

Program EVALPLOT
(Version 2021-1)

by

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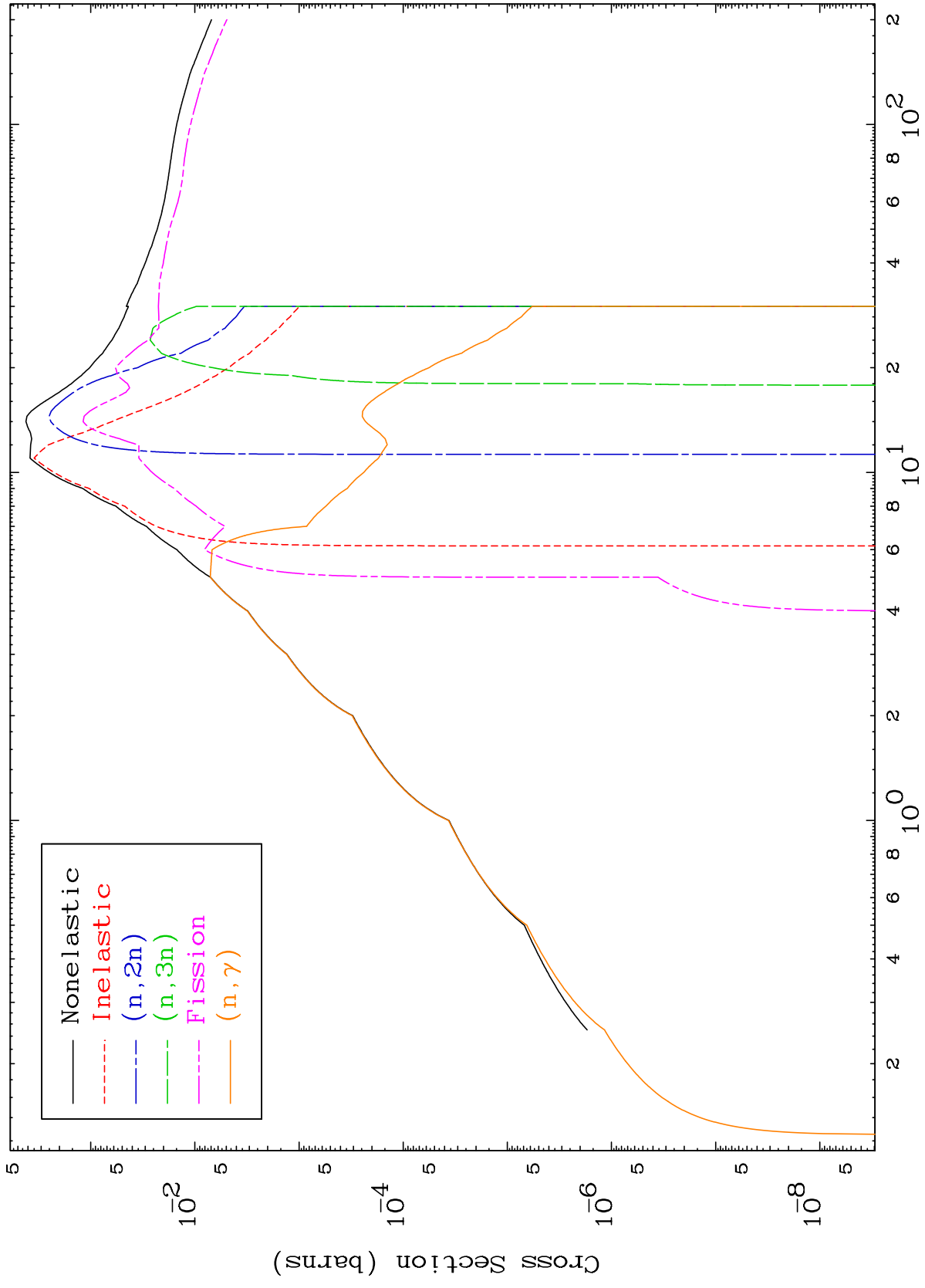
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

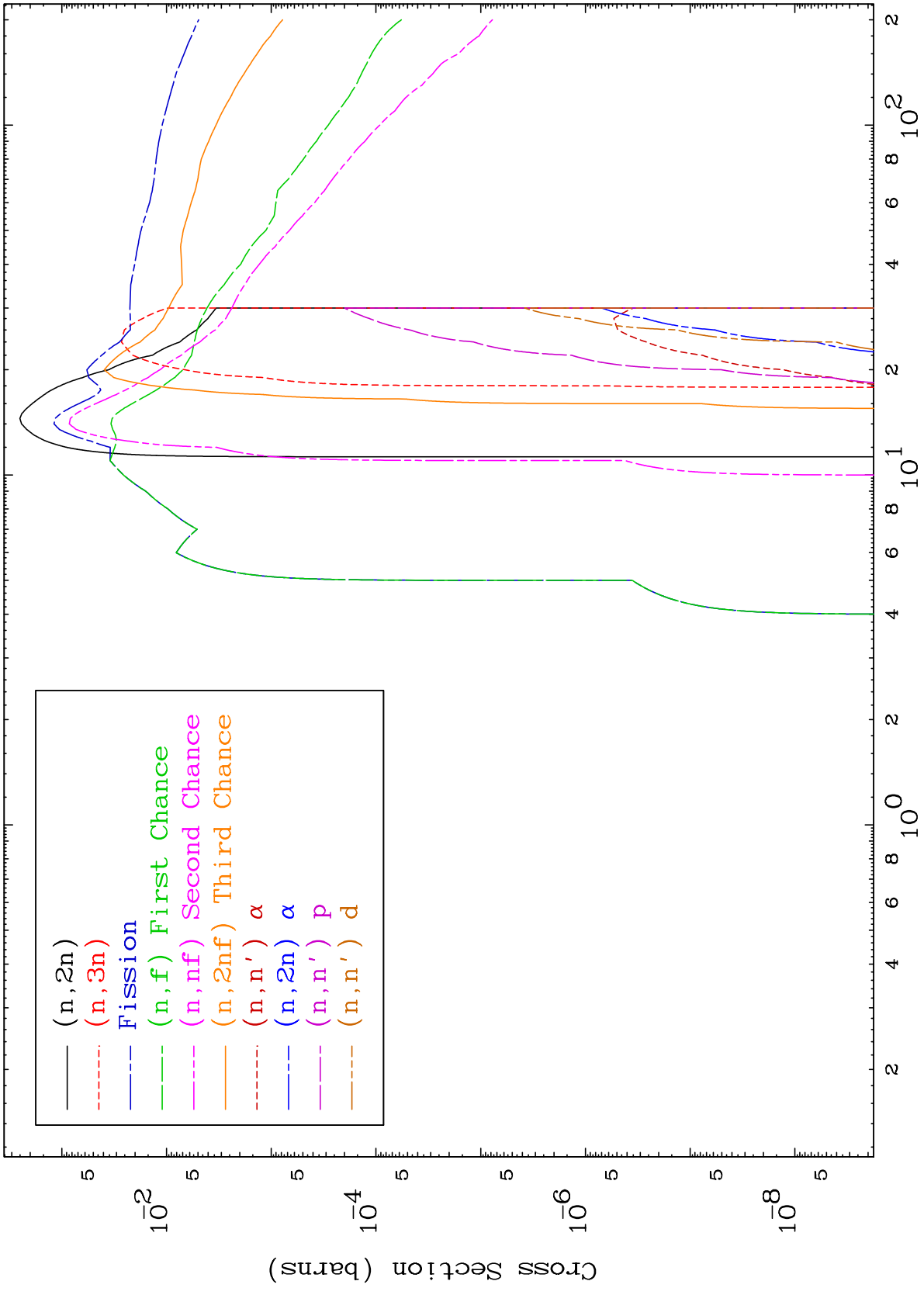
Press Mouse Button to Start

MAT 9237

0 Kelvin Major
Photon Cross Sections

92-U -238

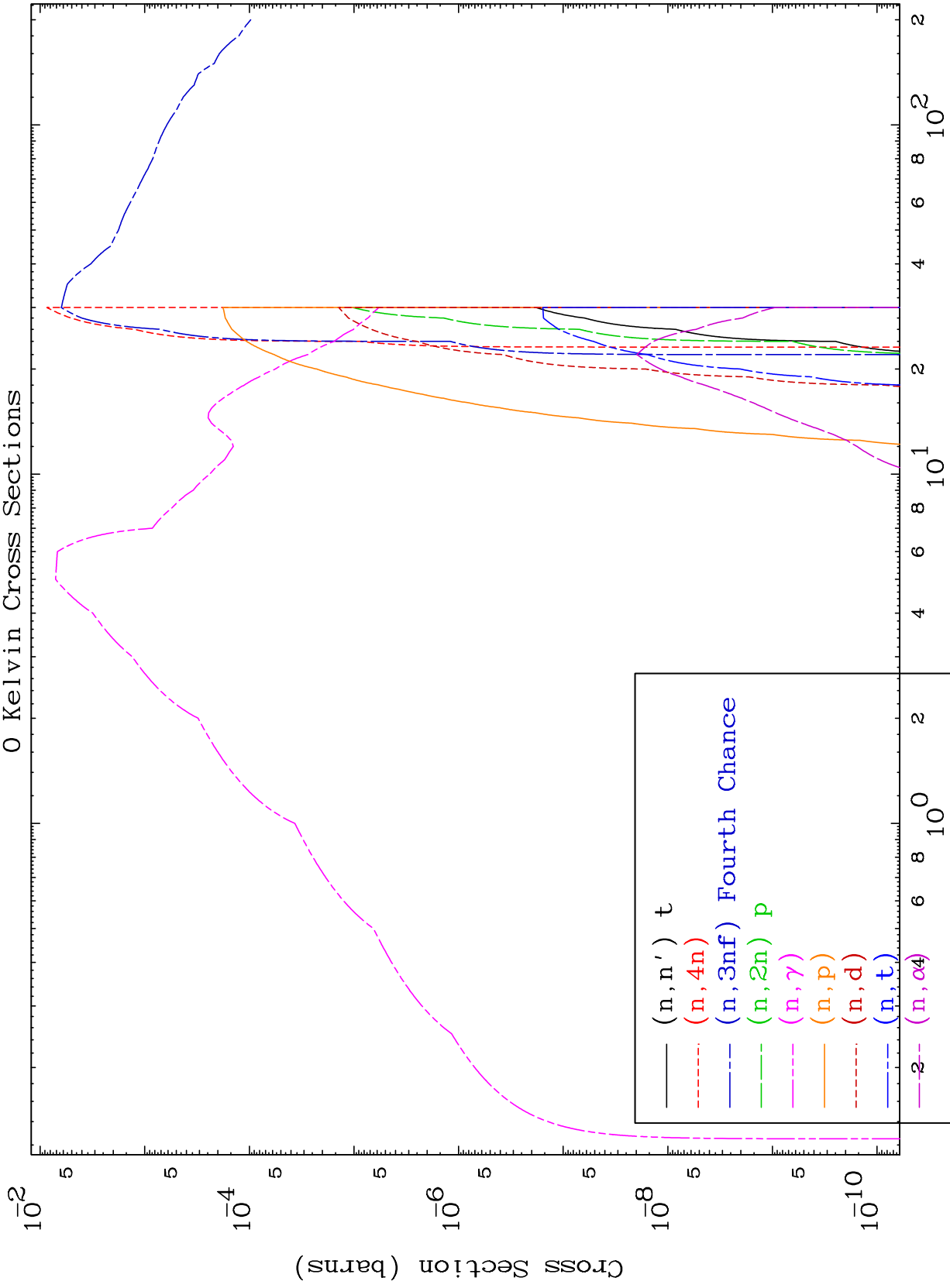




MAT 9237

Photon Neutron Absorption
0 Kelvin Cross Sections

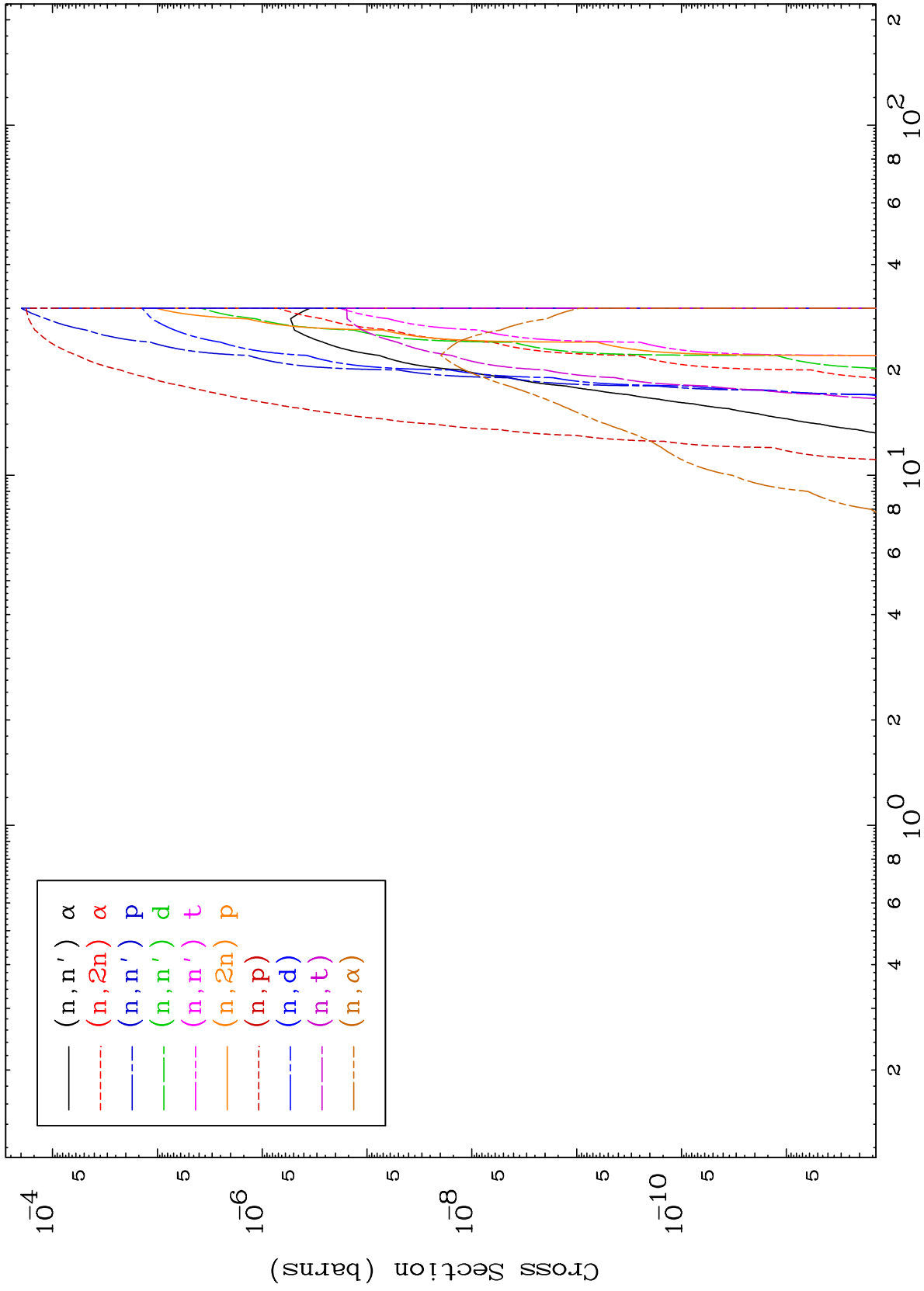
92-U -238

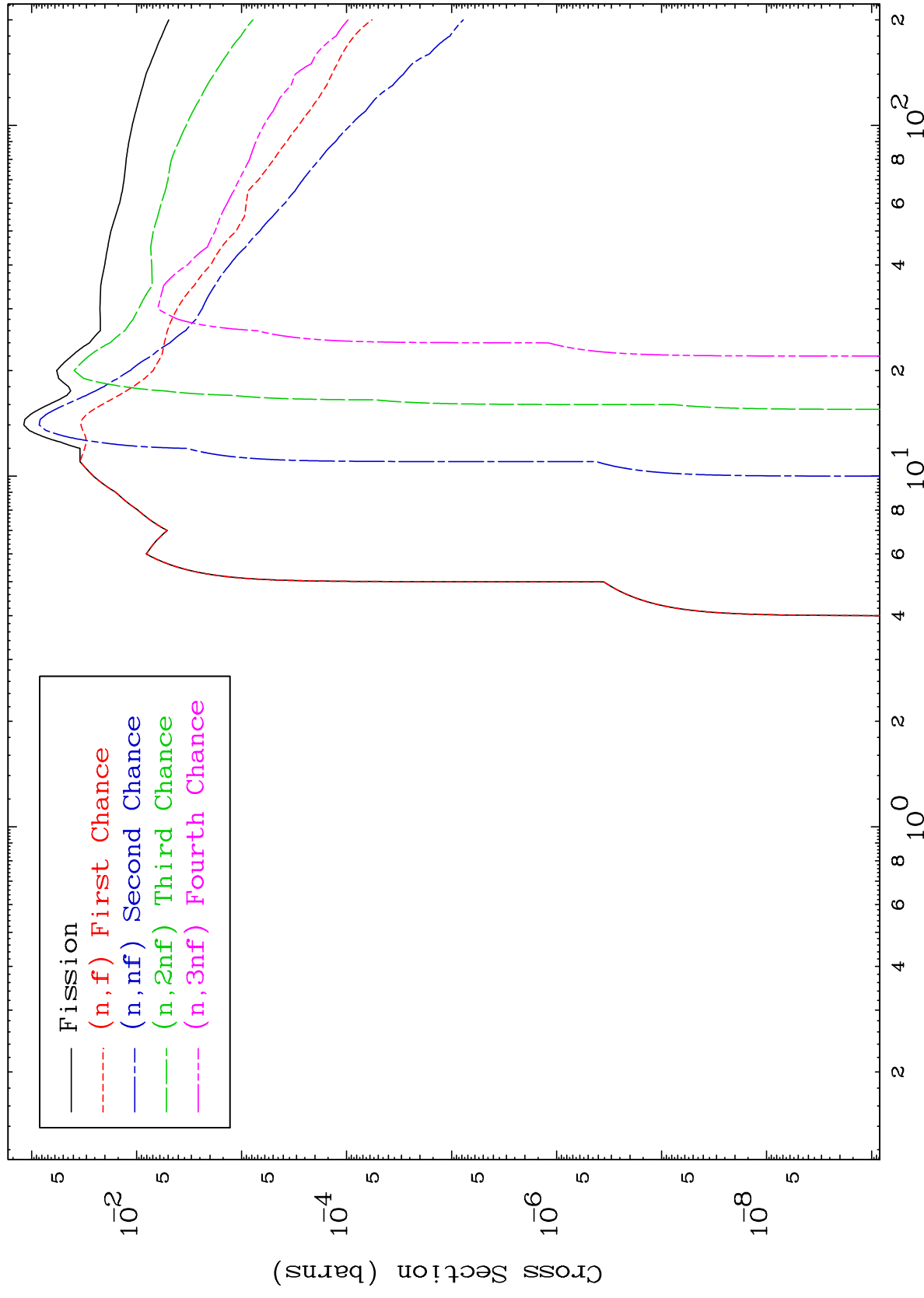


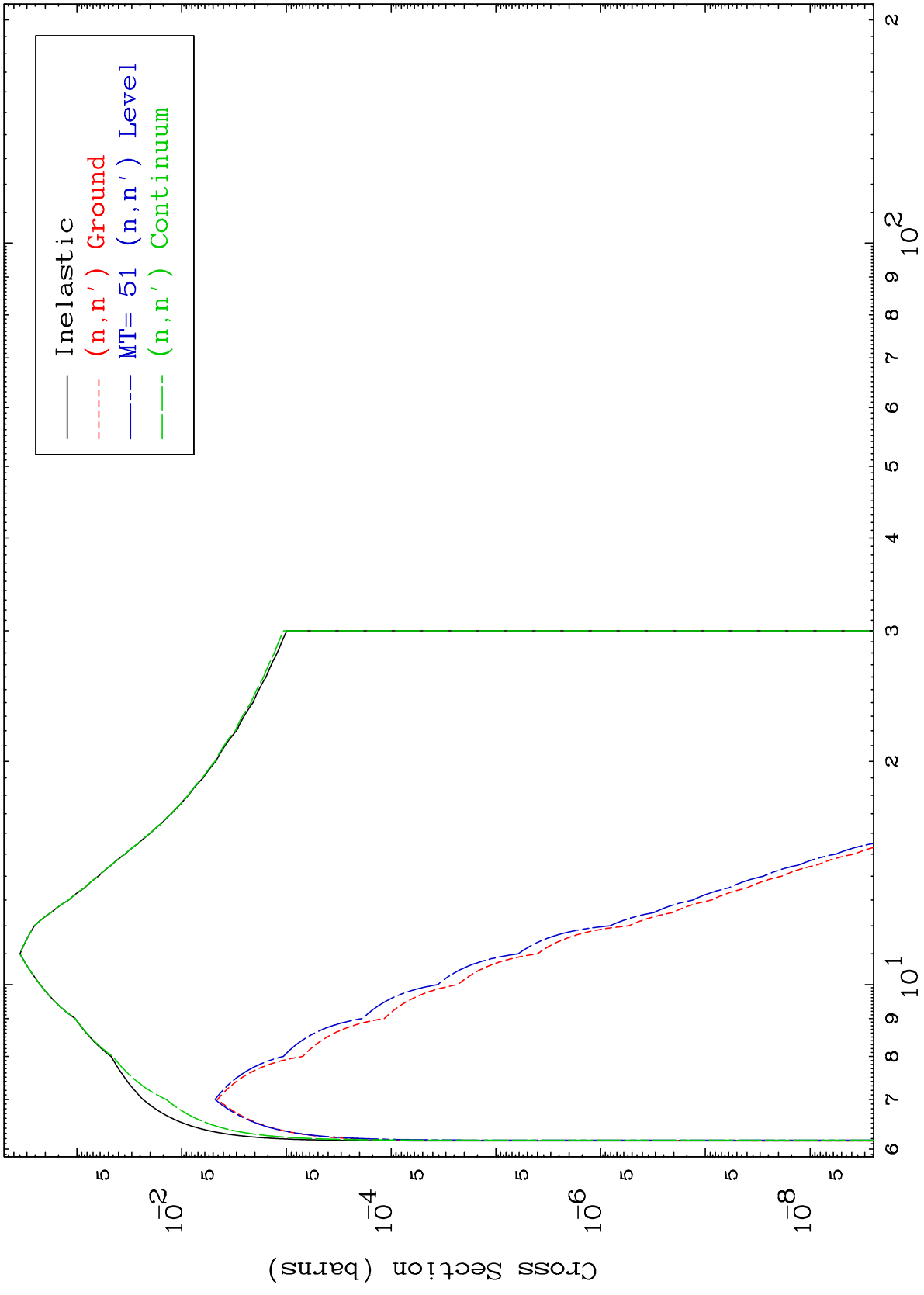
3

Incident Energy (MeV)

92-U -238



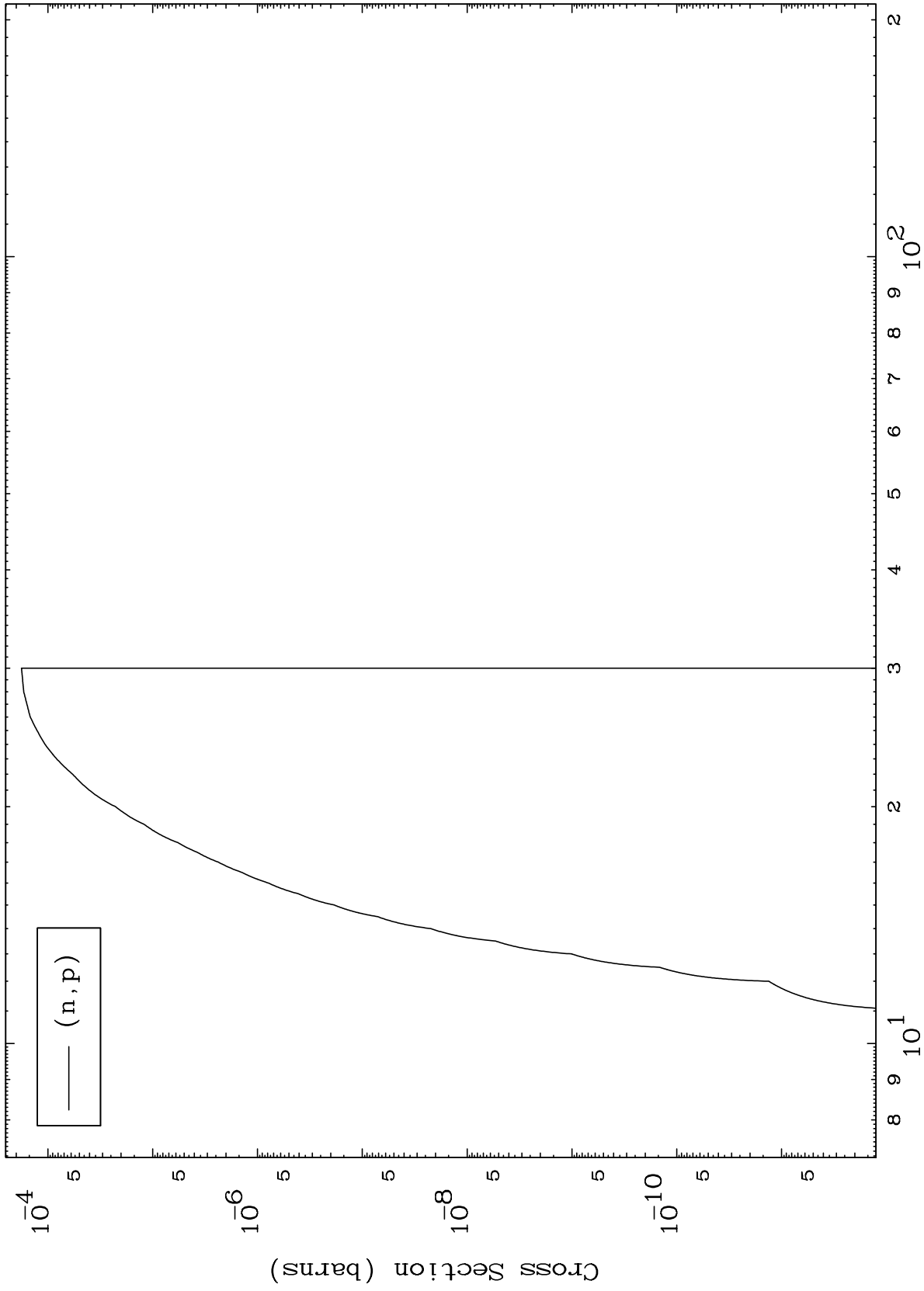




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(γ, p) Levels
0 Kelvin Cross Sections

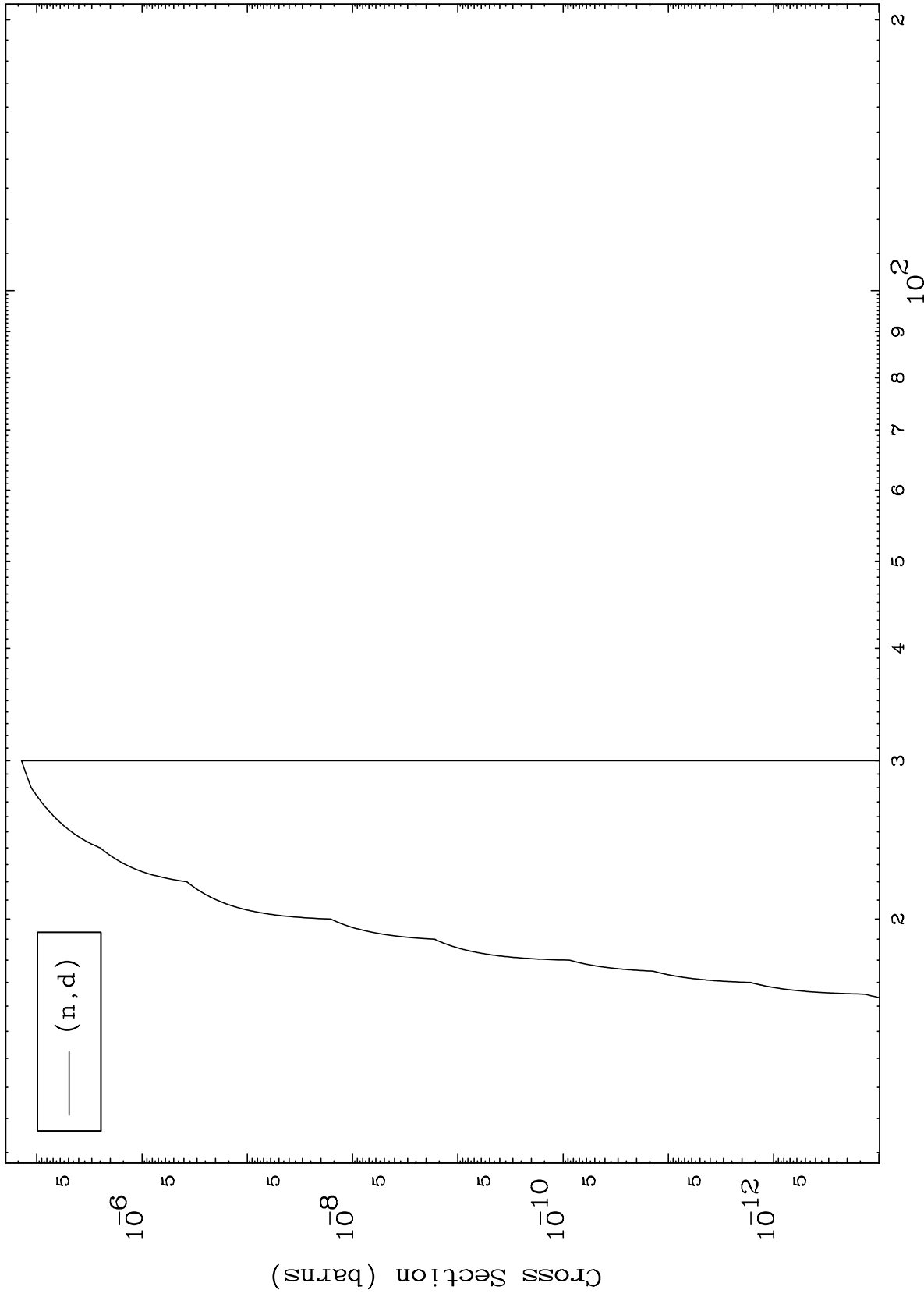
92-U -238



7

Incident Energy (MeV)

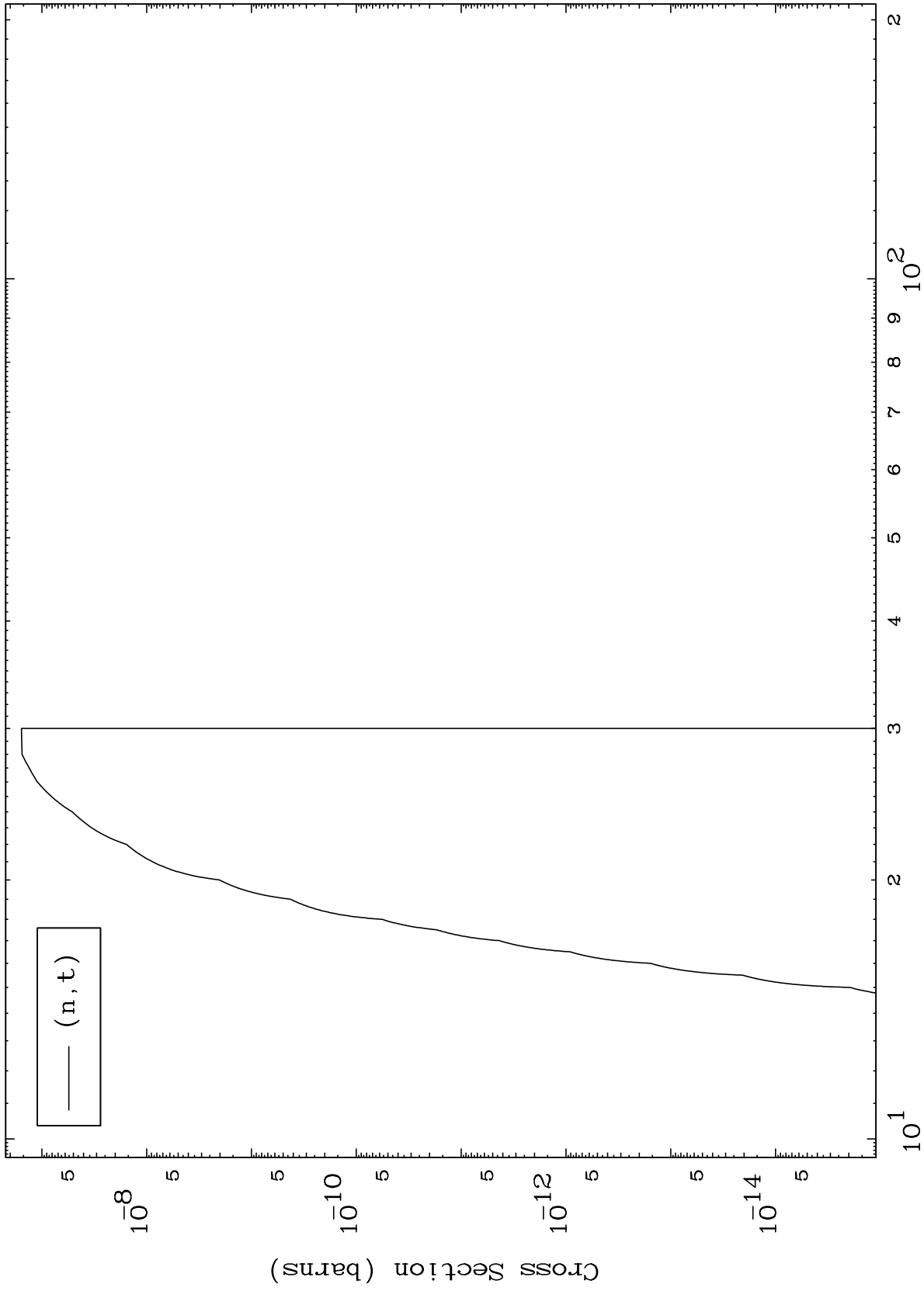
92-U -238



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(γ, t) Levels
0 Kelvin Cross Sections

92-U -238



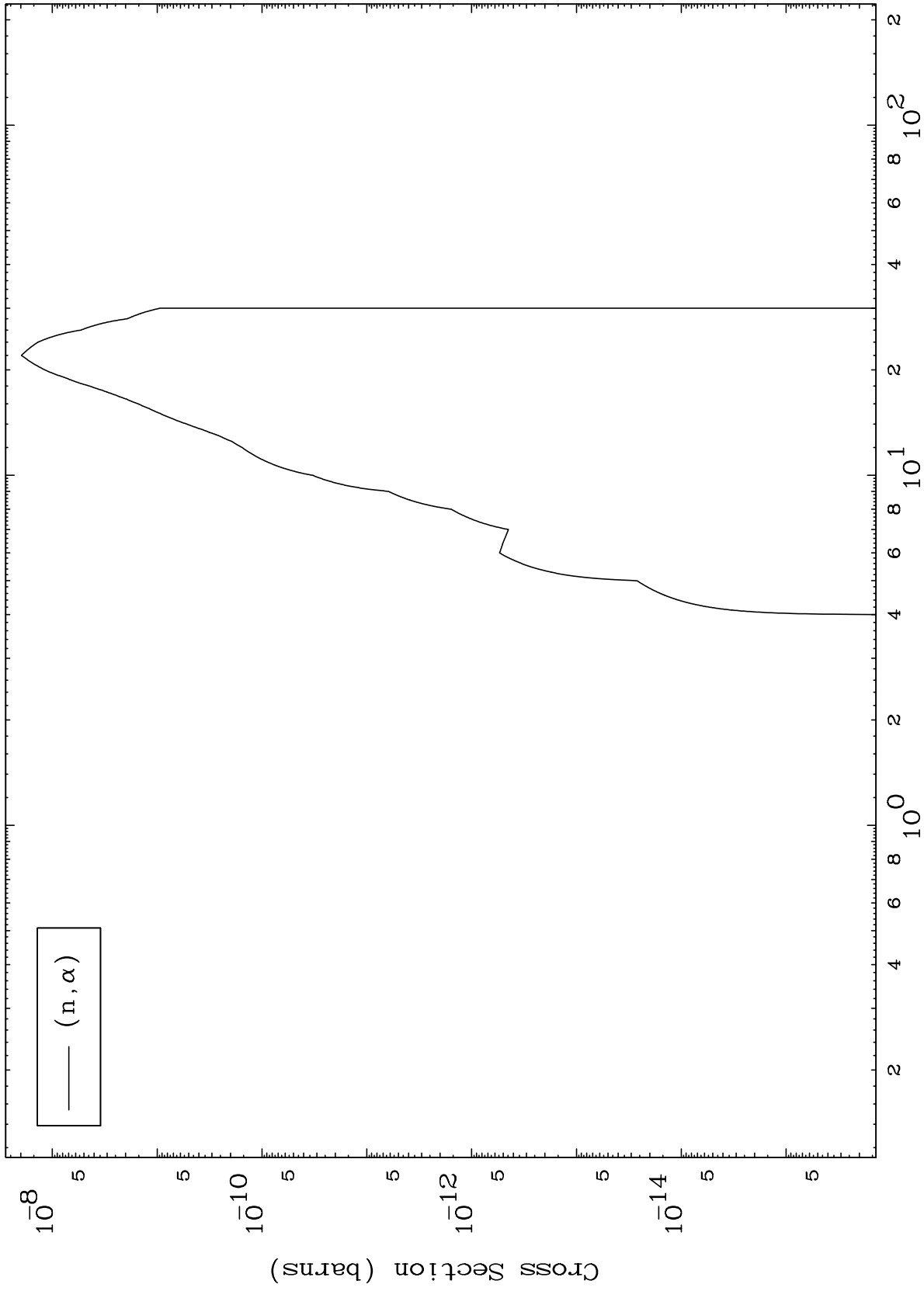
Incident Energy (MeV)

92-U -238

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(γ, α) Levels
0 Kelvin Cross Sections

92-U -238



10

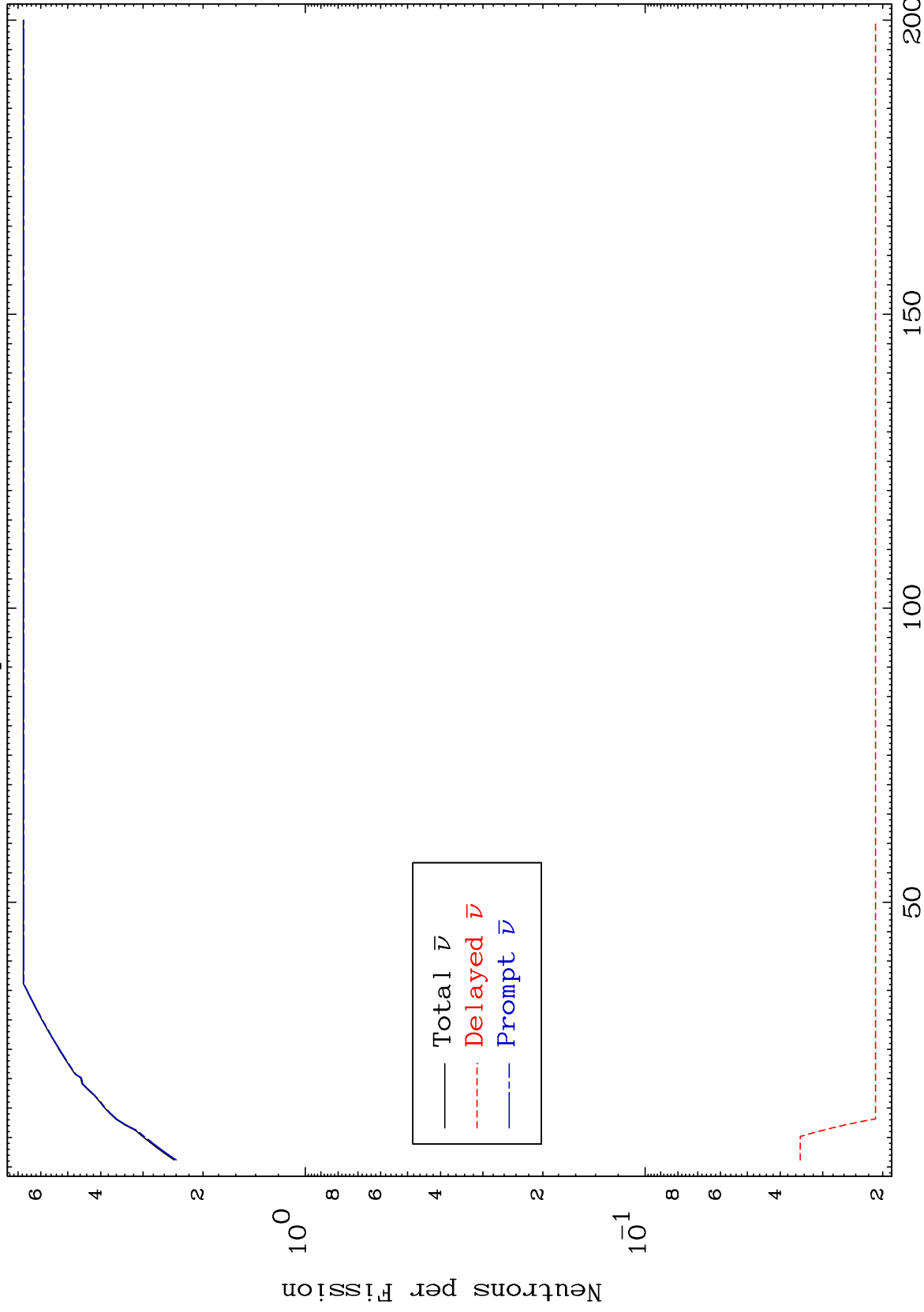
Incident Energy (MeV)

92-U -238

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Photon Neutrons
per Fission

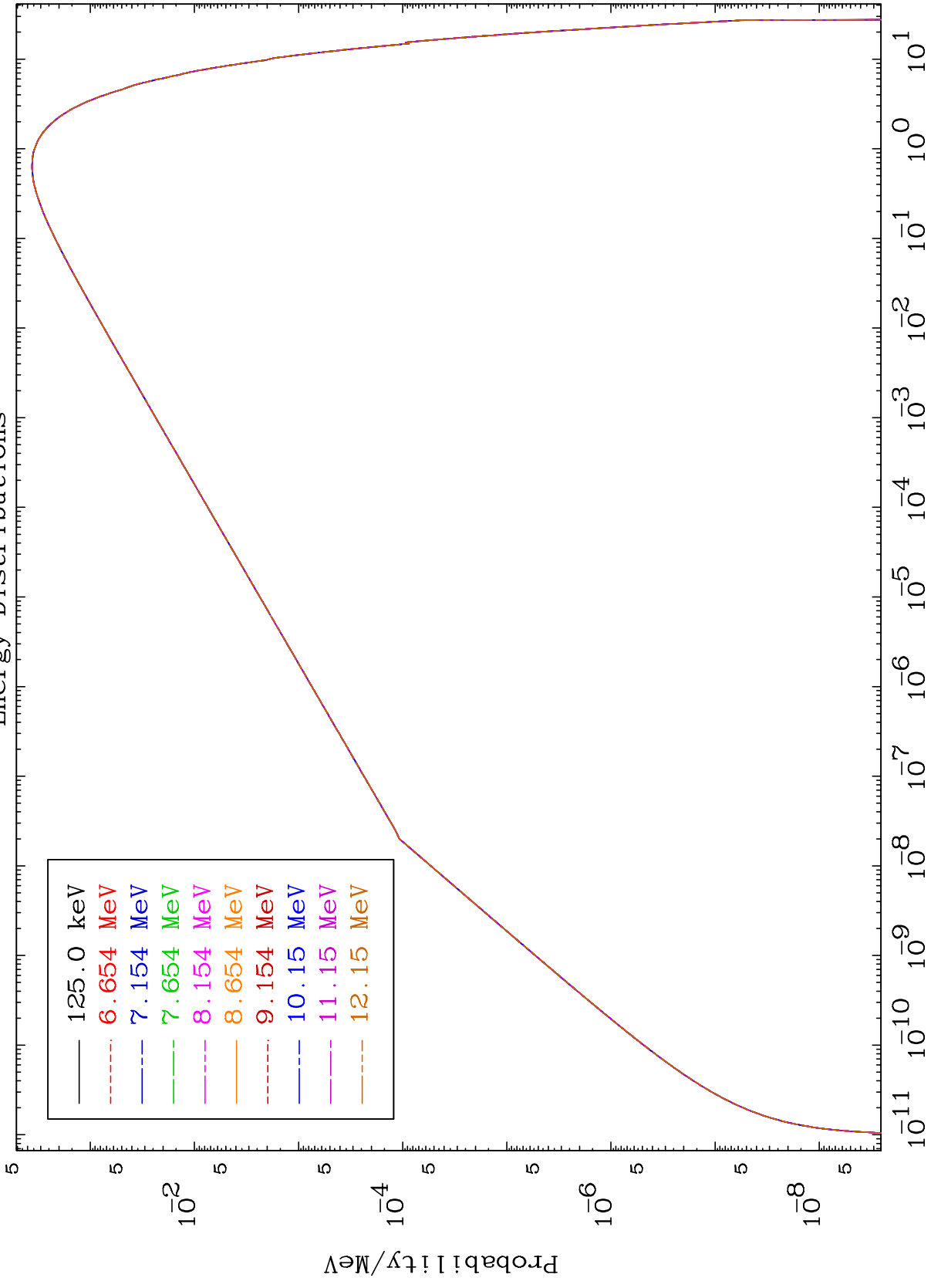
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Fission Energy Distributions

92-U -238



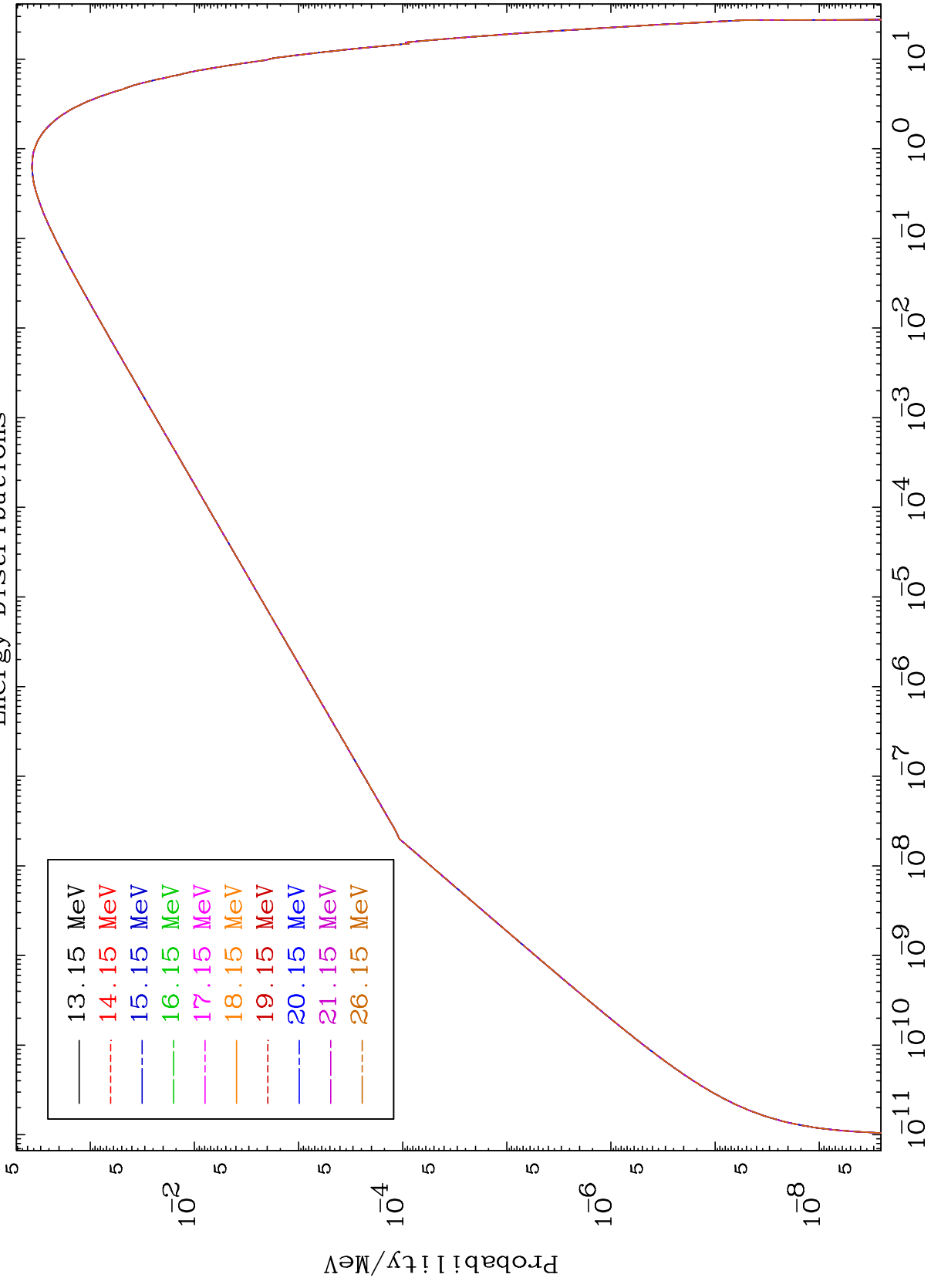
92-U -238

Secondary Energy (MeV)

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Fission Energy Distributions

92-U -238



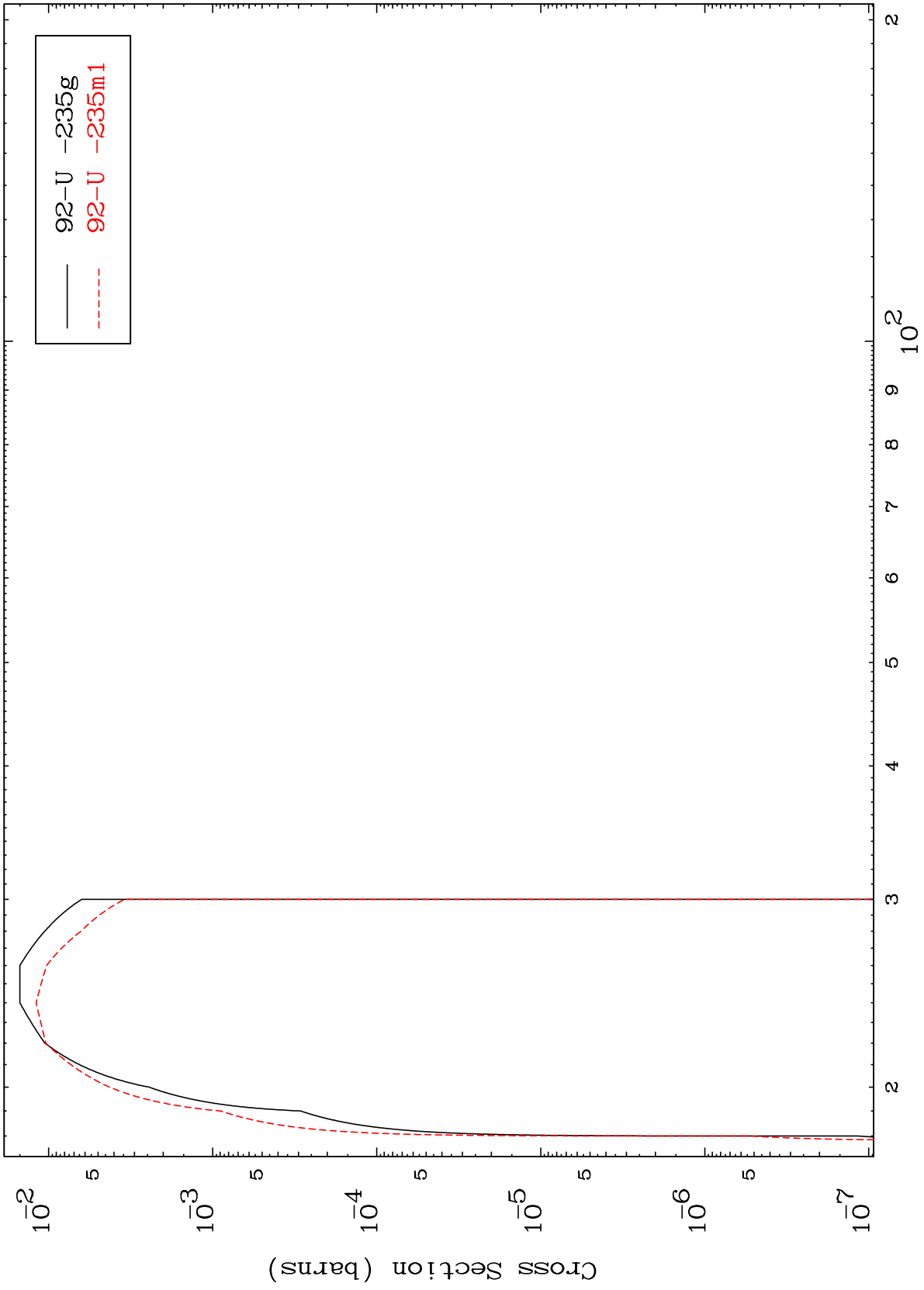
92-U -238

Secondary Energy (MeV)

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92-U -238

(n,3n)
Radionuclide Production Cross Section



14

Incident Energy (MeV)

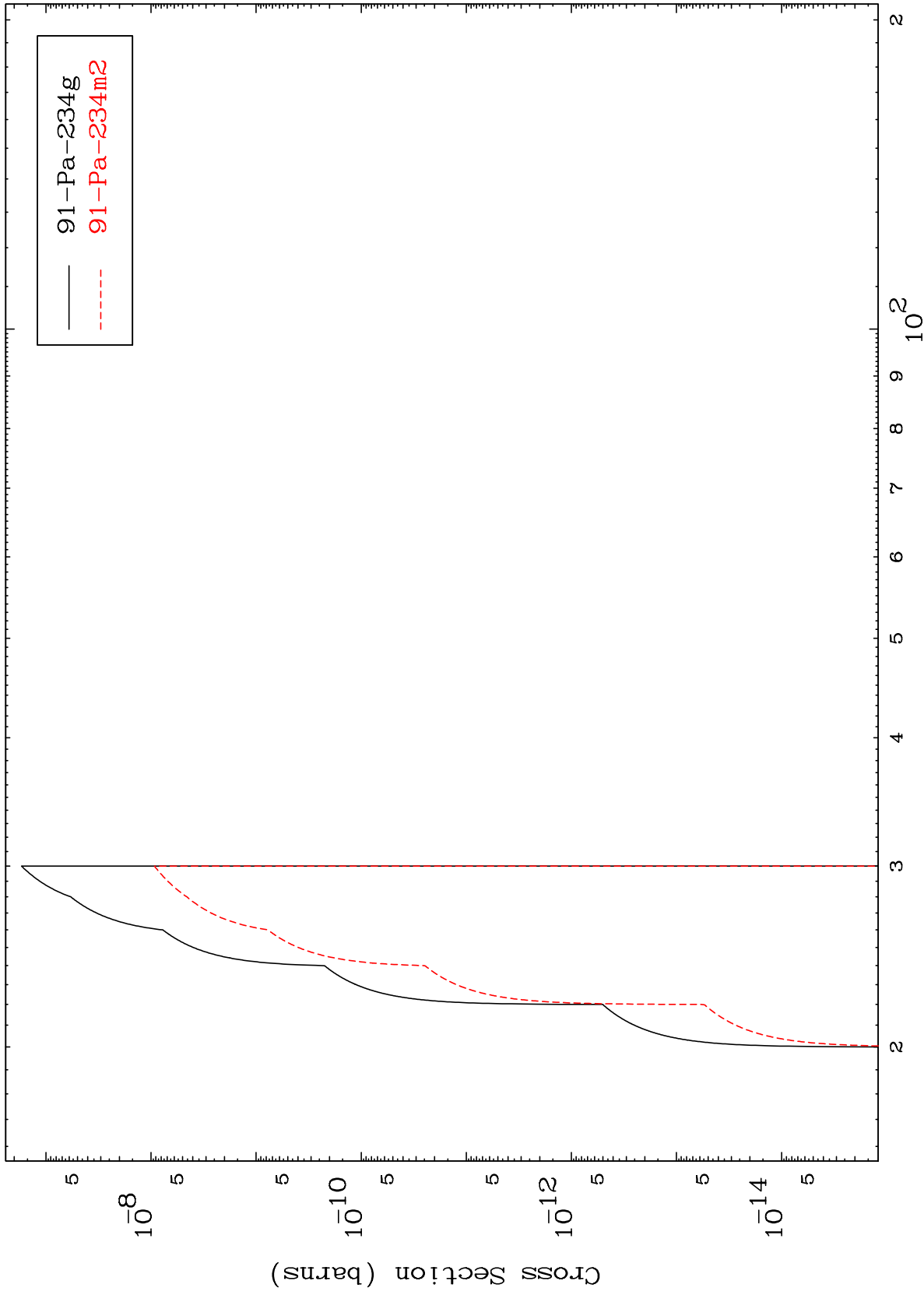
92-U -238

MAT 9237

(n,n') t

92-U -238

Radionuclide Production Cross Section



15

Incident Energy (MeV)

92-U -238