

Program EVALPLOT
(Version 2021-1)

by

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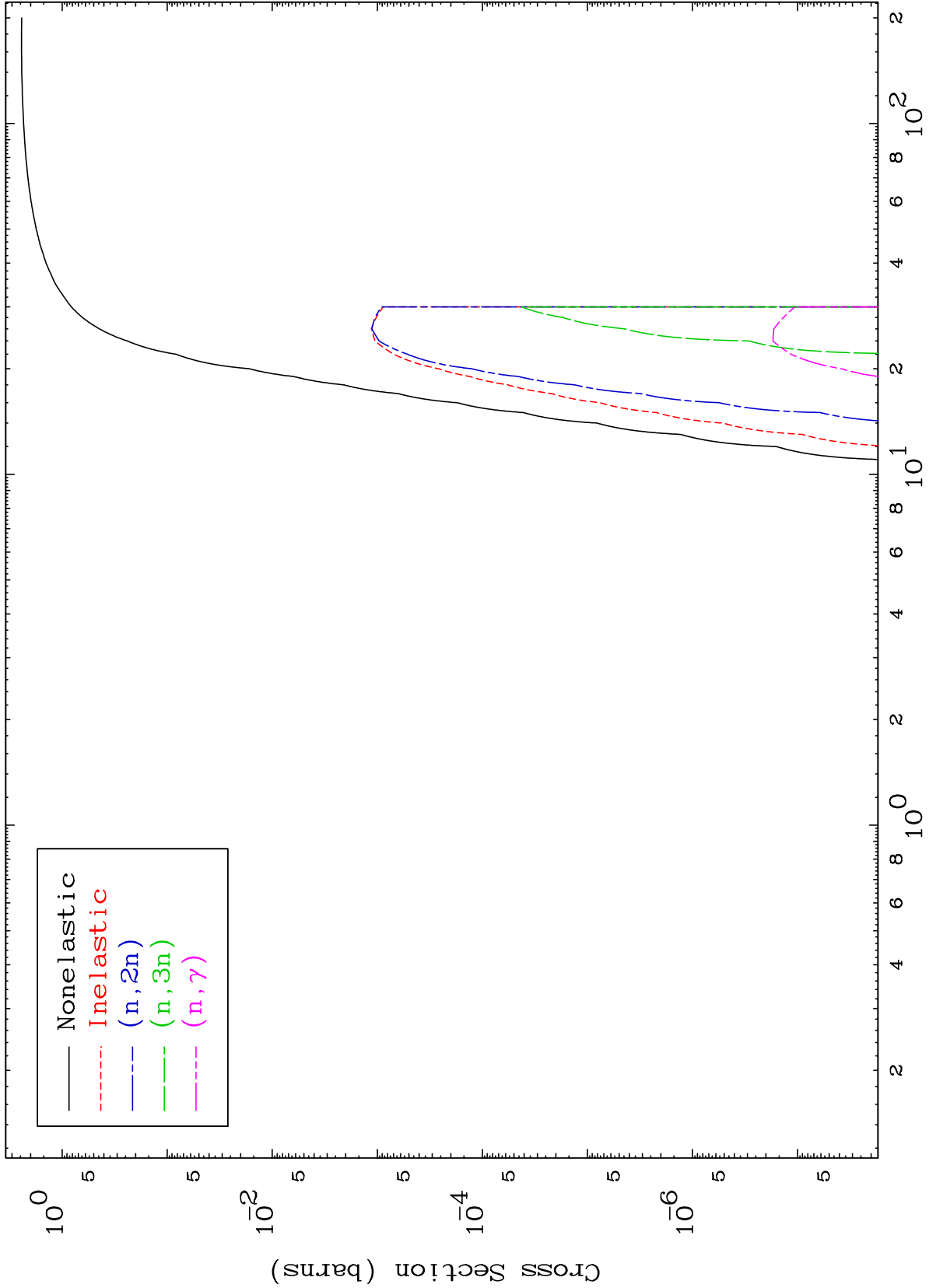
Press Mouse Button to Start

MAT 8613

He-3 Major

86-Rn-207

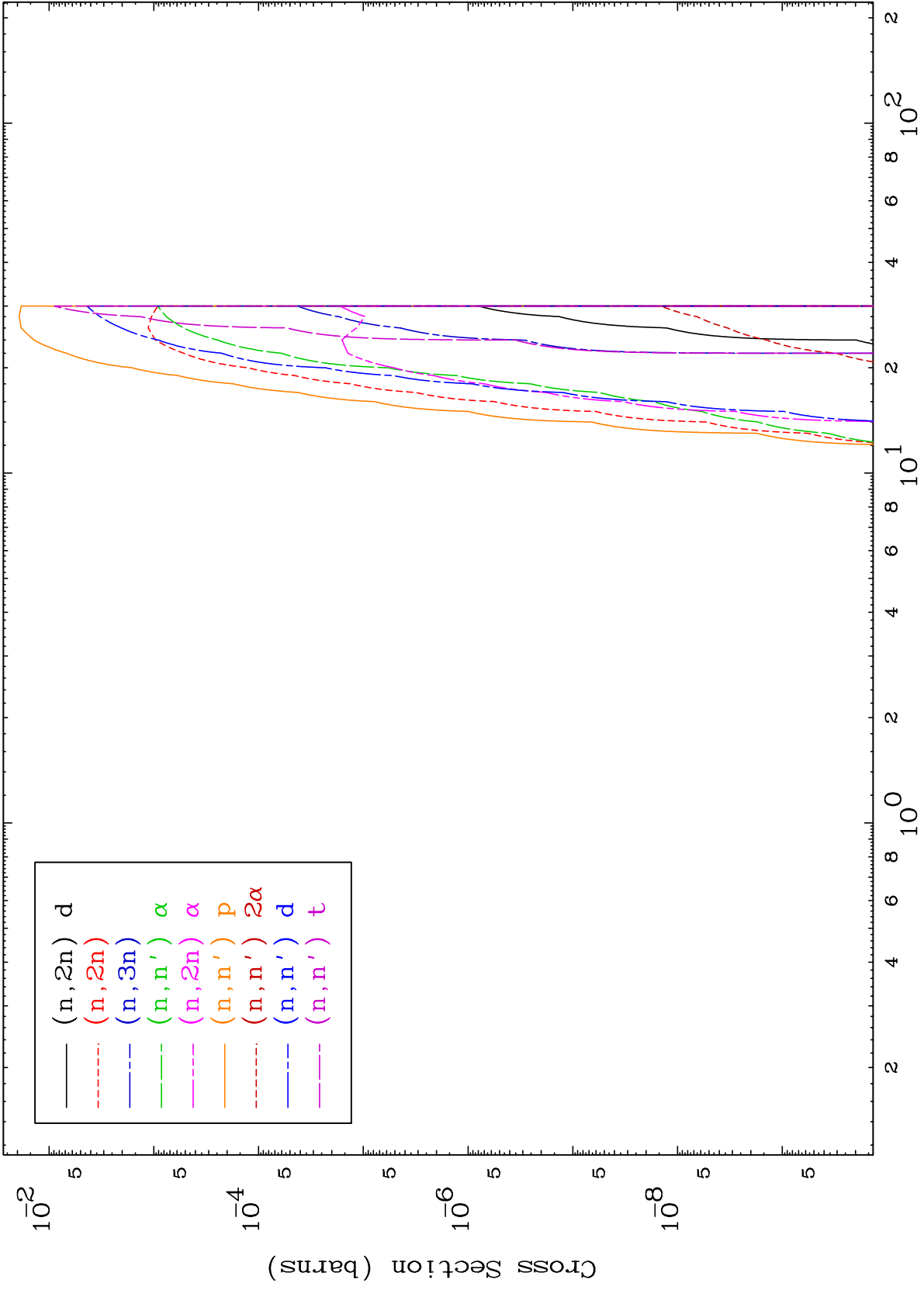
0 Kelvin Cross Sections

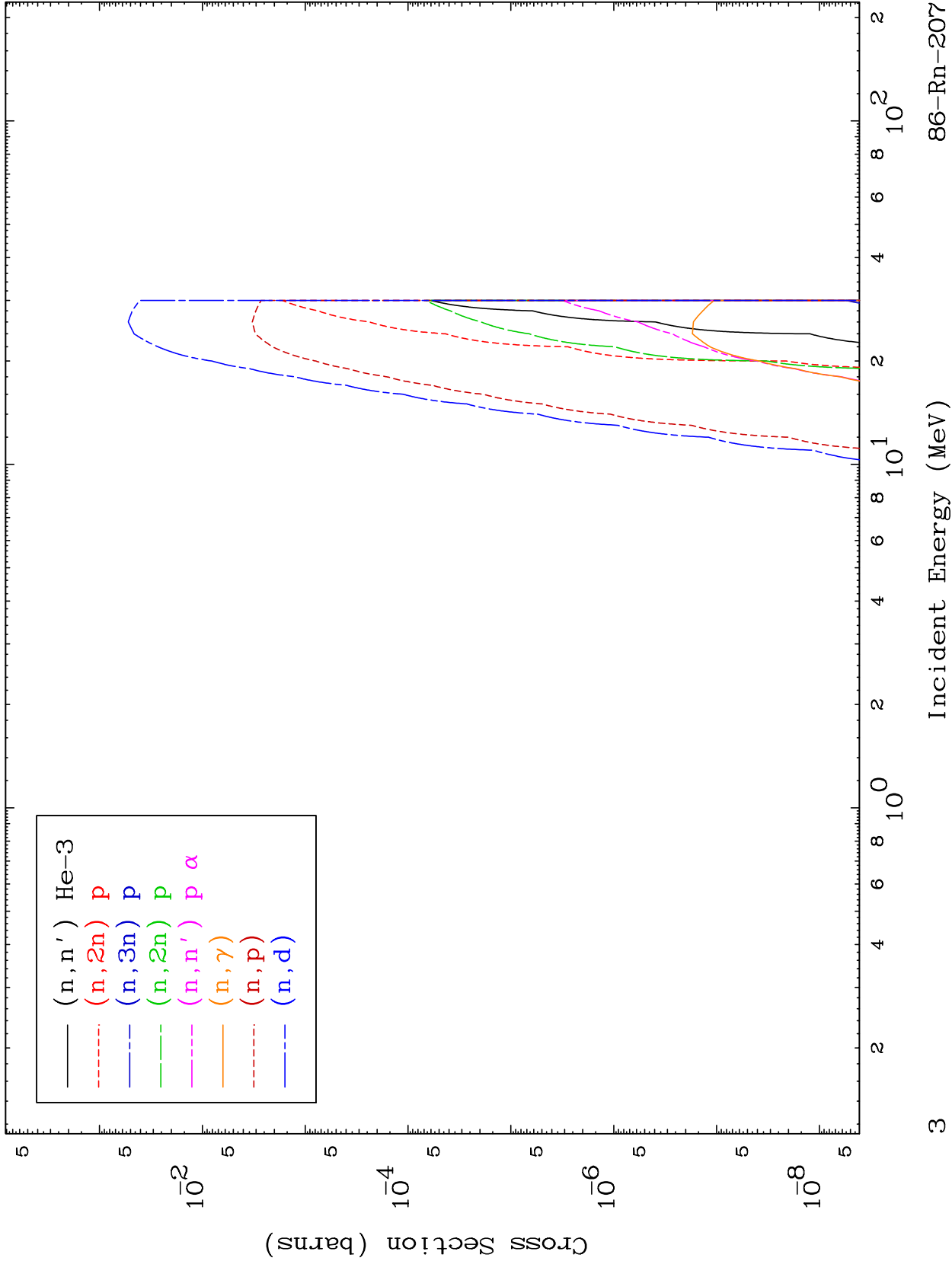


MAT 8613

He-3 Neutron Absorption
0 Kelvin Cross Sections

86-Rn-207

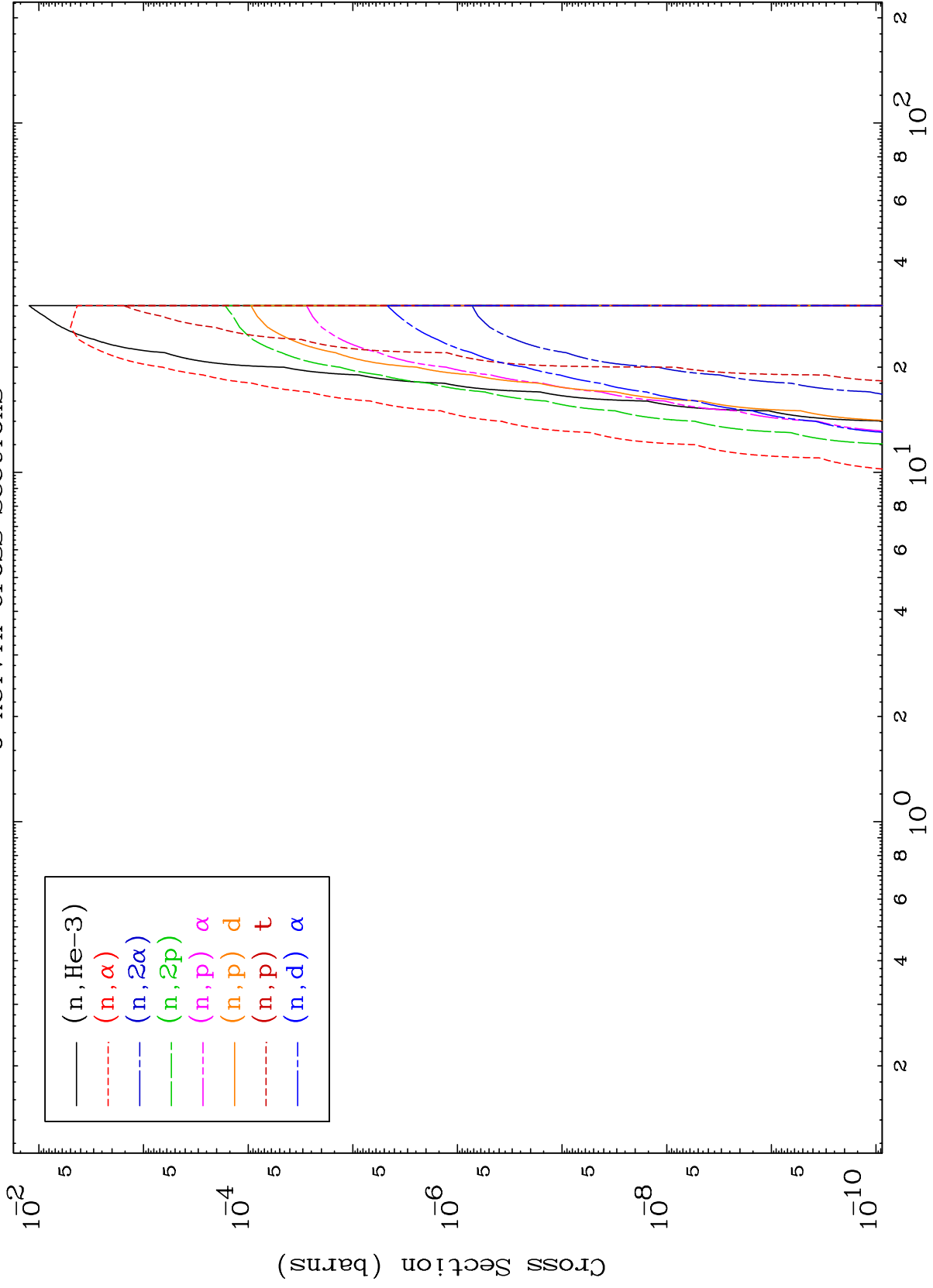


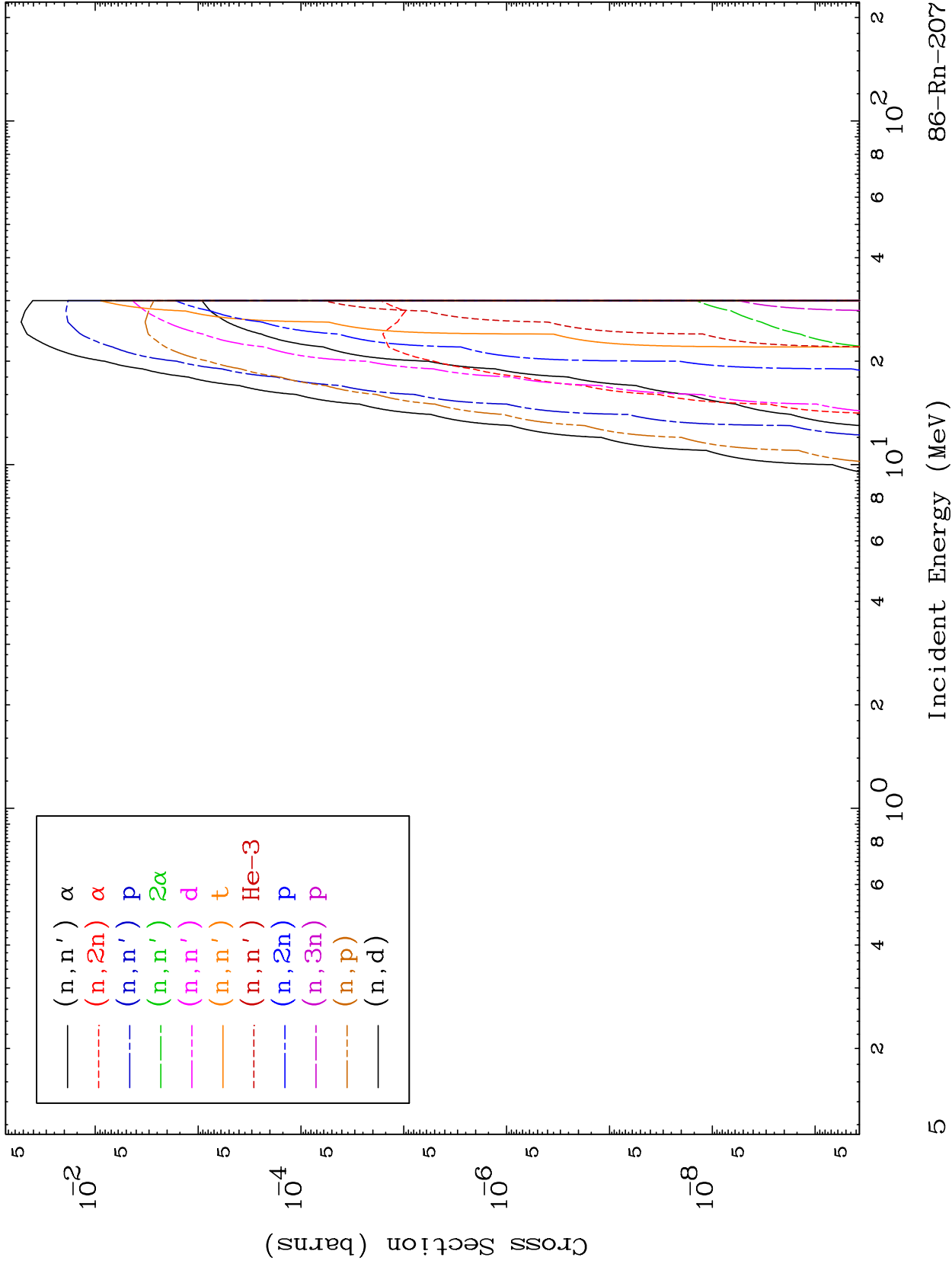


MAT 8613

He-3 Neutron Absorption
0 Kelvin Cross Sections

86-Rn-207

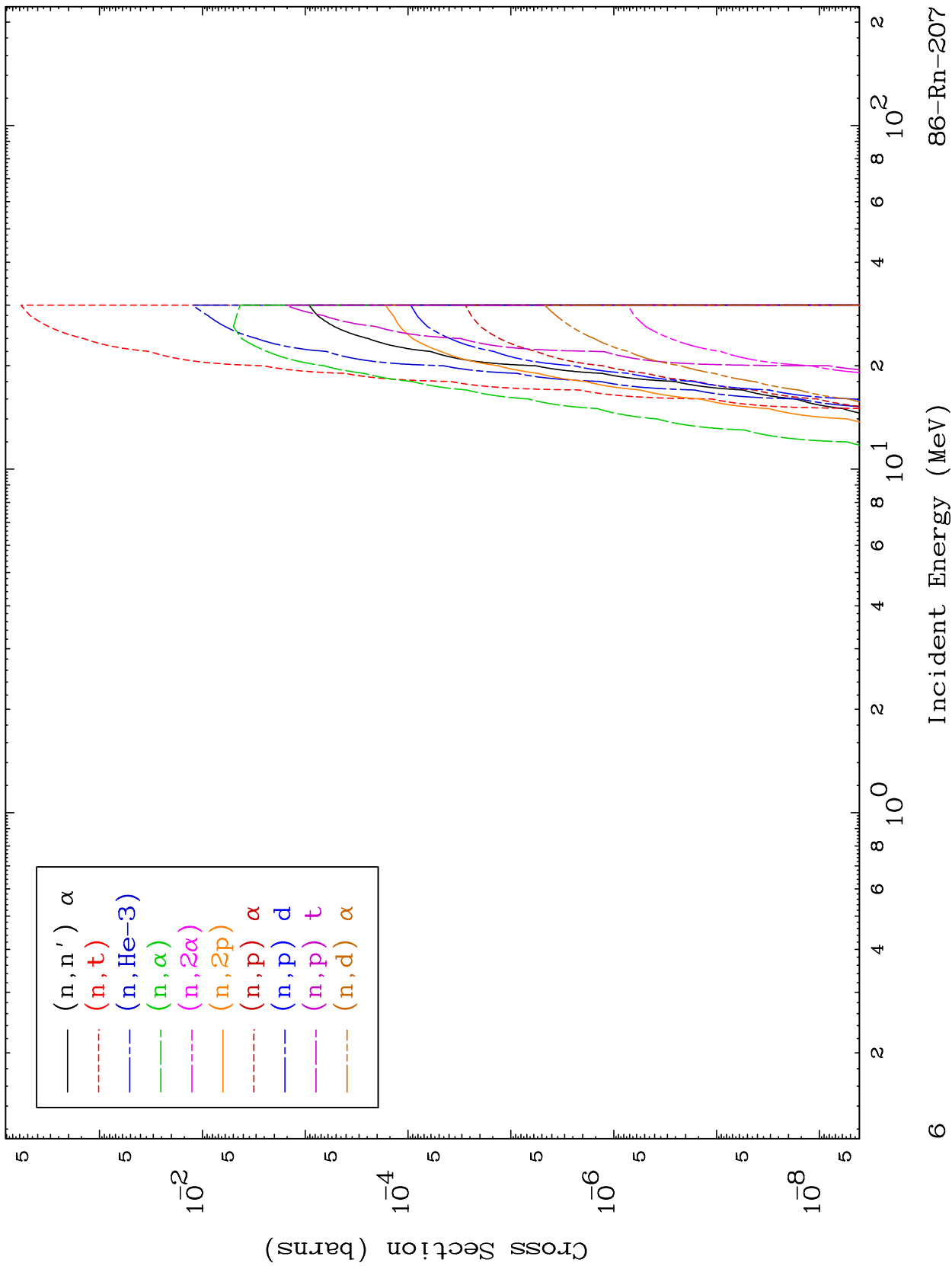




MAT 8613

He-3 Charged Particle
0 Kelvin Cross Sections

86-Rn-207



6

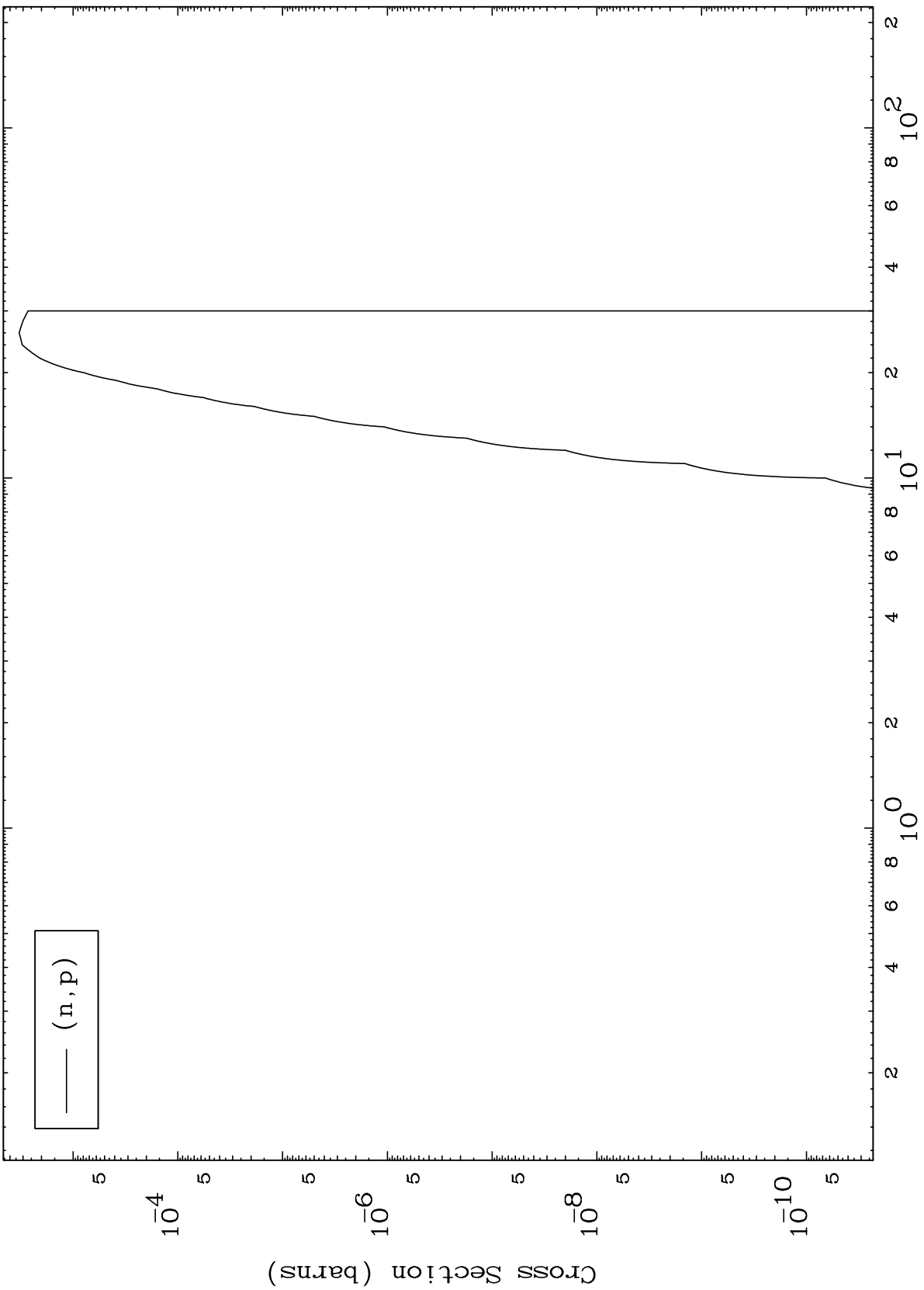
86-Rn-207

MAT 8613

(He-3,p) Levels

86-Rn-207

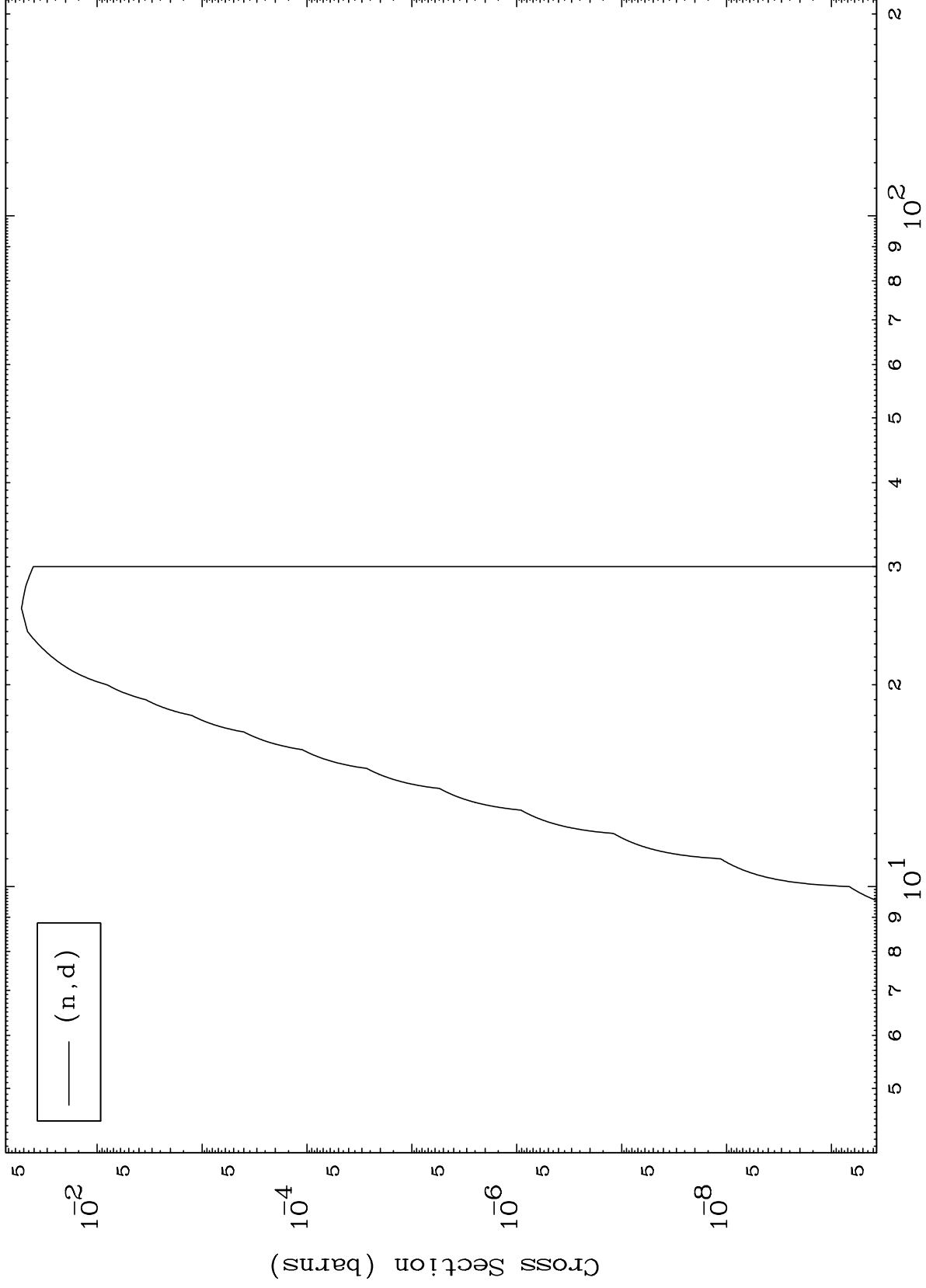
0 Kelvin Cross Sections



MAT 8613

(He-3,d) Levels
0 Kelvin Cross Sections

86-Rn-207



8

Incident Energy (MeV)

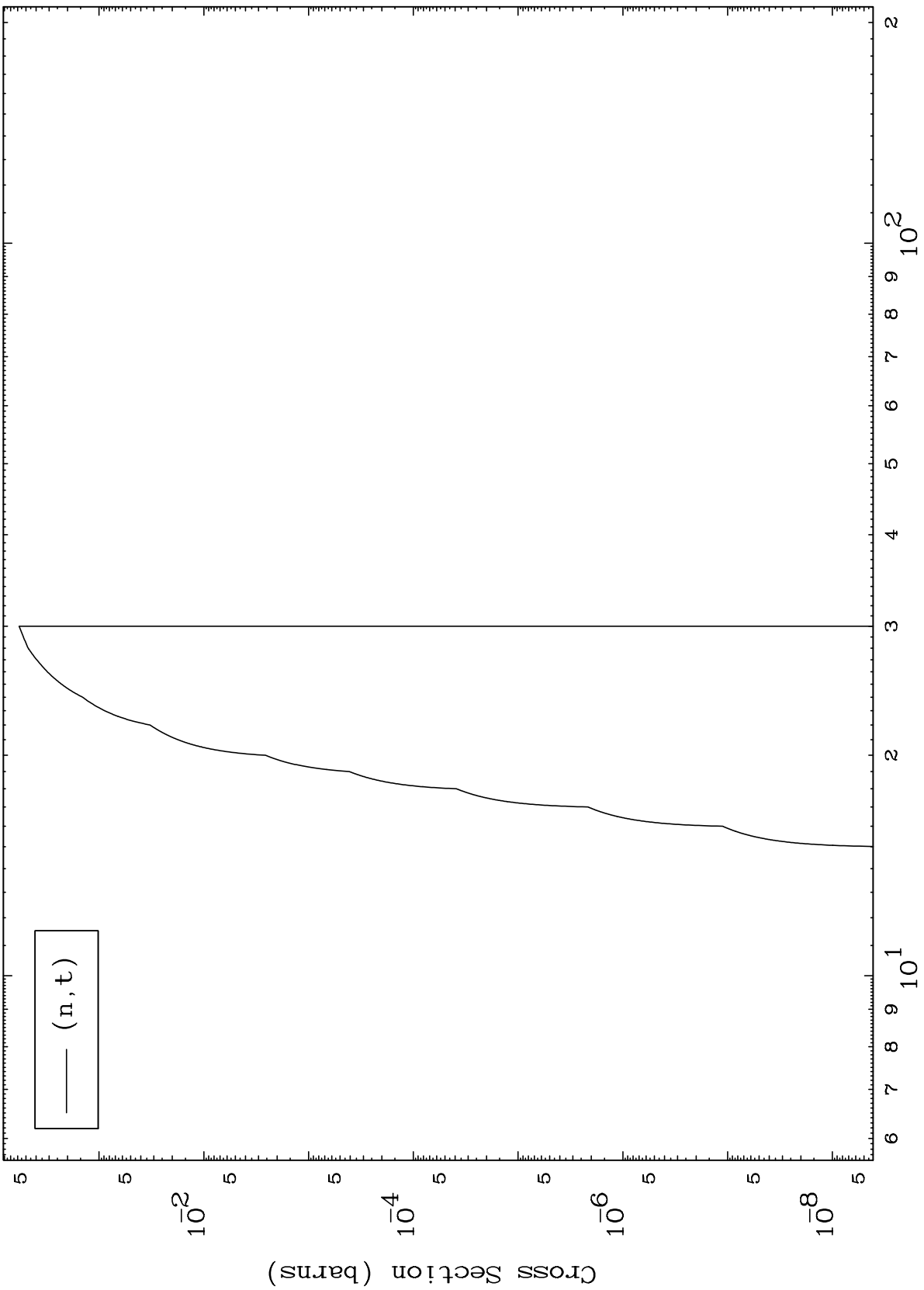
86-Rn-207

MAT 8613

(He-3,t) Levels

86-Rn-207

0 Kelvin Cross Sections



9

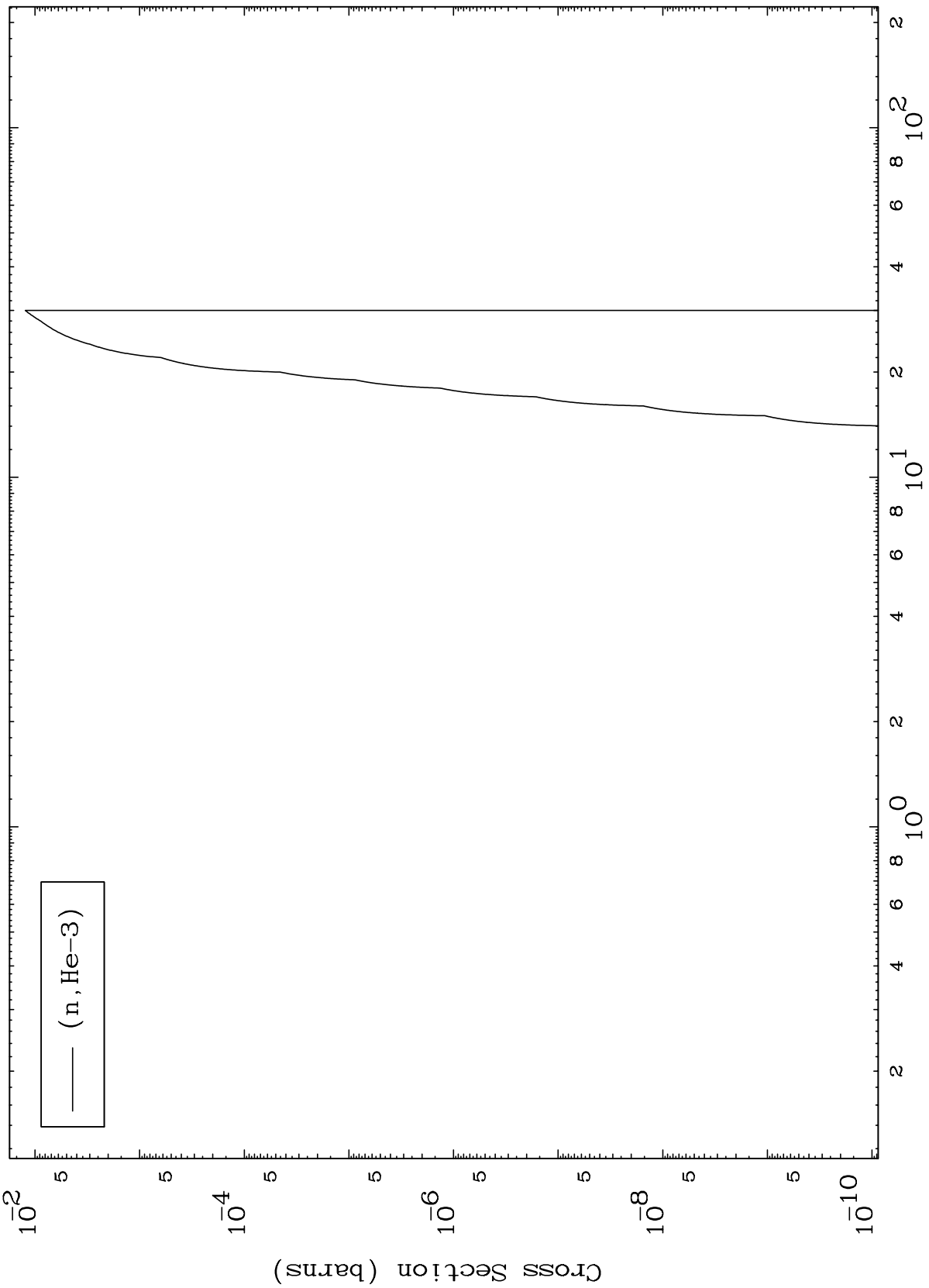
86-Rn-207

MAT 8613

(He-3, He3) Levels

86-Rn-207

0 Kelvin Cross Sections



10

Incident Energy (MeV)

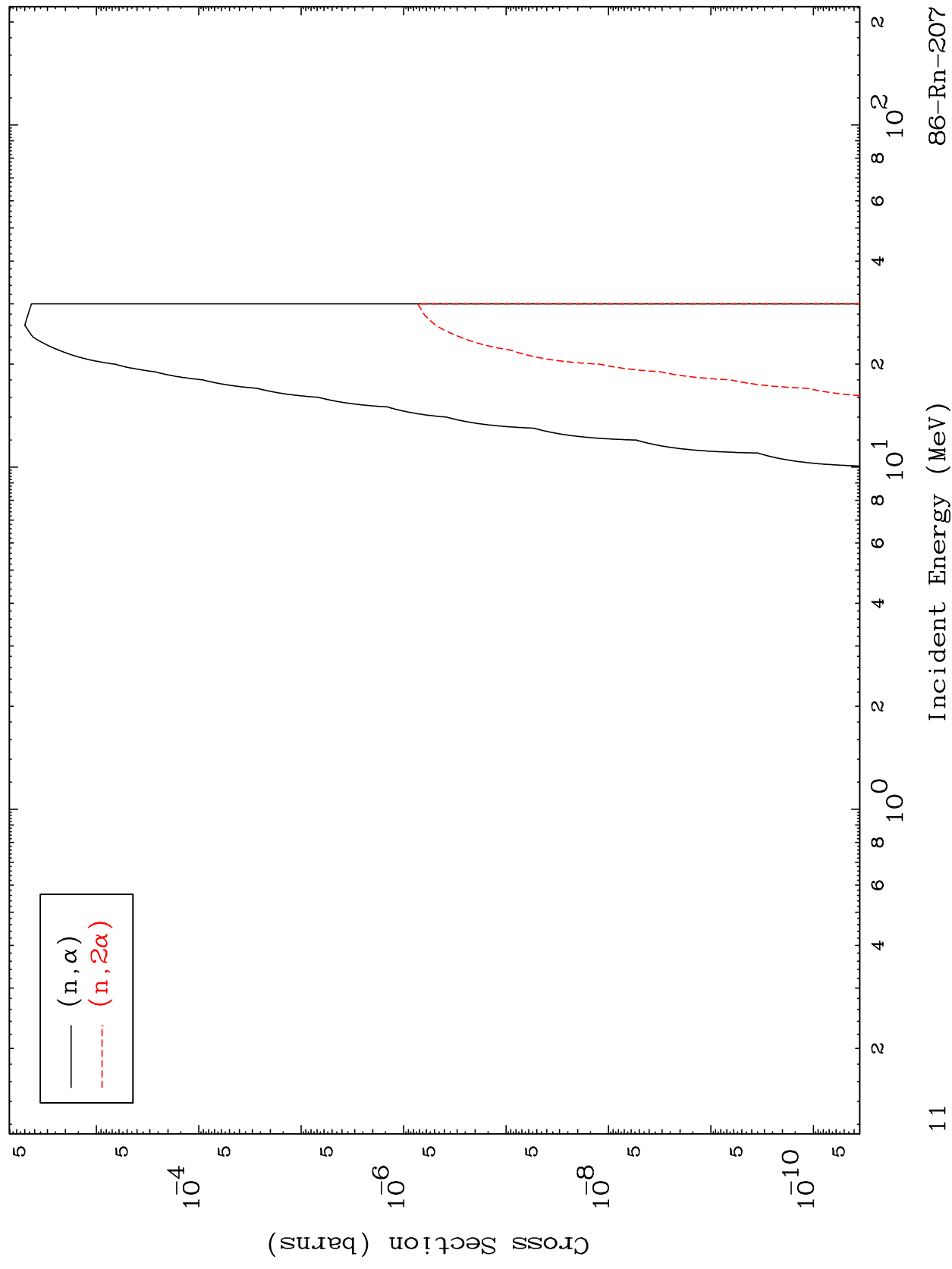
86-Rn-207

MAT 8613

(He-3, α) Levels

86-Rn-207

0 Kelvin Cross Sections

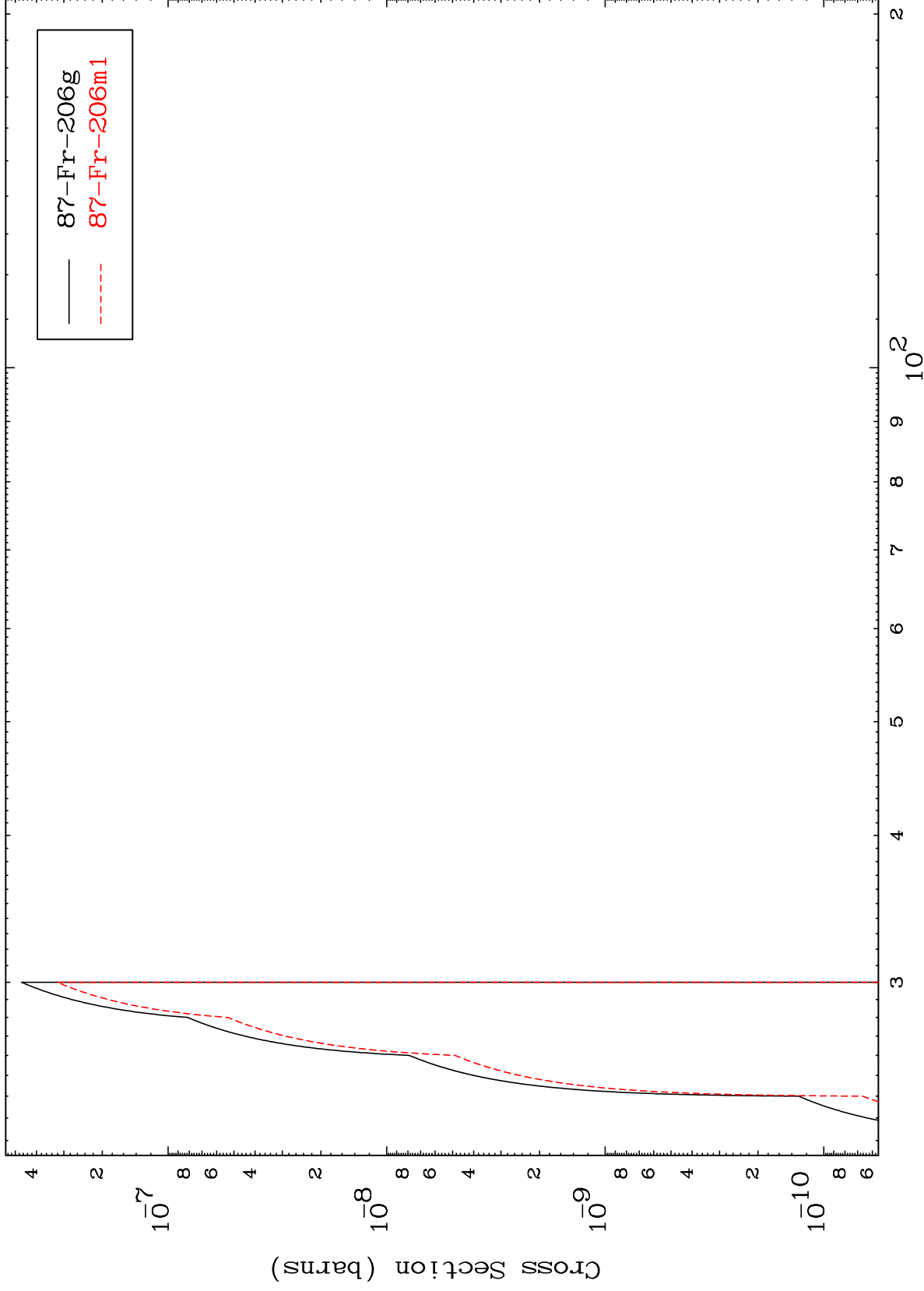


MAT 8613

(n,2n) d

86-Rn-207

Radionuclide Production Cross Section



12

Incident Energy (MeV)

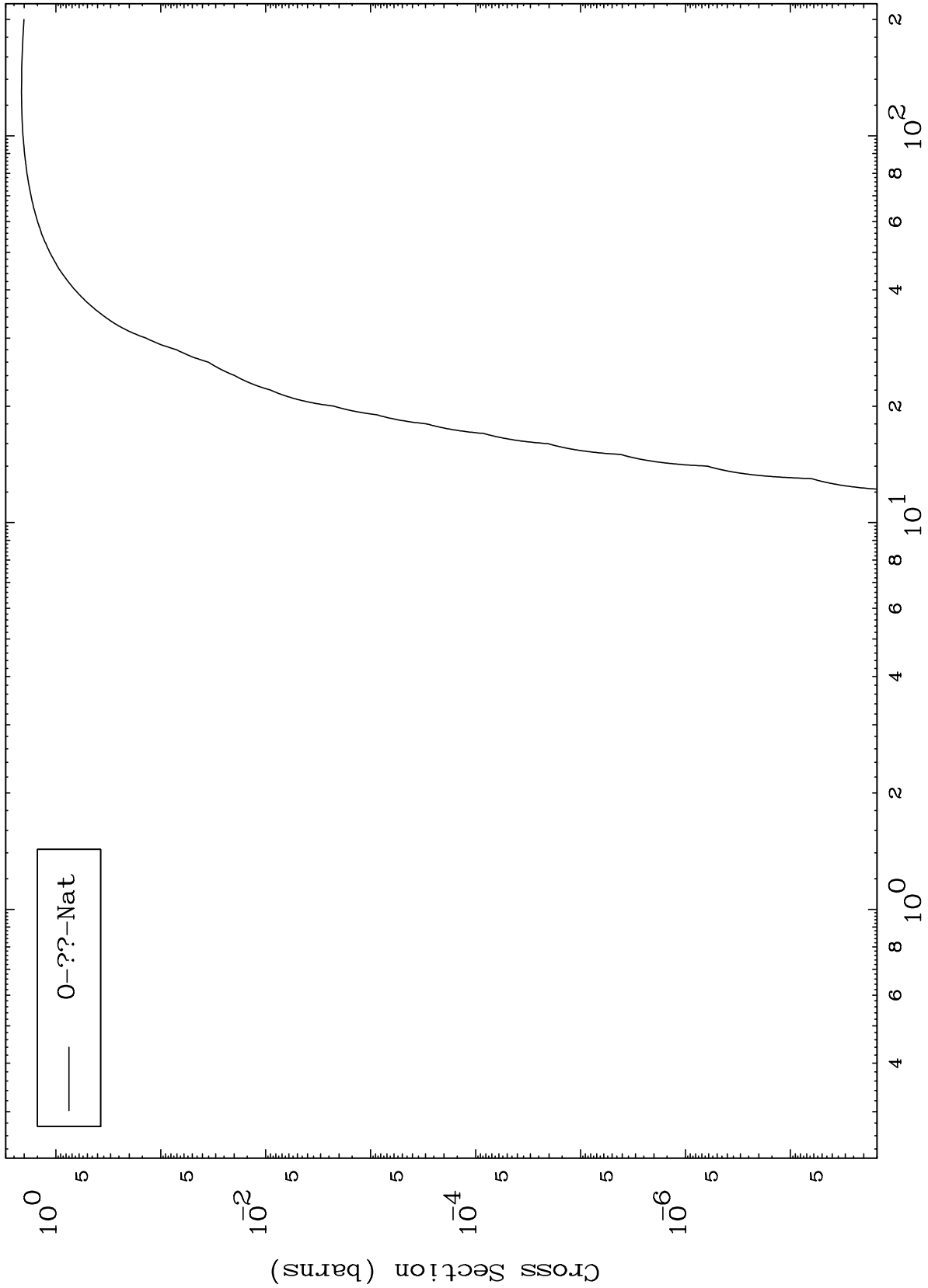
86-Rn-207

MAT 8613

Fission

86-Rn-207

Radionuclide Production Cross Section

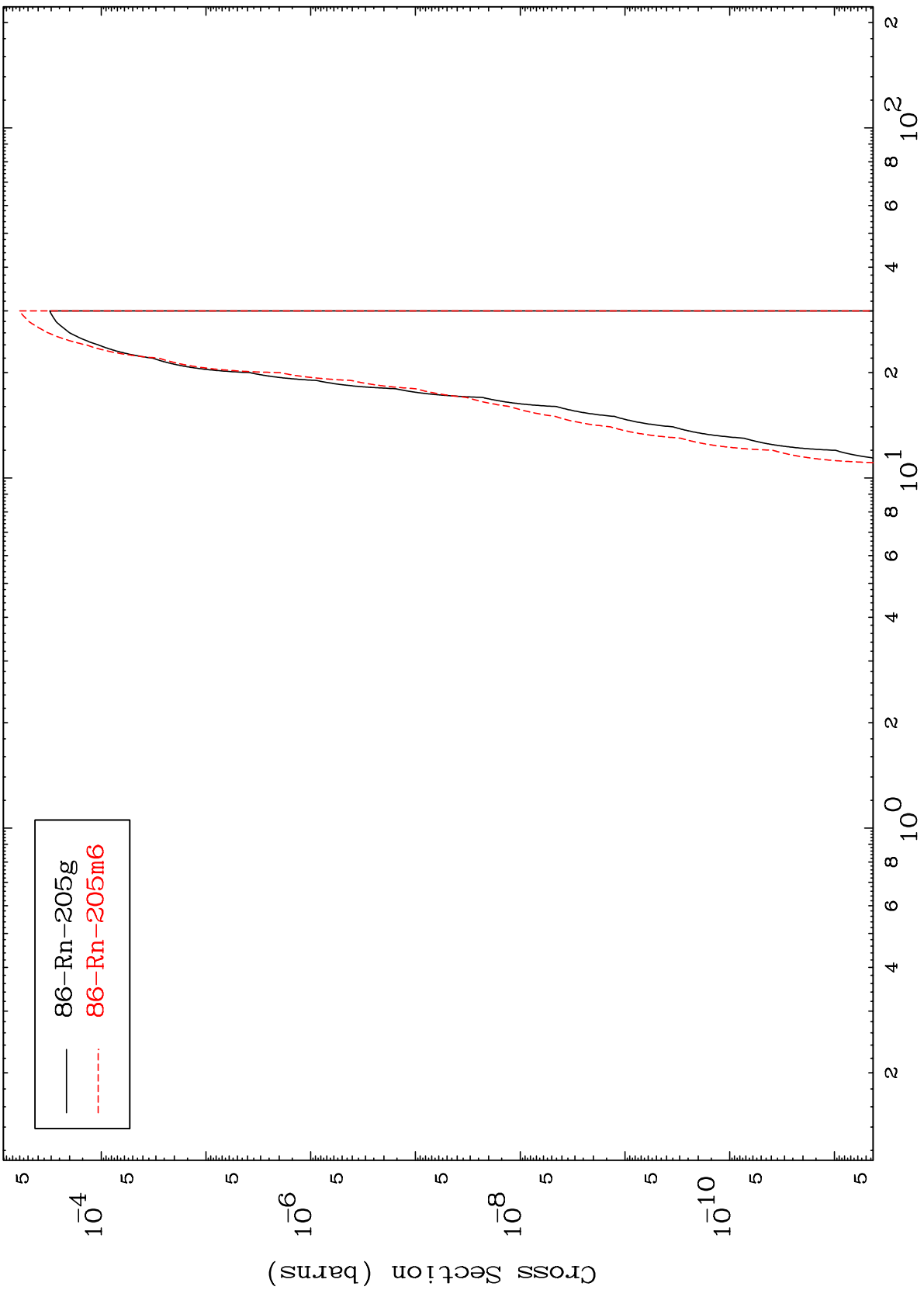


MAT 8613

$(n, n') \alpha$

86-Rn-207

Radionuclide Production Cross Section

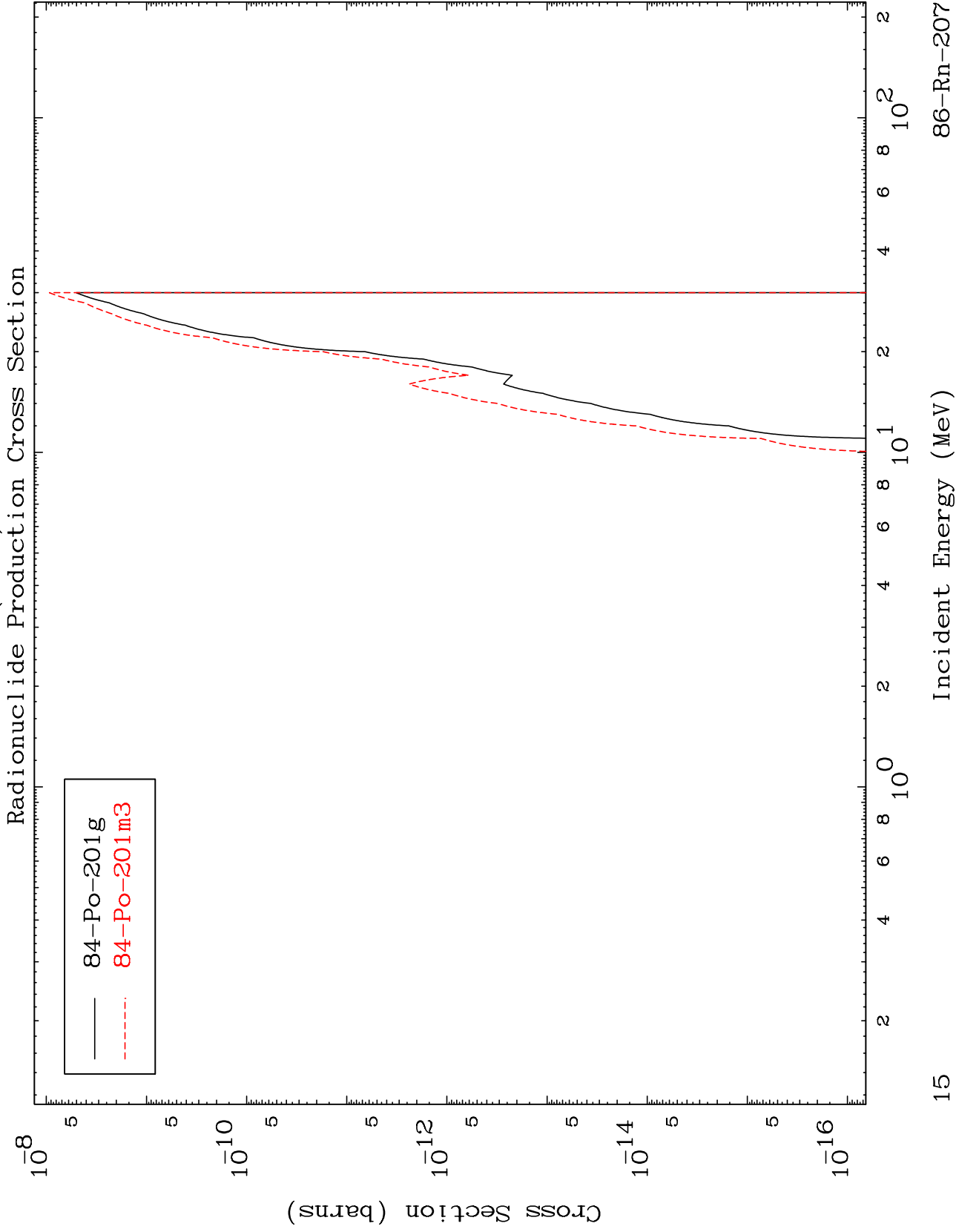


86-Rn-205g
86-Rn-205m6

MAT 8613

(n,n') 2α

86-Rn-207

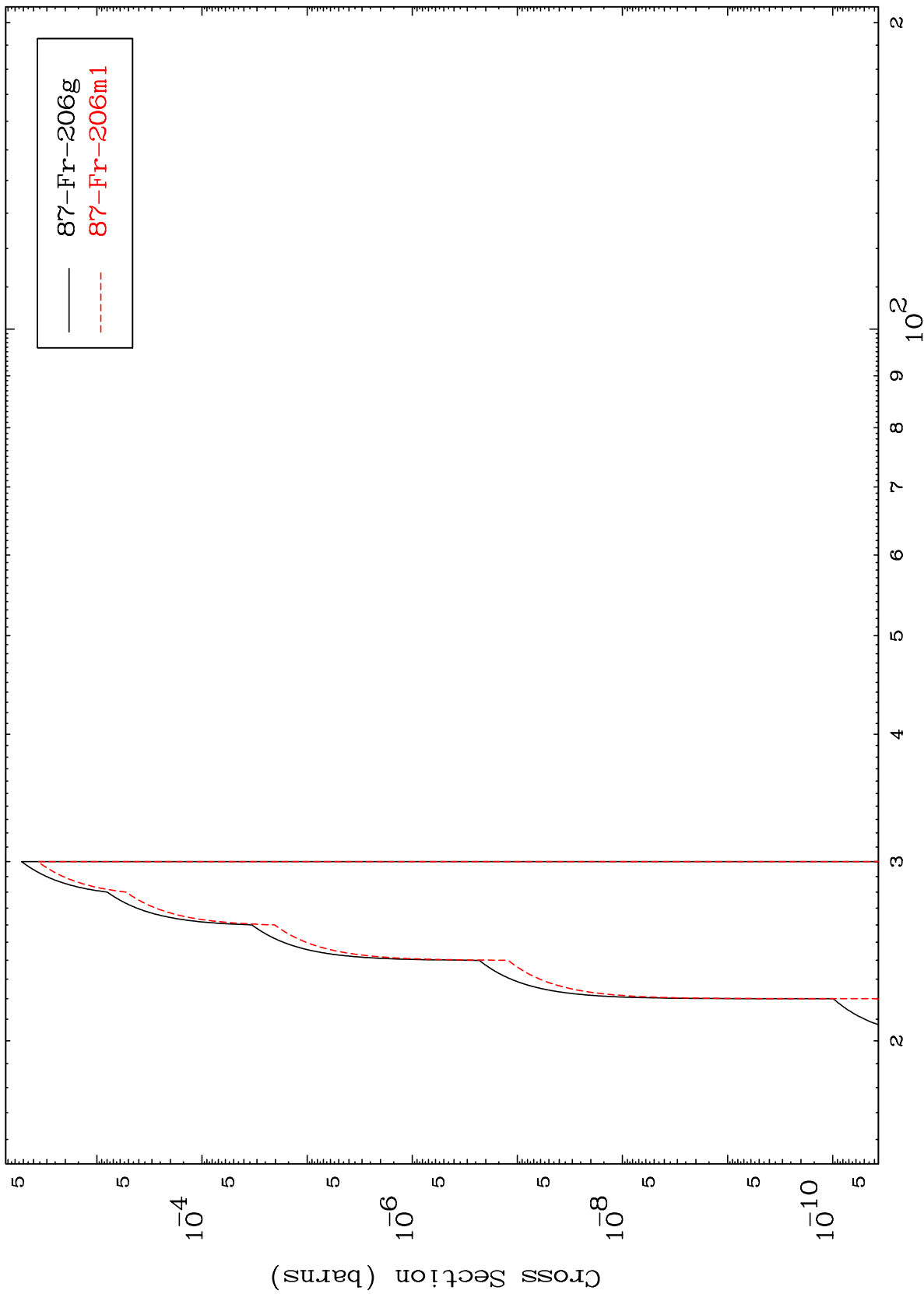


MAT 8613

(n,n') t

86-Rn-207

Radionuclide Production Cross Section



16

Incident Energy (MeV)

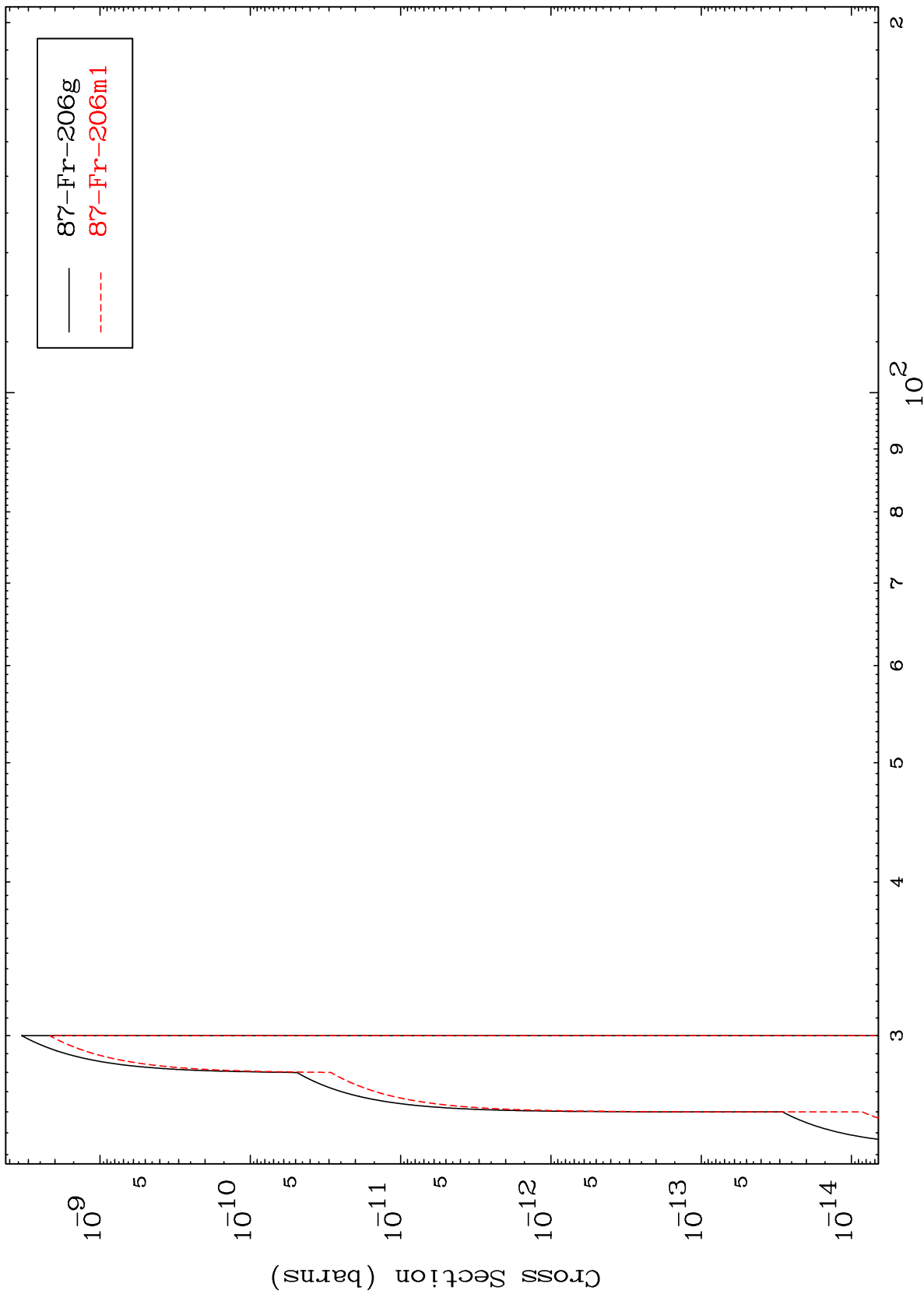
86-Rn-207

MAT 8613

(n,3n) p

86-Rn-207

Radionuclide Production Cross Section



87-Fr-206g
87-Fr-206m1

17

Incident Energy (MeV)

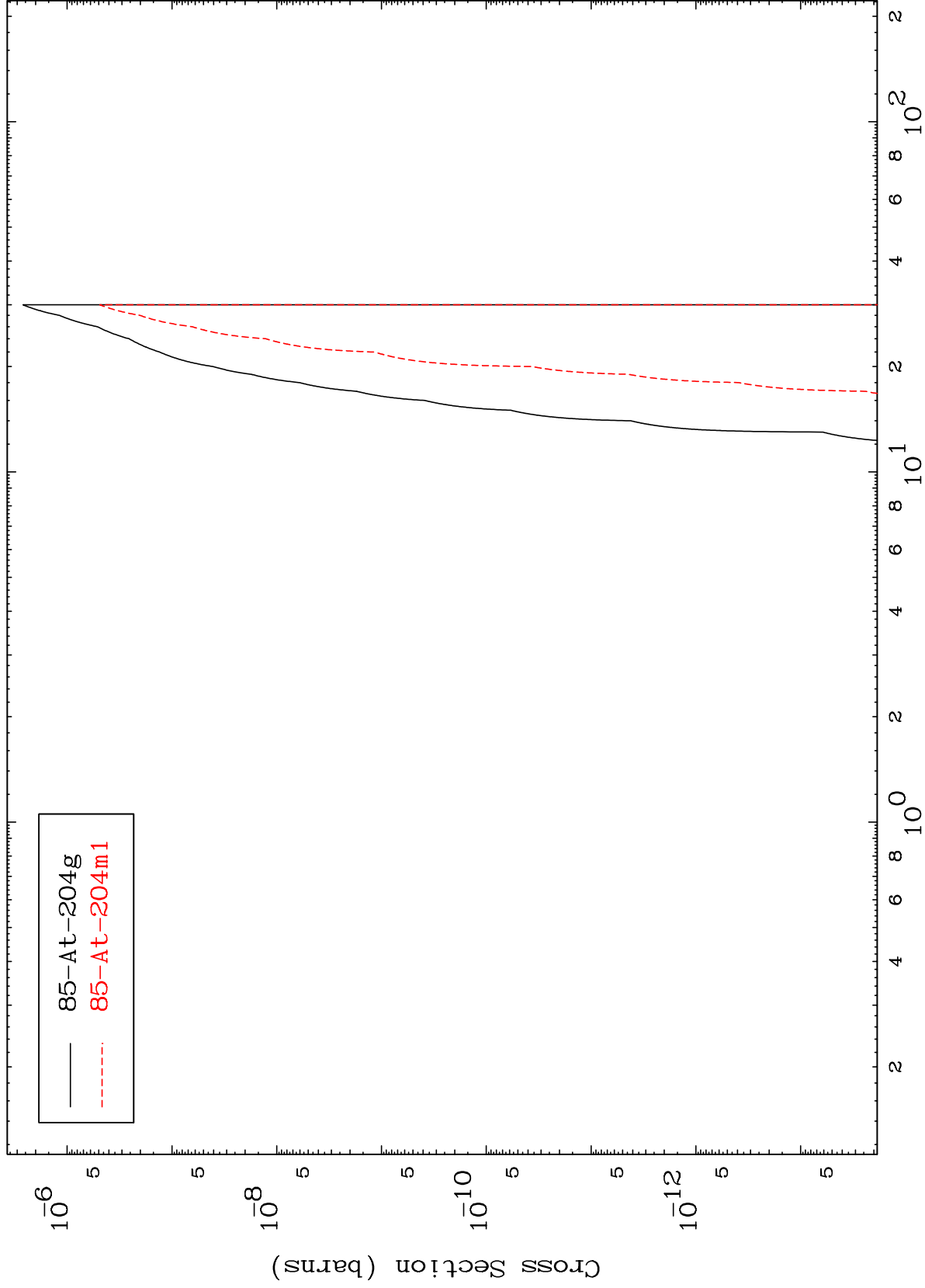
86-Rn-207

MAT 8613

(n,n') p α

86-Rn-207

Radionuclide Production Cross Section



18

Incident Energy (MeV)

86-Rn-207

MAT 8613

(n,d) α

86-Rn-207

