

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

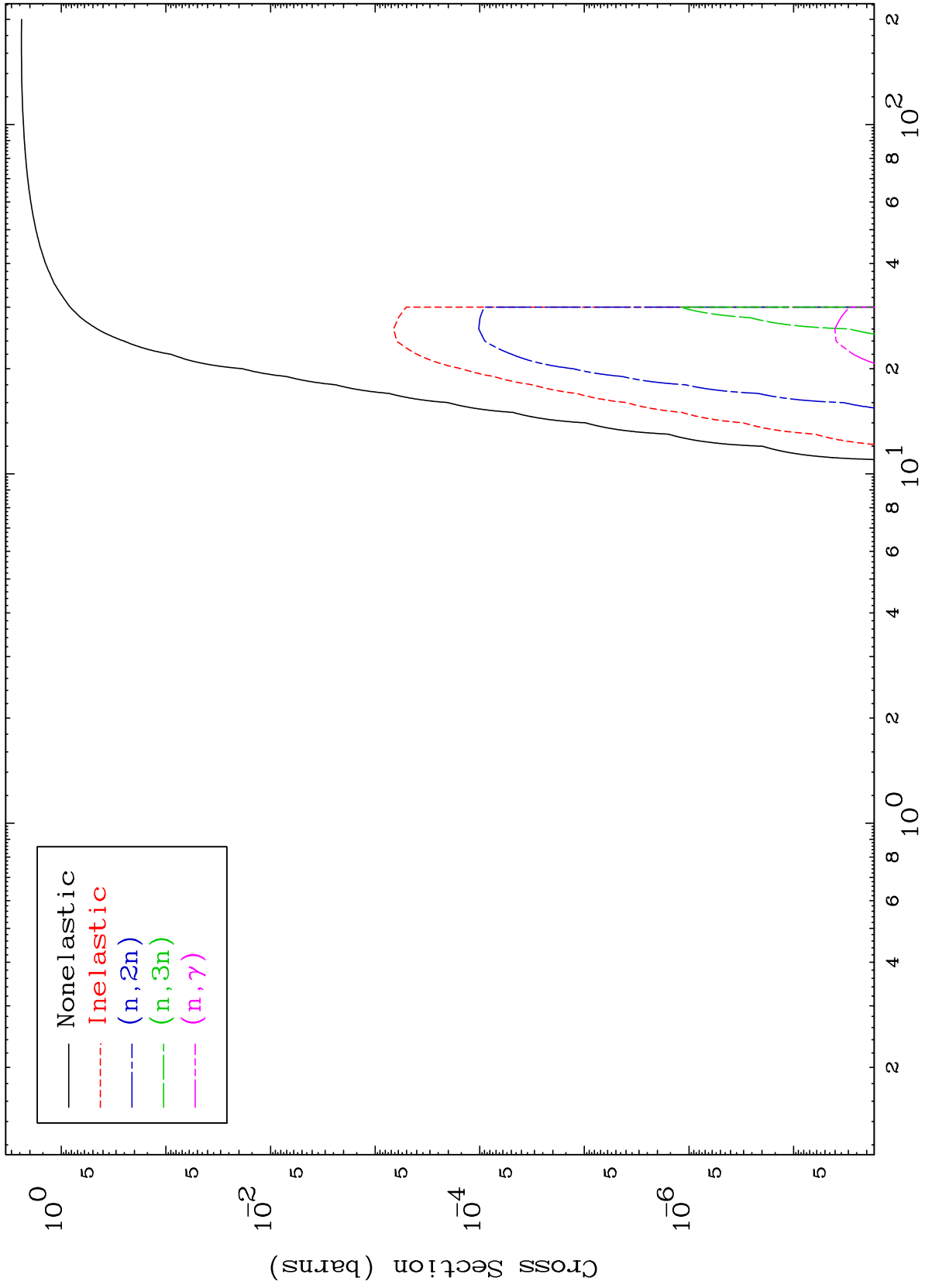
Press Mouse Button to Start

MAT 8523

He-3 Major

85-At-202m

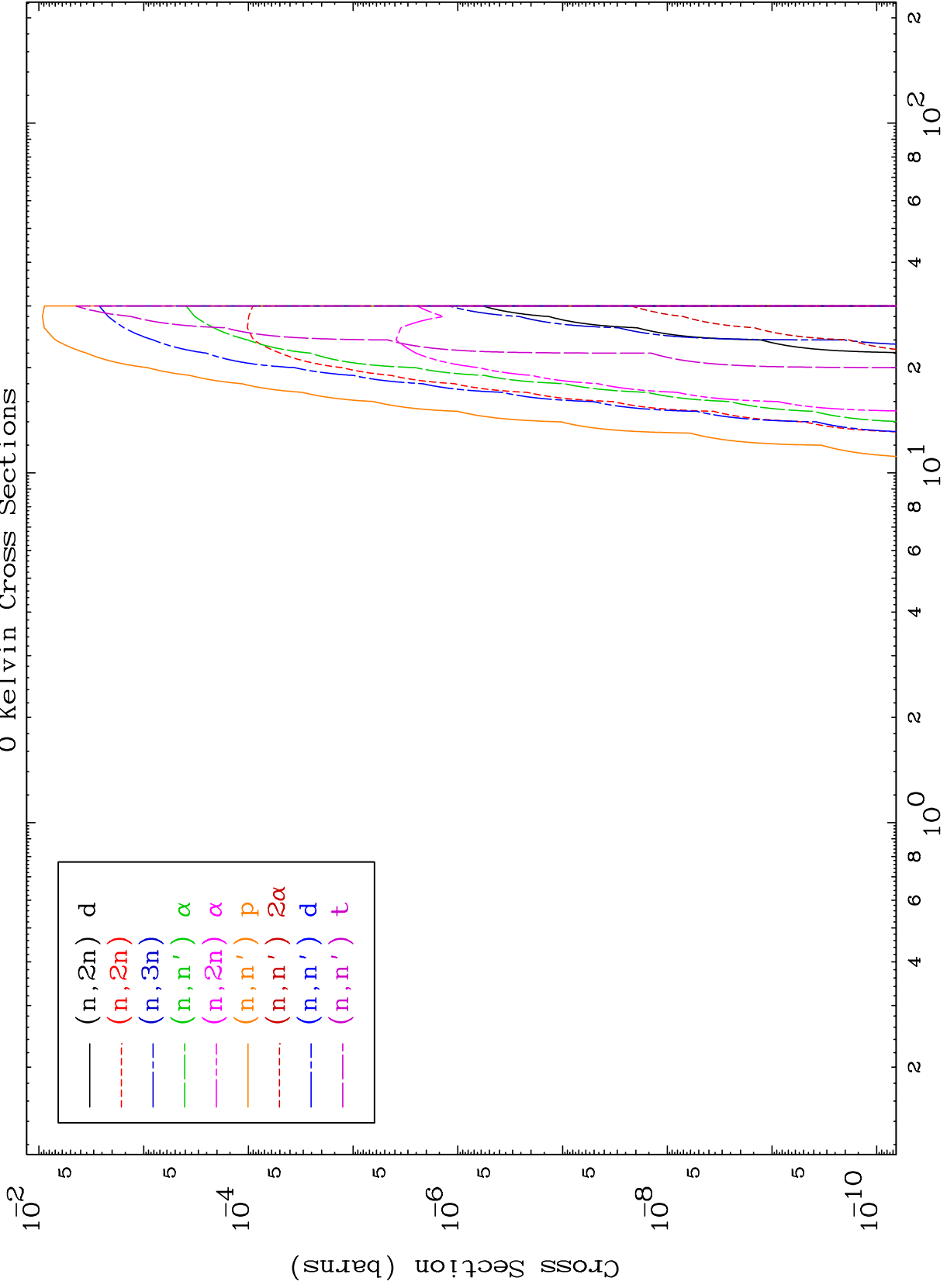
0 Kelvin Cross Sections



MAT 8523

He-3 Neutron Absorption  
0 Kelvin Cross Sections

85-At-202m



Incident Energy (MeV)

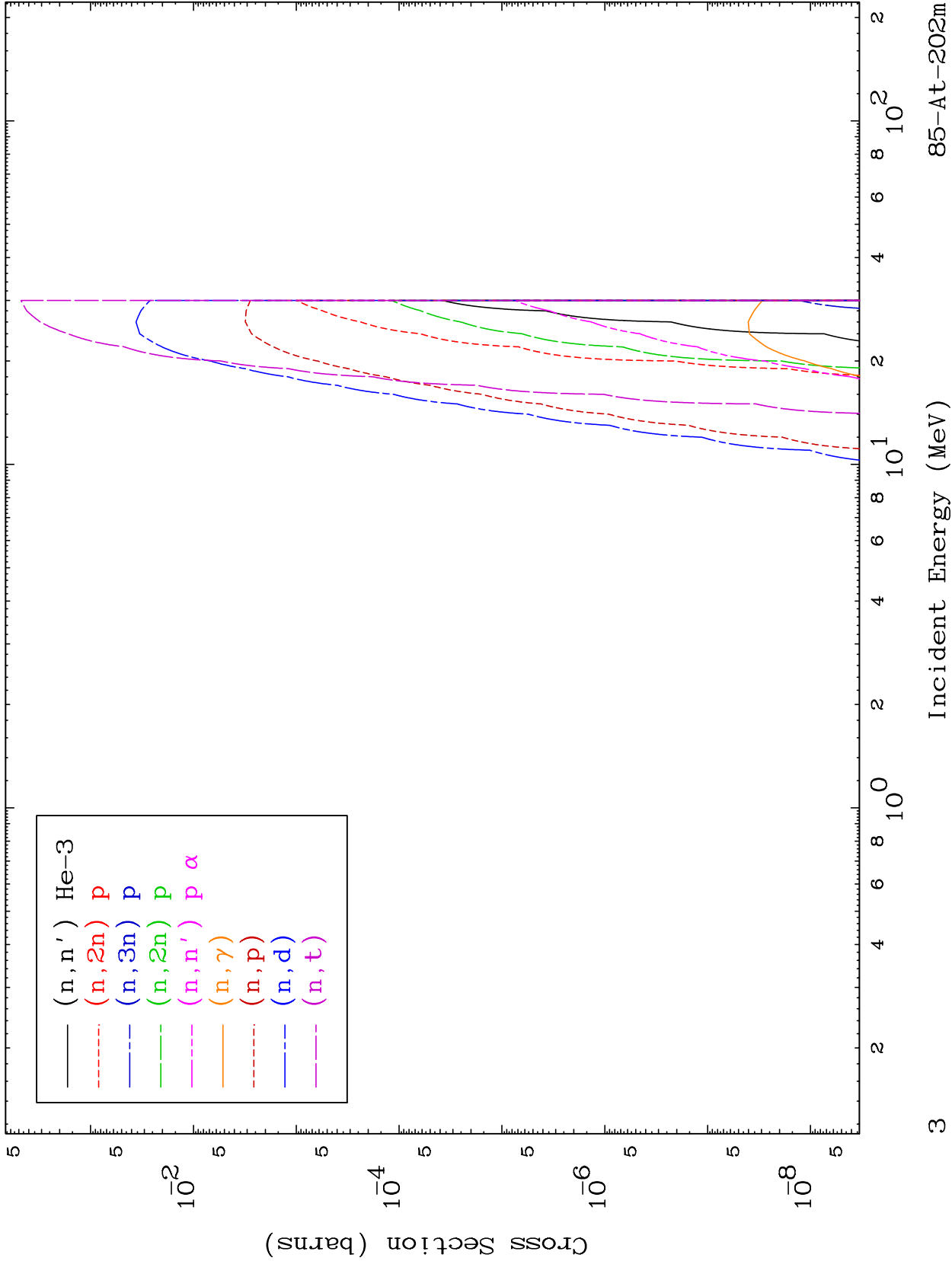
85-At-202m

2

MAT 8523

He-3 Neutron Absorption  
0 Kelvin Cross Sections

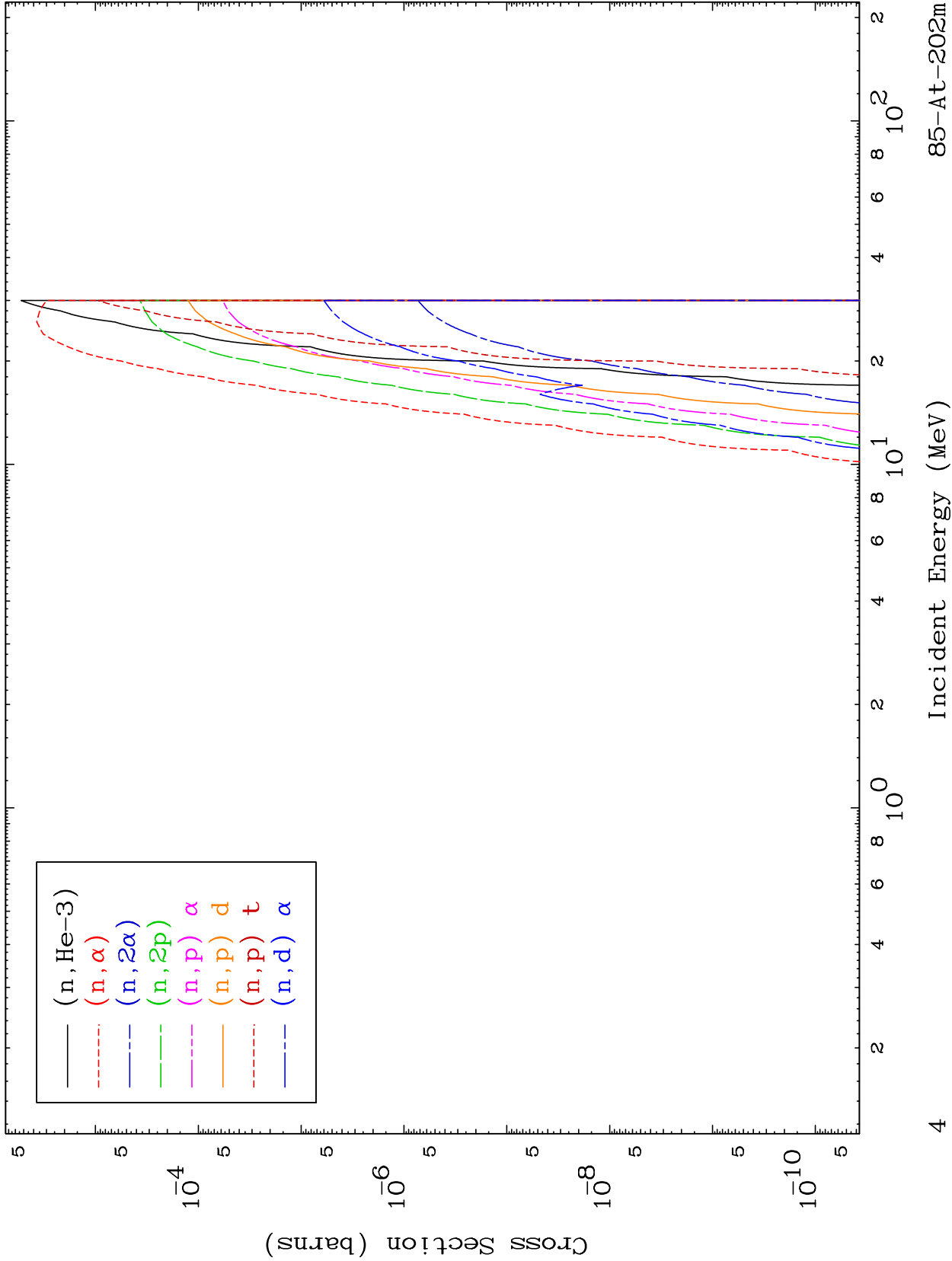
85-At-202m



MAT 8523

He-3 Neutron Absorption  
0 Kelvin Cross Sections

85-At-202m

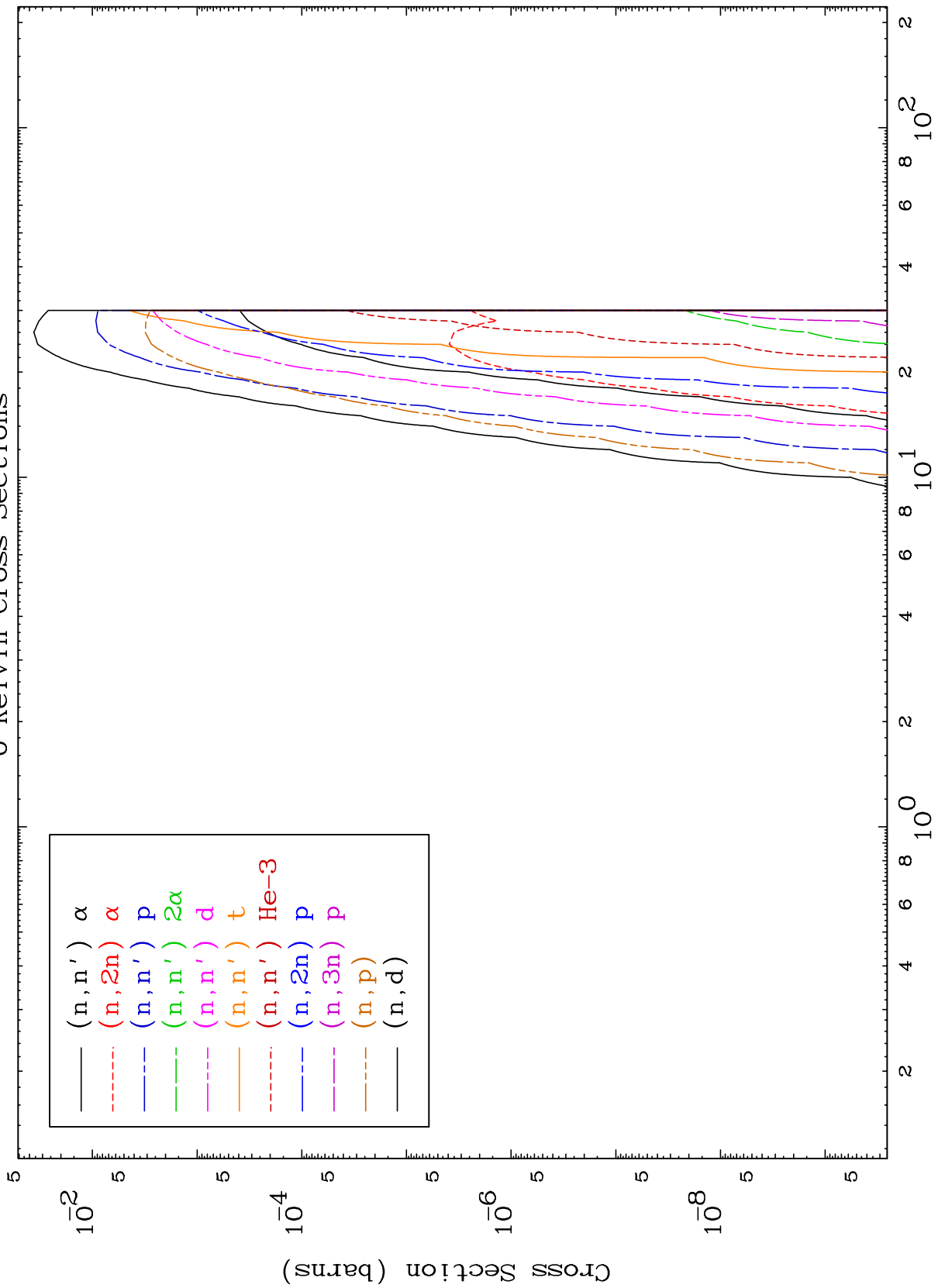


85-At-202m

MAT 8523

He-3 Charged Particle  
0 Kelvin Cross Sections

85-At-202m



5

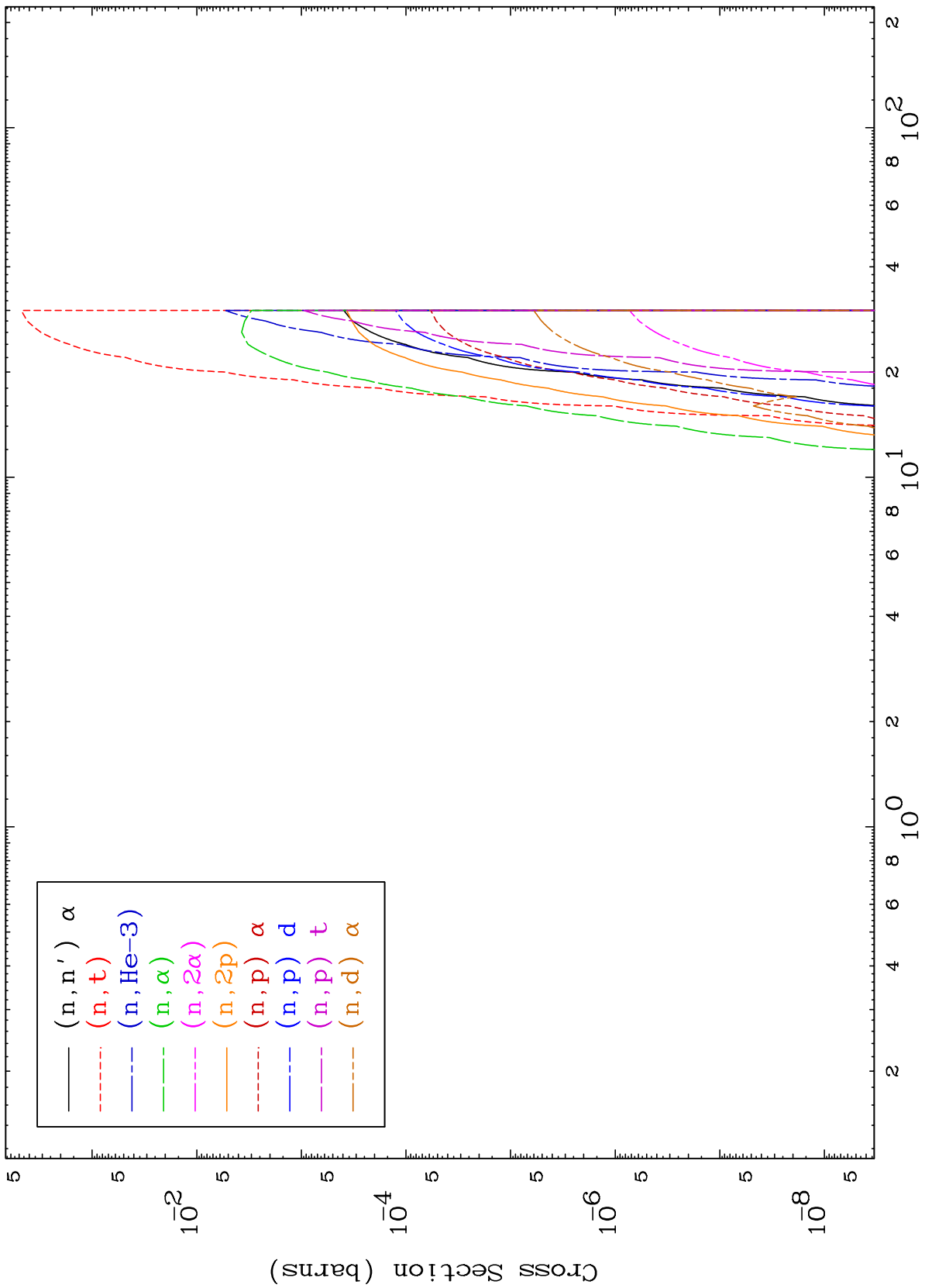
Incident Energy (MeV)

85-At-202m

MAT 8523

He-3 Charged Particle  
0 Kelvin Cross Sections

85-At-202m

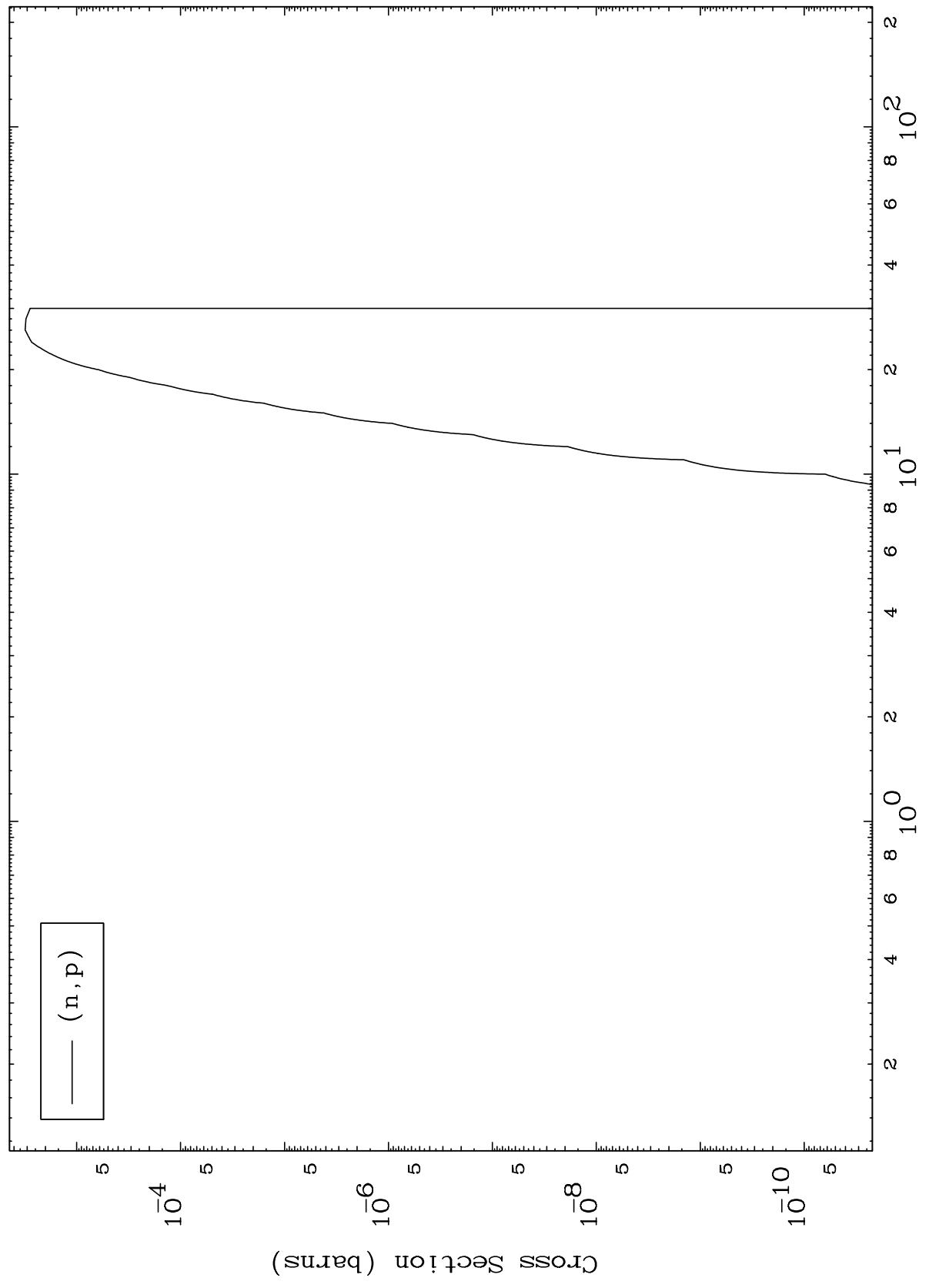


MAT 8523

(He-3,p) Levels

85-At-202m

0 Kelvin Cross Sections

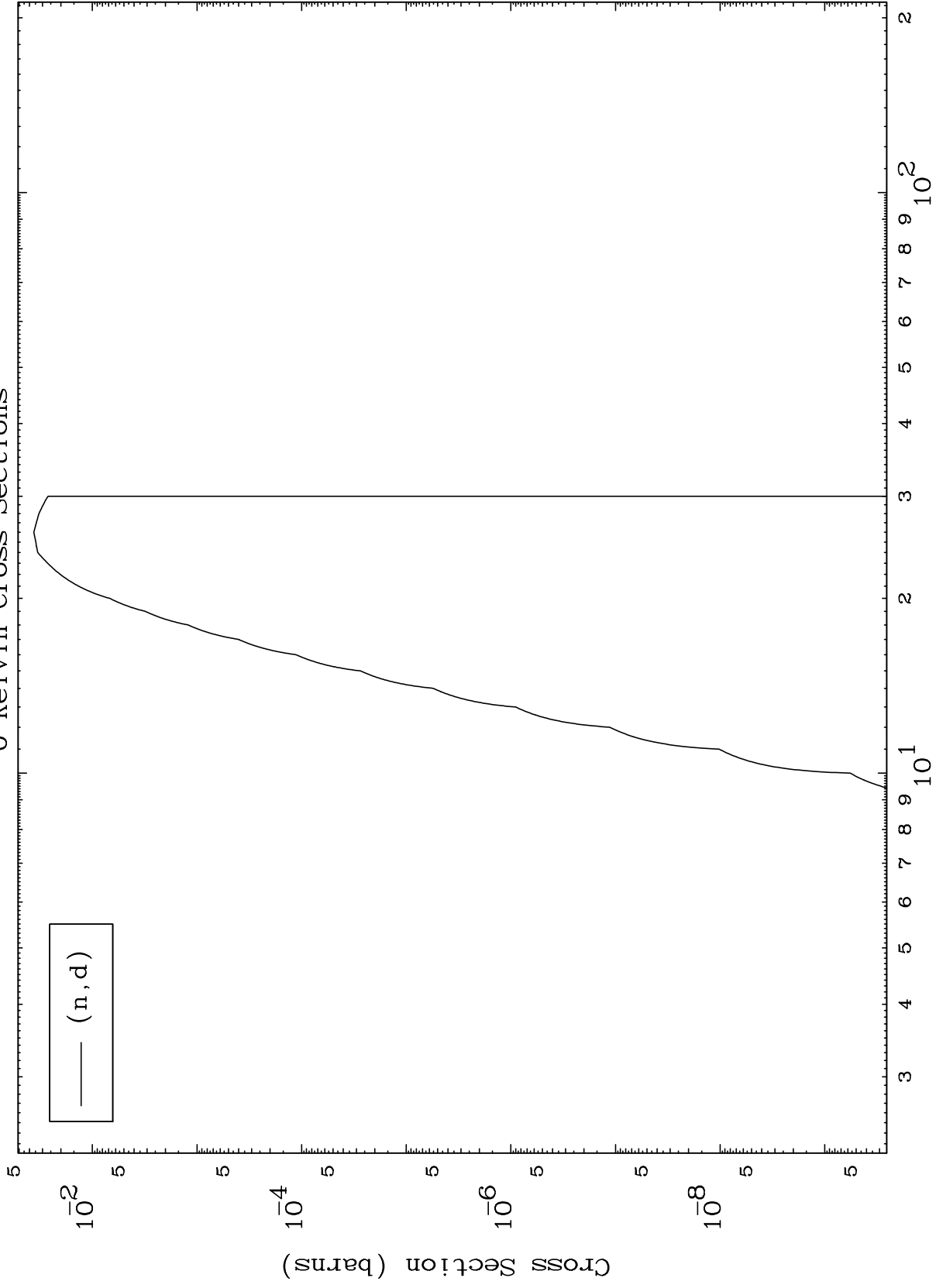




MAT 8523

(He-3,d) Levels  
0 Kelvin Cross Sections

85-At-202m

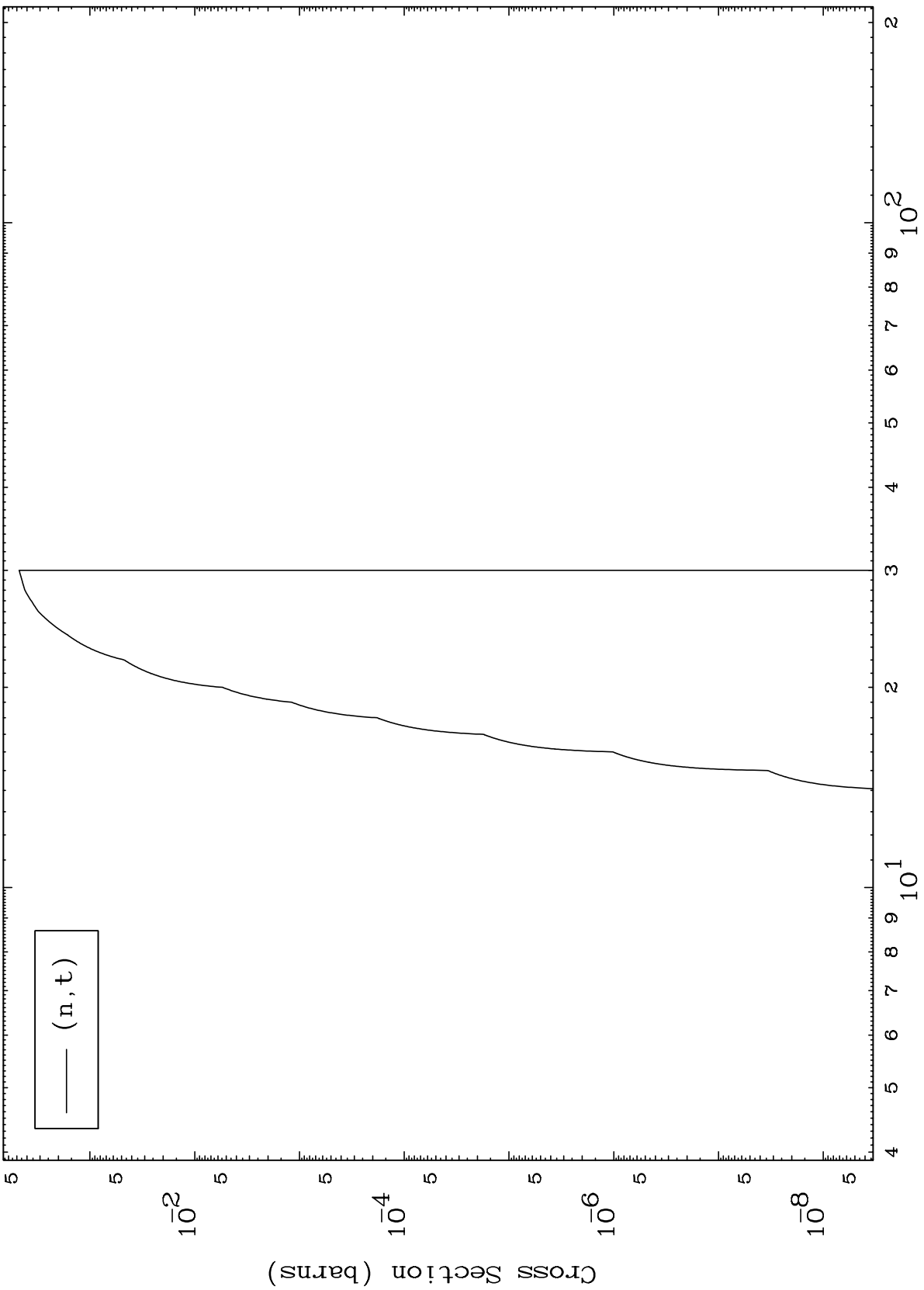


MAT 8523

(He-3,t) Levels

85-At-202m

0 Kelvin Cross Sections



9

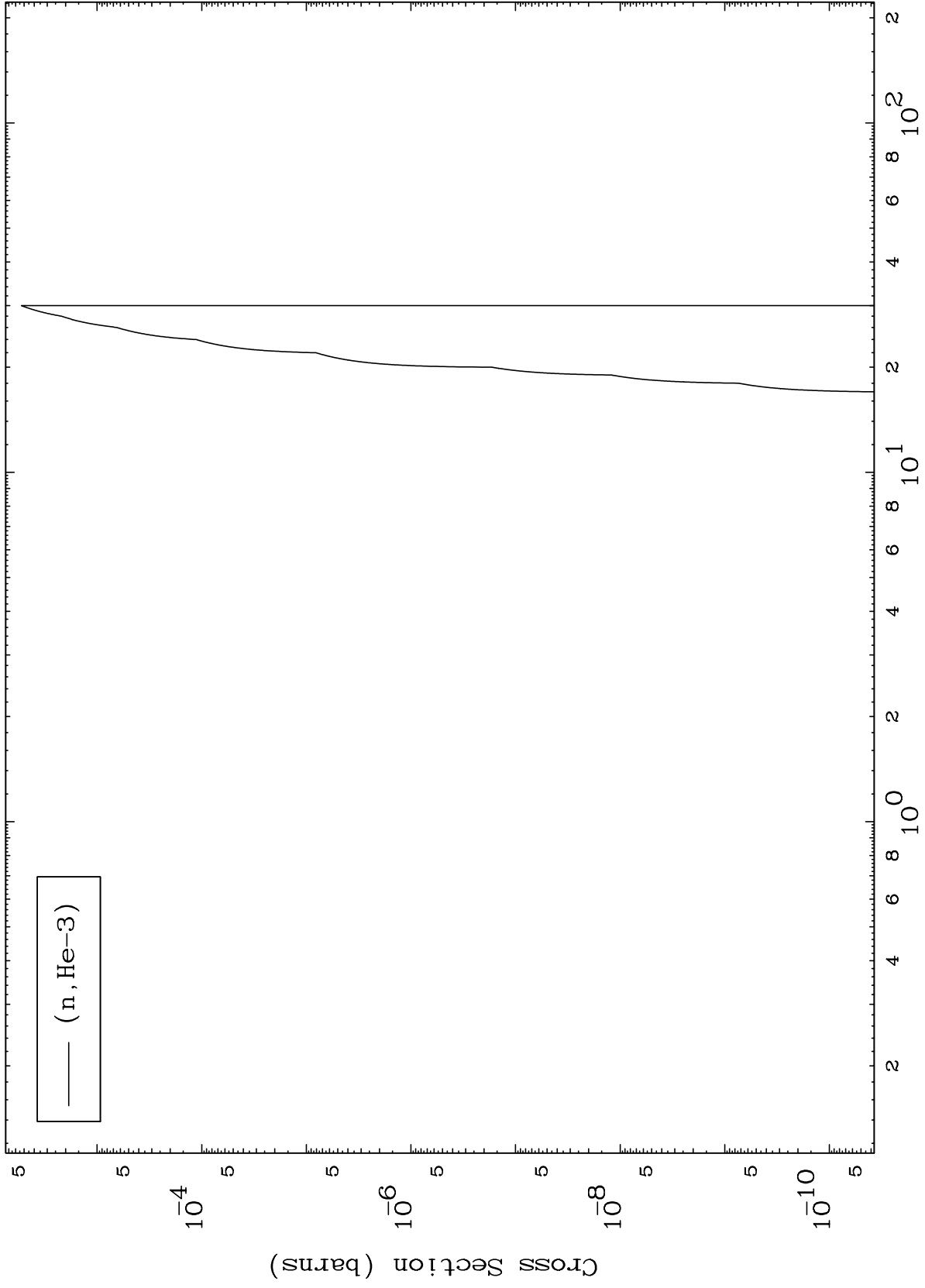
Incident Energy (MeV)

85-At-202m

MAT 8523

(He-3, He3) Levels  
0 Kelvin Cross Sections

85-At-202m



10

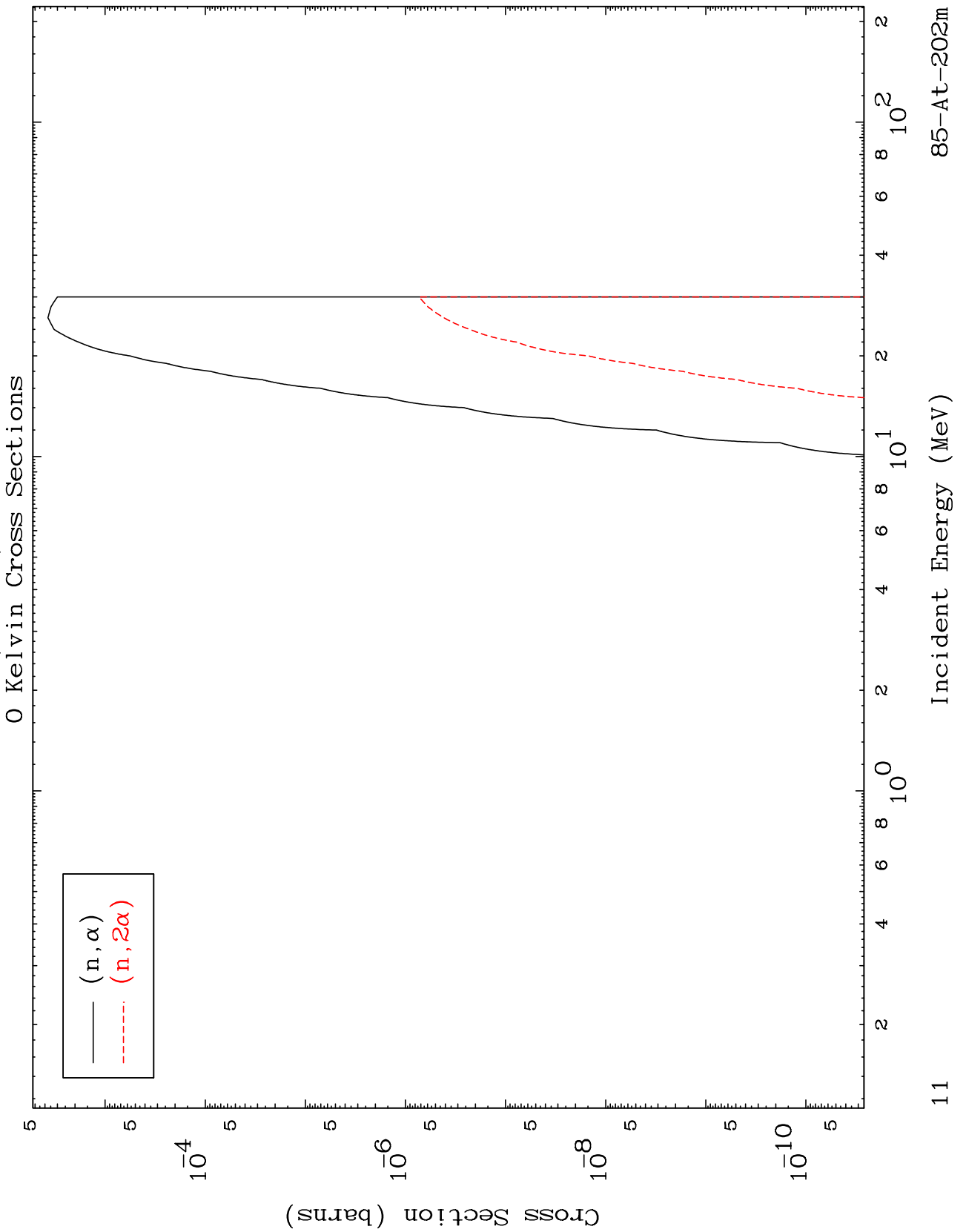
Incident Energy (MeV)

85-At-202m

MAT 8523

(He-3,  $\alpha$ ) Levels

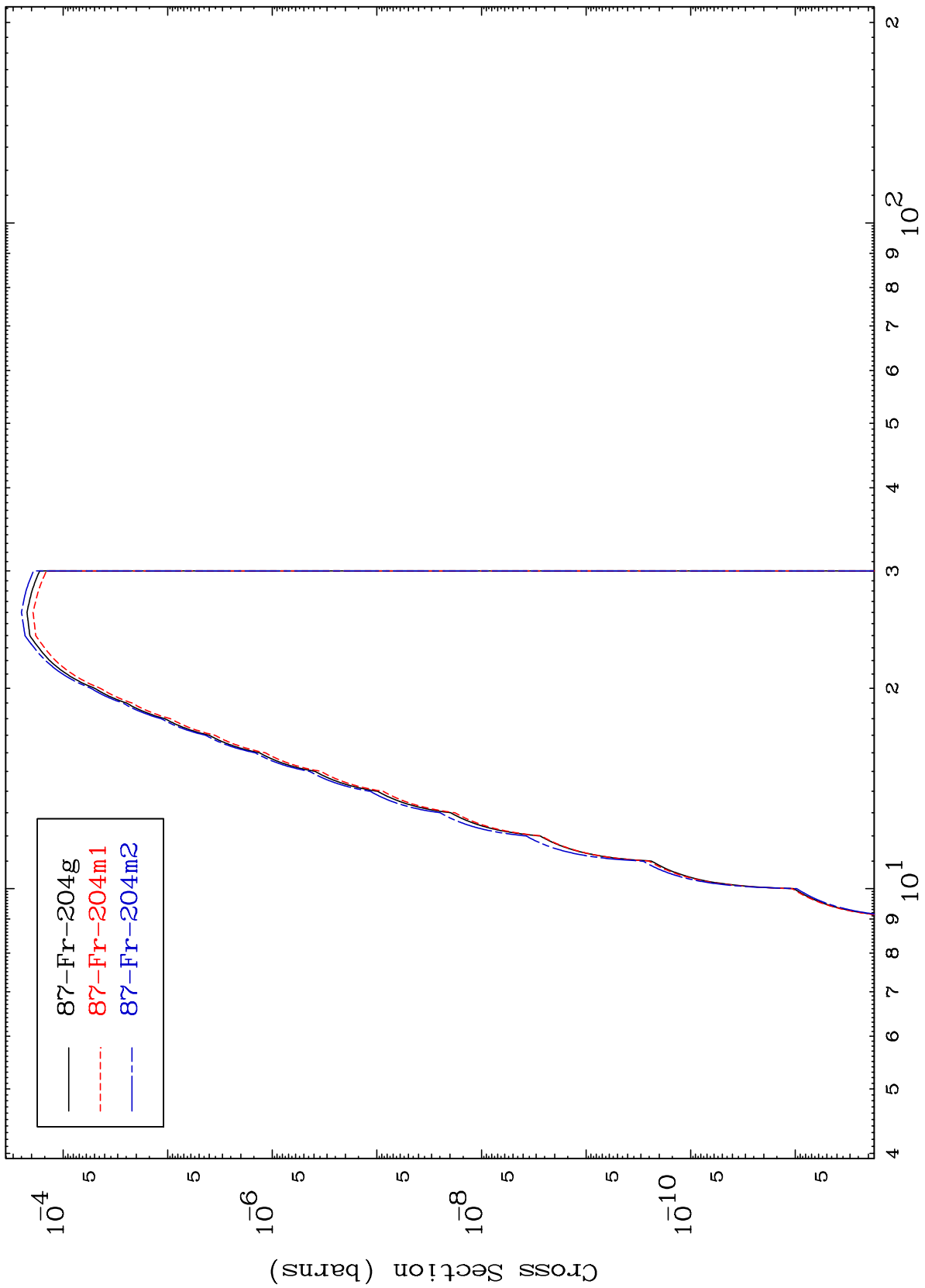
85-At-202m



MAT 8523

85-At-202m

### Radionuclide Production Cross Section



85-At-202m

Incident Energy (MeV)

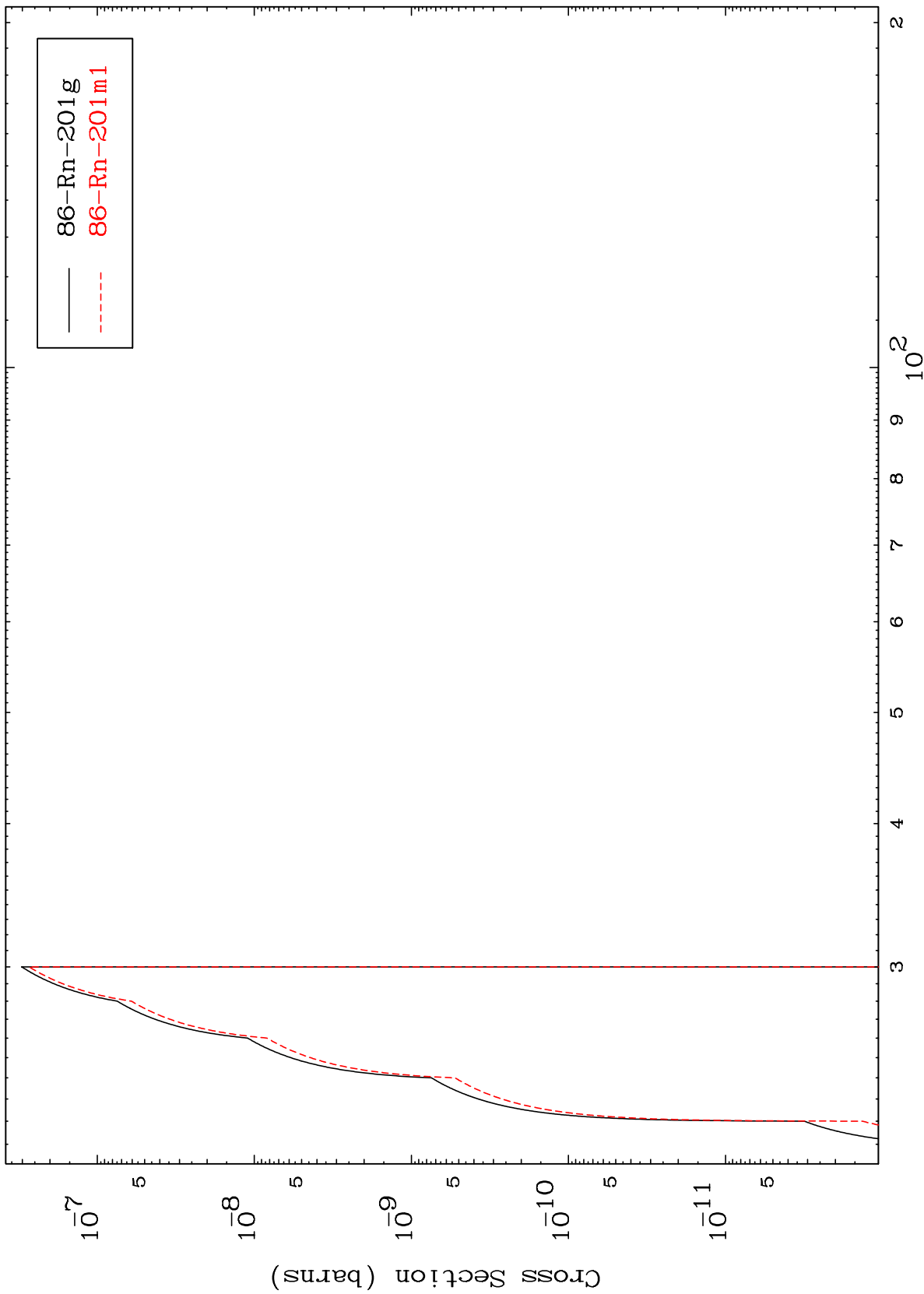
12

MAT 8523

(n,2n) d

85-At-202m

Radionuclide Production Cross Section



13

Incident Energy (MeV)

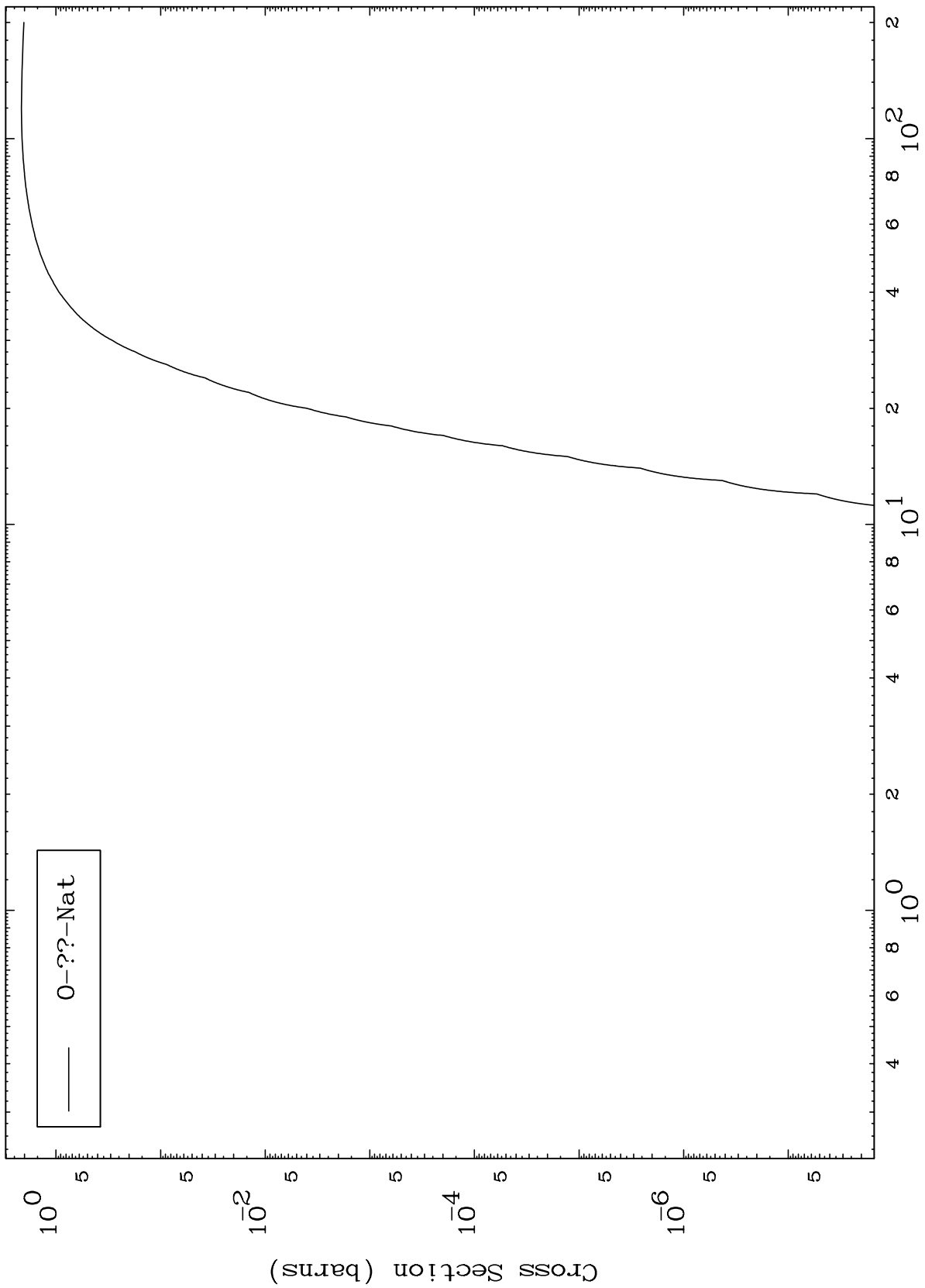
85-At-202m

MAT 8523

Fission

85-At-202m

Radionuclide Production Cross Section



Incident Energy (MeV)

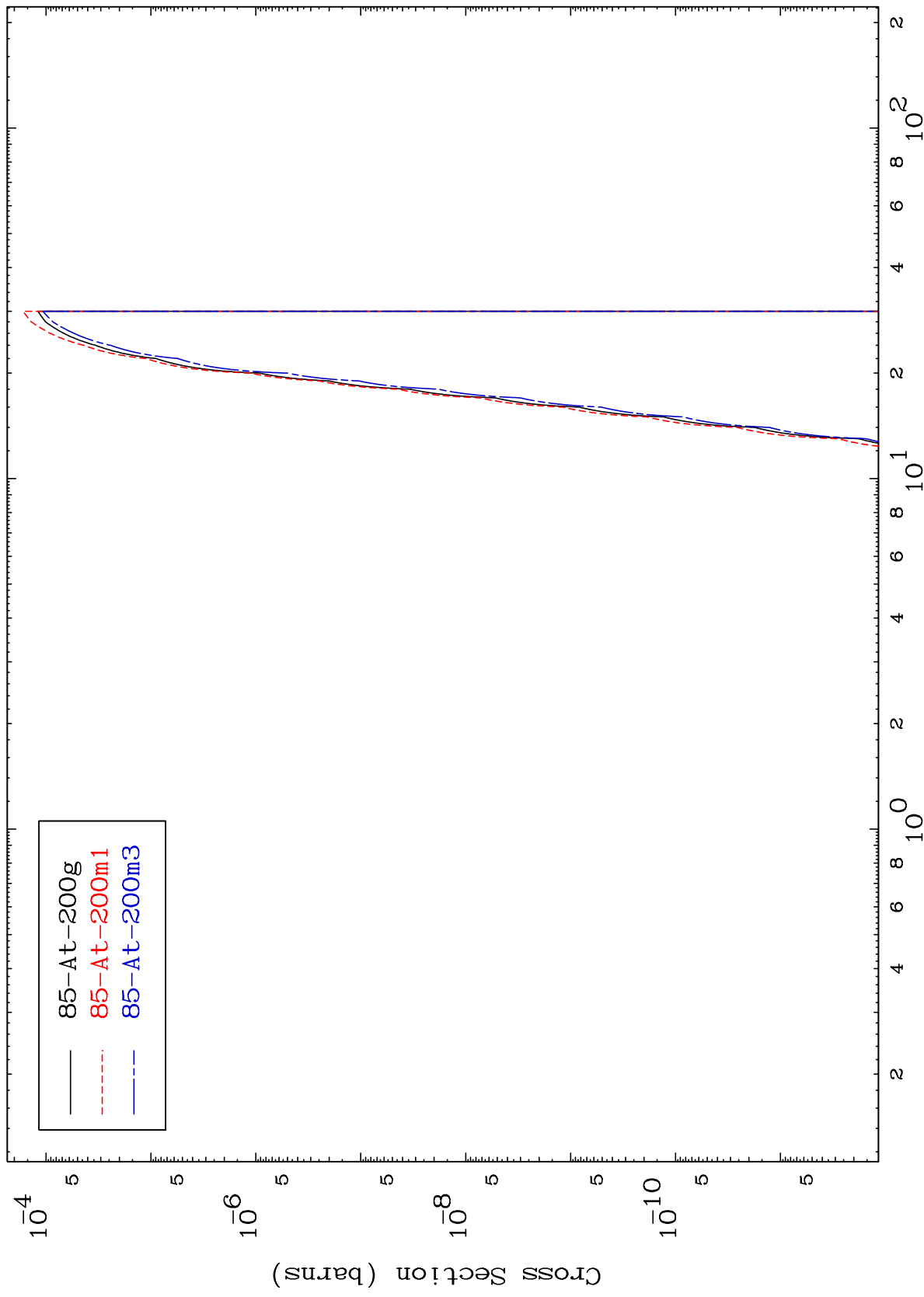
85-At-202m

MAT 8523

85-At-202m

(n,n')  $\alpha$

Radionuclide Production Cross Section



15

85-At-202m

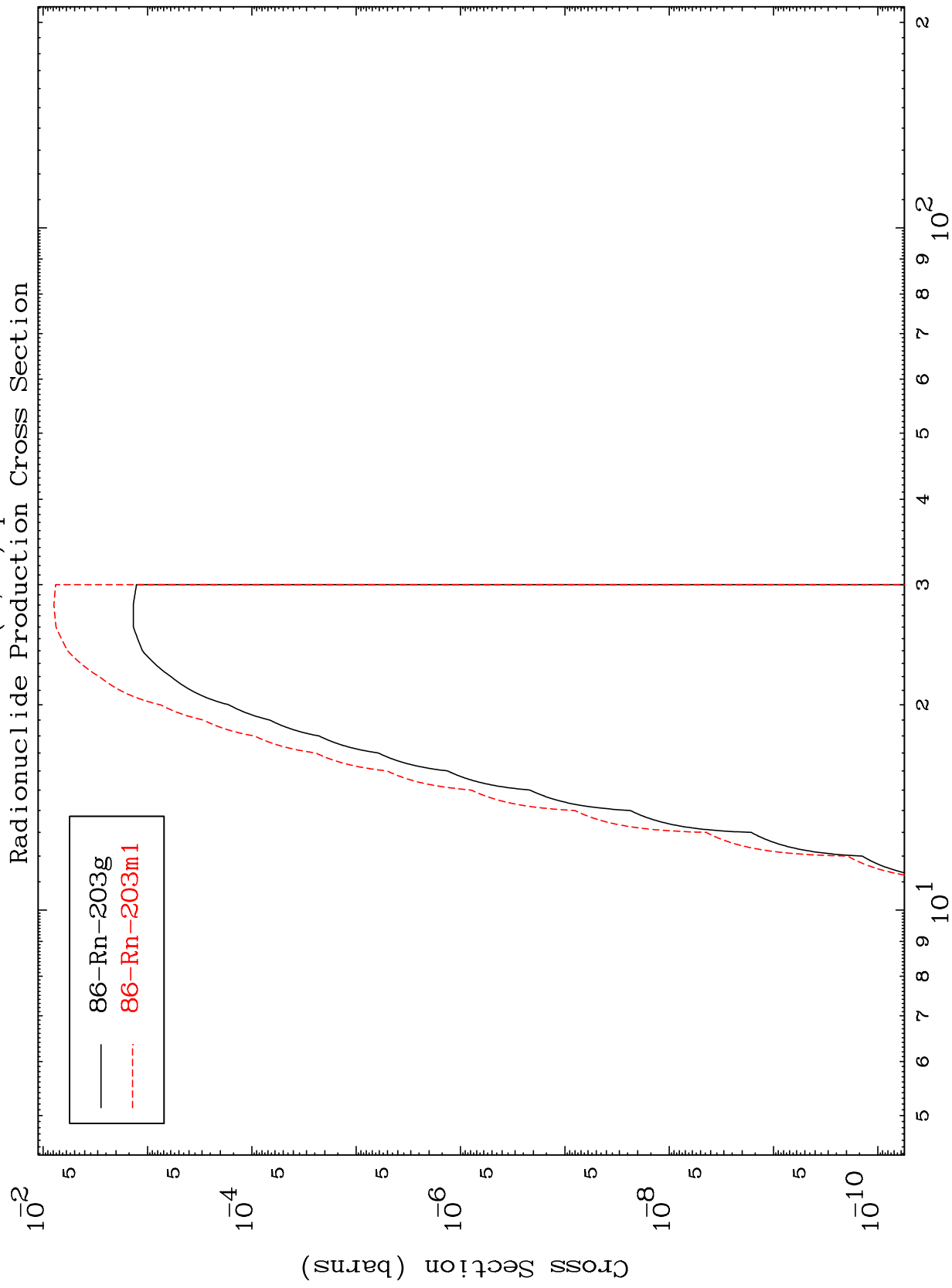
Incident Energy (MeV)



MAT 8523

(n,n') p

85-At-202m



16

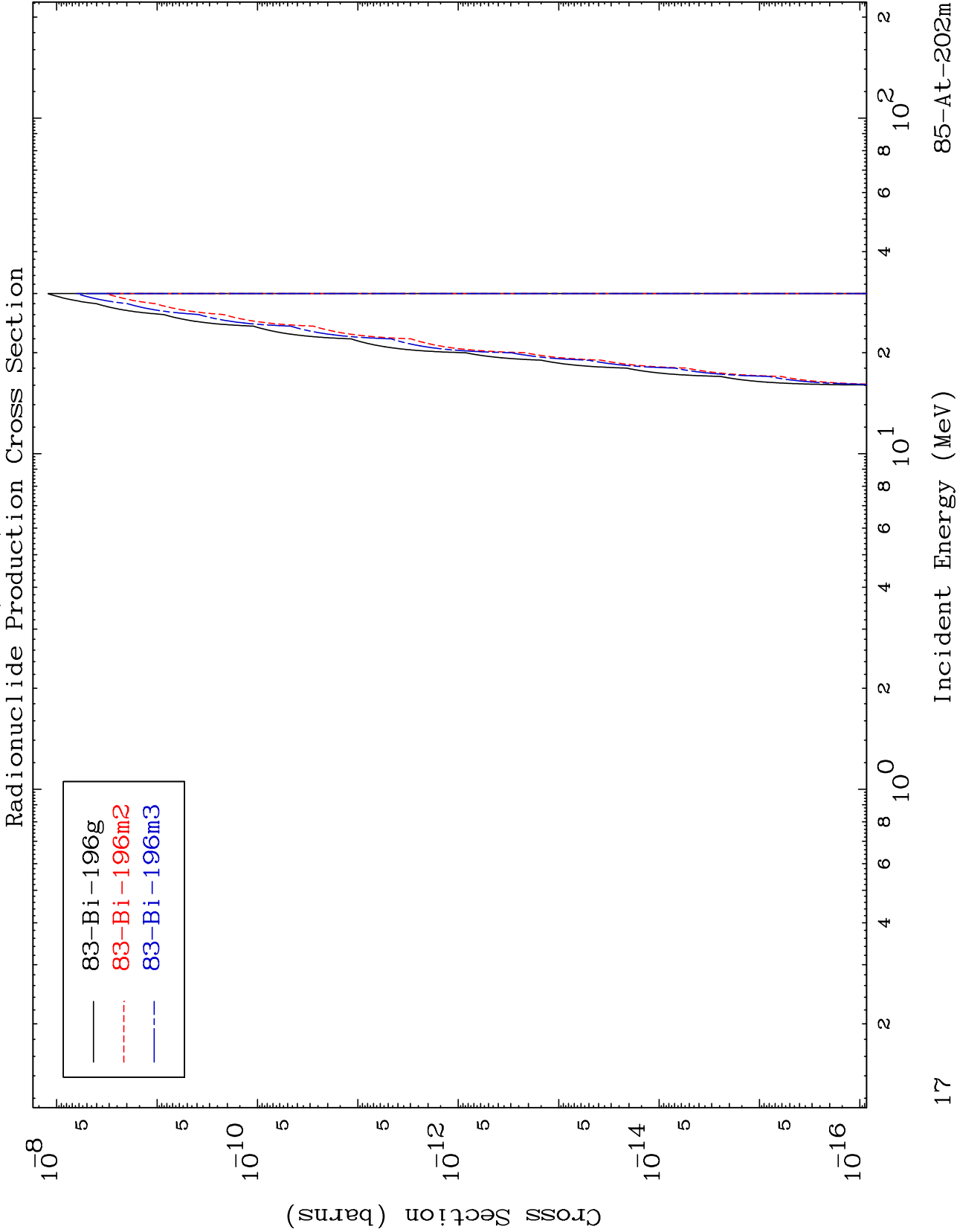
Incident Energy (MeV)

85-At-202m

MAT 8523

(n,n') 2α

85-At-202m



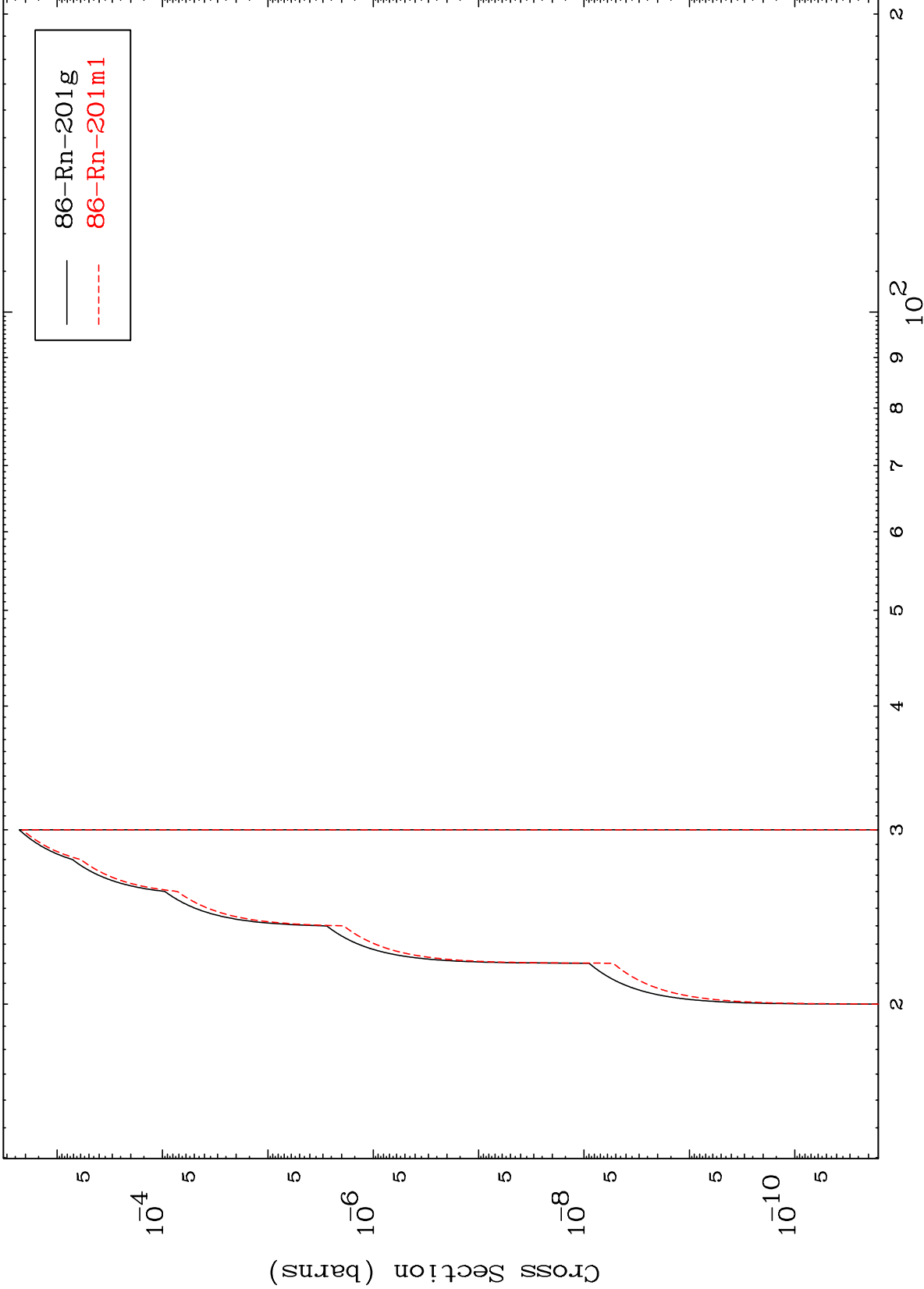
83-Bi-196g  
83-Bi-196m2  
83-Bi-196m3

MAT 8523

(n,n') t

85-At-202m

Radionuclide Production Cross Section



18

Incident Energy (MeV)

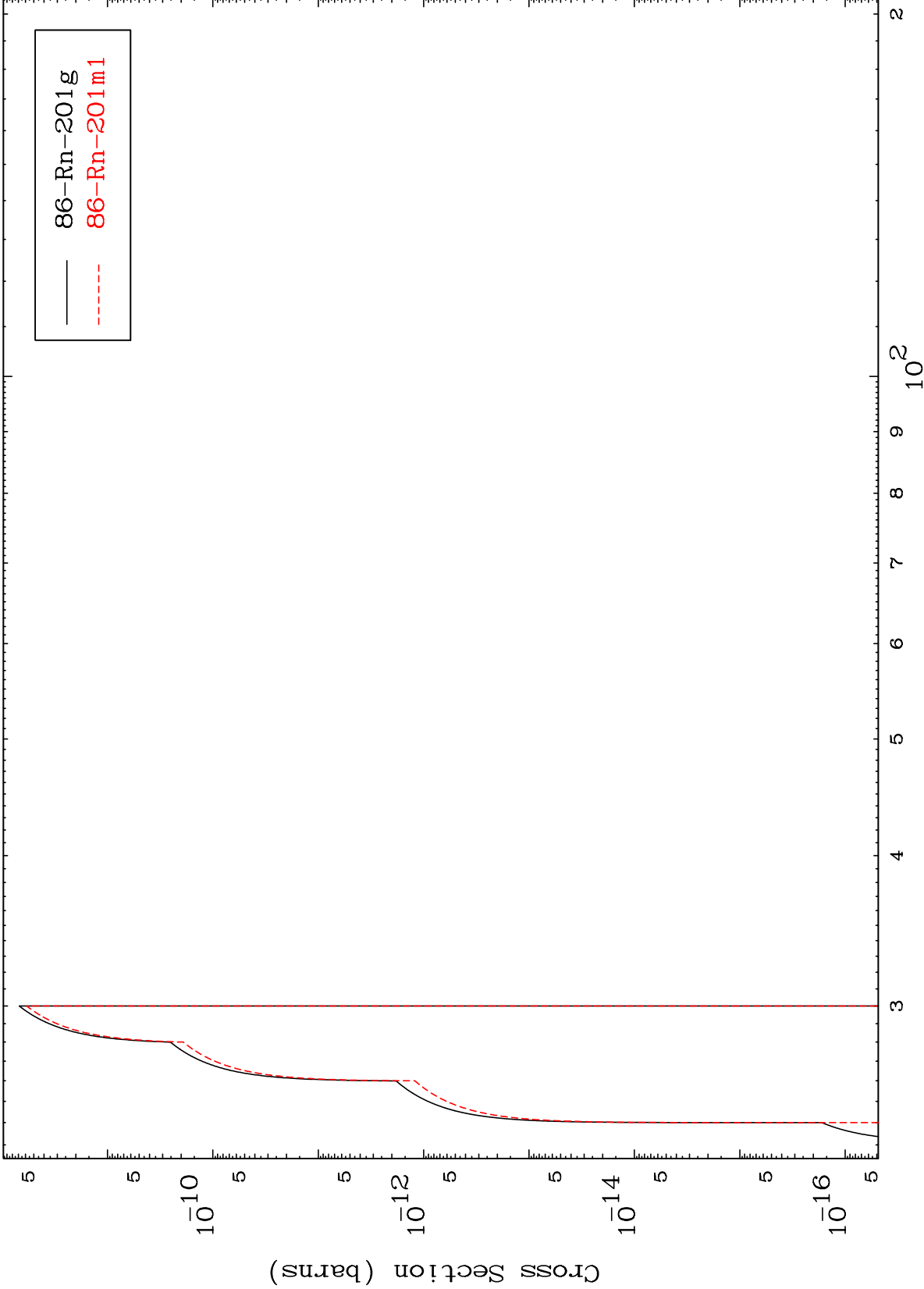
85-At-202m

MAT 8523

(n,3n) p

85-At-202m

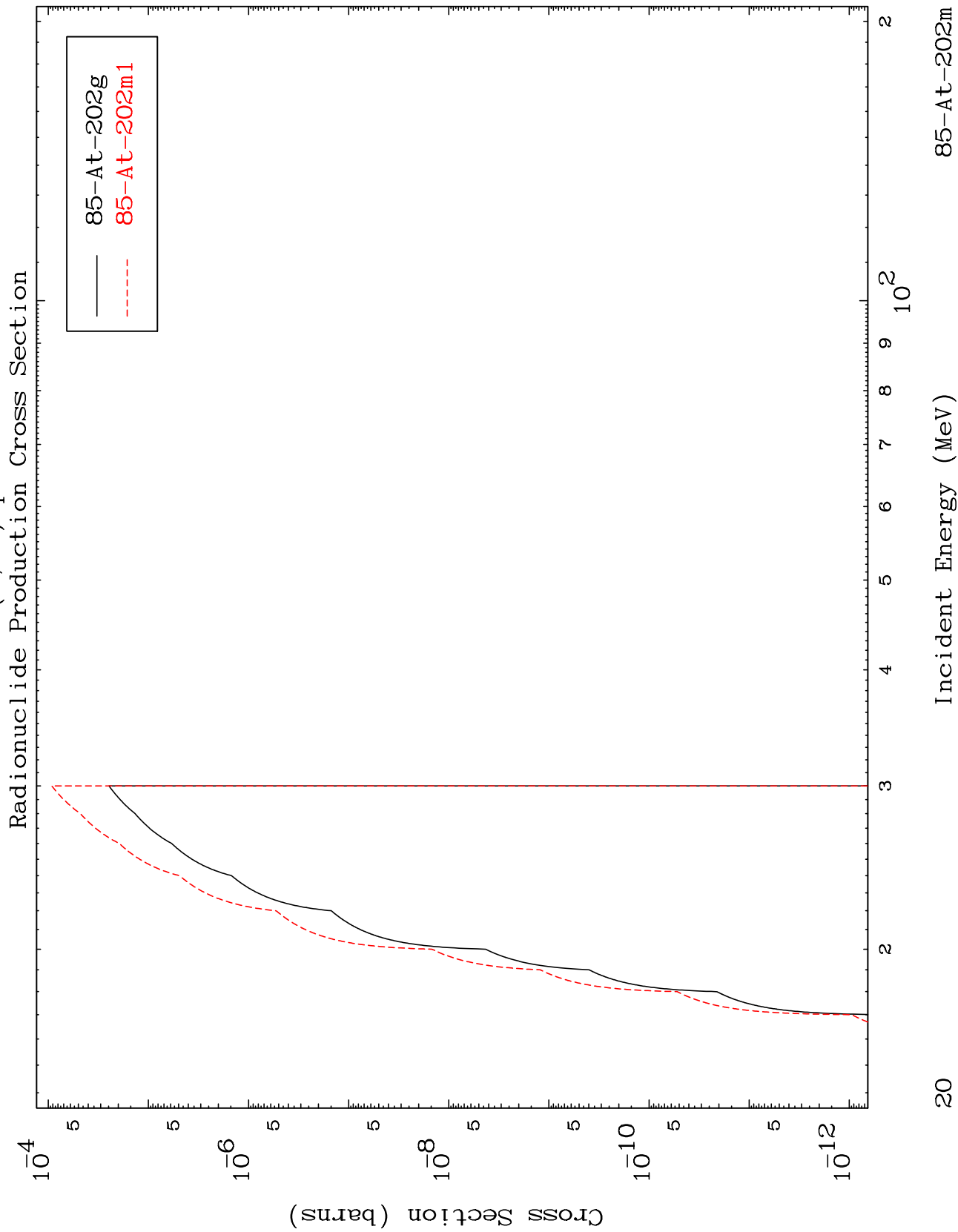
Radionuclide Production Cross Section



MAT 8523

(n,2n) p

85-At-202m



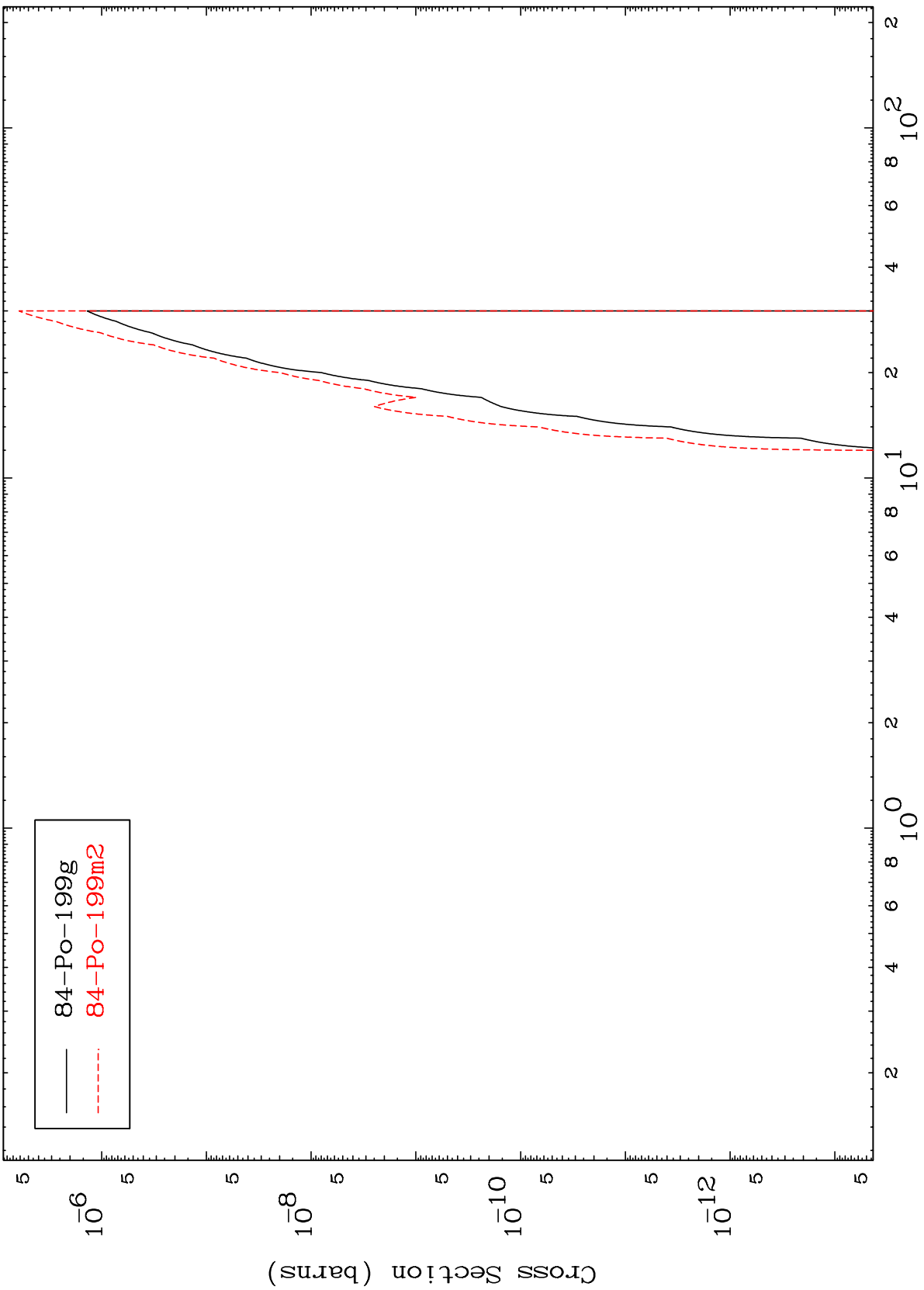
20

MAT 8523

(n,n') p  $\alpha$

85-At-202m

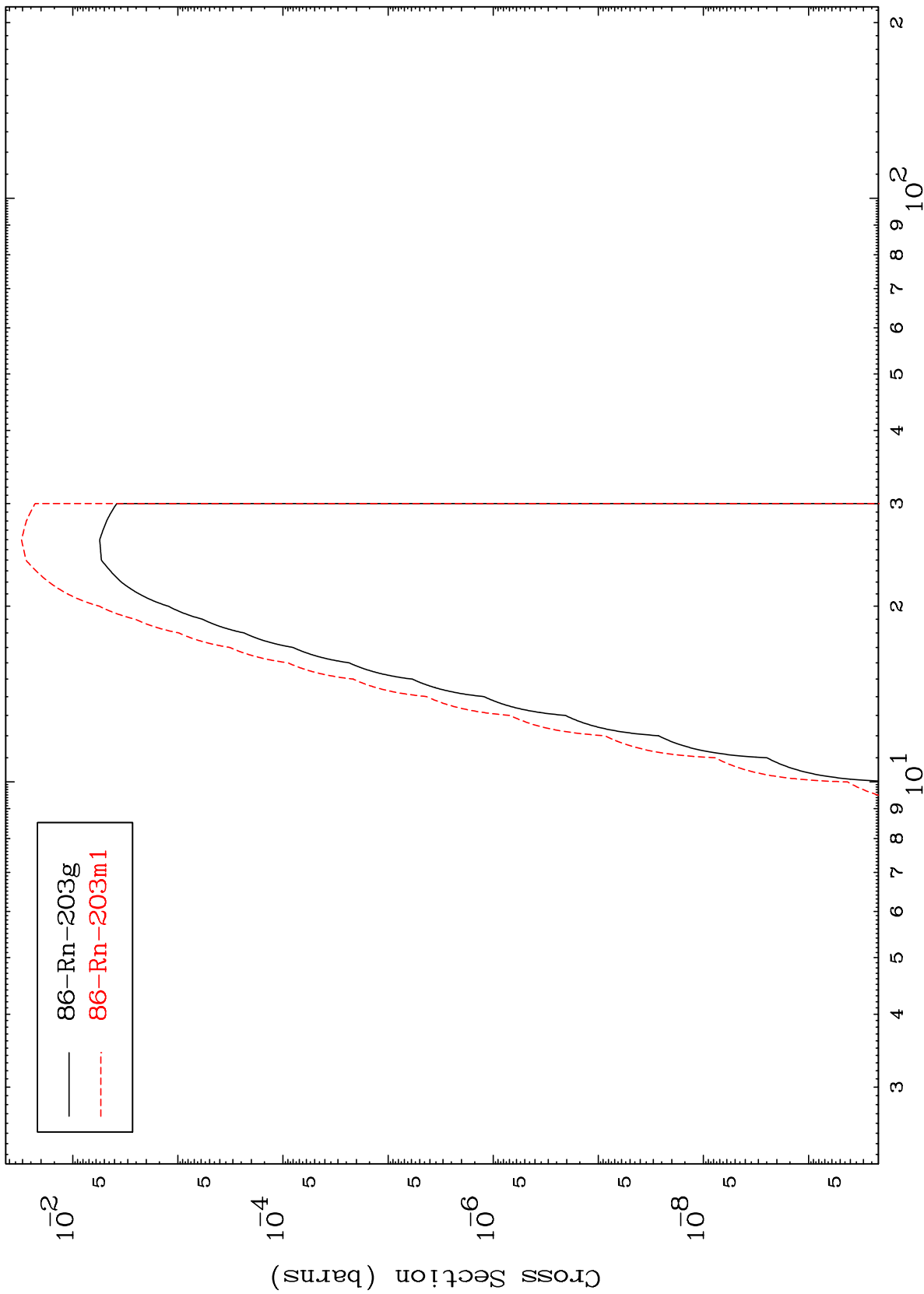
Radionuclide Production Cross Section



MAT 8523

85-At-202m

(n,d)  
Radionuclide Production Cross Section



Incident Energy (MeV)

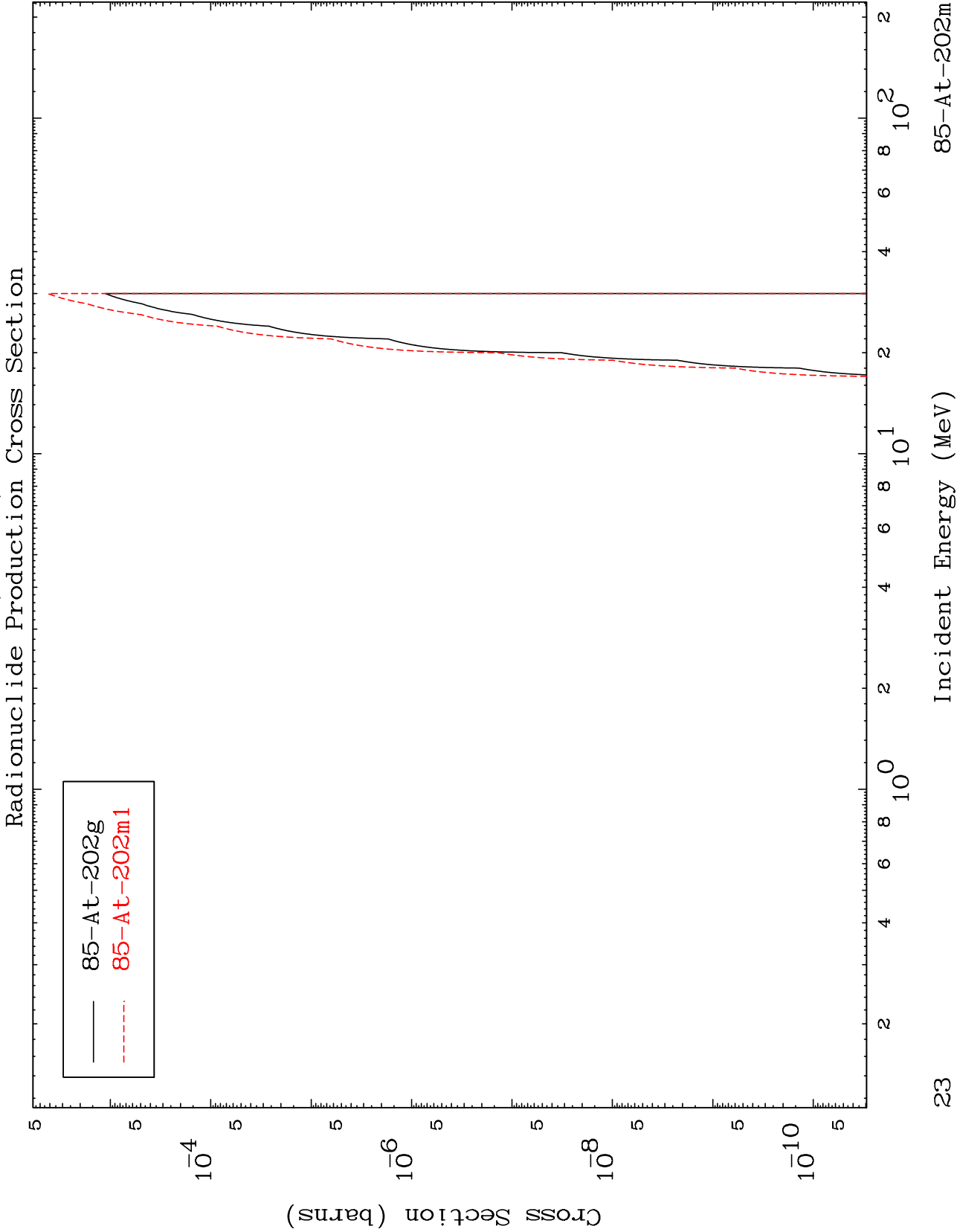
85-At-202m

22

MAT 8523

(n,He-3)

85-At-202m



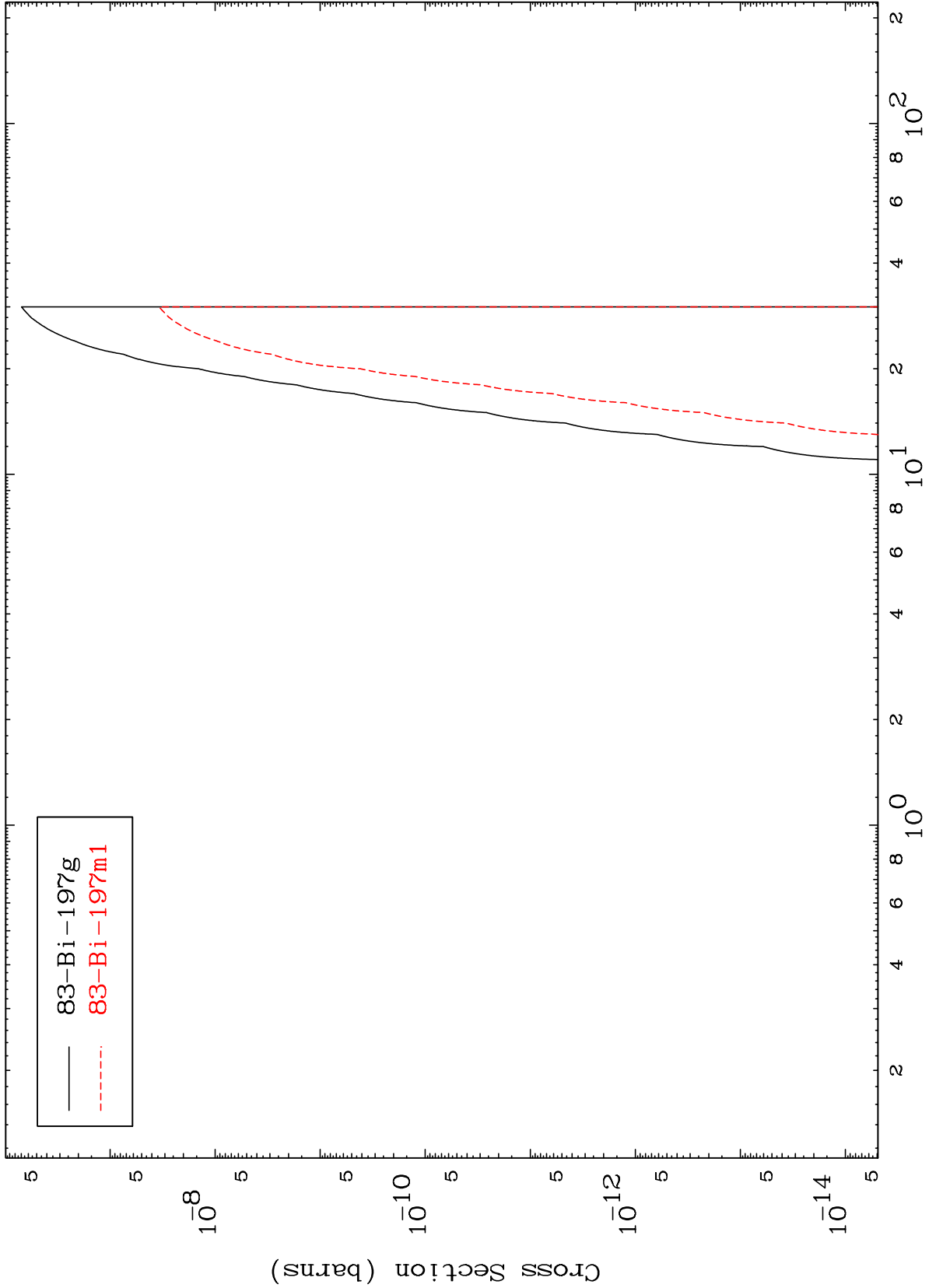


MAT 8523

(n,2α)

85-At-202m

Radionuclide Production Cross Section

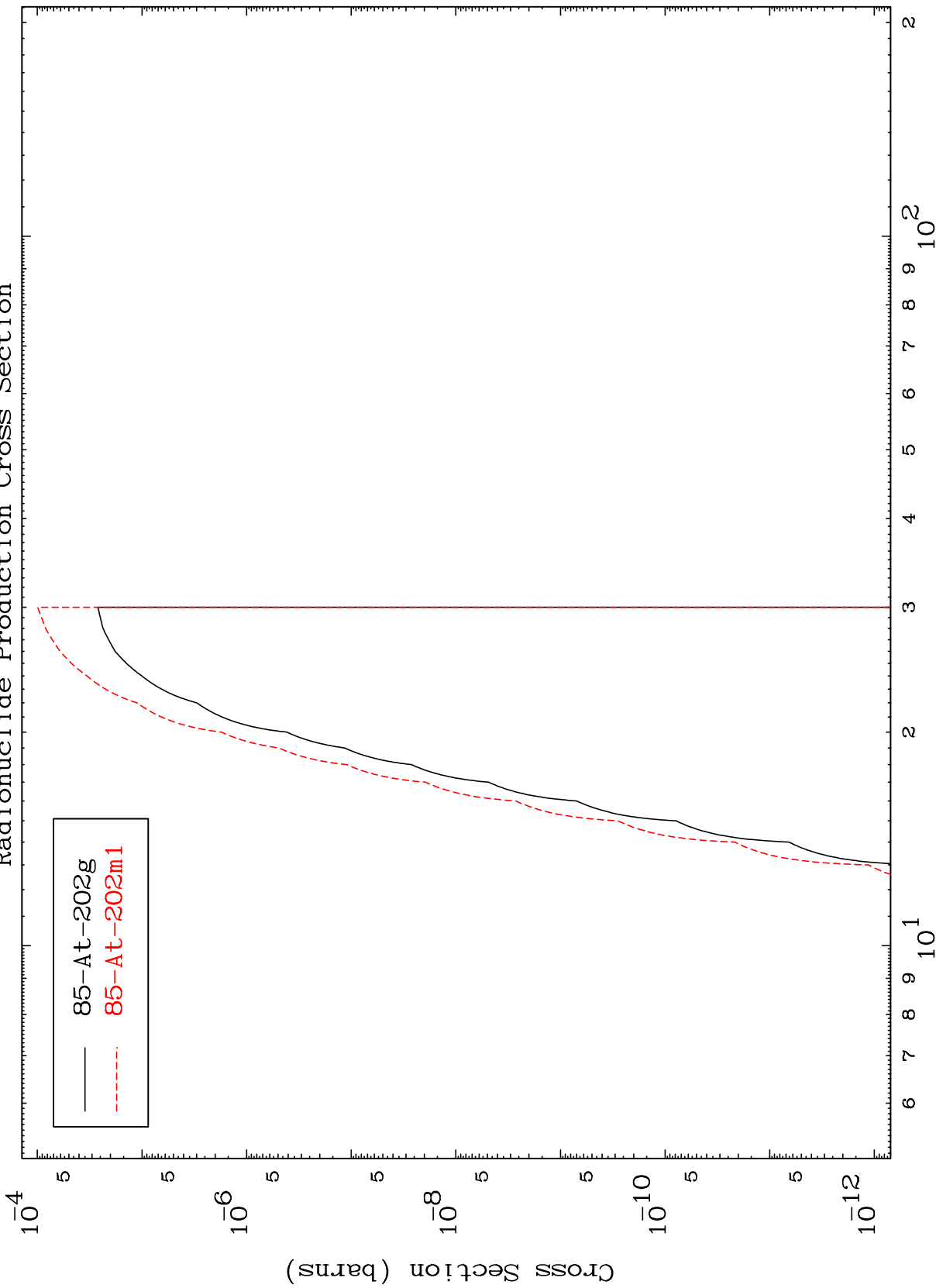


MAT 8523

(n,p) d

85-At-202m

Radionuclide Production Cross Section



Incident Energy (MeV)

85-At-202m

25

MAT 8523

(n,d)  $\alpha$

85-At-202m

