

Program EVALPLOT
(Version 2021-1)

by

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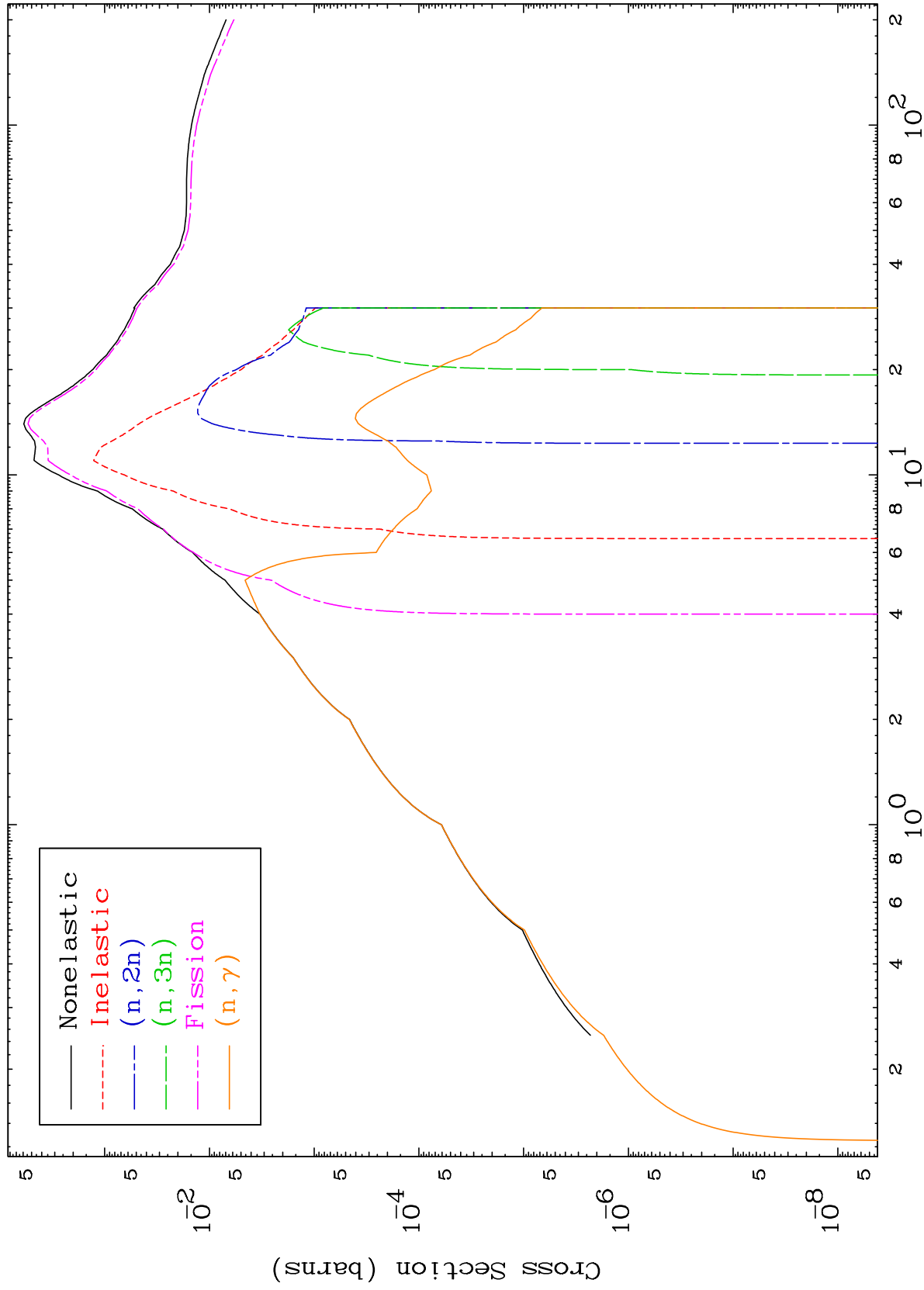
Press Mouse Button to Start

MAT 9346

Photon Major

93-Np-237

0 Kelvin Cross Sections



1

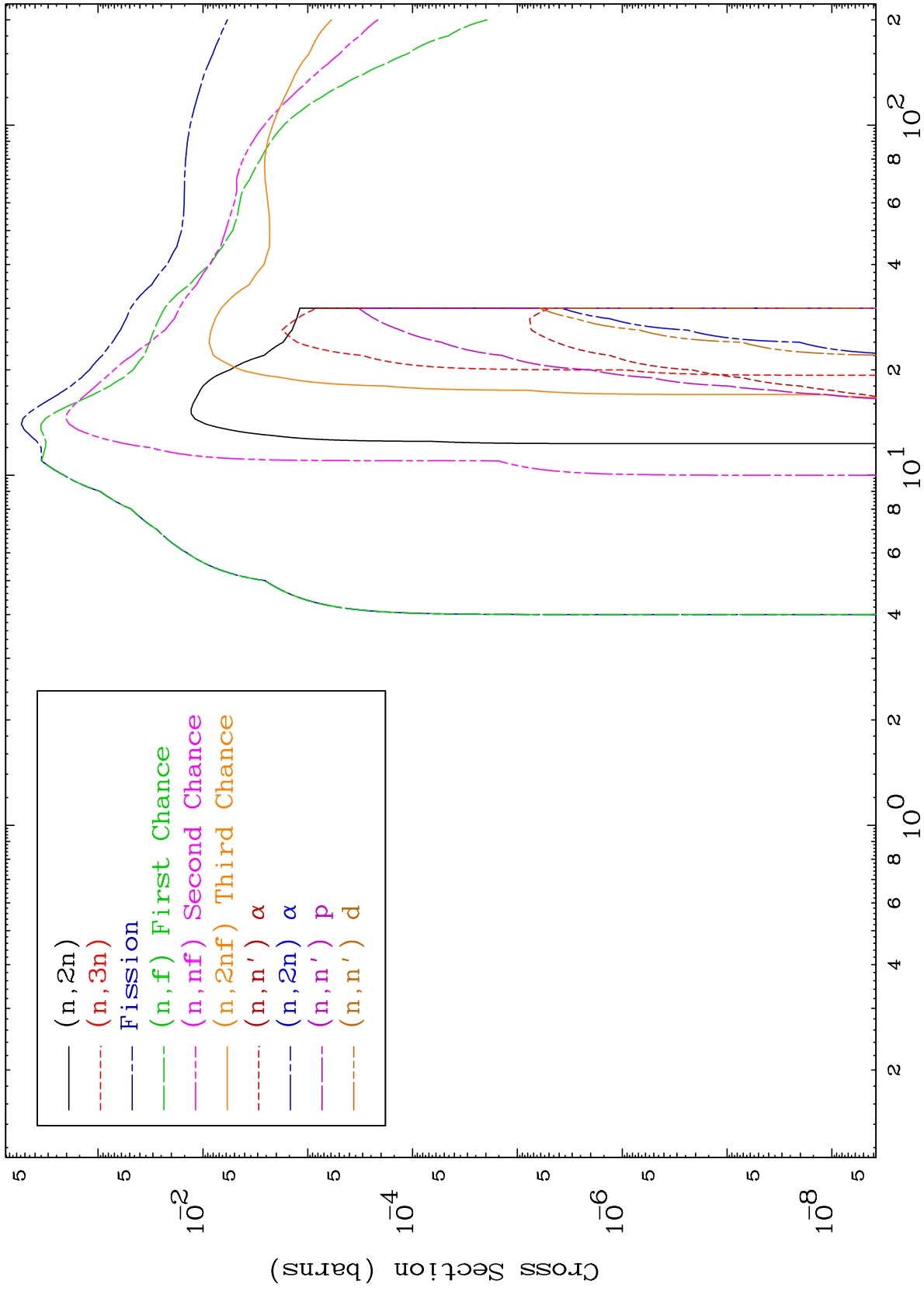
Incident Energy (MeV)

93-Np-237

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Photon Neutron Absorption
0 Kelvin Cross Sections

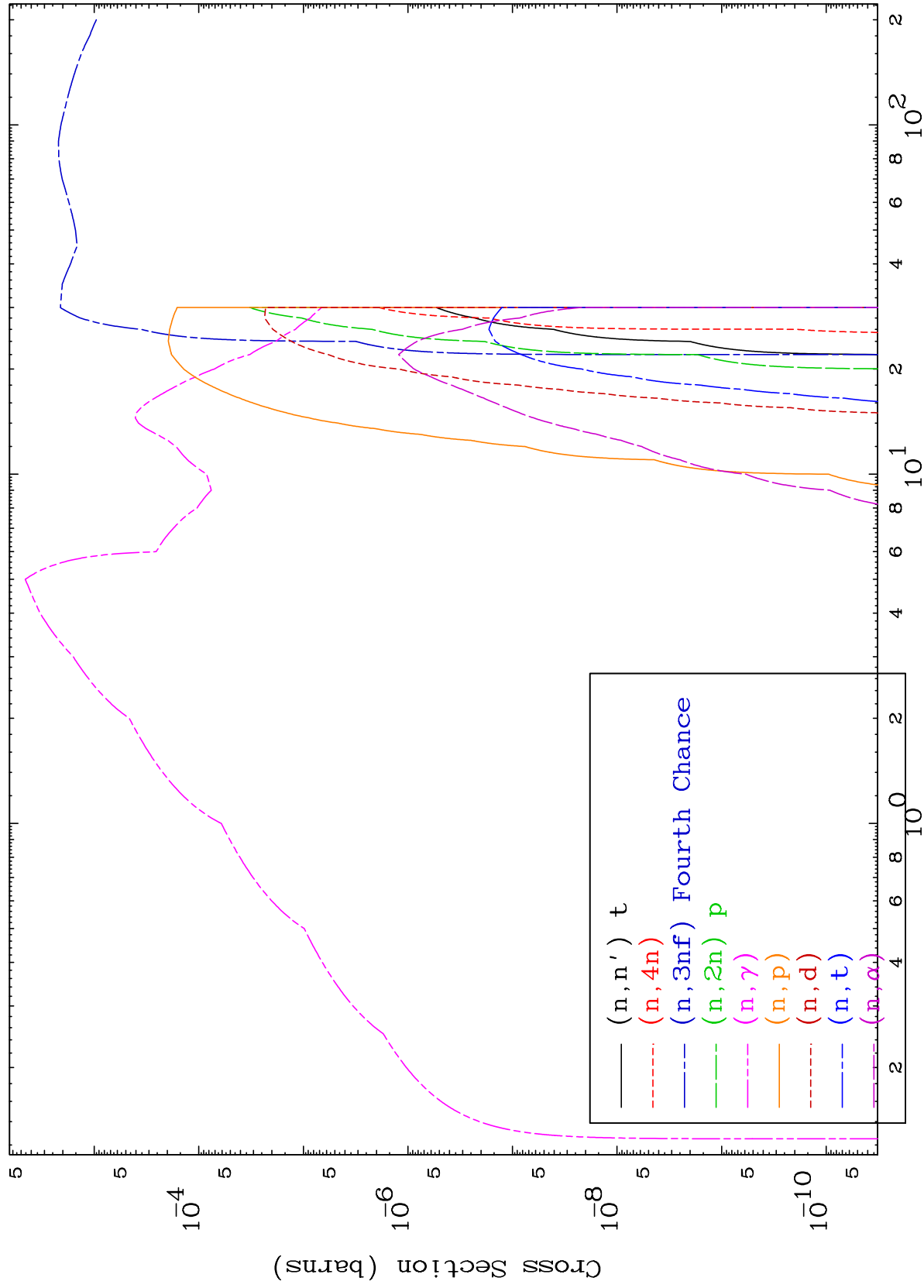
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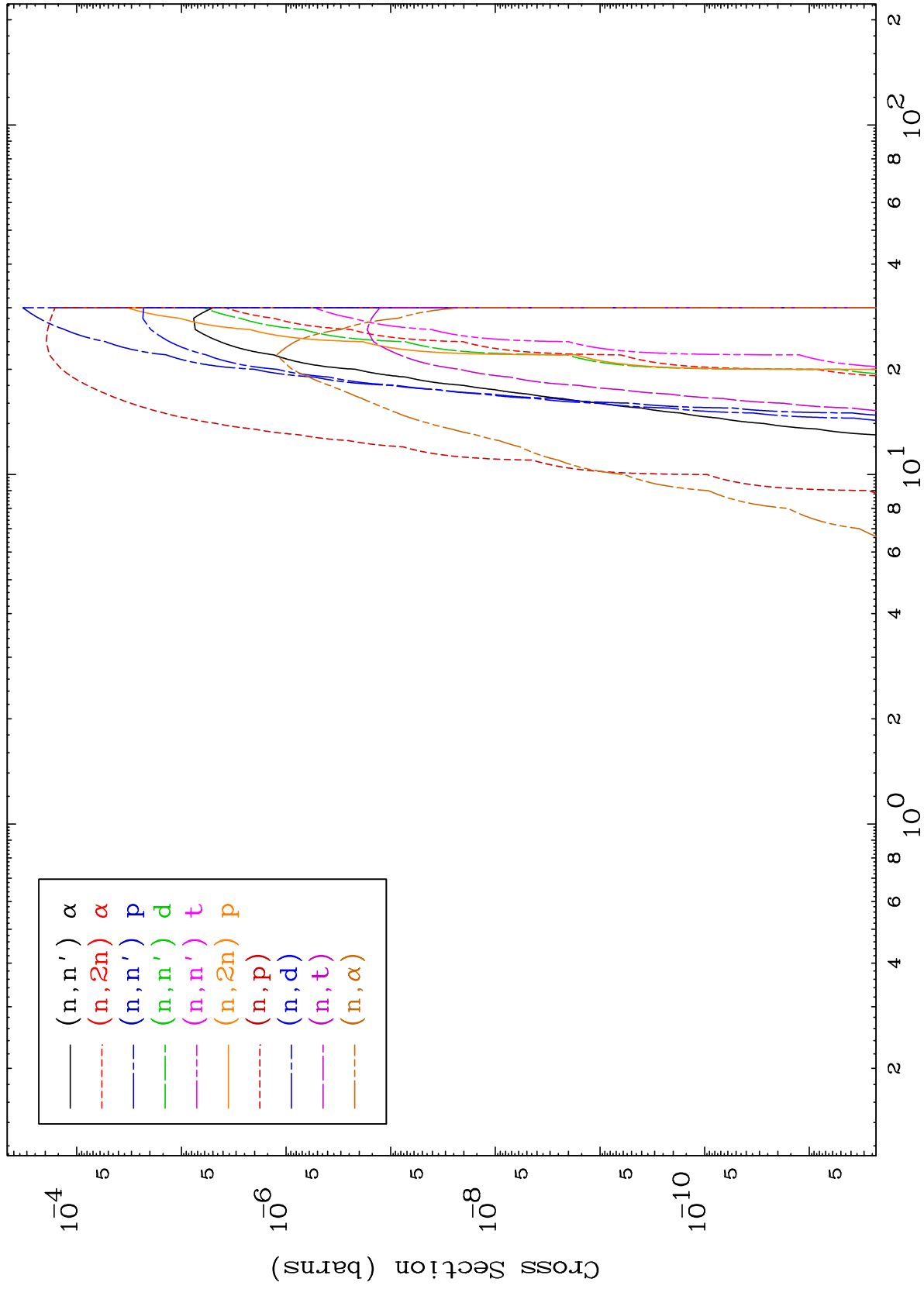


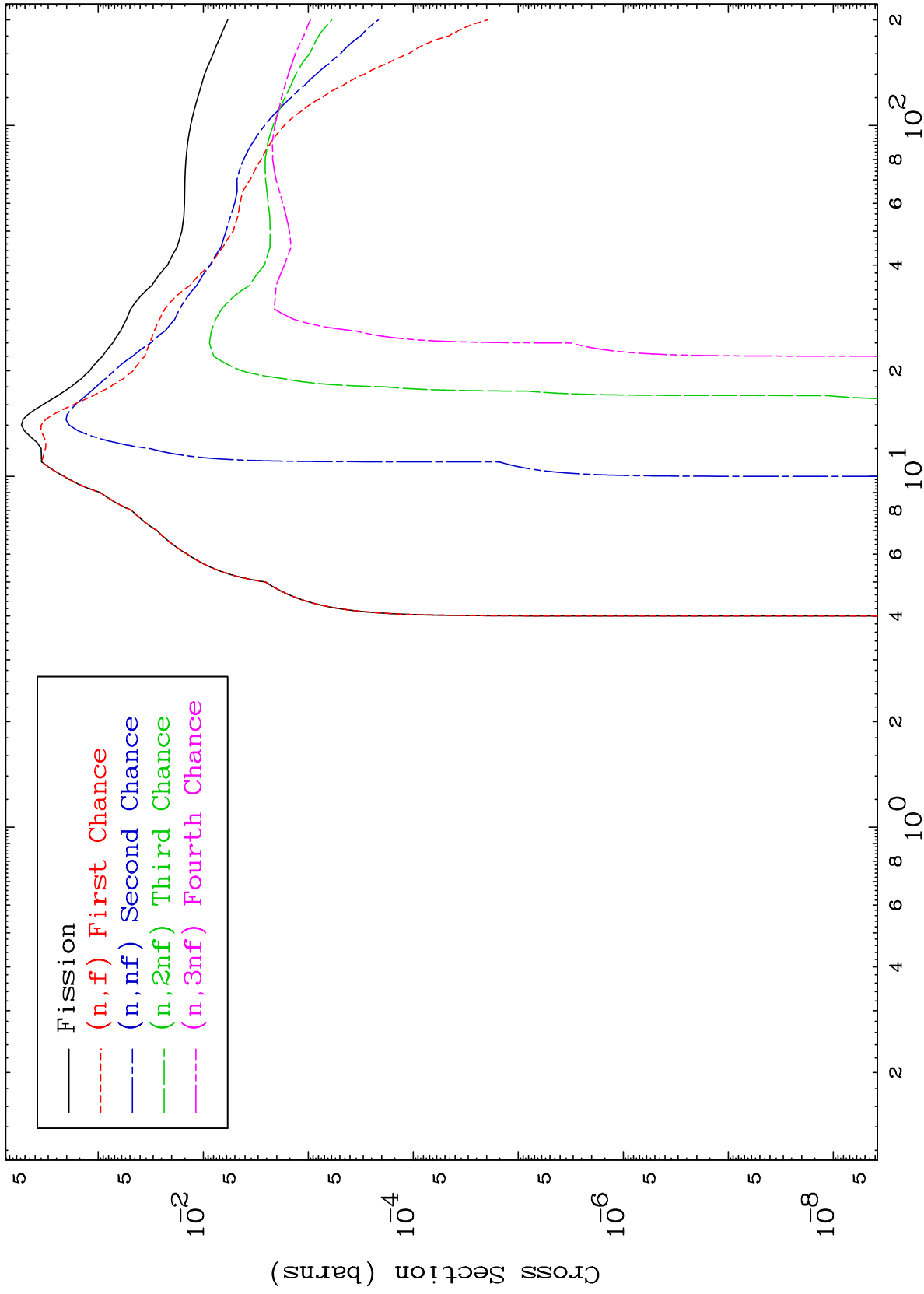
2

Incident Energy (MeV)

93-Np-237



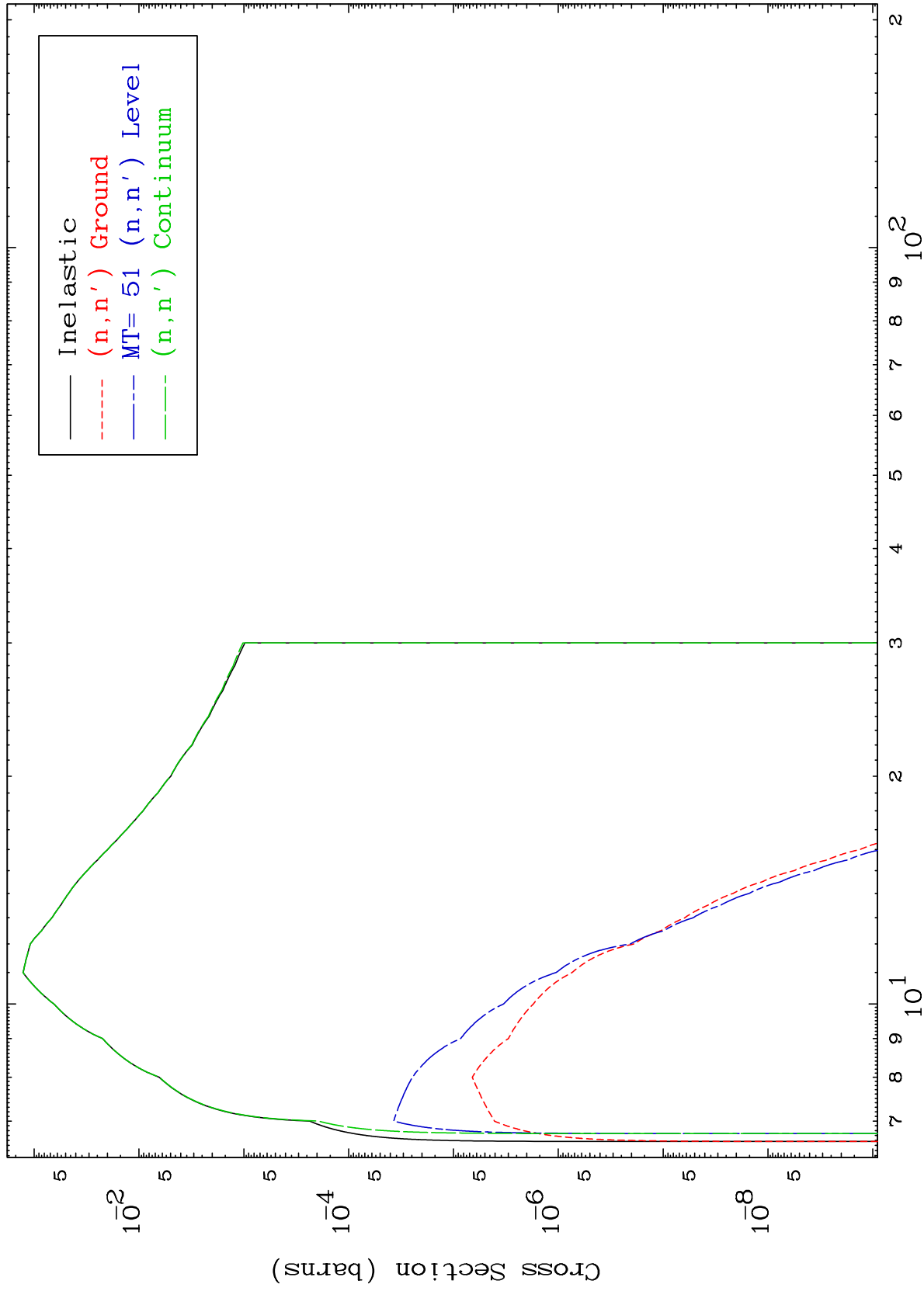




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(γ, n') Levels
0 Kelvin Cross Sections

93-Np-237



6

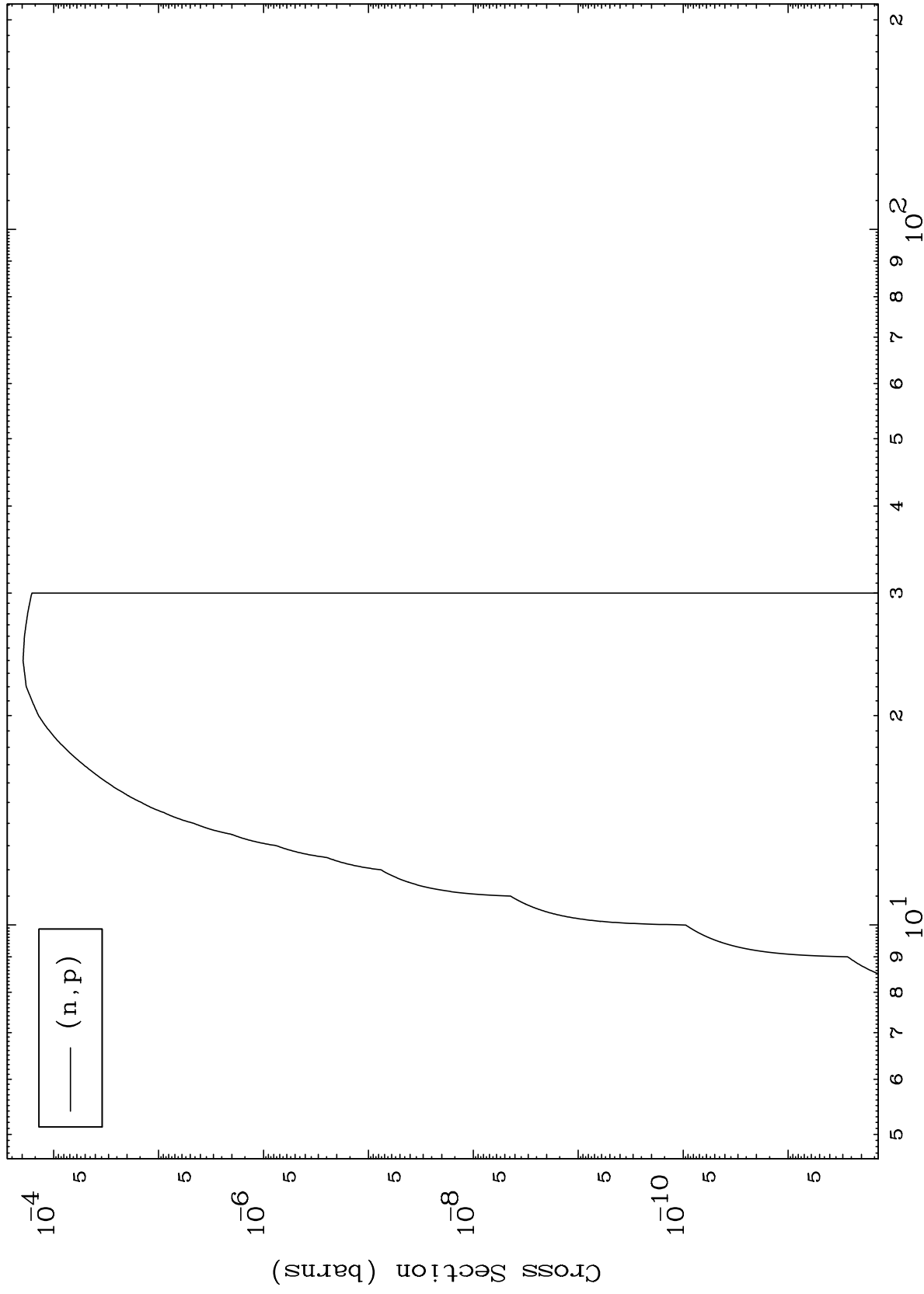
Incident Energy (MeV)

93-Np-237

MAT 9346

93-Np-237

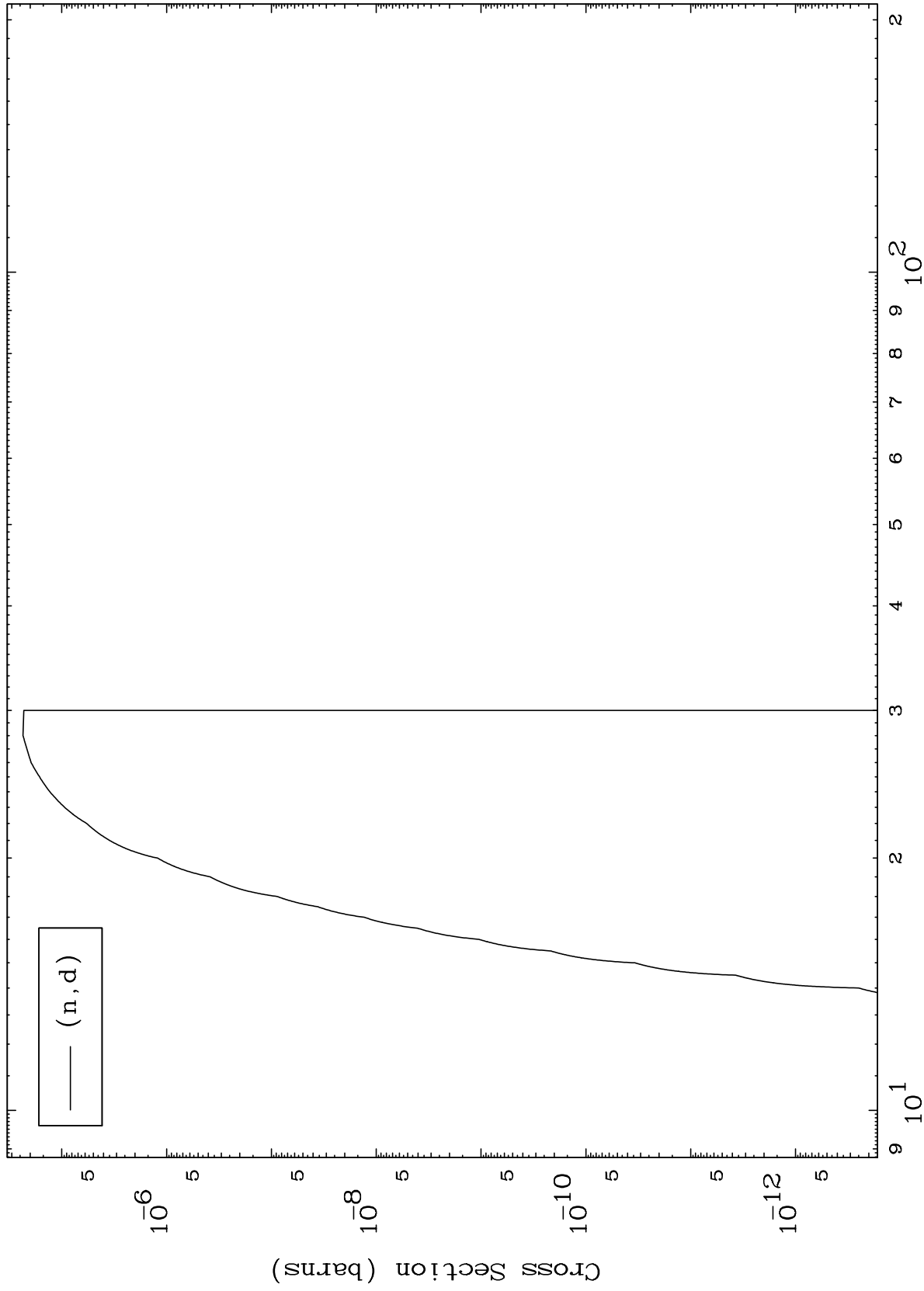
(γ ,p) Levels
0 Kelvin Cross Sections



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93-Np-237

(γ ,d) Levels
0 Kelvin Cross Sections



8

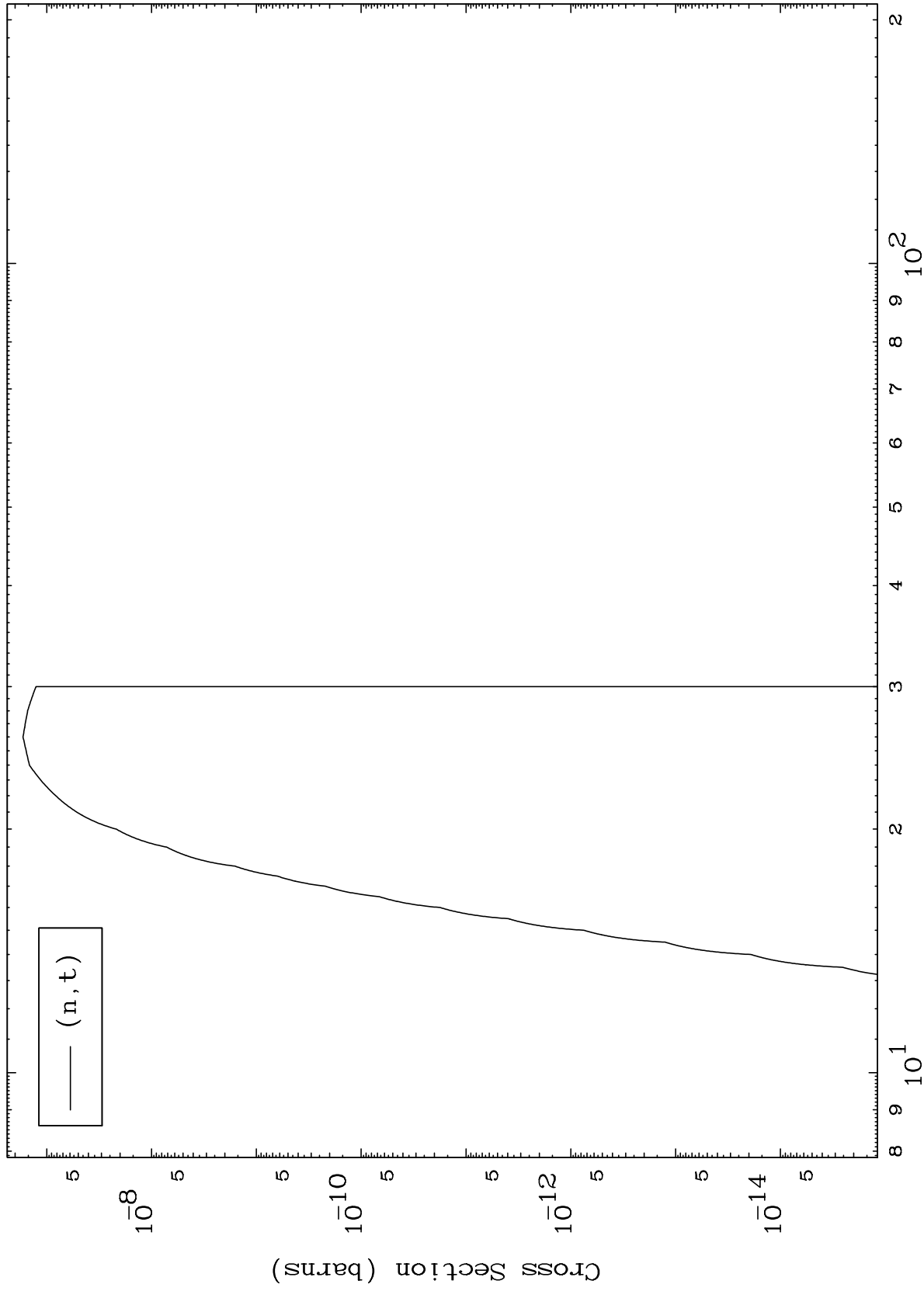
Incident Energy (MeV)

93-Np-237

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(γ, t) Levels
0 Kelvin Cross Sections

93-Np-237



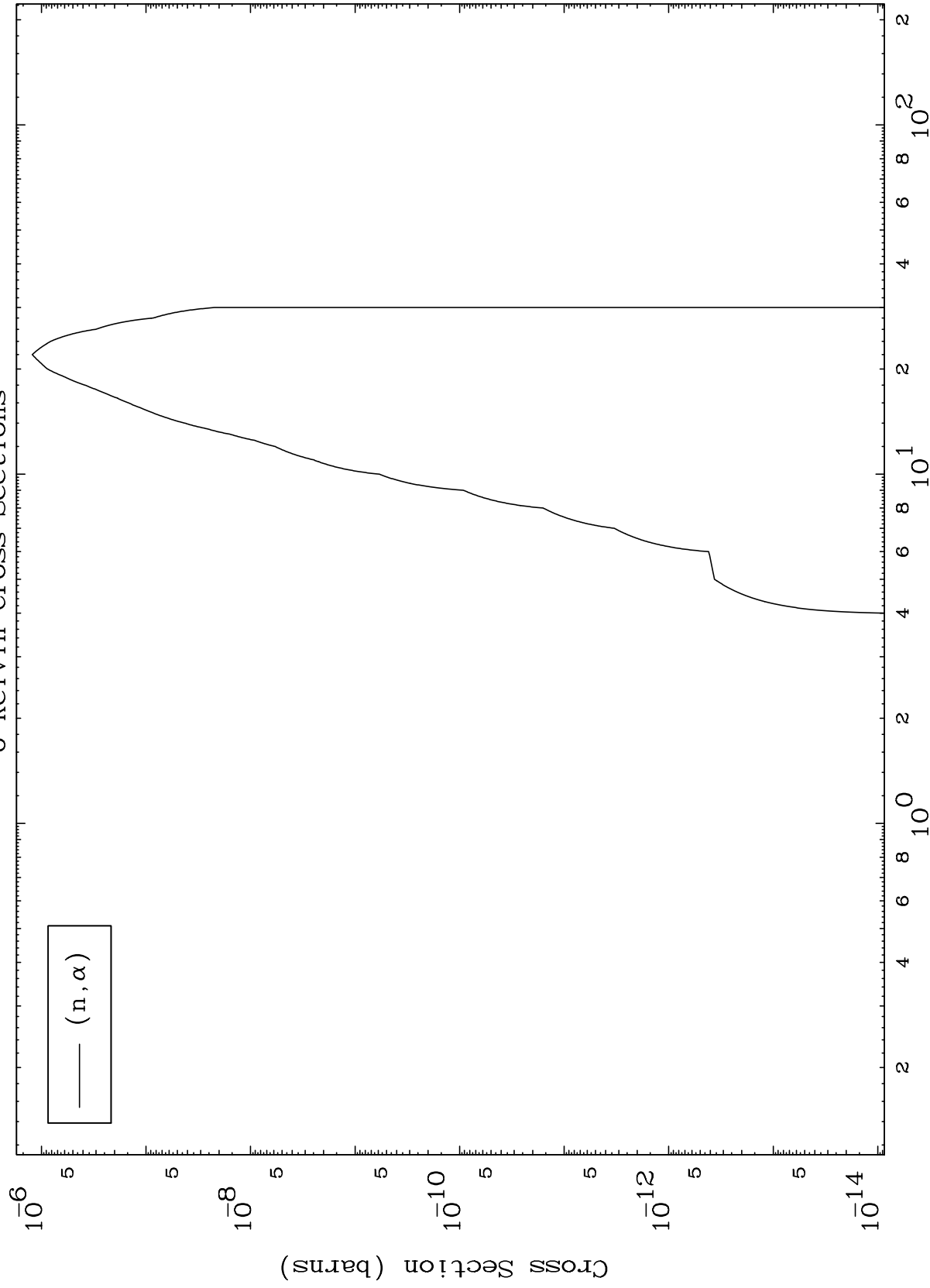
9

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(γ, α) Levels

93-Np-237

0 Kelvin Cross Sections



(n, α)

10

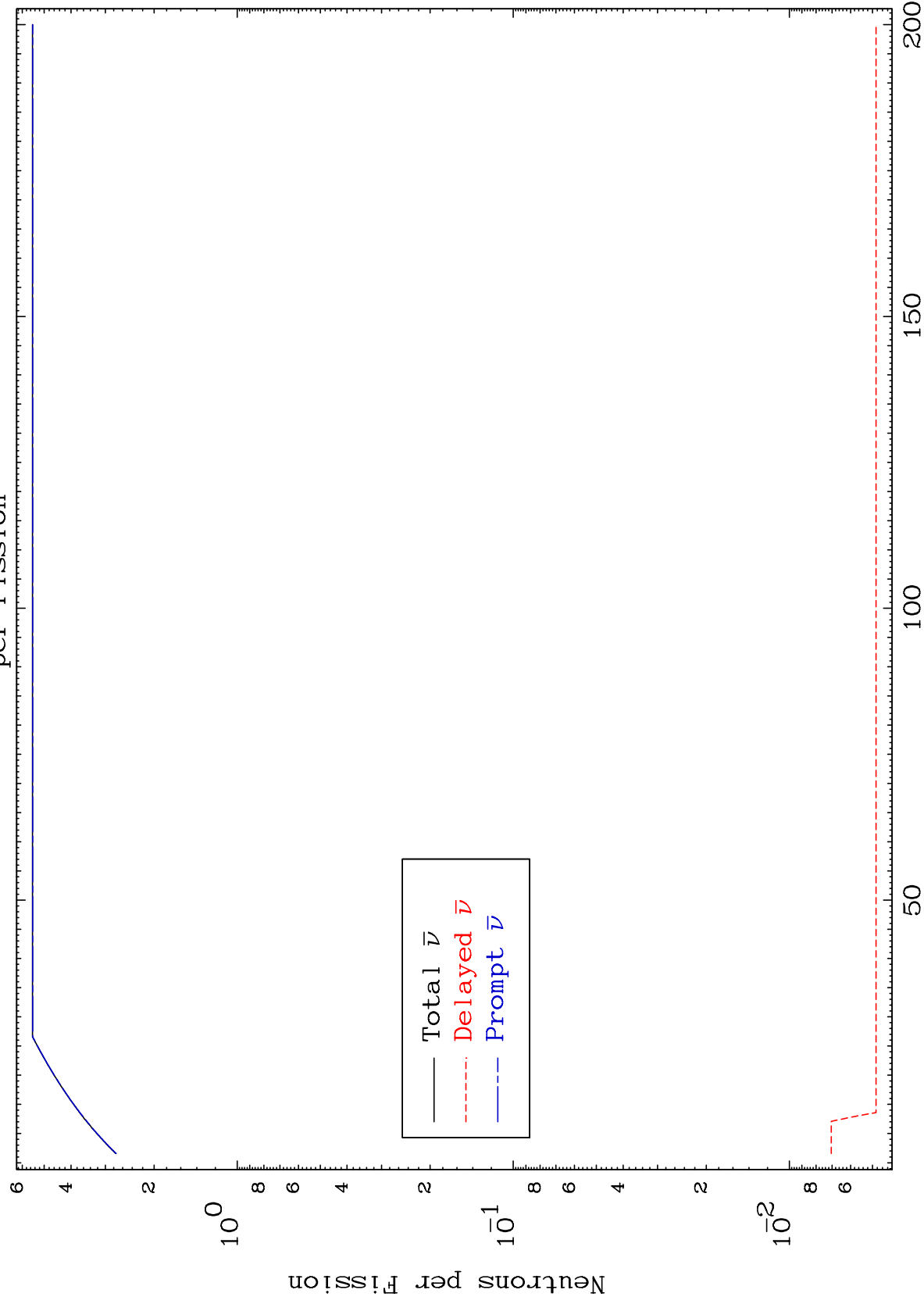
Incident Energy (MeV)

93-Np-237

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Photon Neutrons
per Fission

93-Np-237

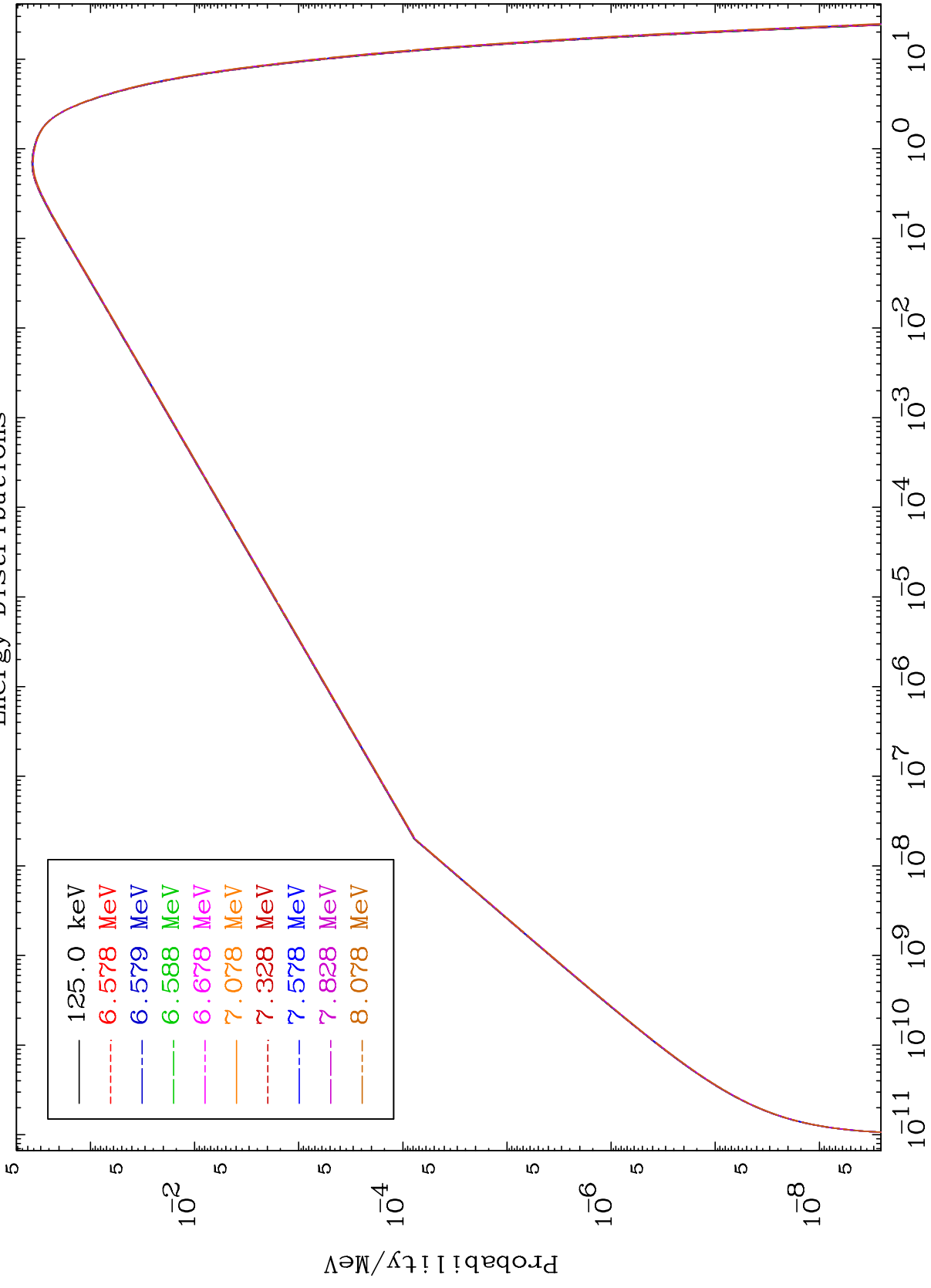


11

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Fission
Energy Distributions

93-Np-237



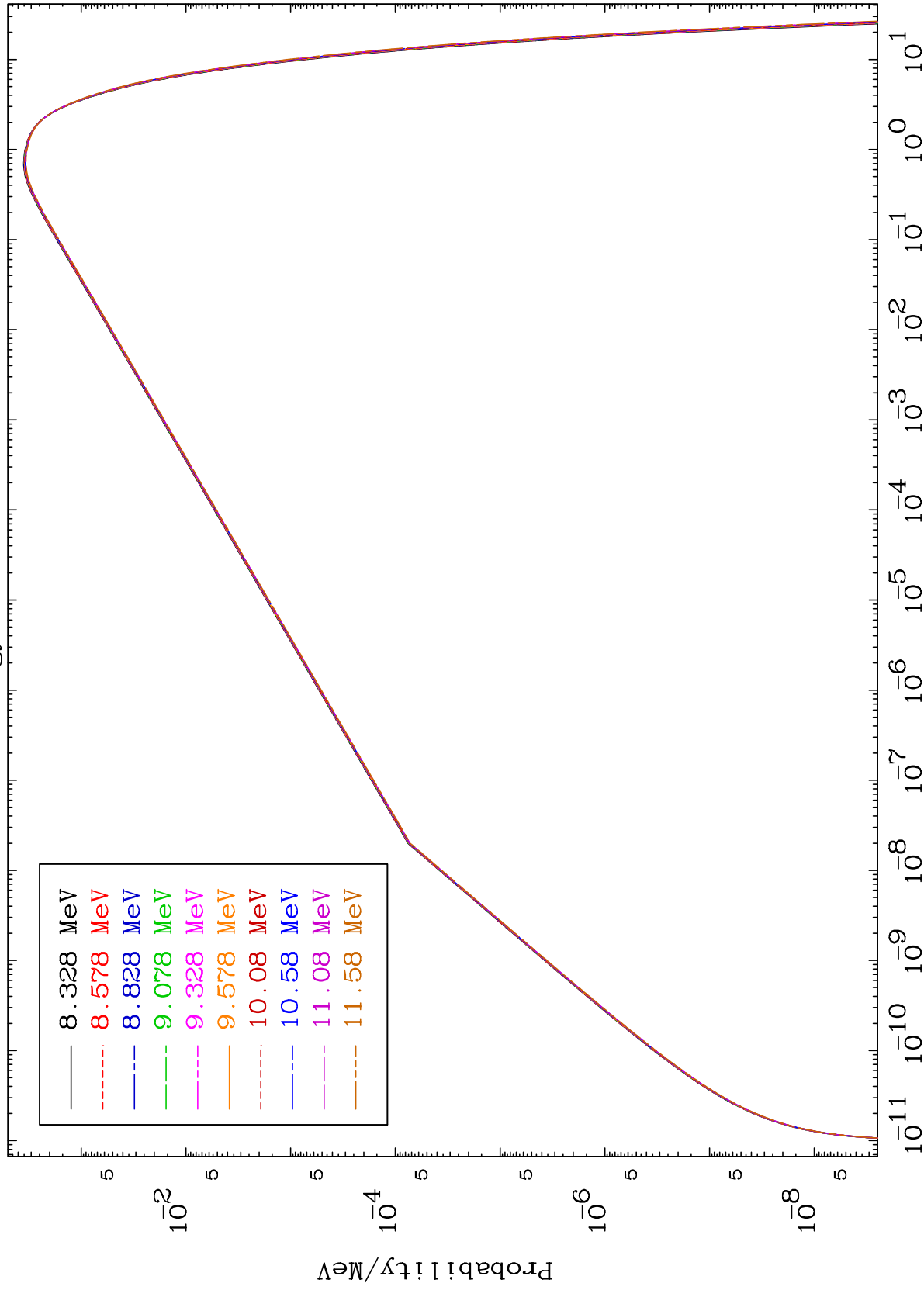
93-Np-237

Secondary Energy (MeV)

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Fission
Energy Distributions

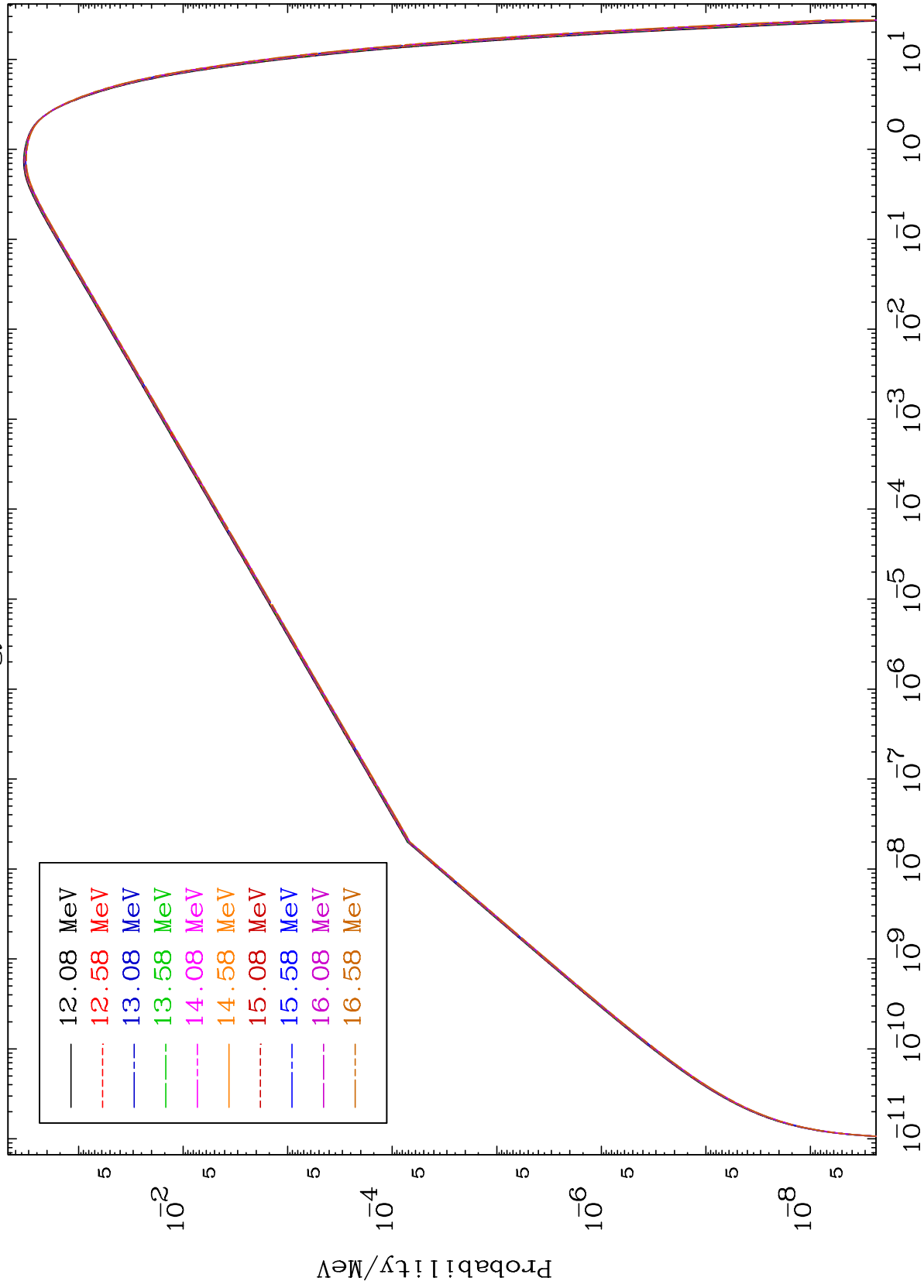
93-Np-237



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Fission
Energy Distributions

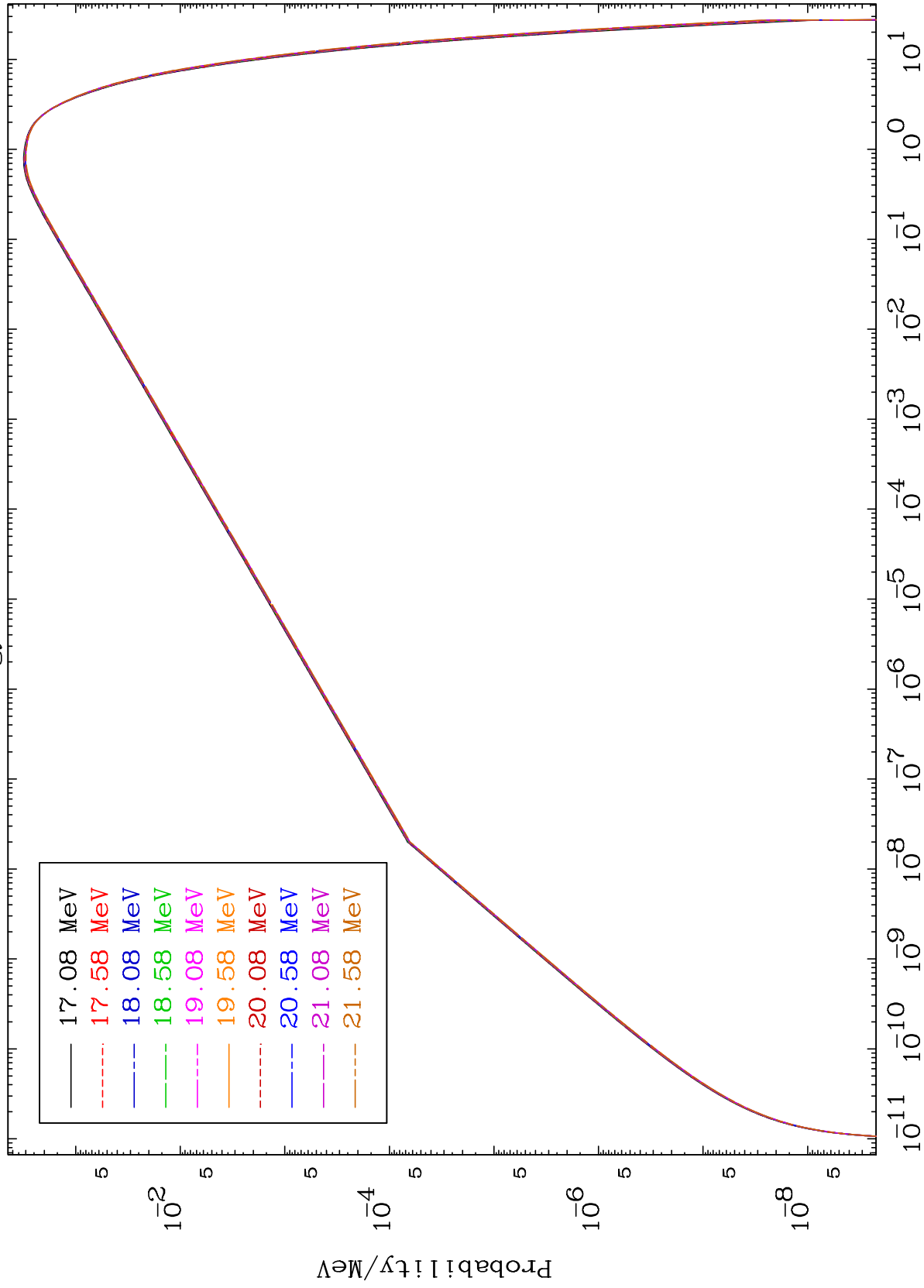
93-Np-237



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Fission
Energy Distributions

93-Np-237



15

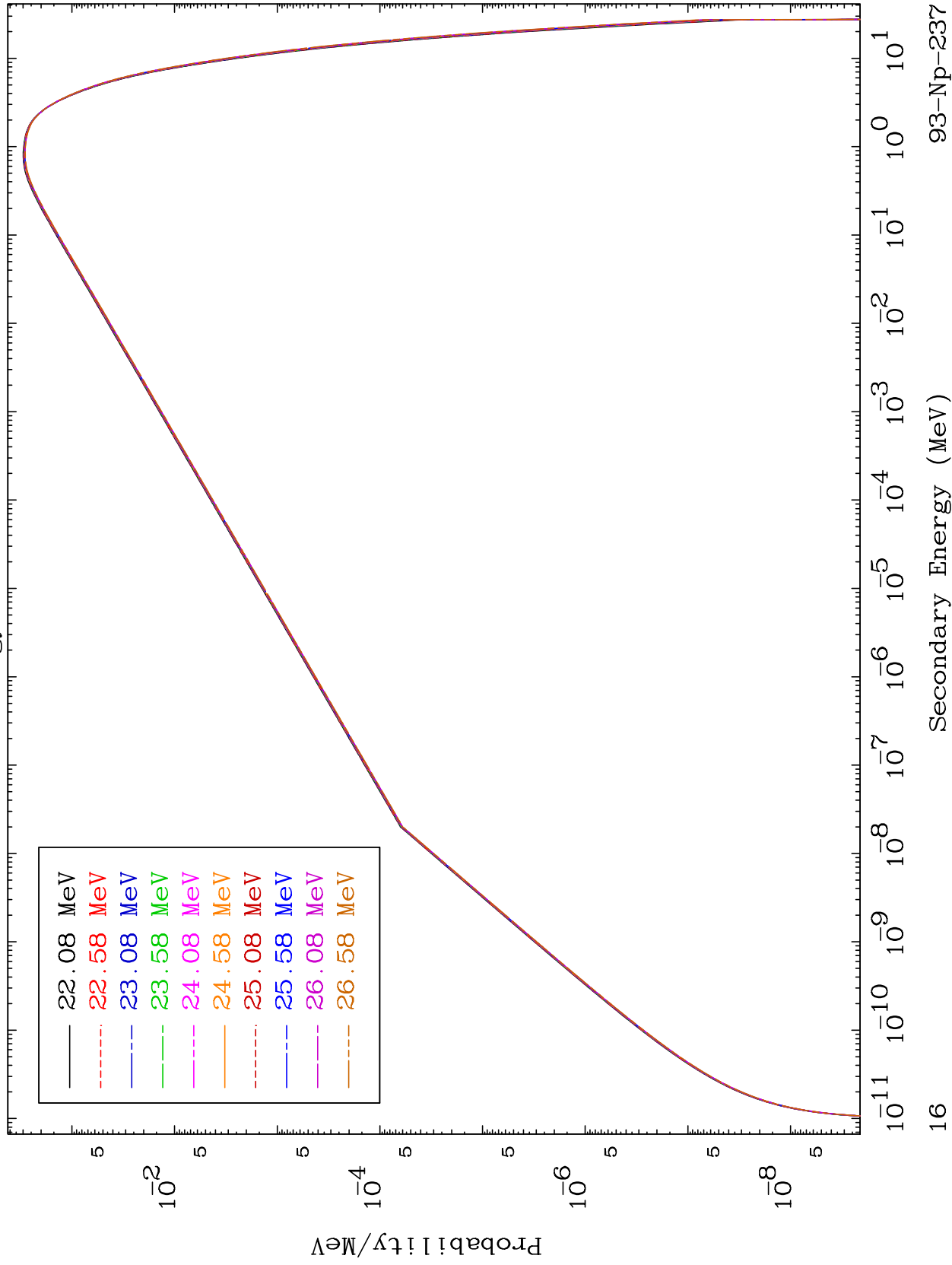
Secondary Energy (MeV)

93-Np-237

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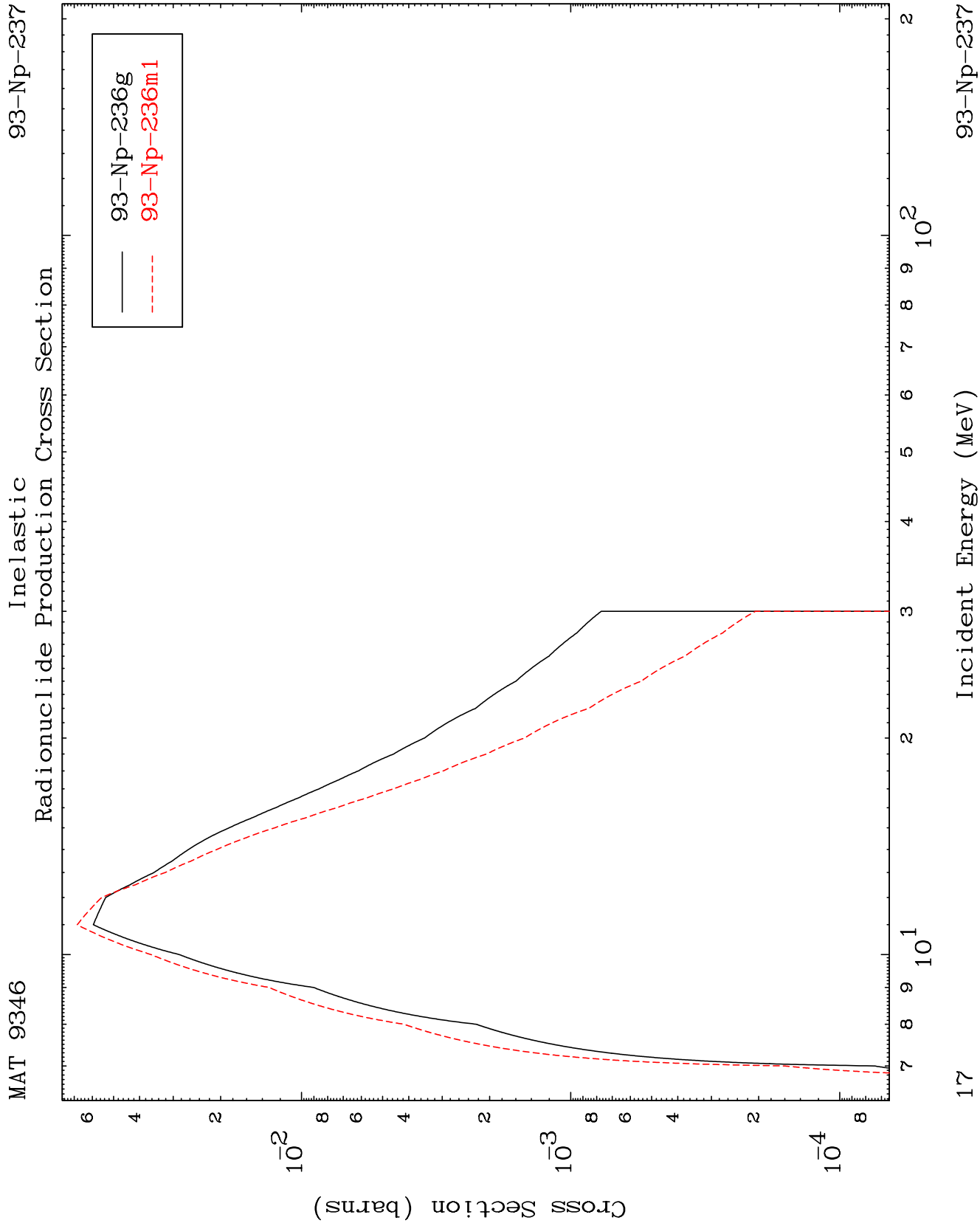
Fission
Energy Distributions

93-Np-237



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93-Np-237

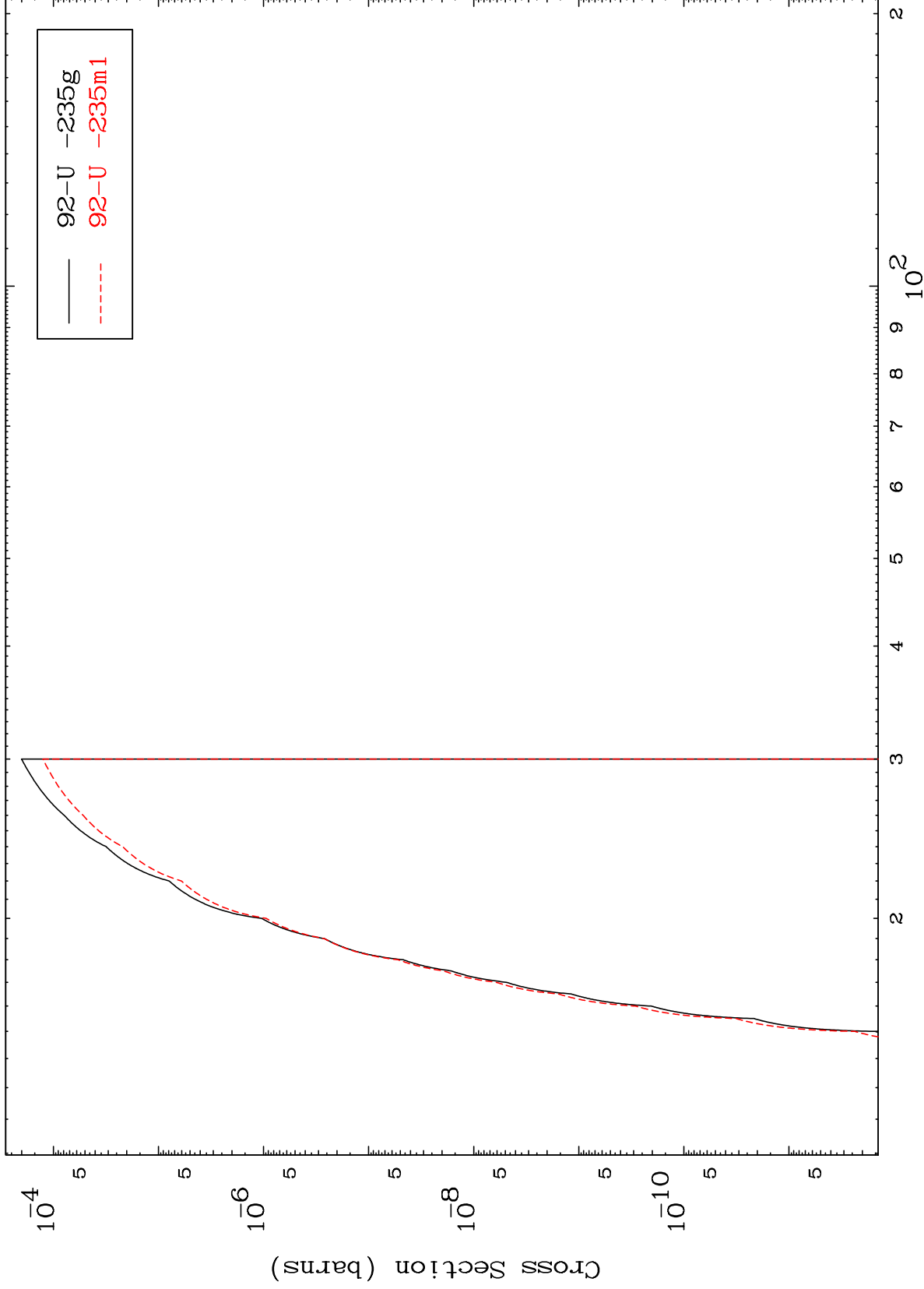


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(n,n') p

93-Np-237

Radionuclide Production Cross Section



18

Incident Energy (MeV)

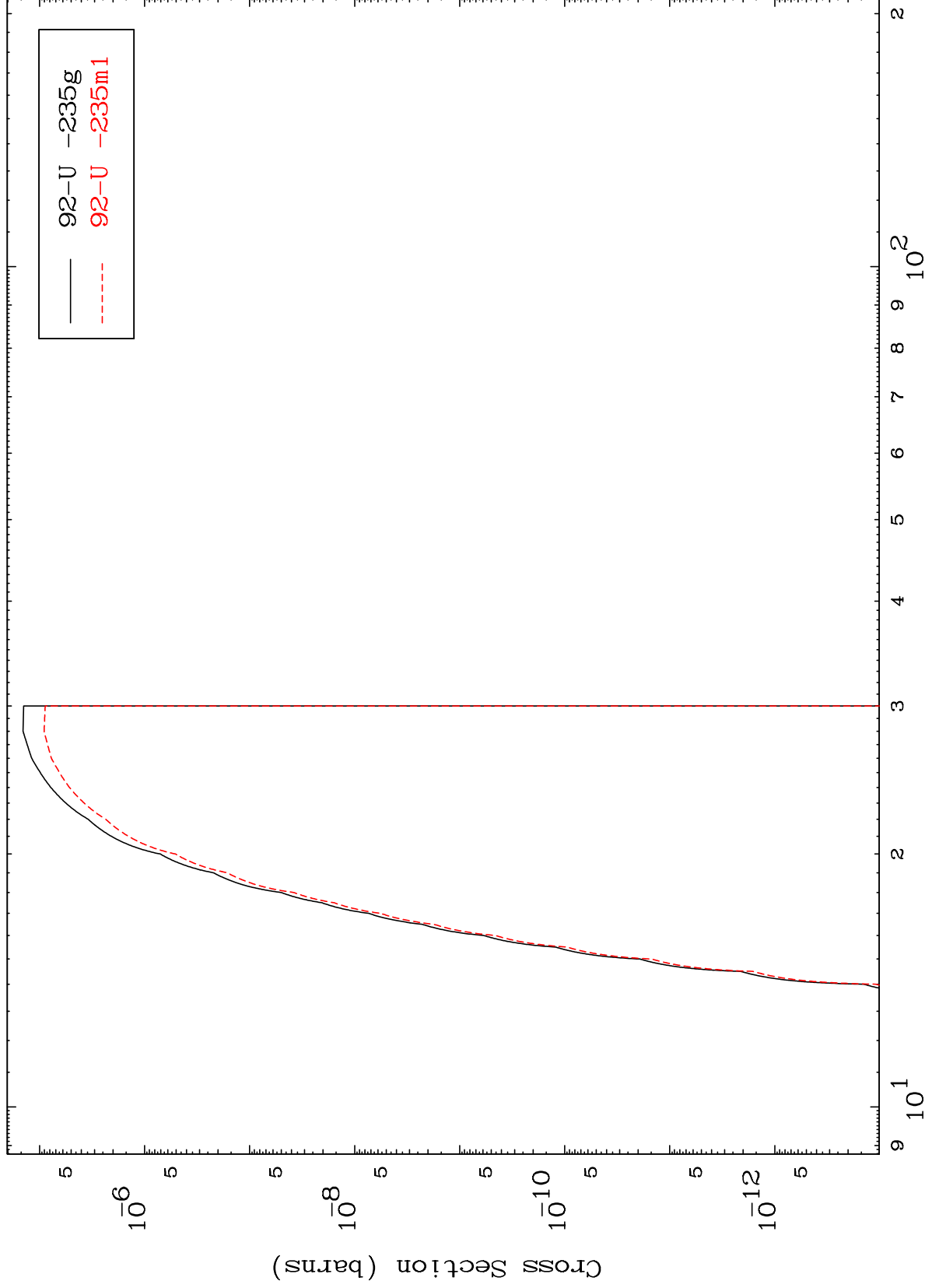
93-Np-237

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(n,d)

93-Np-237

Radionuclide Production Cross Section



19

Incident Energy (MeV)

93-Np-237