

Program EVALPLOT
(Version 2021-1)

by

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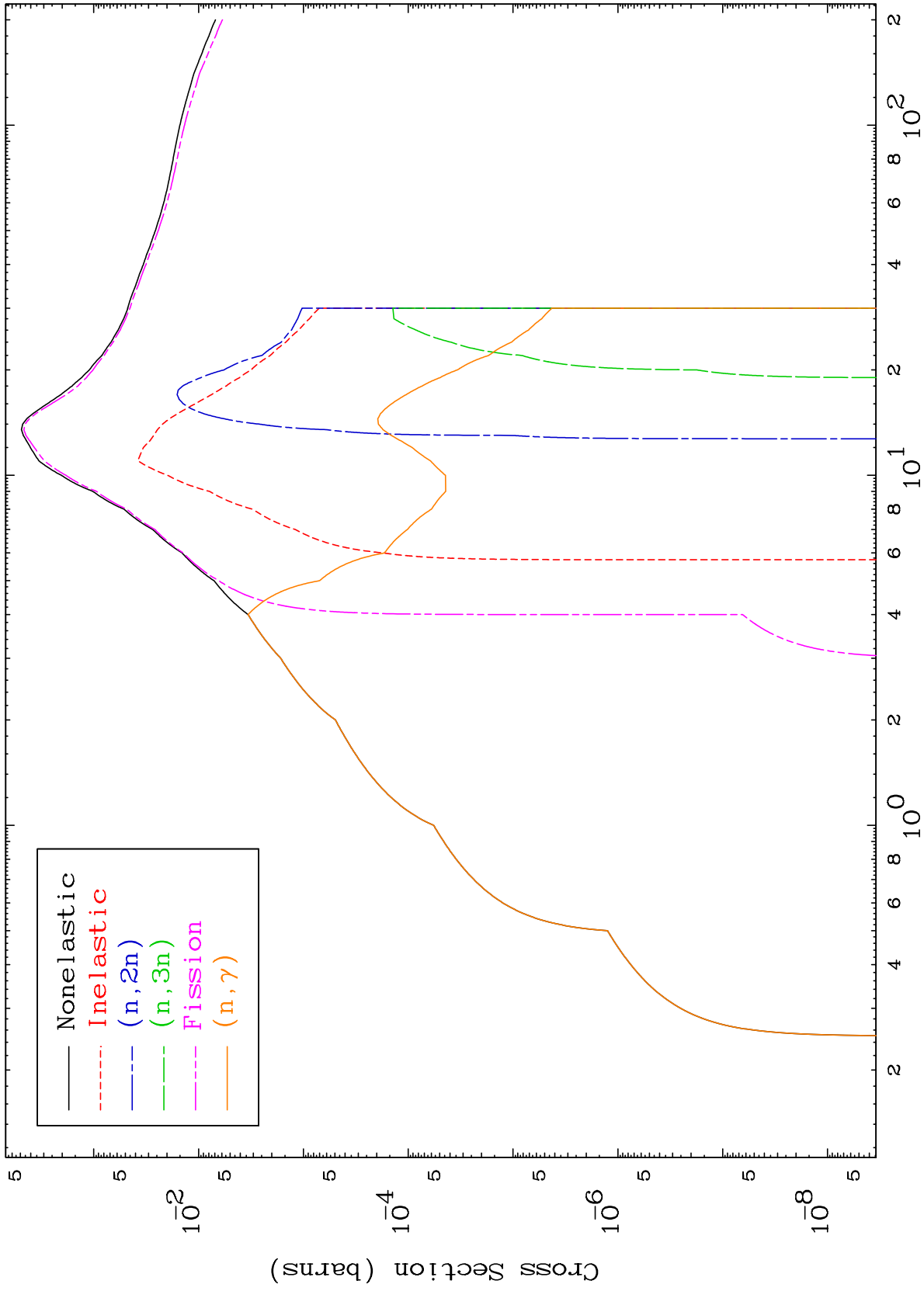
Press Mouse Button to Start

MAT 9343

Photon Major

93-Np-236

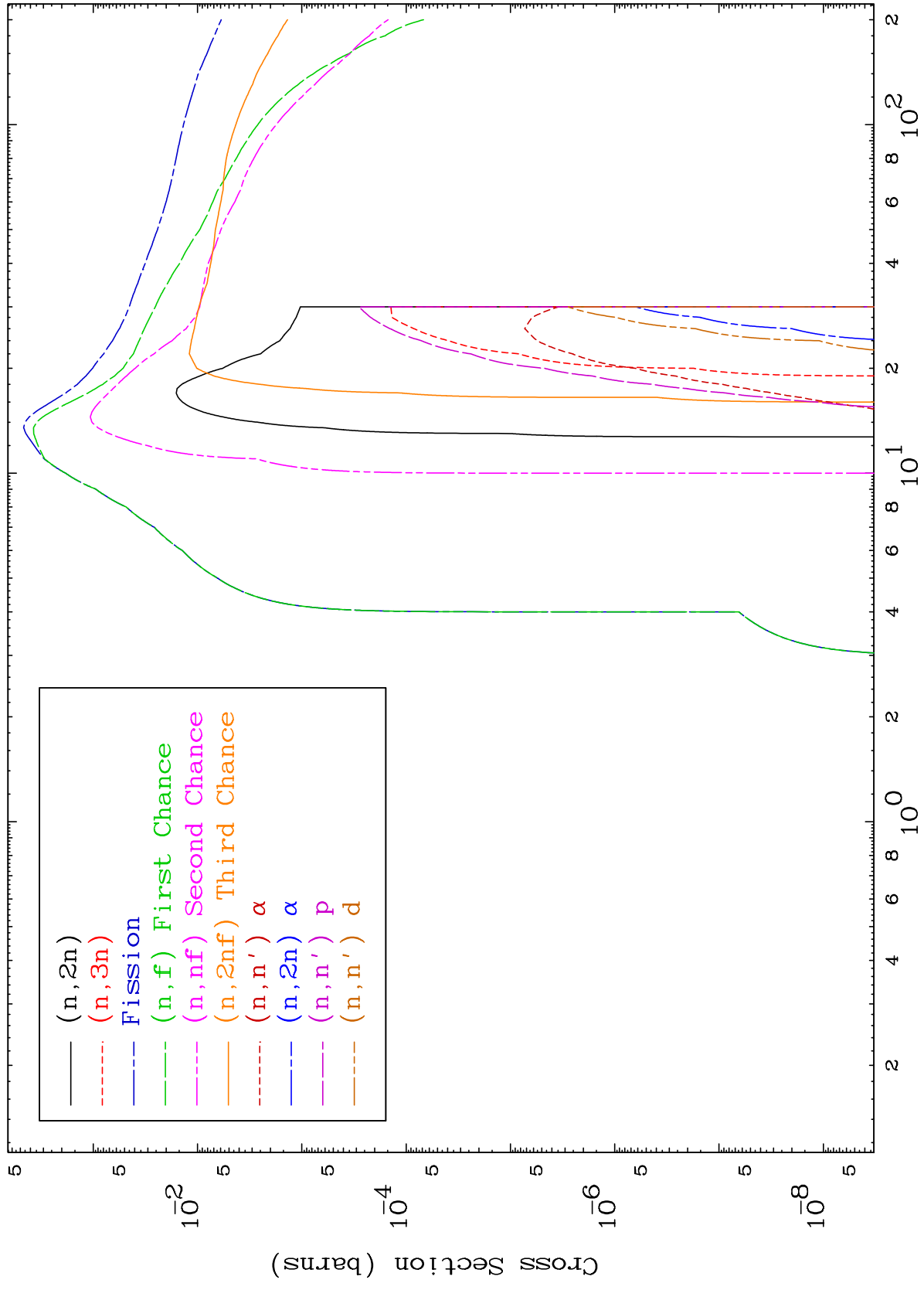
0 Kelvin Cross Sections

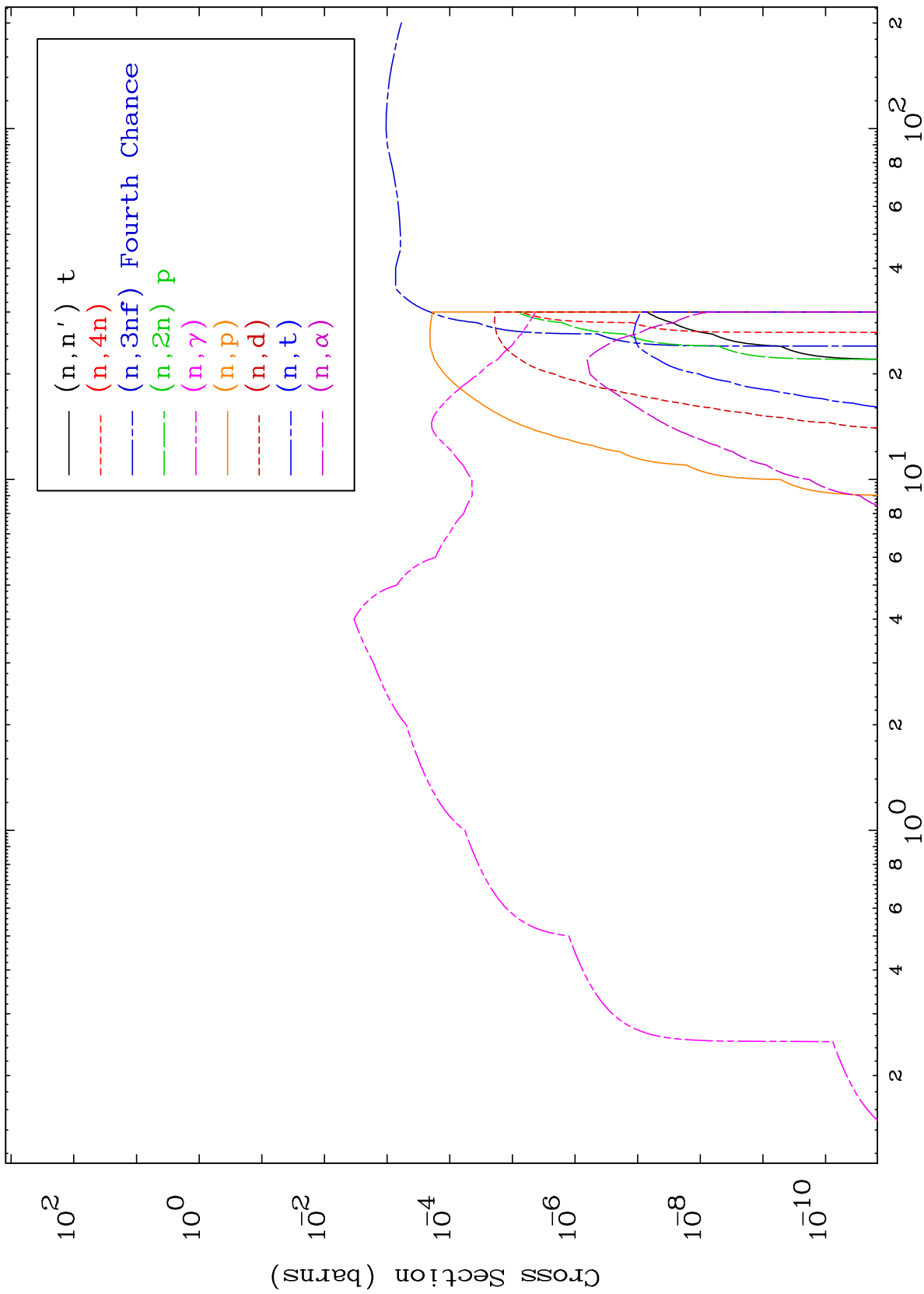


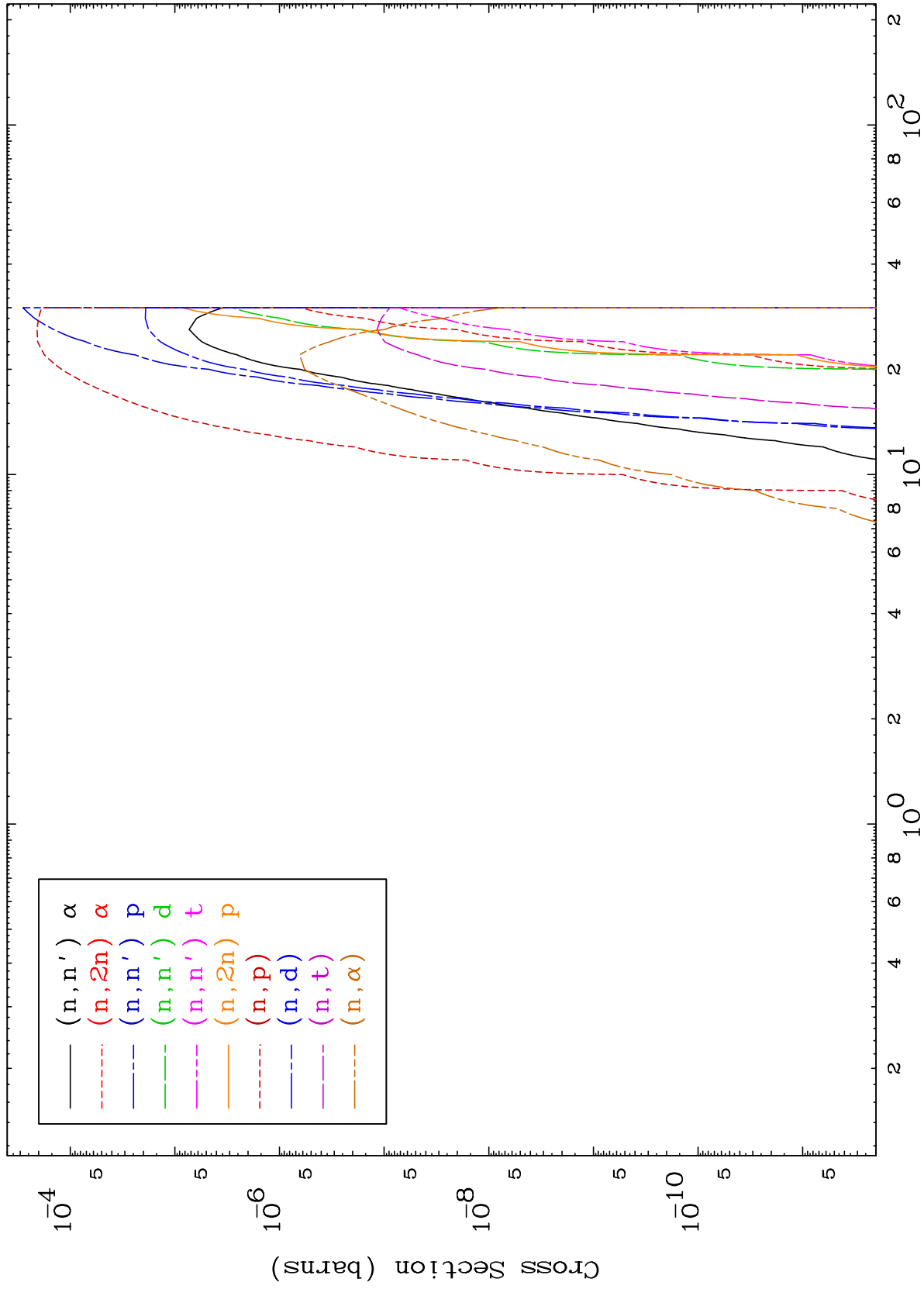
1

Incident Energy (MeV)

93-Np-236



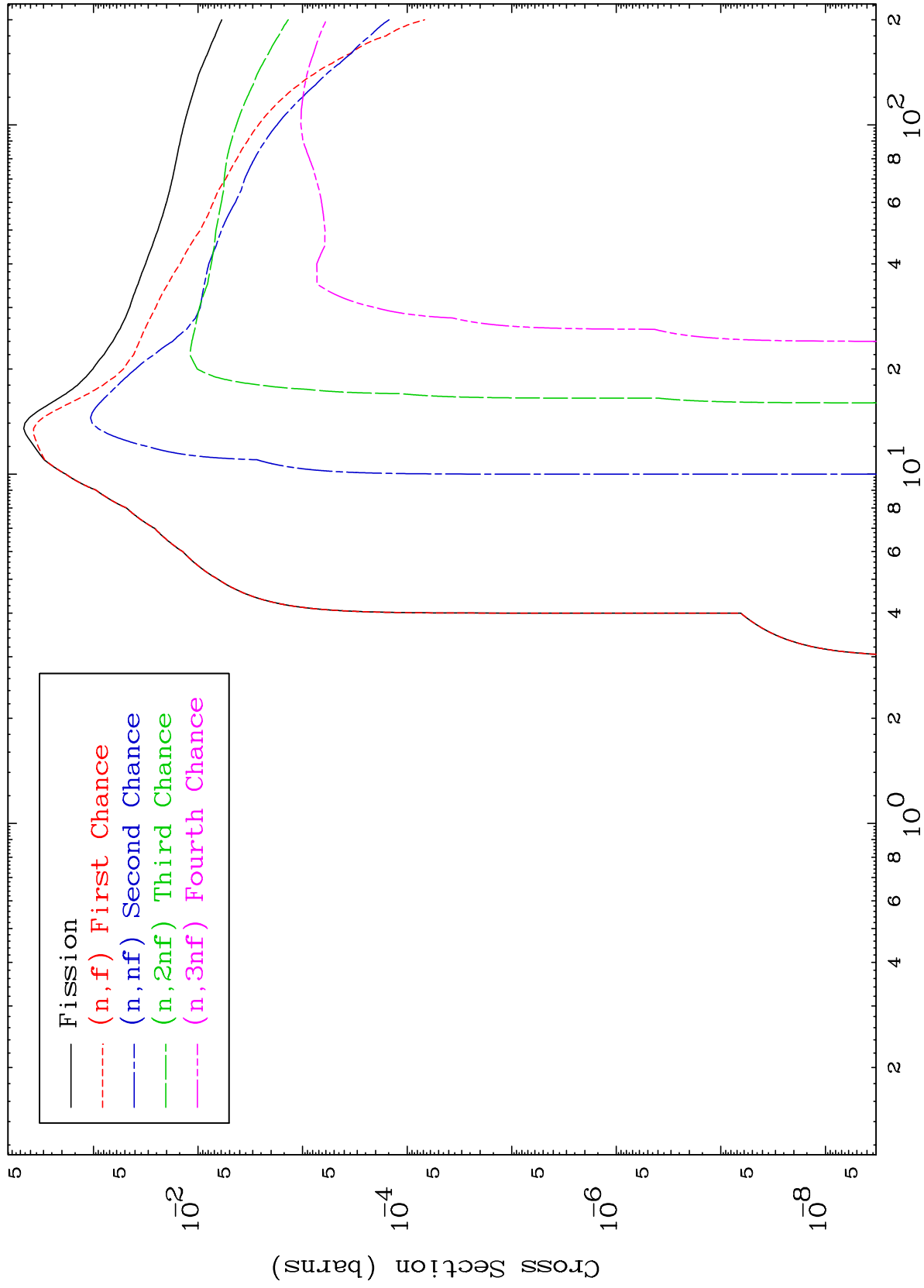




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Photon Fission
0 Kelvin Cross Sections

93-Np-236



5

Incident Energy (MeV)

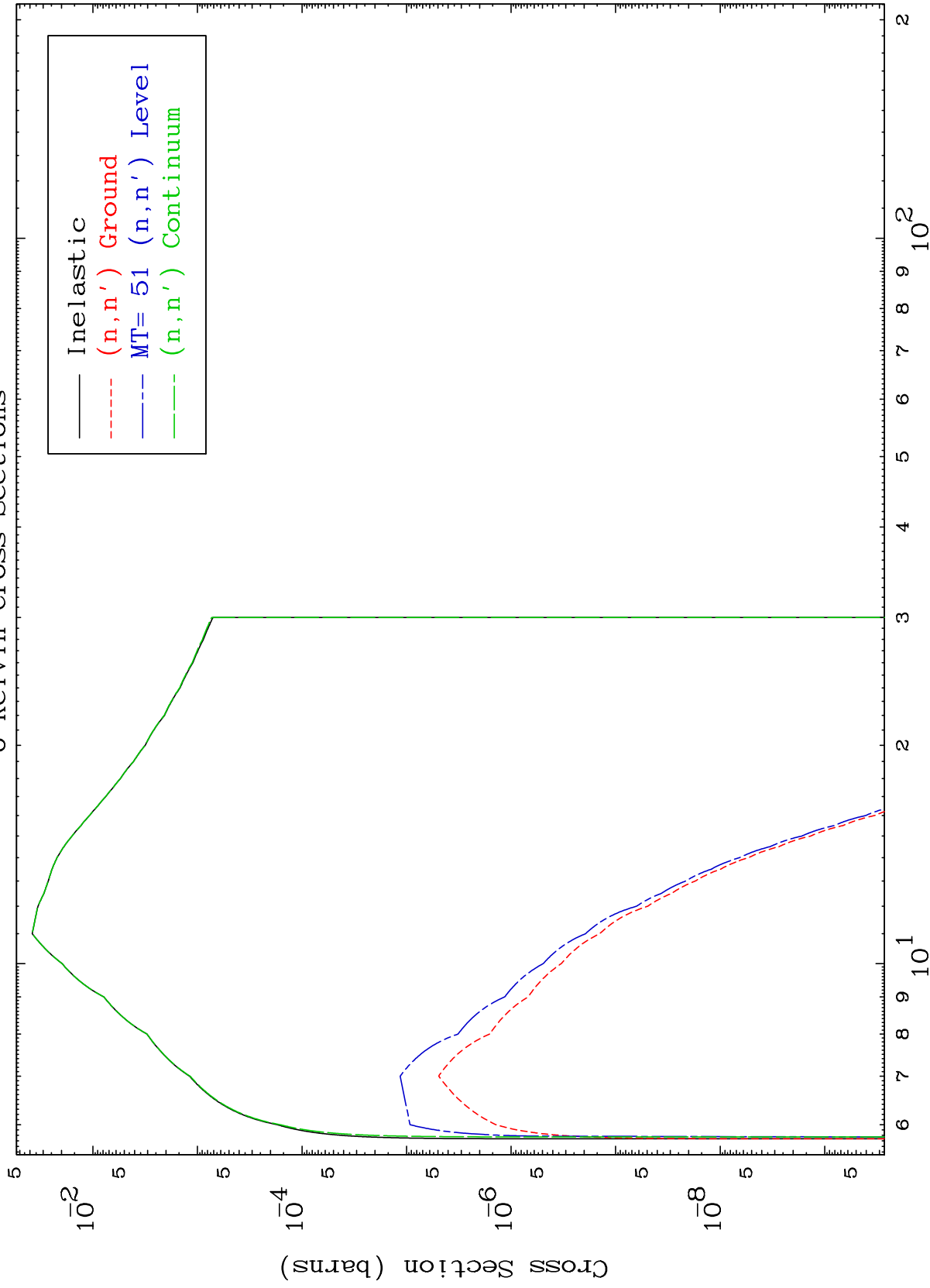
93-Np-236

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(γ, n') Levels

0 Kelvin Cross Sections

93-Np-236



6

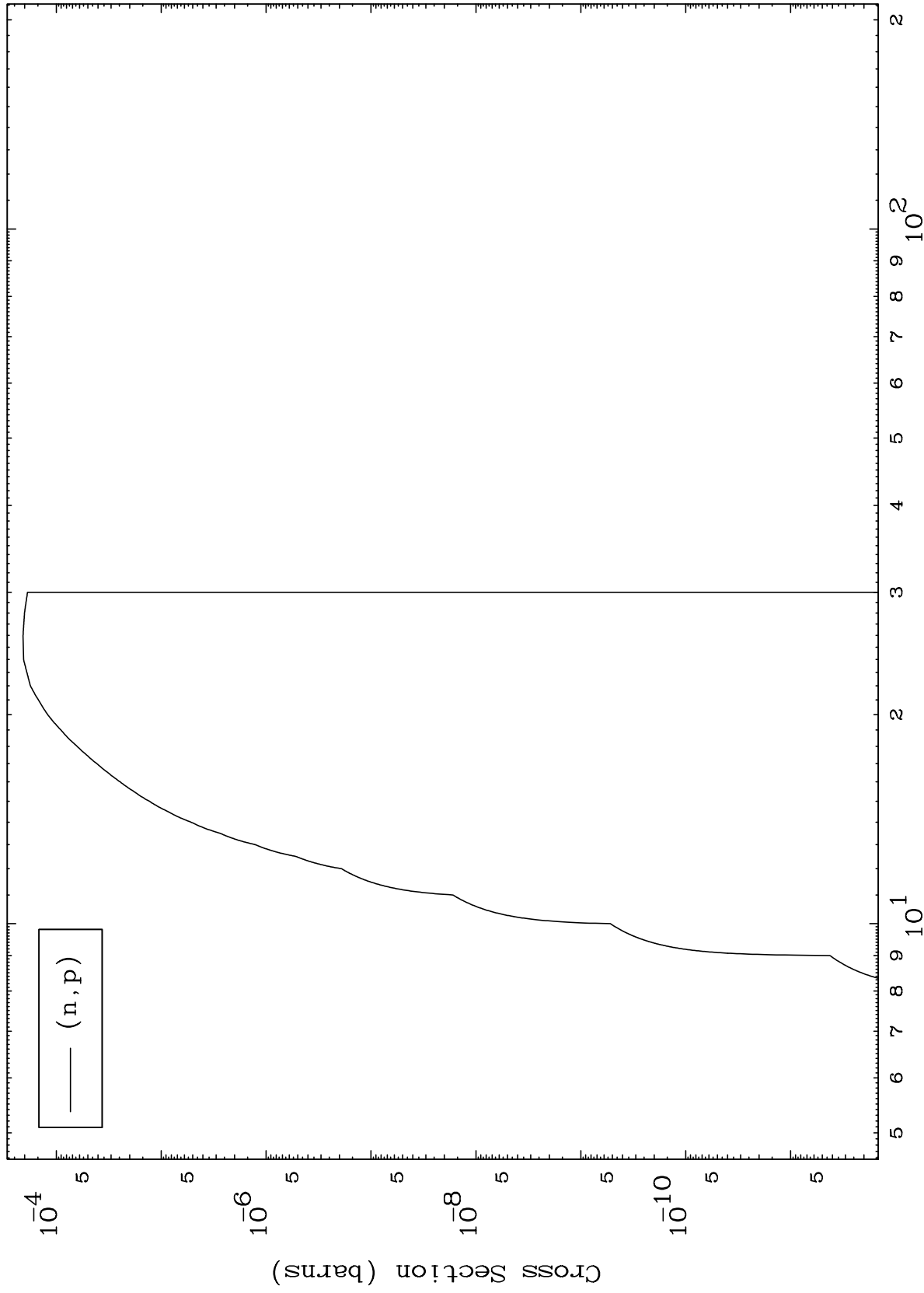
Incident Energy (MeV)

93-Np-236

MAT 9343

93-Np-236

(γ ,p) Levels
0 Kelvin Cross Sections



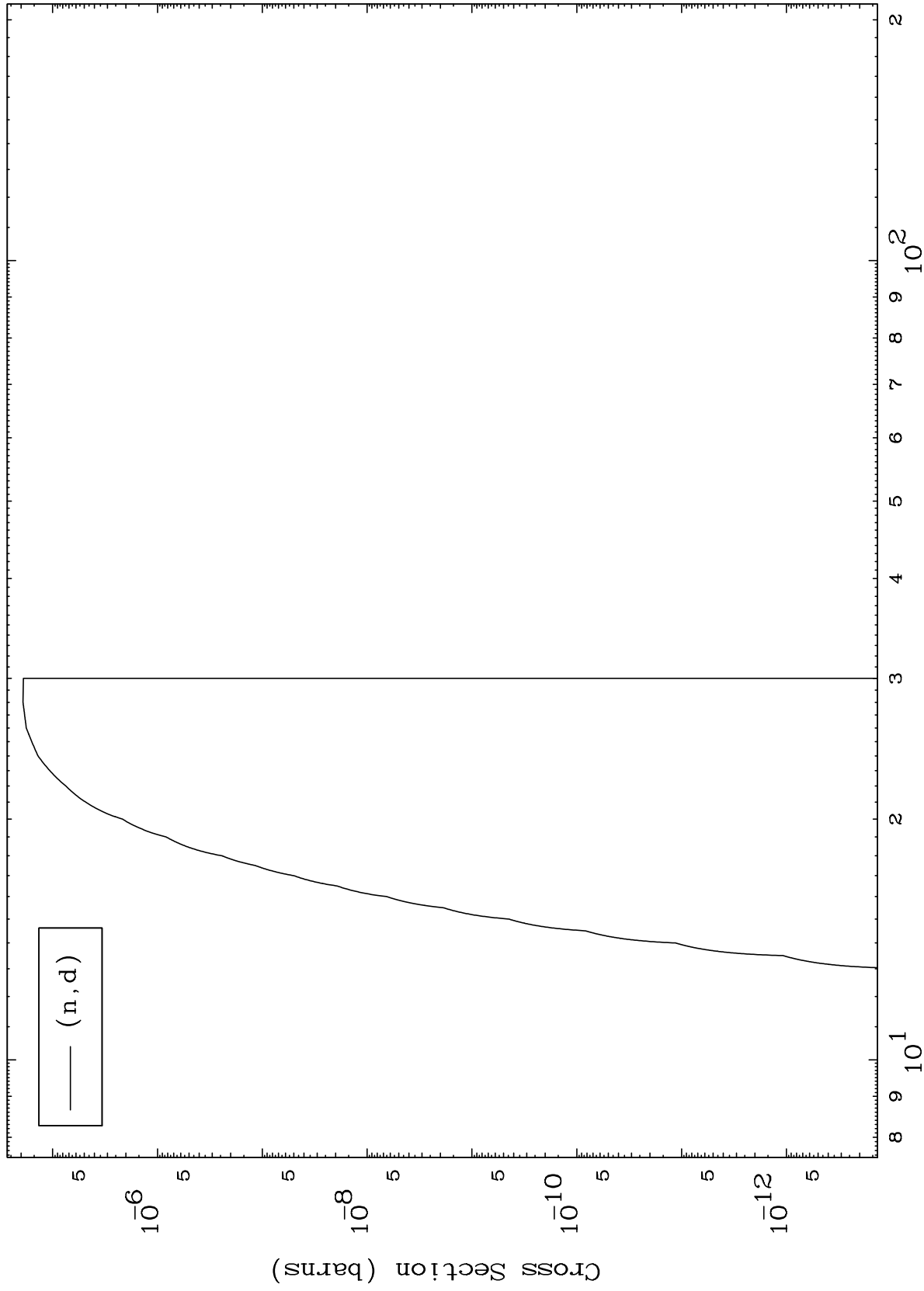
7

93-Np-236

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93-Np-236

(γ ,d) Levels
0 Kelvin Cross Sections



8

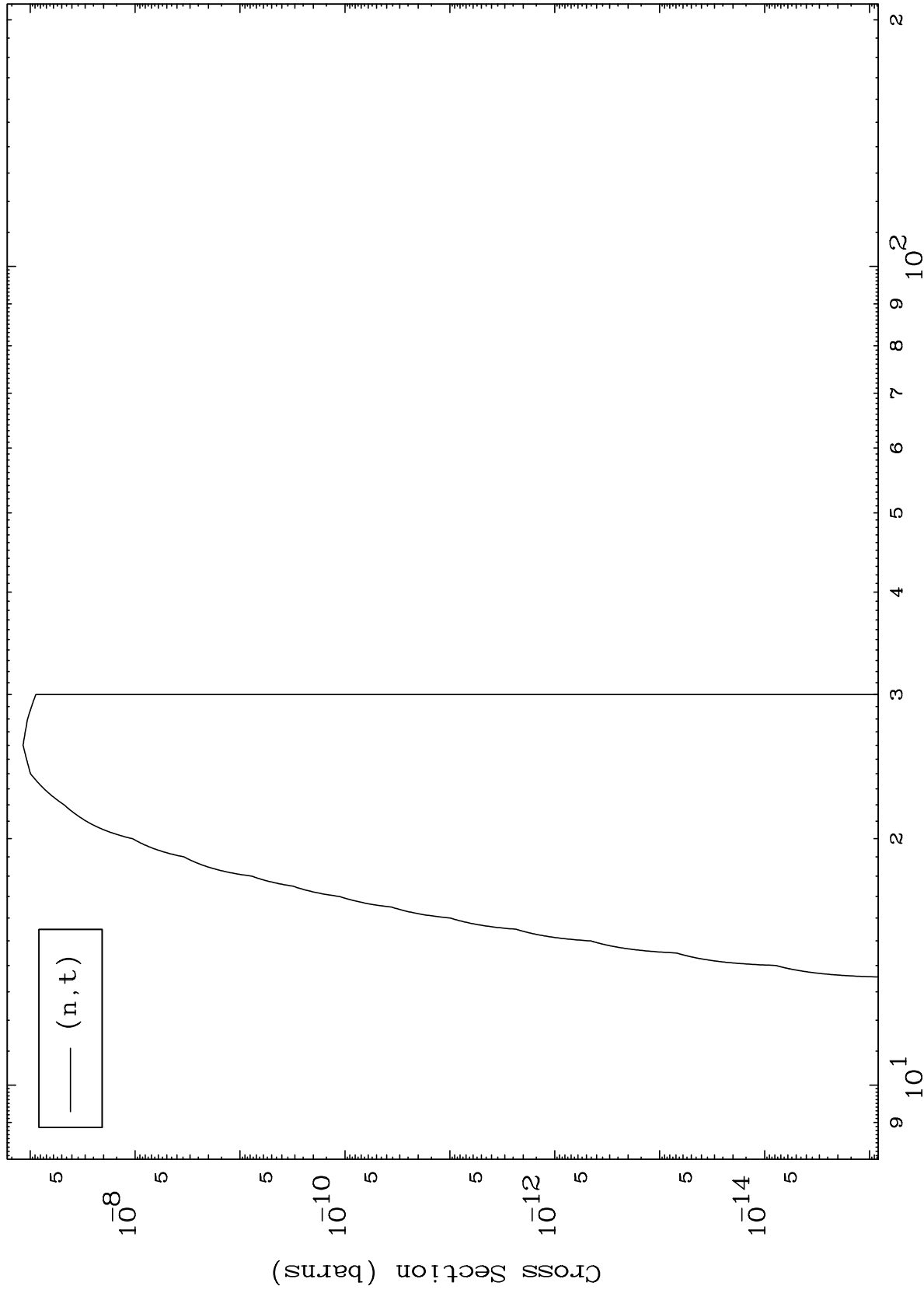
Incident Energy (MeV)

93-Np-236

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93-Np-236

(γ, t) Levels
0 Kelvin Cross Sections



9

Incident Energy (MeV)

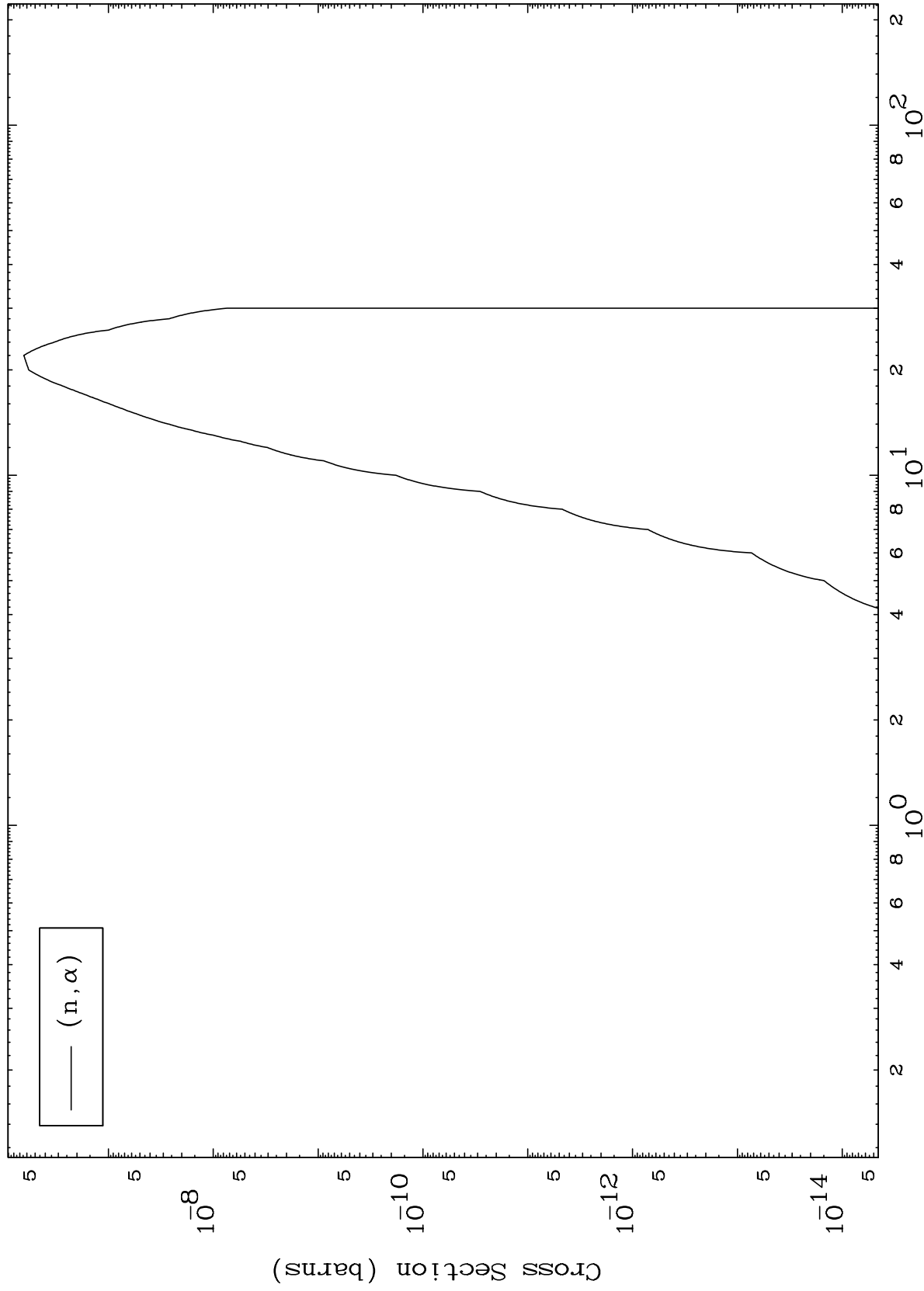
93-Np-236

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(γ, α) Levels

93-Np-236

0 Kelvin Cross Sections



10

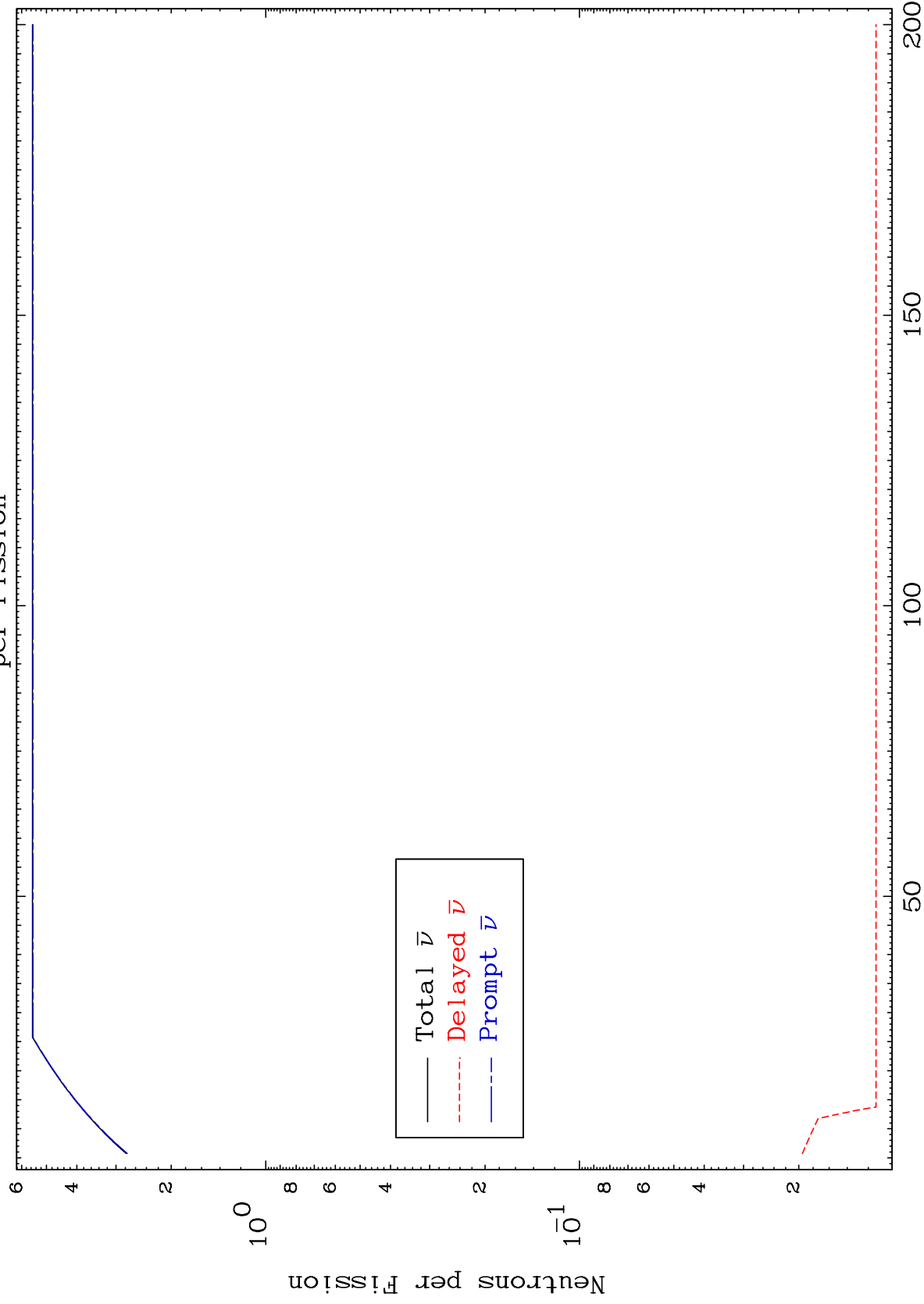
Incident Energy (MeV)

93-Np-236

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Photon Neutrons
per Fission

93-Np-236



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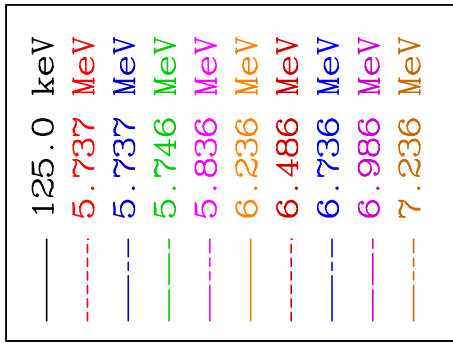
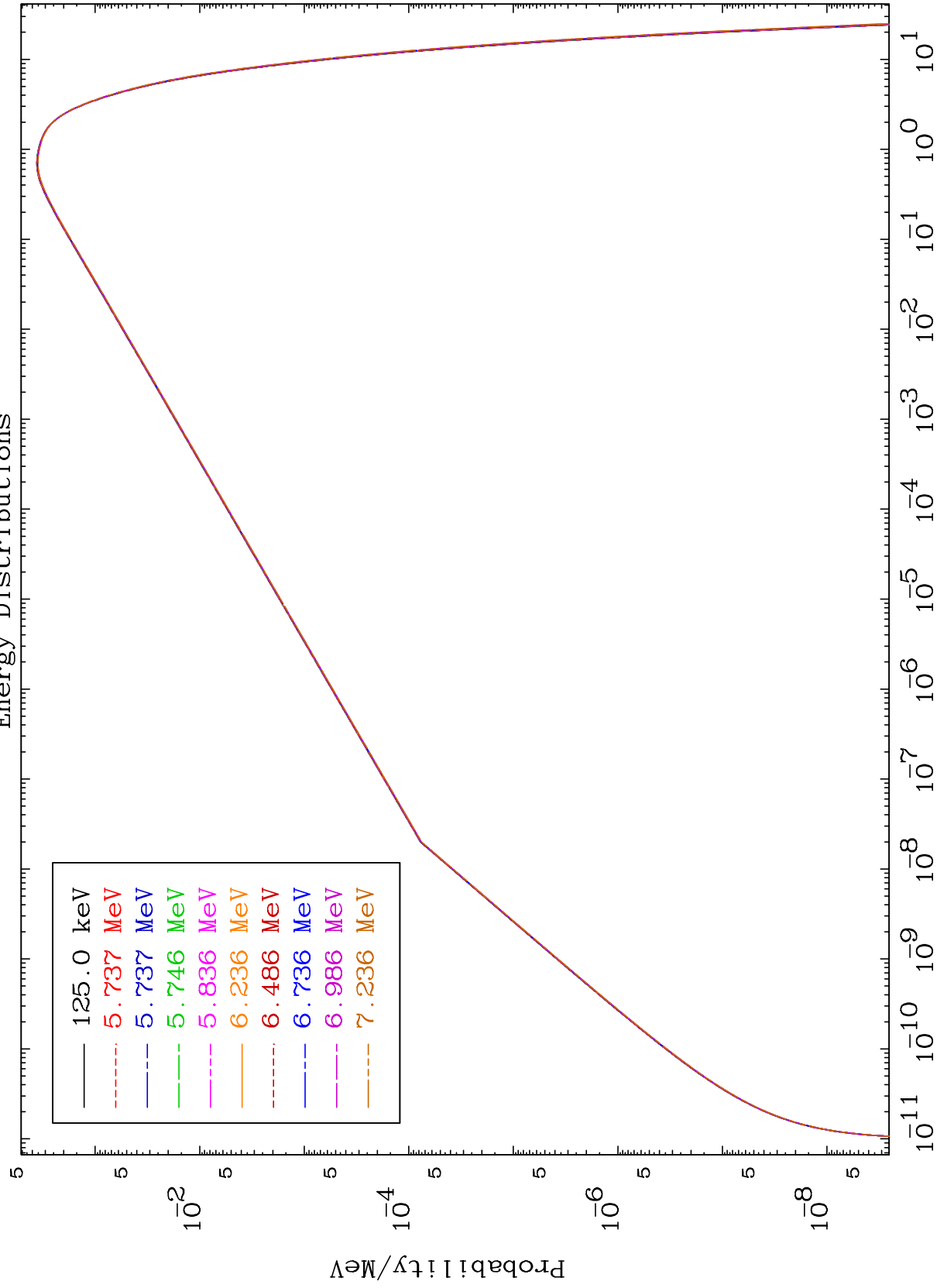
Incident Energy (MeV)

93-Np-236

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Fission
Energy Distributions

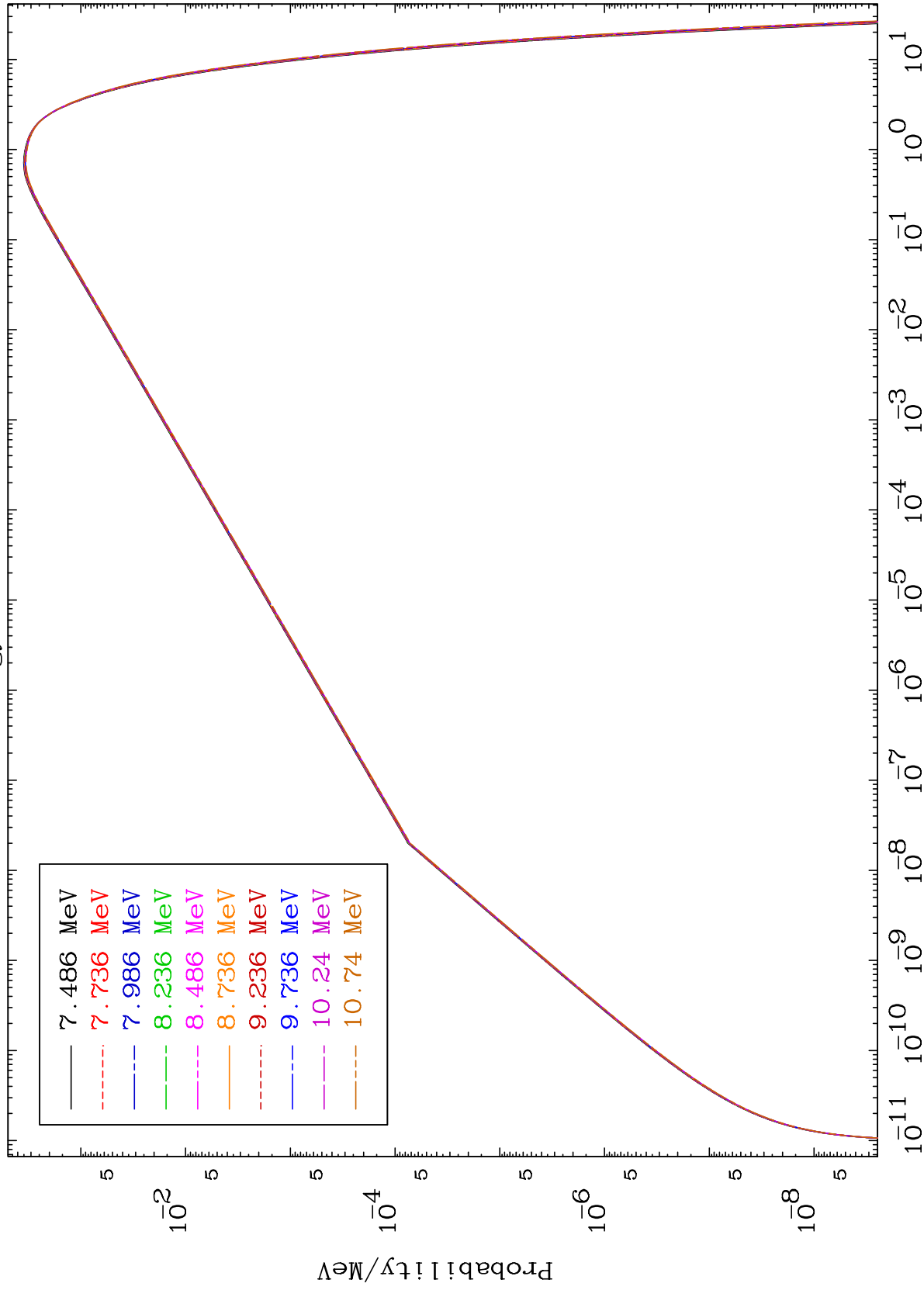
93-Np-236



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Fission
Energy Distributions

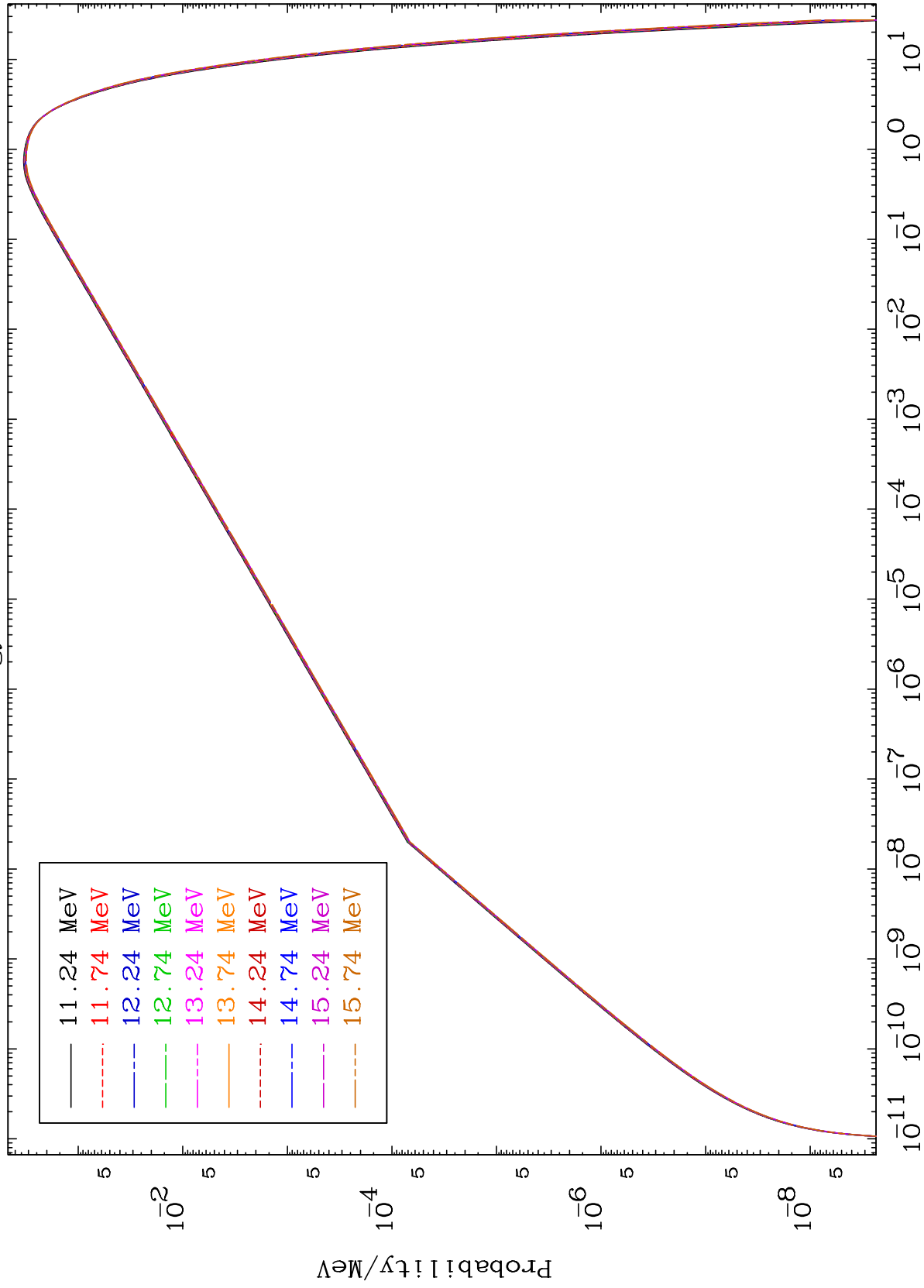
93-Np-236



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Fission
Energy Distributions

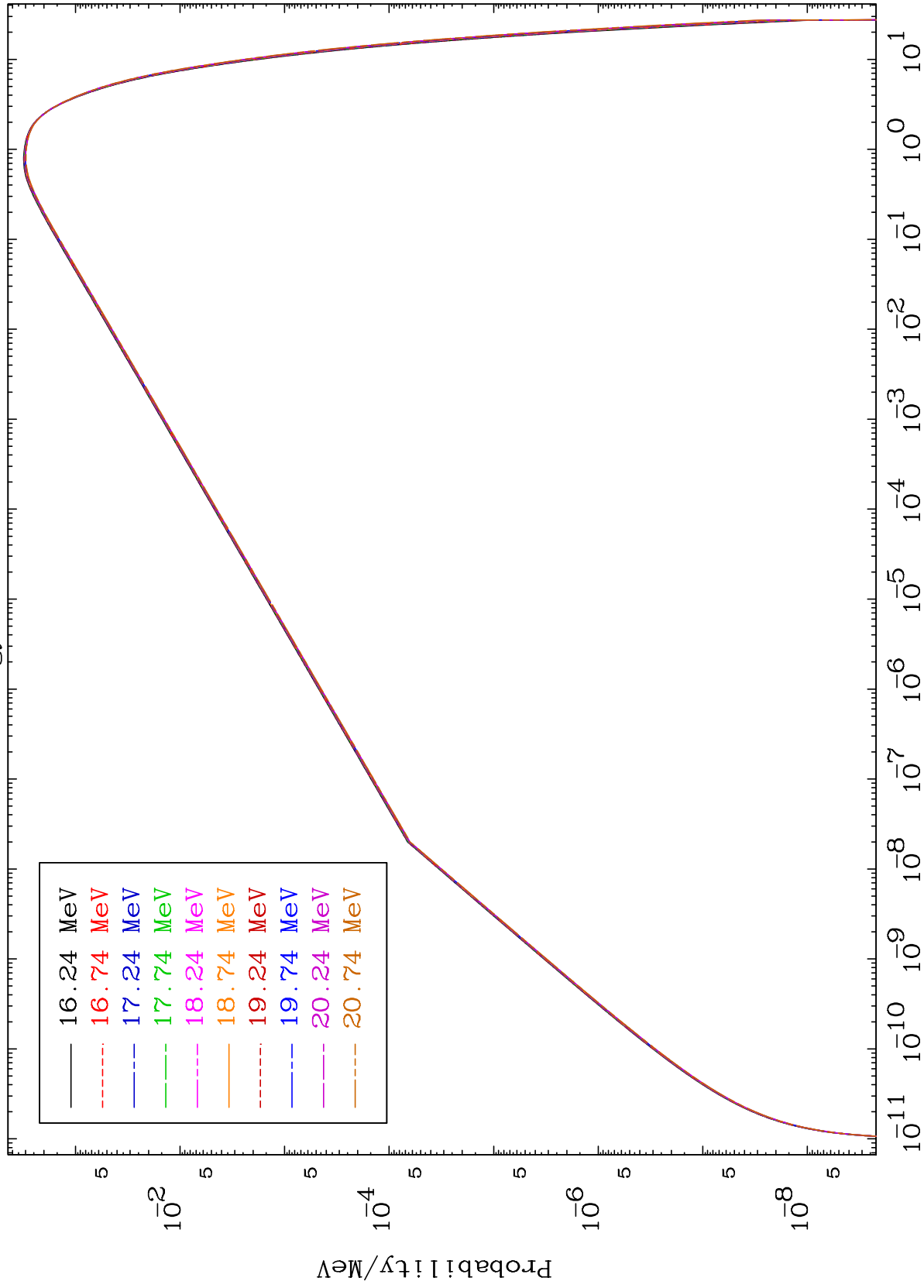
93-Np-236



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Fission
Energy Distributions

93-Np-236



15

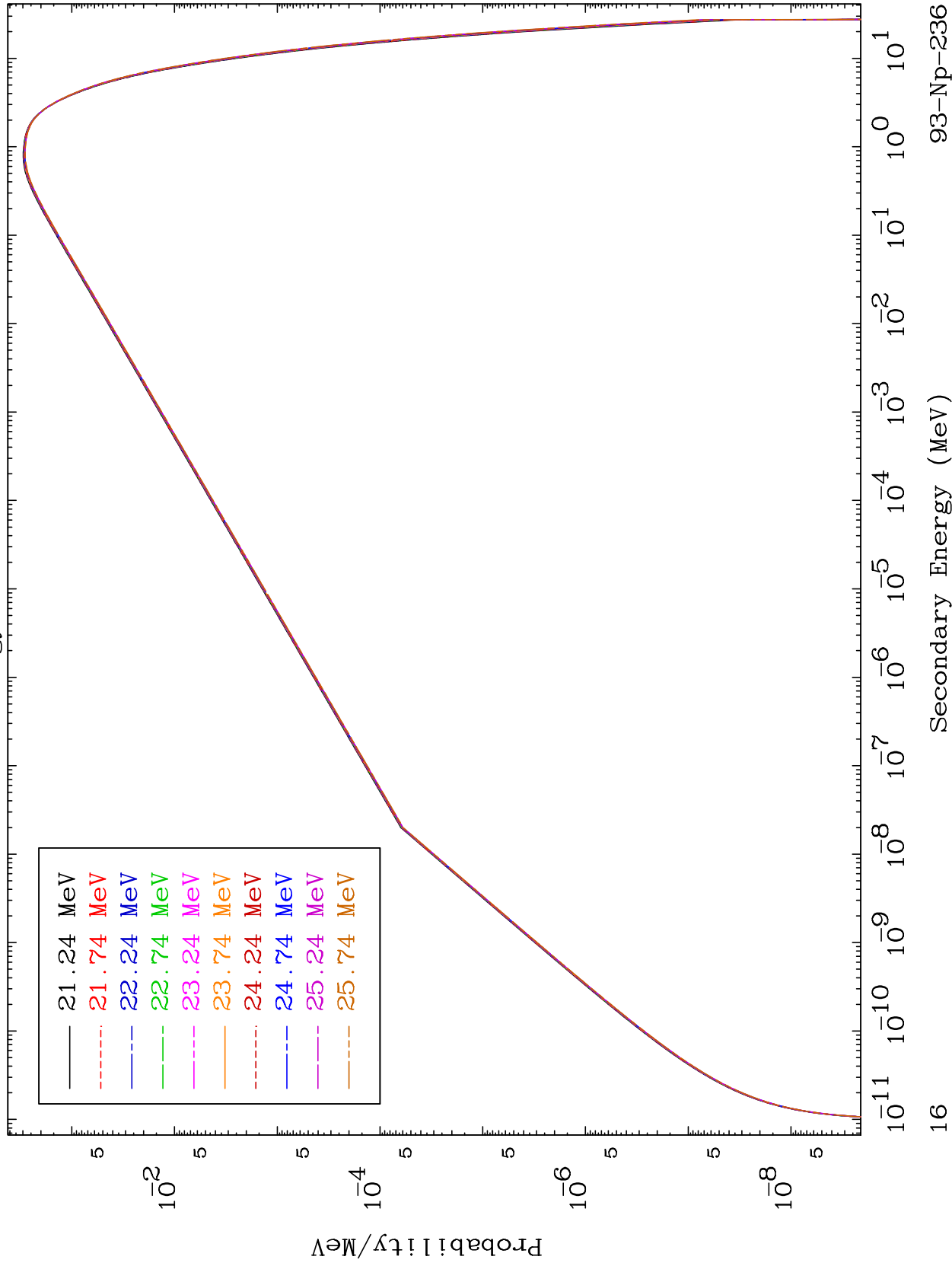
Secondary Energy (MeV)

93-Np-236

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Fission
Energy Distributions

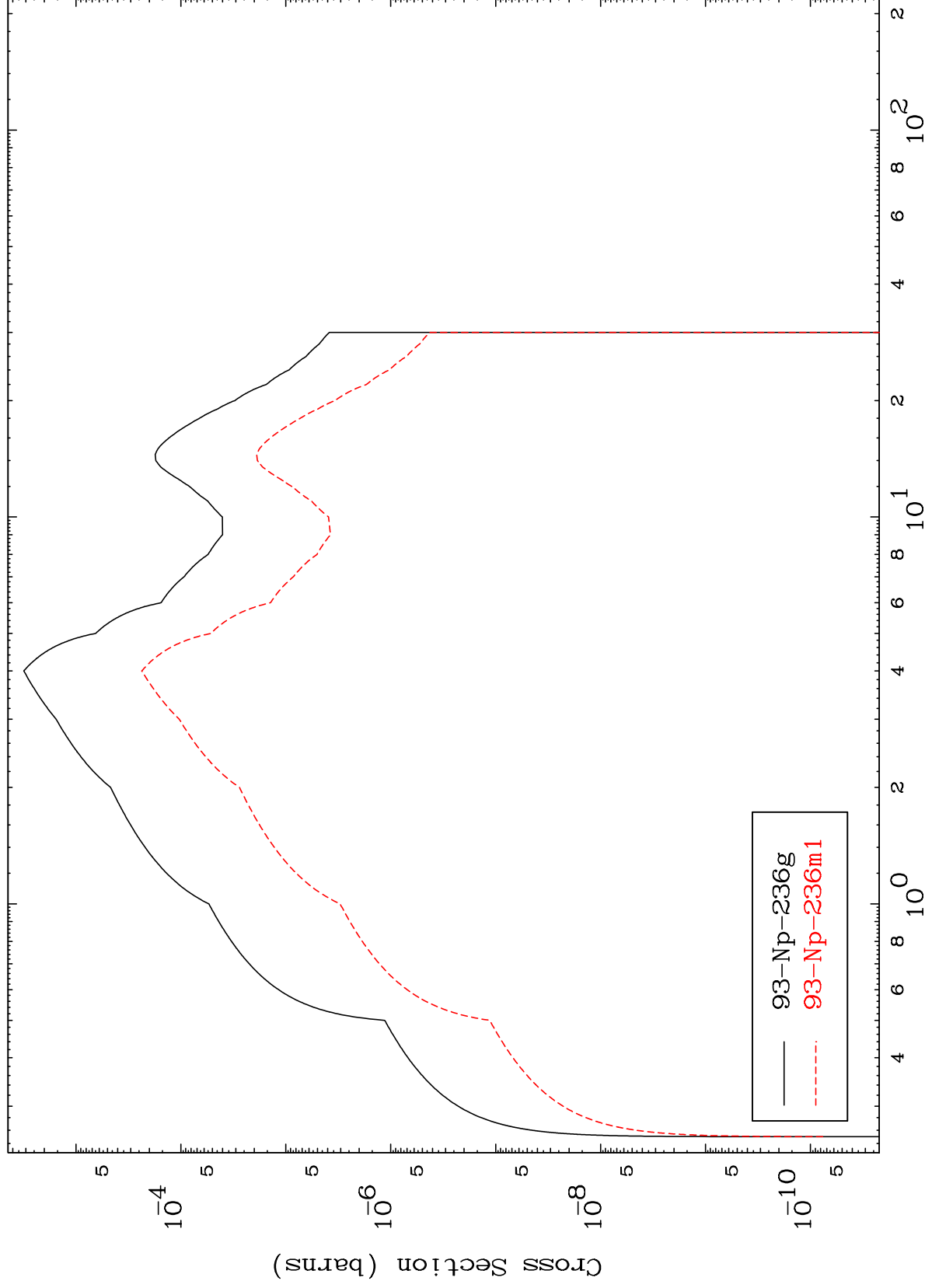
93-Np-236



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93-Np-236

(n, γ)
Radionuclide Production Cross Section



17

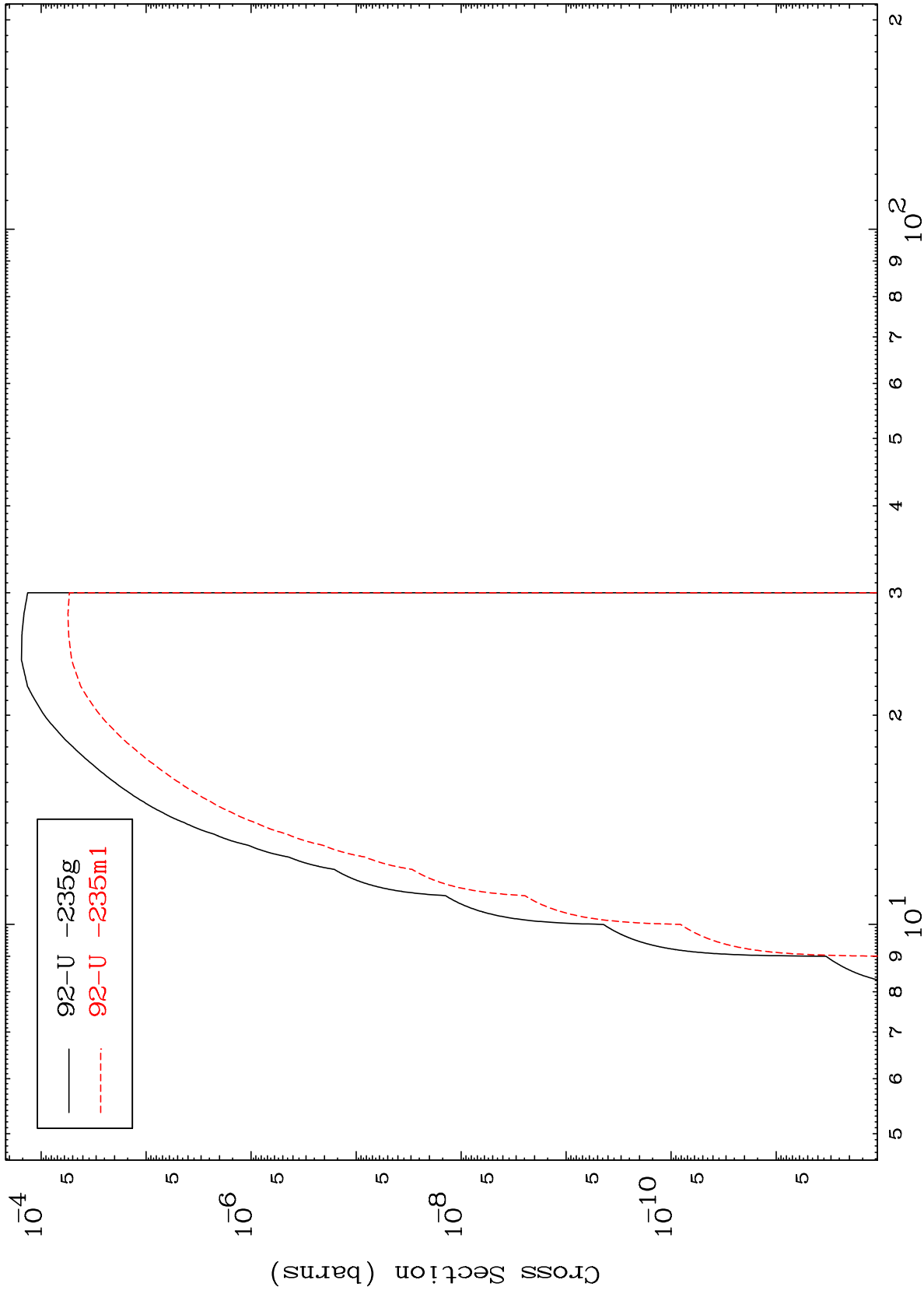
Incident Energy (MeV)

93-Np-236

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93-Np-236

(n,p)
Radionuclide Production Cross Section



18

Incident Energy (MeV)

93-Np-236