

Program EVALPLOT
(Version 2021-1)

by

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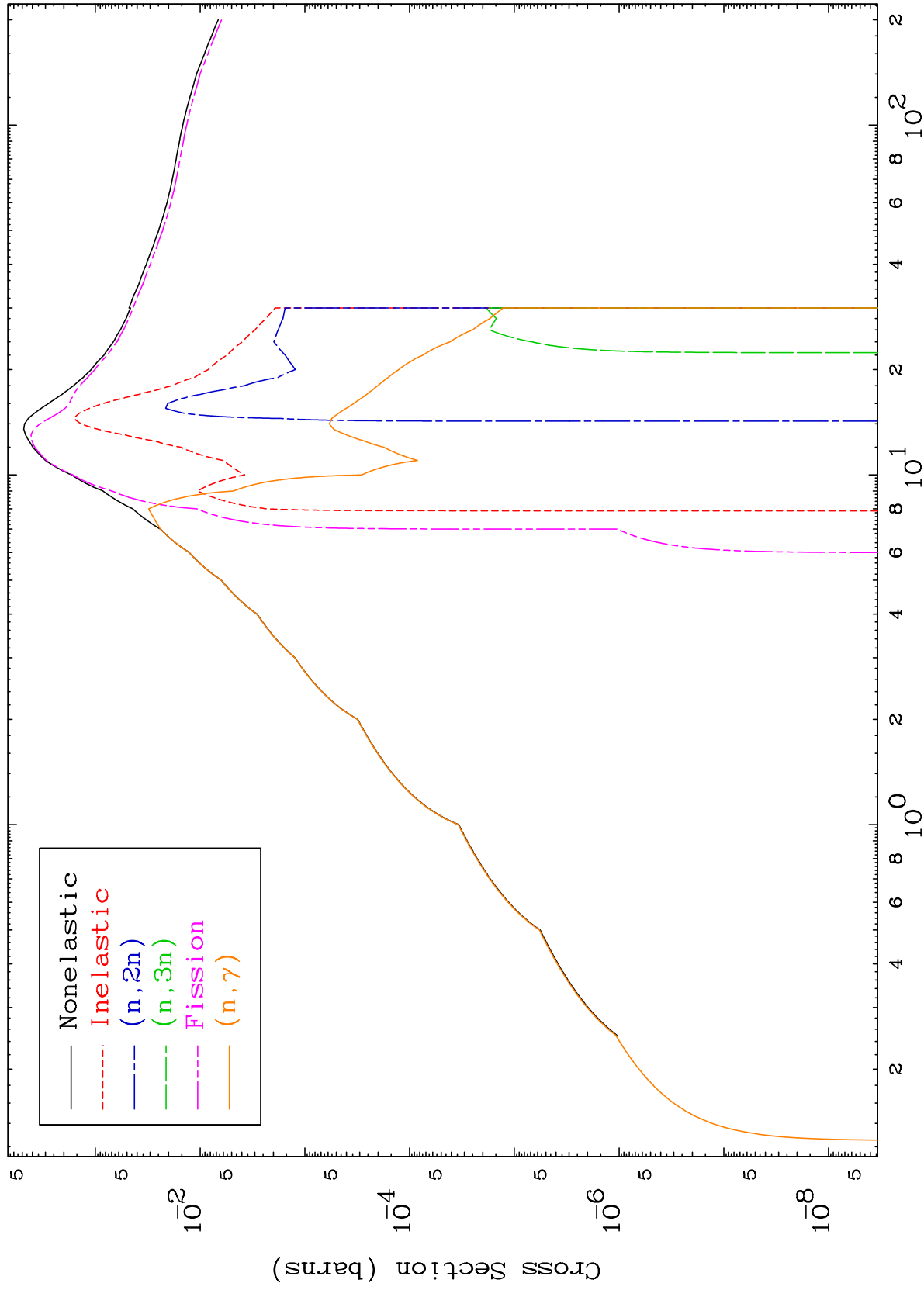
Press Mouse Button to Start

MAT 9207

Photon Major

92-U -228

0 Kelvin Cross Sections



1

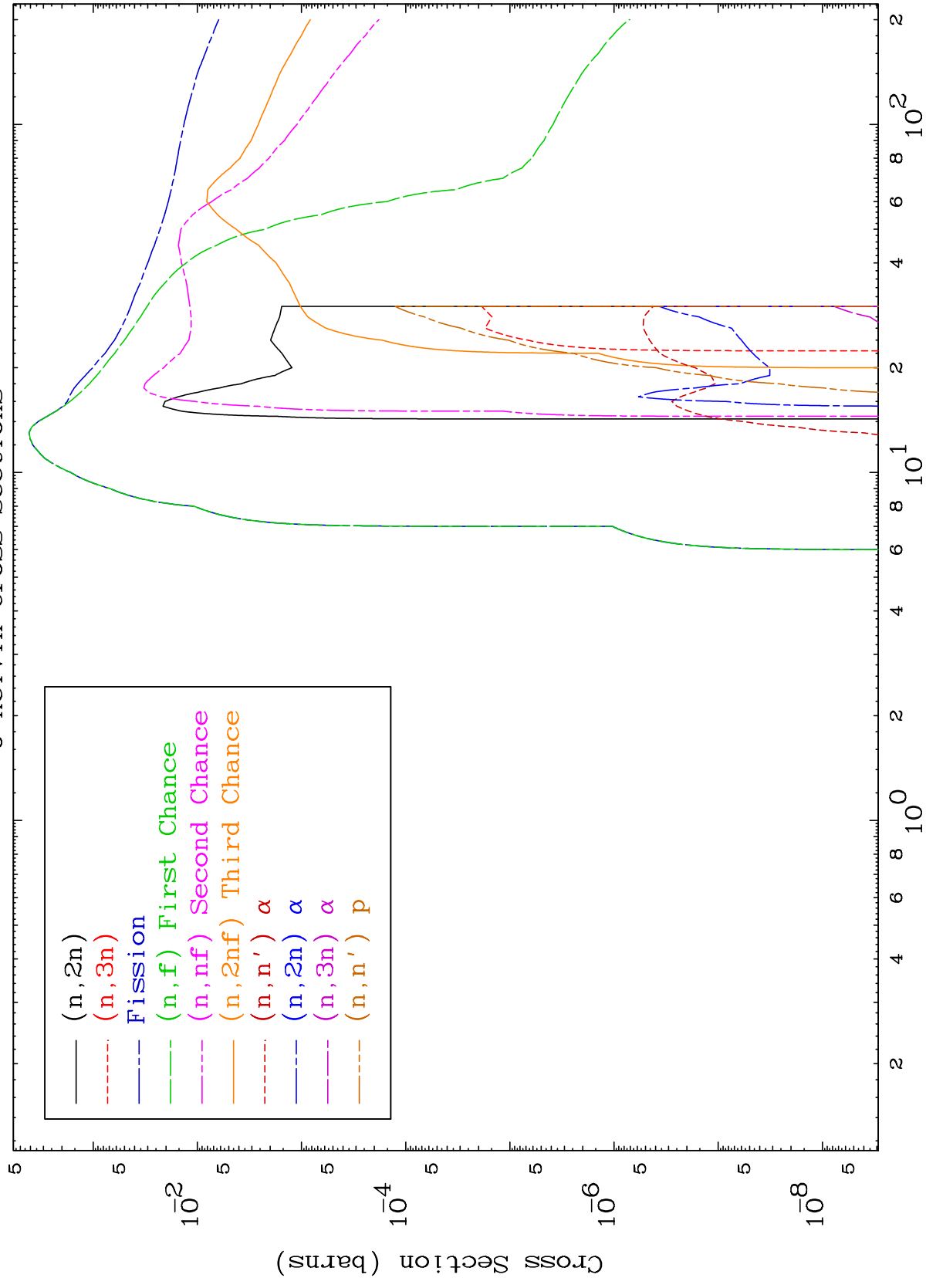
Incident Energy (MeV)

92-U -228

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Photon Neutron Absorption
0 Kelvin Cross Sections

92-U -228



2

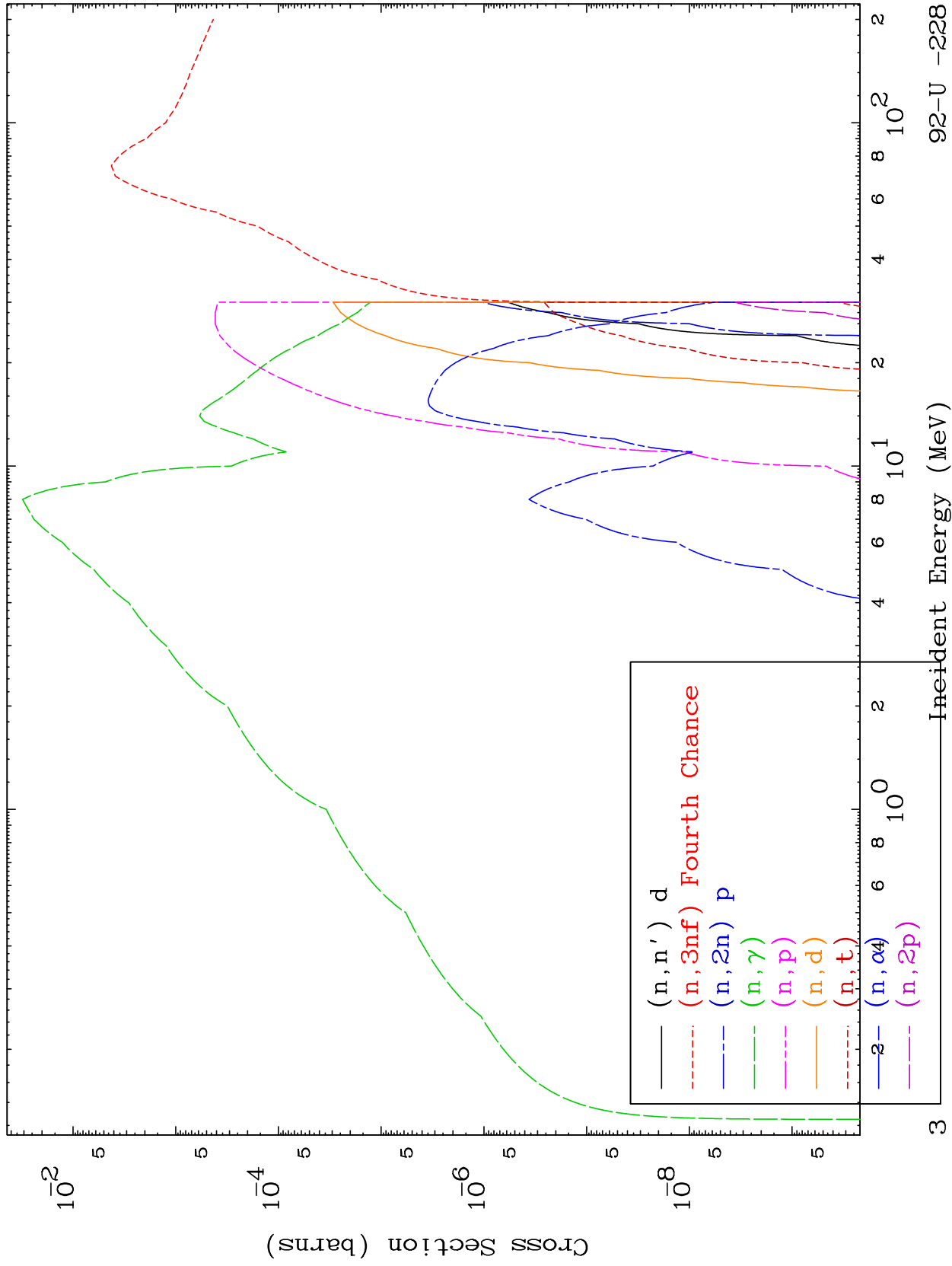
Incident Energy (MeV)

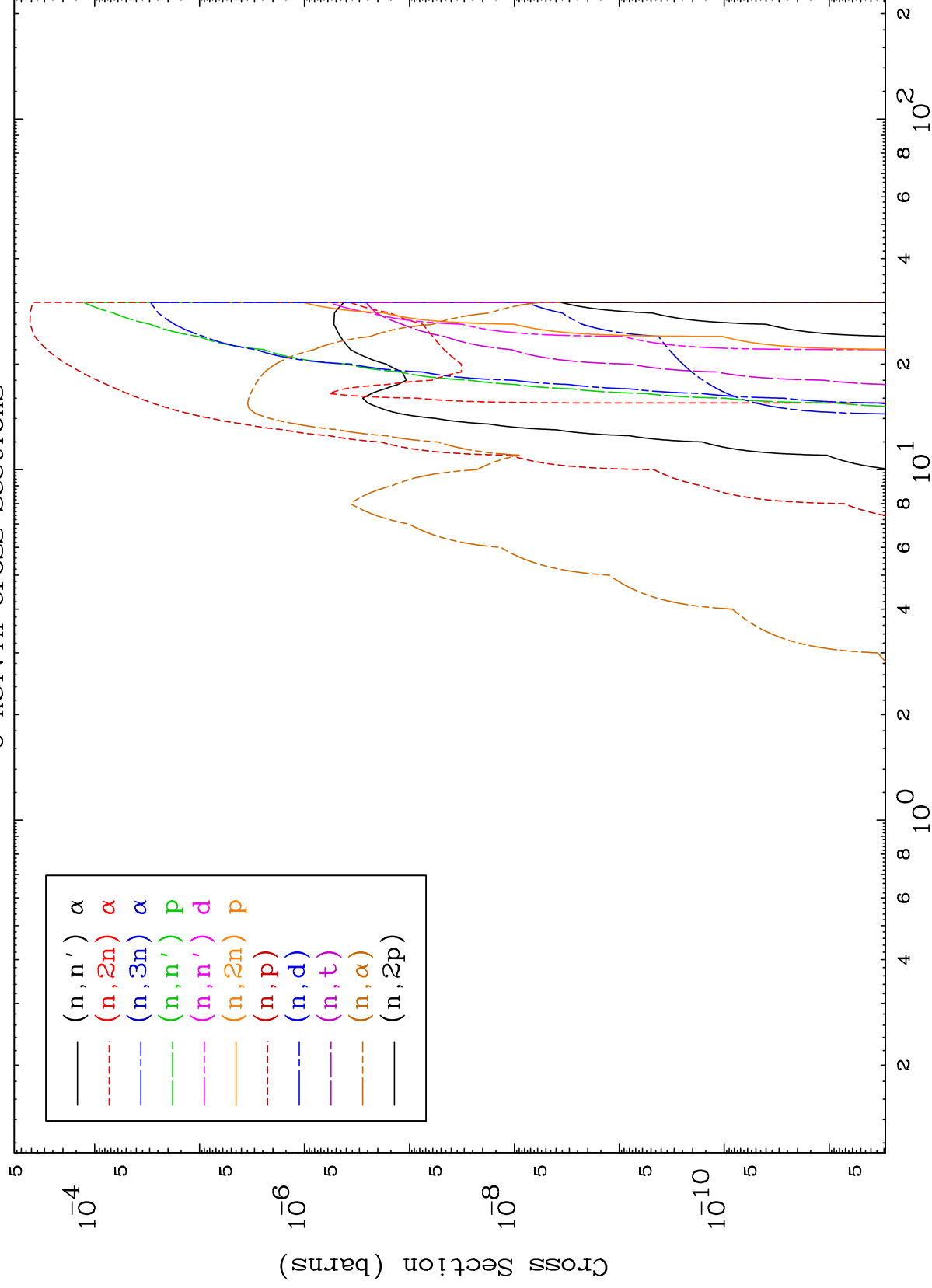
92-U -228

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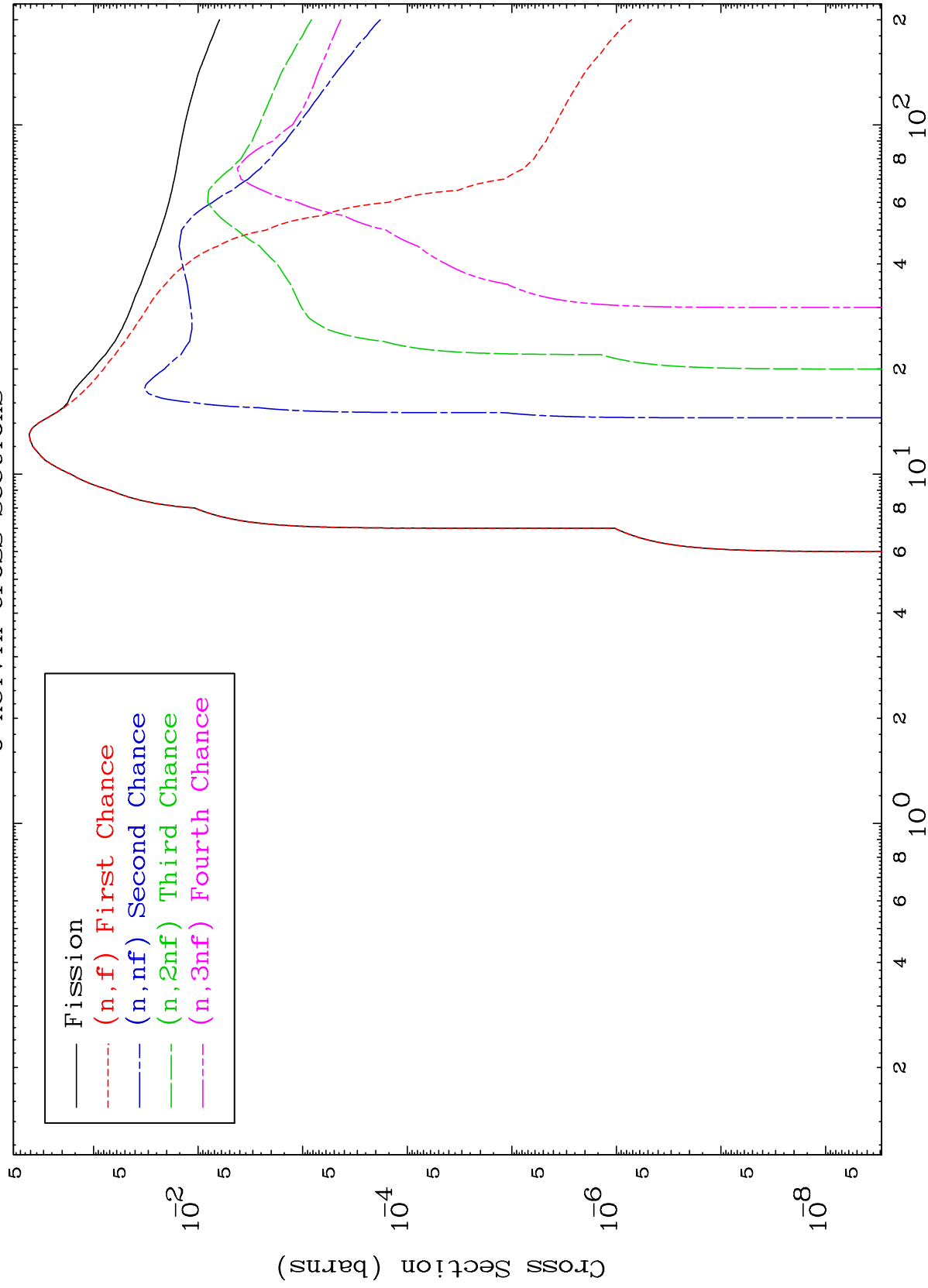
Photon Neutron Absorption
0 Kelvin Cross Sections

92-U -228





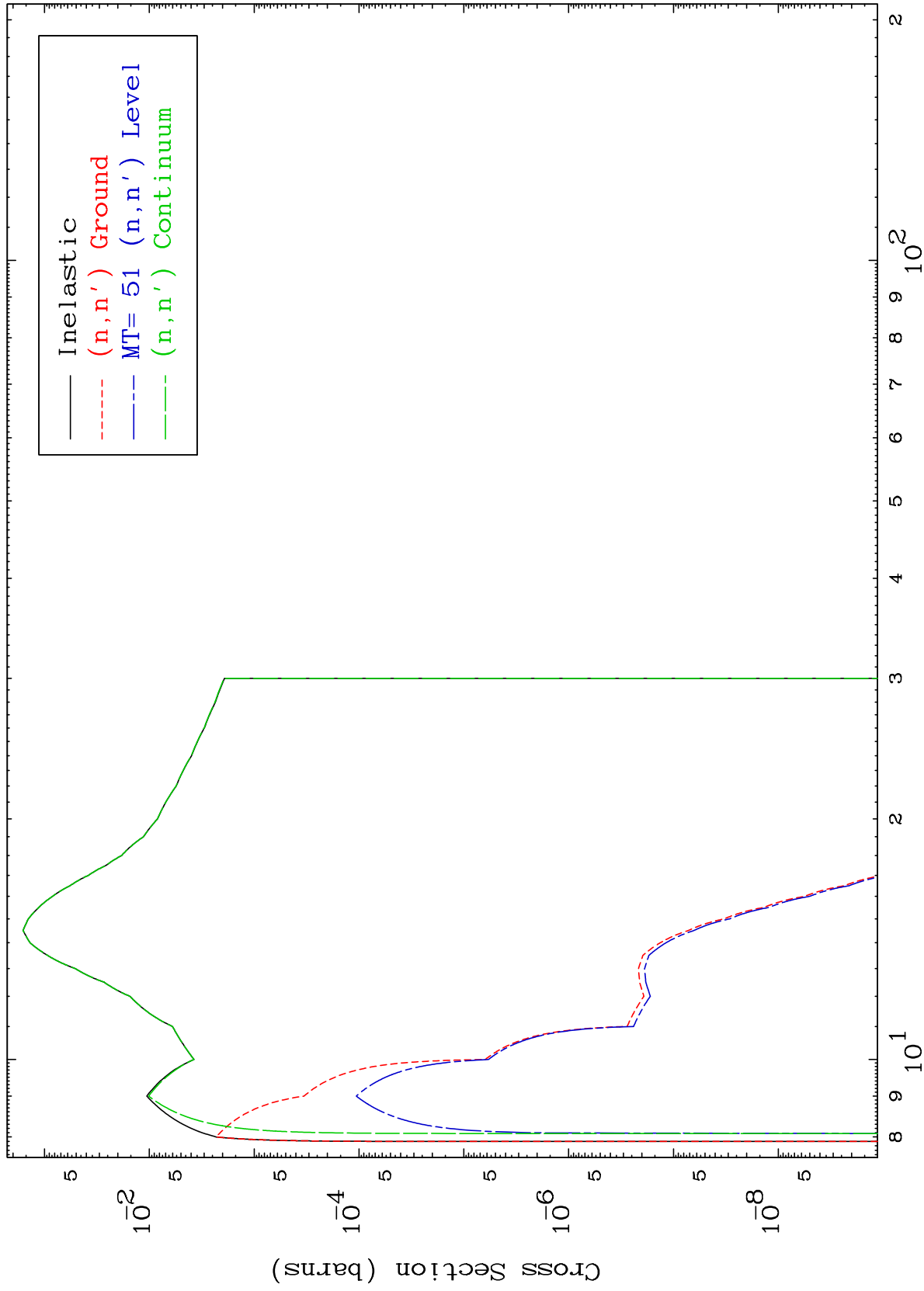
0 Kelvin Cross Sections



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(γ, n') Levels
0 Kelvin Cross Sections

92-U -228



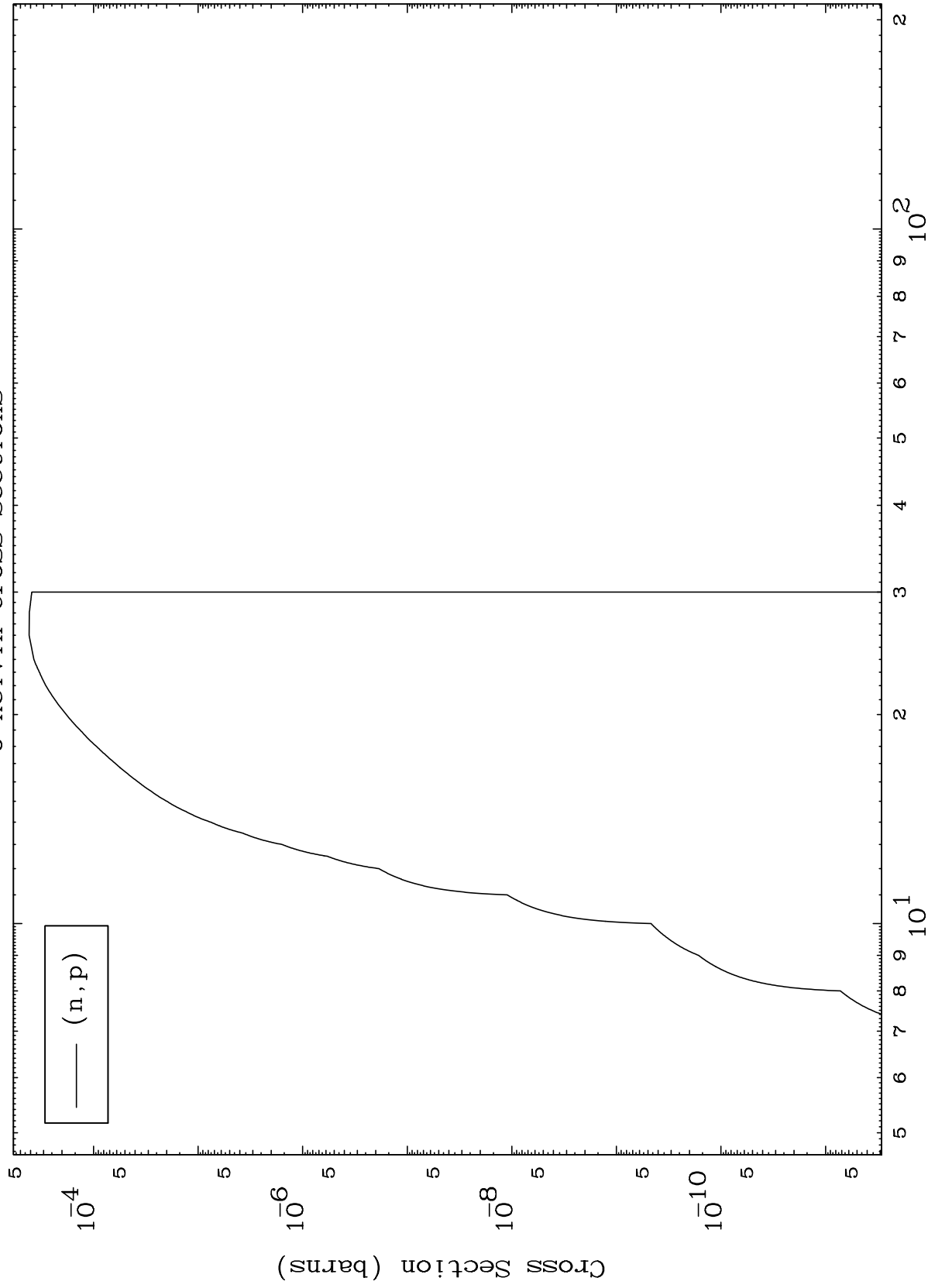
Incident Energy (MeV)

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92-U -228

(γ ,p) Levels
0 Kelvin Cross Sections



7

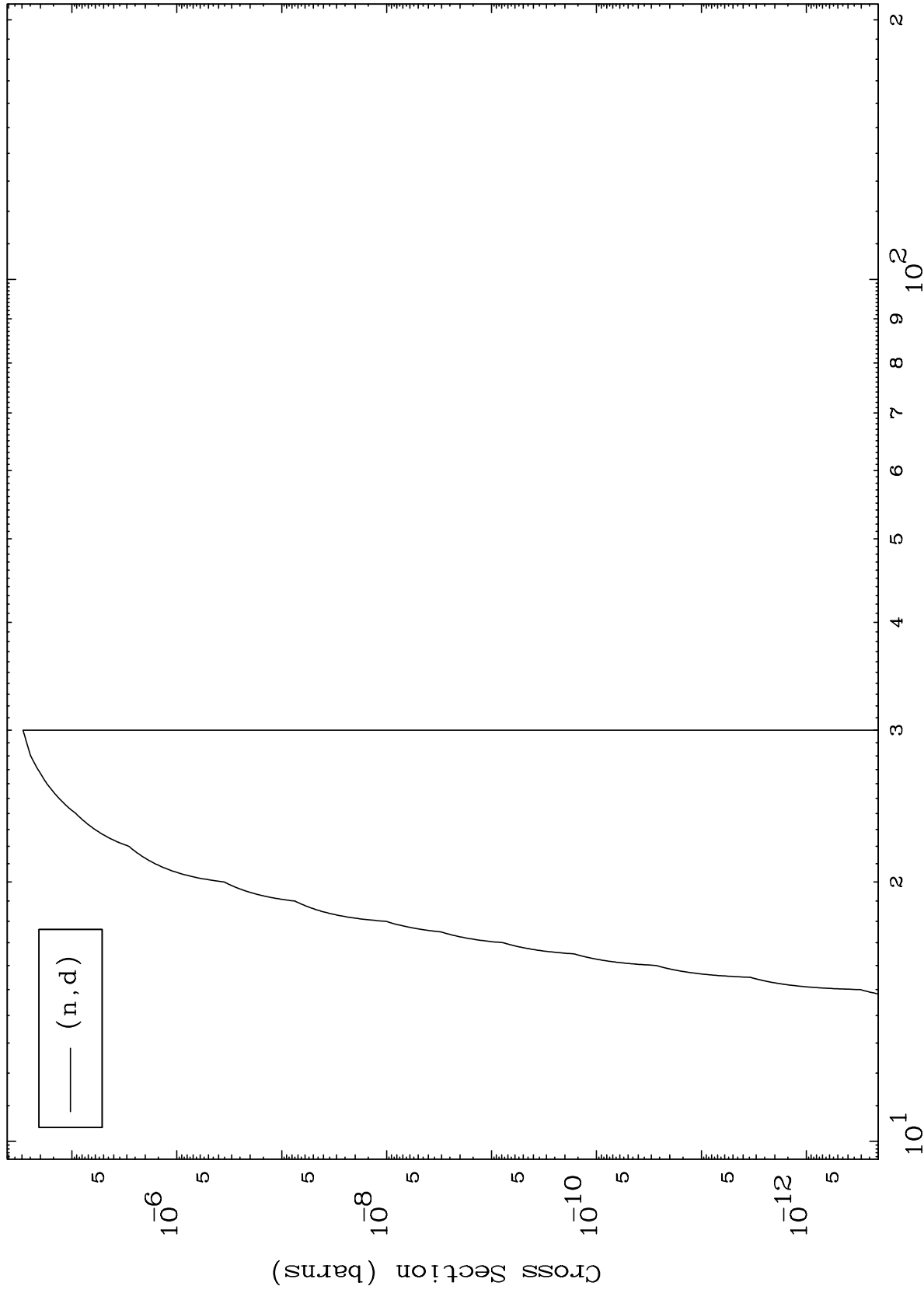
Incident Energy (MeV)

92-U -228

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(γ ,d) Levels
0 Kelvin Cross Sections

92-U -228



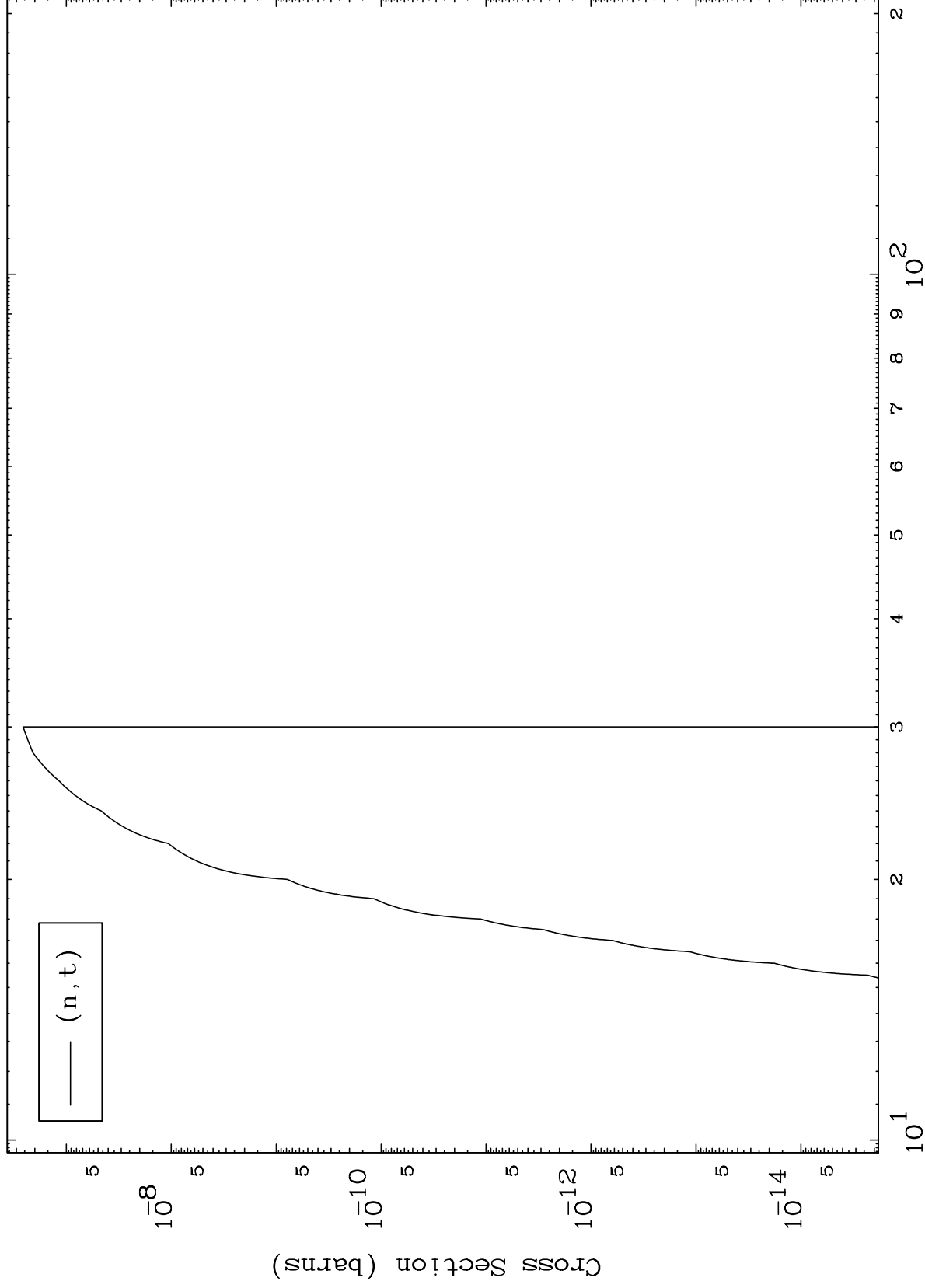
Incident Energy (MeV)

92-U -228

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(γ, t) Levels
0 Kelvin Cross Sections

92-U -228



Incident Energy (MeV)

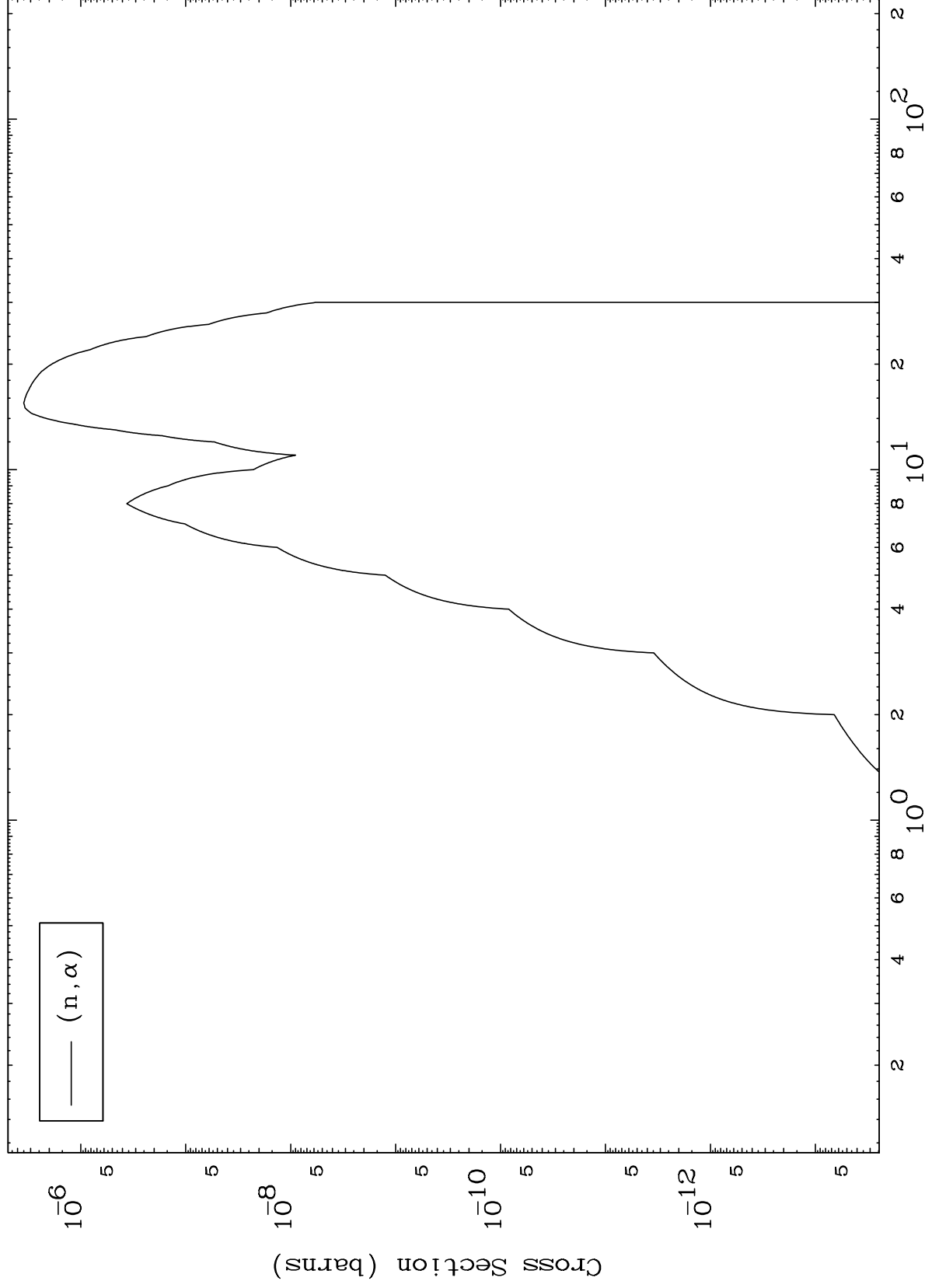
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(γ, α) Levels

92-U -228

0 Kelvin Cross Sections



10

Incident Energy (MeV)

92-U -228

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Photon Neutrons
per Fission

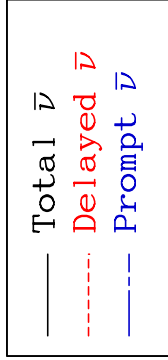
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10^0

10^{-1}

10^{-2}

Neutrons per Fission



150

50

200

11

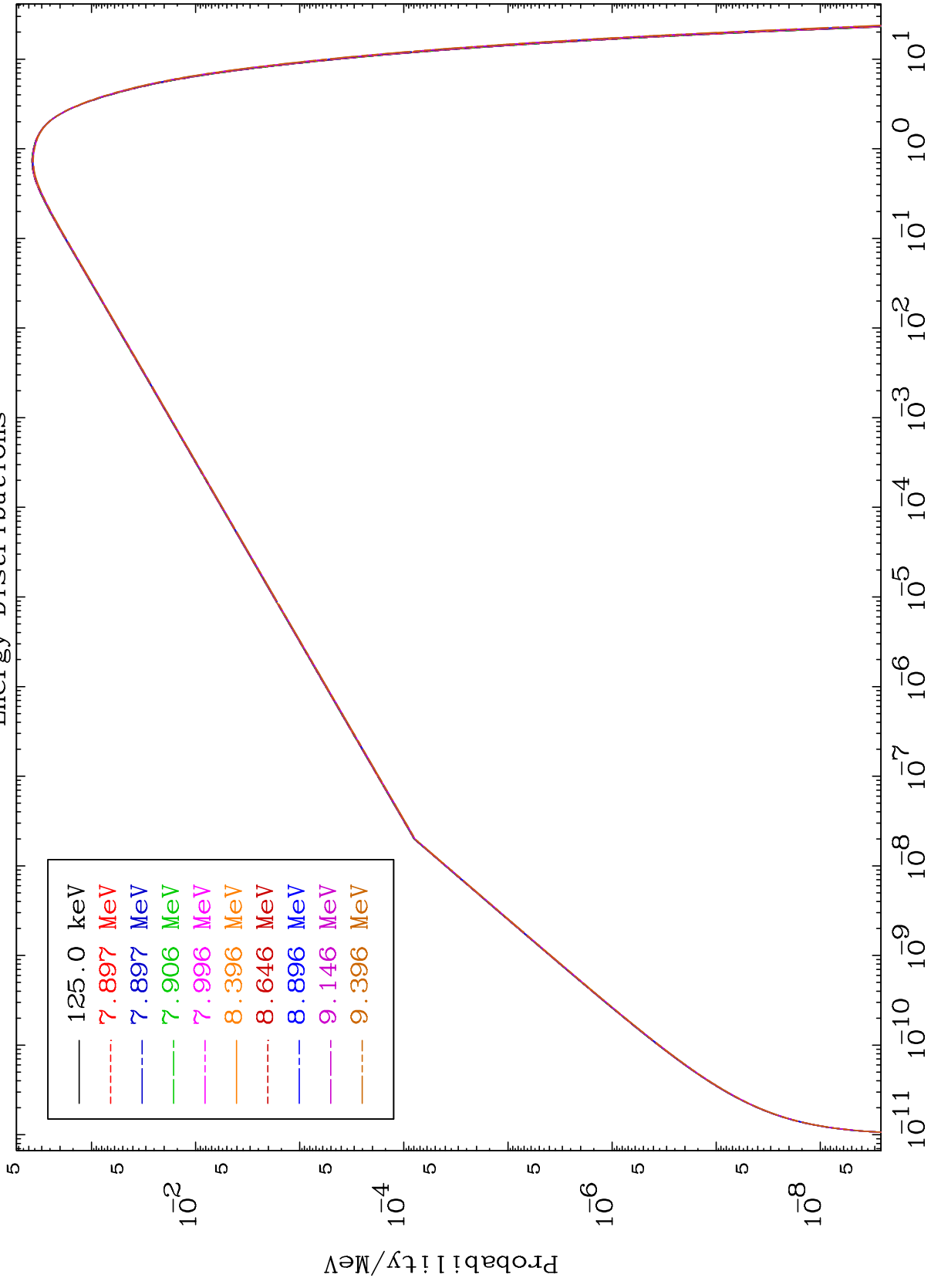
Incident Energy (MeV)

92-U -228

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Fission
Energy Distributions

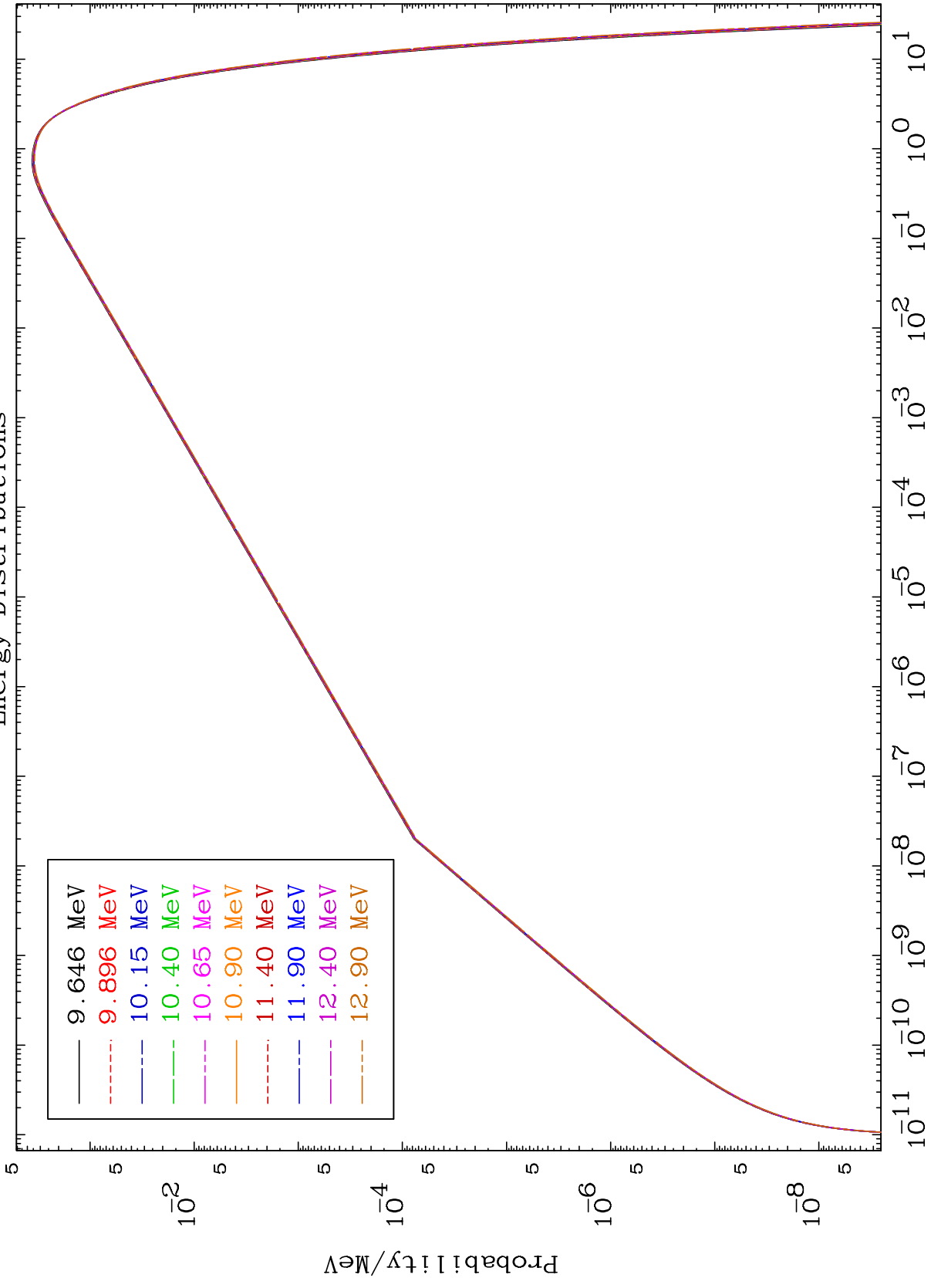
92-U -228



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Fission
Energy Distributions

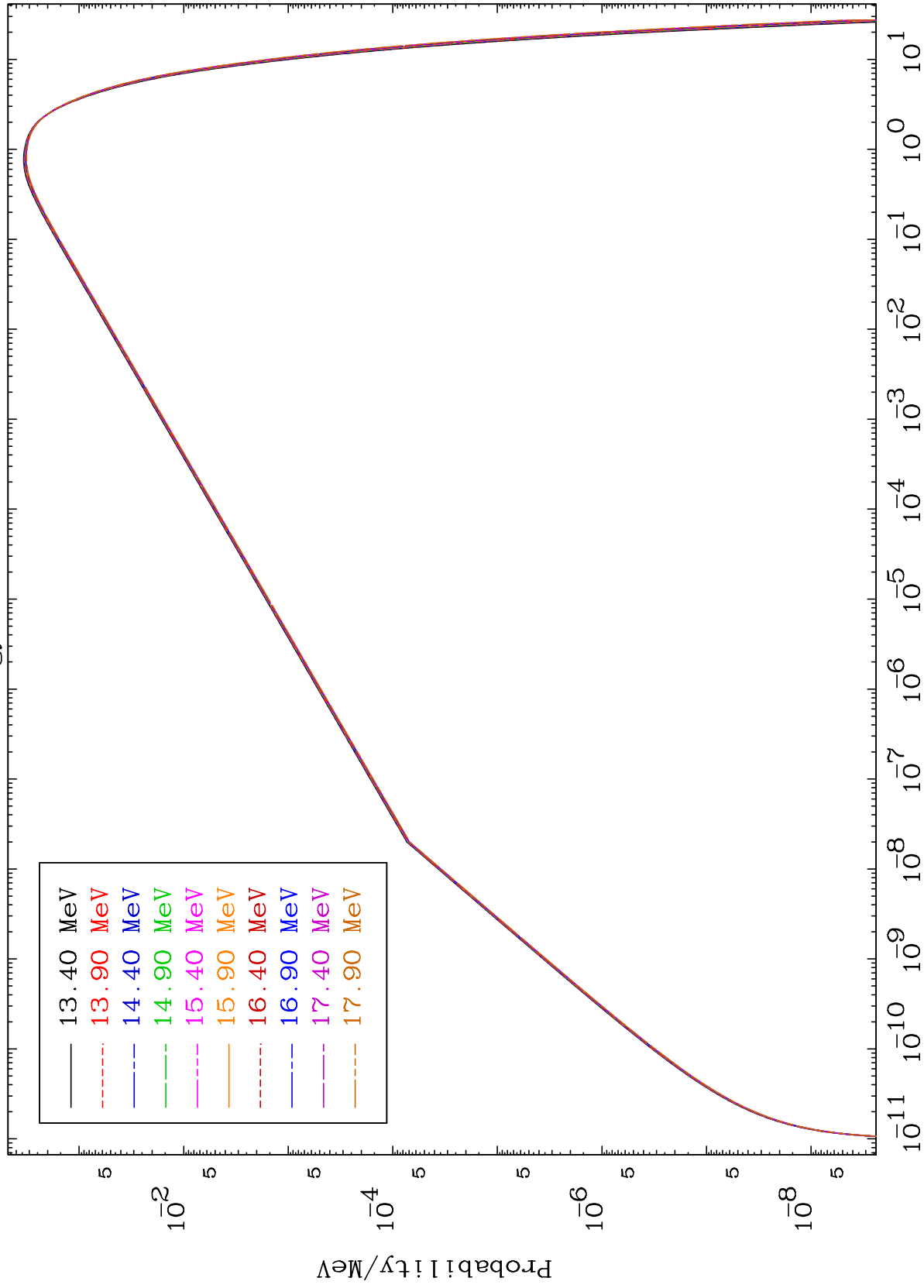
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Fission
Energy Distributions

92-U -228



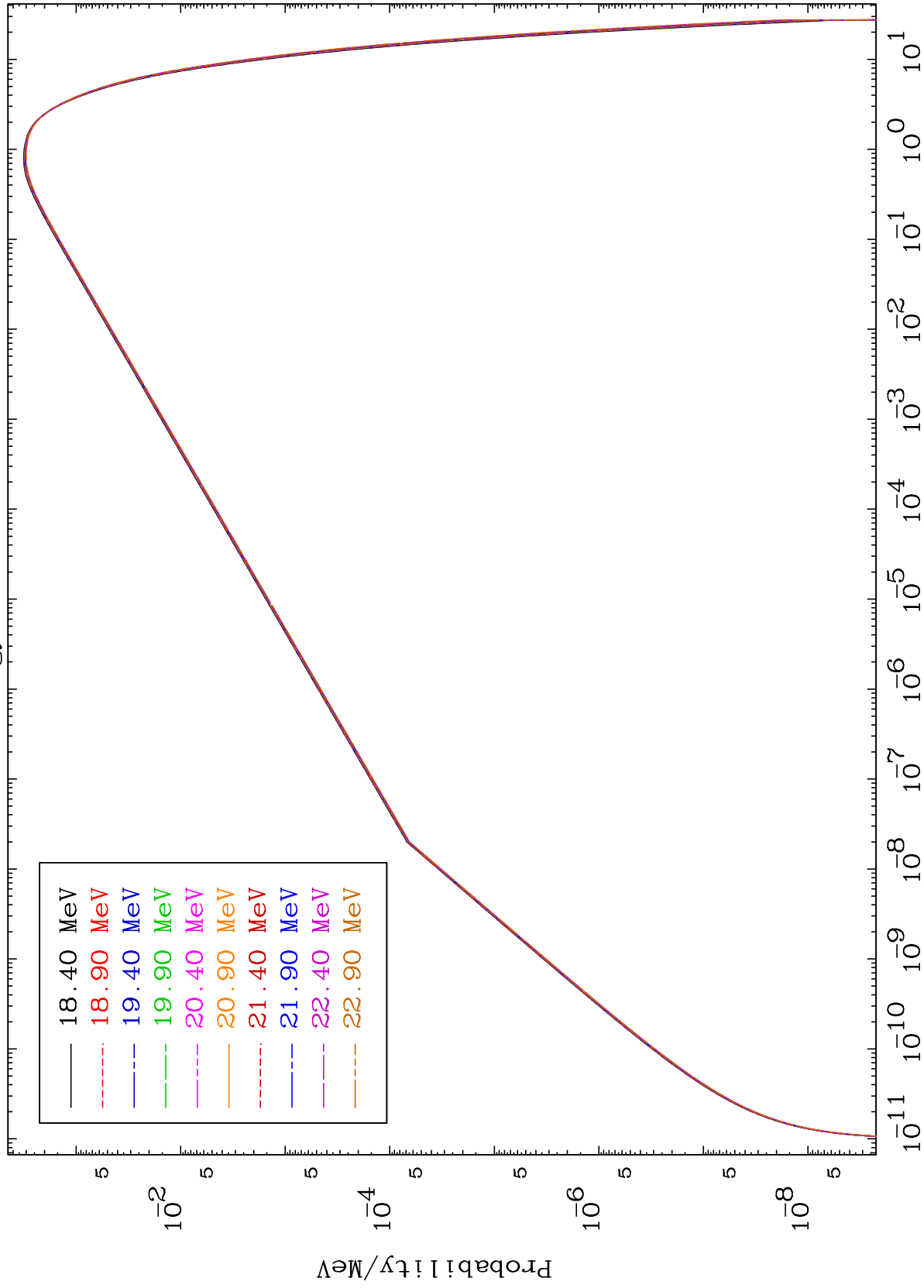
14

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Fission
Energy Distributions

92-U -228



15

Secondary Energy (MeV)

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Fission
Energy Distributions

92-U -228

