

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

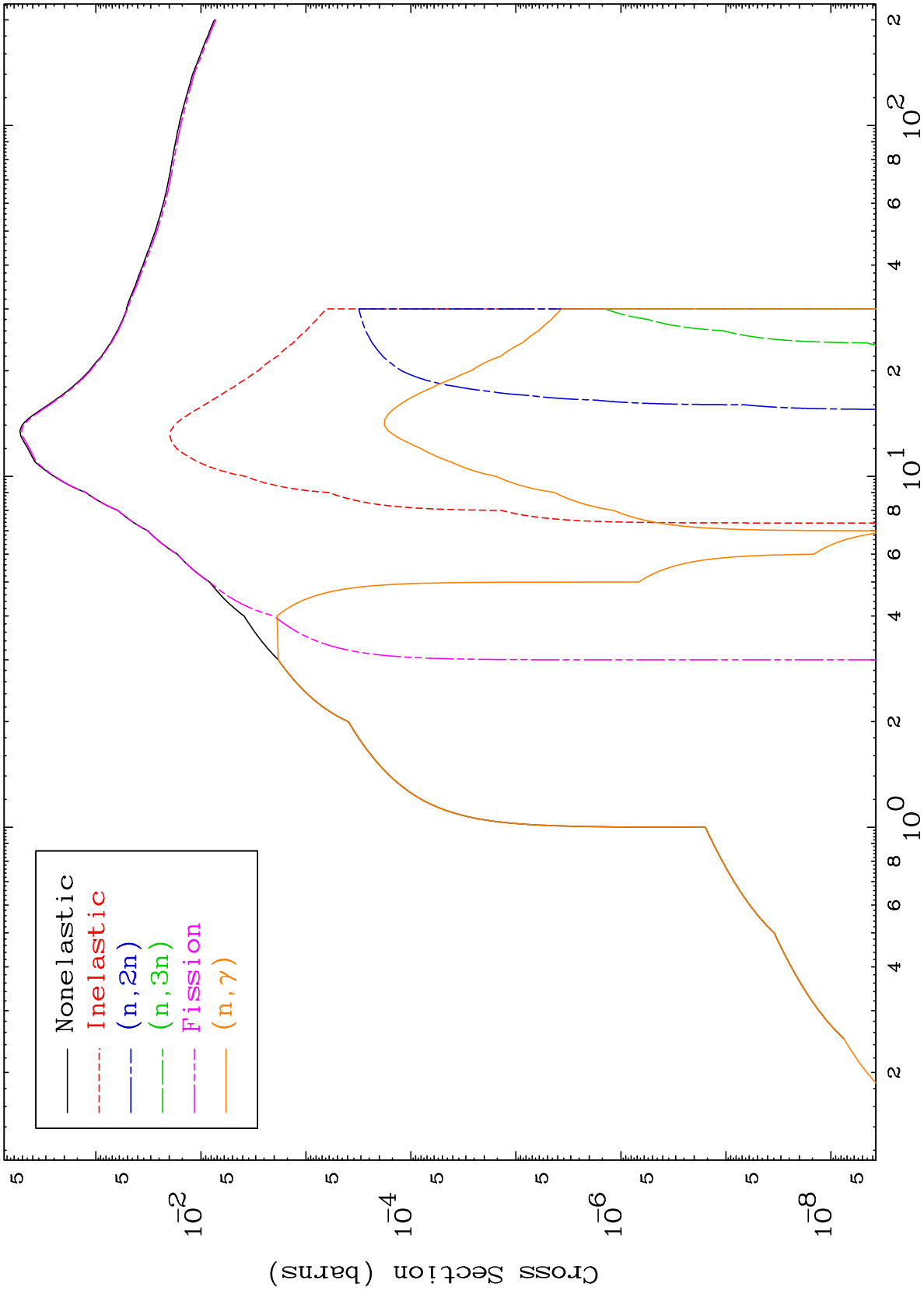
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9954

Photon Major
0 Kelvin Cross Sections

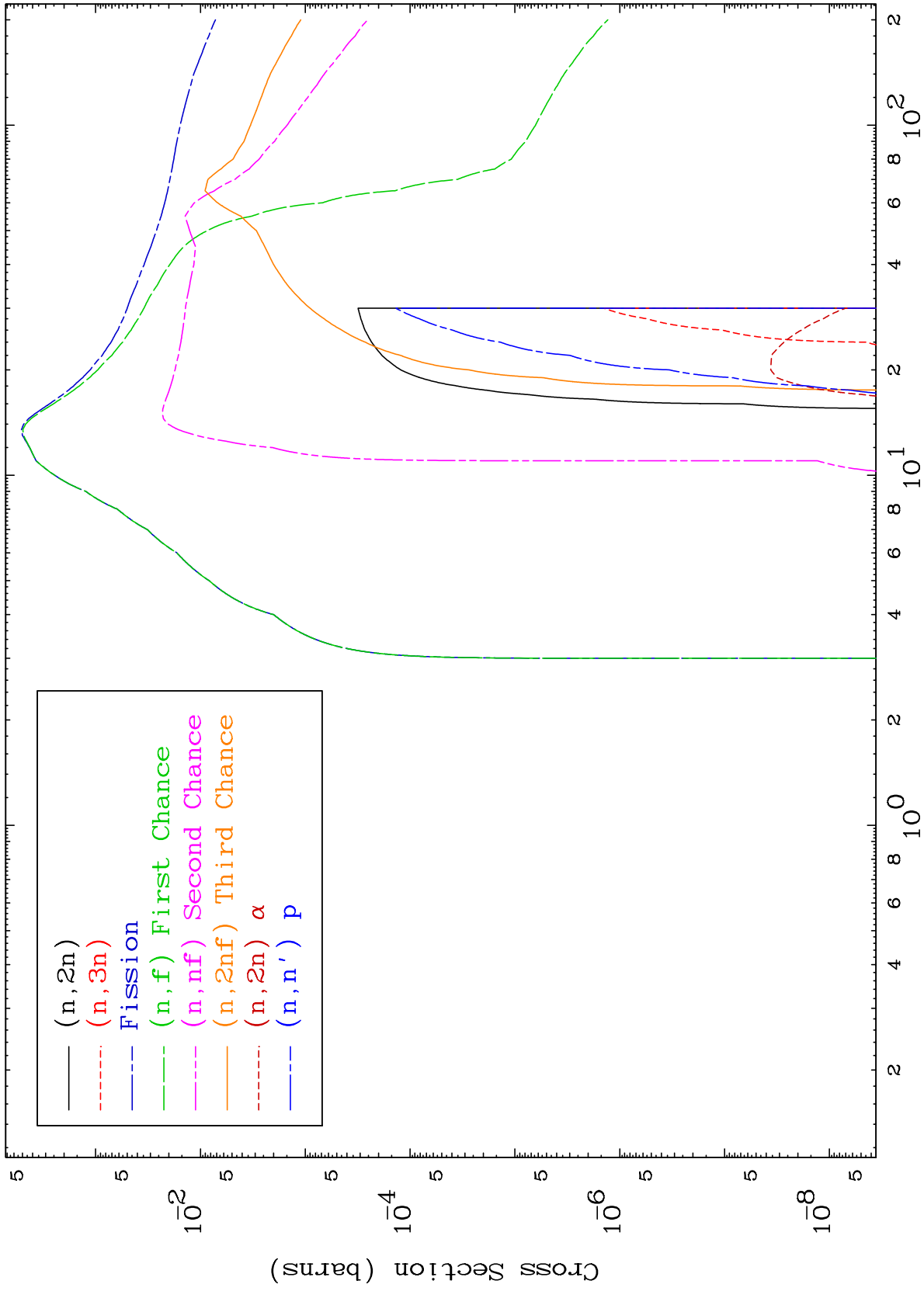
101-Md-253



MAT 9954

Photon Neutron Absorption
0 Kelvin Cross Sections

101-Md-253



2

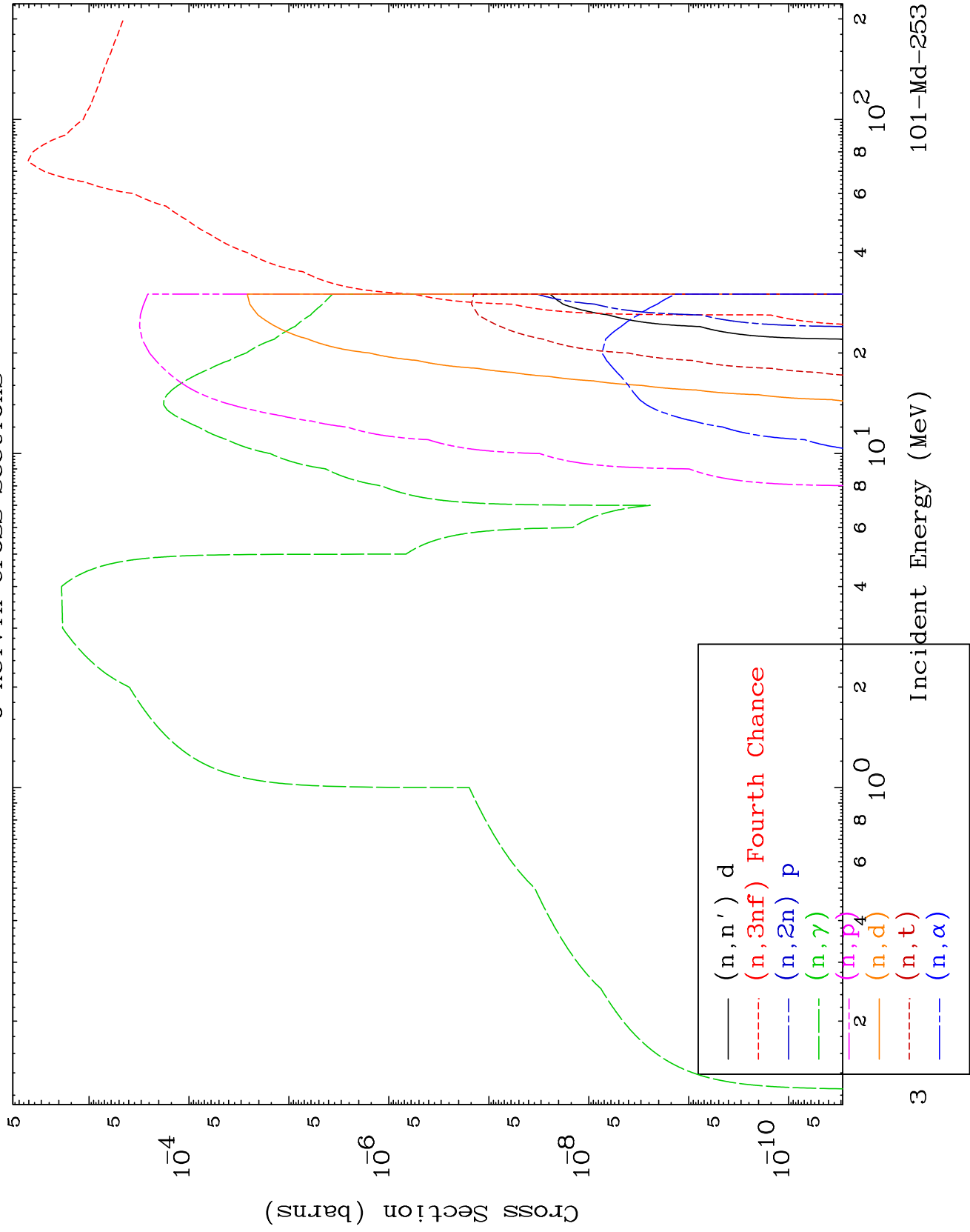
Incident Energy (MeV)

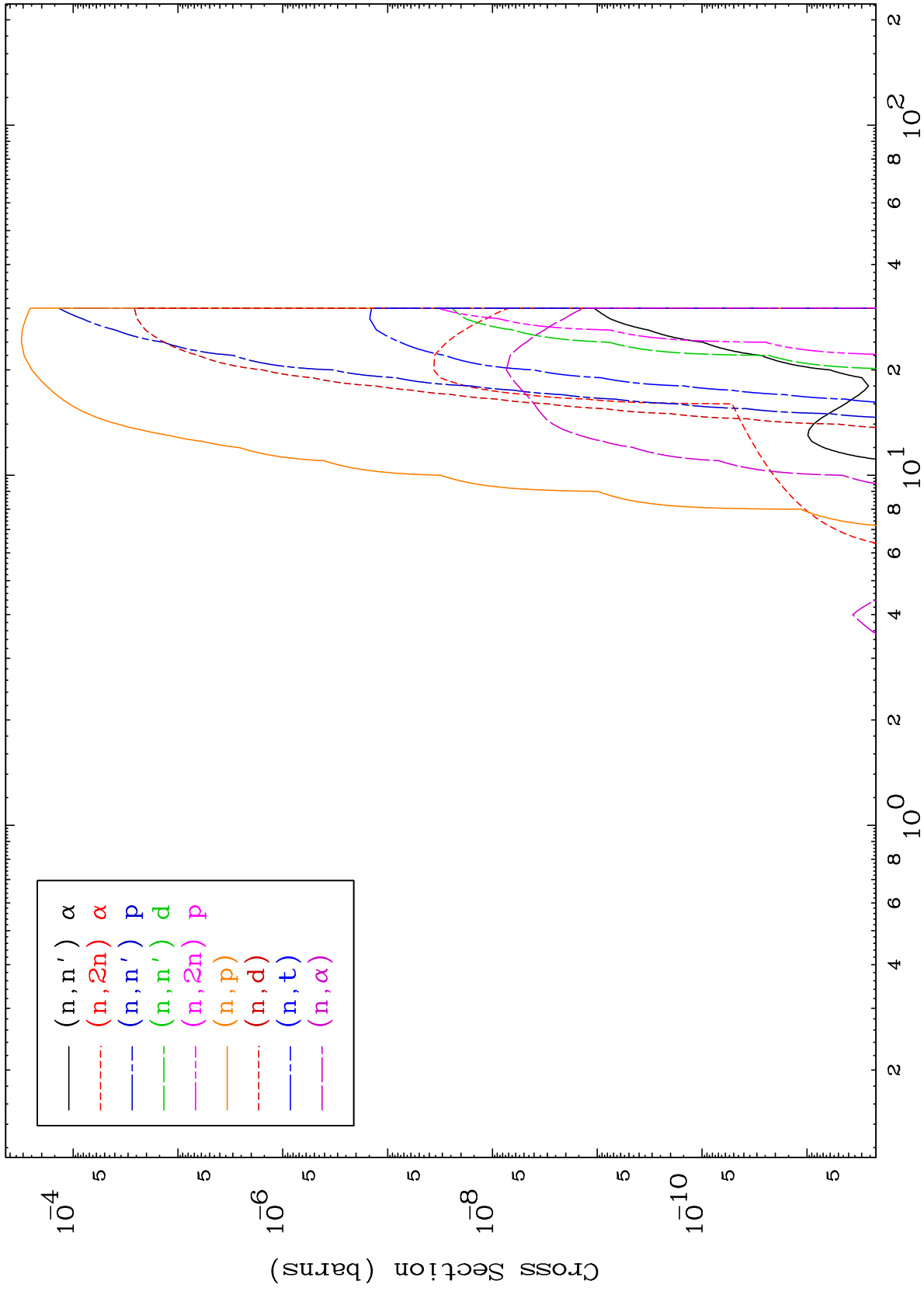
101-Md-253

MAT 9954

Photon Neutron Absorption
0 Kelvin Cross Sections

101-Md-253

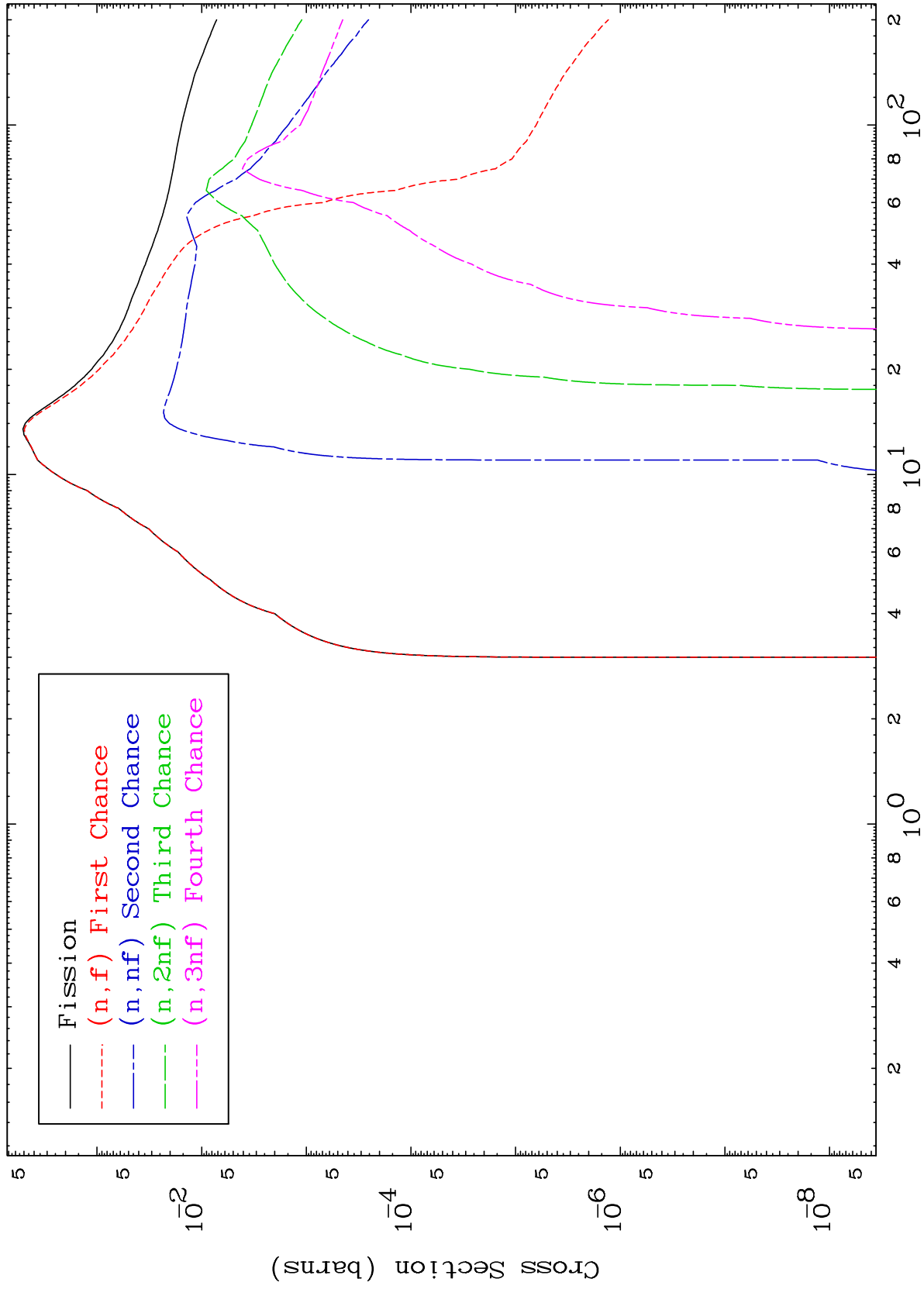




MAT 9954

Photon Fission
0 Kelvin Cross Sections

101-Md-253



5

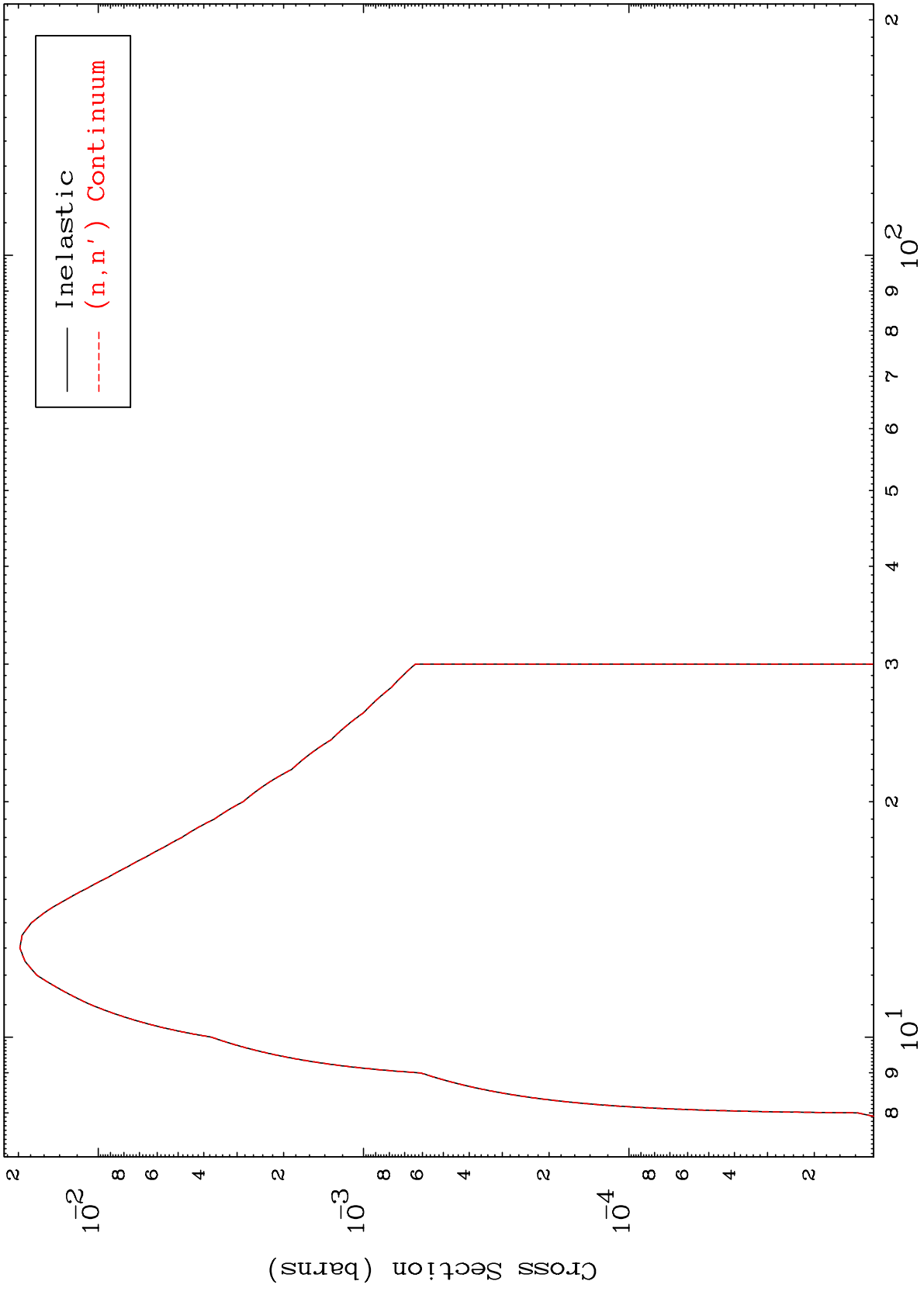
Incident Energy (MeV)

101-Md-253

MAT 9954

(γ, n') Levels
0 Kelvin Cross Sections

101-Md-253



6

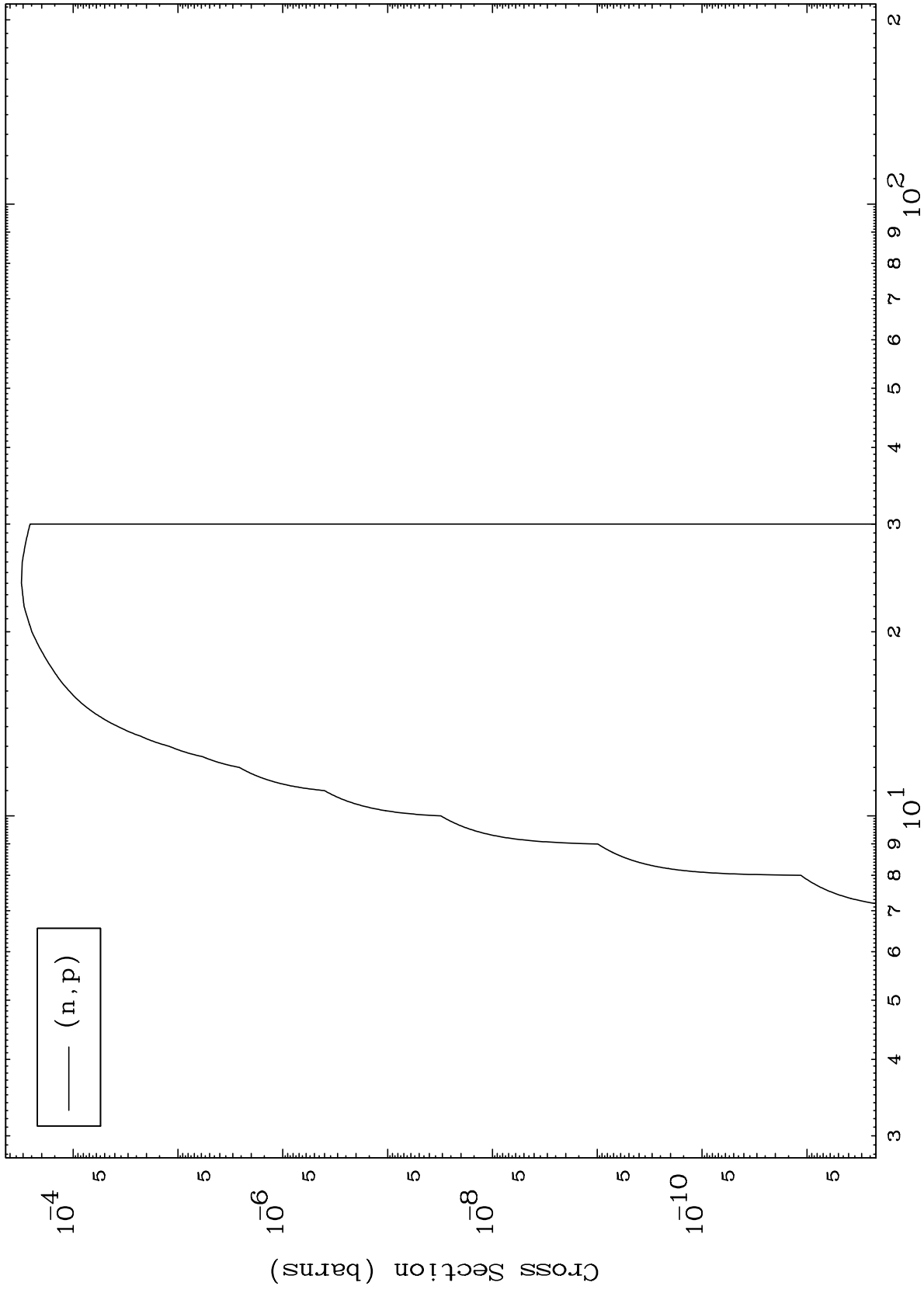
Incident Energy (MeV)

101-Md-253

MAT 9954

(γ, p) Levels
0 Kelvin Cross Sections

101-Md-253



7

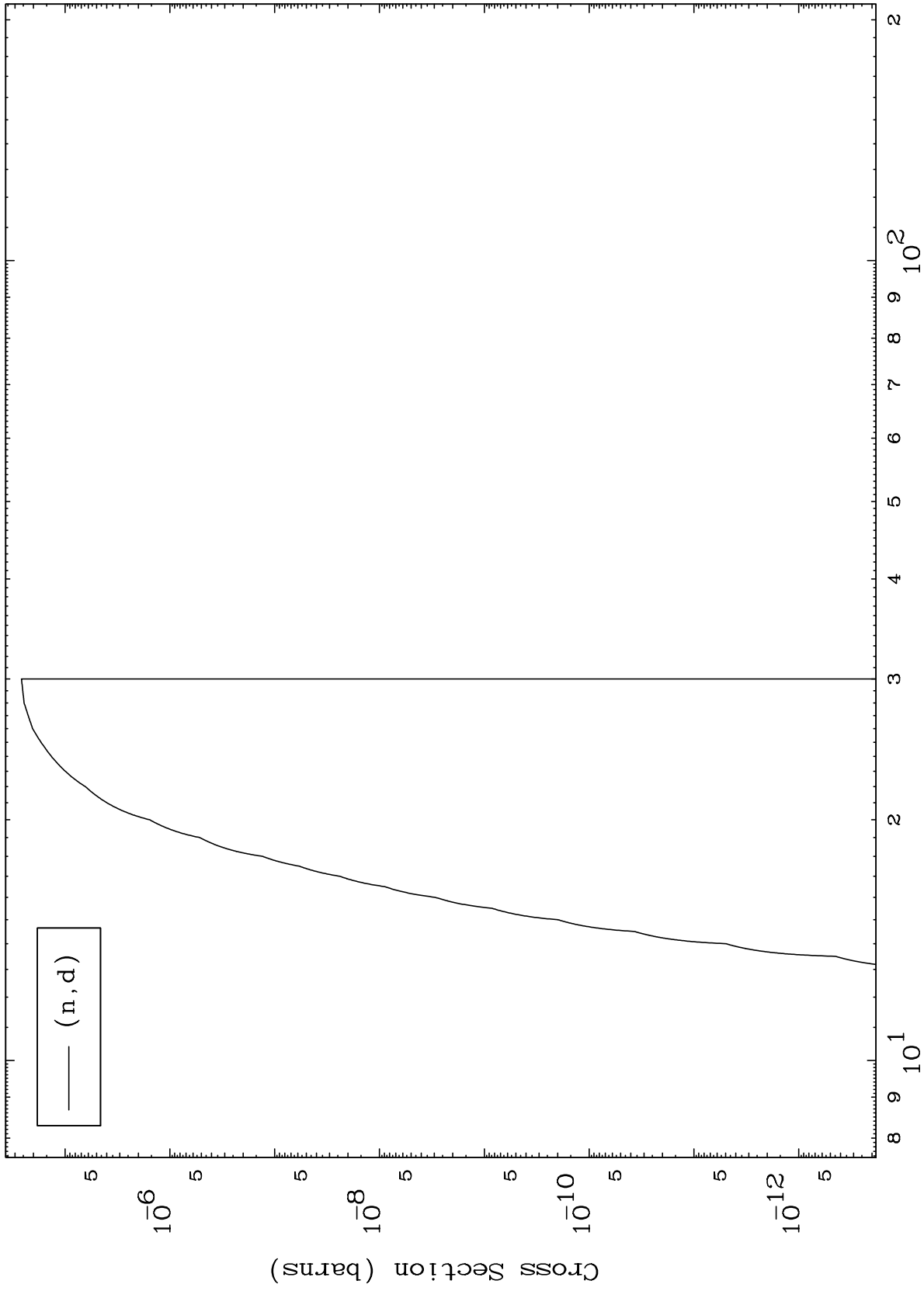
Incident Energy (MeV)

101-Md-253

MAT 9954

(γ, d) Levels
0 Kelvin Cross Sections

101-Md-253



8

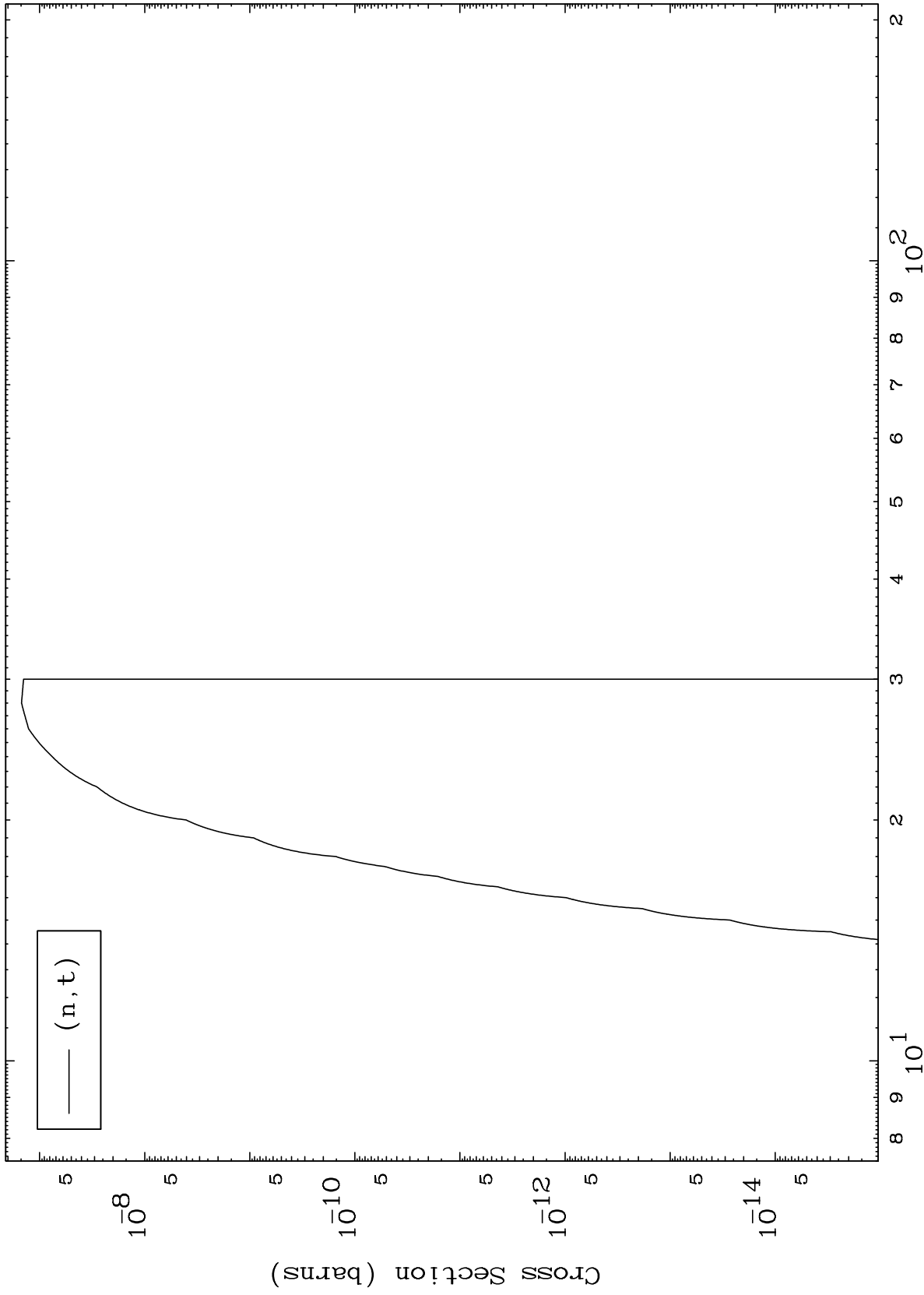
Incident Energy (MeV)

101-Md-253

MAT 9954

(γ, t) Levels
0 Kelvin Cross Sections

101-Md-253



9

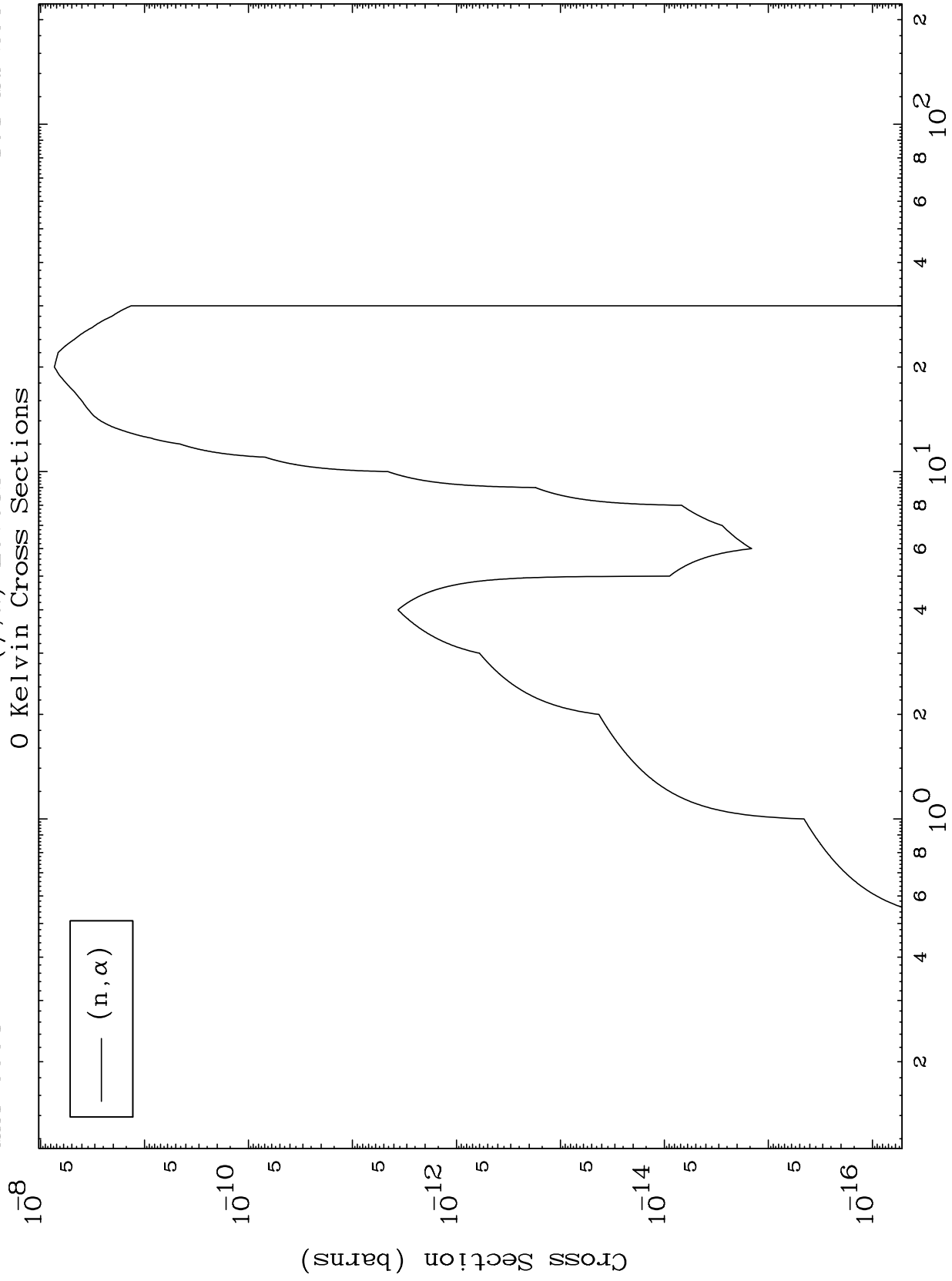
Incident Energy (MeV)

101-Md-253

MAT 9954

(γ, α) Levels
0 Kelvin Cross Sections

101-Md-253



10

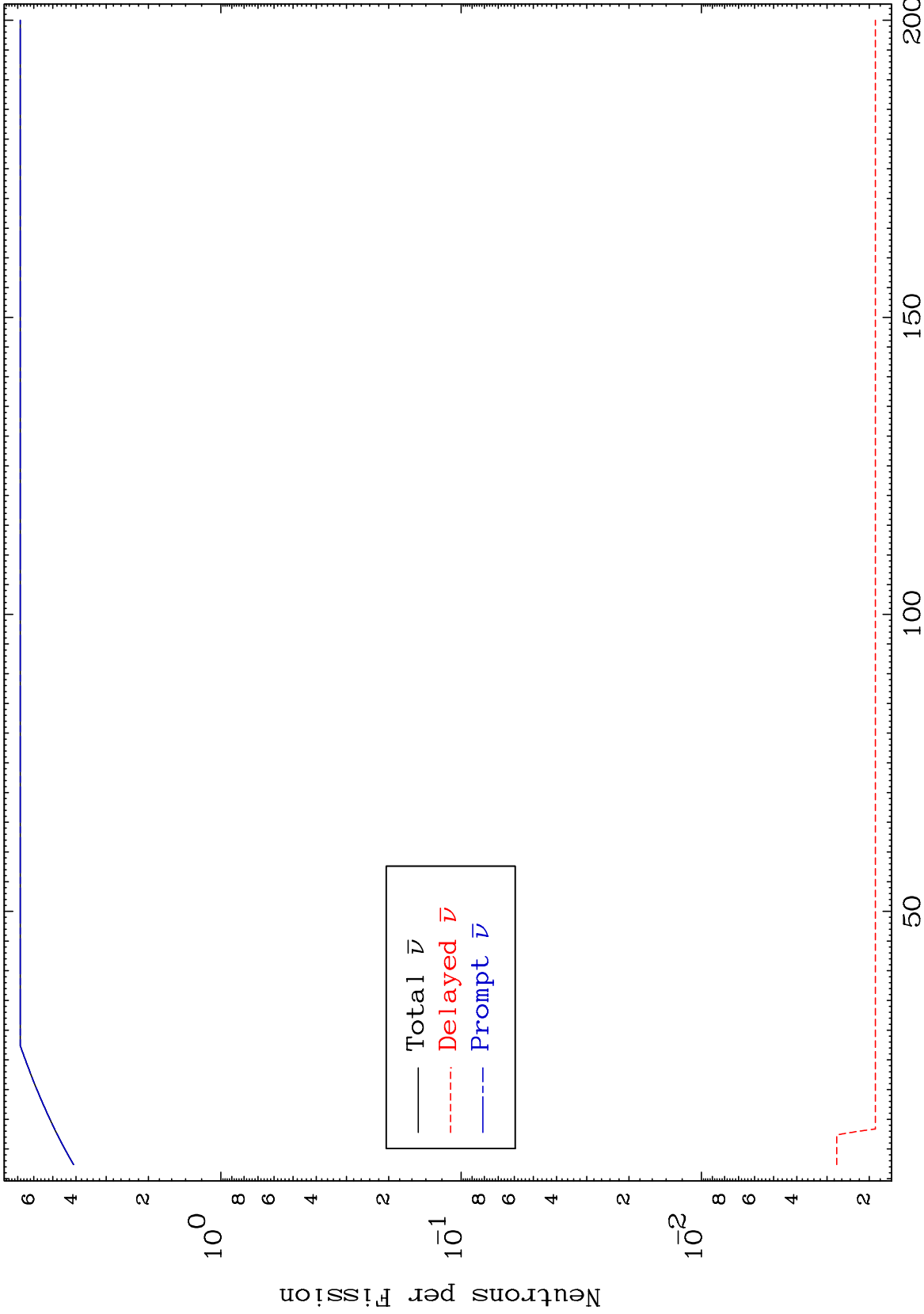
Incident Energy (MeV)

101-Md-253

MAT 9954

Photon Neutrons
per Fission

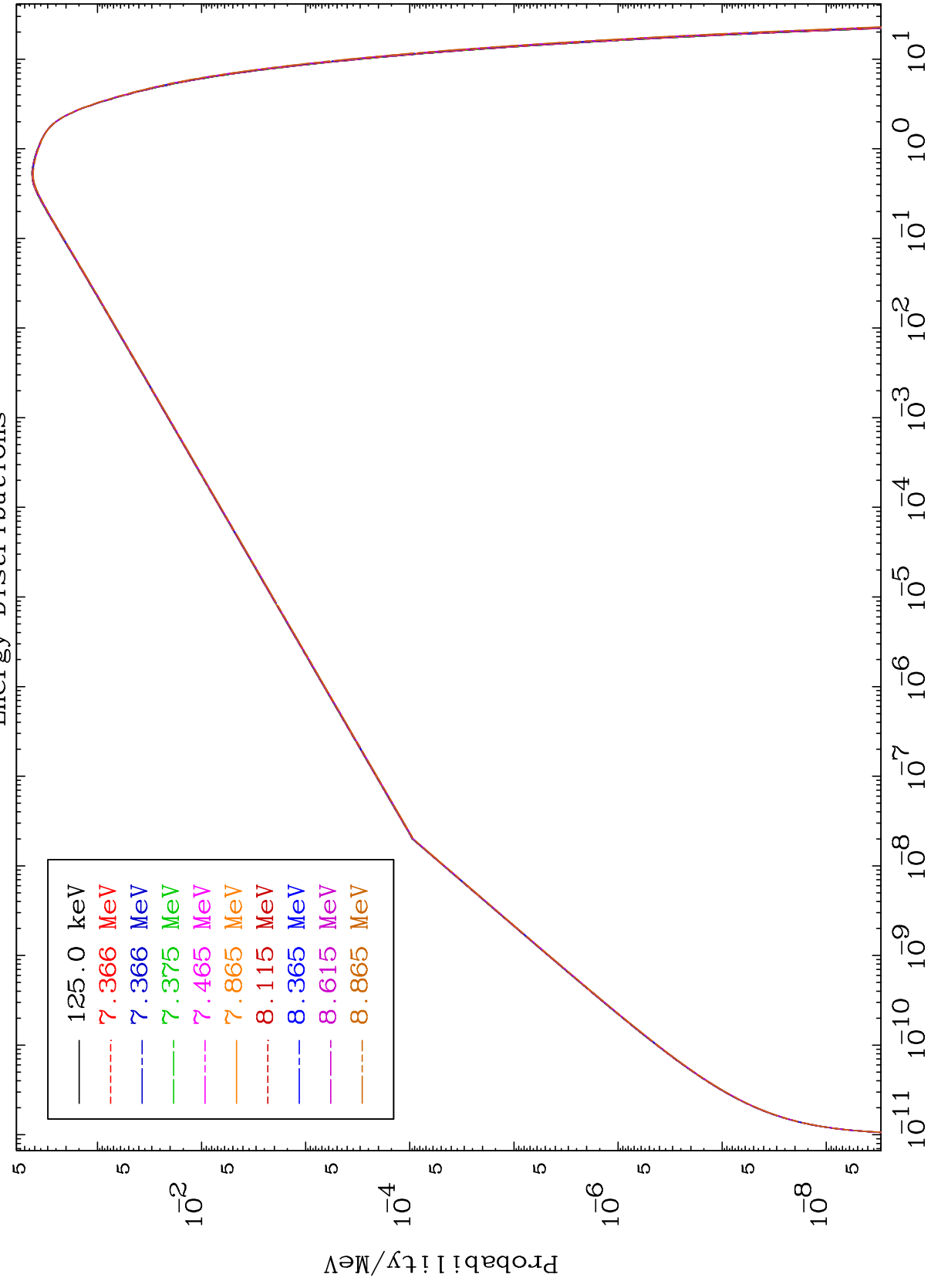
101-Md-253



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Fission Energy Distributions

101-Md-253



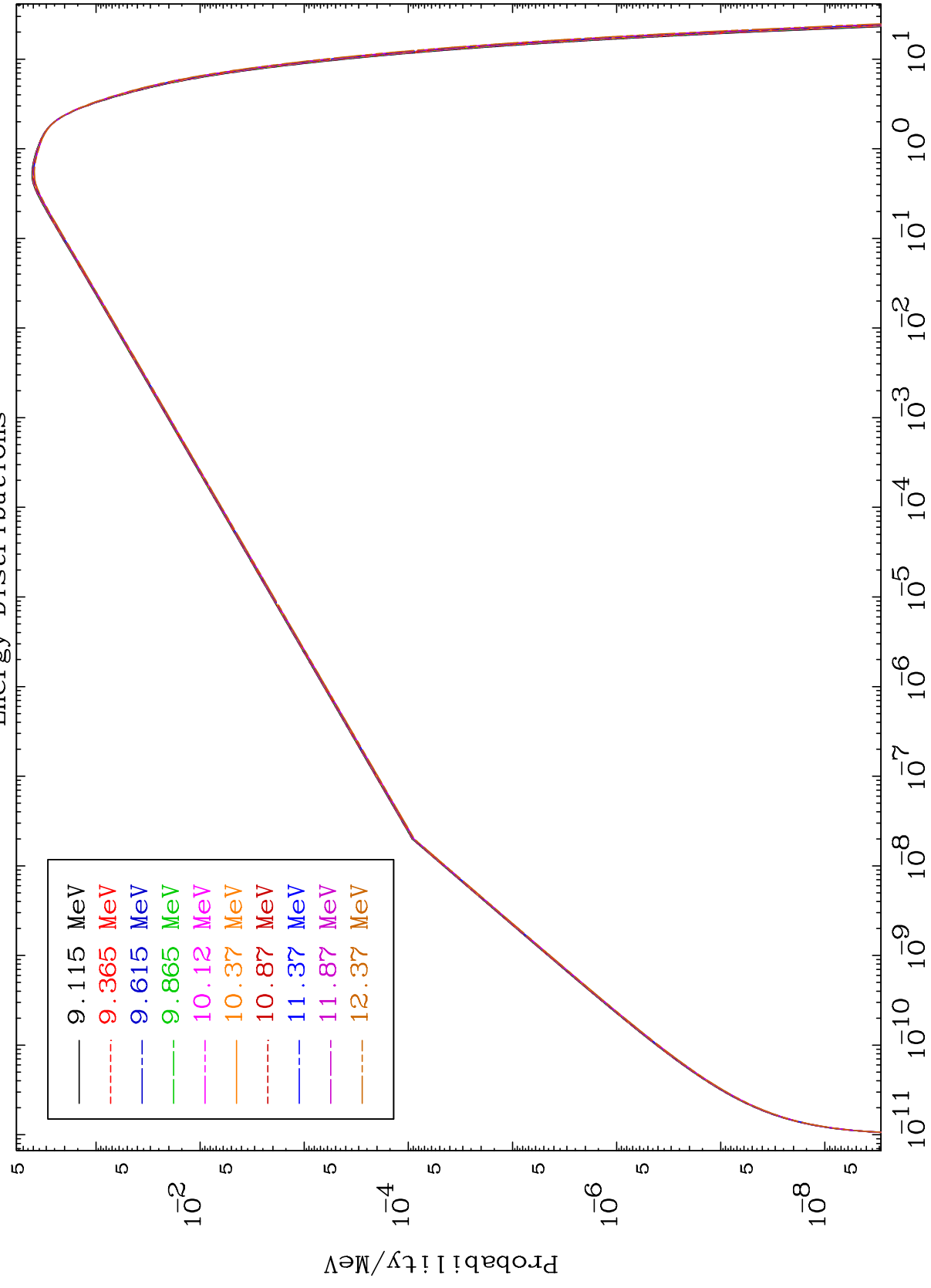
101-Md-253

Secondary Energy (MeV)

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Fission Energy Distributions

101-Md-253

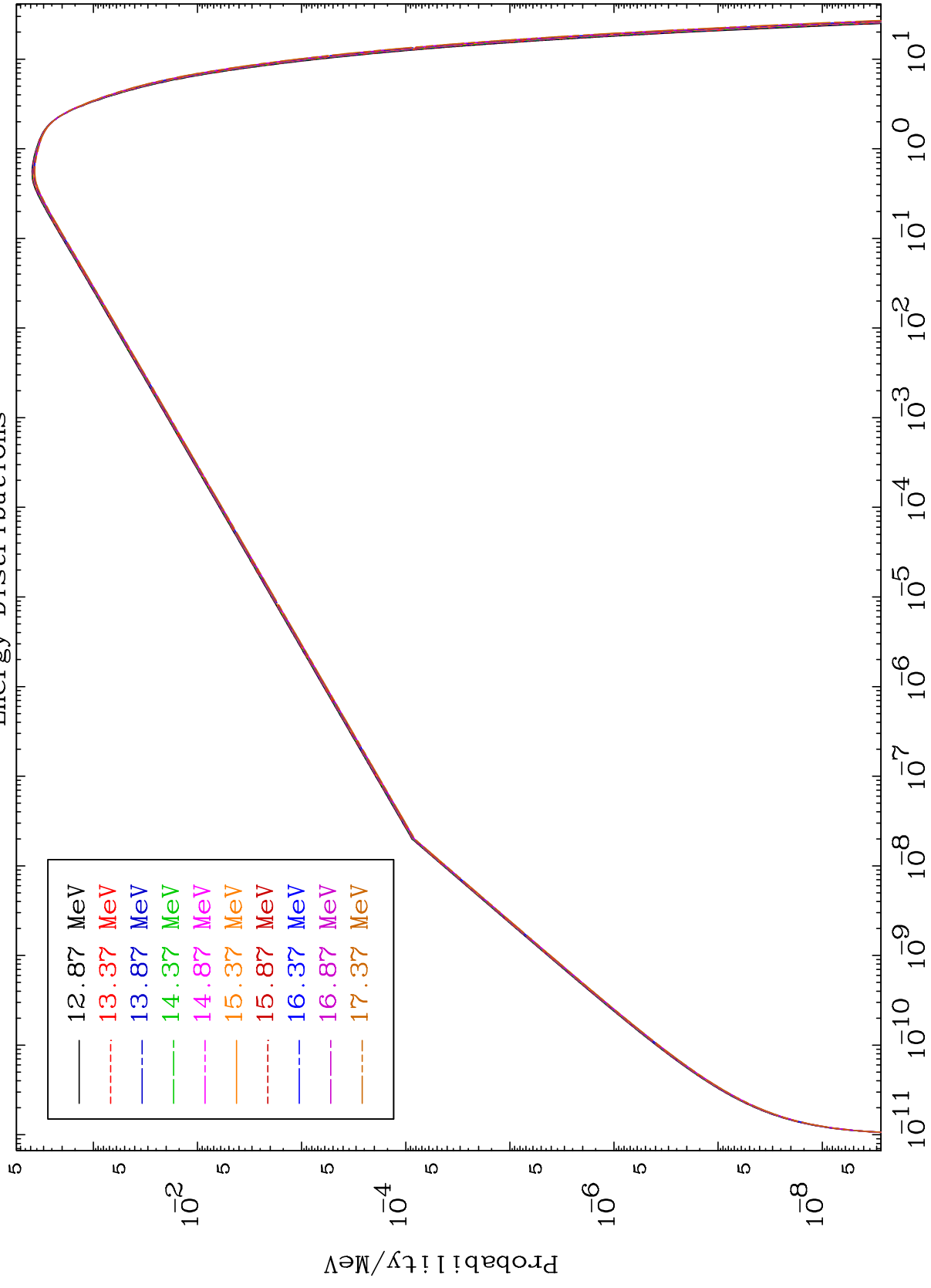


101-Md-253

MAT 9954

Fission Energy Distributions

101-Md-253

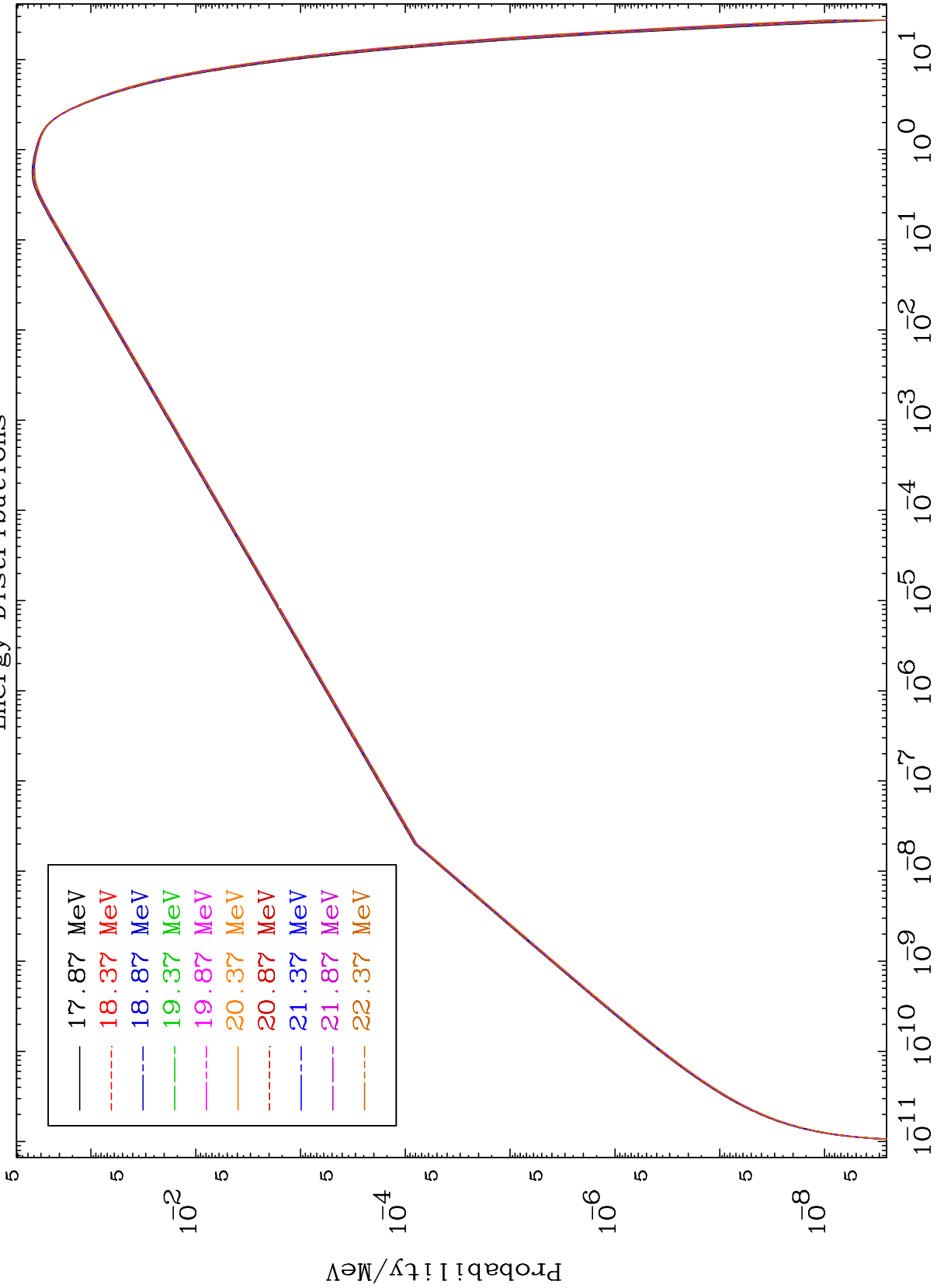


101-Md-253

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Fission Energy Distributions

101-Md-253



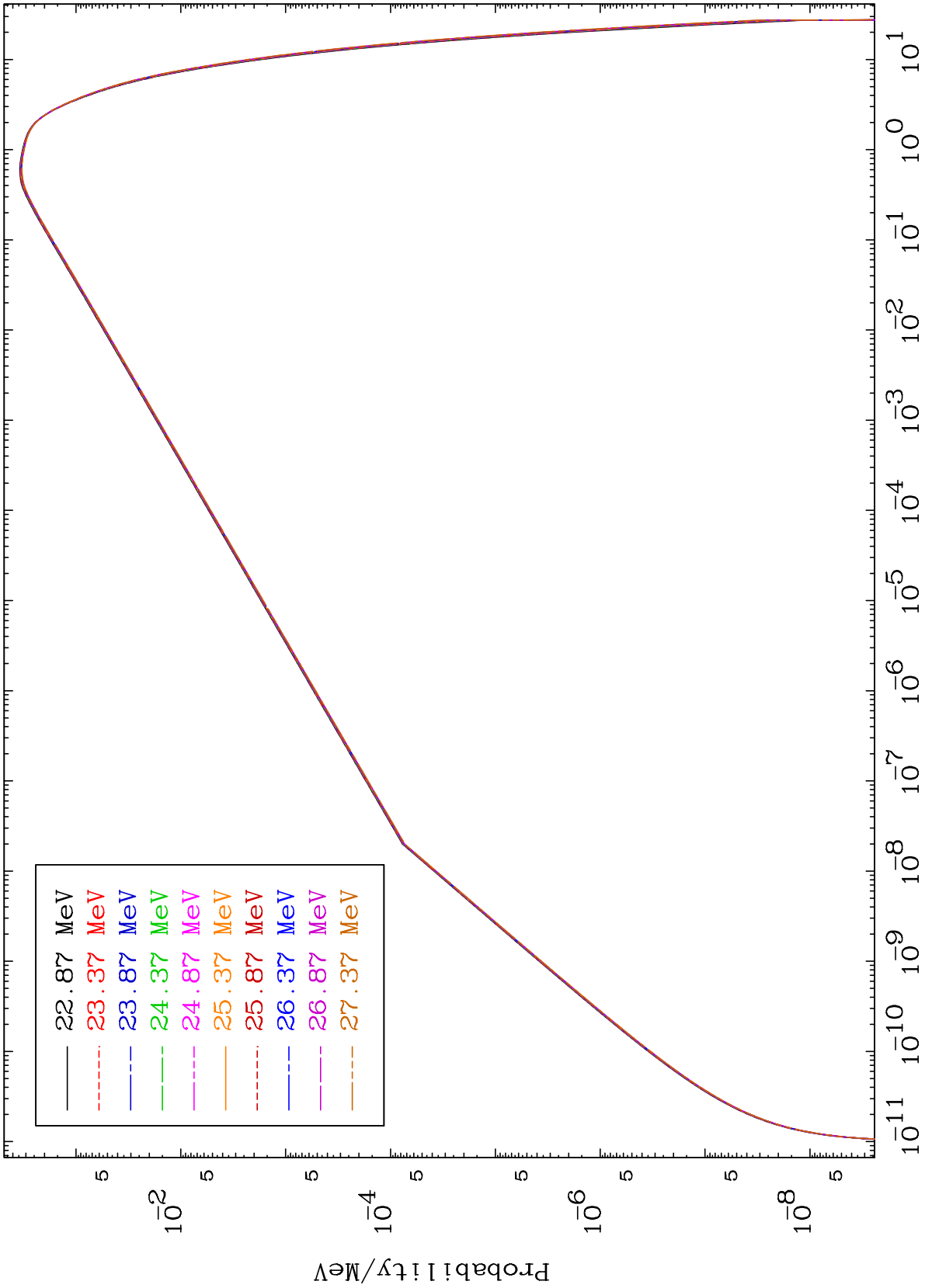
101-Md-253

Secondary Energy (MeV)

MAT 9954

Fission Energy Distributions

101-Md-253

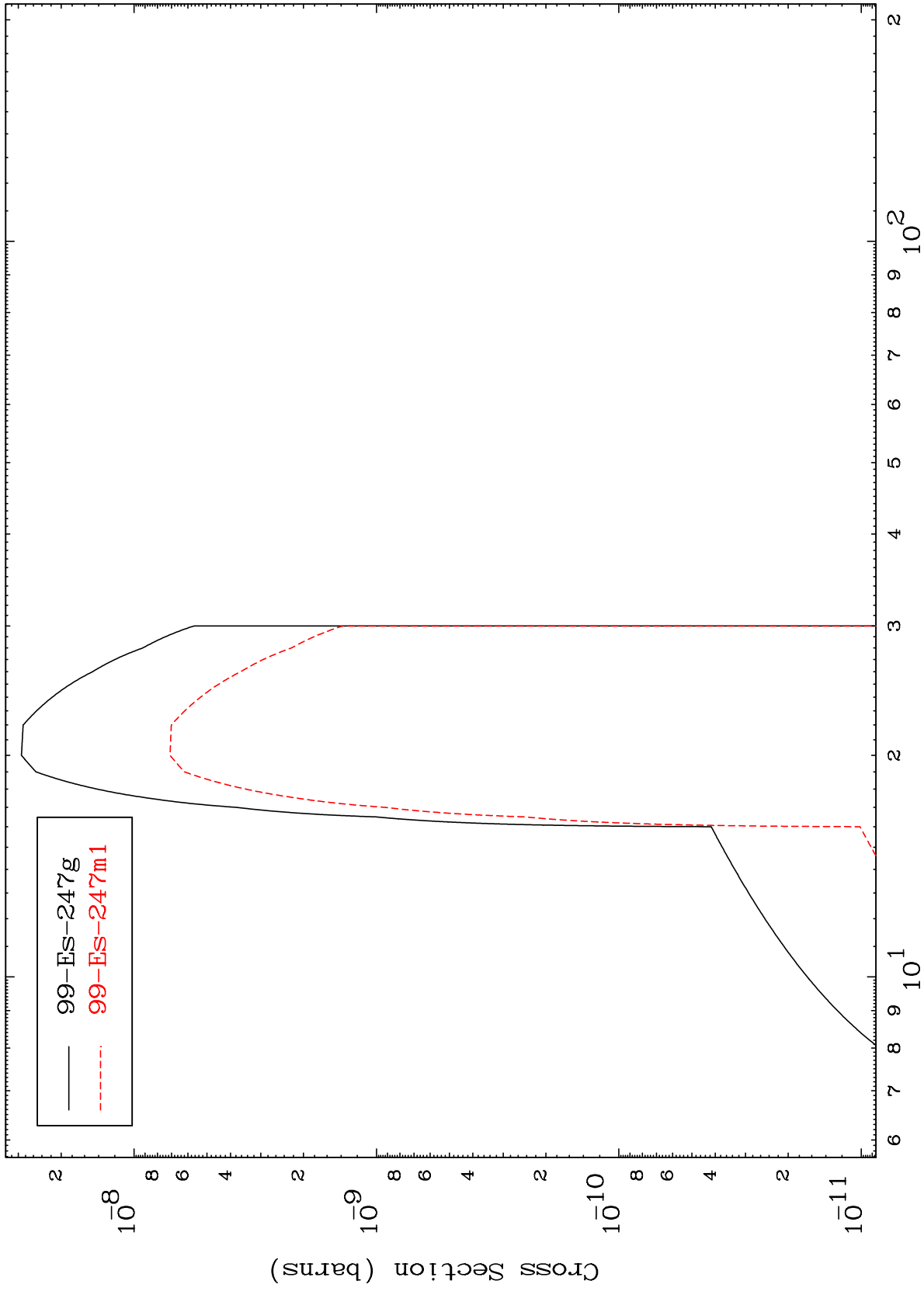


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(n,2n) α

101-Md-253

Radionuclide Production Cross Section



17

Incident Energy (MeV)

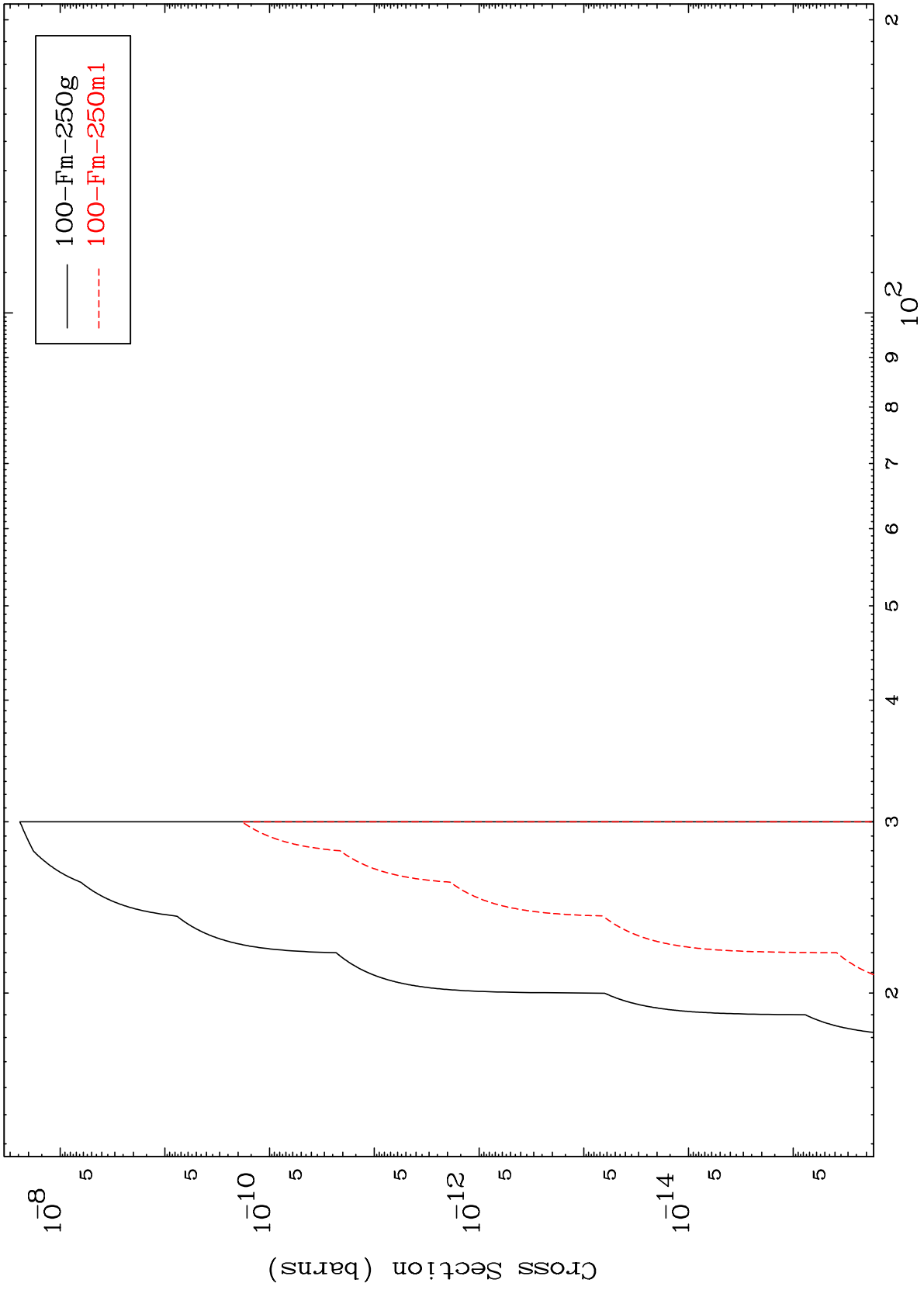
101-Md-253

MAT 9954

(n,n') d

101-Md-253

Radionuclide Production Cross Section



18

Incident Energy (MeV)

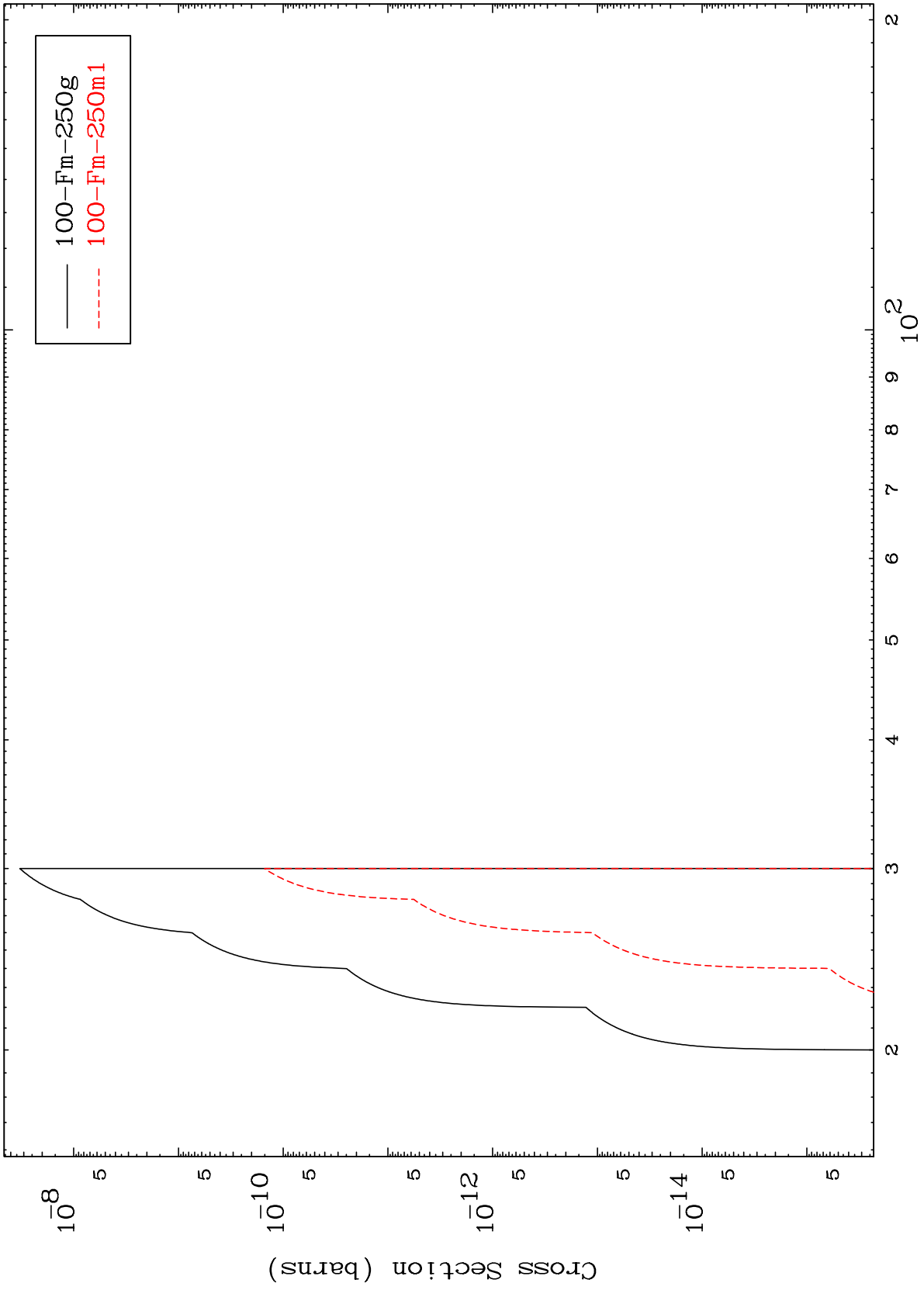
101-Md-253

MAT 9954

(n,2n) p

101-Md-253

Radionuclide Production Cross Section



19

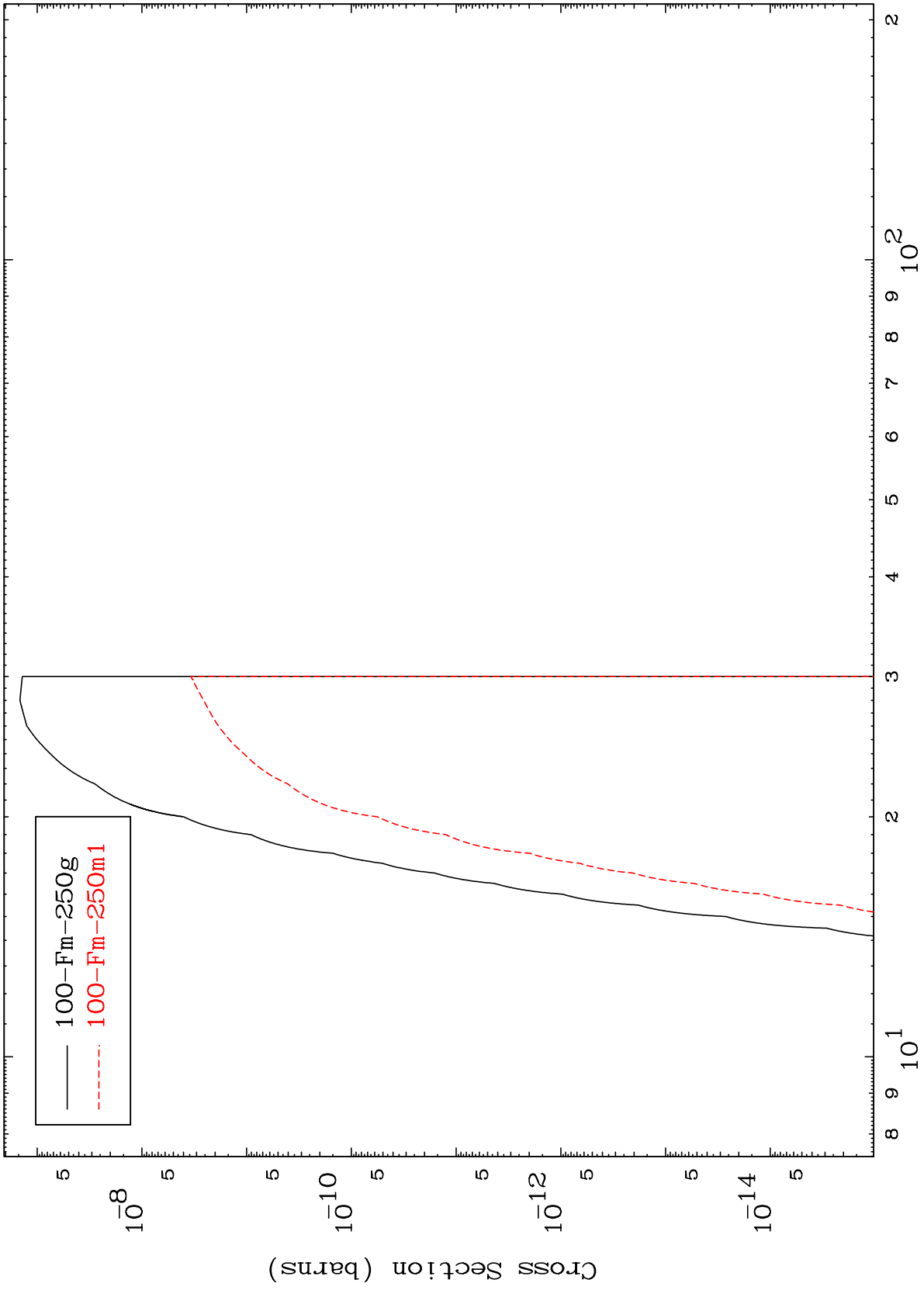
Incident Energy (MeV)

101-Md-253

MAT 9954

101-Md-253

(n, t)
Radionuclide Production Cross Section



20

Incident Energy (MeV)

101-Md-253