

Program EVALPLOT  
(Version 2021-1)

by

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(Present Contact Information)

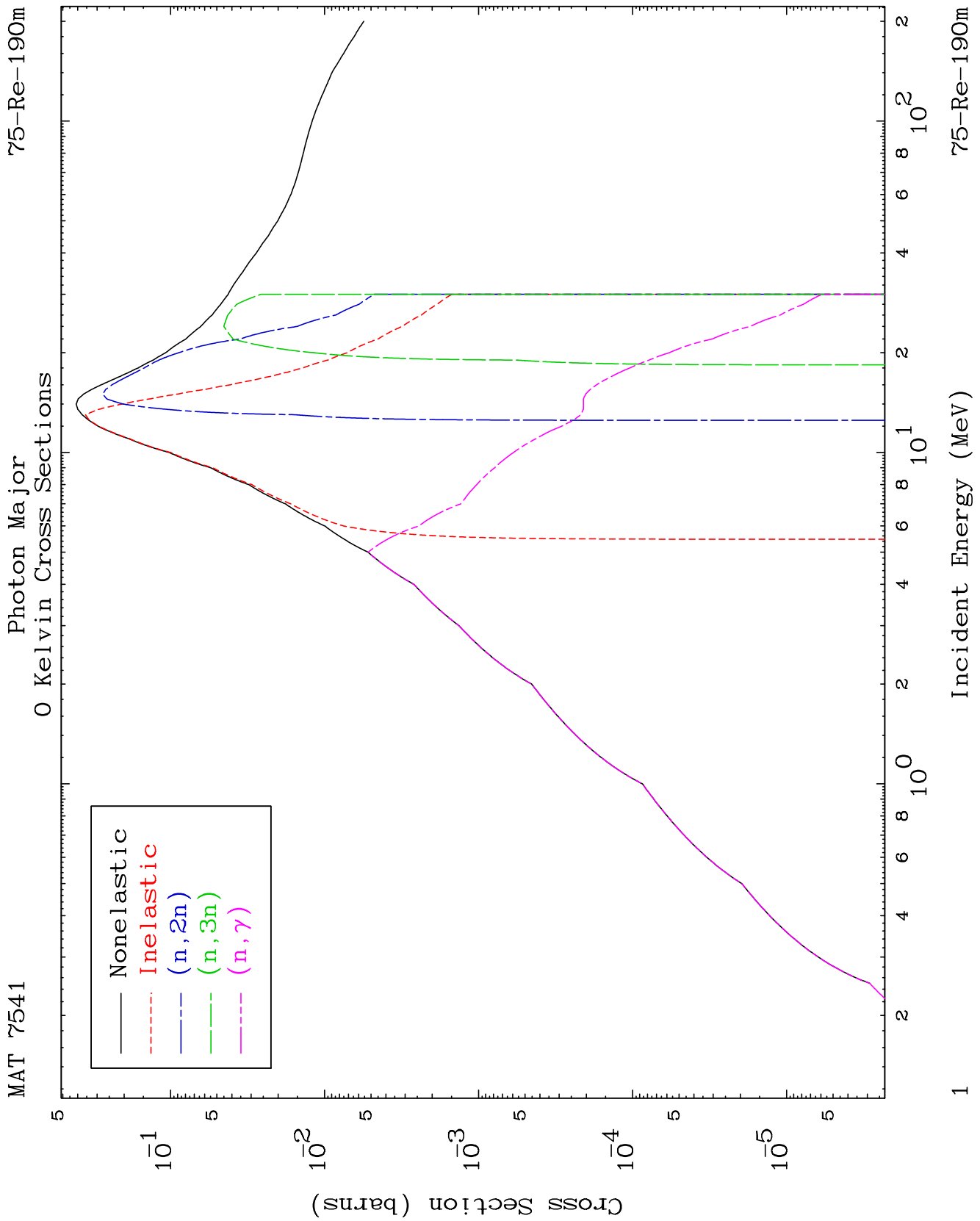
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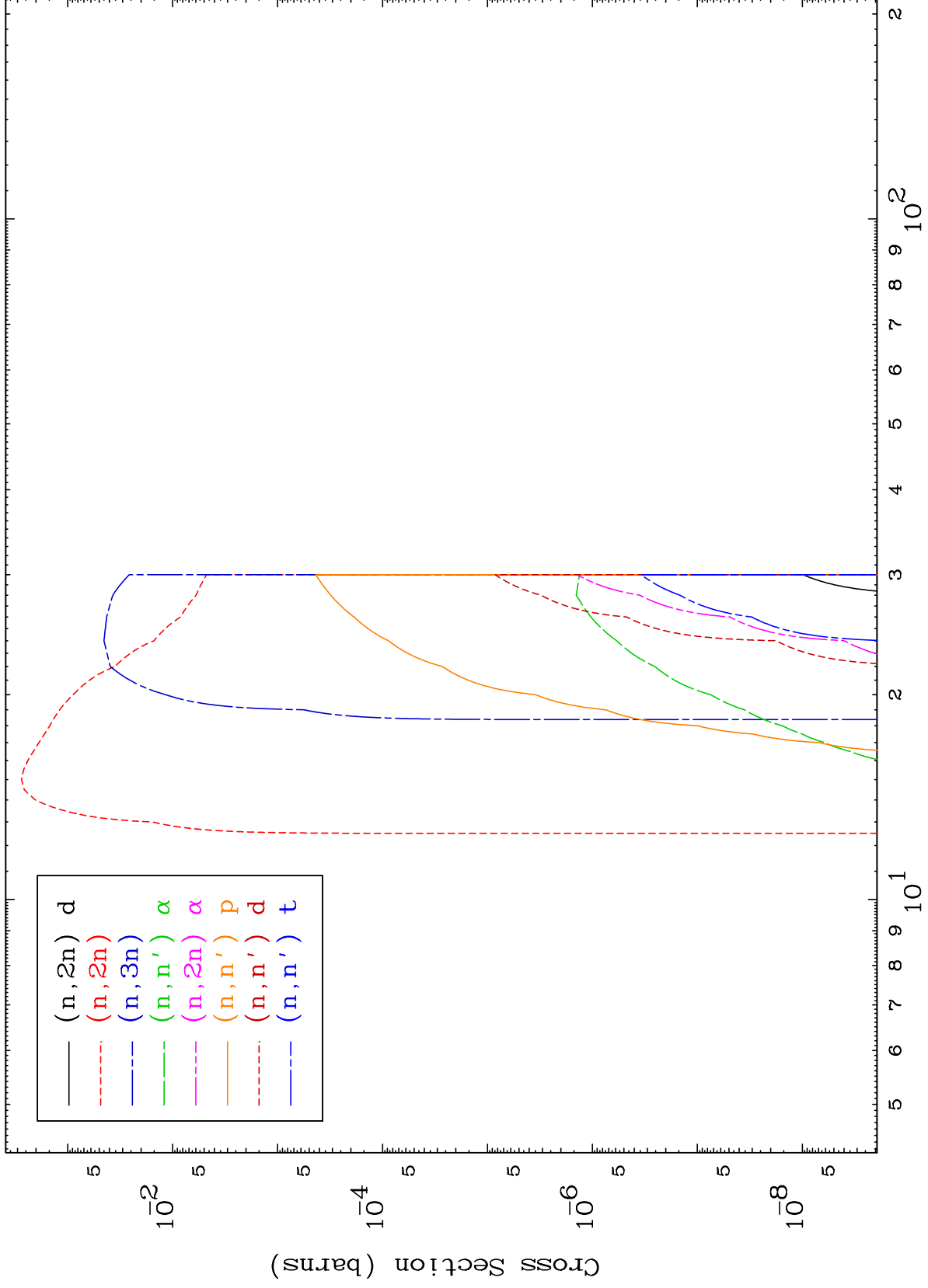
Press Mouse Button to Start



MAT 7541

Photon Neutron Absorption  
0 Kelvin Cross Sections

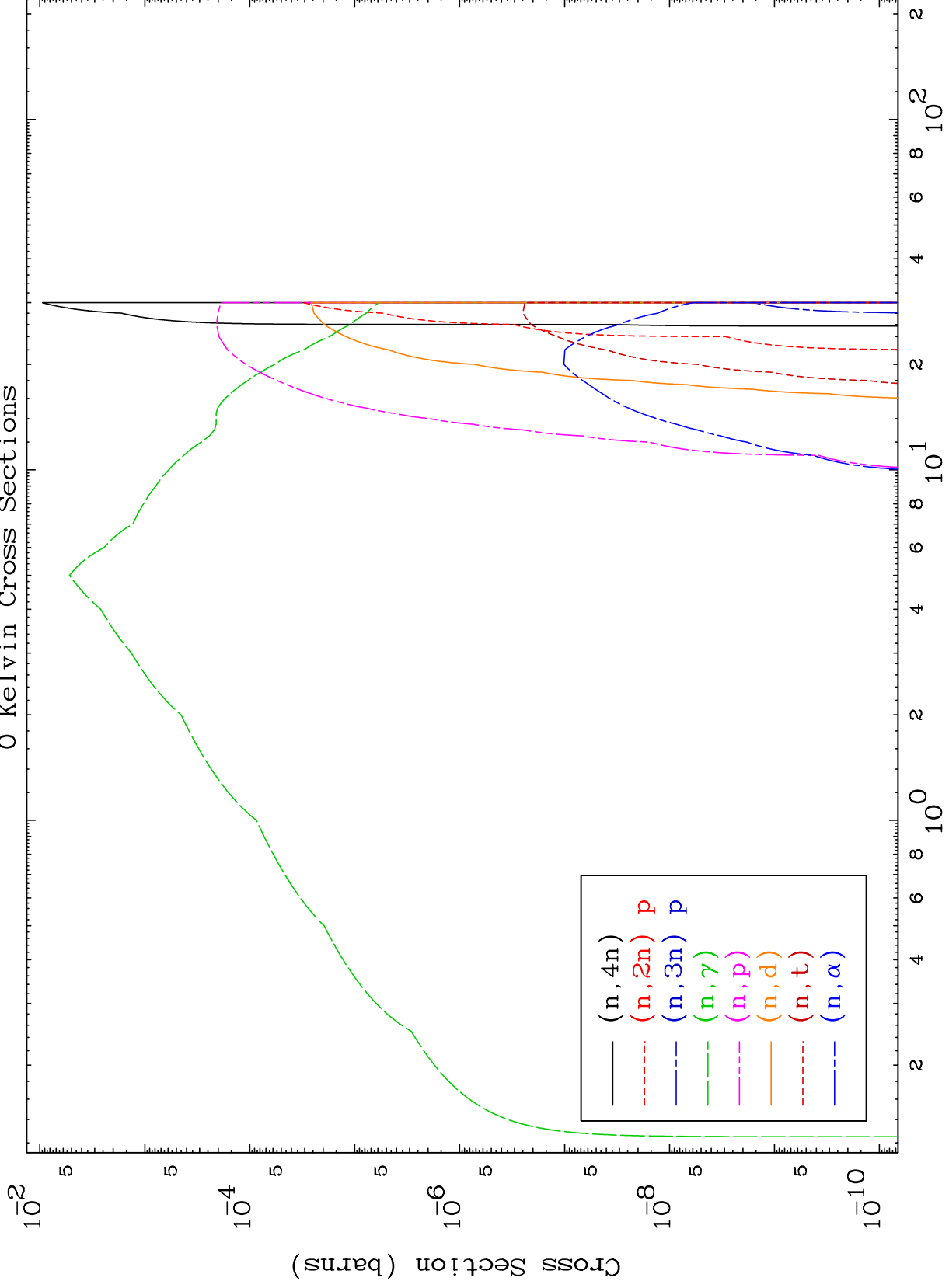
75-Re-190m



MAT 7541

Photon Neutron Absorption  
0 Kelvin Cross Sections

75-Re-190m



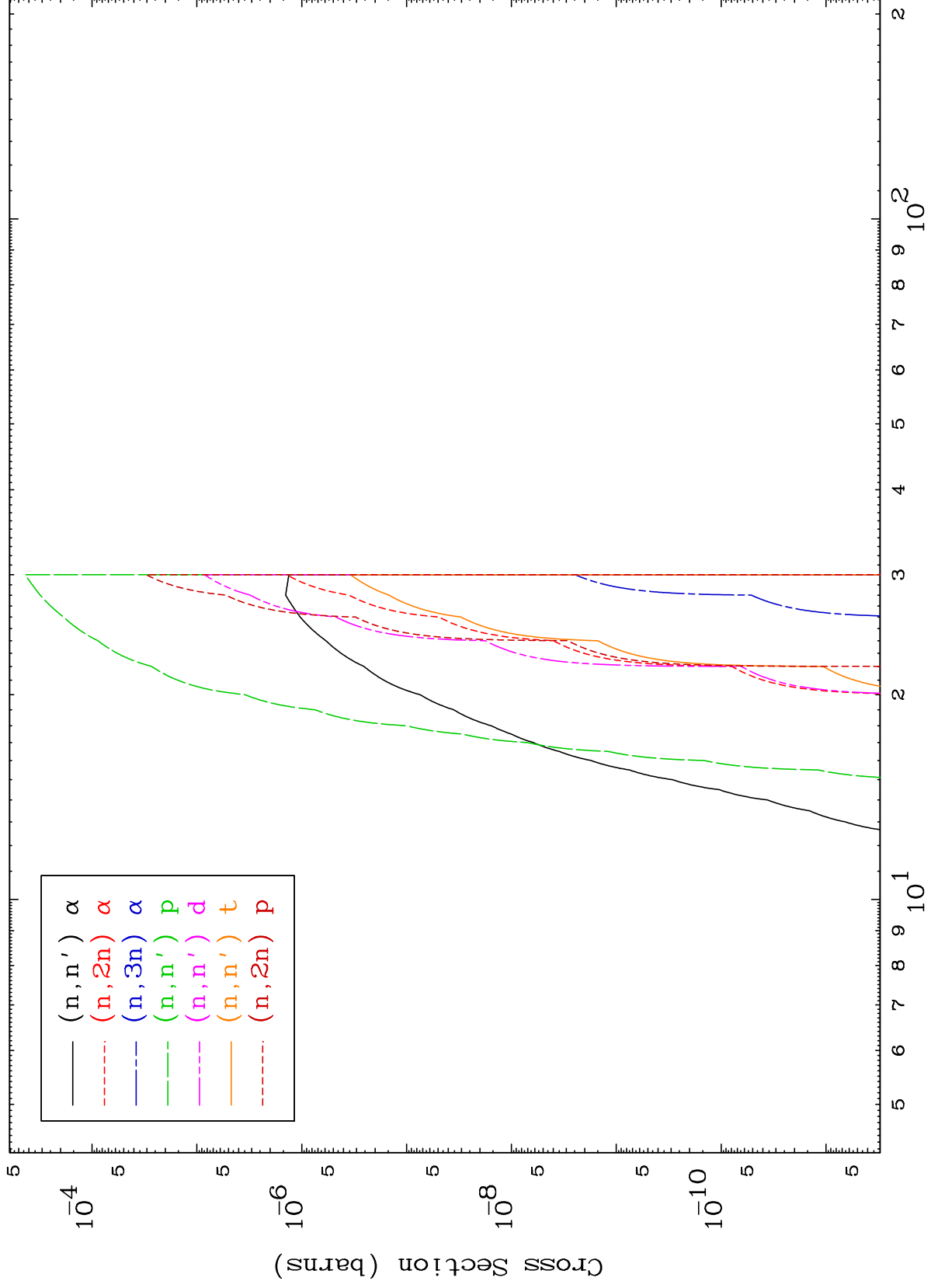
75-Re-190m

Incident Energy (MeV)

MAT 7541

Photon Charged Particle  
0 Kelvin Cross Sections

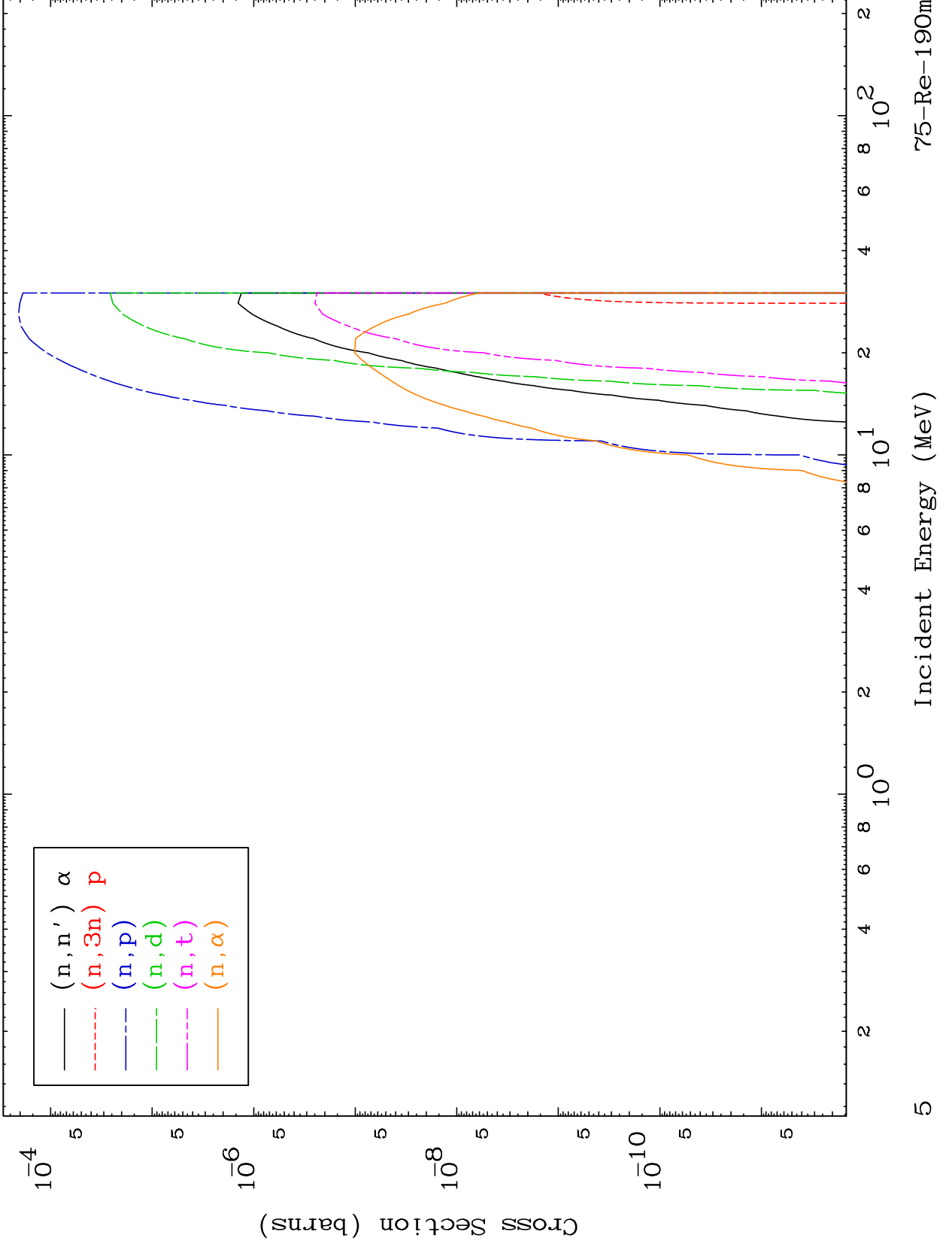
75-Re-190m



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Photon Charged Particle  
0 Kelvin Cross Sections

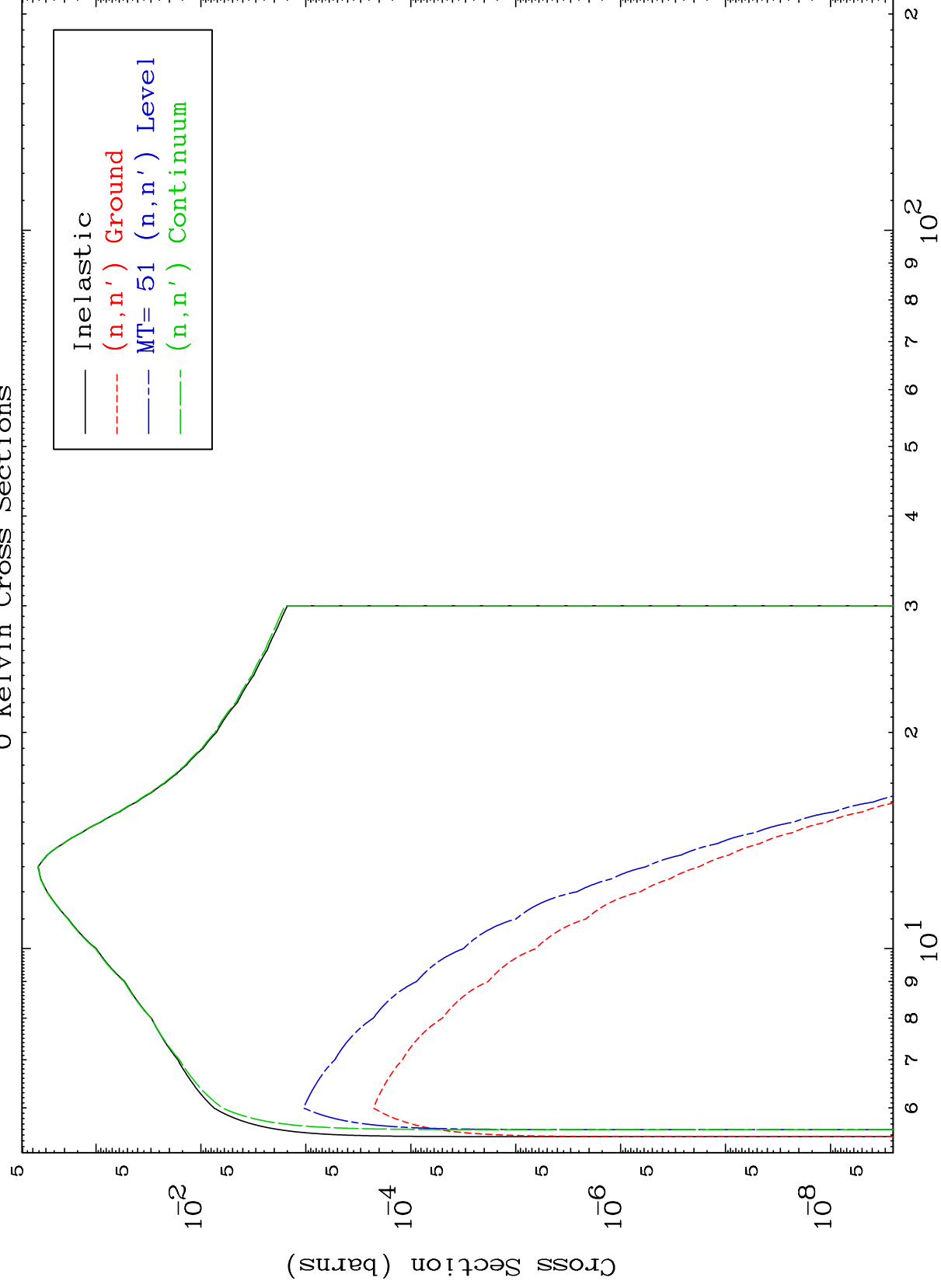
75-Re-190m



MAT 7541

75-Re-190m

( $\gamma, n'$ ) Levels  
0 Kelvin Cross Sections



75-Re-190m

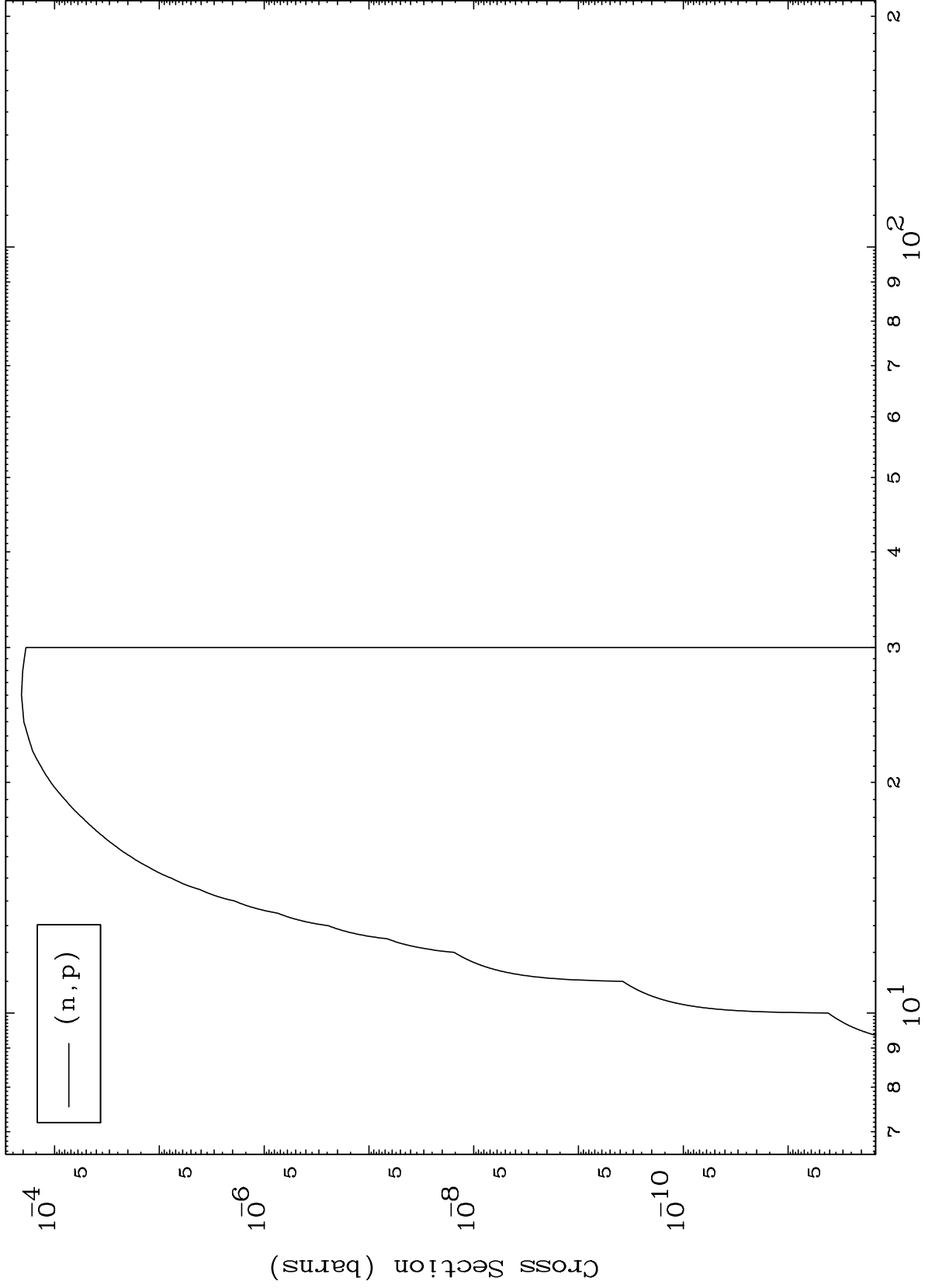
Incident Energy (MeV)

6

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( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

<sup>75</sup>Re-190m



7

Incident Energy (MeV)

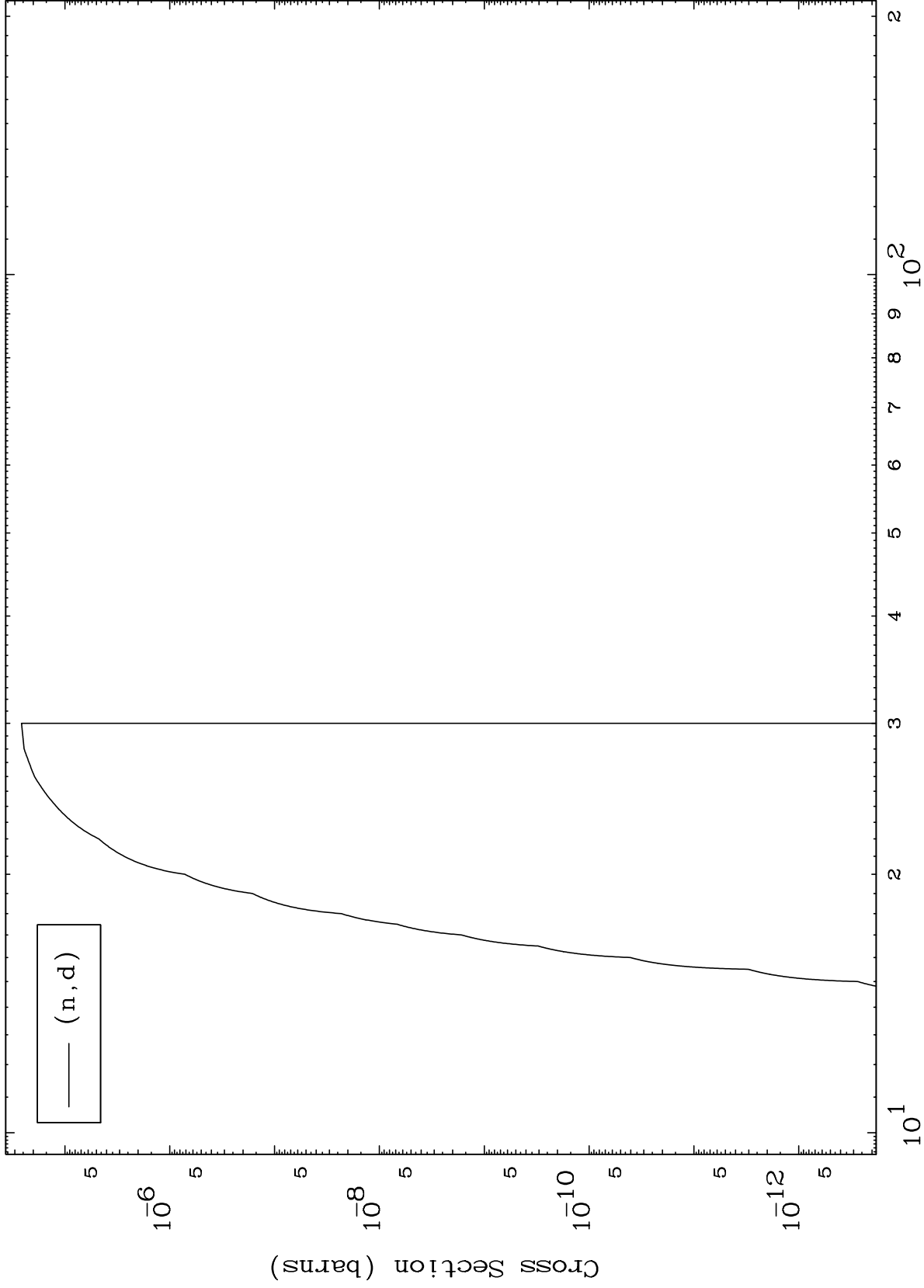
<sup>75</sup>Re-190m



MAT 7541

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

<sup>75</sup>Re-190m



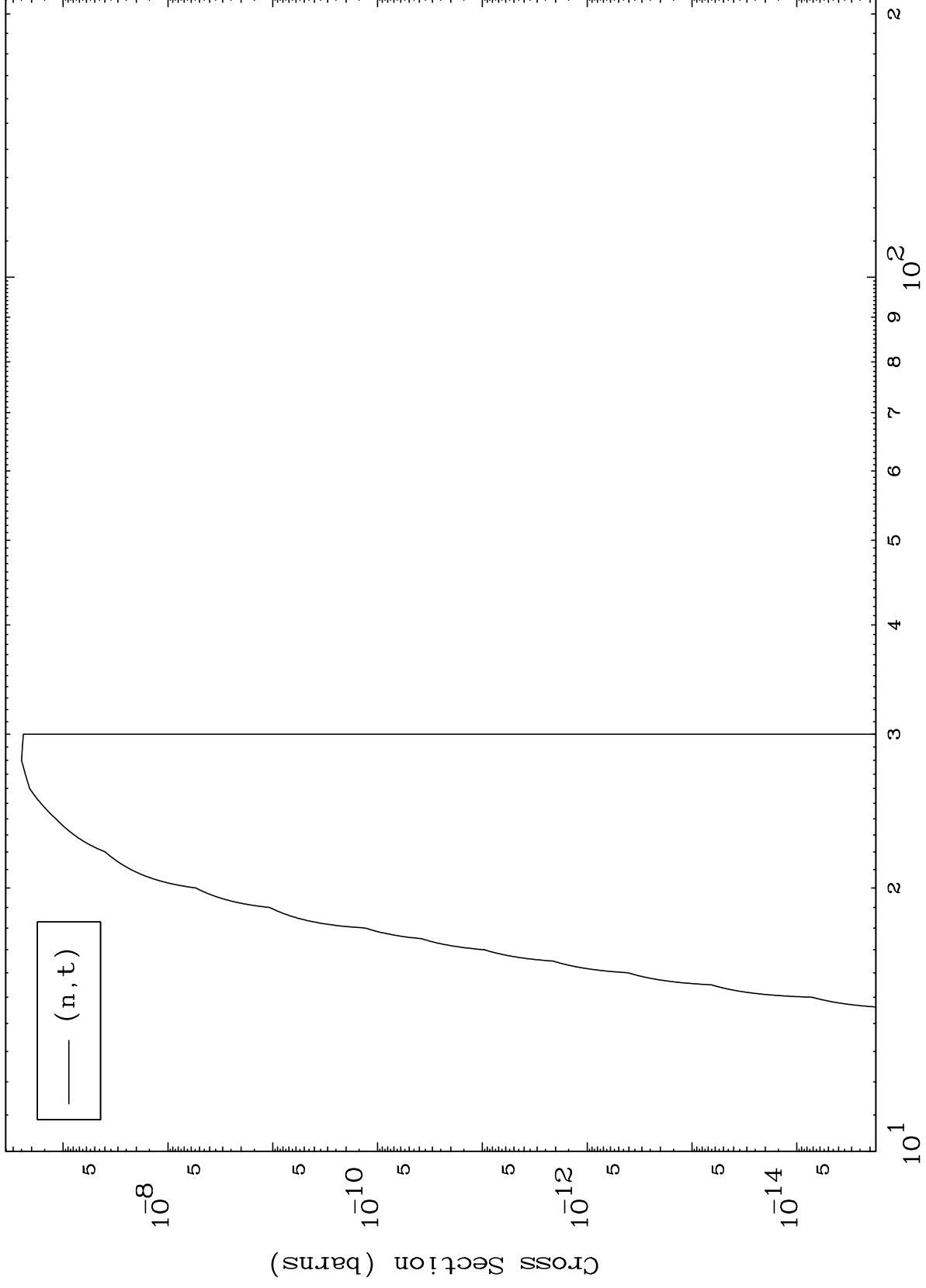
Incident Energy (MeV)

<sup>75</sup>Re-190m

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( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

75-Re-190m



Incident Energy (MeV)

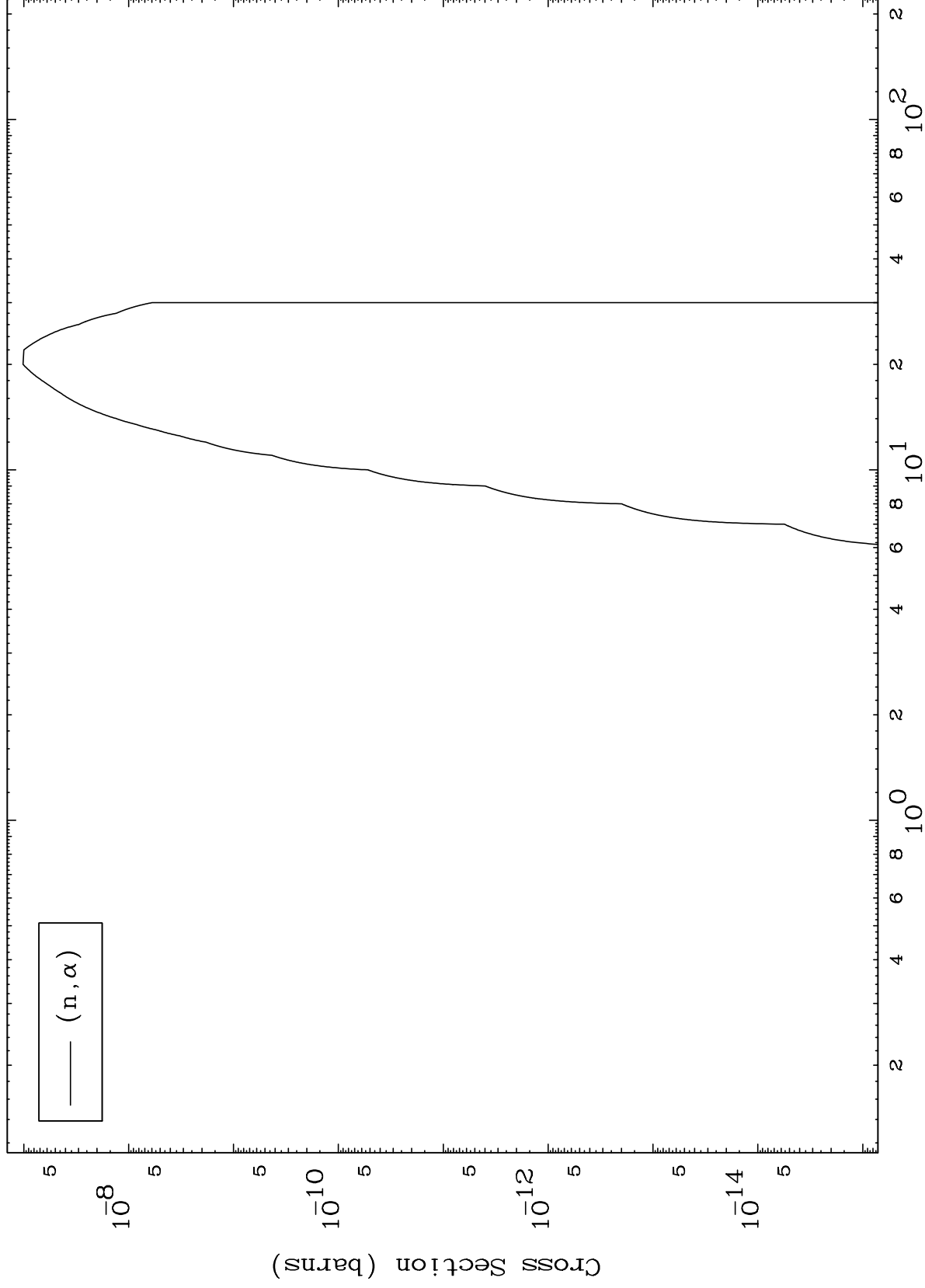
75-Re-190m

MAT 7541

( $\gamma, \alpha$ ) Levels

75-Re-190m

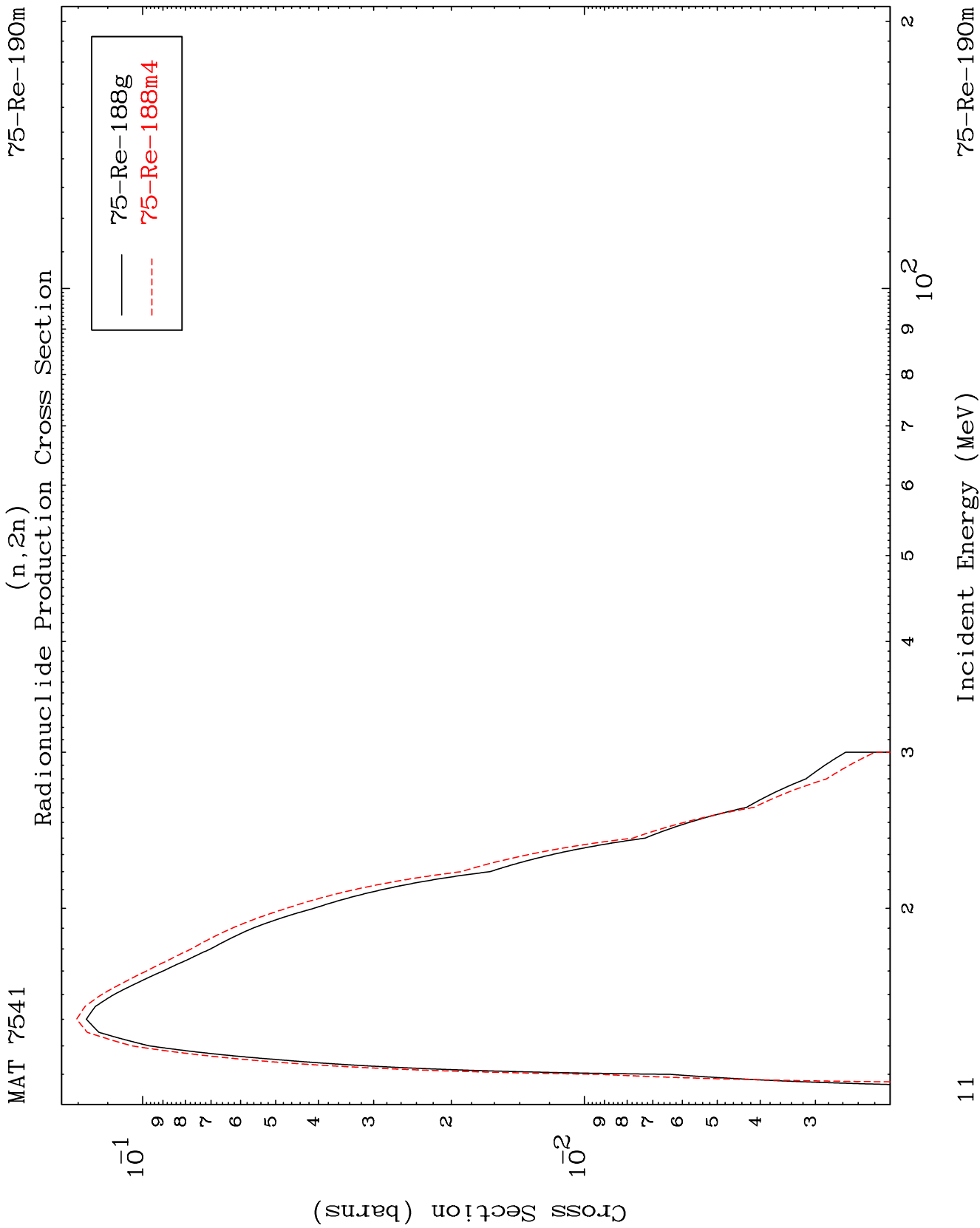
0 Kelvin Cross Sections



10

Incident Energy (MeV)

75-Re-190m

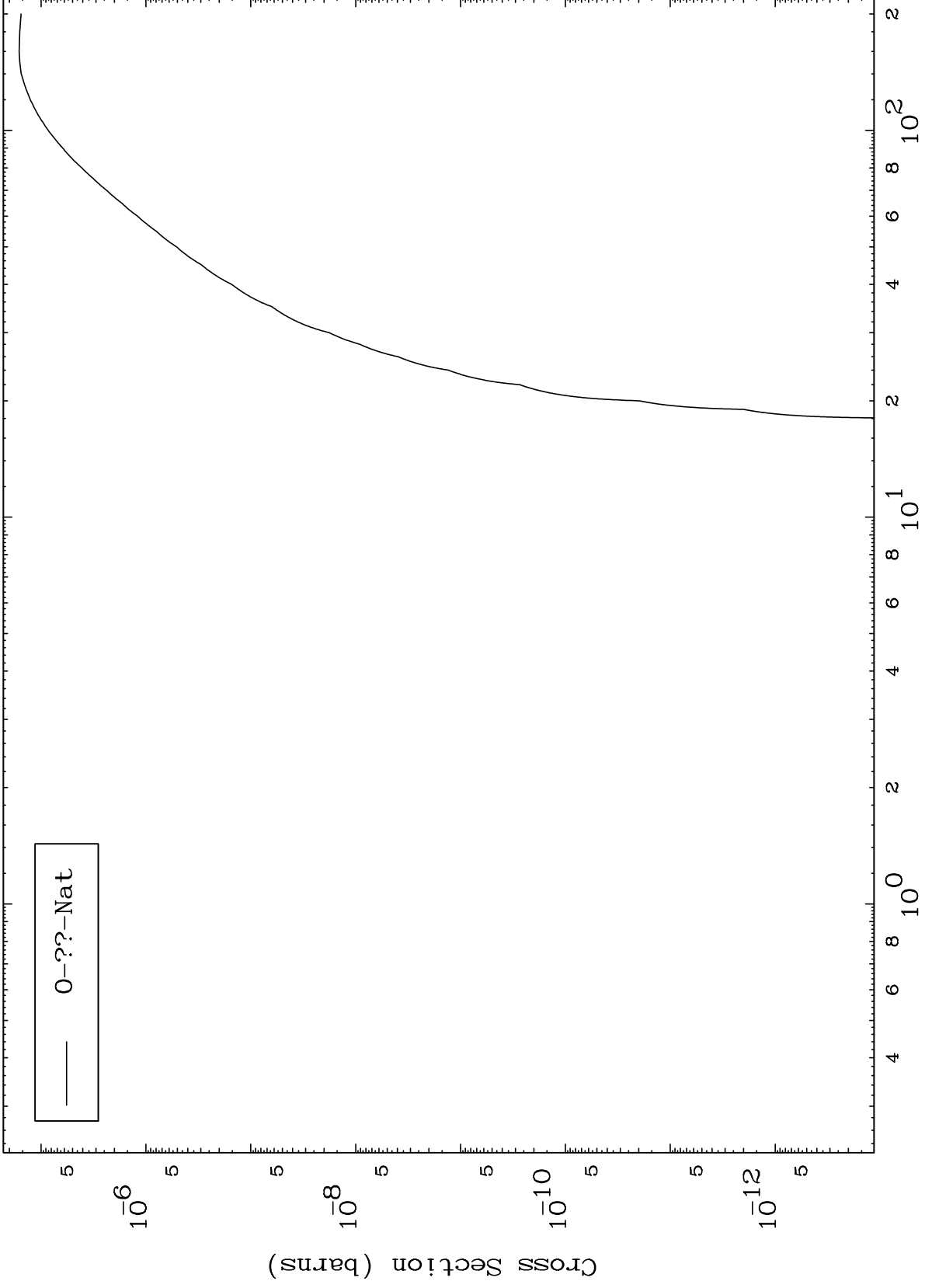


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Fission

75-Re-190m

Radionuclide Production Cross Section



12

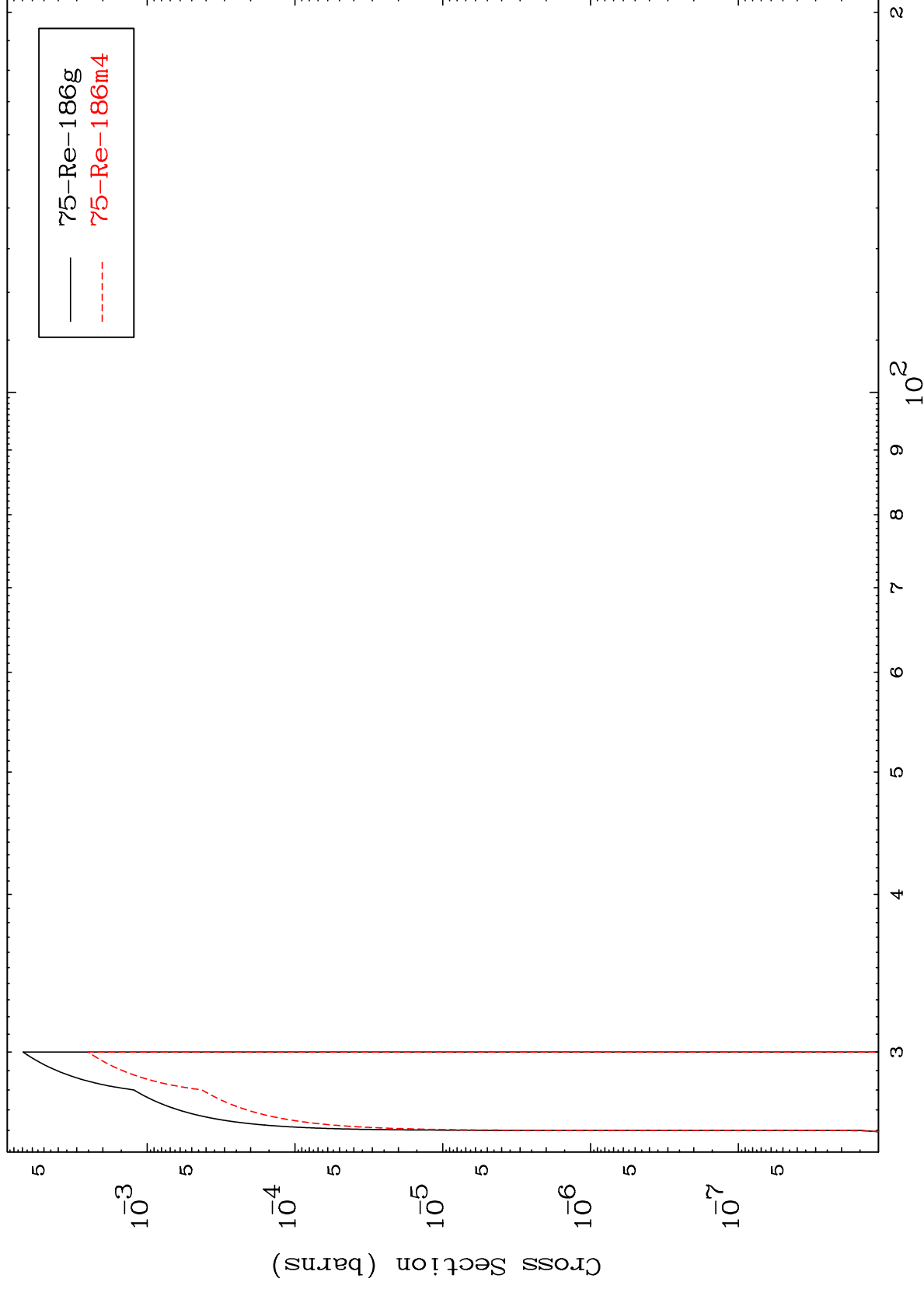
Incident Energy (MeV)

75-Re-190m

MAT 7541

<sup>75</sup>Re-190m

(n,4n)  
Radionuclide Production Cross Section



13

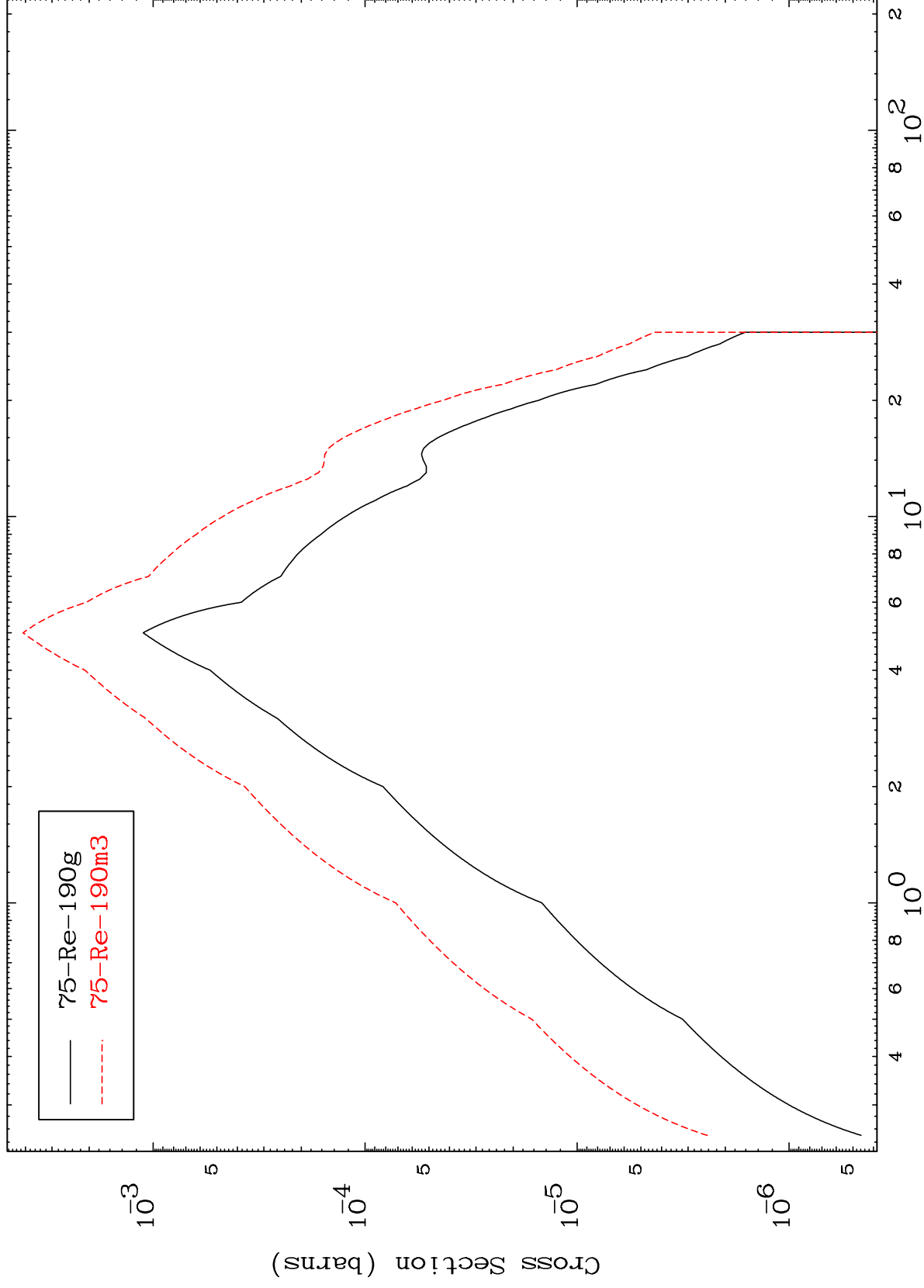
<sup>75</sup>Re-190m

Incident Energy (MeV)

MAT 7541

75-Re-190m

Radionuclide Production Cross Section



14

Incident Energy (MeV)

75-Re-190m