

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

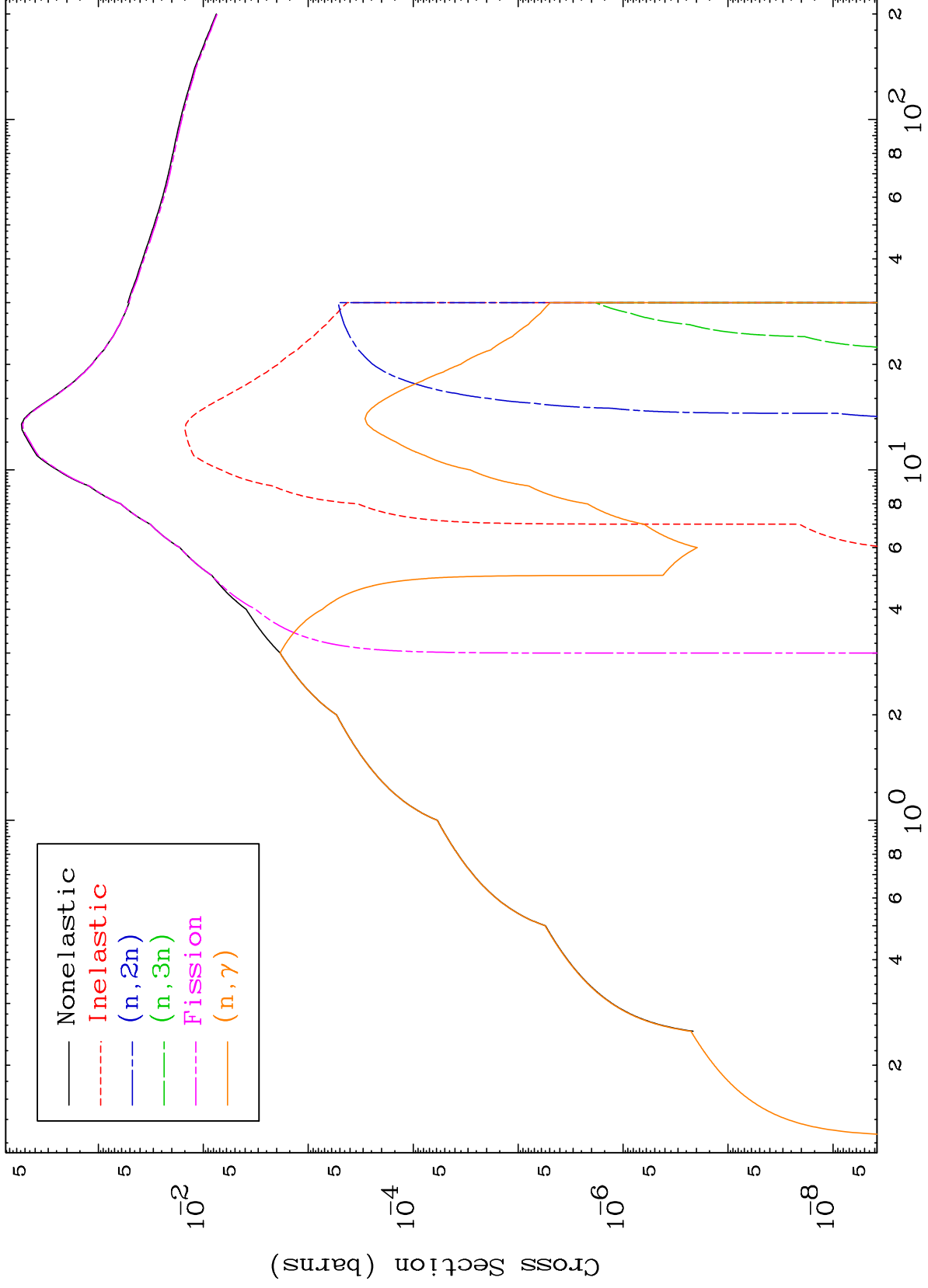
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9973

Photon Major  
0 Kelvin Cross Sections

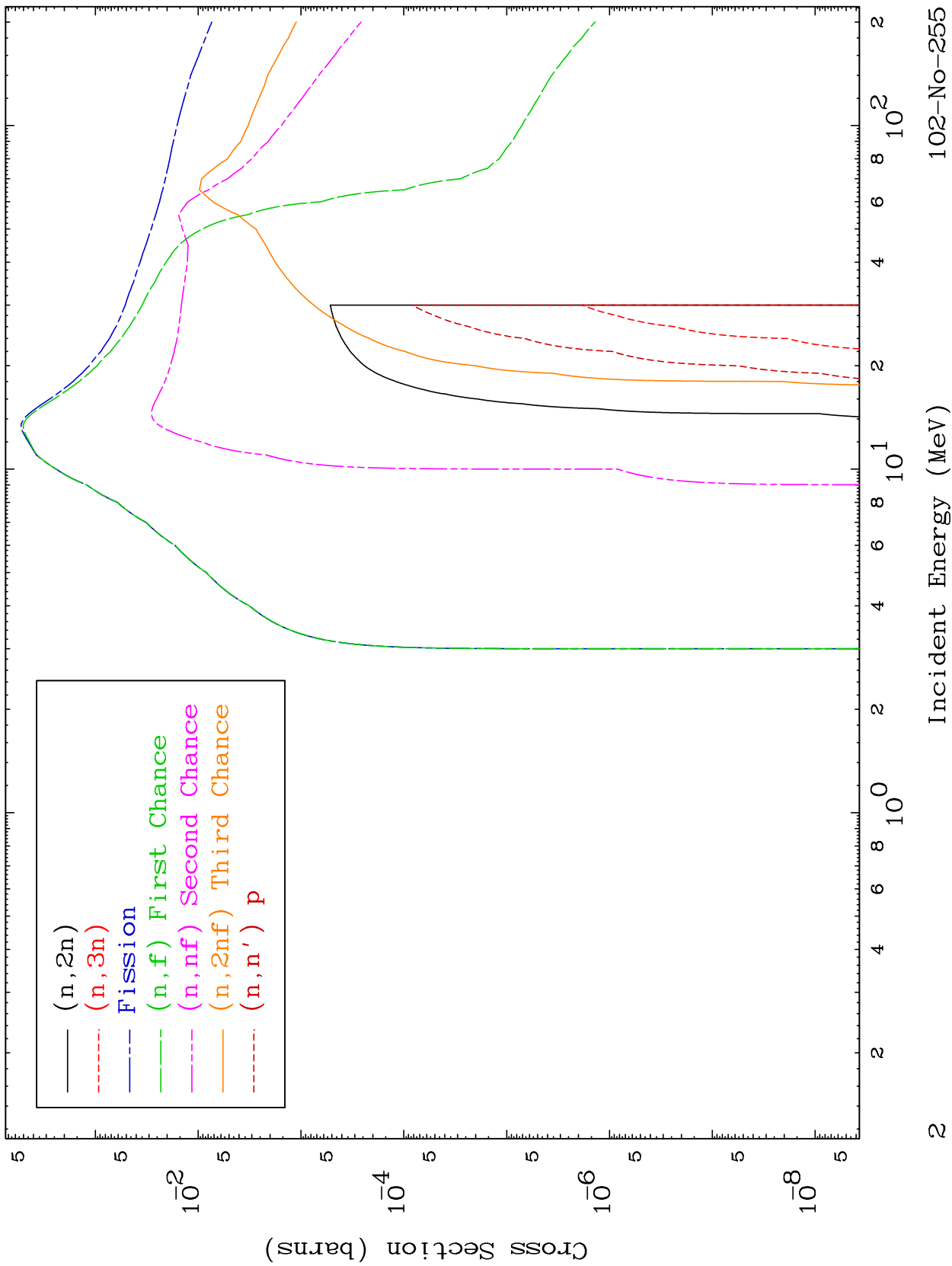
102-No-255



MAT 9973

Photon Neutron Absorption  
0 Kelvin Cross Sections

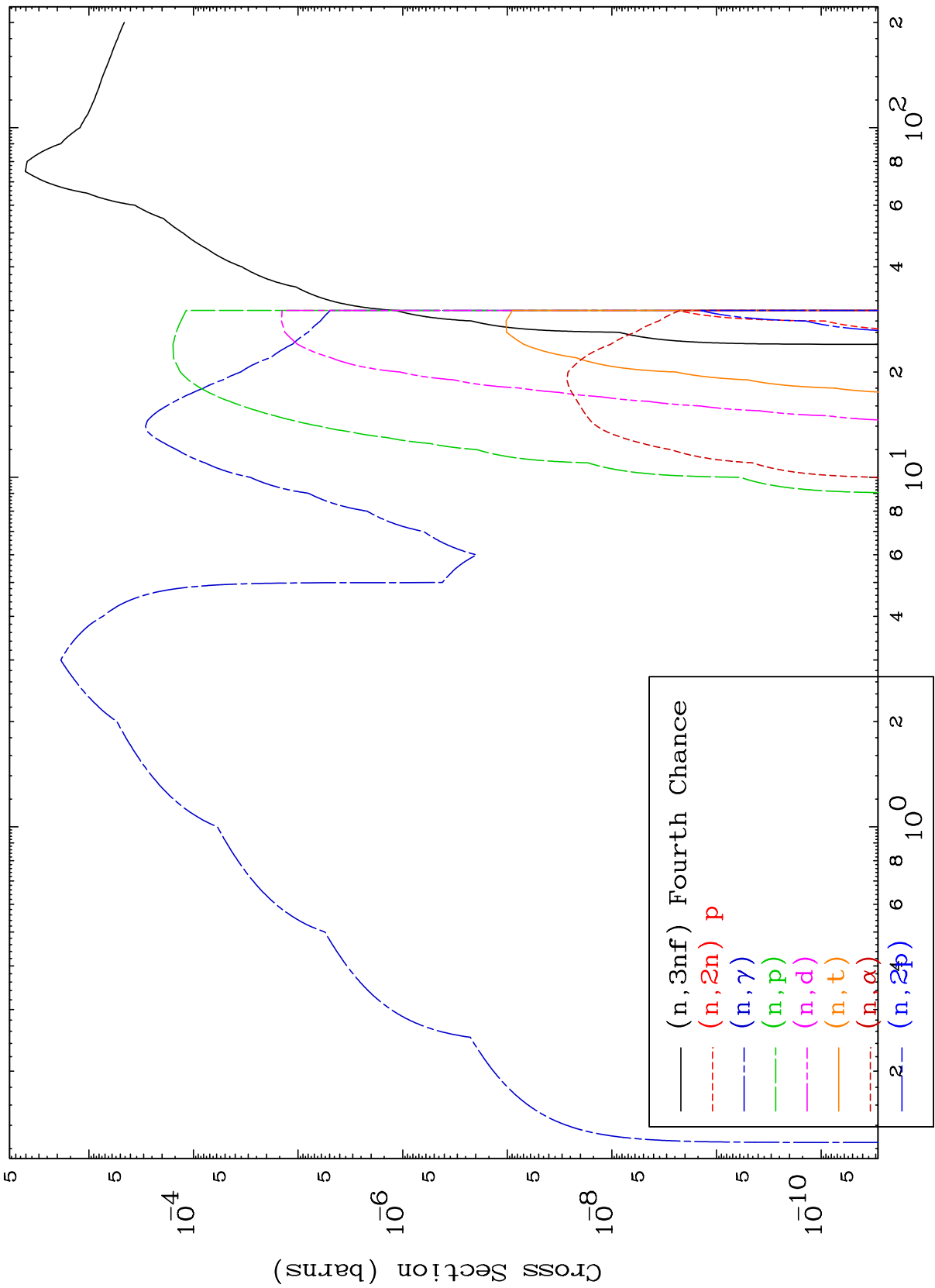
102-No-255



MAT 9973

Photon Neutron Absorption  
0 Kelvin Cross Sections

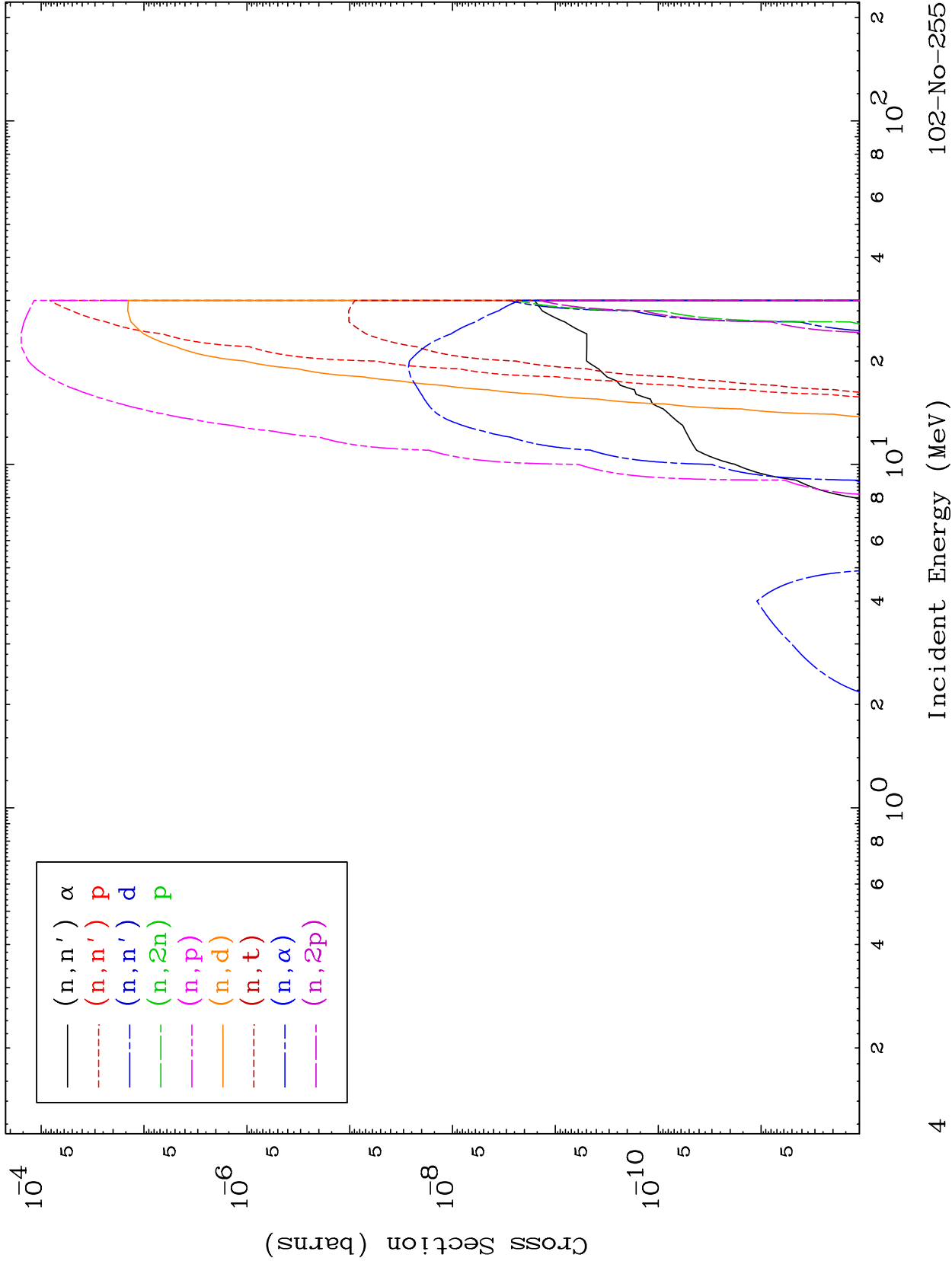
102-No-255

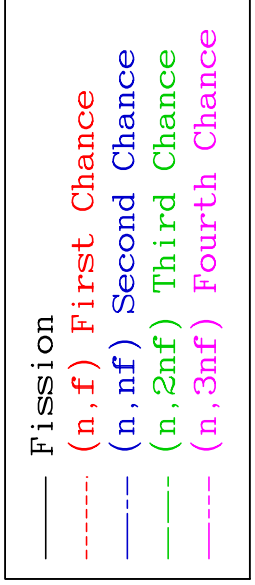
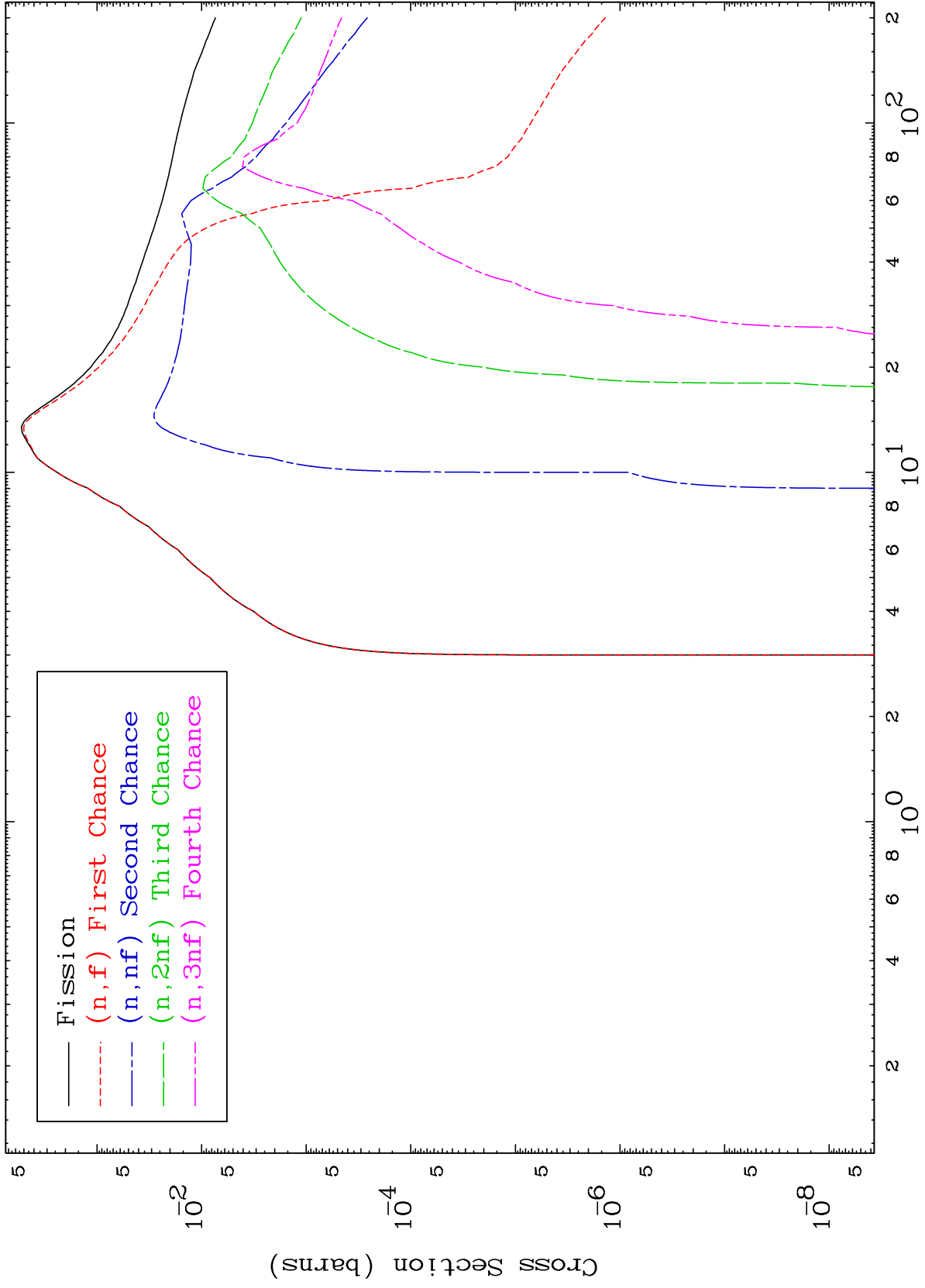


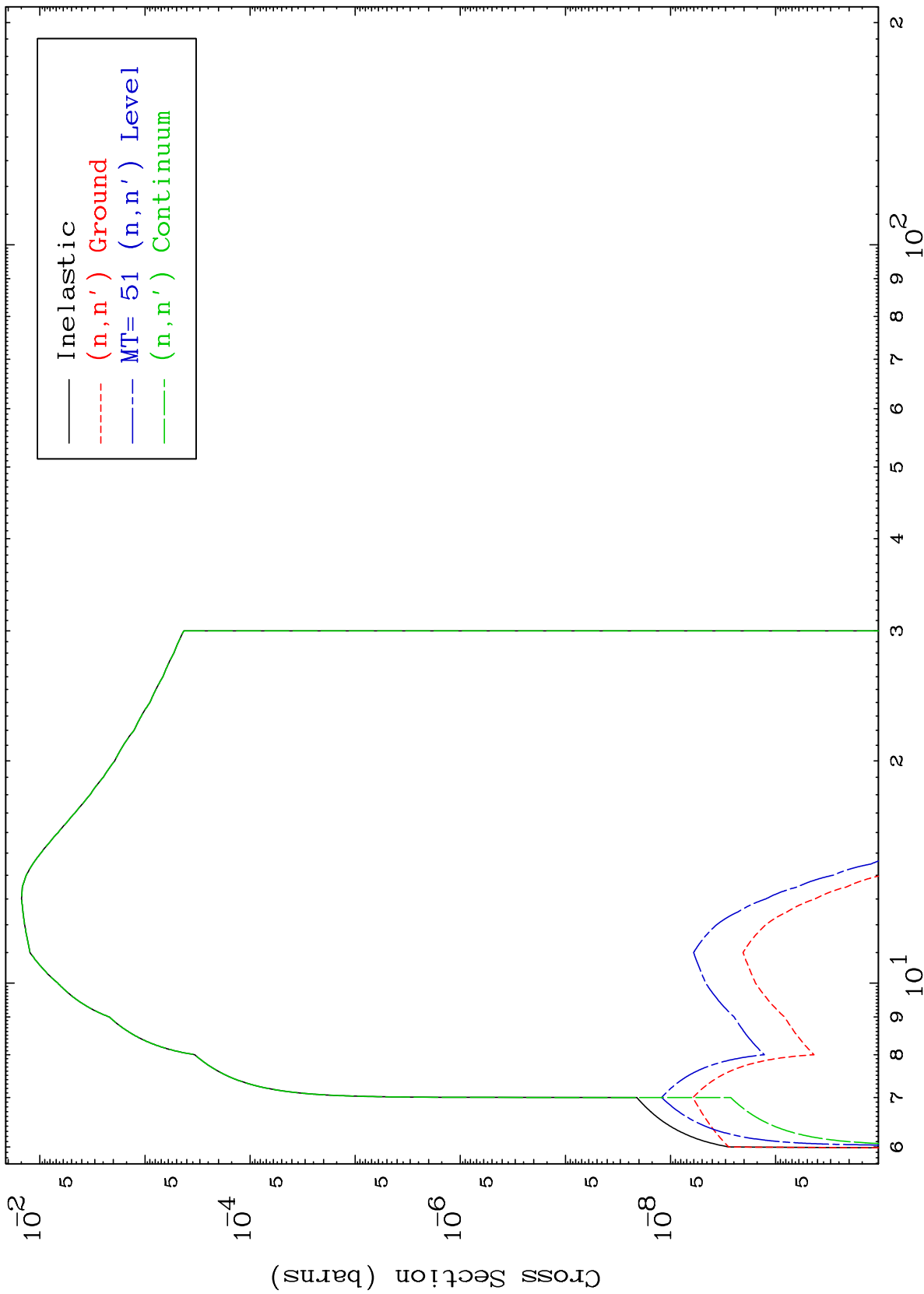
102-No-255

Incident Energy (MeV)

3



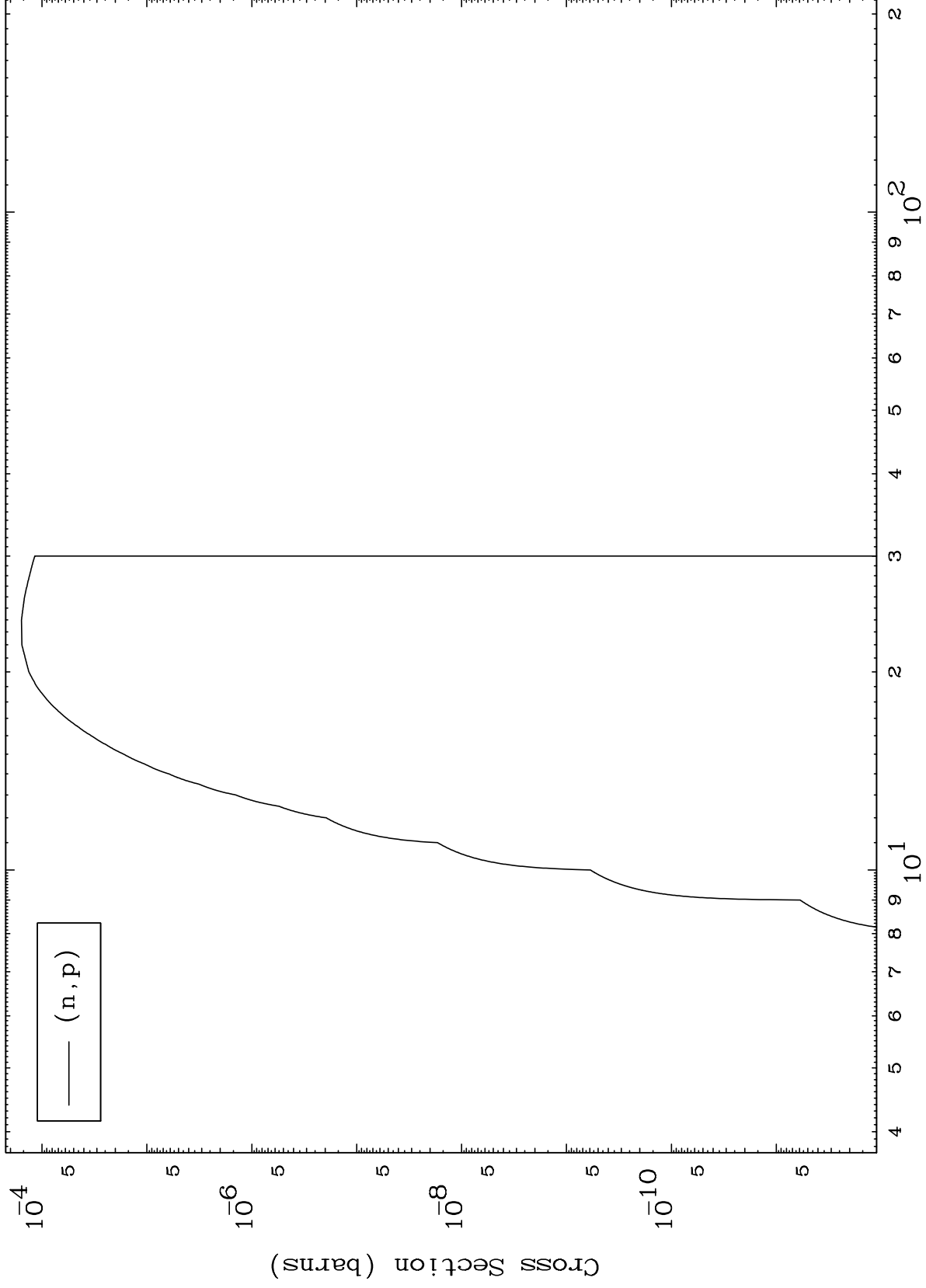




MAT 9973

( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

102-No-255

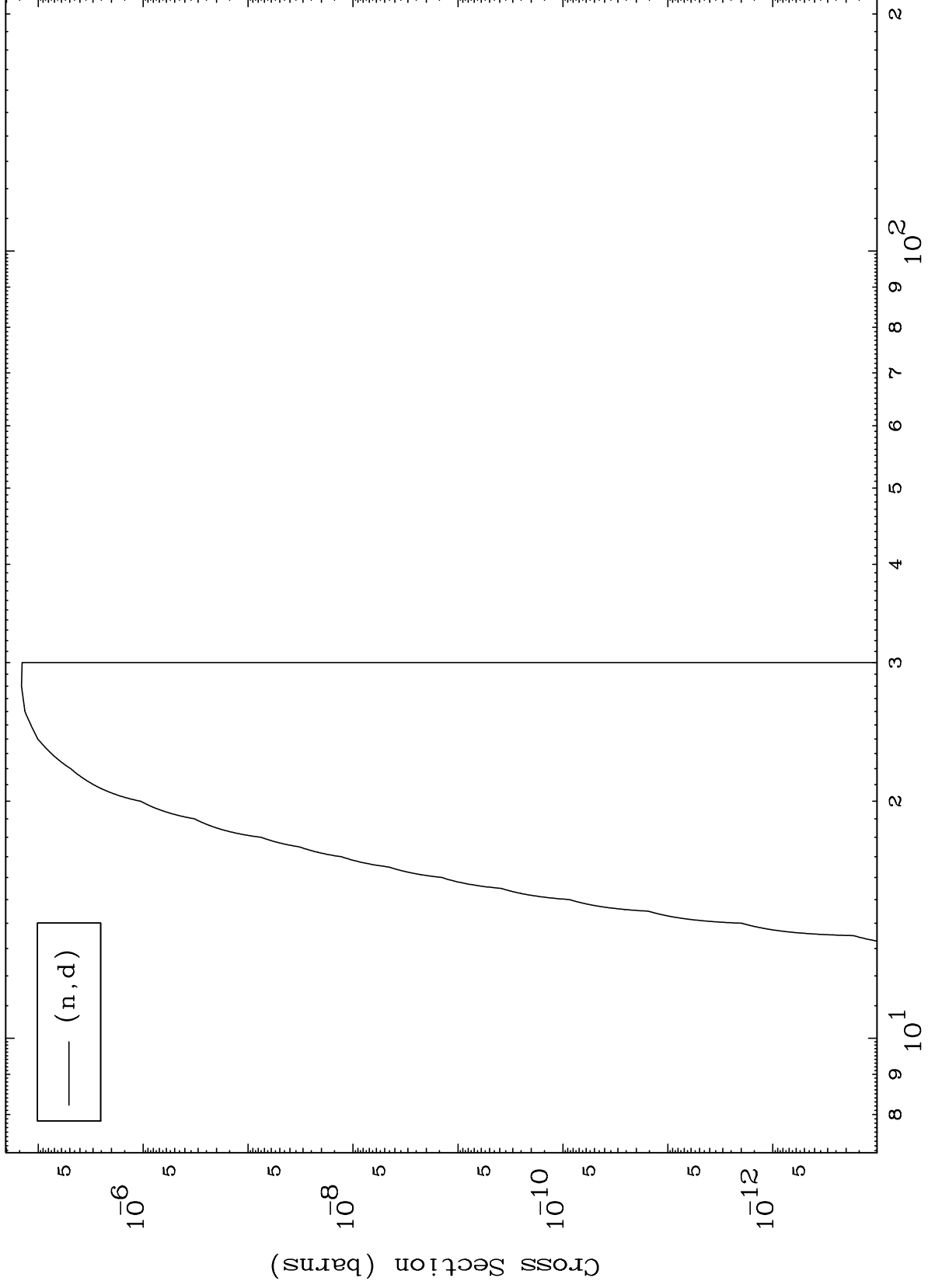


7

Incident Energy (MeV)

102-No-255

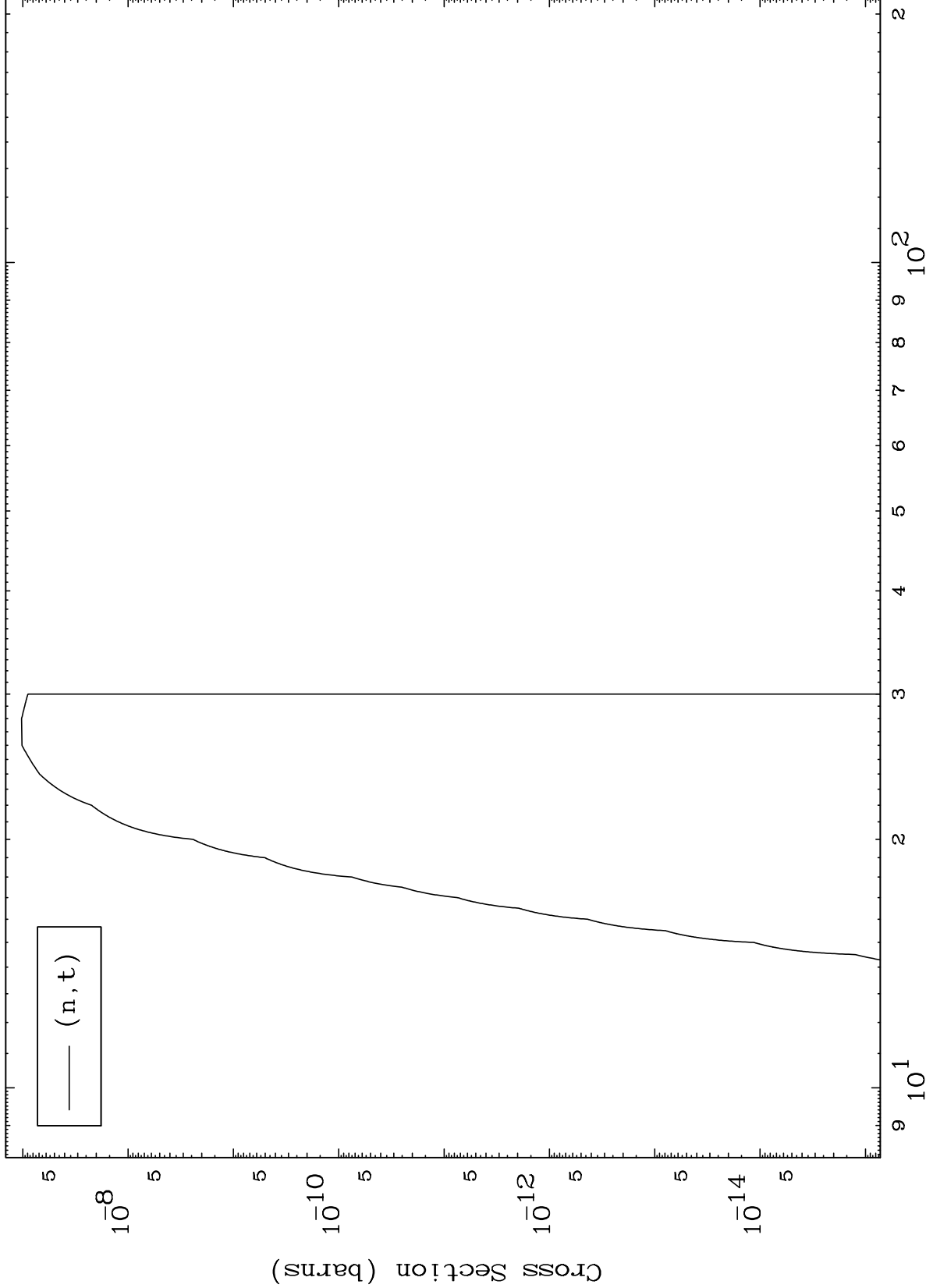




MAT 9973

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

102-No-255



9

Incident Energy (MeV)

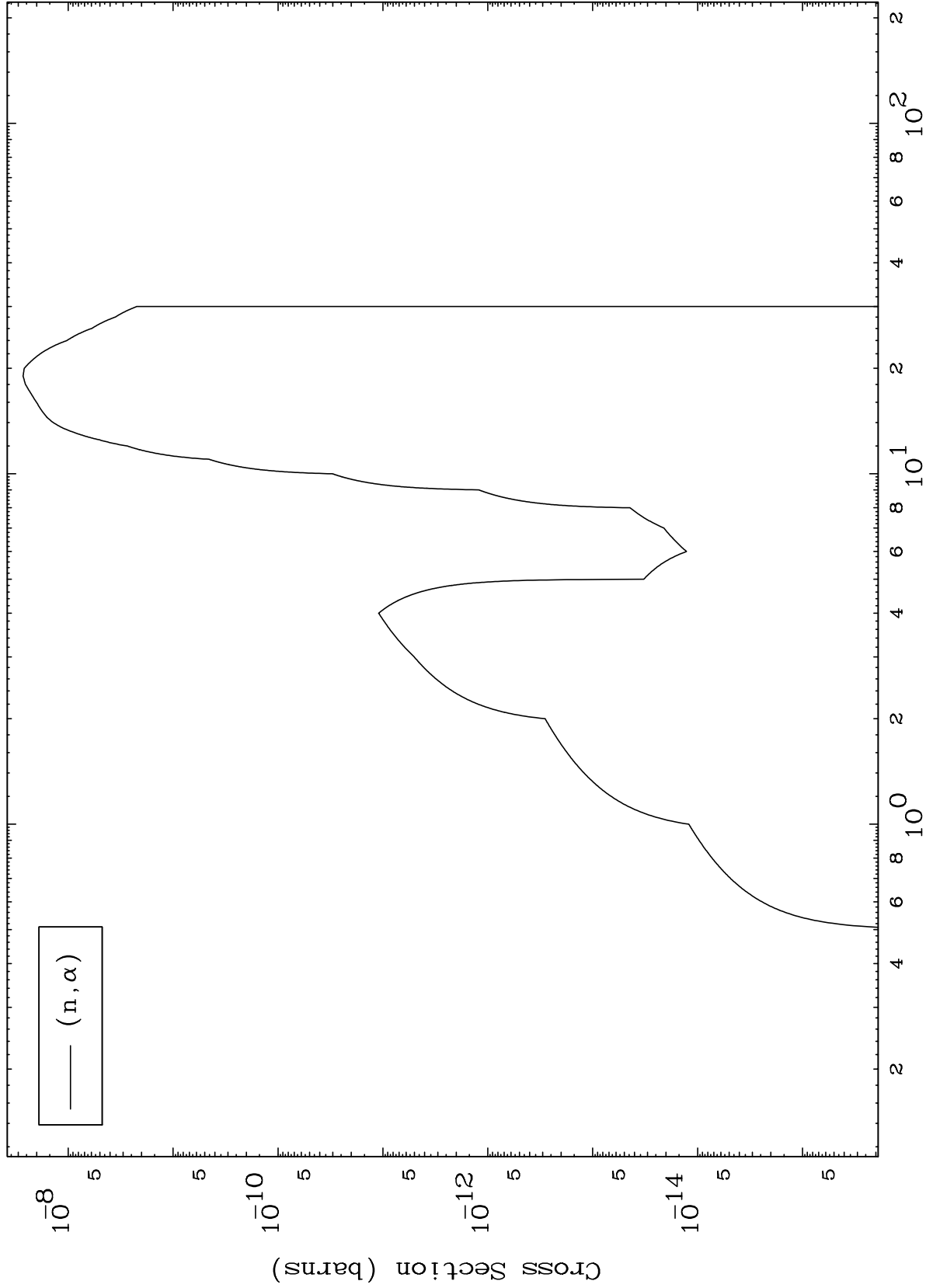
102-No-255

MAT 9973

( $\gamma, \alpha$ ) Levels

102-No-255

0 Kelvin Cross Sections

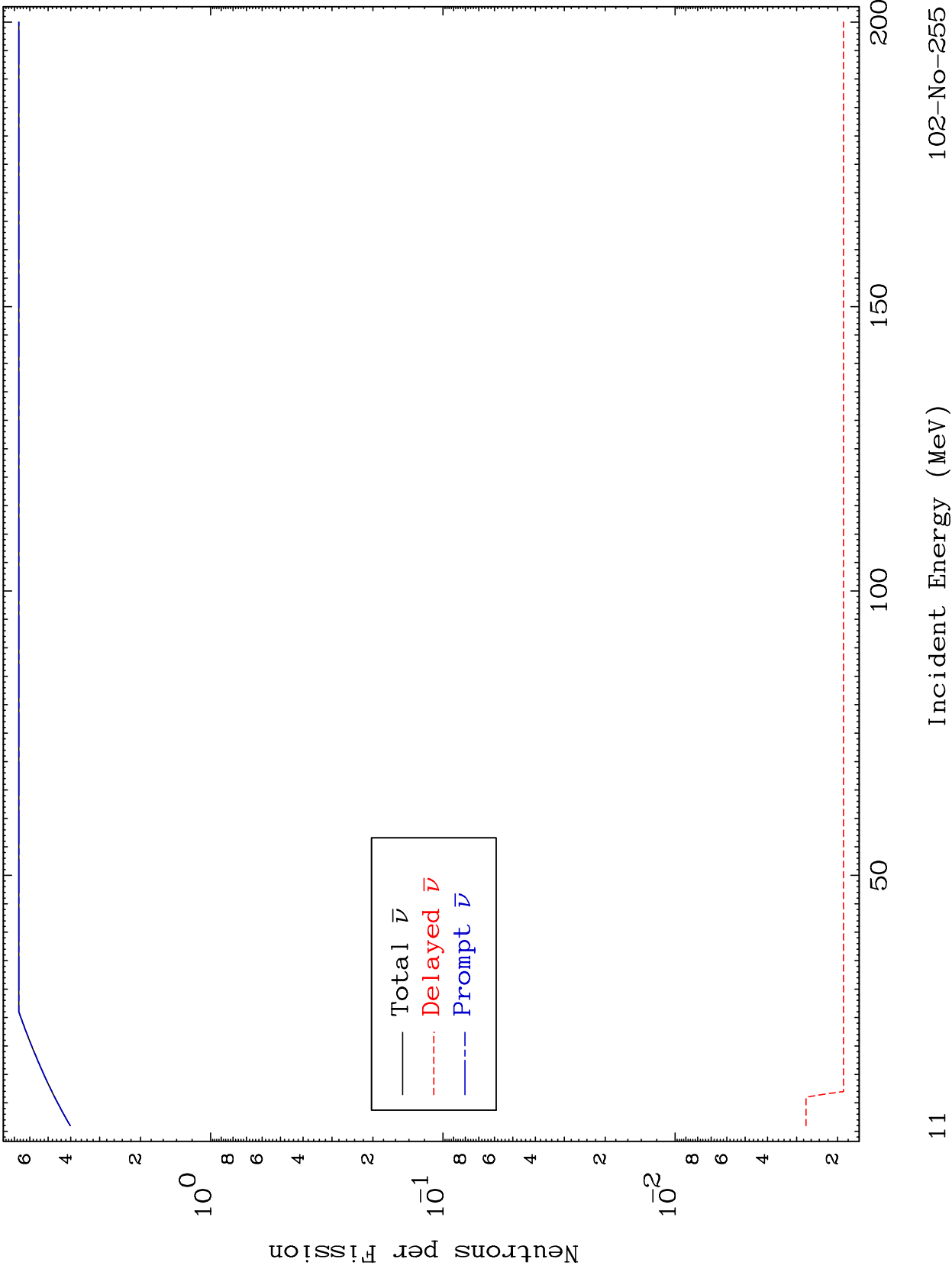


10

Incident Energy (MeV)

102-No-255

Photon Neutrons  
per Fission

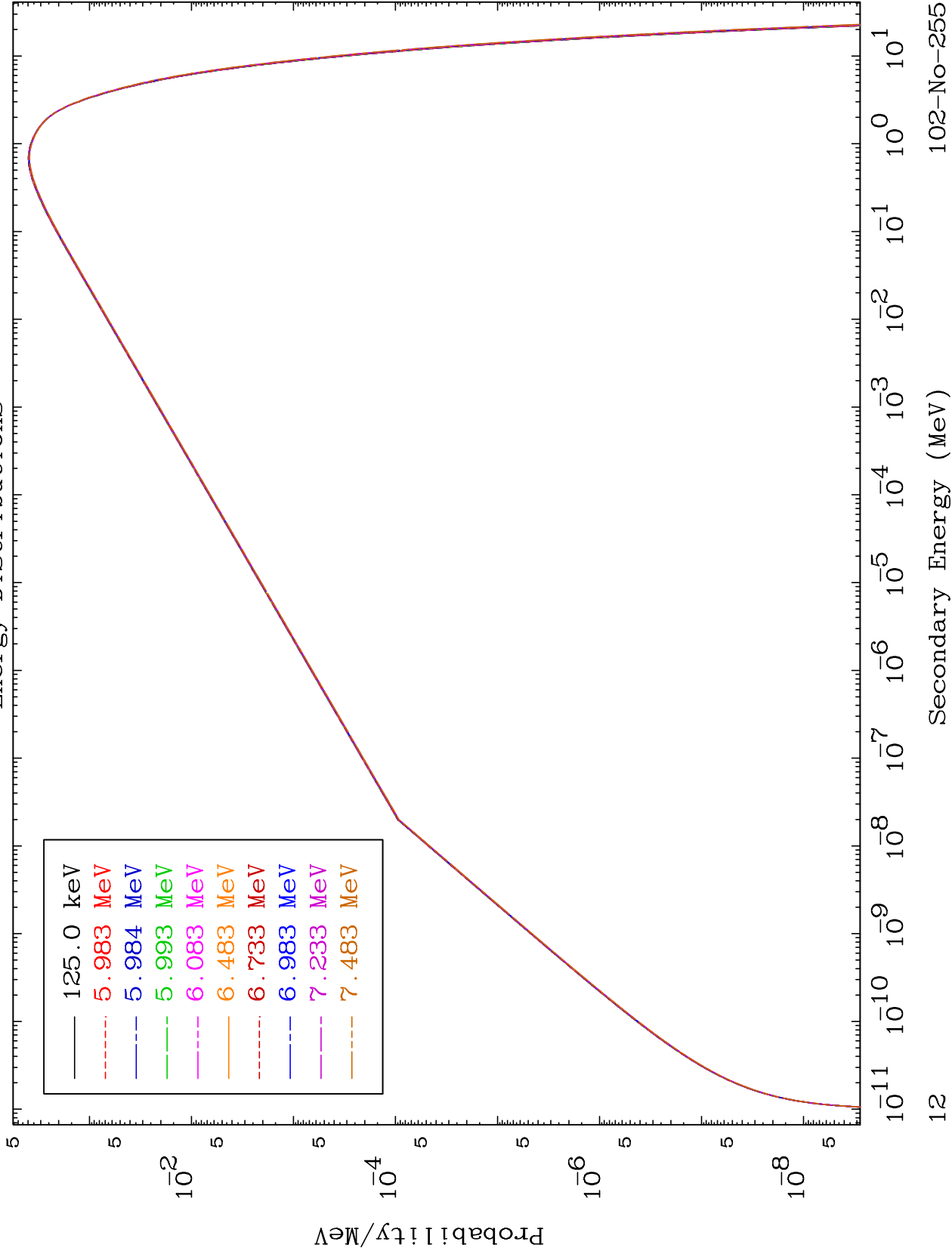


Incident Energy (MeV)

MAT 9973

### Fission Energy Distributions

102-No-255



102-No-255

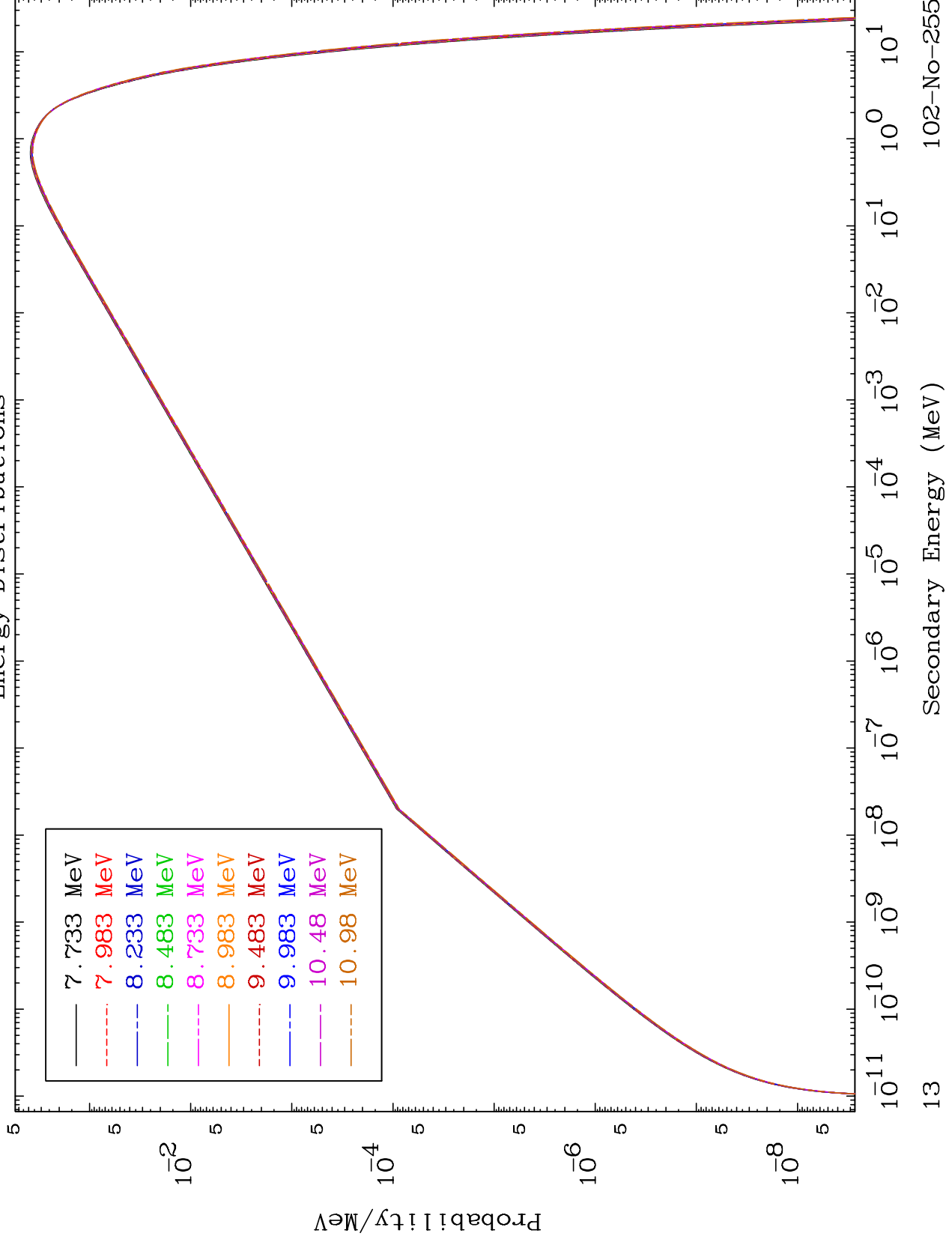
Secondary Energy (MeV)

12

MAT 9973

Fission  
Energy Distributions

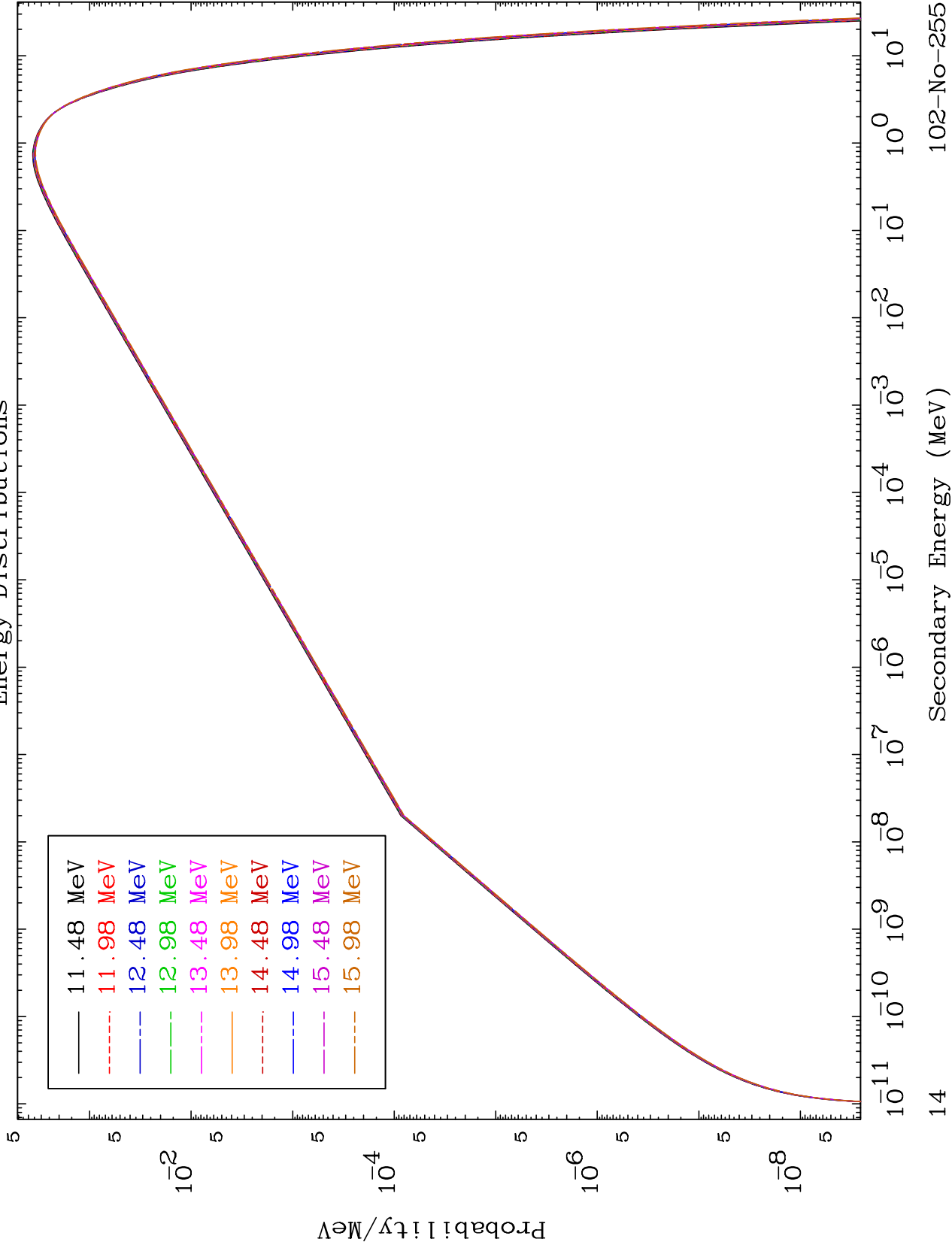
102-No-255



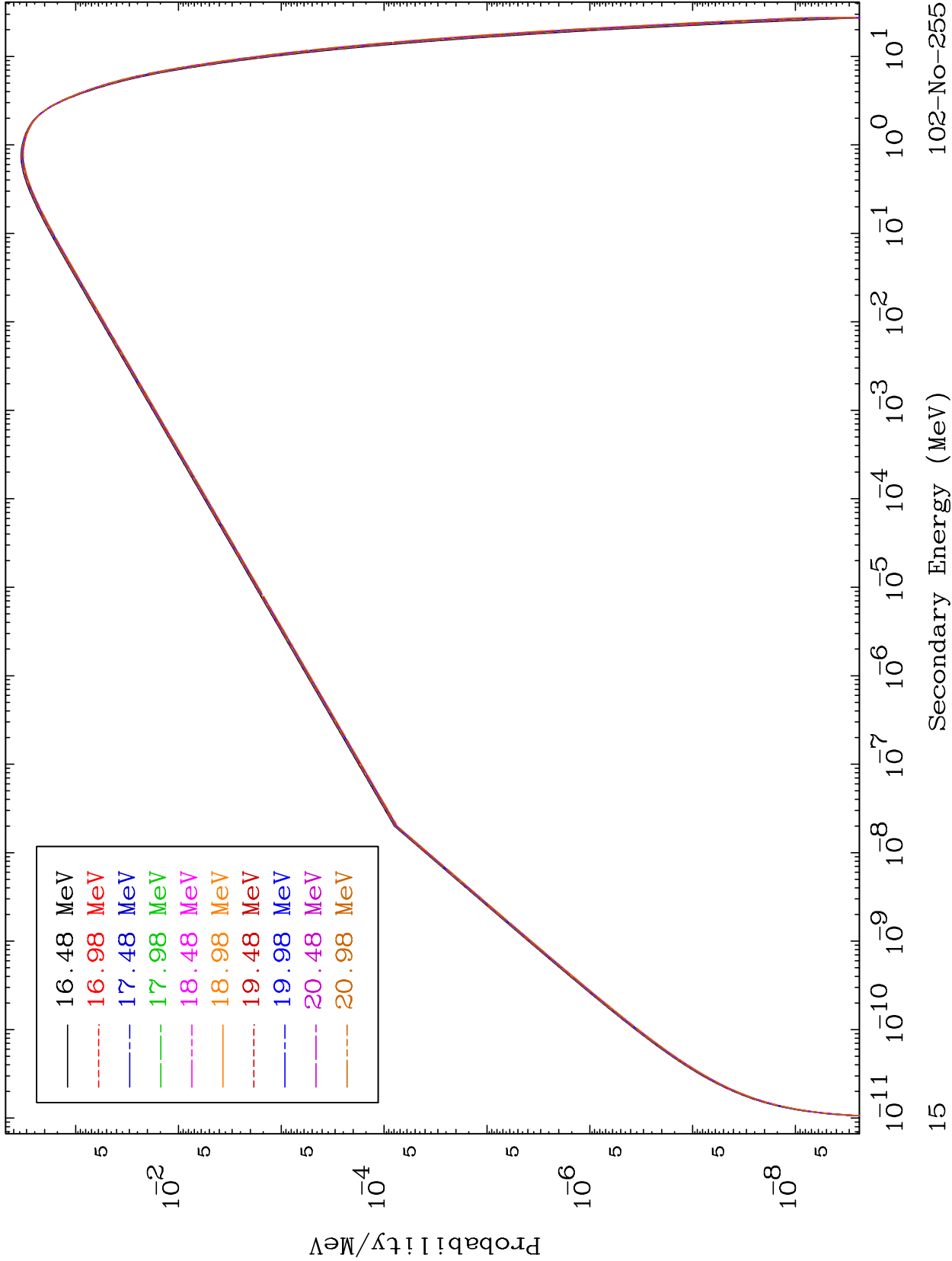
MAT 9973

### Fission Energy Distributions

102-No-255

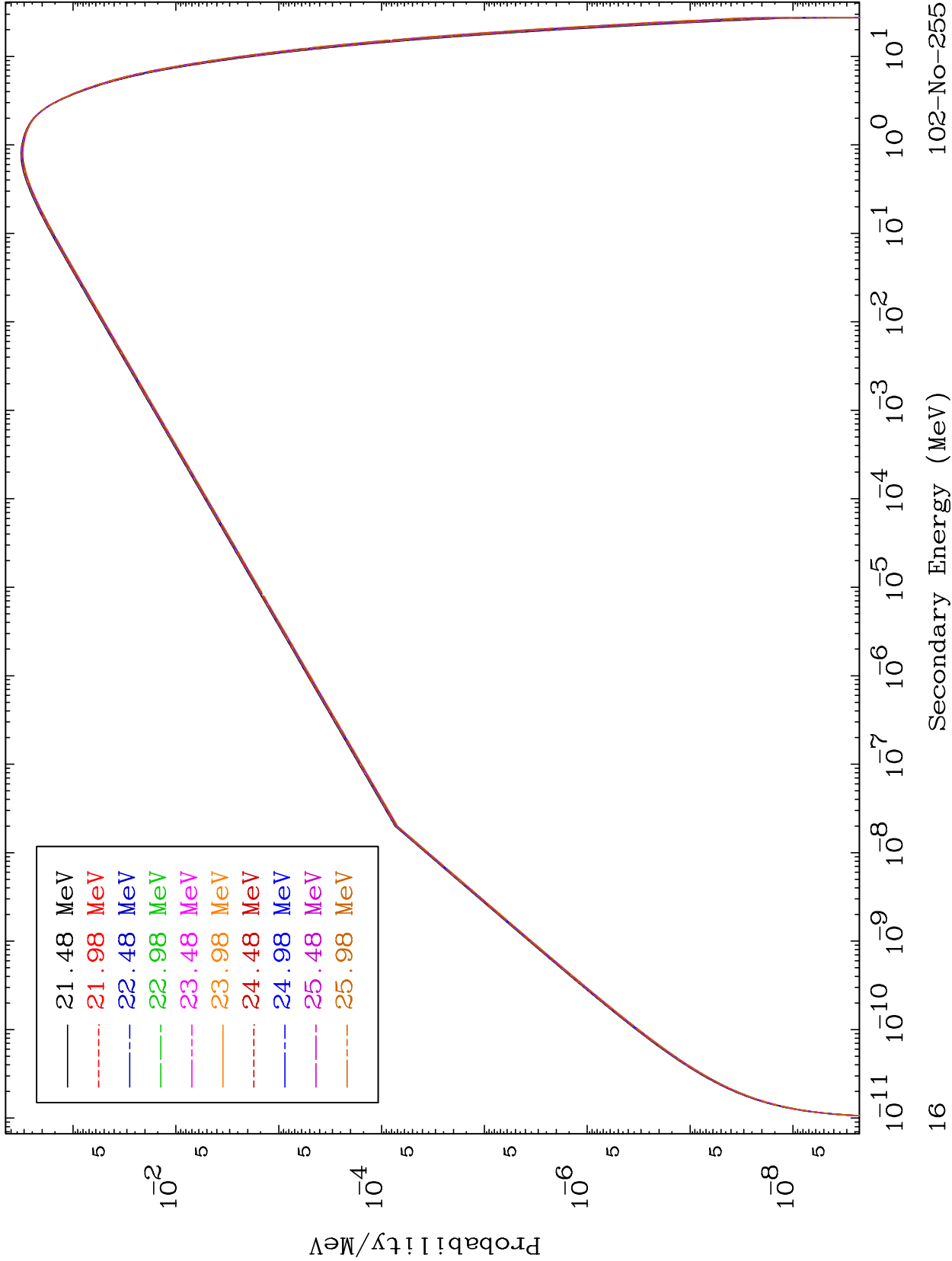


Fission Energy Distributions





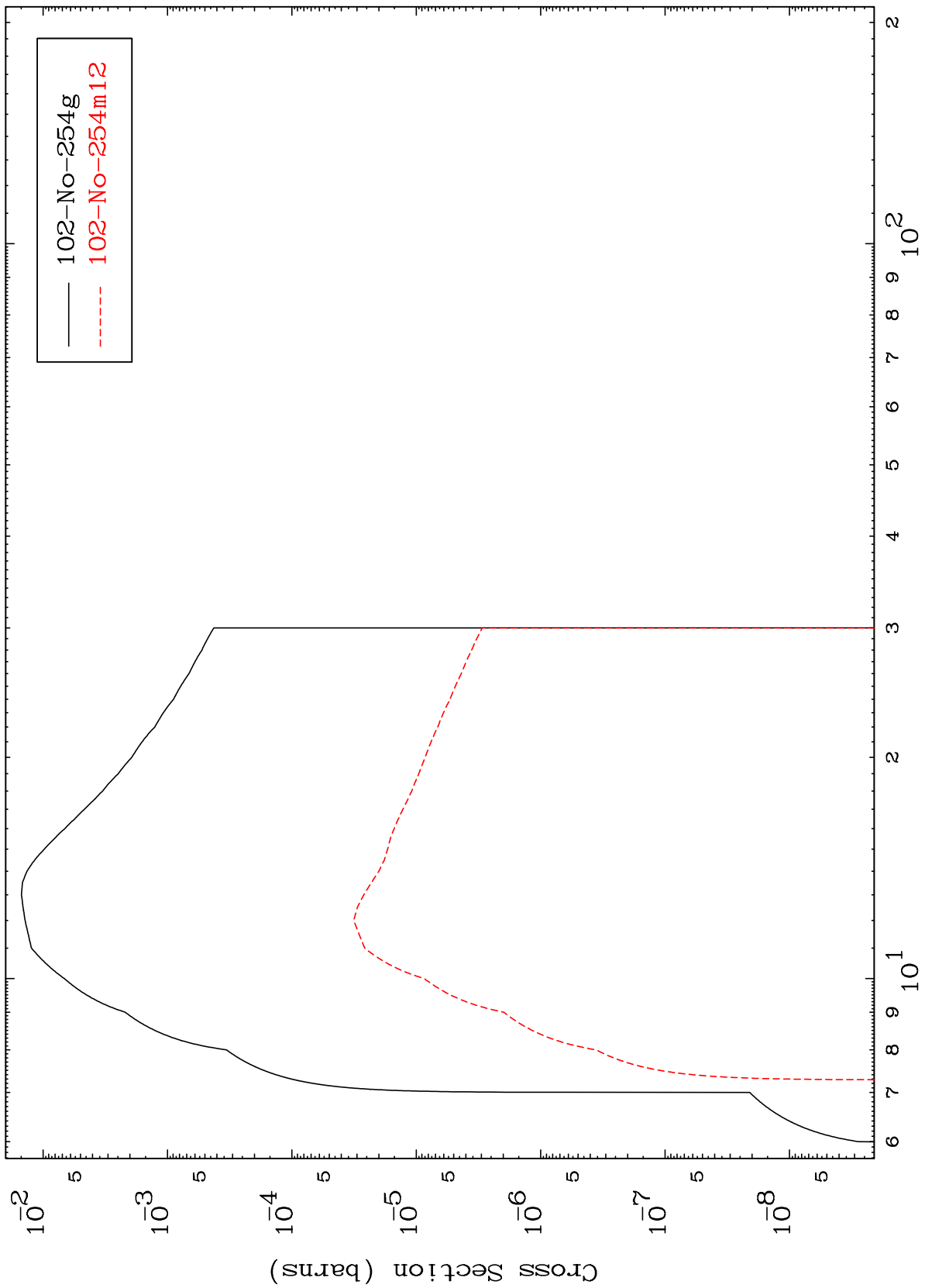
Fission  
Energy Distributions



MAT 9973

102-No-255

Inelastic  
Radionuclide Production Cross Section



102-No-255

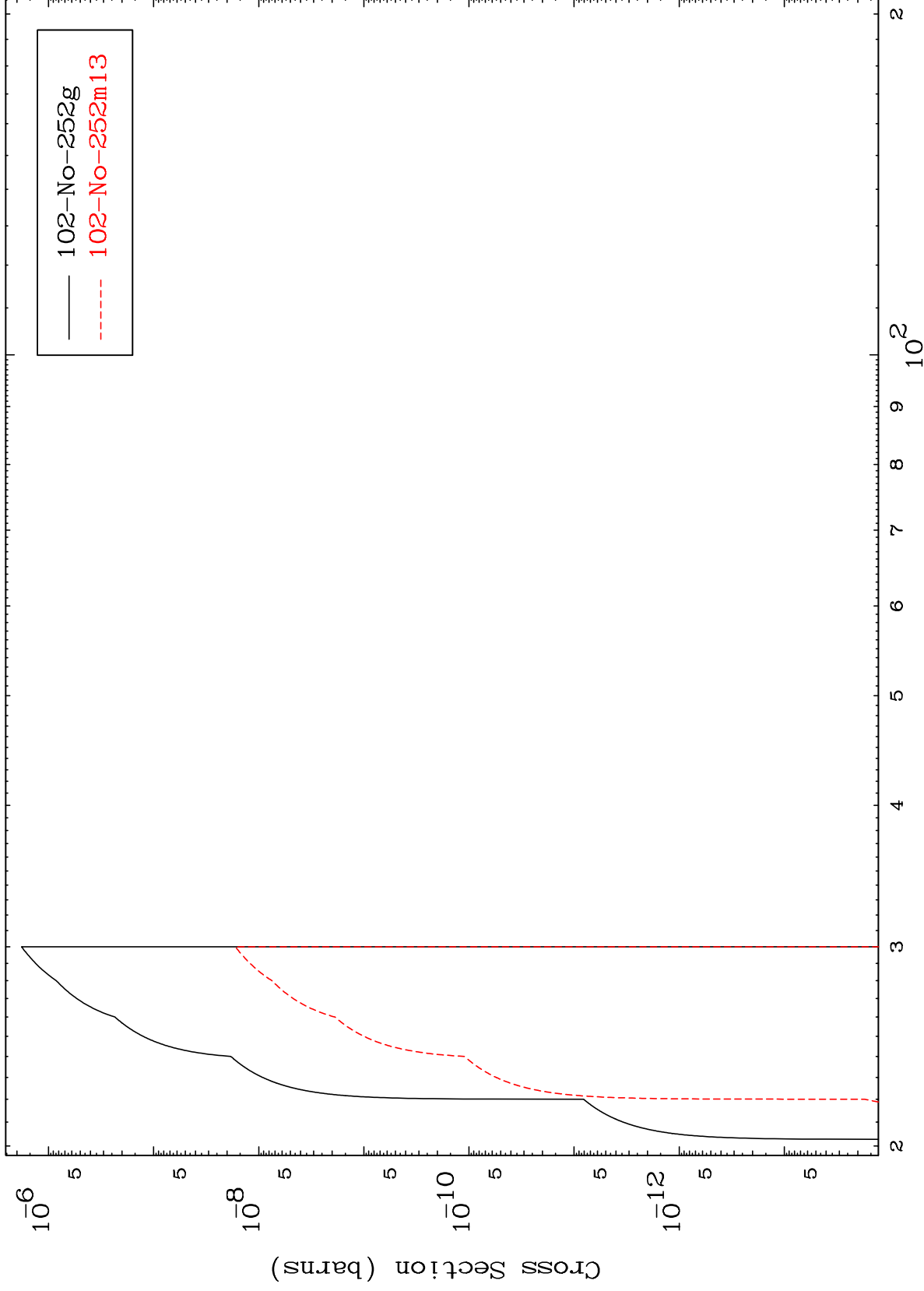
Incident Energy (MeV)

17

MAT 9973

102-No-255

(n,3n)  
Radionuclide Production Cross Section



102-No-255

Incident Energy (MeV)

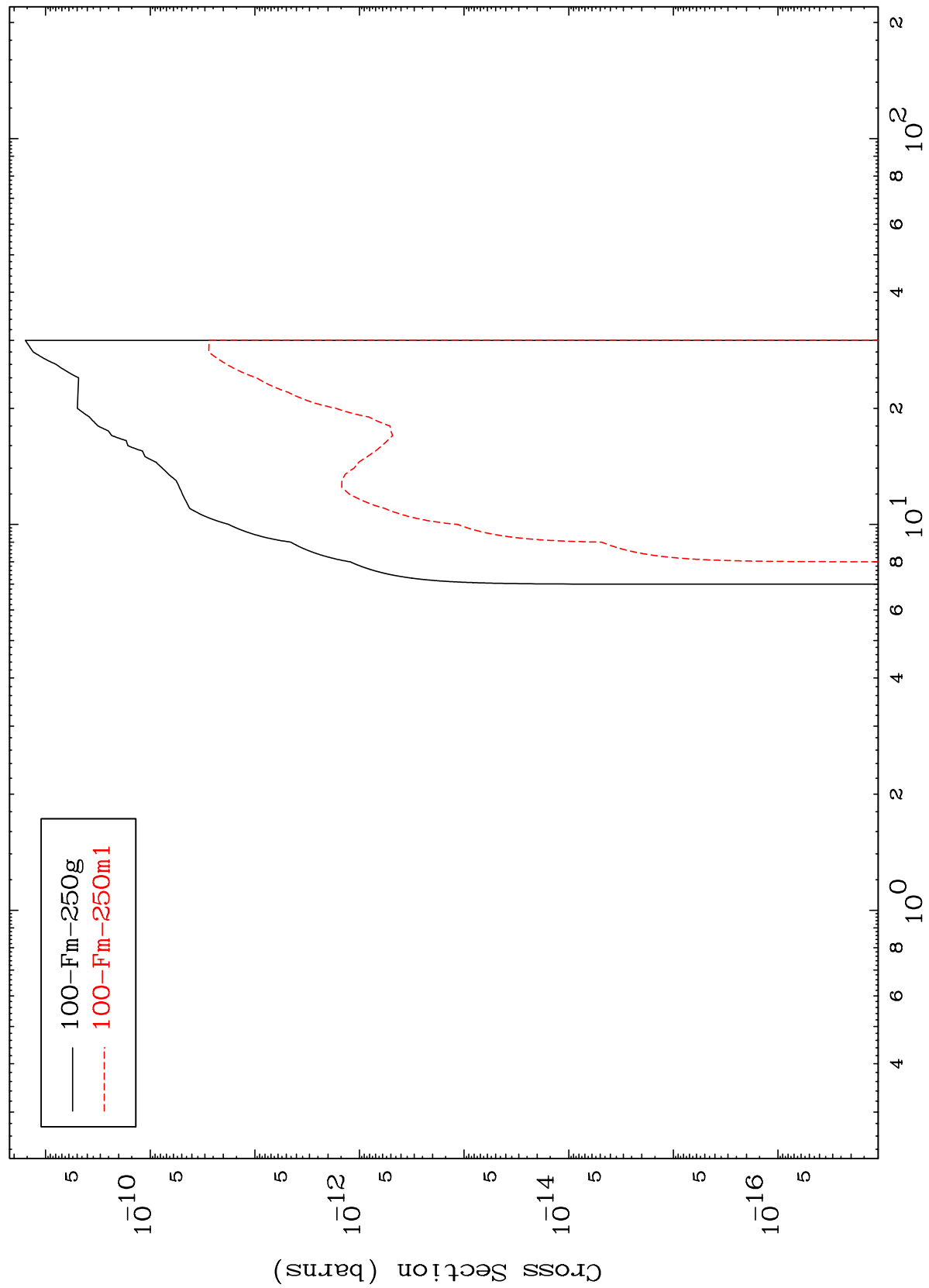
18

MAT 9973

(n,n')  $\alpha$

102-No-255

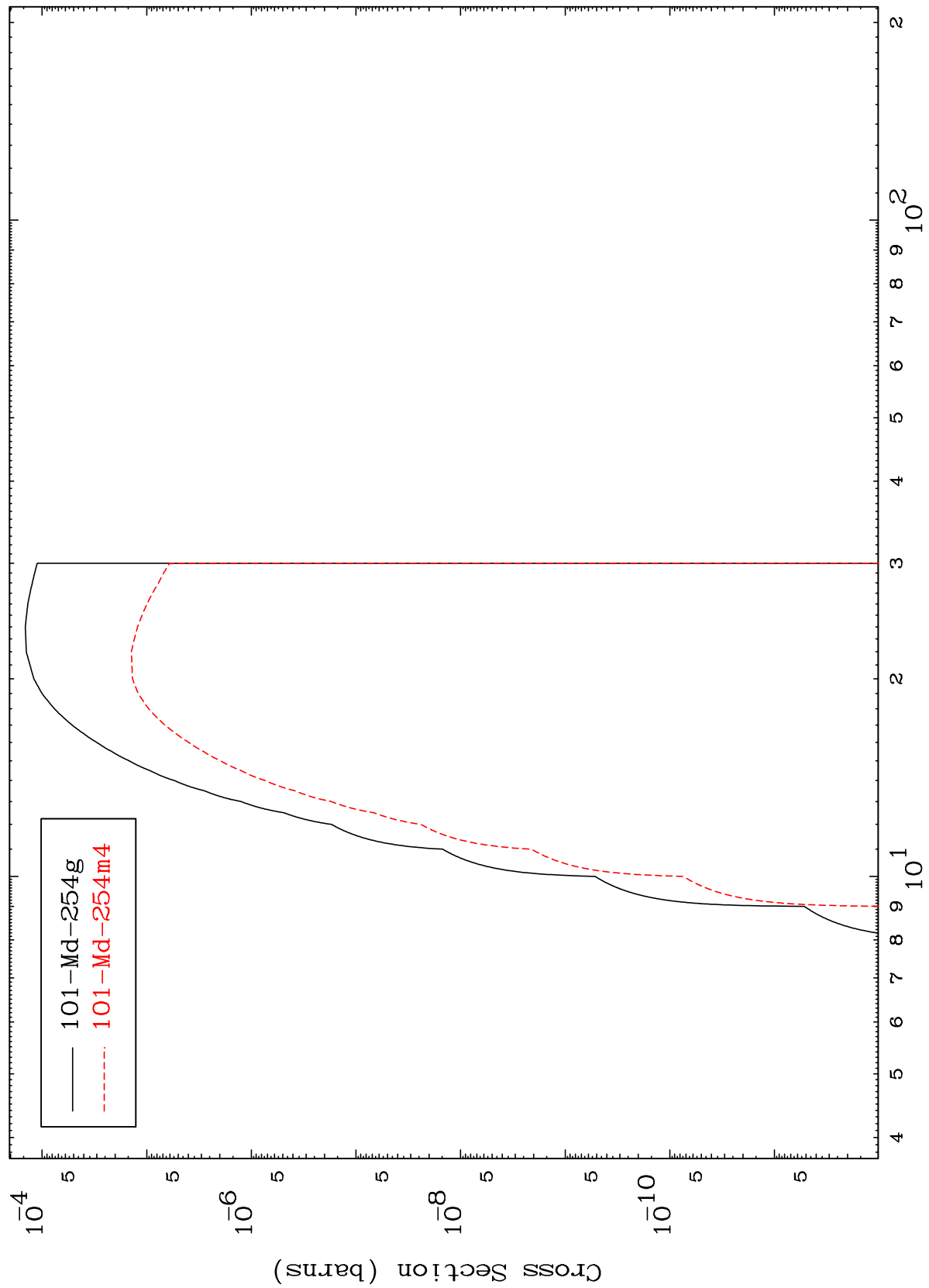
Radionuclide Production Cross Section



MAT 9973

102-No-255

(n,p)  
Radionuclide Production Cross Section



20

Incident Energy (MeV)

102-No-255