

Program EVALPLOT
(Version 2021-1)

by

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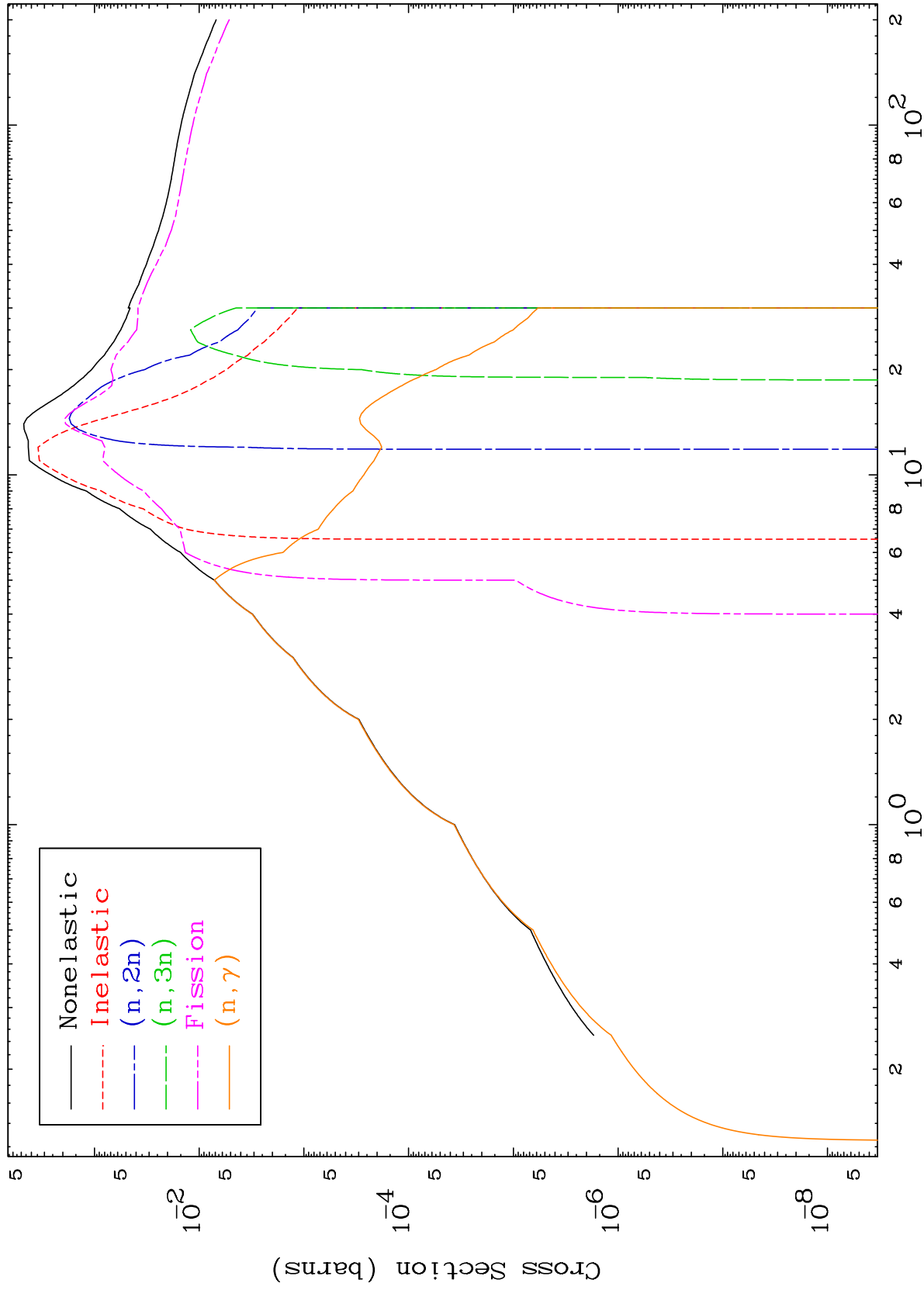
Press Mouse Button to Start

MAT 9231

Photon Major

92-U -236

0 Kelvin Cross Sections



1

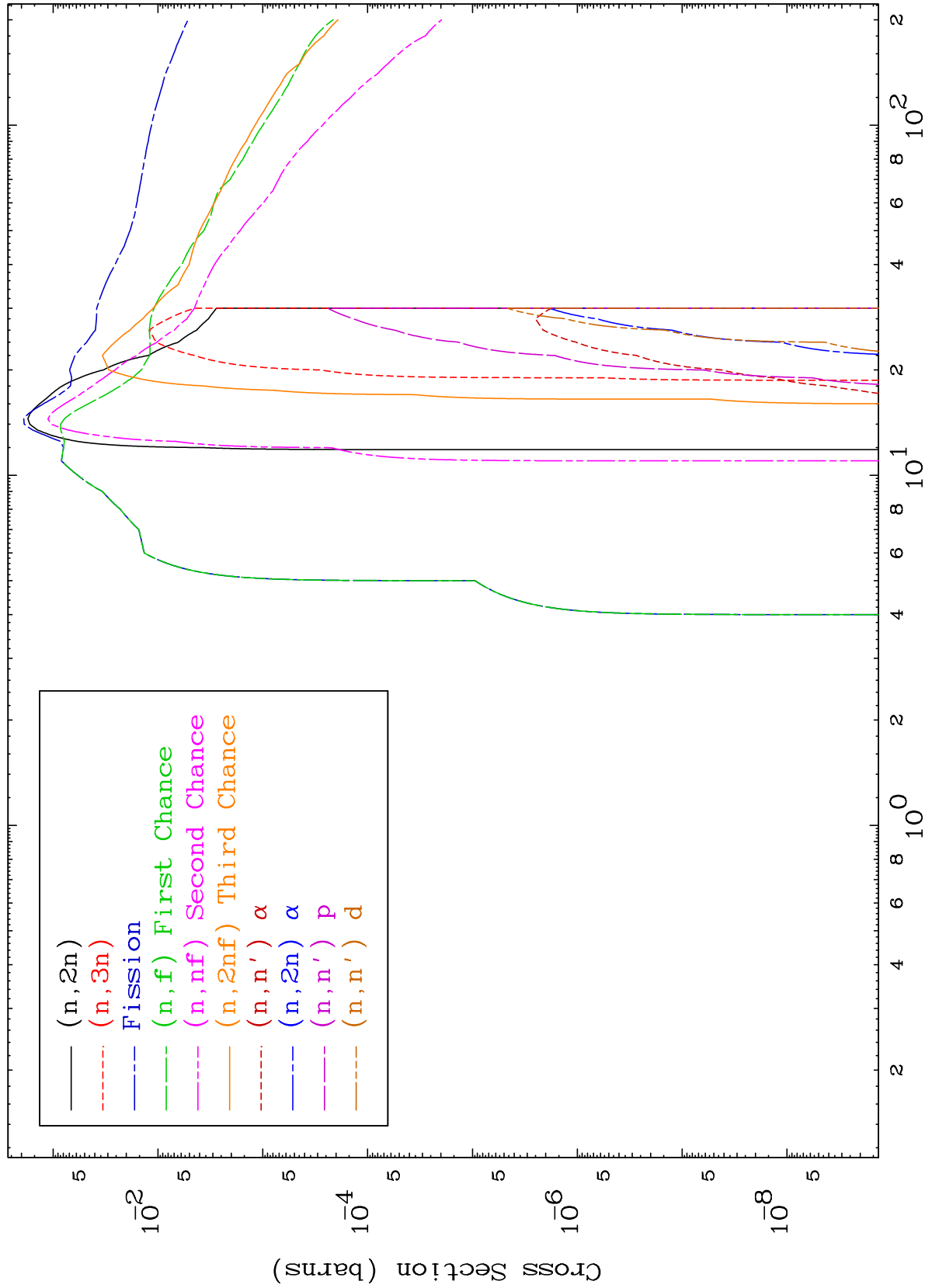
Incident Energy (MeV)

92-U -236

MAT 9231

Photon Neutron Absorption
0 Kelvin Cross Sections

92-U -236



2

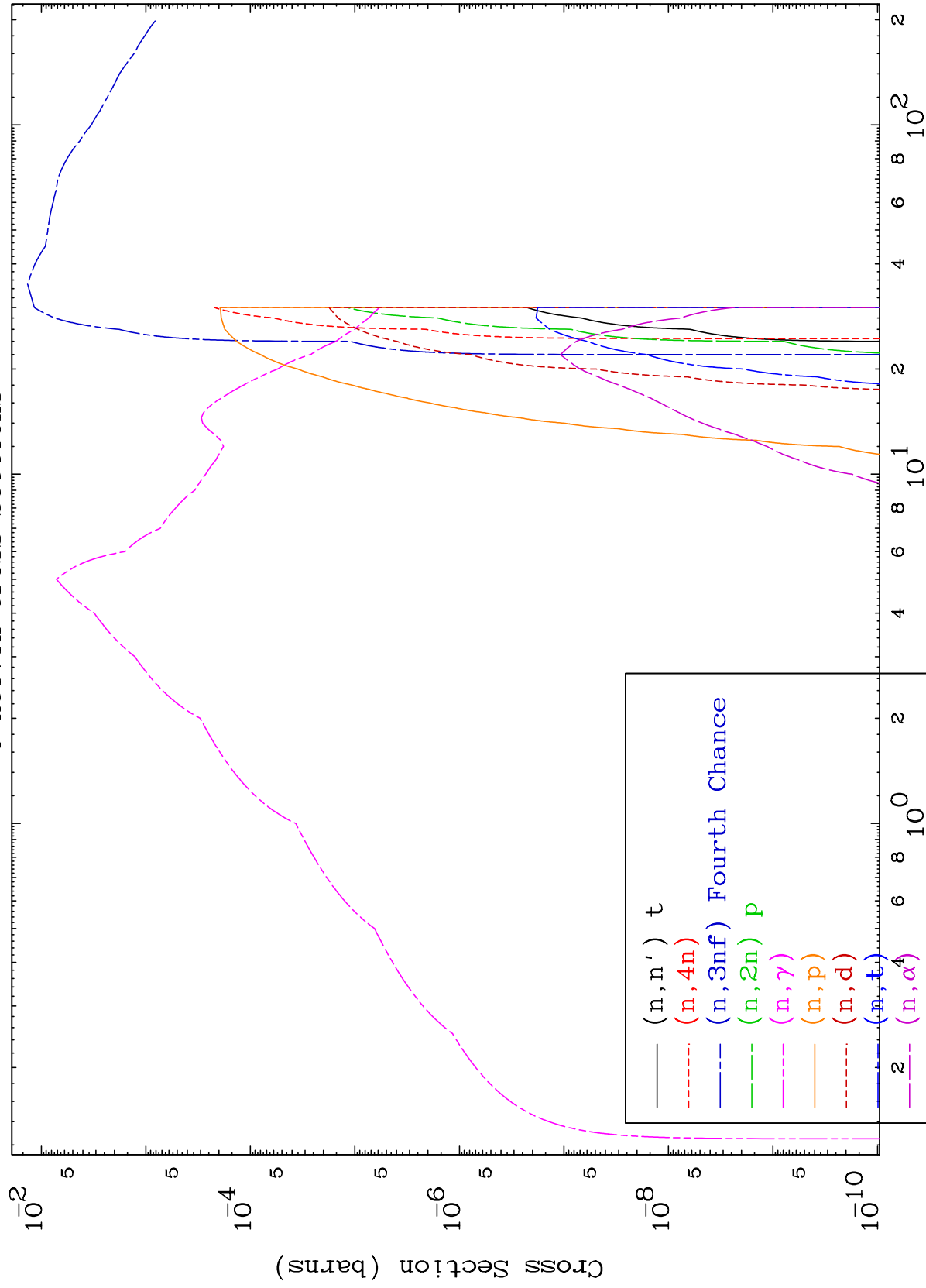
Incident Energy (MeV)

92-U -236

MAT 9231

Photon Neutron Absorption
0 Kelvin Cross Sections

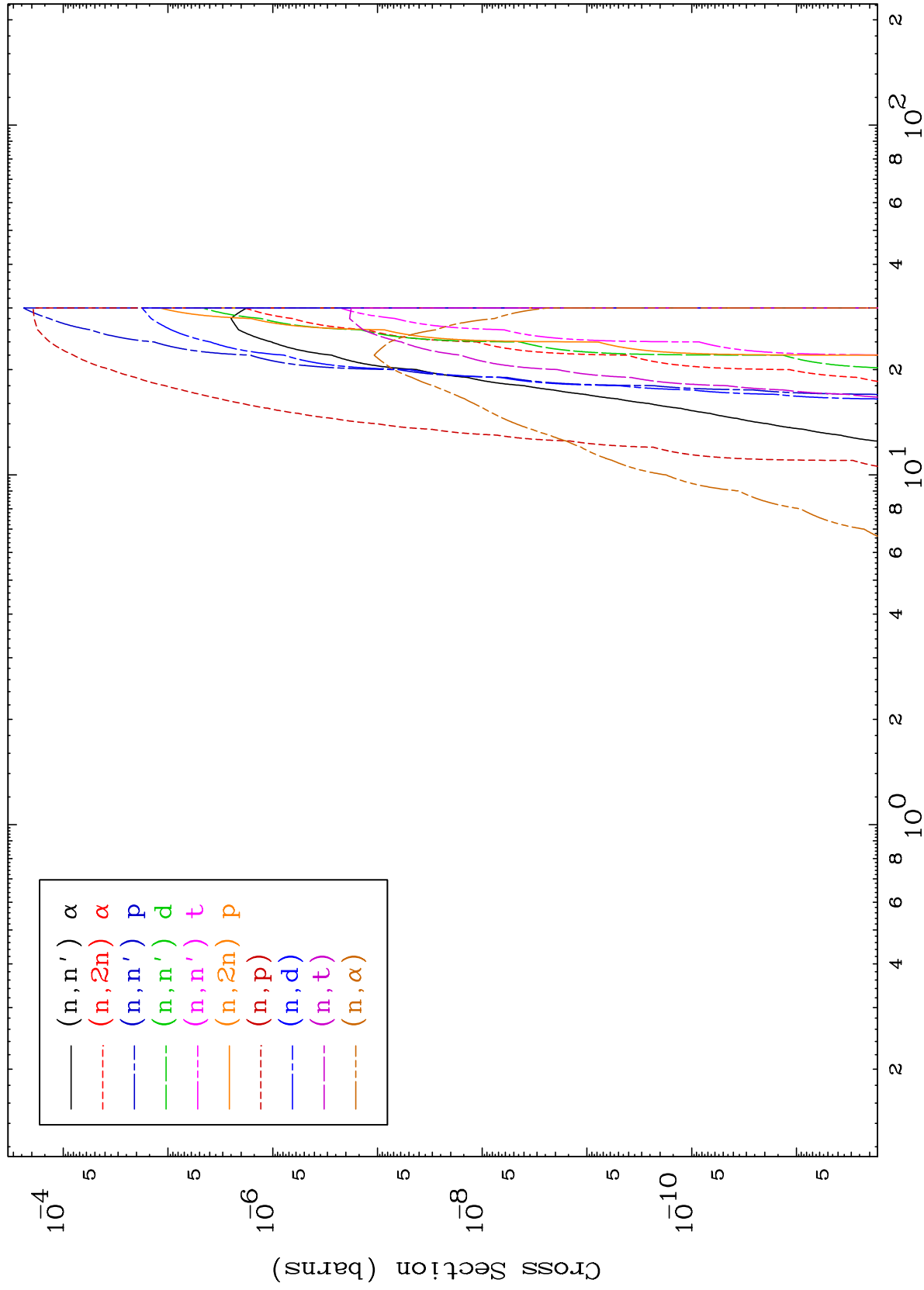
92-U -236

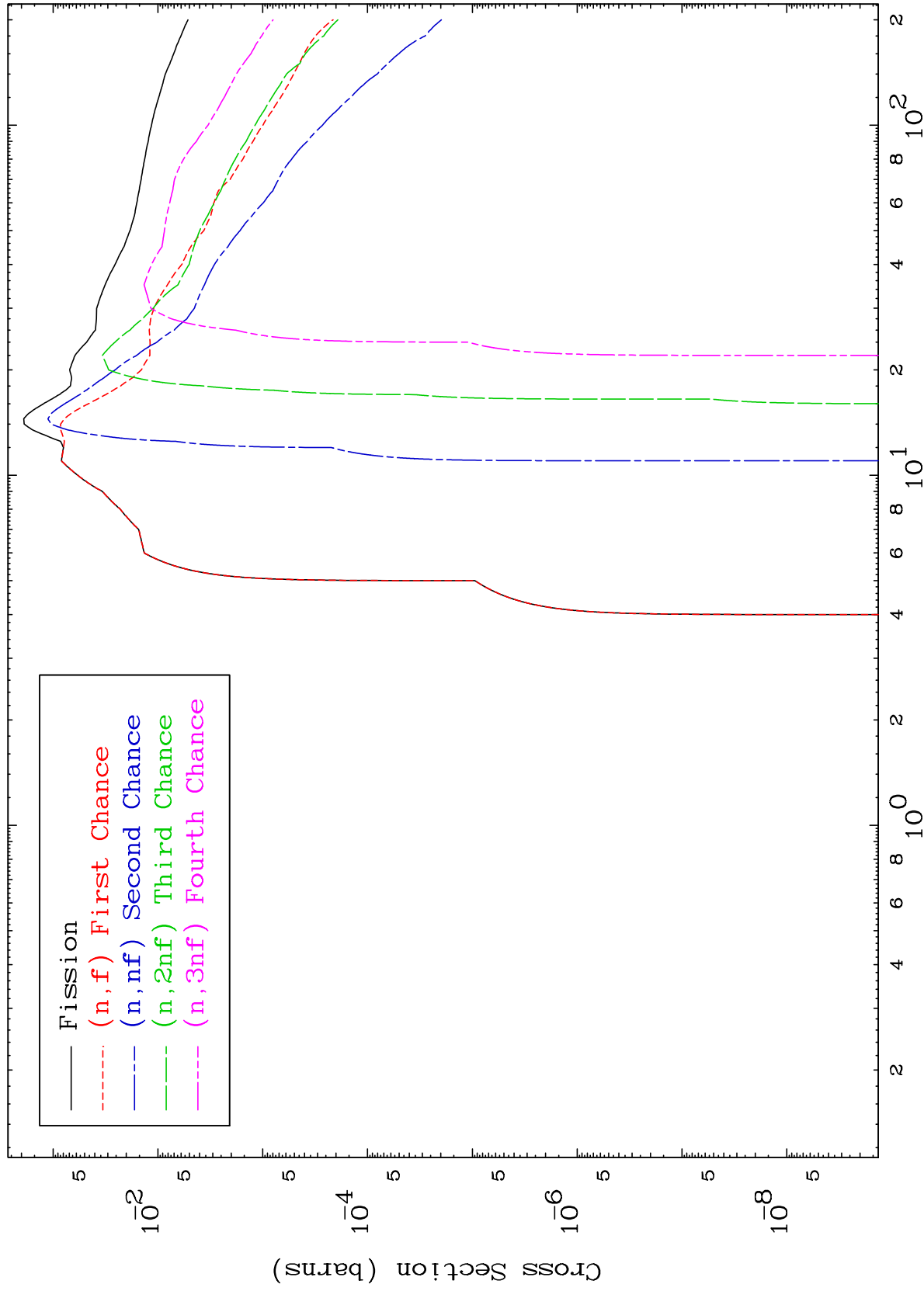


3

Incident Energy (MeV)

92-U -236

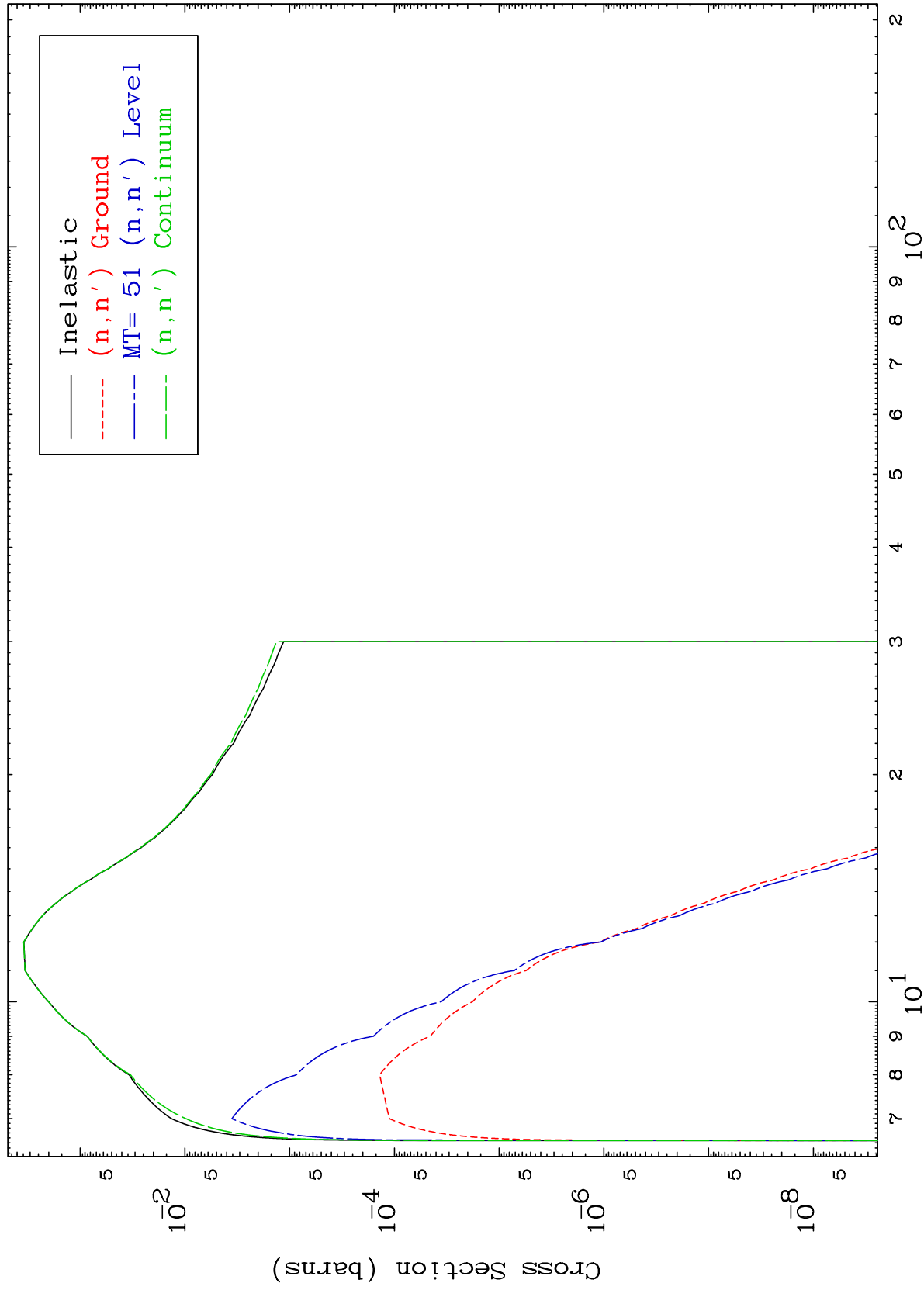




MAT 9231

(γ, n') Levels
0 Kelvin Cross Sections

92-U -236



6

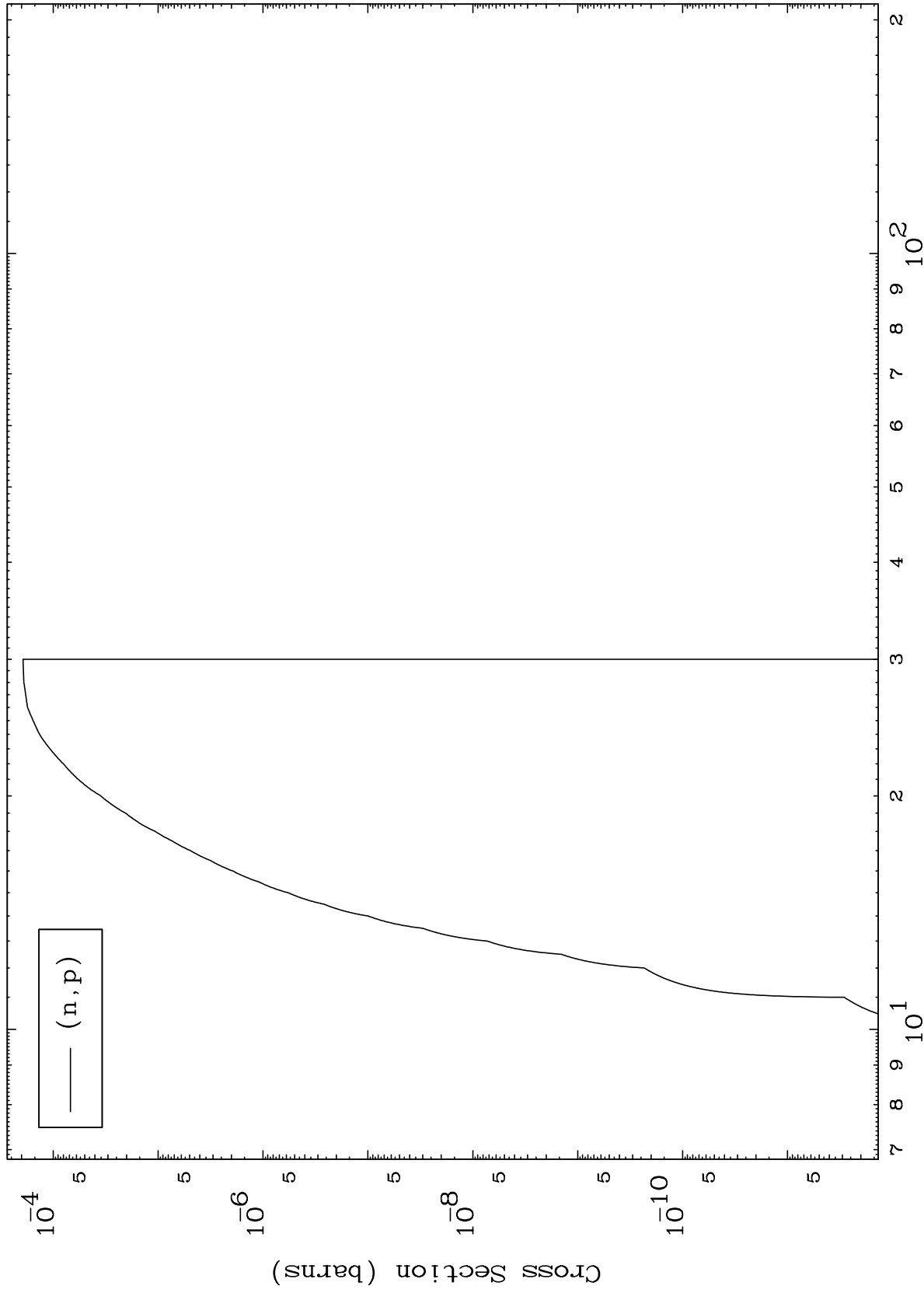
Incident Energy (MeV)

92-U -236

MAT 9231

92-U -236

(γ ,p) Levels
0 Kelvin Cross Sections



7

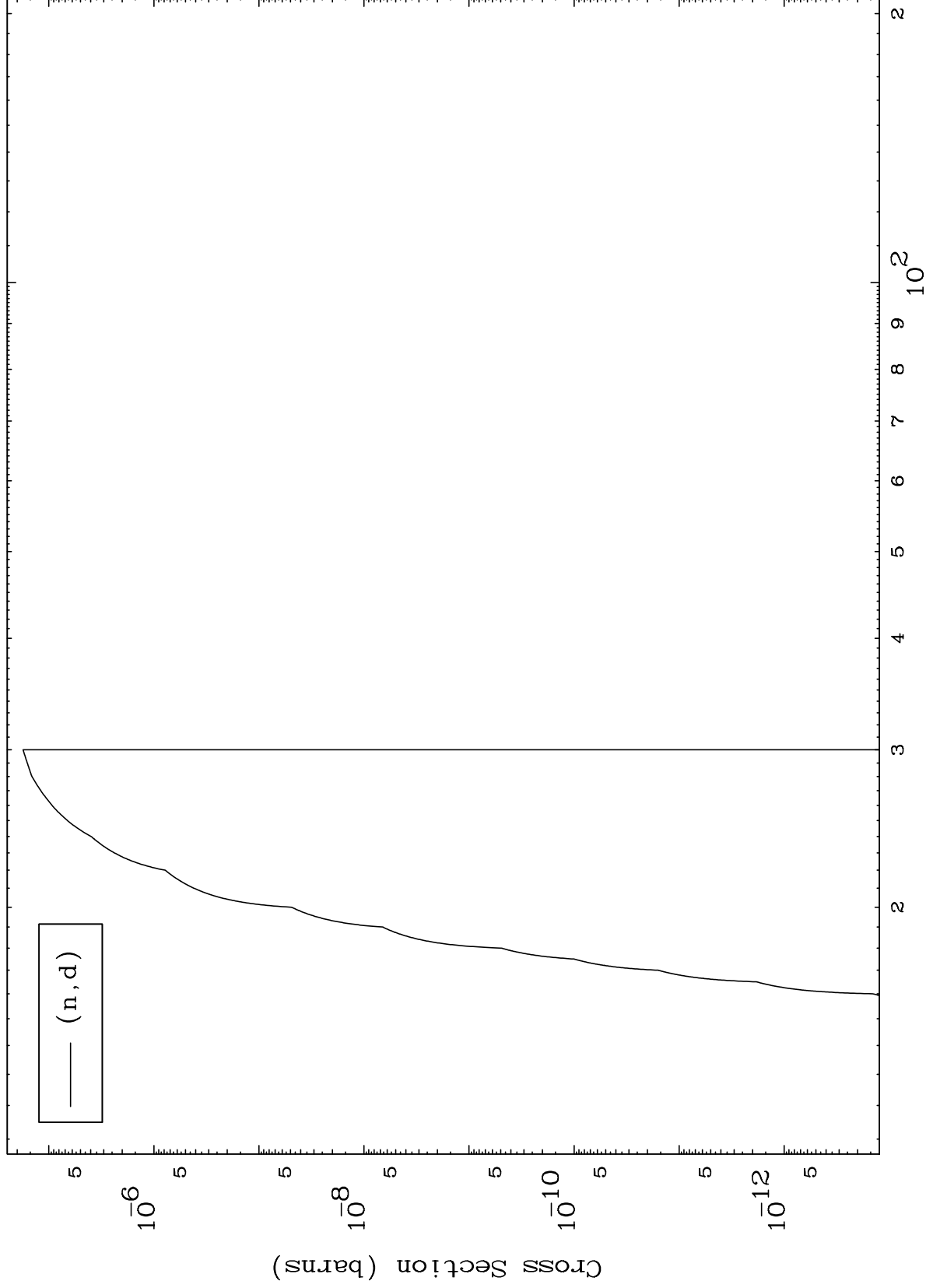
Incident Energy (MeV)

92-U -236

MAT 9231

(γ, d) Levels
0 Kelvin Cross Sections

92-U -236



8

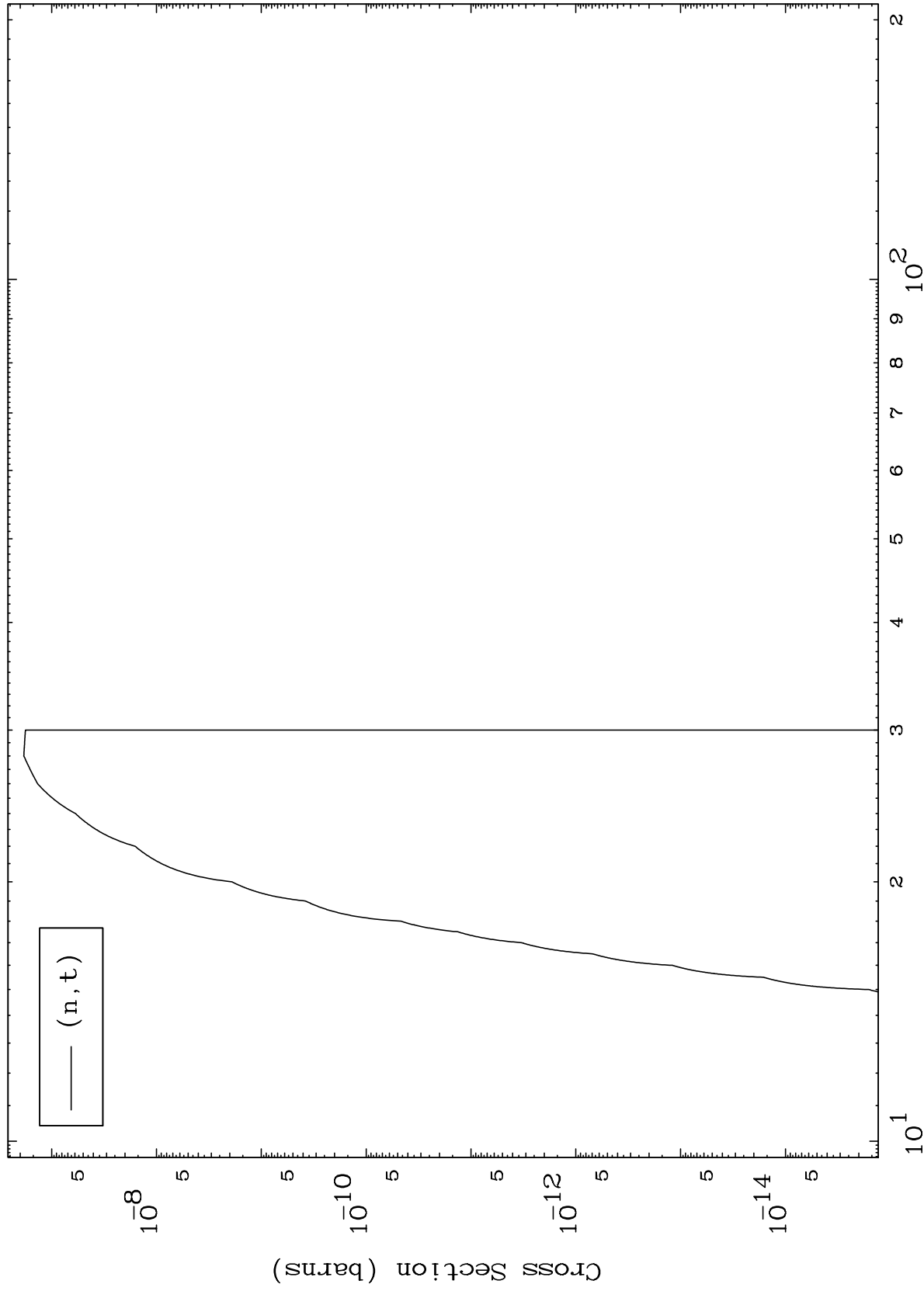
Incident Energy (MeV)

92-U -236

MAT 9231

92-U -236

(γ, t) Levels
0 Kelvin Cross Sections



Incident Energy (MeV)

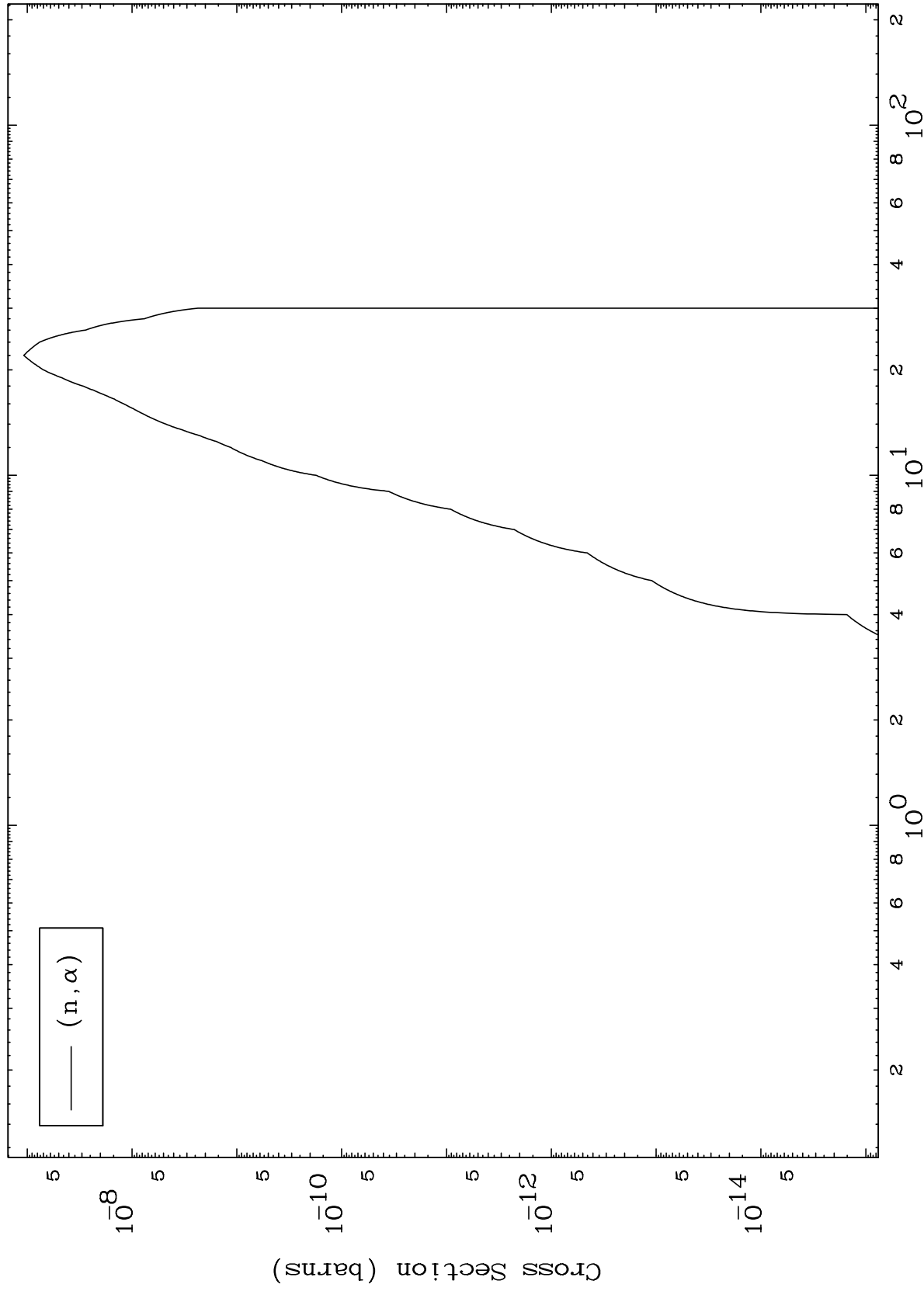
92-U -236

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(γ, α) Levels

92-U -236

0 Kelvin Cross Sections



10

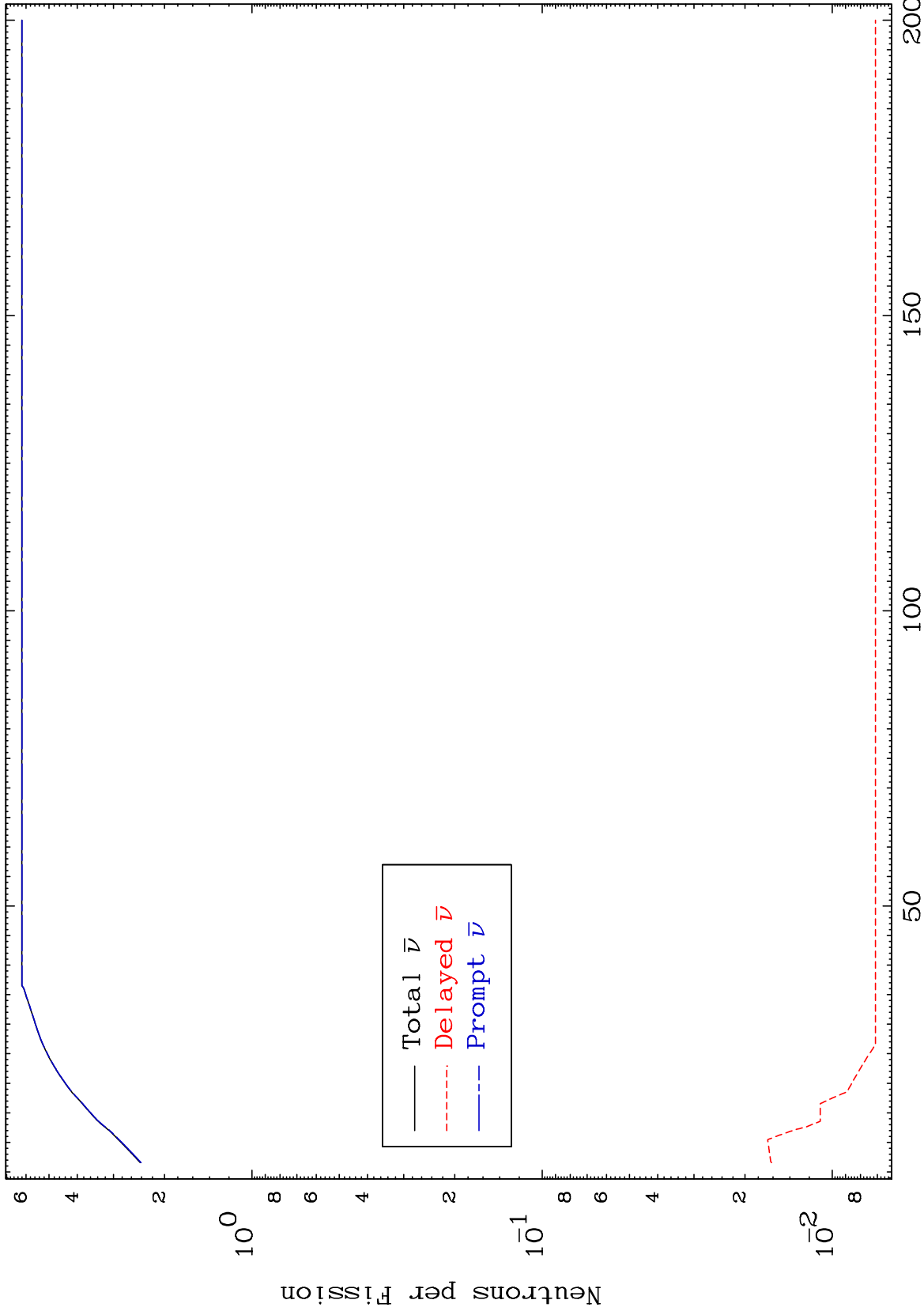
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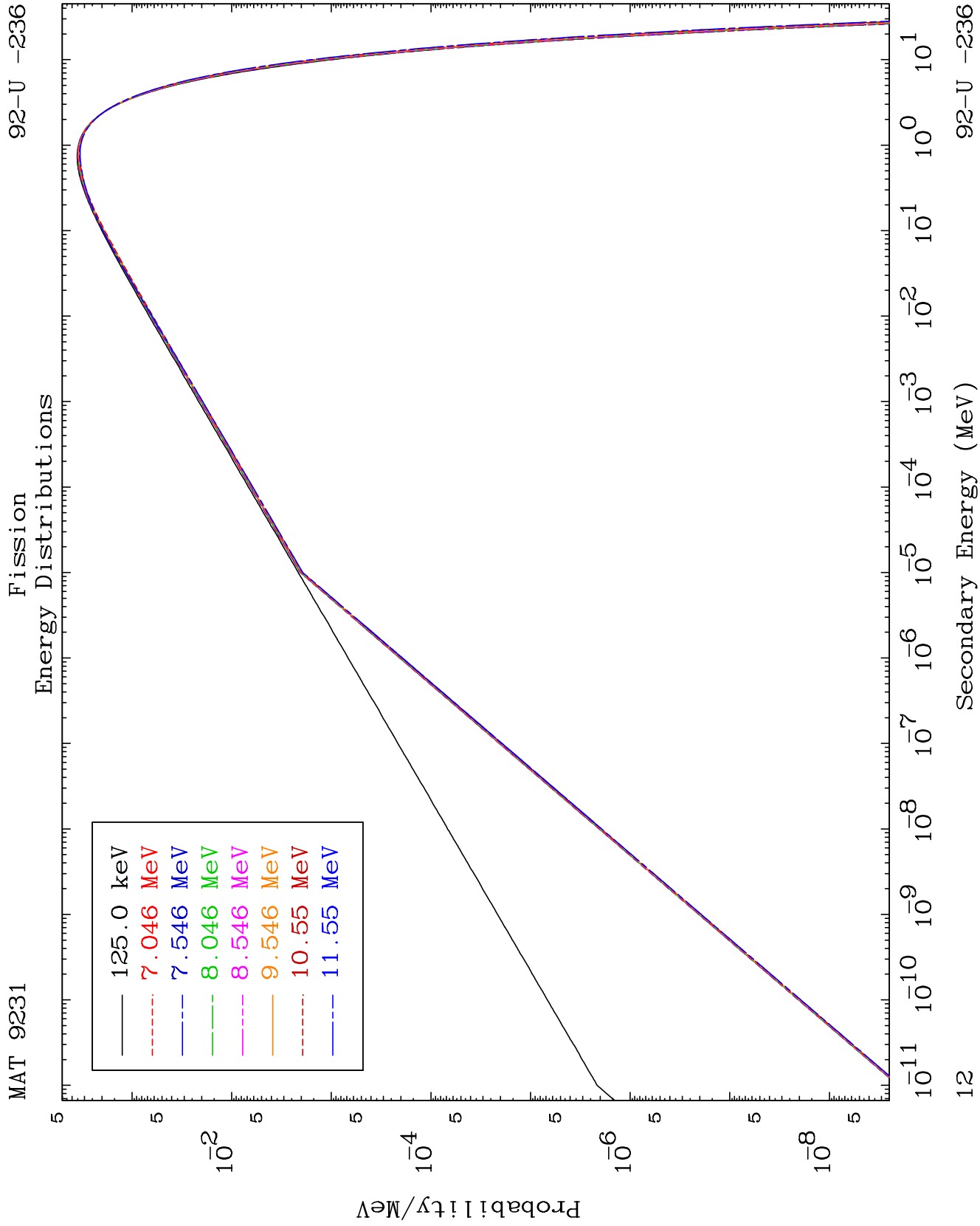
92-U -236

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Photon Neutrons
per Fission

92-U -236

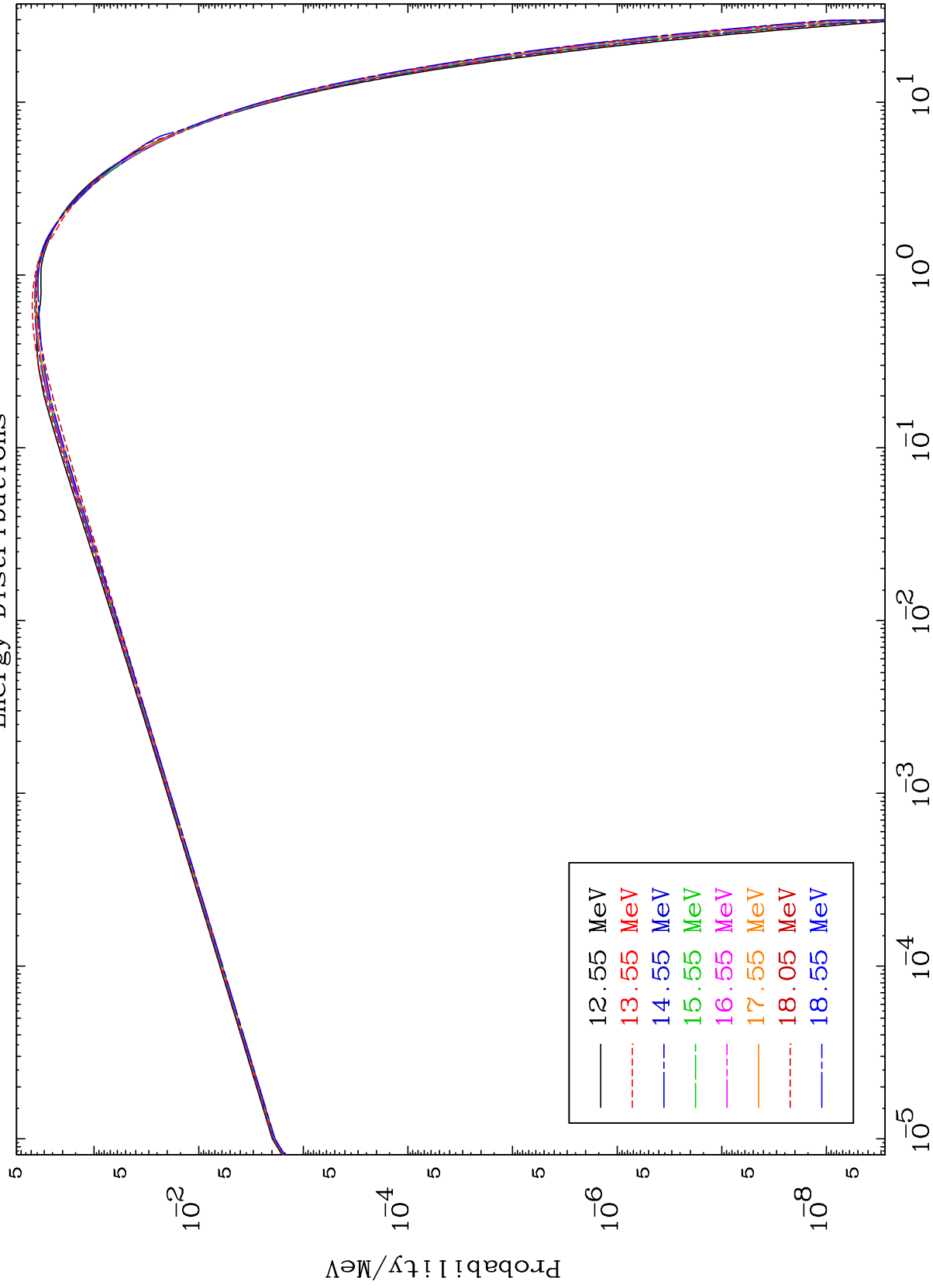




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Fission
Energy Distributions

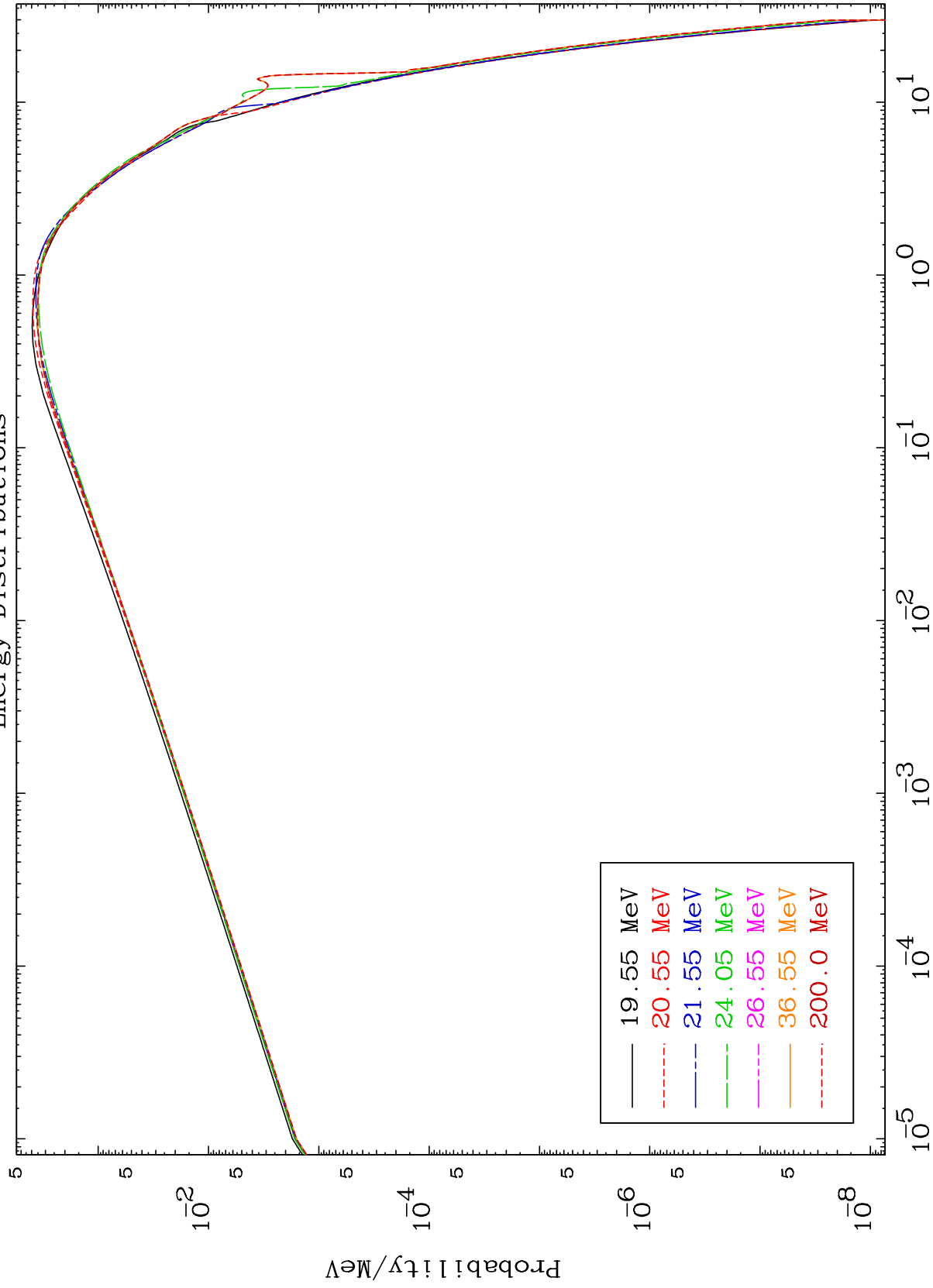
92-U -236



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Fission
Energy Distributions

92-U -236



14

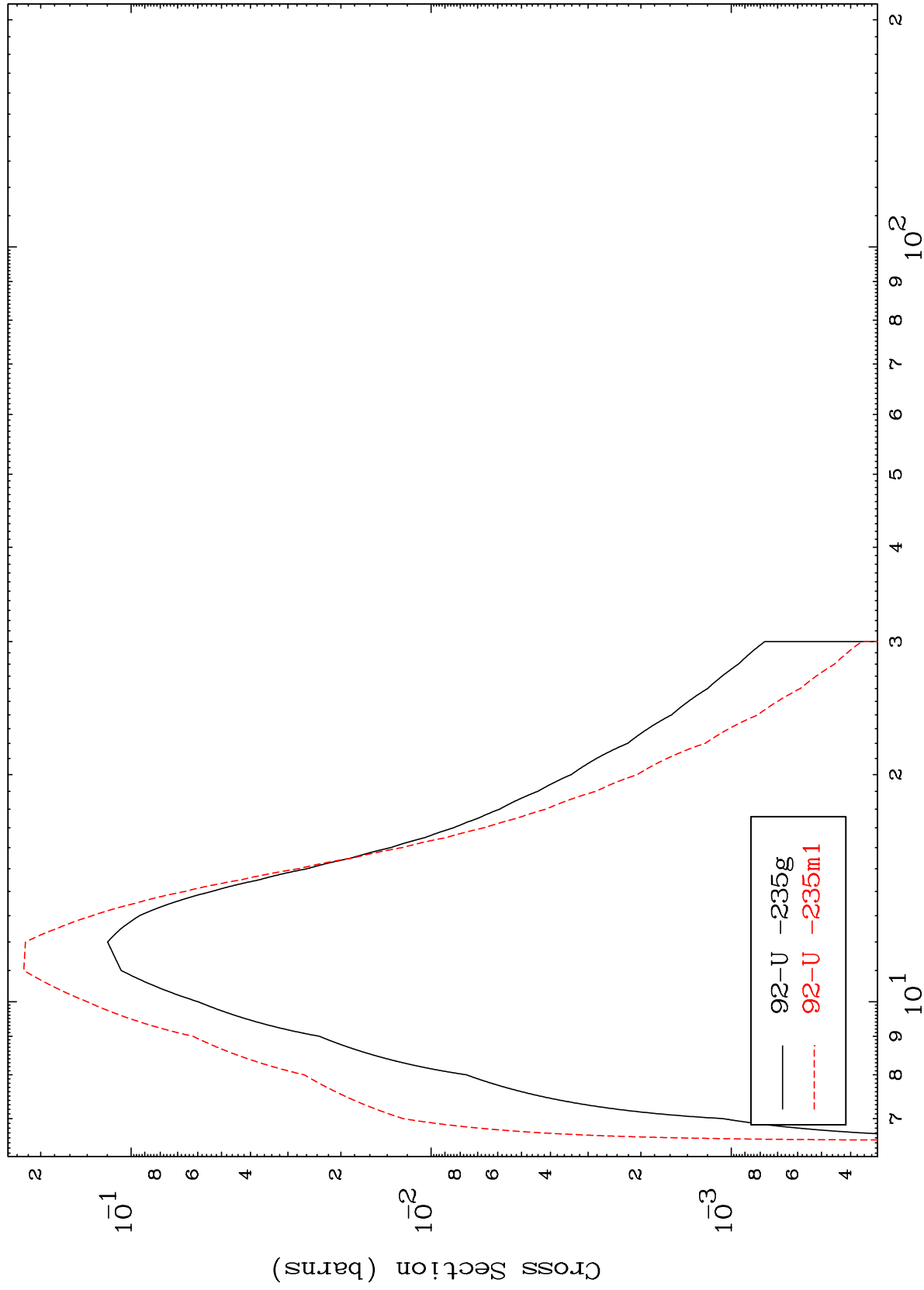
Secondary Energy (MeV)

92-U -236

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92-U -236

Inelastic
Radionuclide Production Cross Section



15

Incident Energy (MeV)

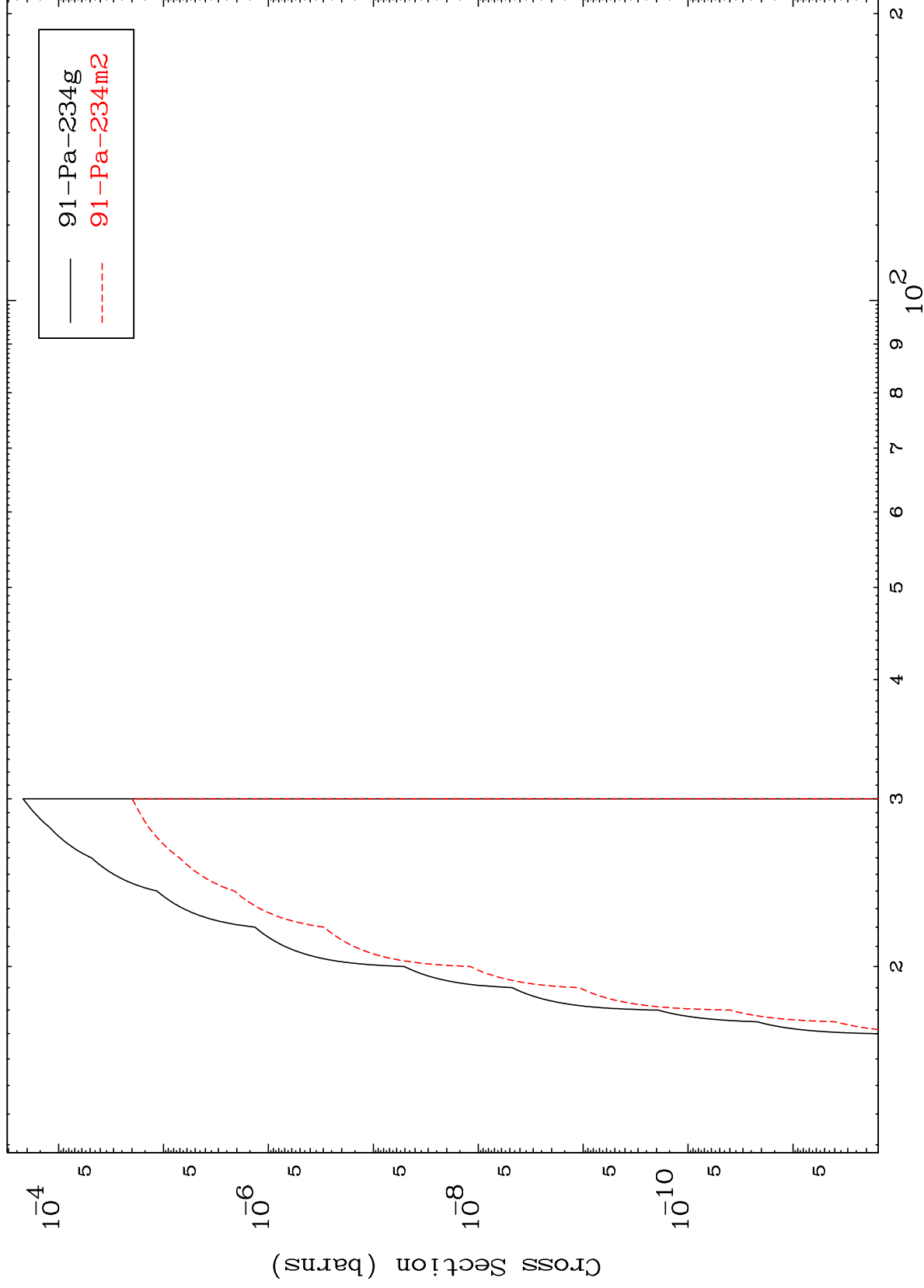
92-U -236

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(n,n') p

92-U -236

Radionuclide Production Cross Section



16

Incident Energy (MeV)

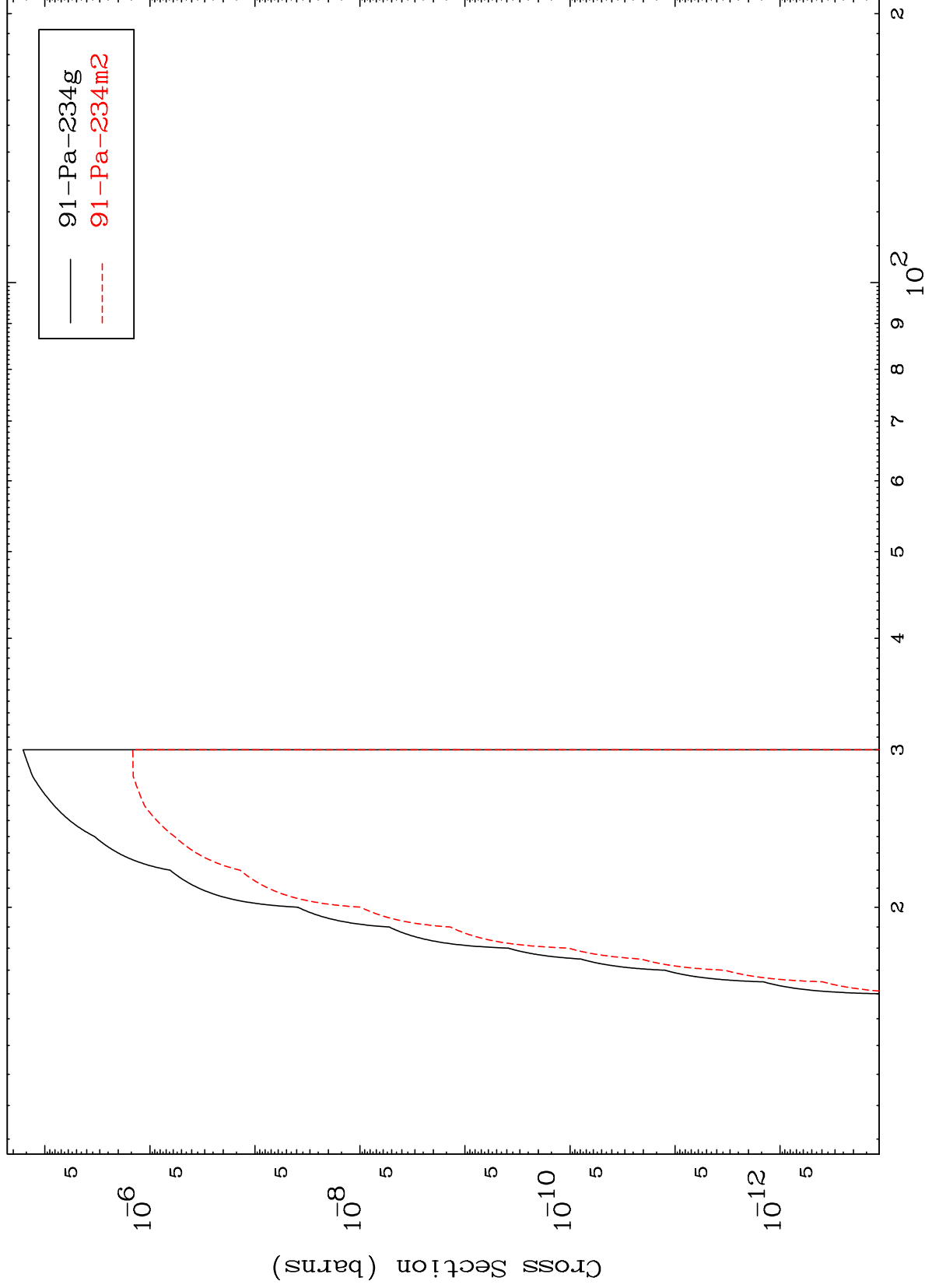
92-U -236

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(n,d)

92-U -236

Radionuclide Production Cross Section



17

Incident Energy (MeV)

92-U -236