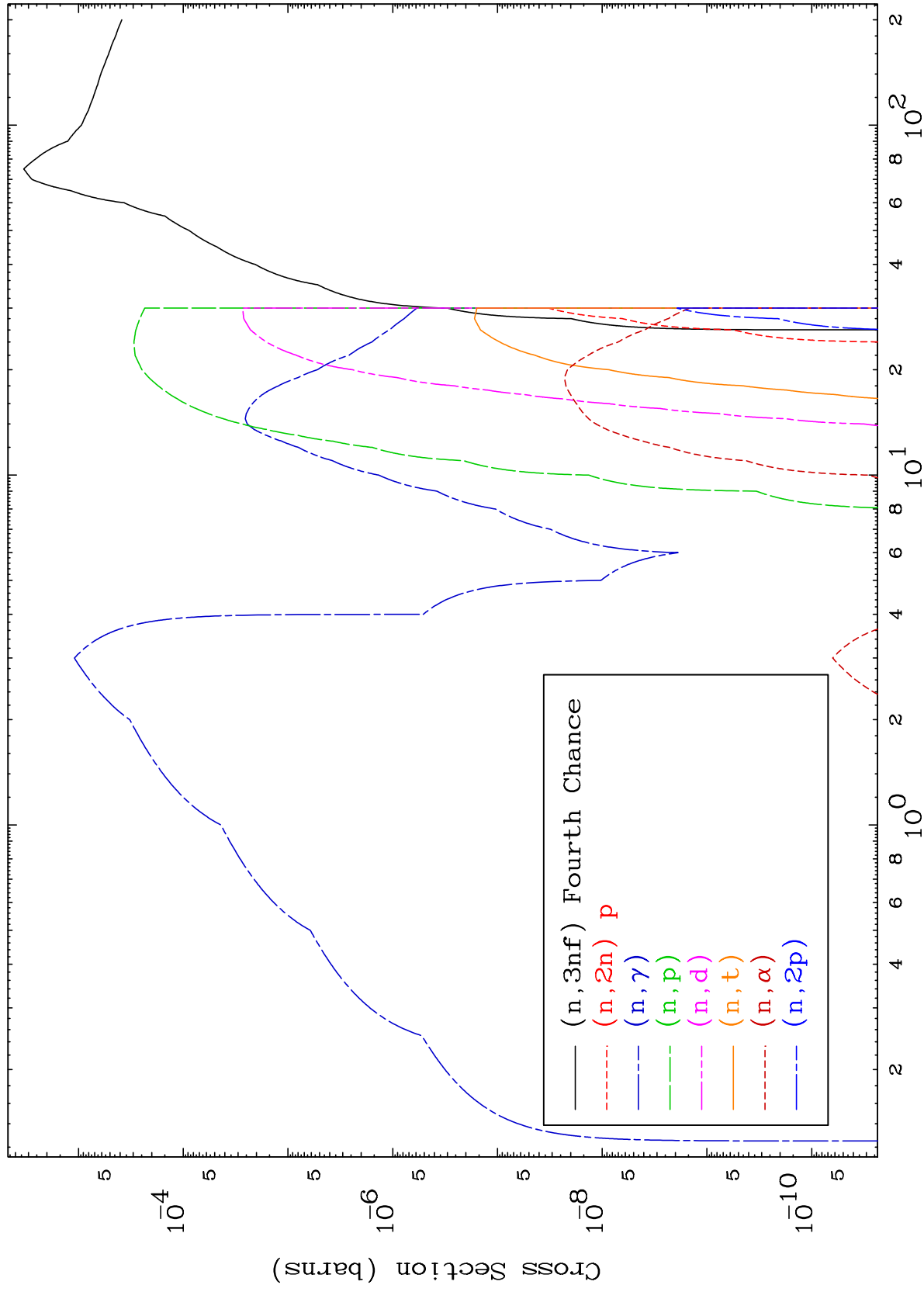
Incident Energy (MeV)

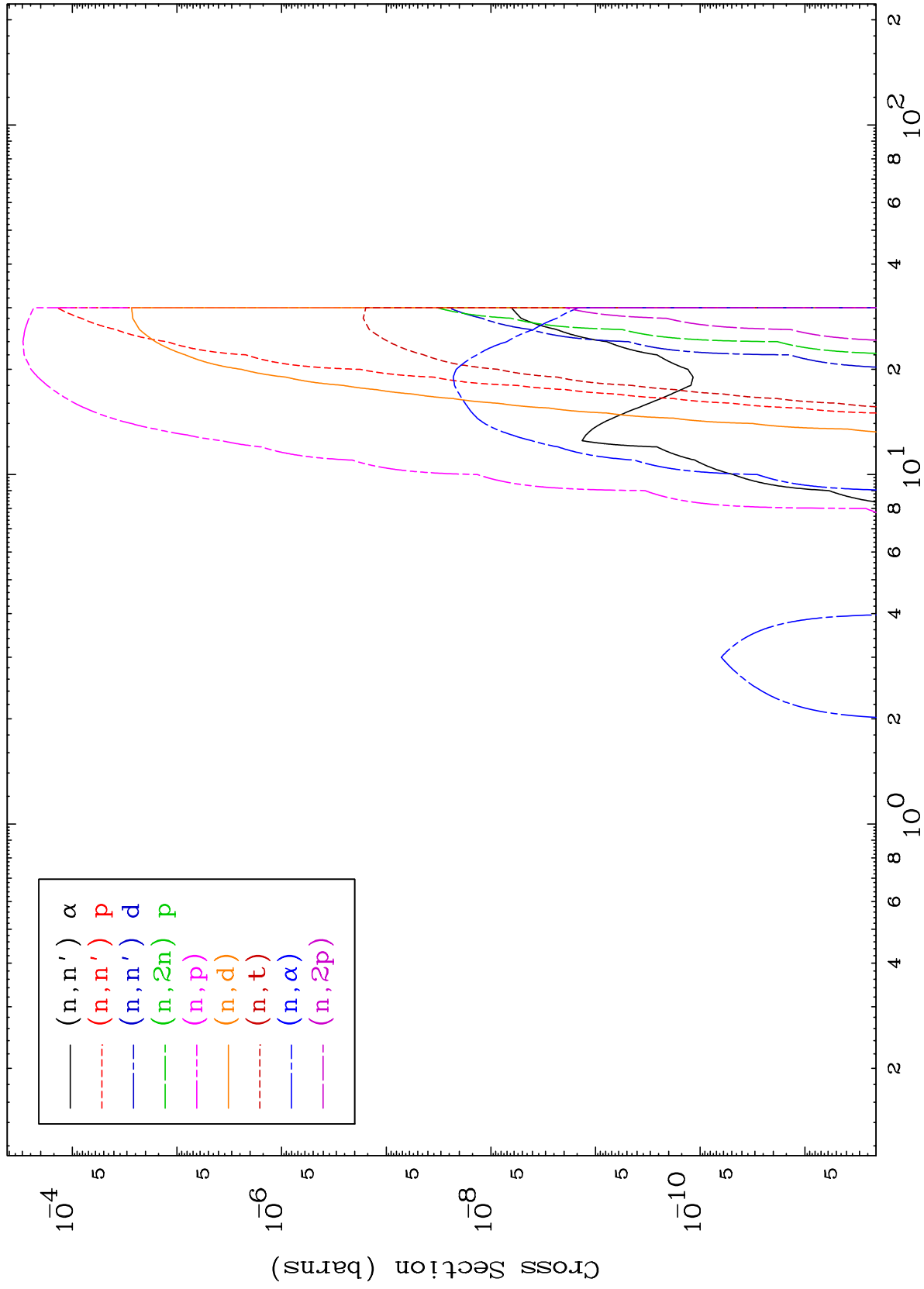
101-Md-251

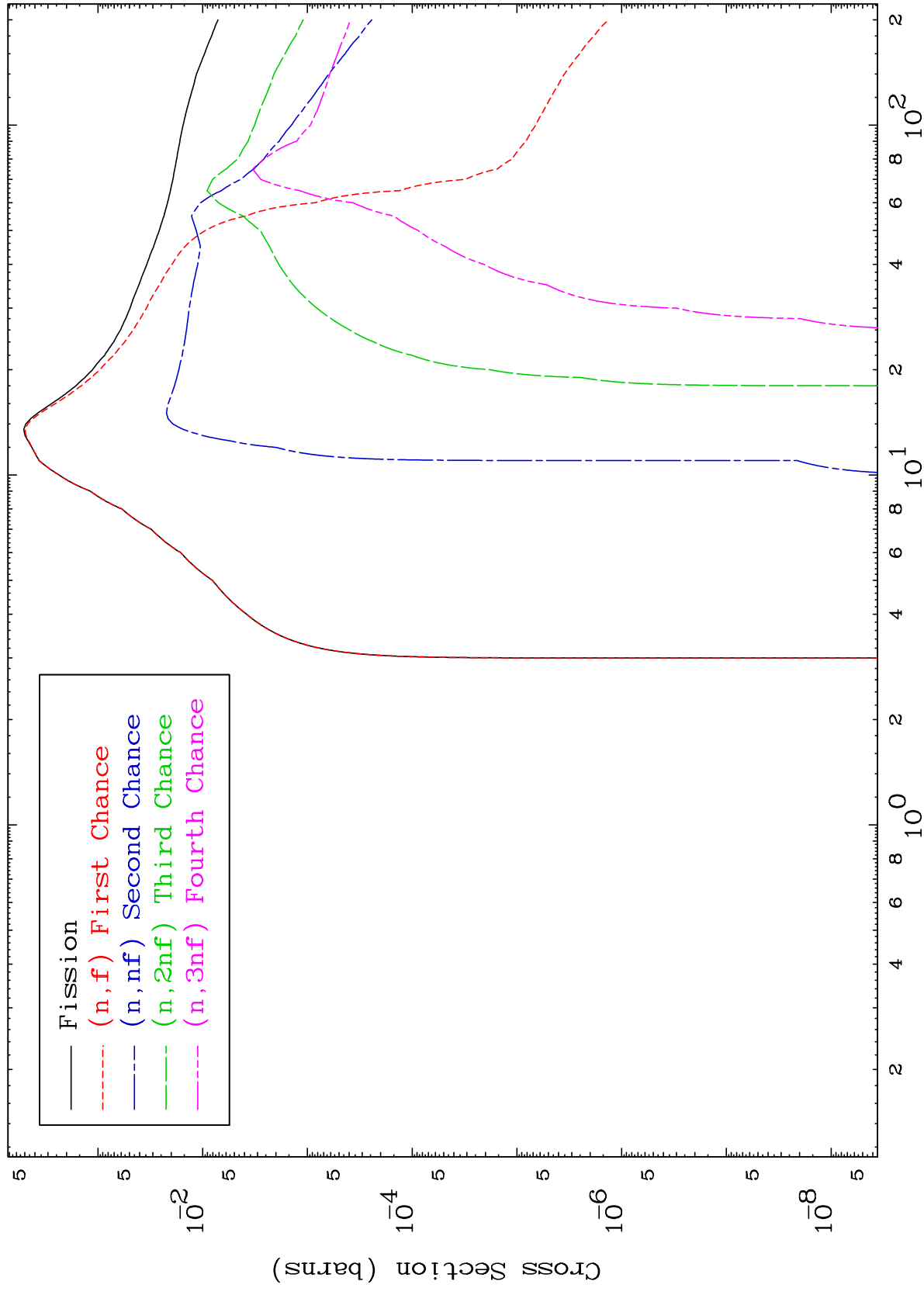
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Photon Neutron Absorption
0 Kelvin Cross Sections

101-Md-251



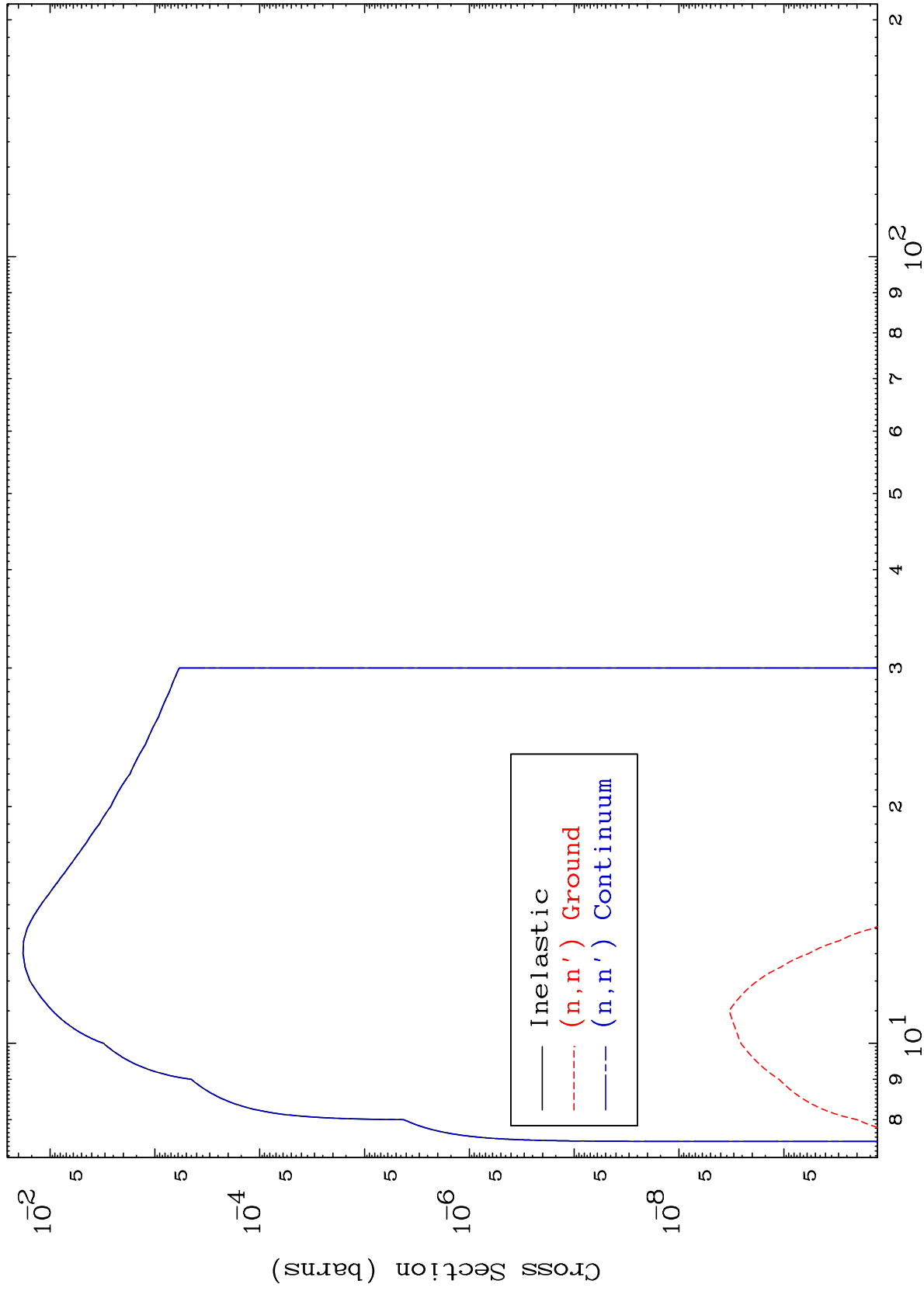




MAT 9952

(γ, n') Levels
0 Kelvin Cross Sections

101-Md-251



6

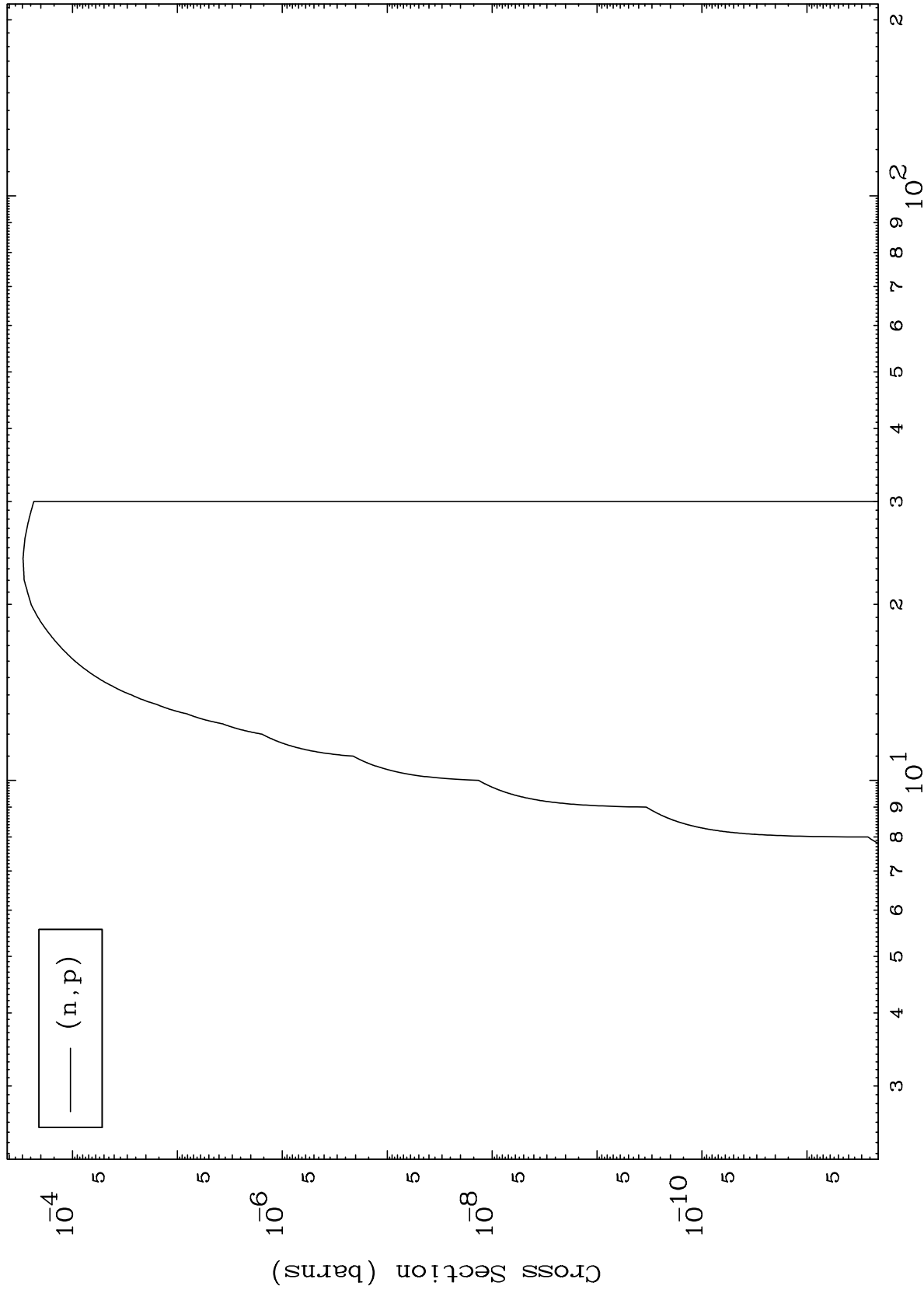
Incident Energy (MeV)

101-Md-251

MAT 9952

101-Md-251

(γ ,p) Levels
0 Kelvin Cross Sections



7

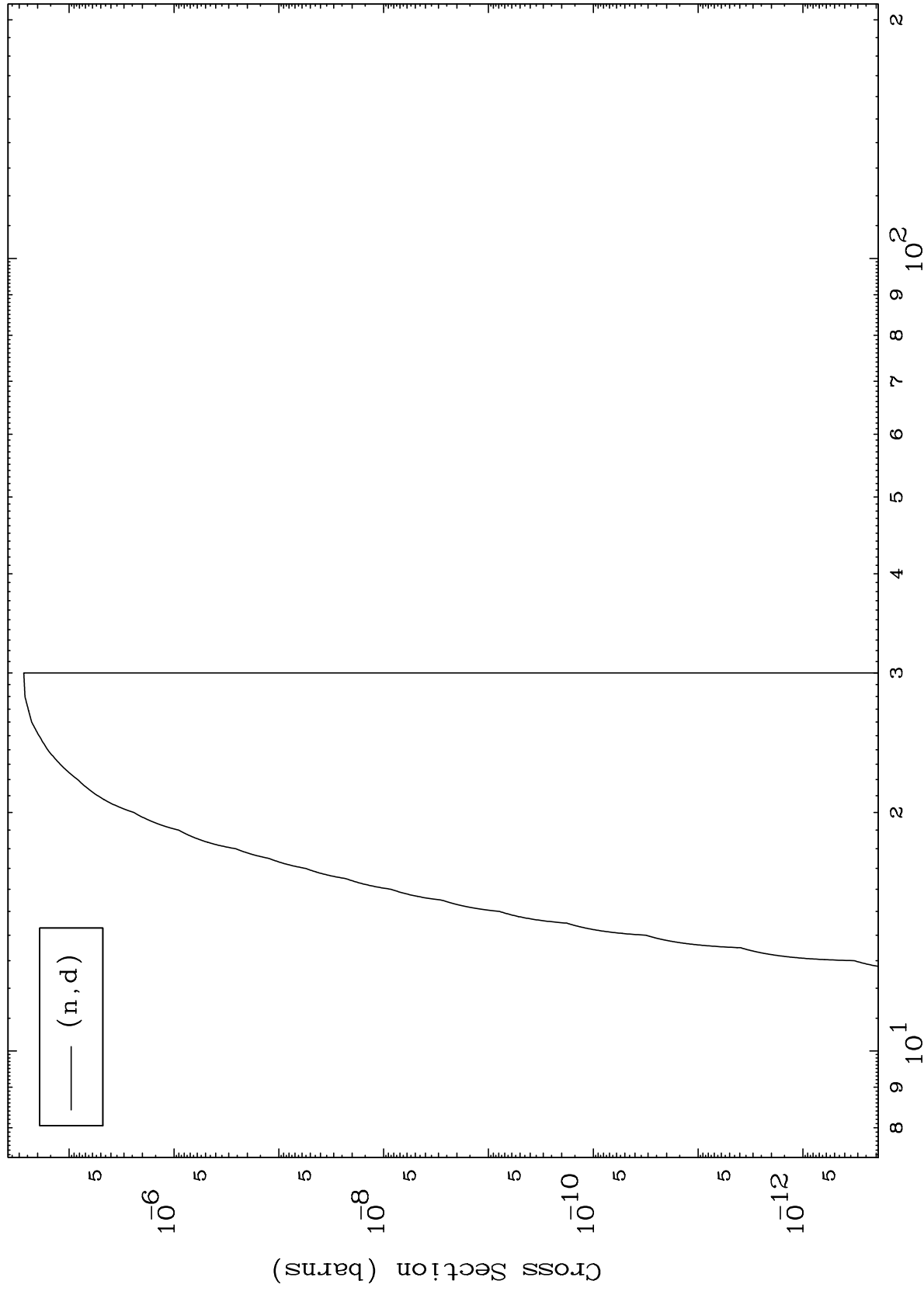
Incident Energy (MeV)

101-Md-251

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101-Md-251

(γ, d) Levels
0 Kelvin Cross Sections



8

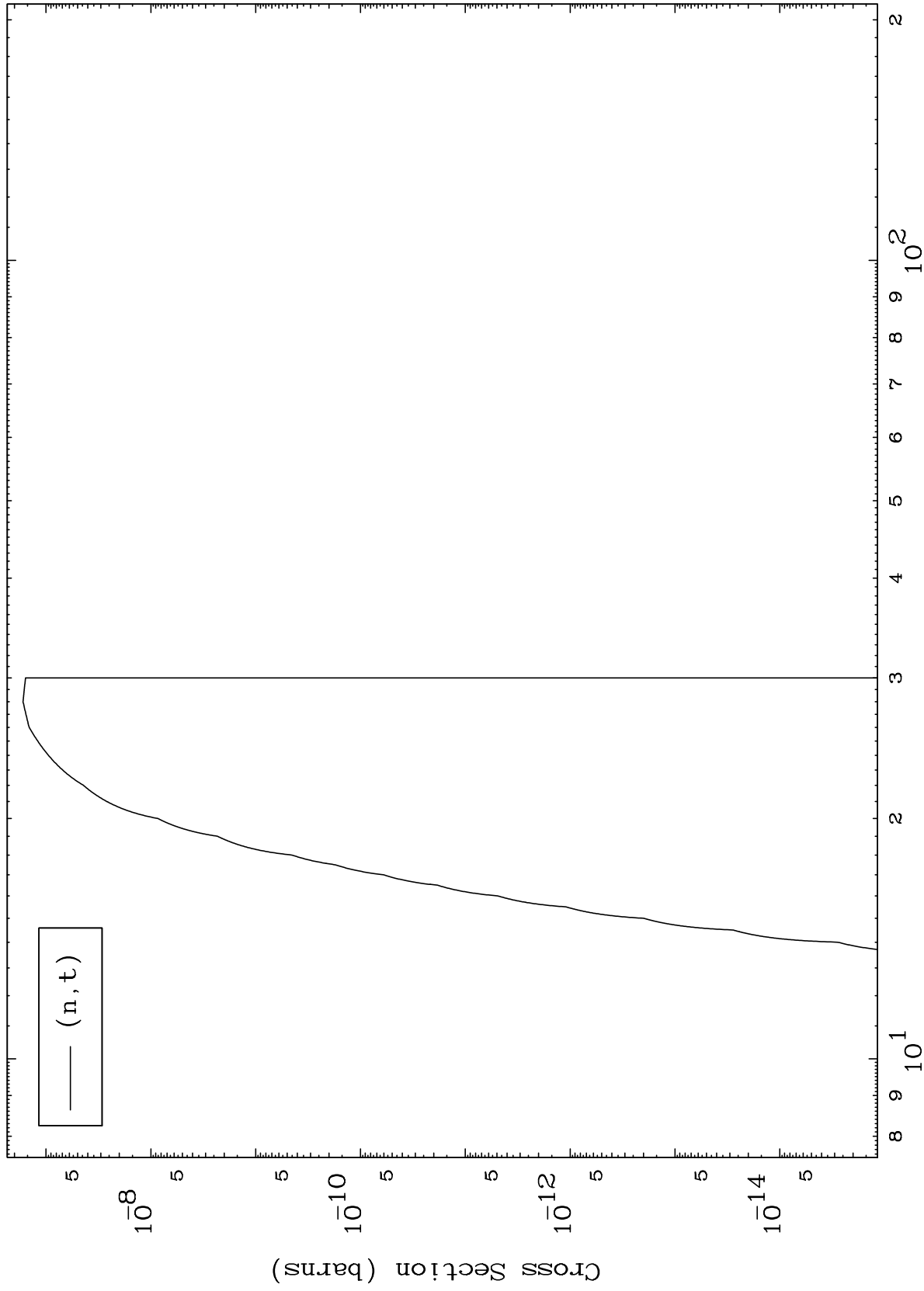
Incident Energy (MeV)

101-Md-251

MAT 9952

101-Md-251

(γ, t) Levels
0 Kelvin Cross Sections



9

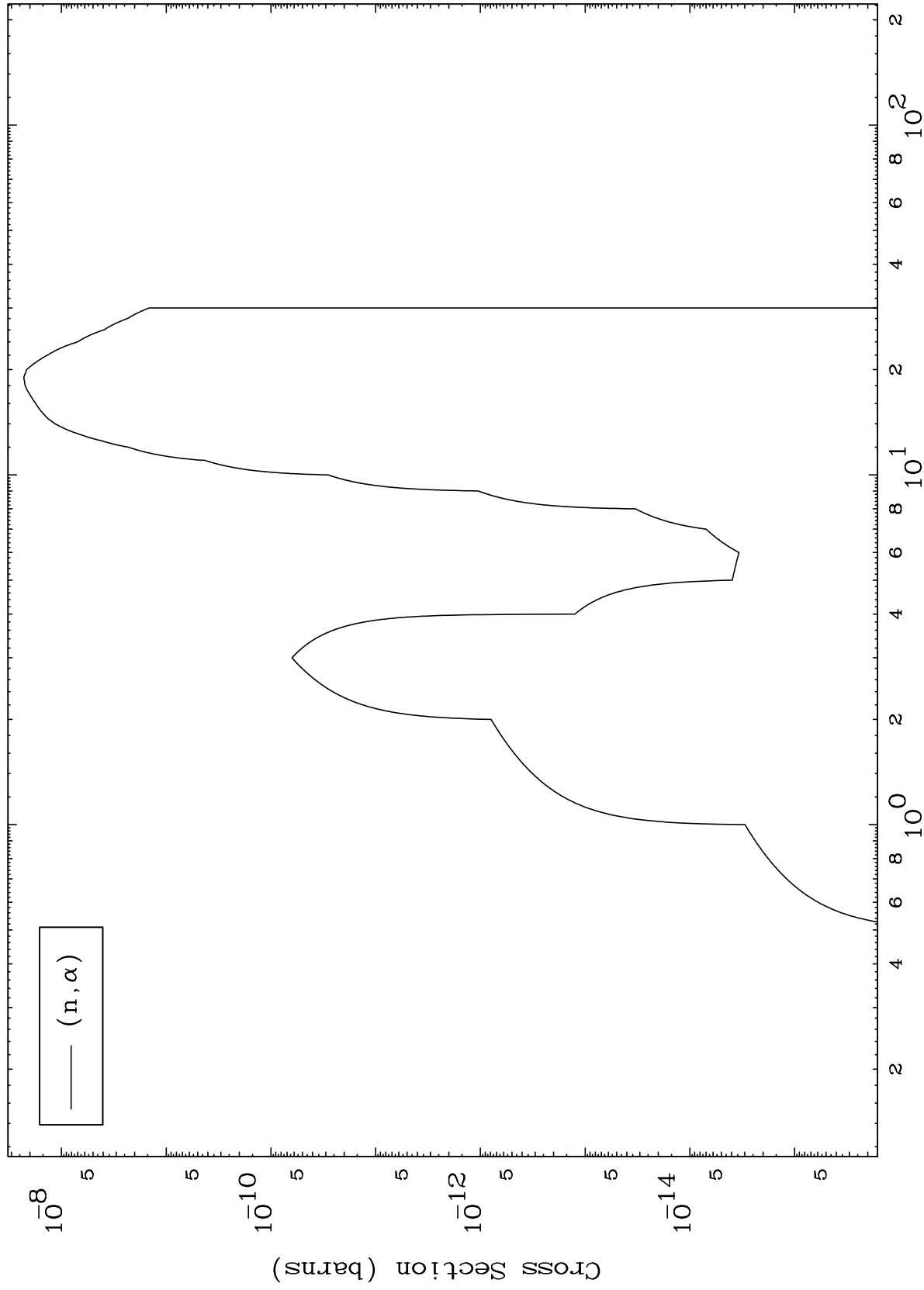
Incident Energy (MeV)

101-Md-251

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101-Md-251

(γ, α) Levels
0 Kelvin Cross Sections



10

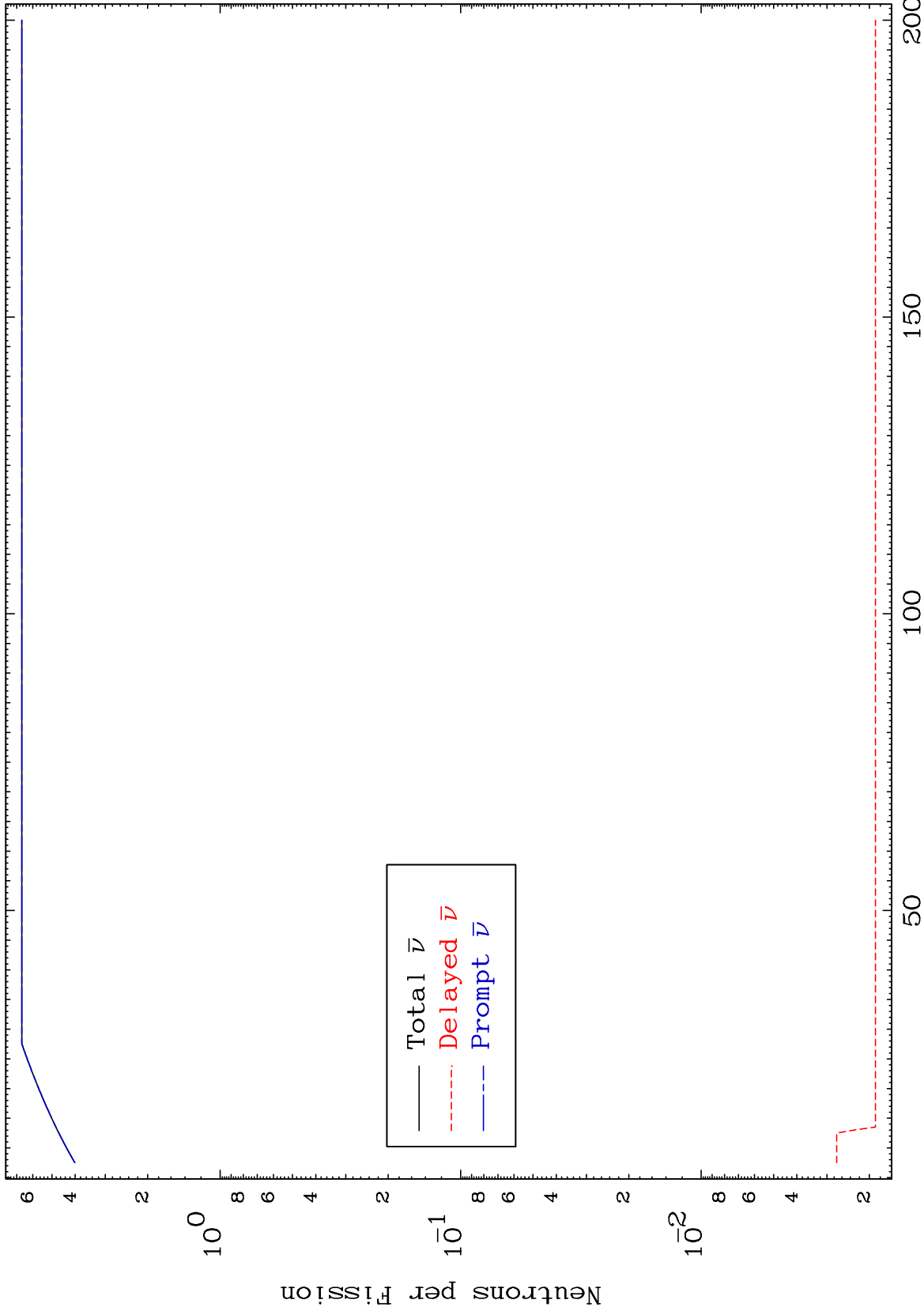
Incident Energy (MeV)

101-Md-251

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Photon Neutrons
per Fission

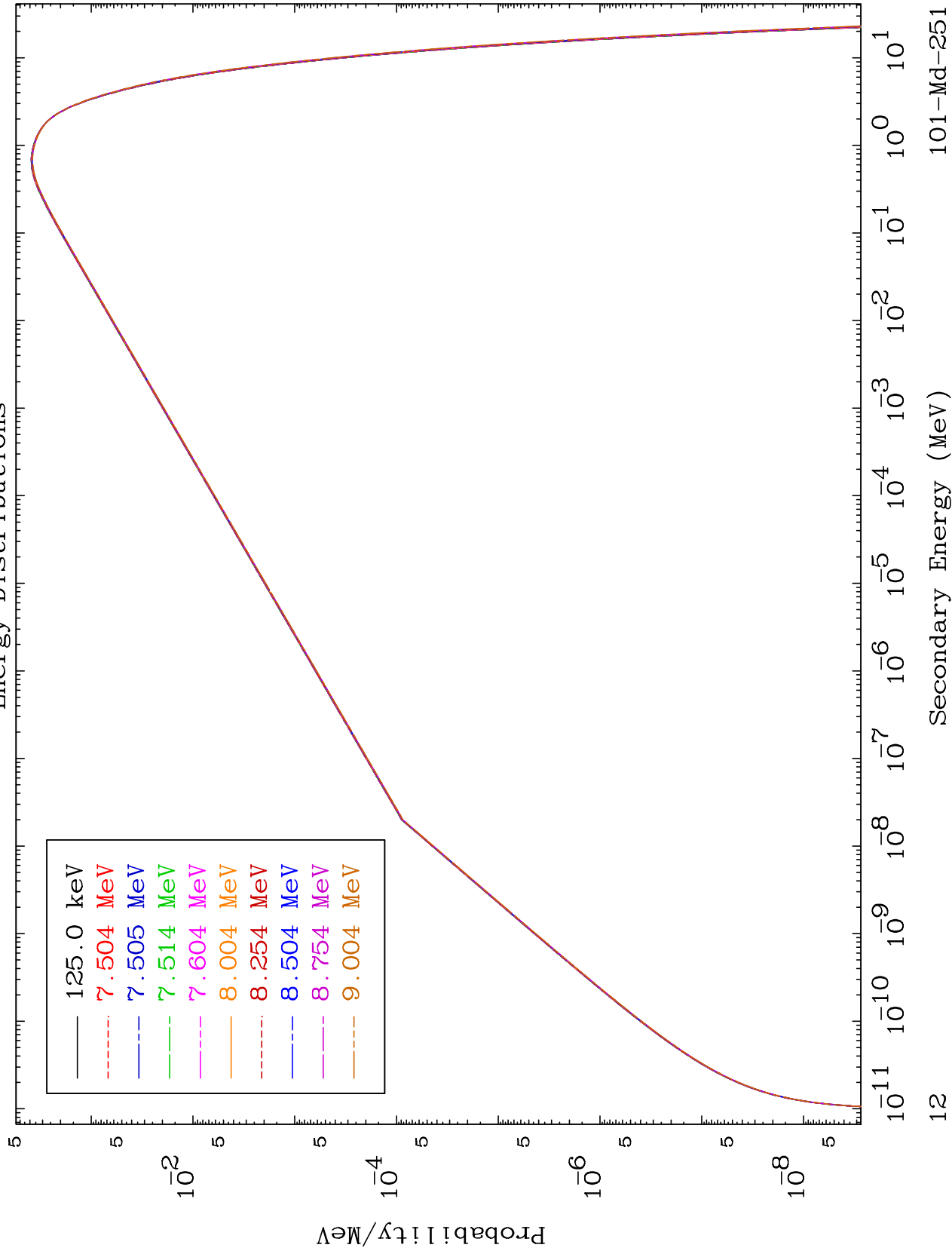
101-Md-251

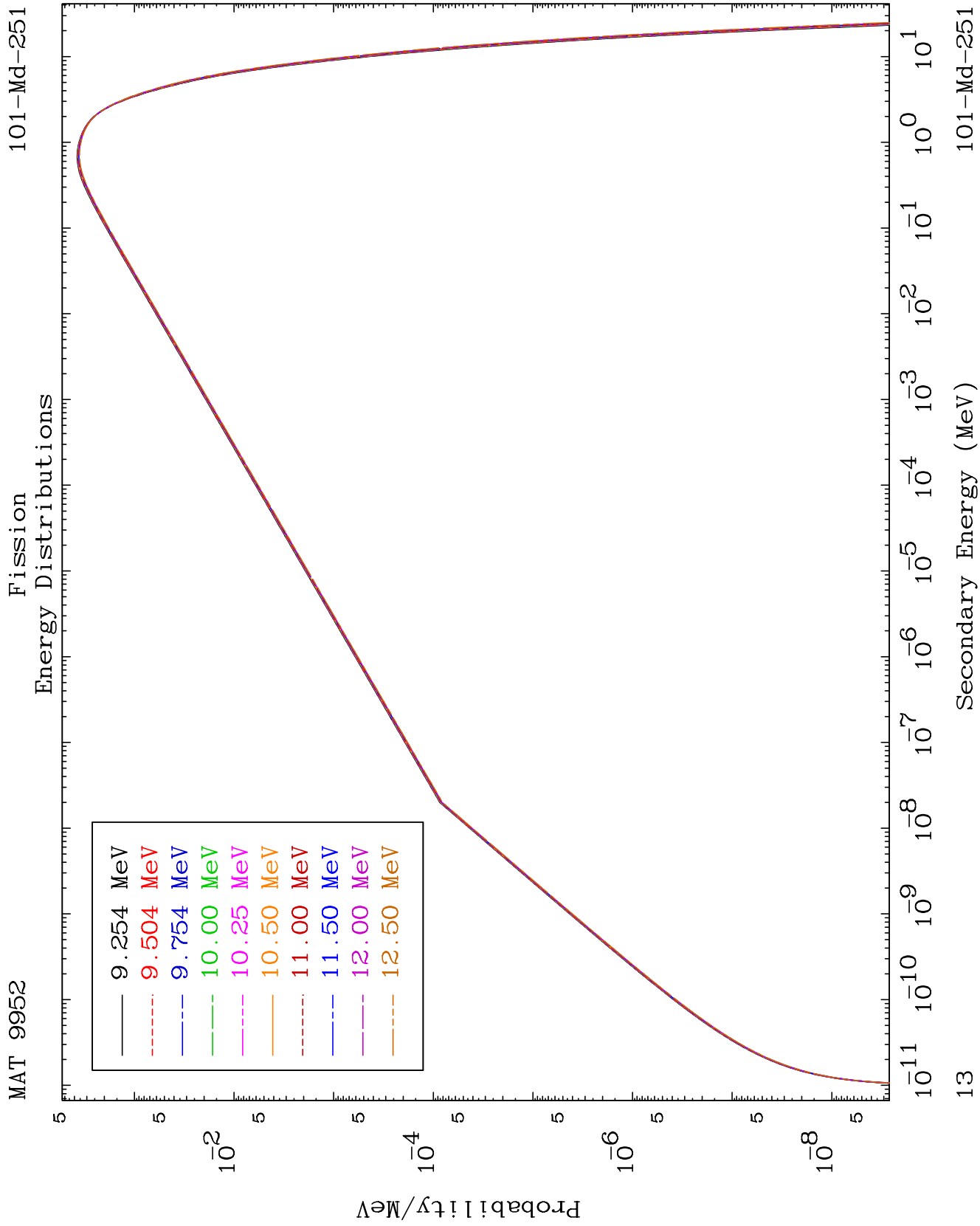


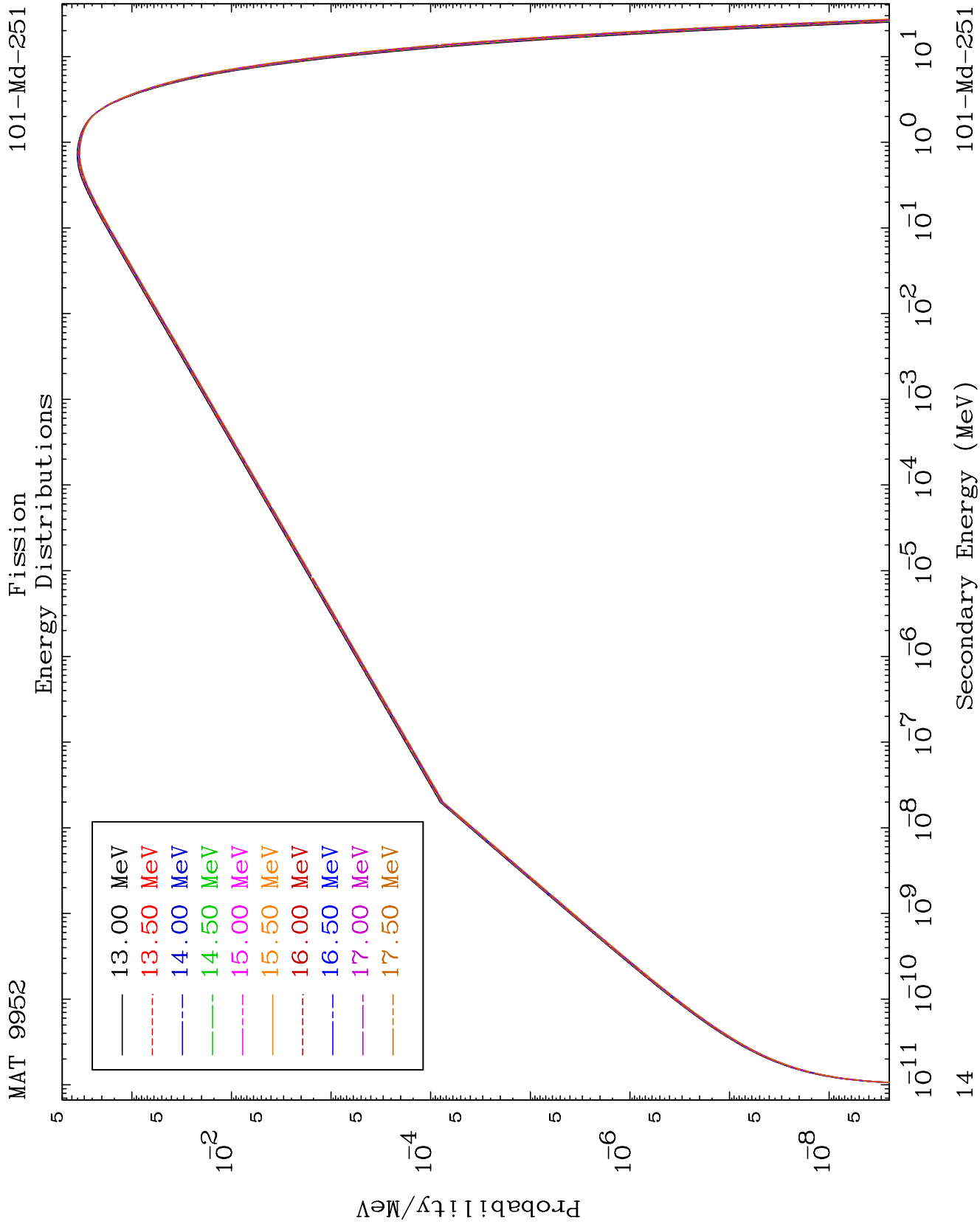
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Fission
Energy Distributions

101-Md-251



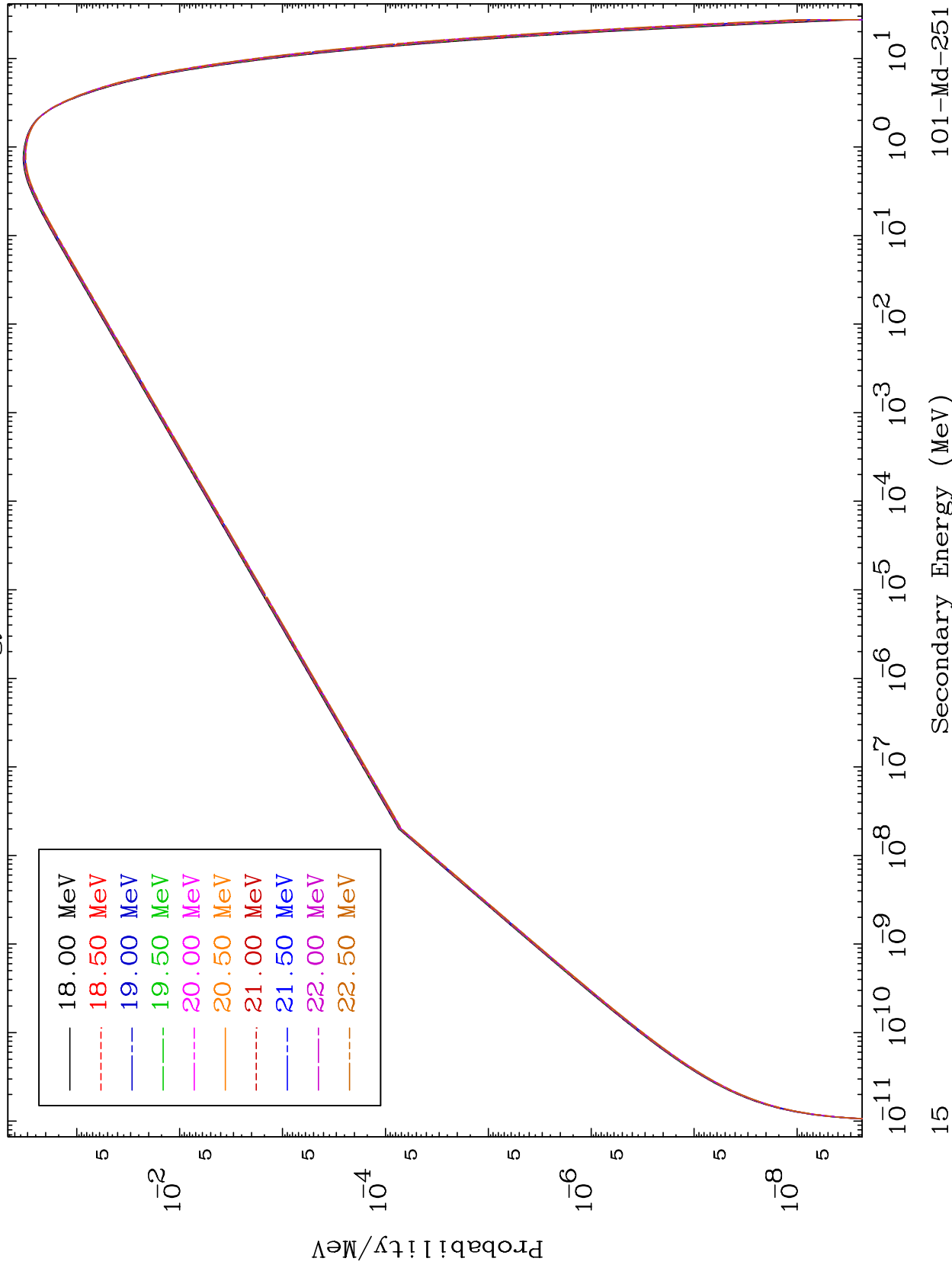




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Fission
Energy Distributions

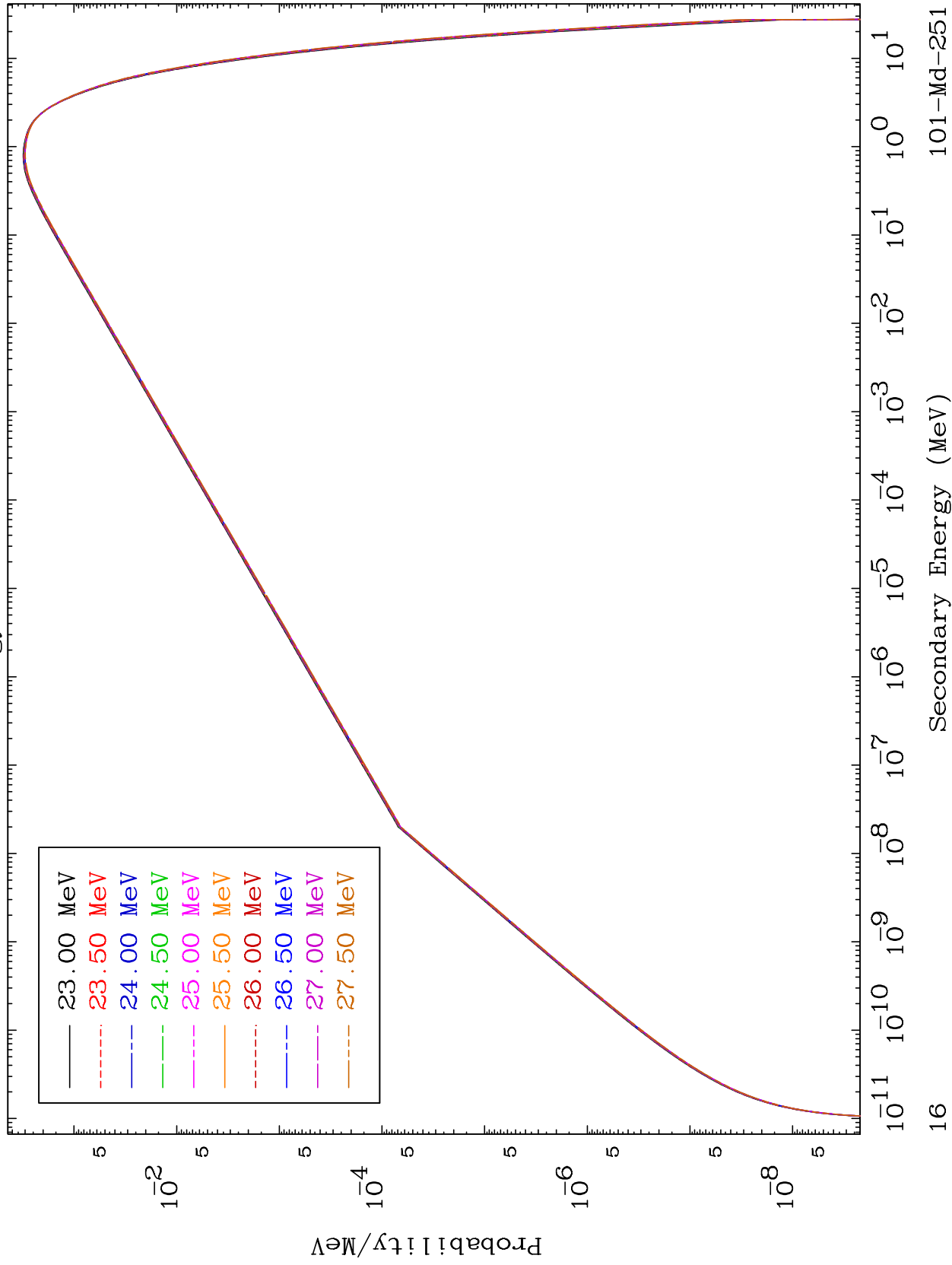
101-Md-251



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Fission
Energy Distributions

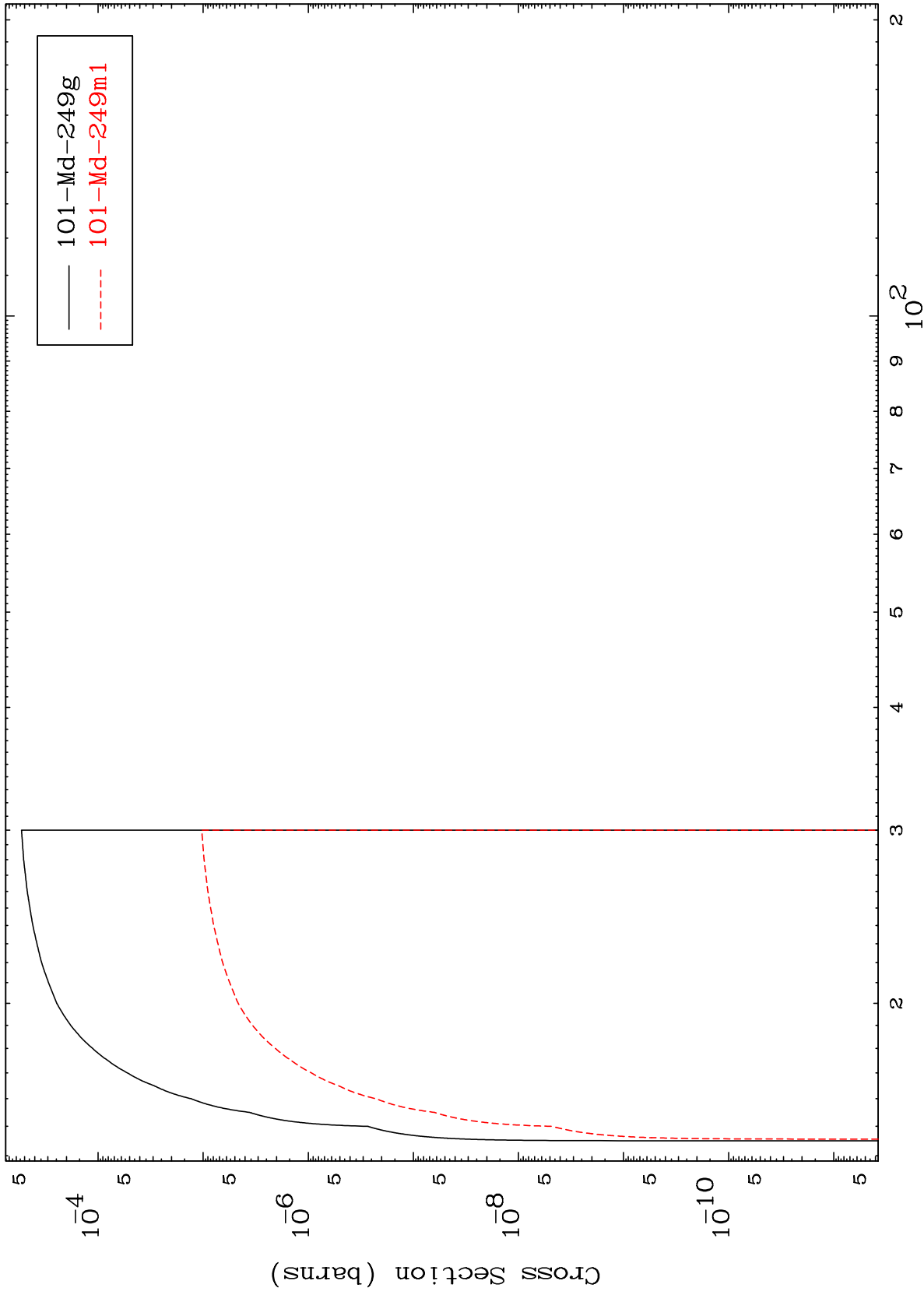
101-Md-251



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101-Md-251

(n,2n)
Radionuclide Production Cross Section

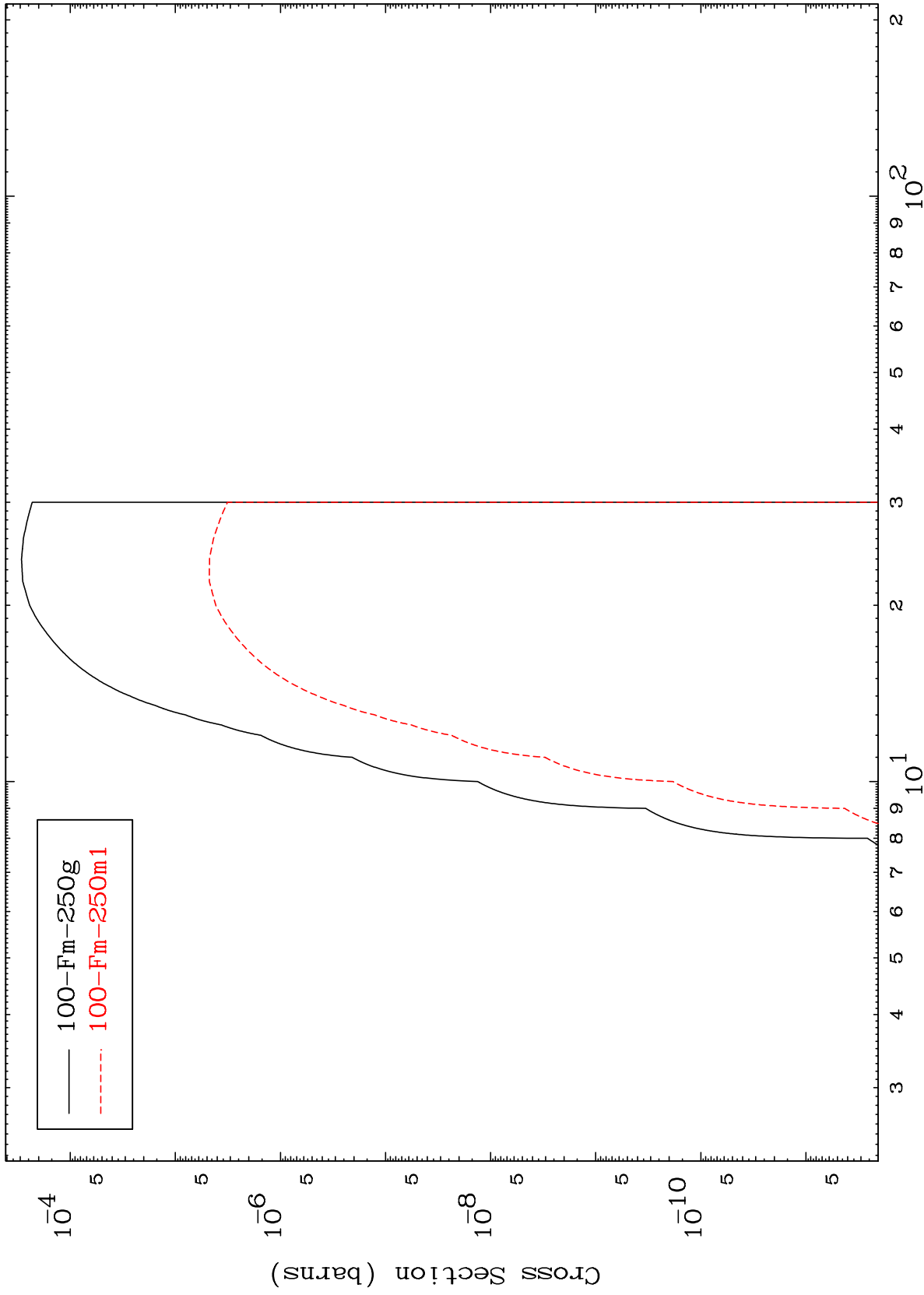


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Incident Energy (MeV)

101-Md-251

Radionuclide Production Cross Section
(n,p)

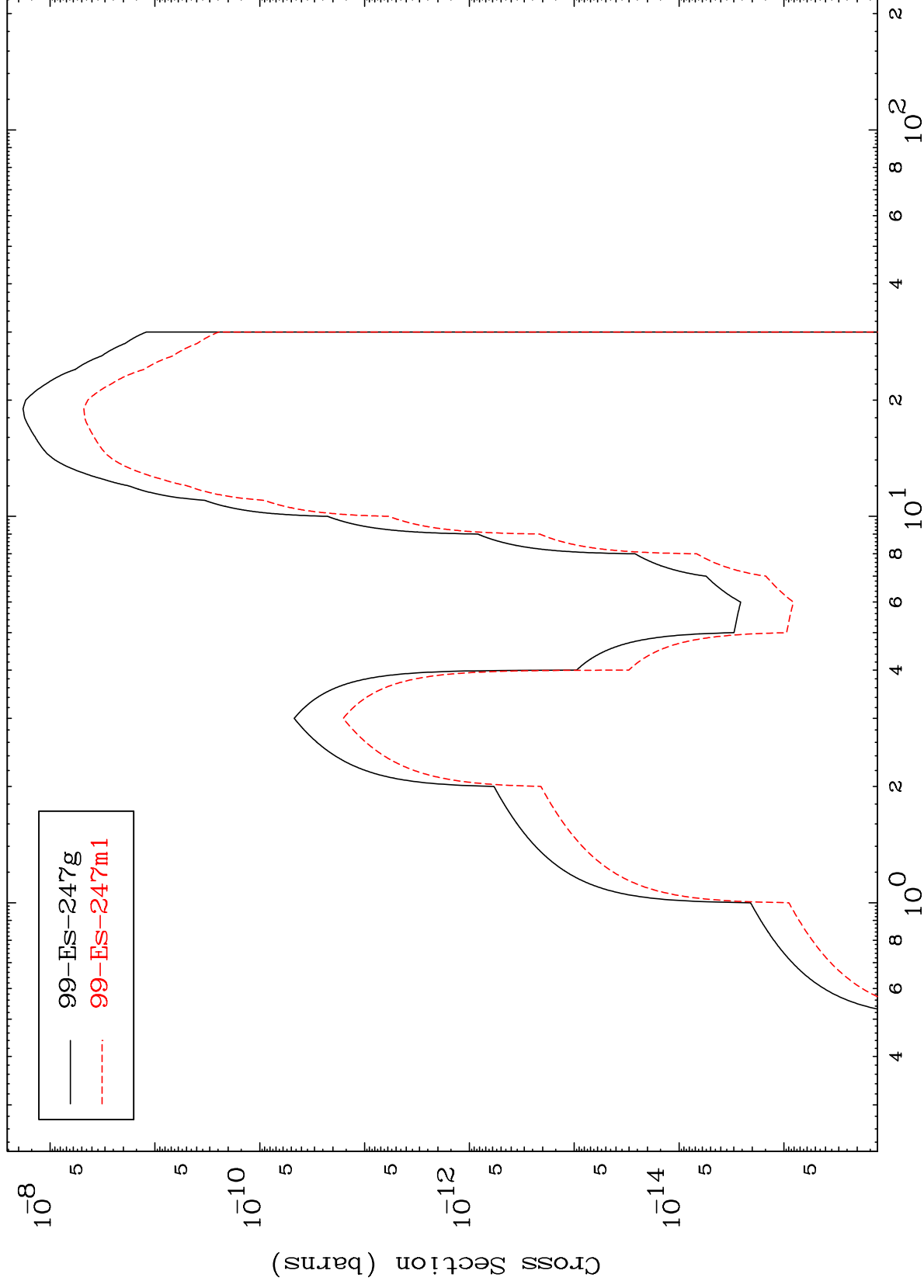


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(n,α)

101-Md-251

Radionuclide Production Cross Section



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Incident Energy (MeV)

101-Md-251