

Program EVALPLOT
(Version 2021-1)

by

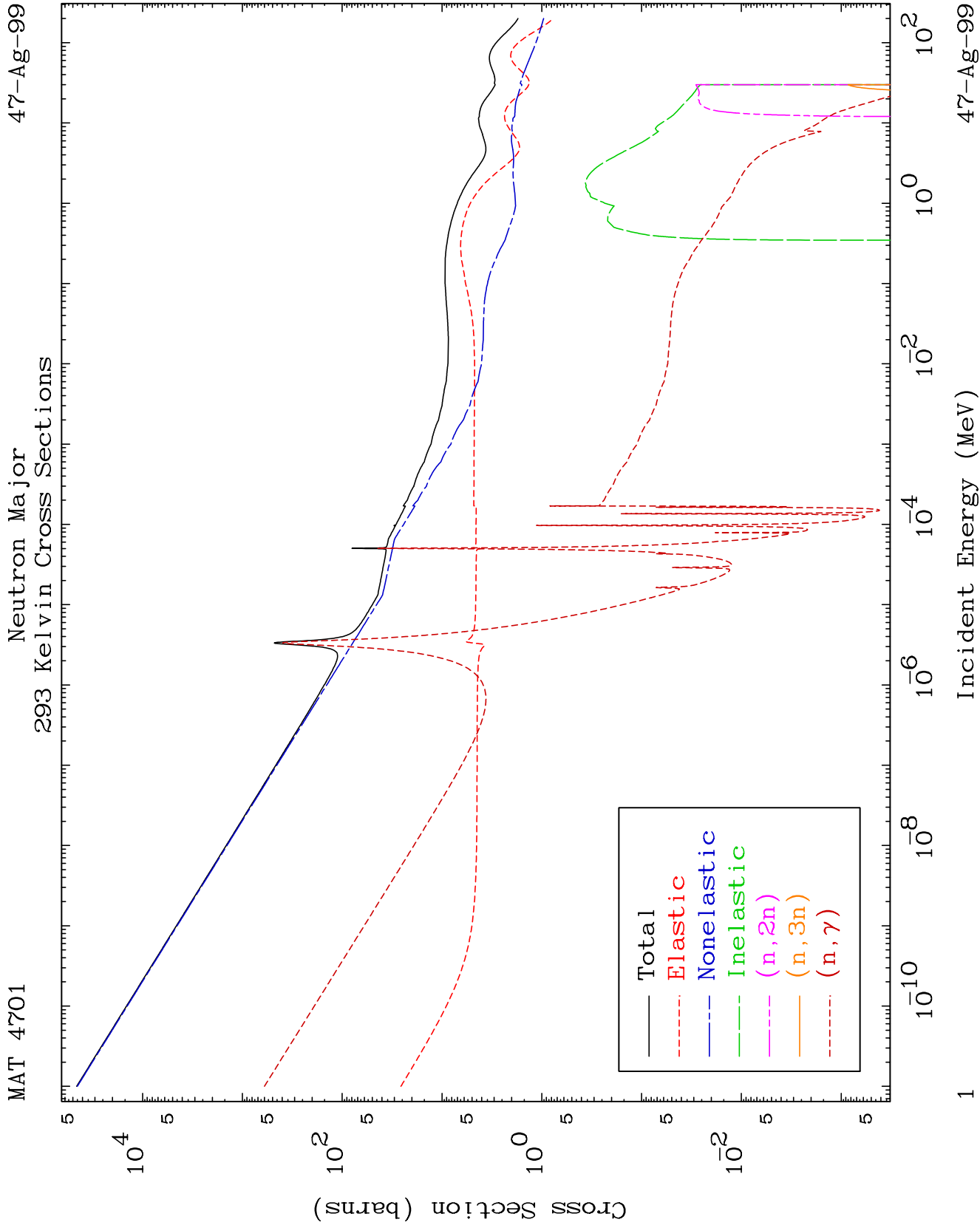
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(Present Contact Information)

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Press Mouse Button to Start

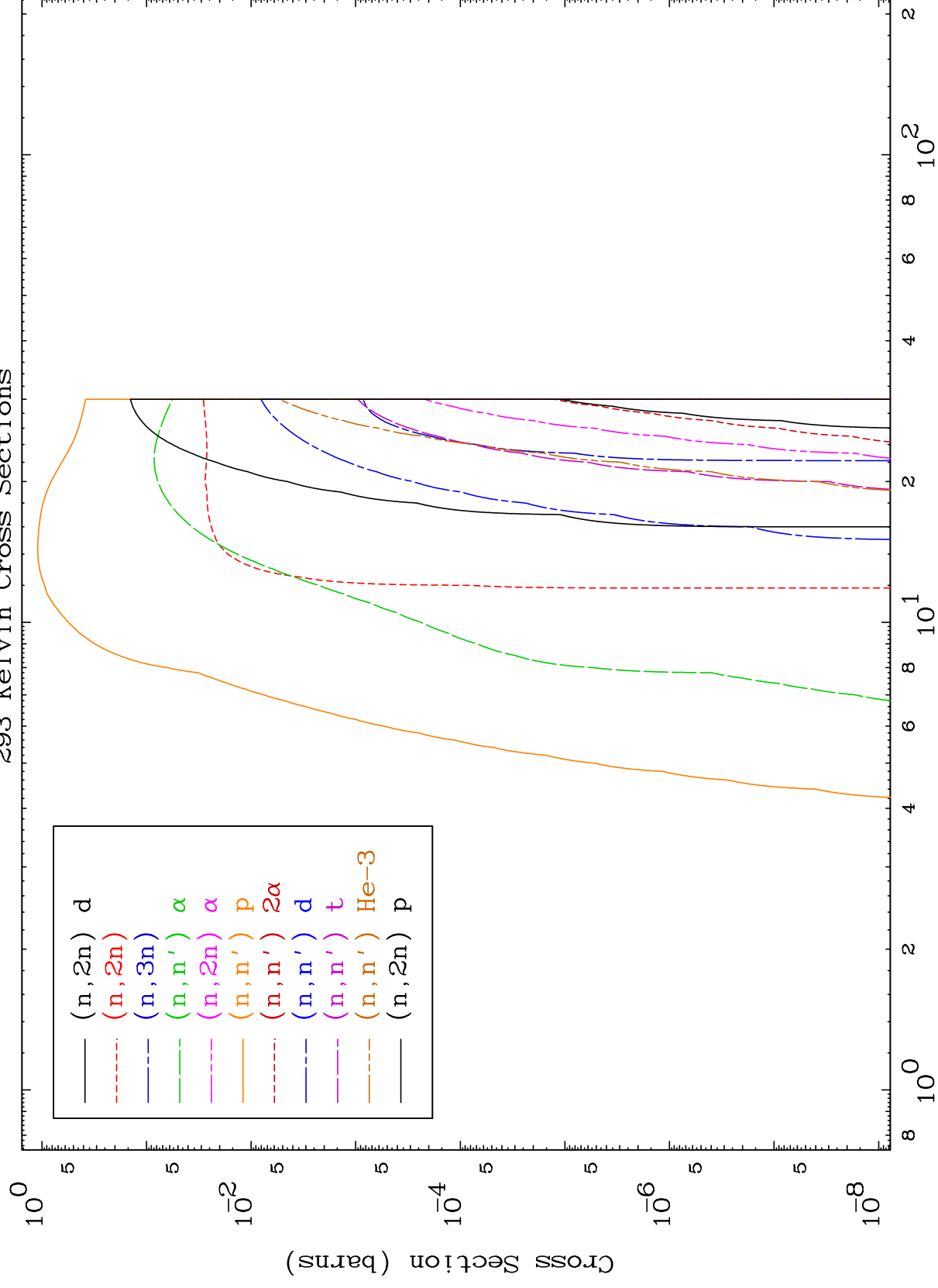


MAT 4701

Neutron Absorption

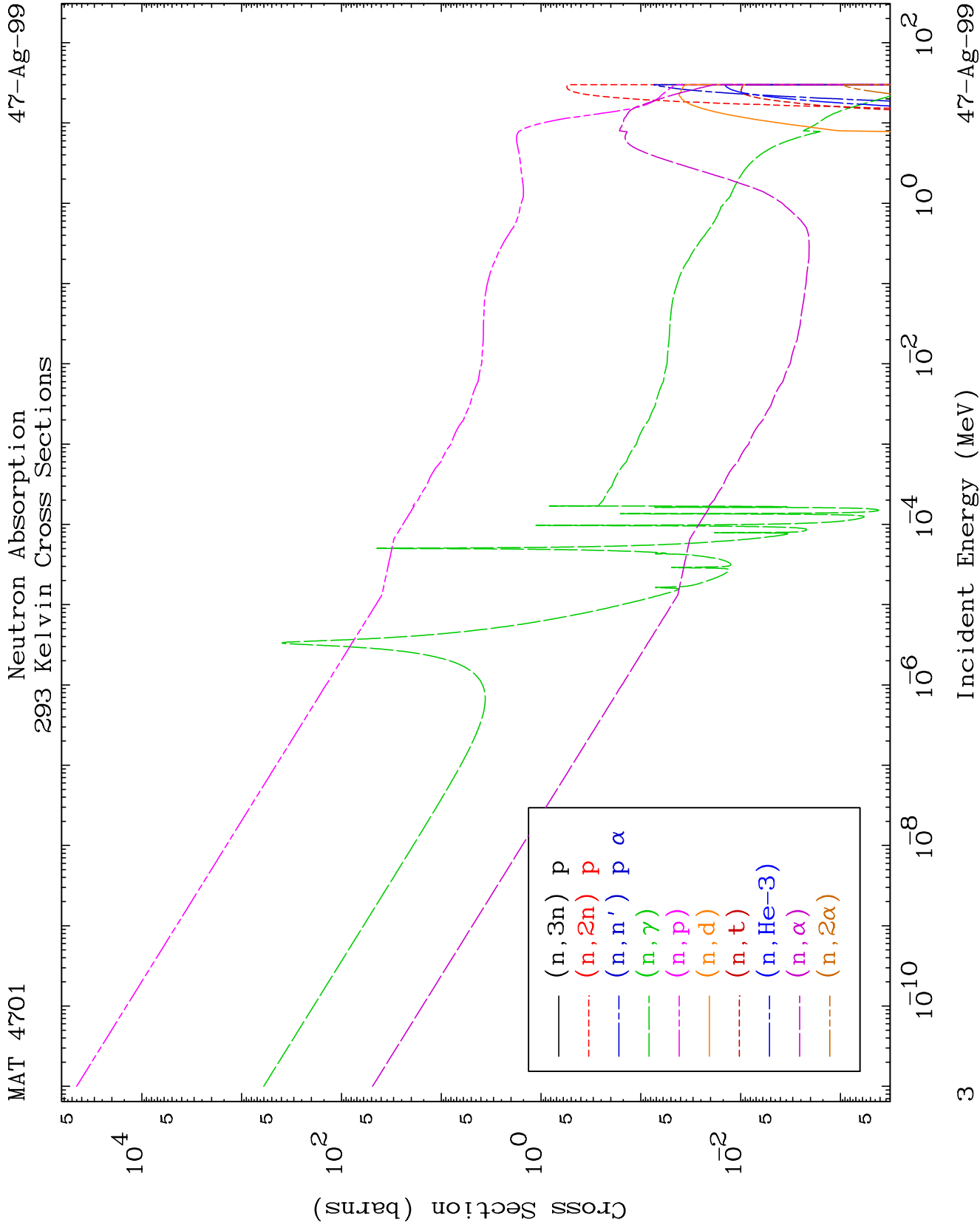
Neutron Absorption
293 Kelvin Cross Sections

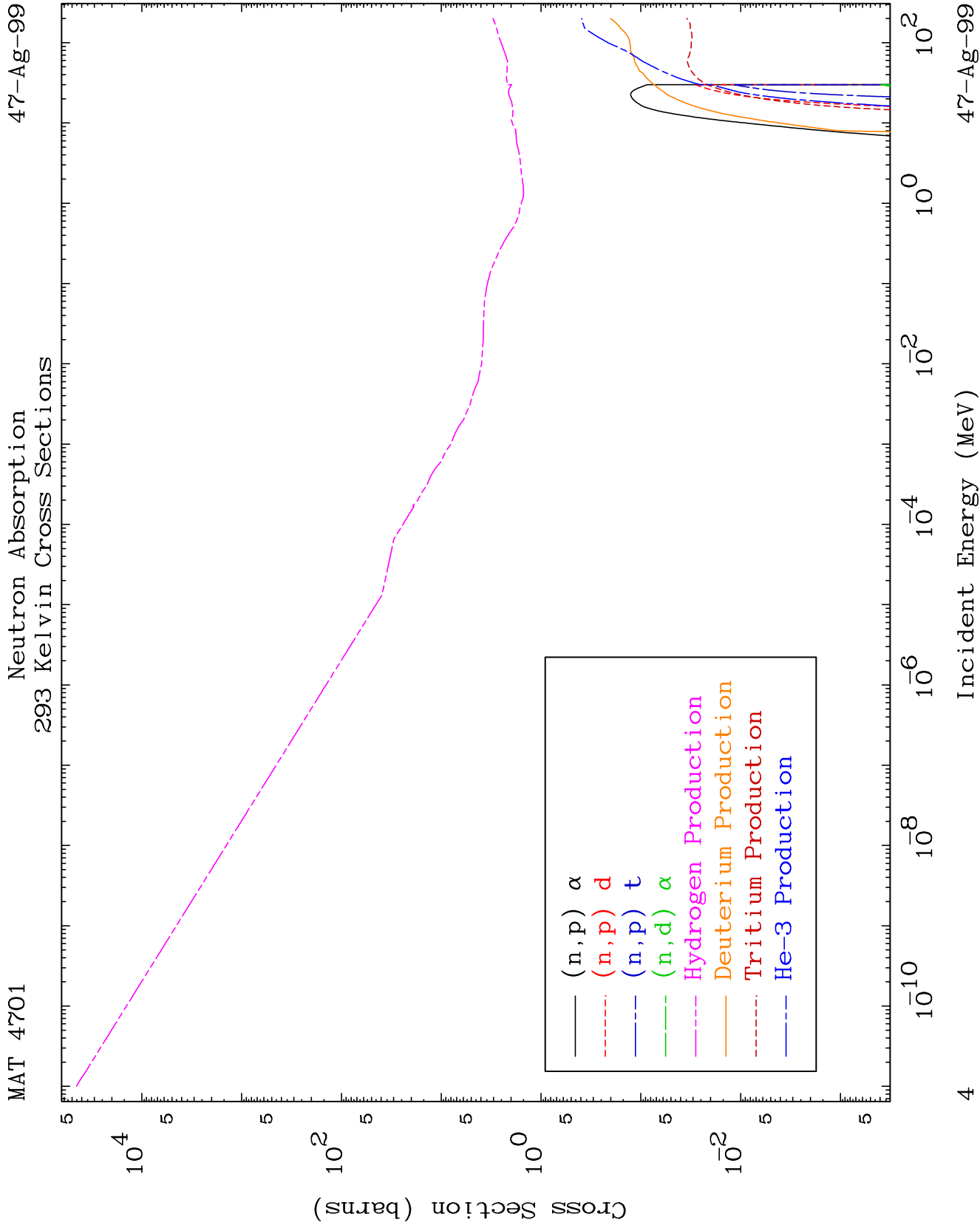
47-Ag-99

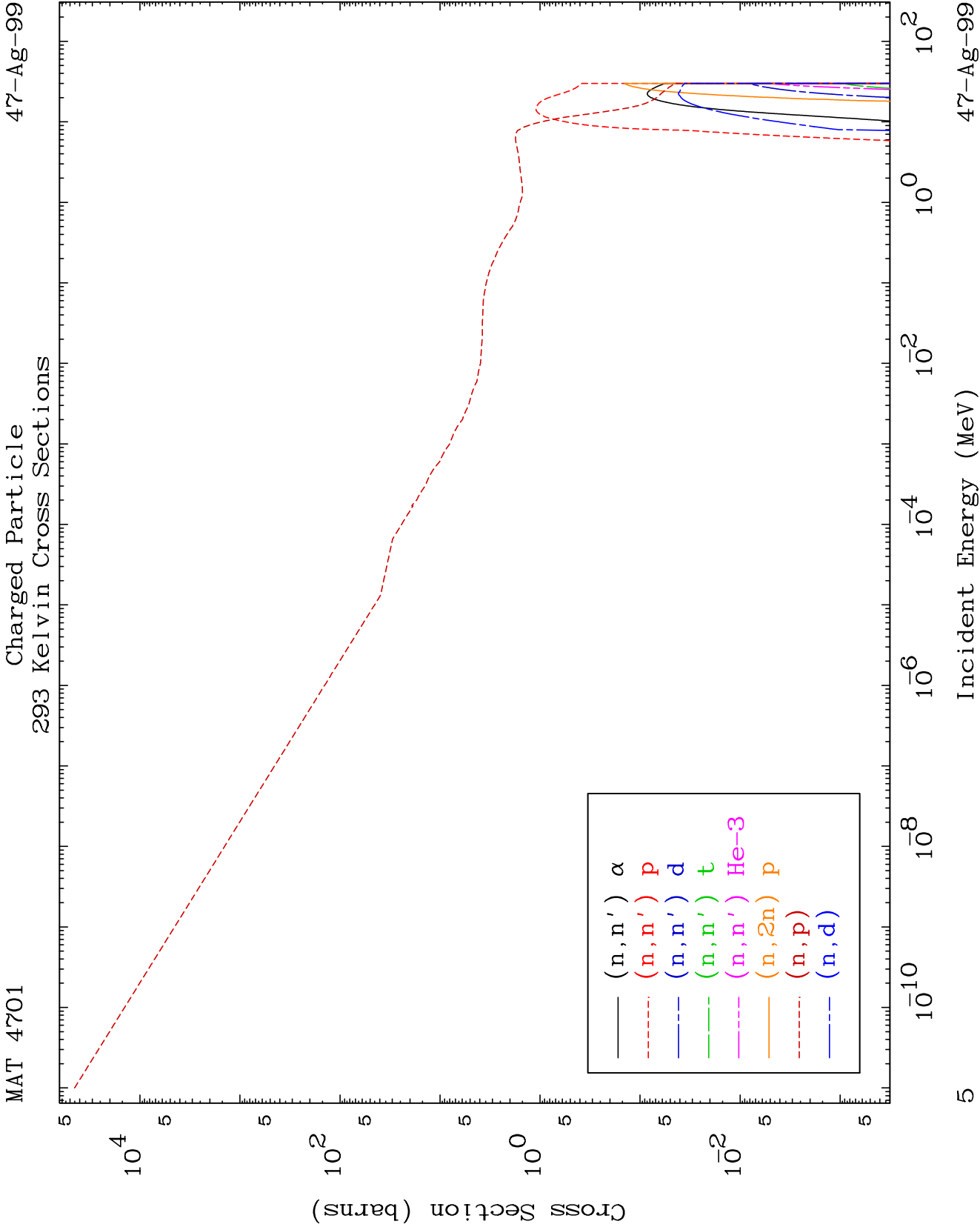


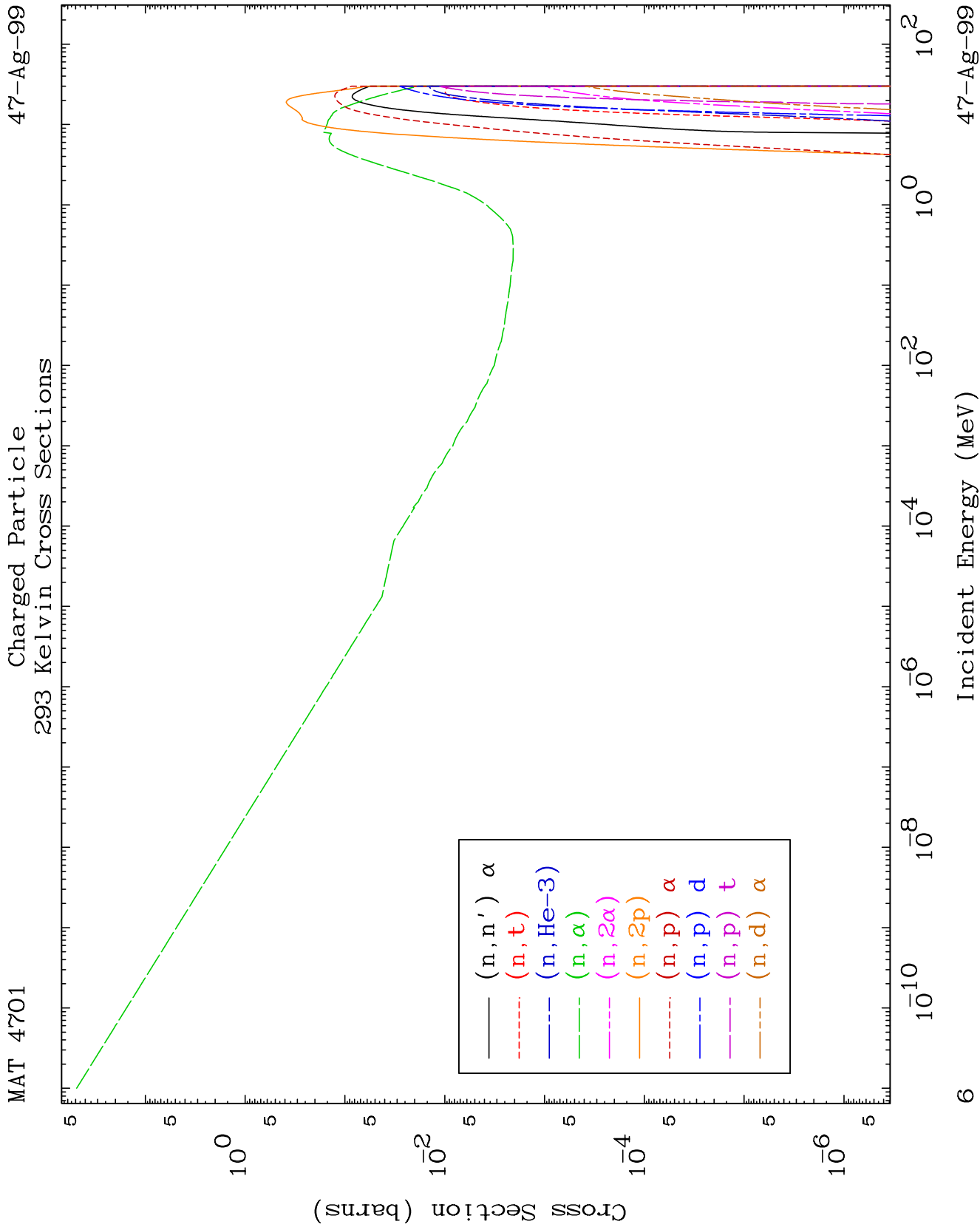
Incident Energy (MeV)

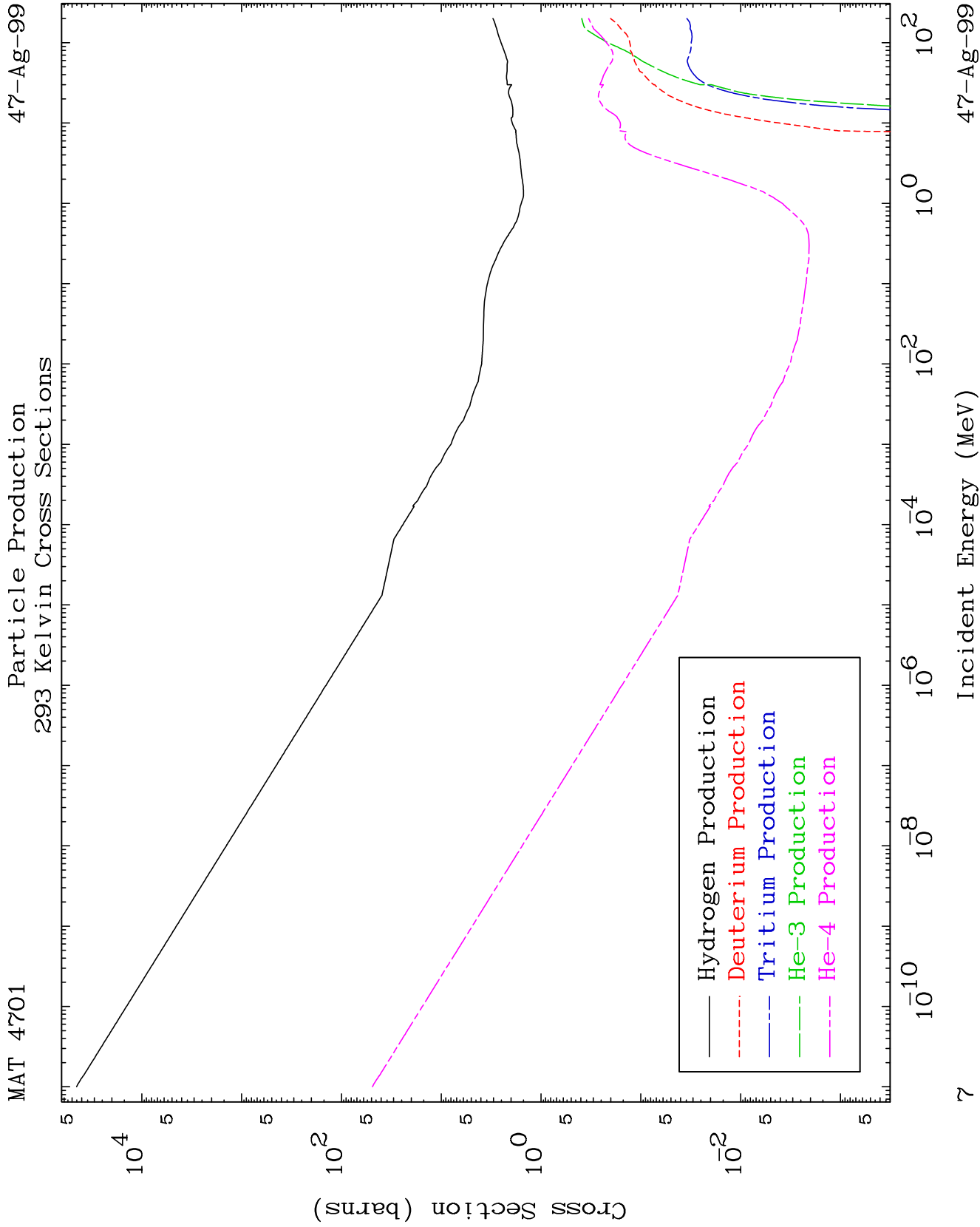
47-A8-99









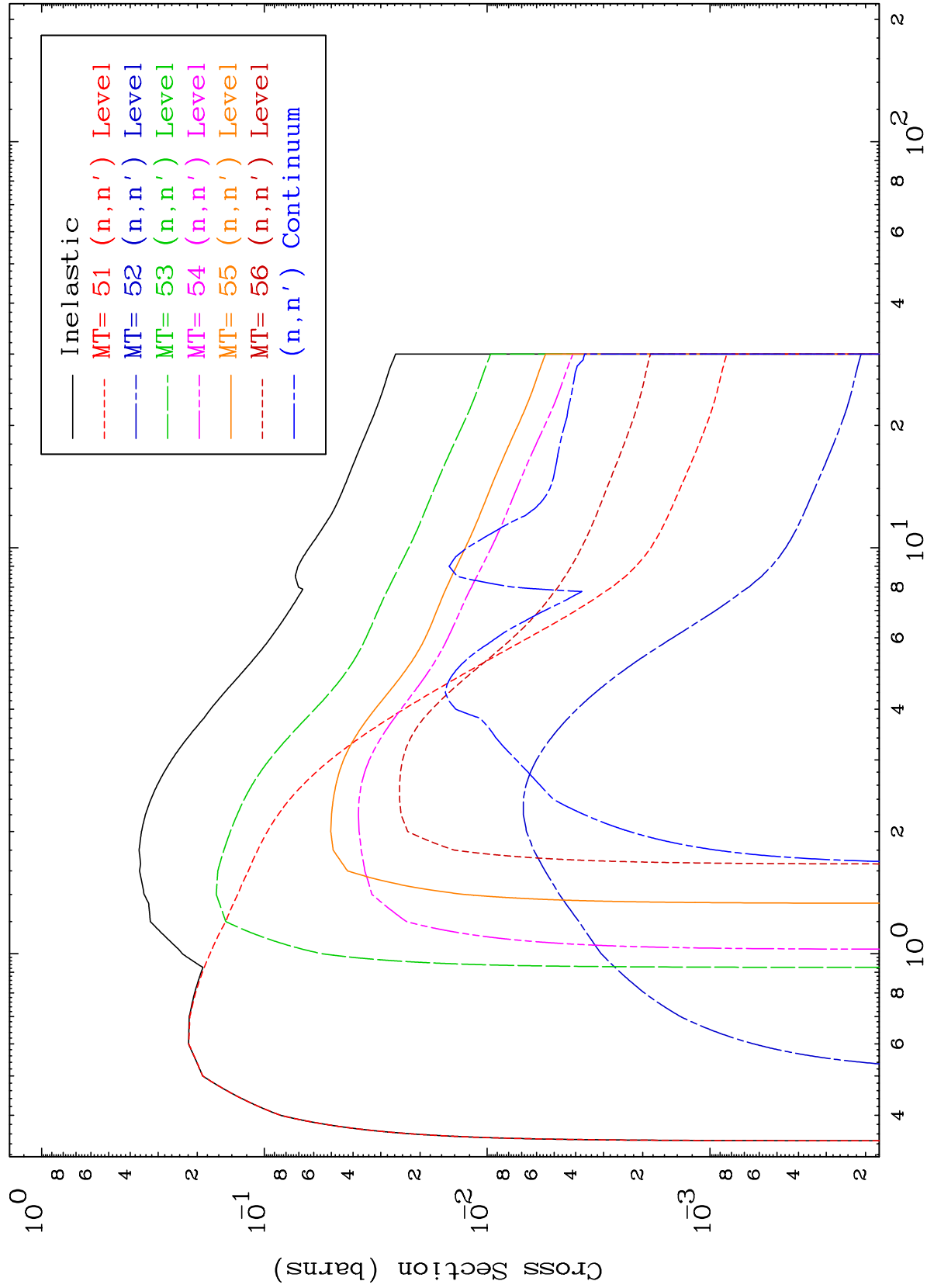


MAT 4701

(n,n') Levels

47-Ag-99

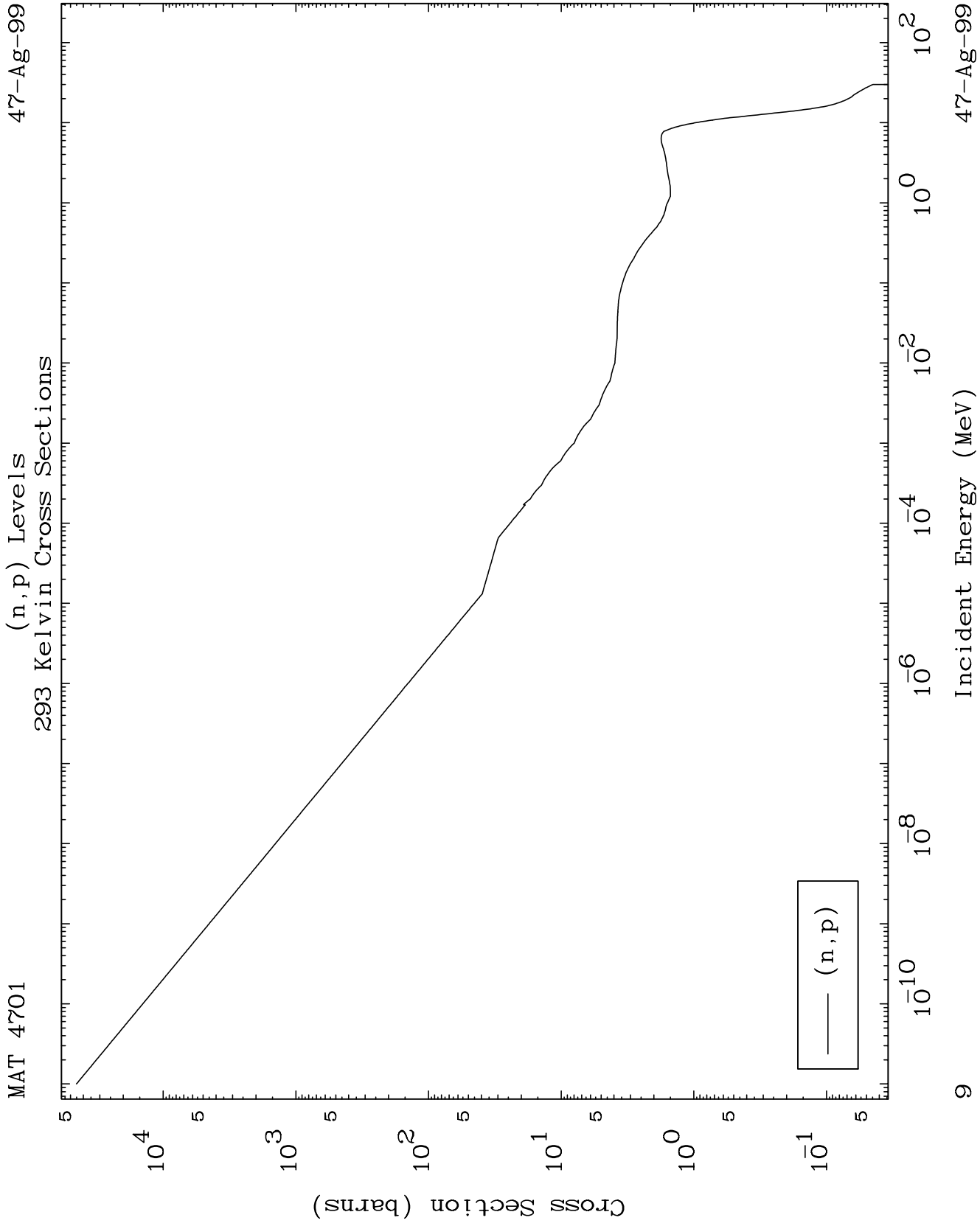
293 Kelvin Cross Sections

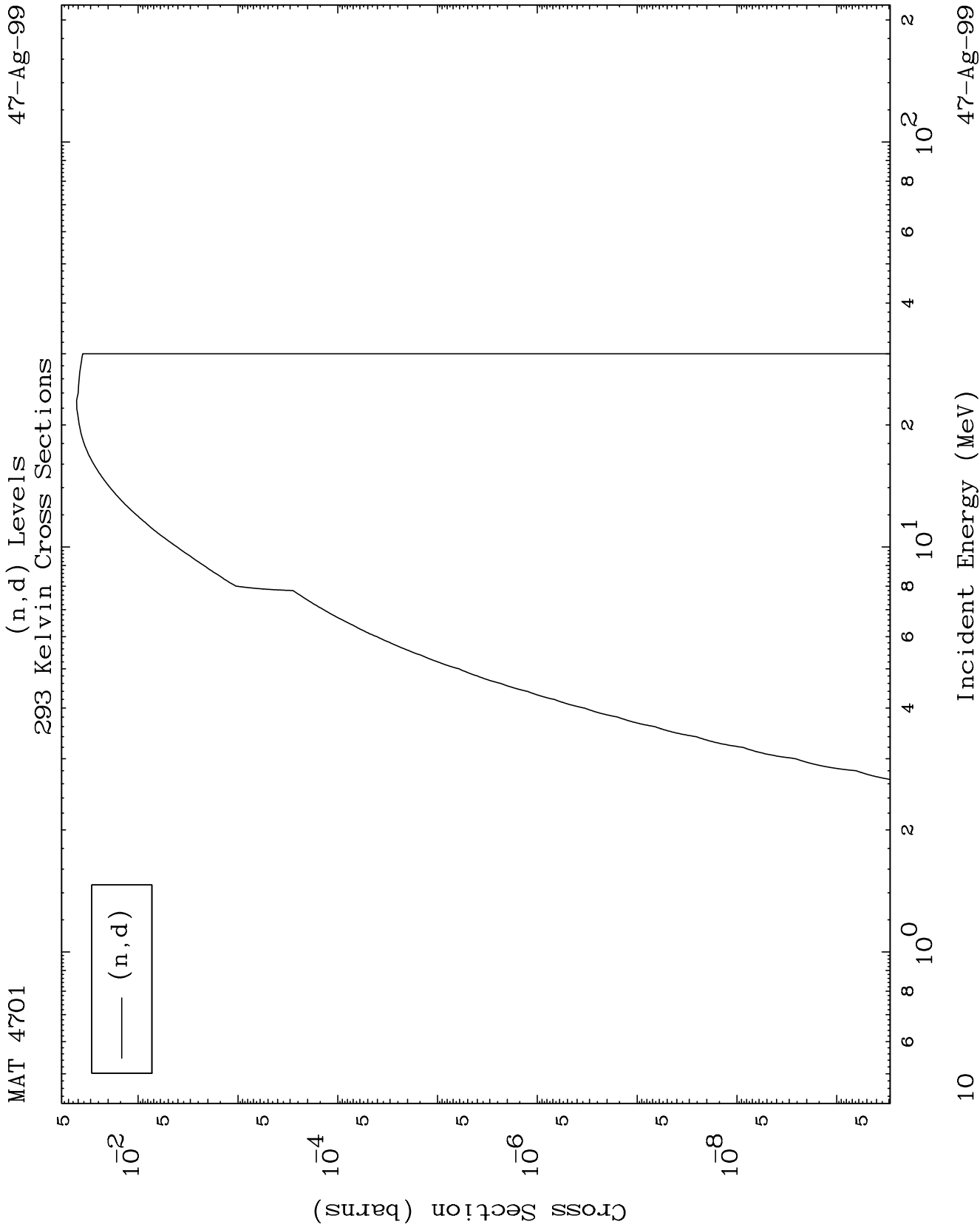


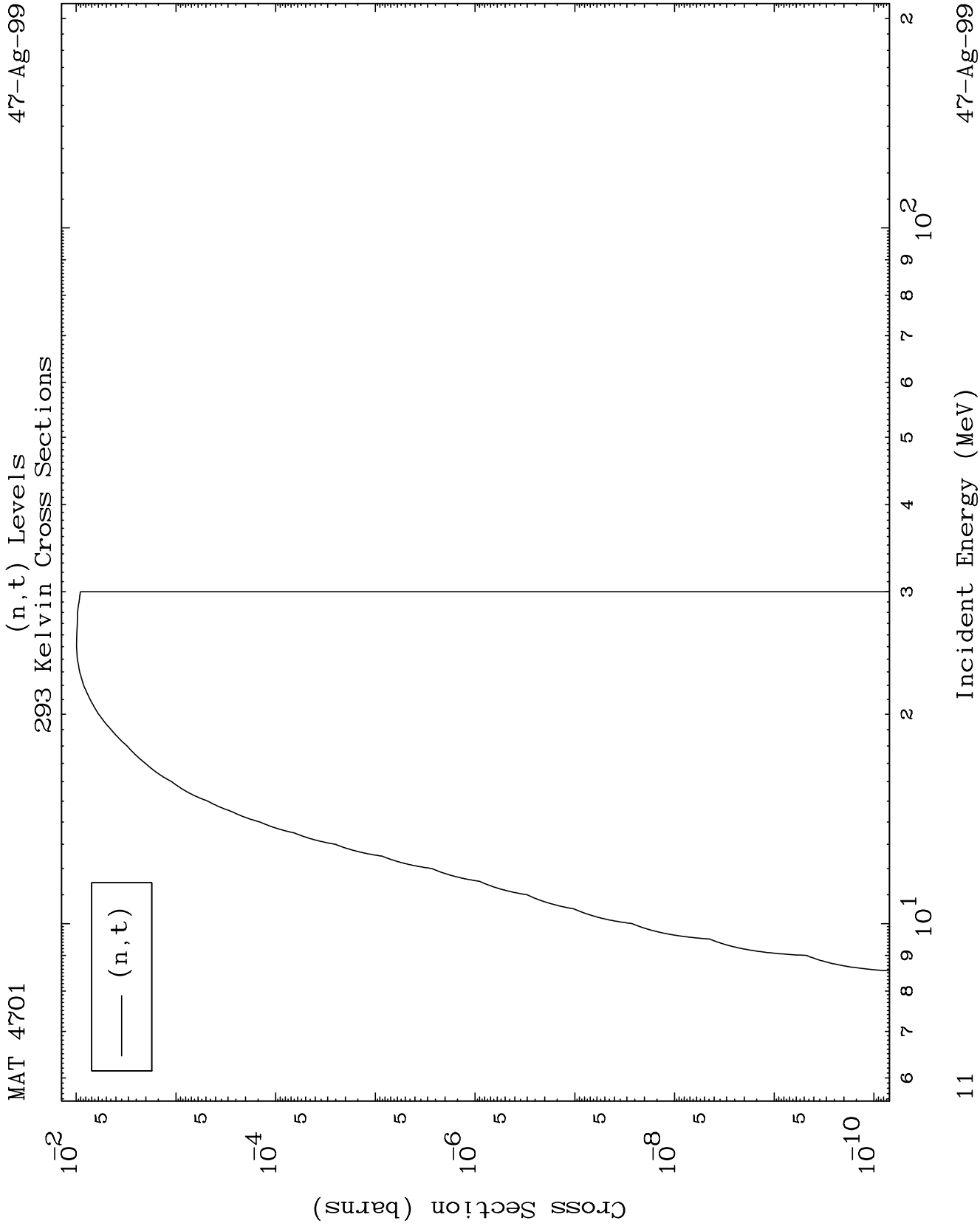
8

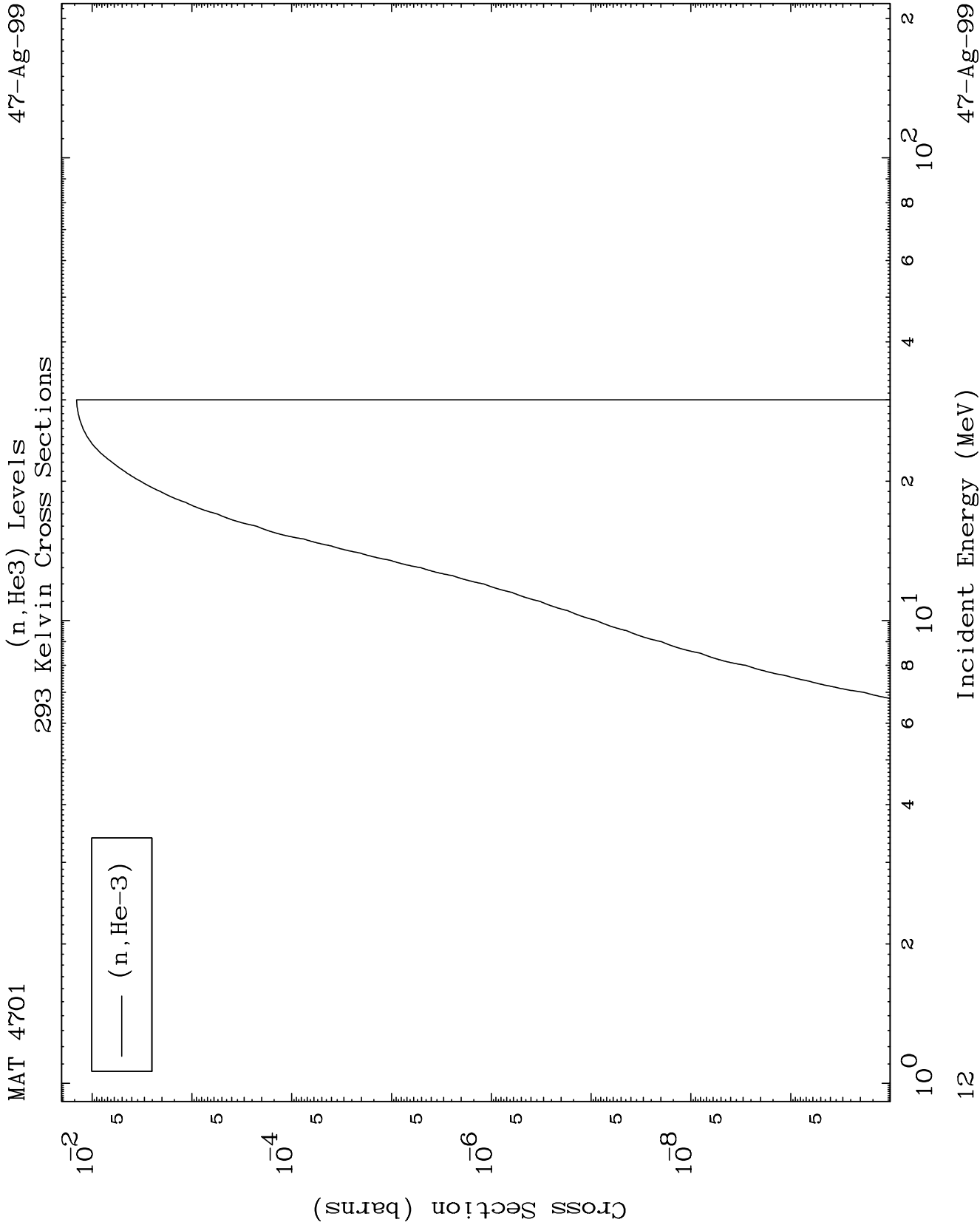
Incident Energy (MeV)

47-Ag-99





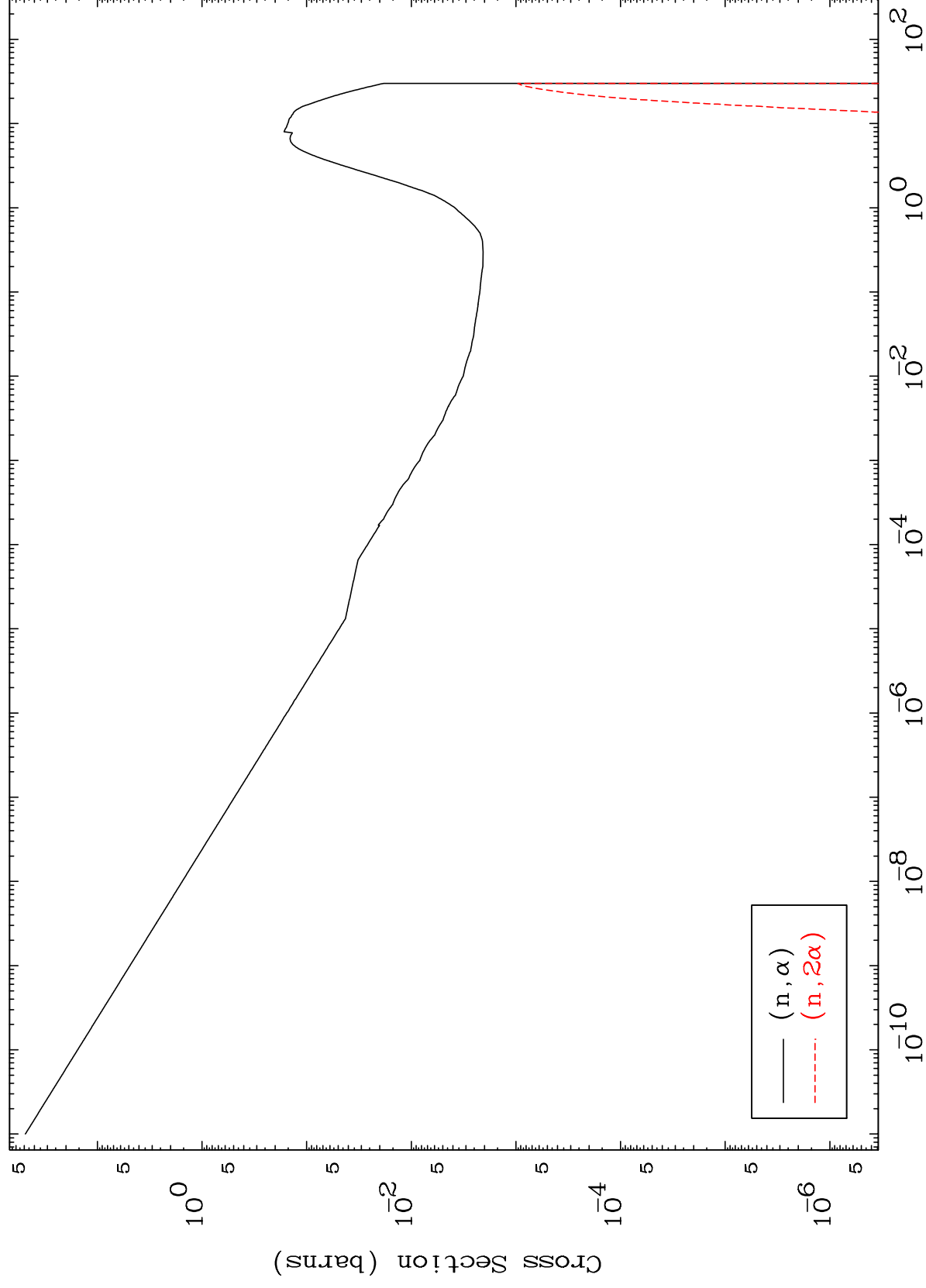




MAT 4701

(n, α) Levels
293 Kelvin Cross Sections

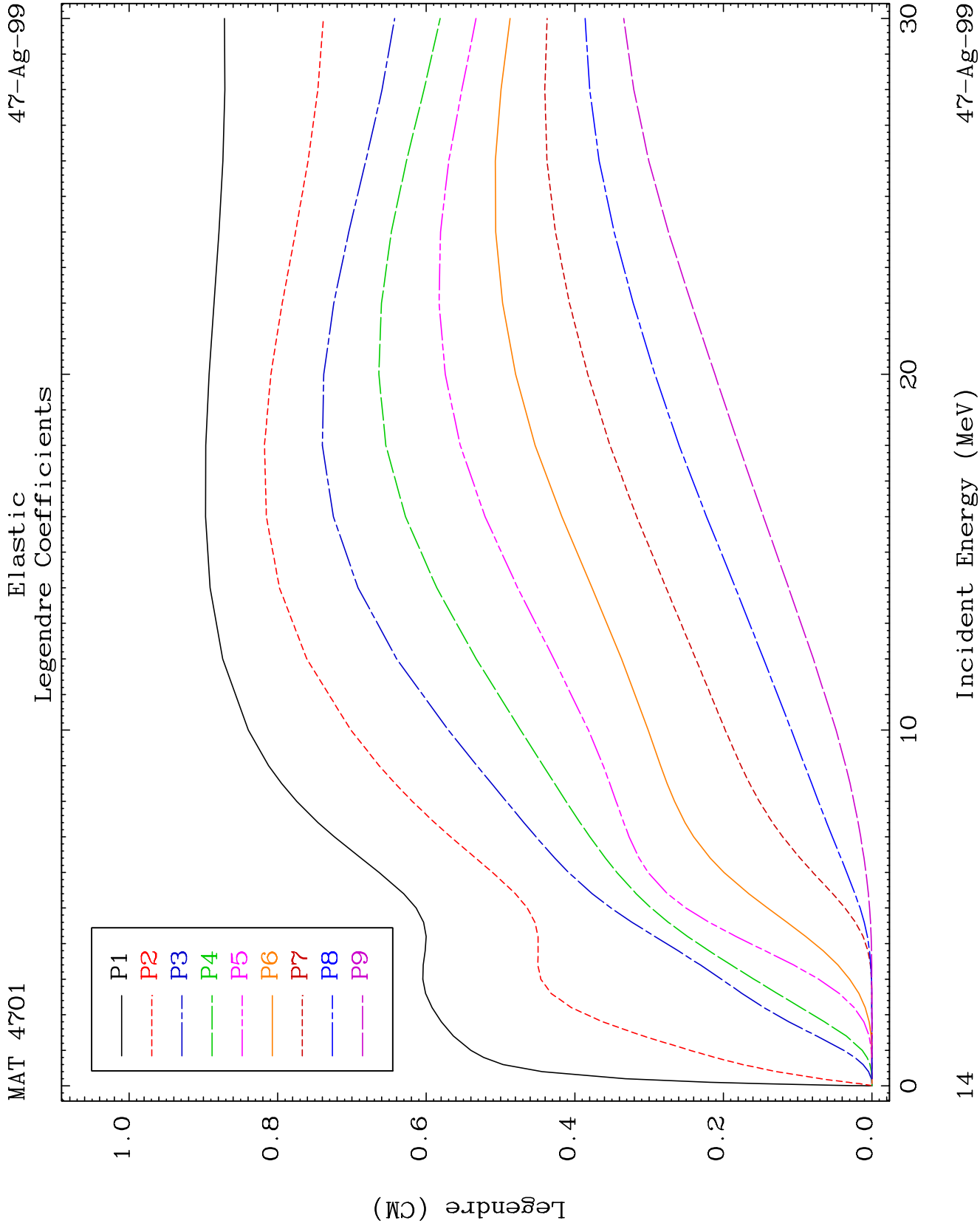
47-Ag-99

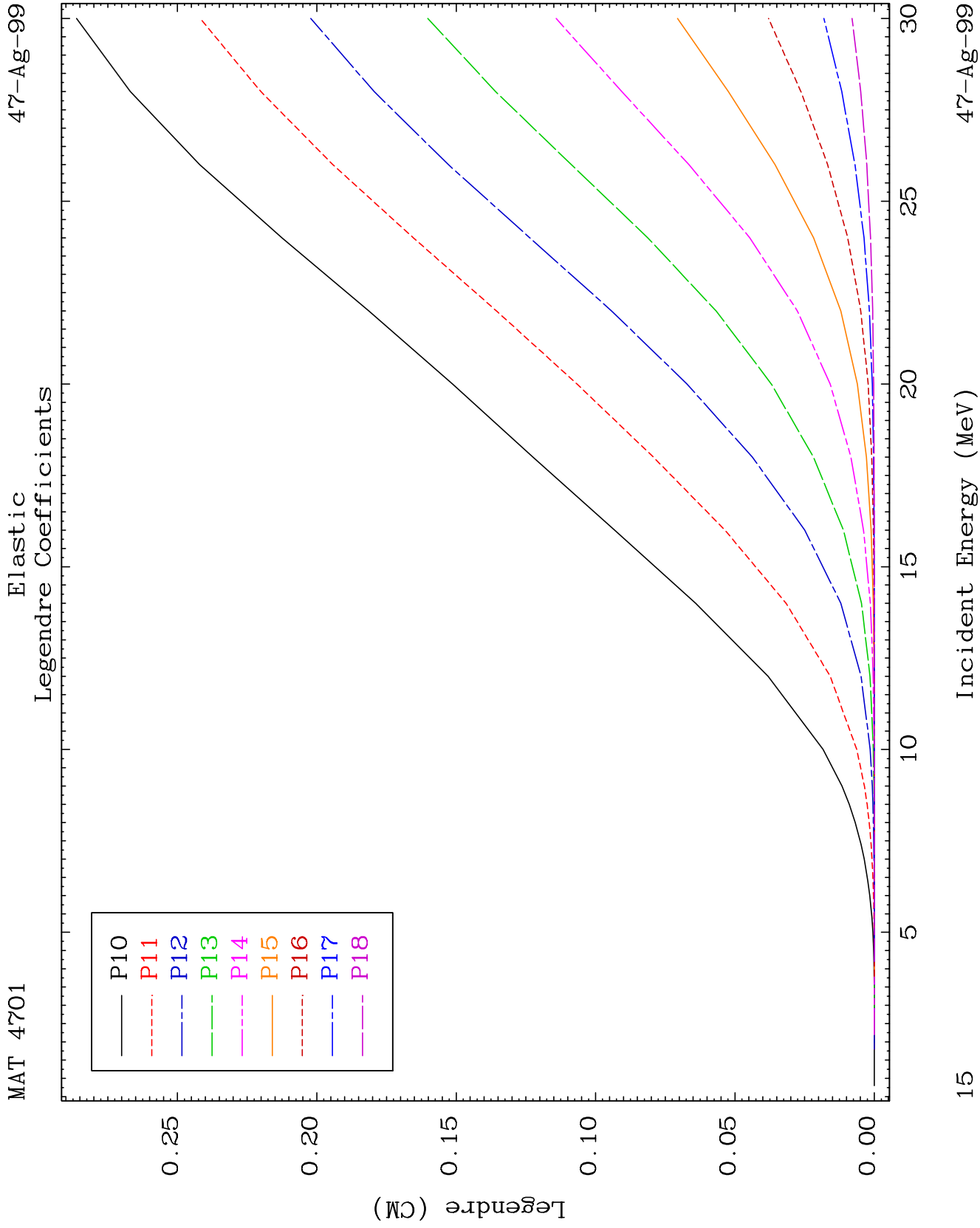


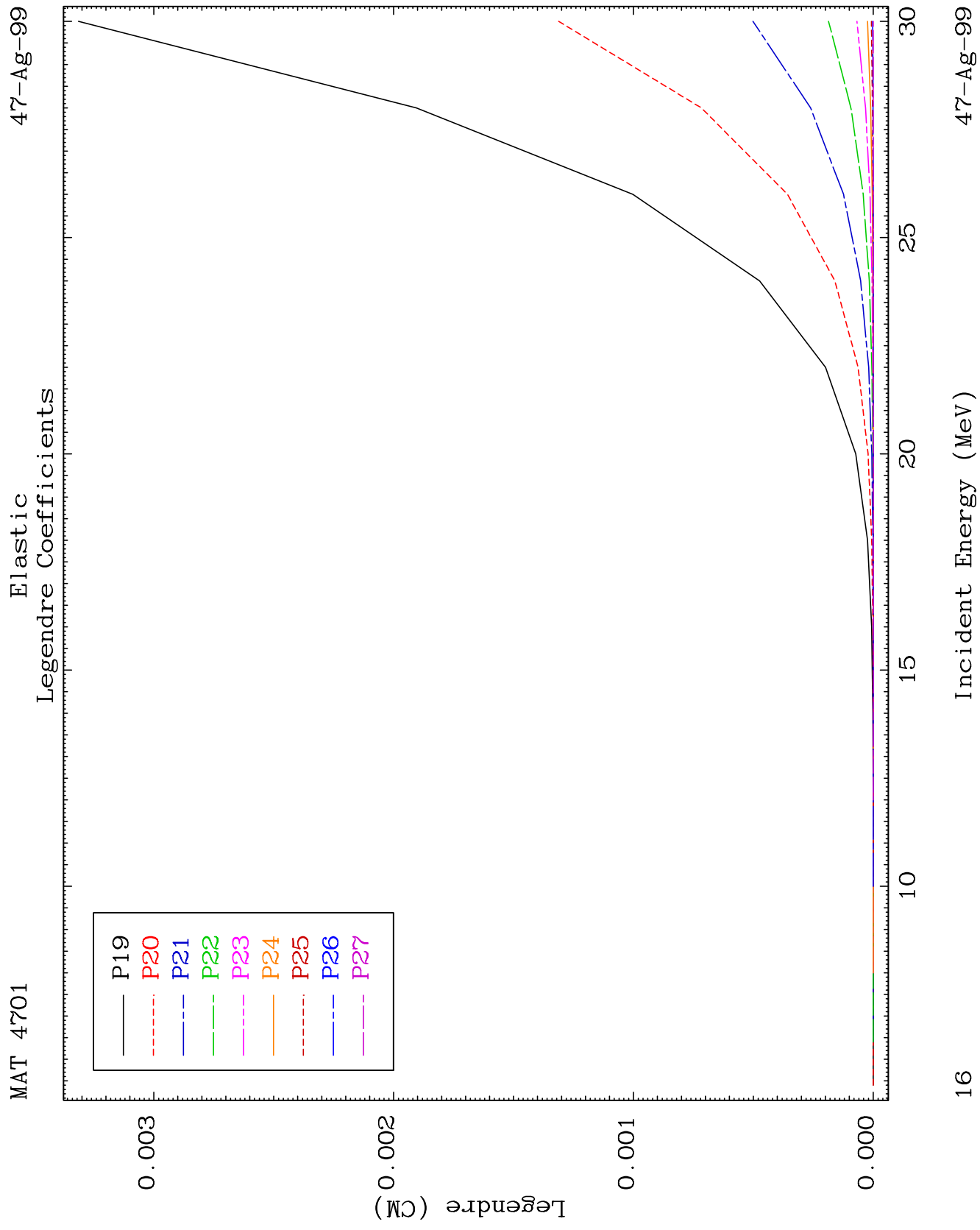
13

Incident Energy (MeV)

47-Ag-99



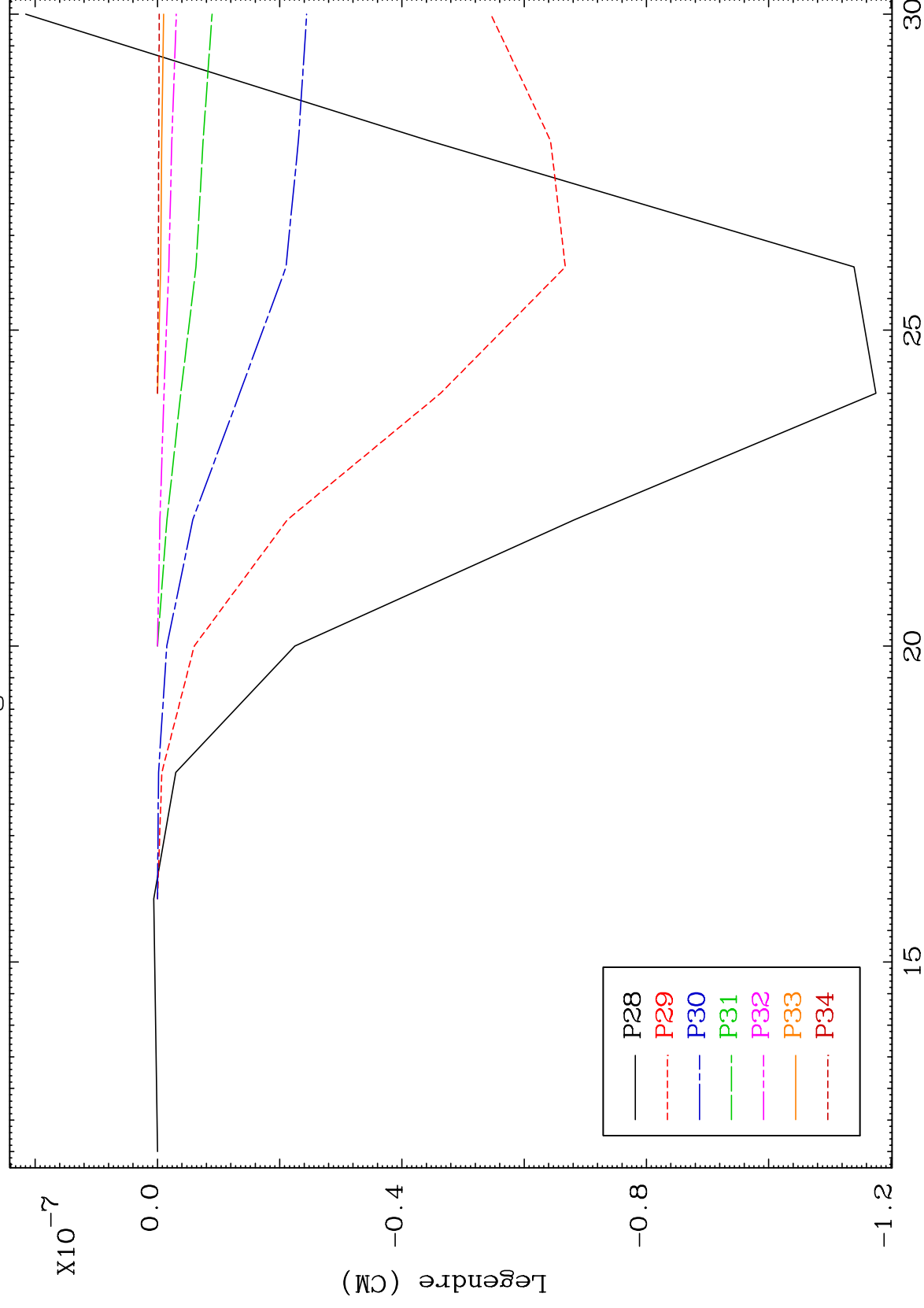




MAT 4701

47-Ag-99

Elastic Legendre Coefficients



21

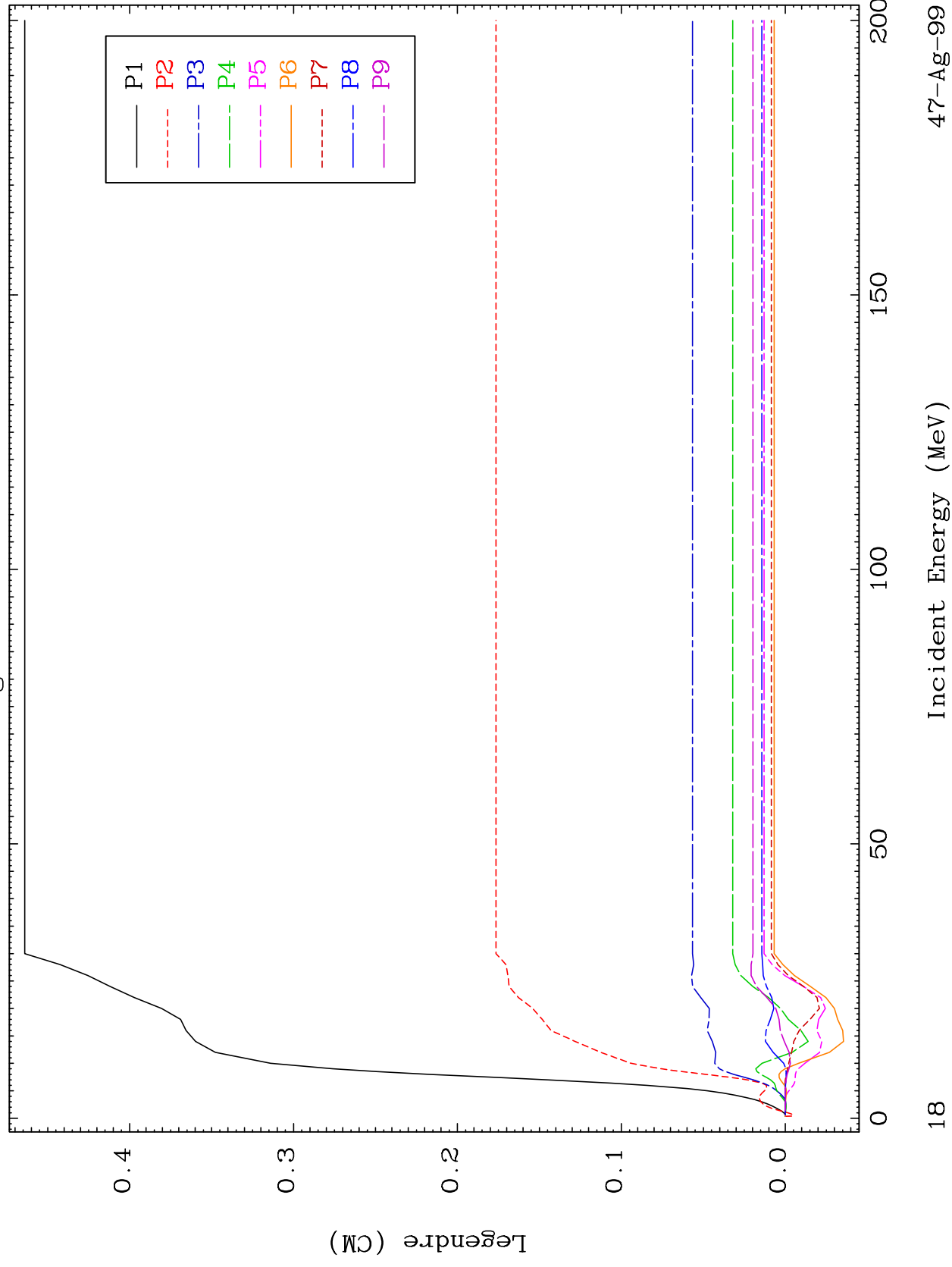
Incident Energy (MeV)

47-A8-99

MAT 4701

MT= 51 (n, n') Level
Legendre Coefficients

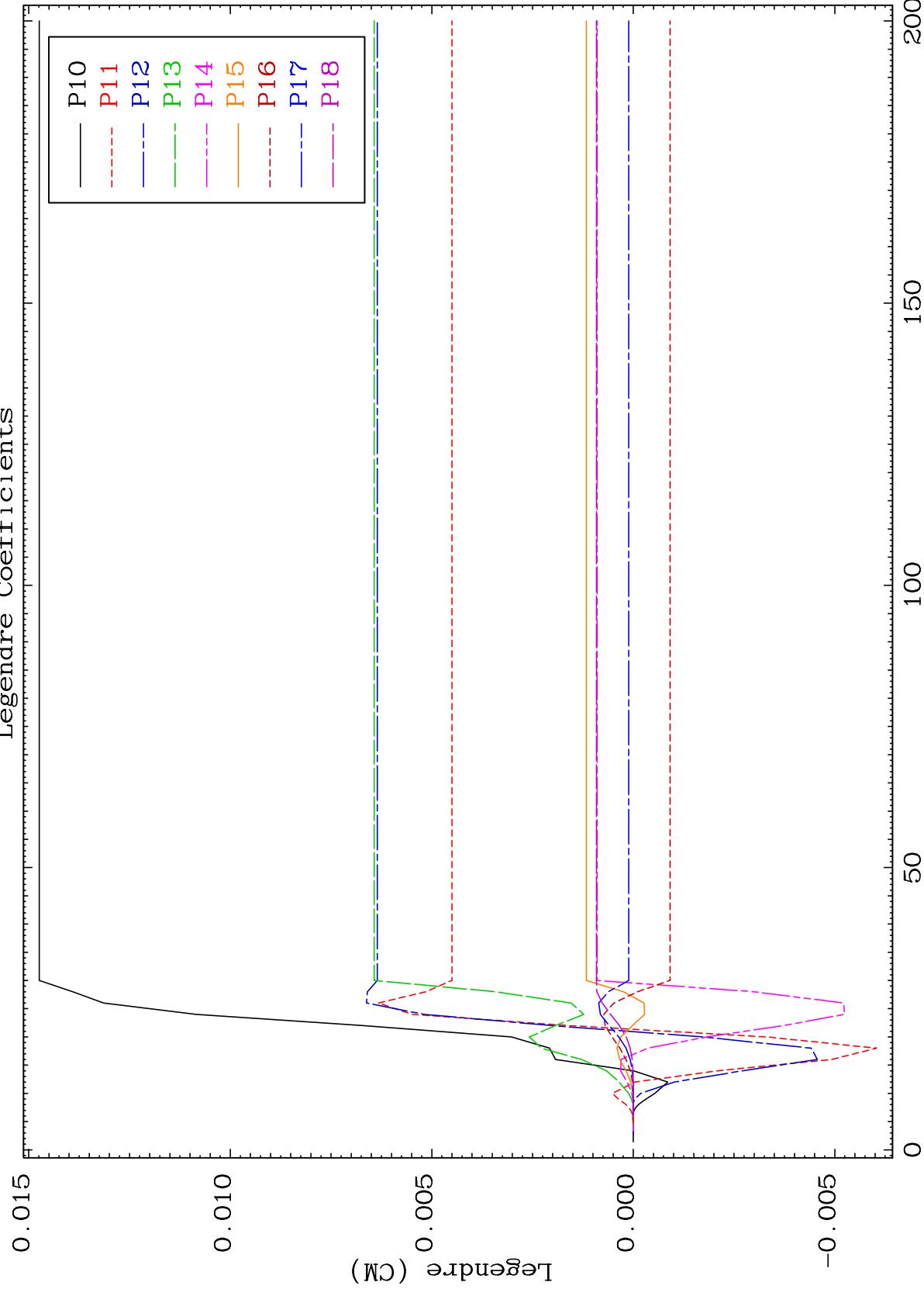
47-Ag-99



MAT 4701

MT= 51 (n, n') Level
Legendre Coefficients

47-Ag-99



19

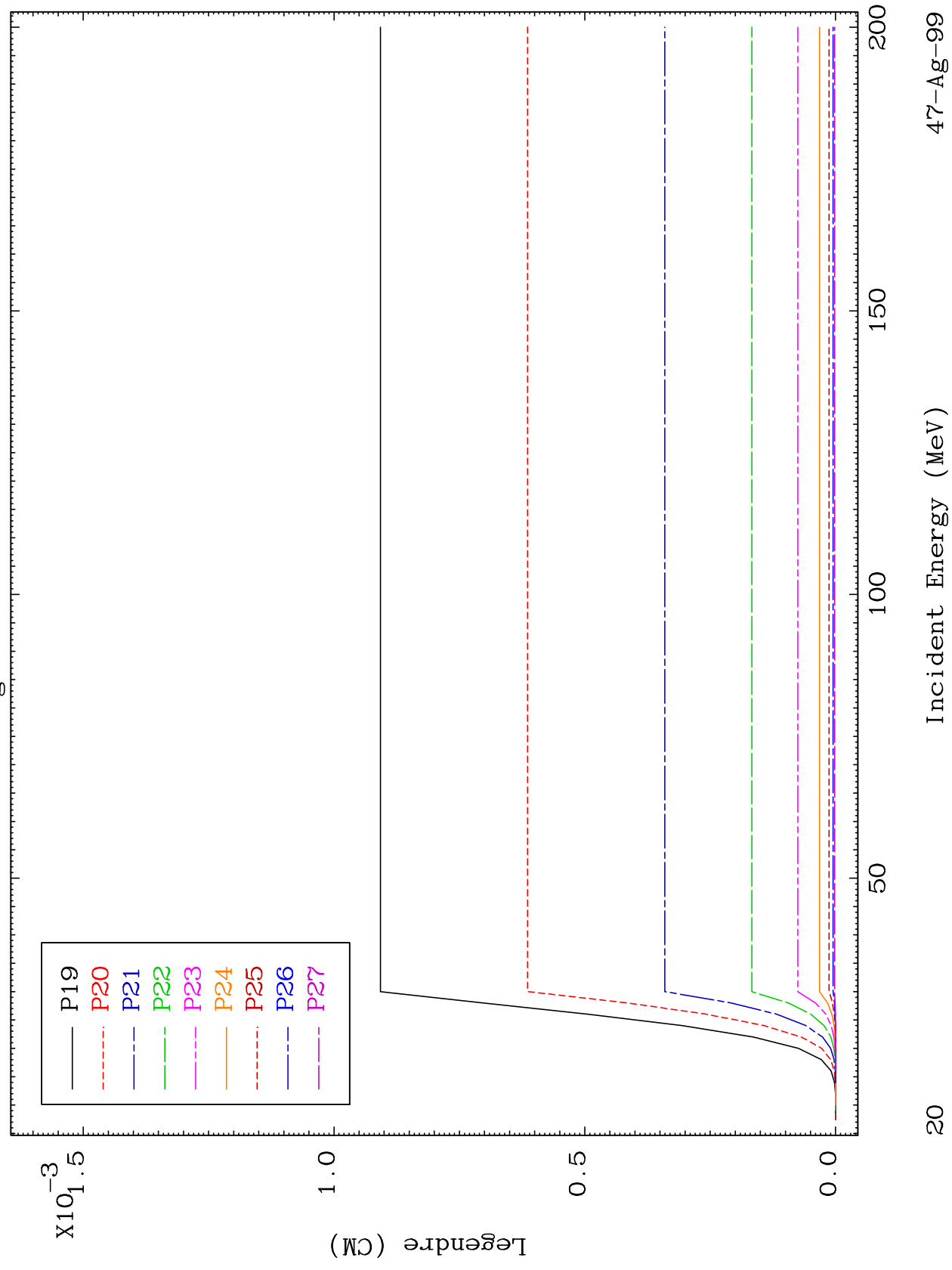
Incident Energy (MeV)

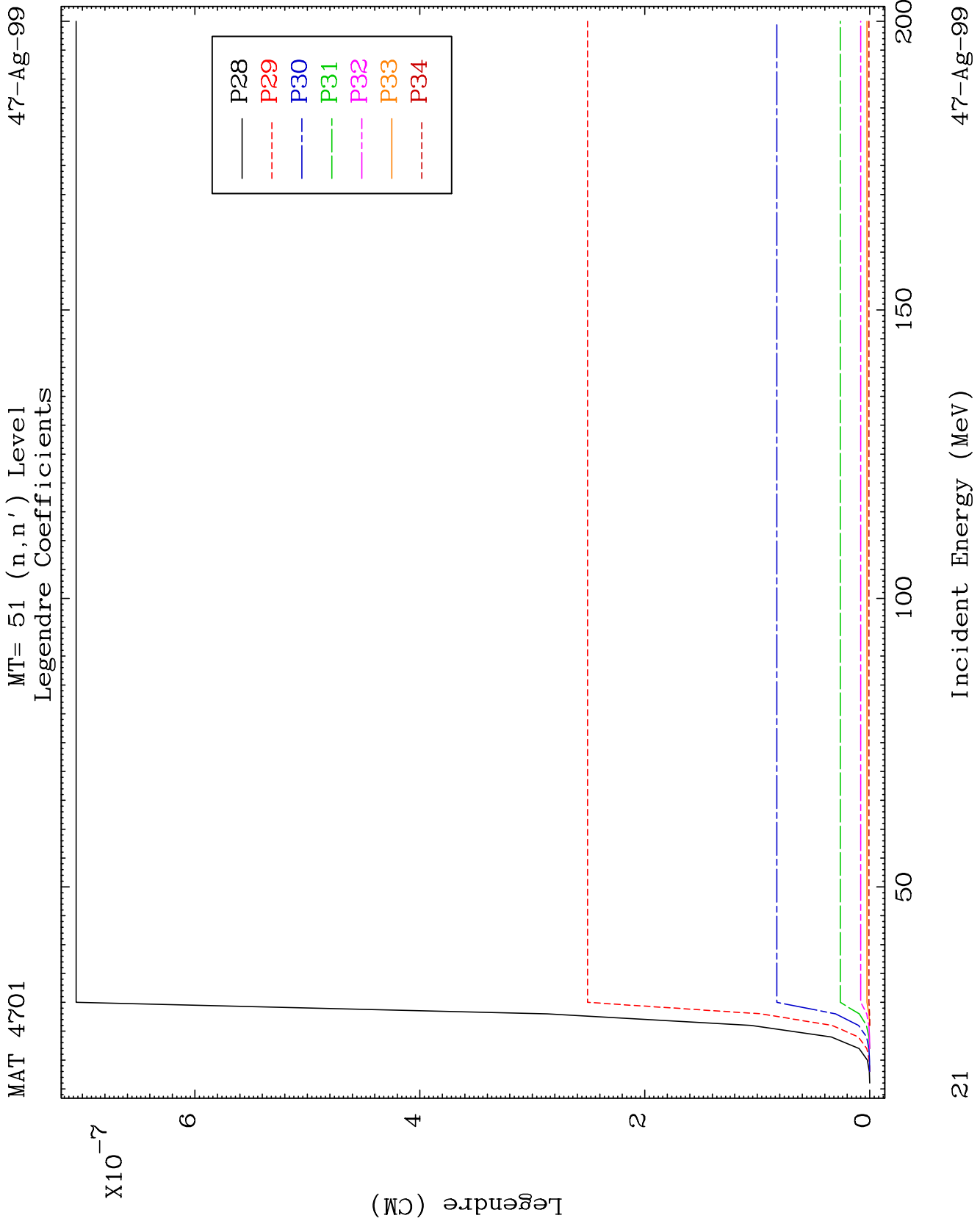
47-A8-99

MAT 4701

MT= 51 (n, n') Level
Legendre Coefficients

47-Ag-99

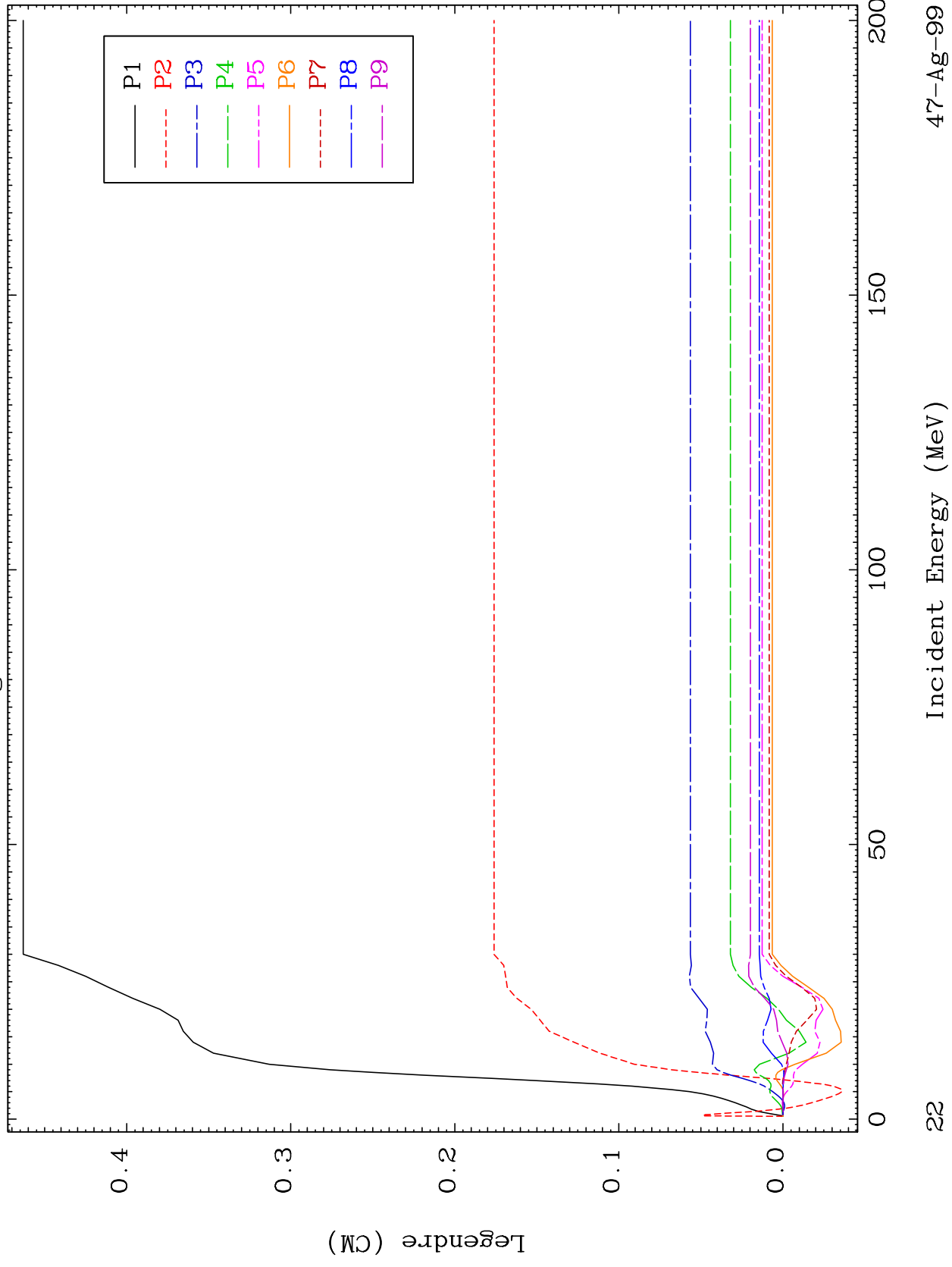




MAT 4701

MT= 52 (n, n') Level
Legendre Coefficients

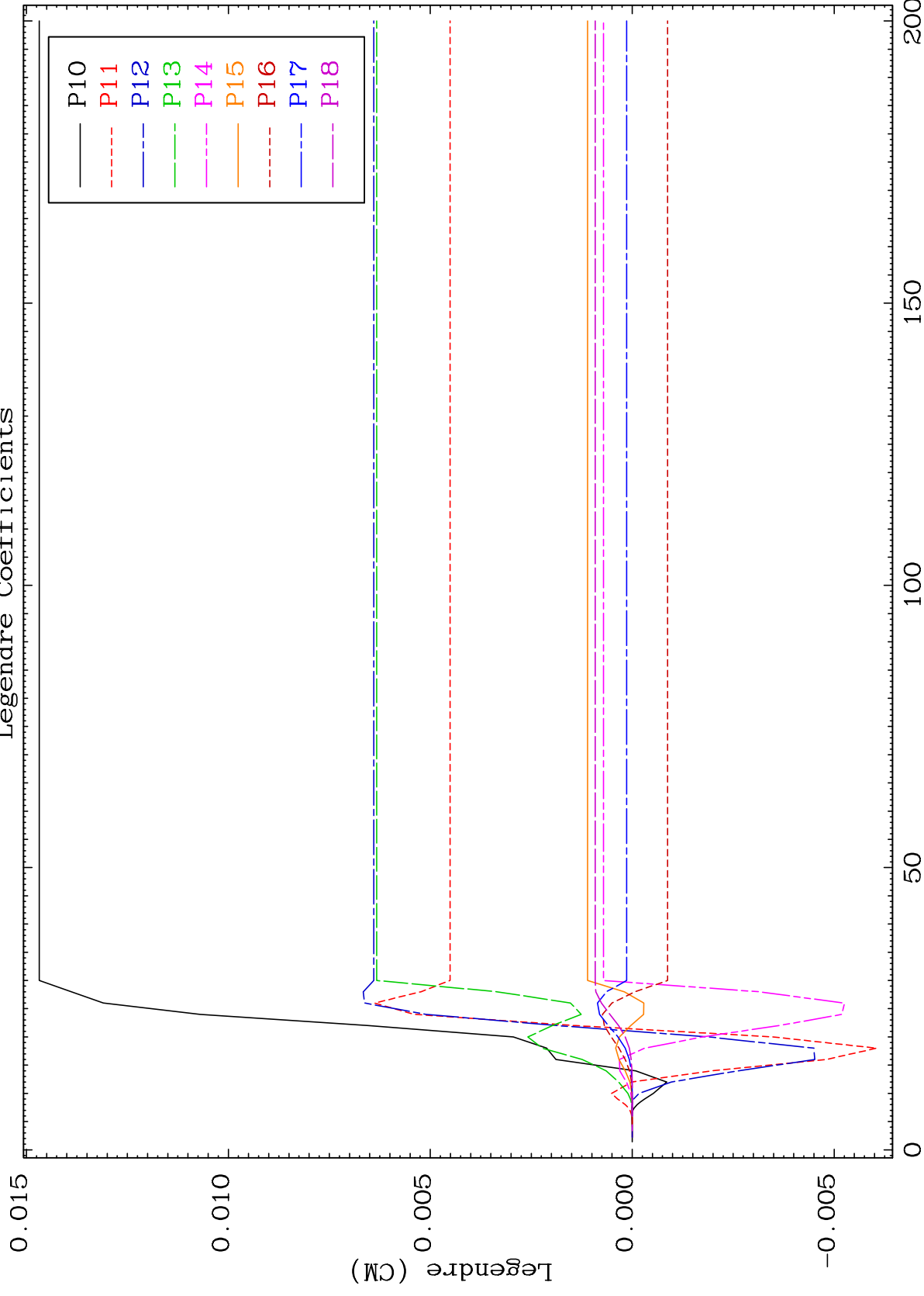
47-Ag-99



MAT 4701

MT= 52 (n, n') Level
Legendre Coefficients

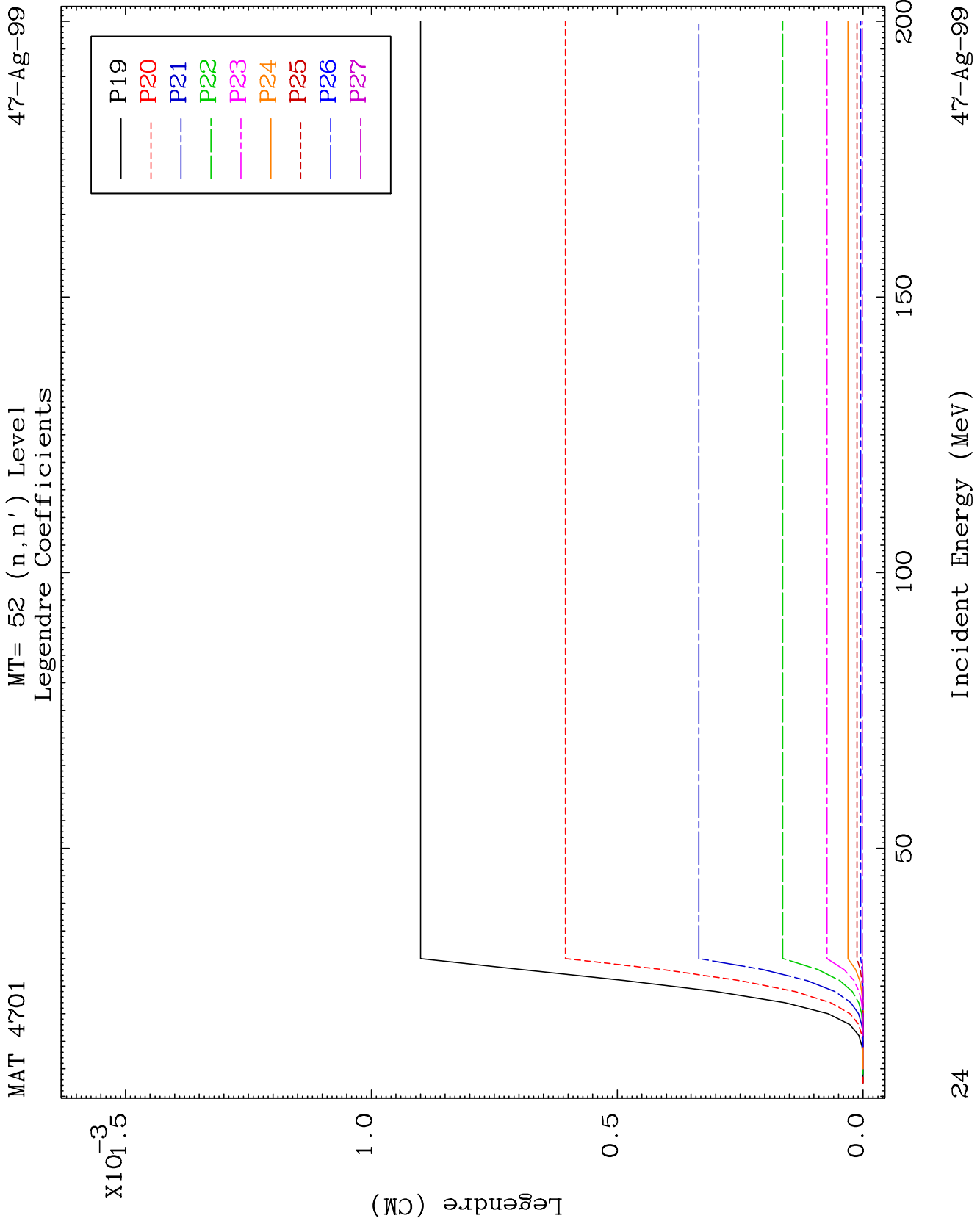
47-A8-99

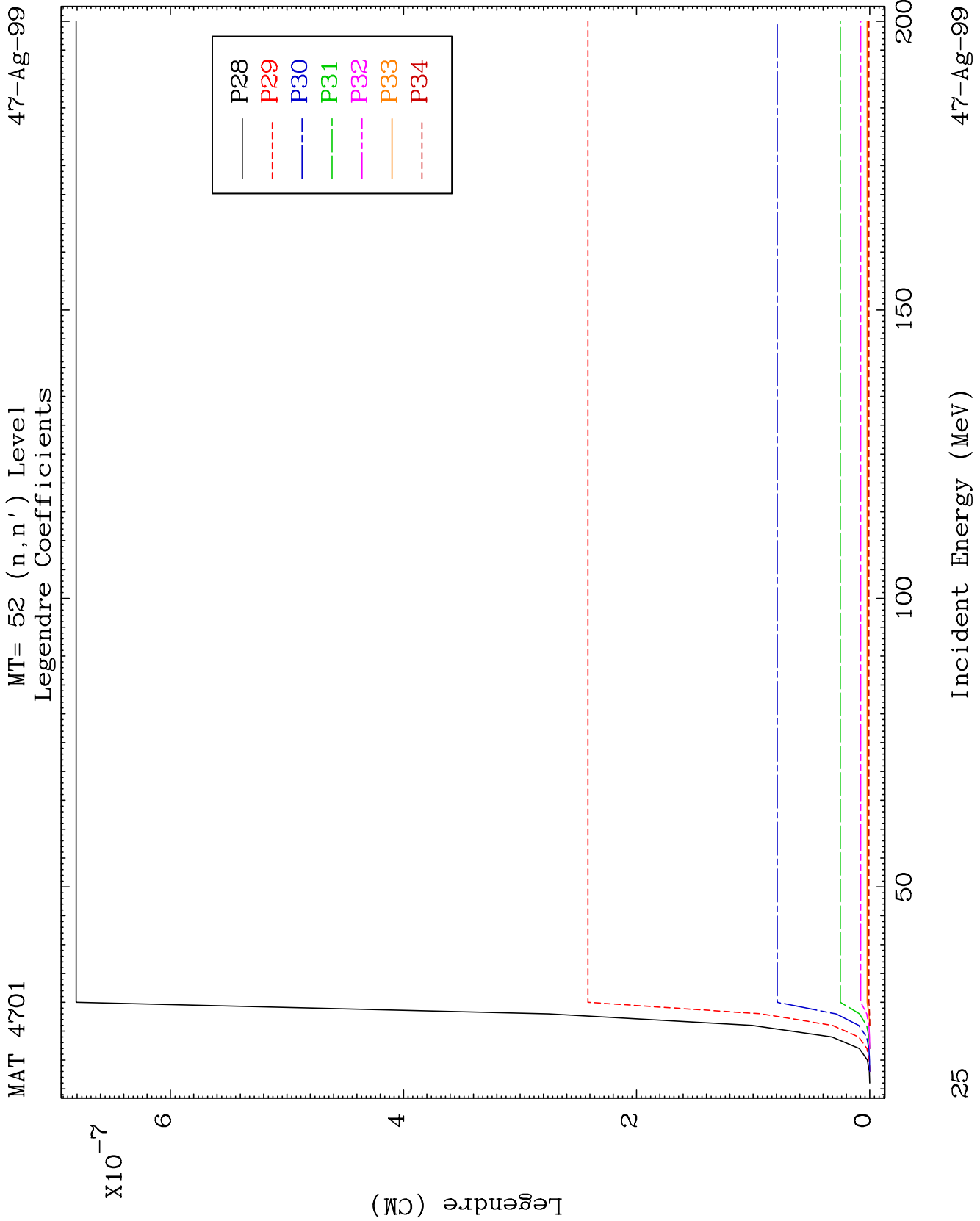


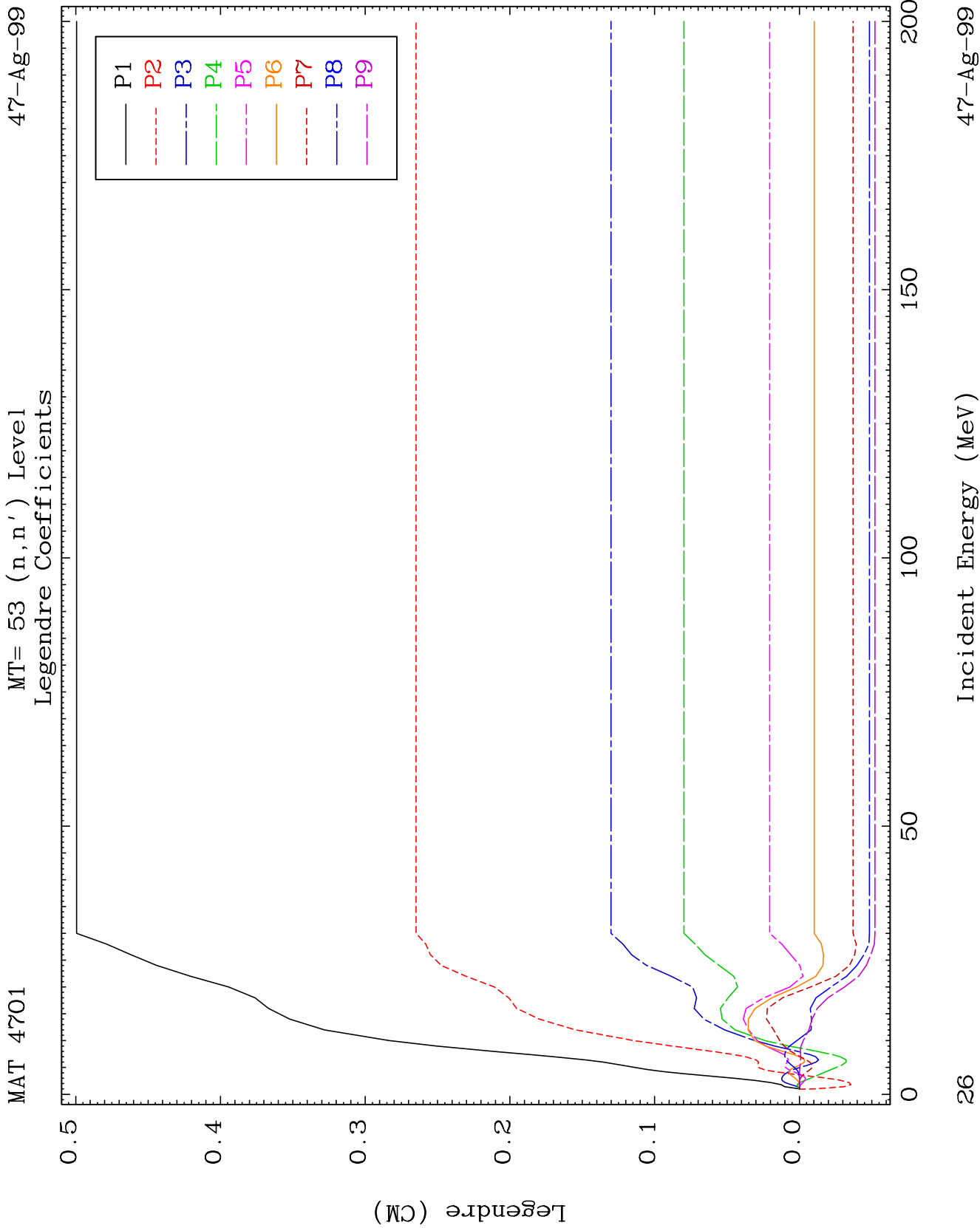
32

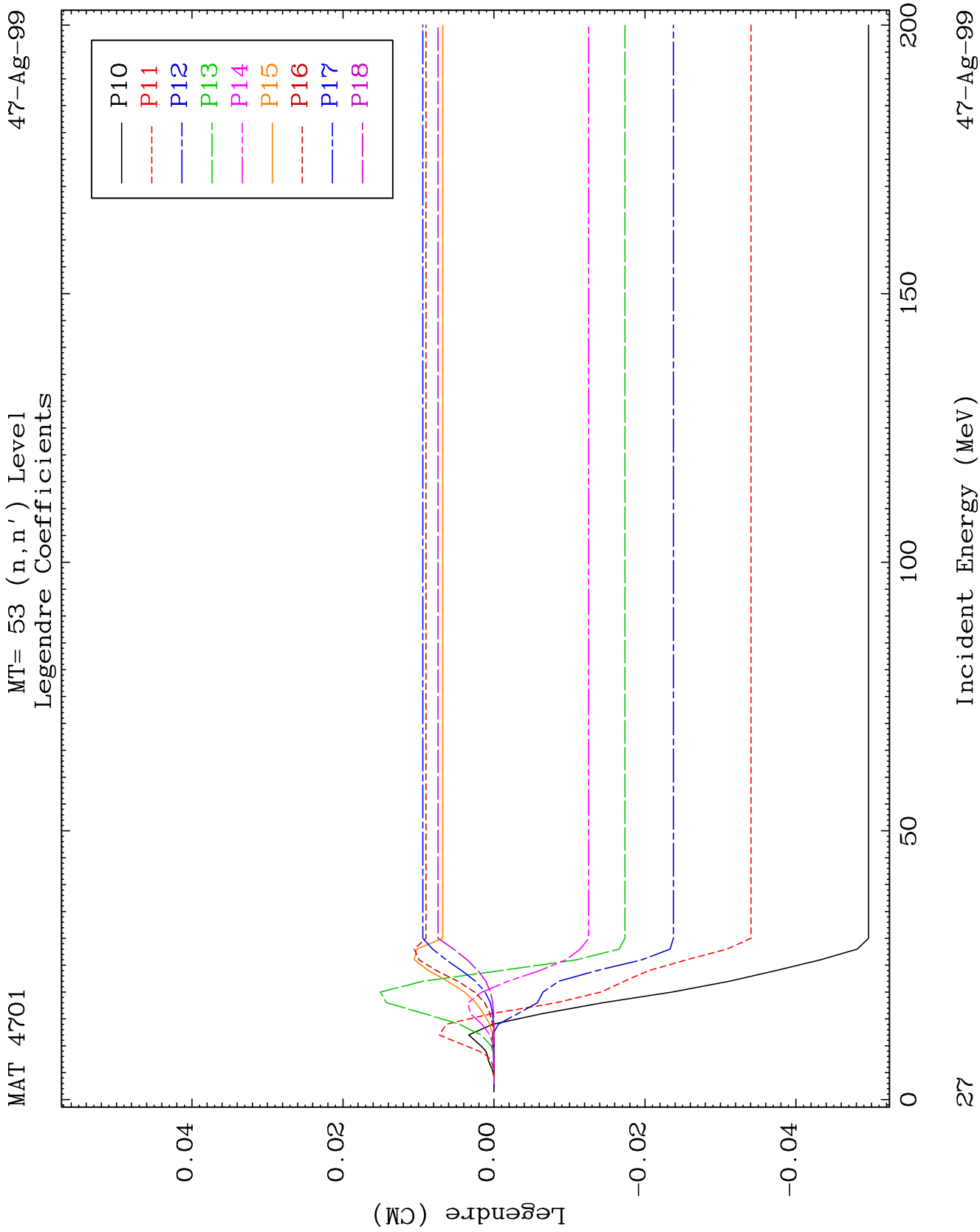
Incident Energy (MeV)

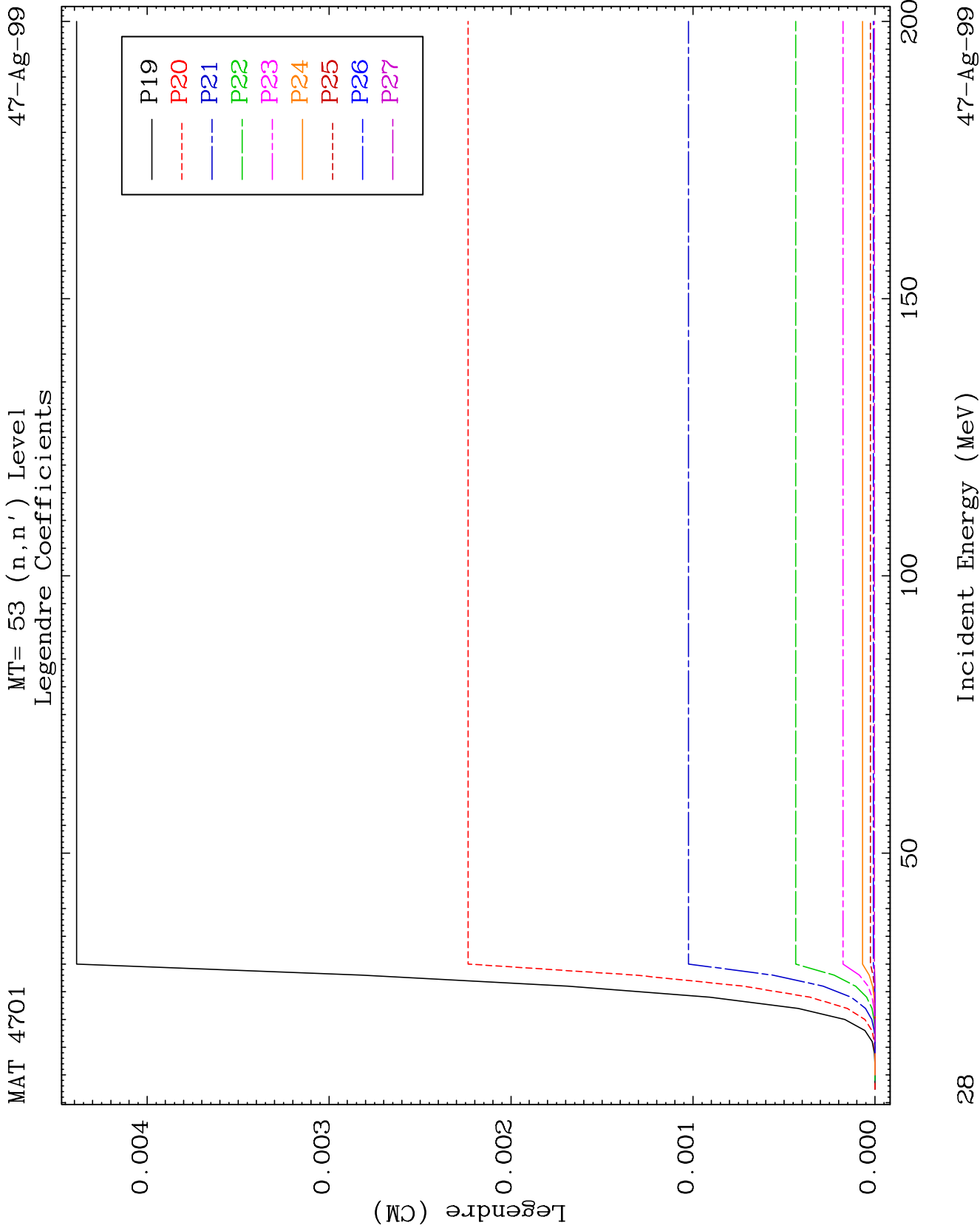
47-A8-99

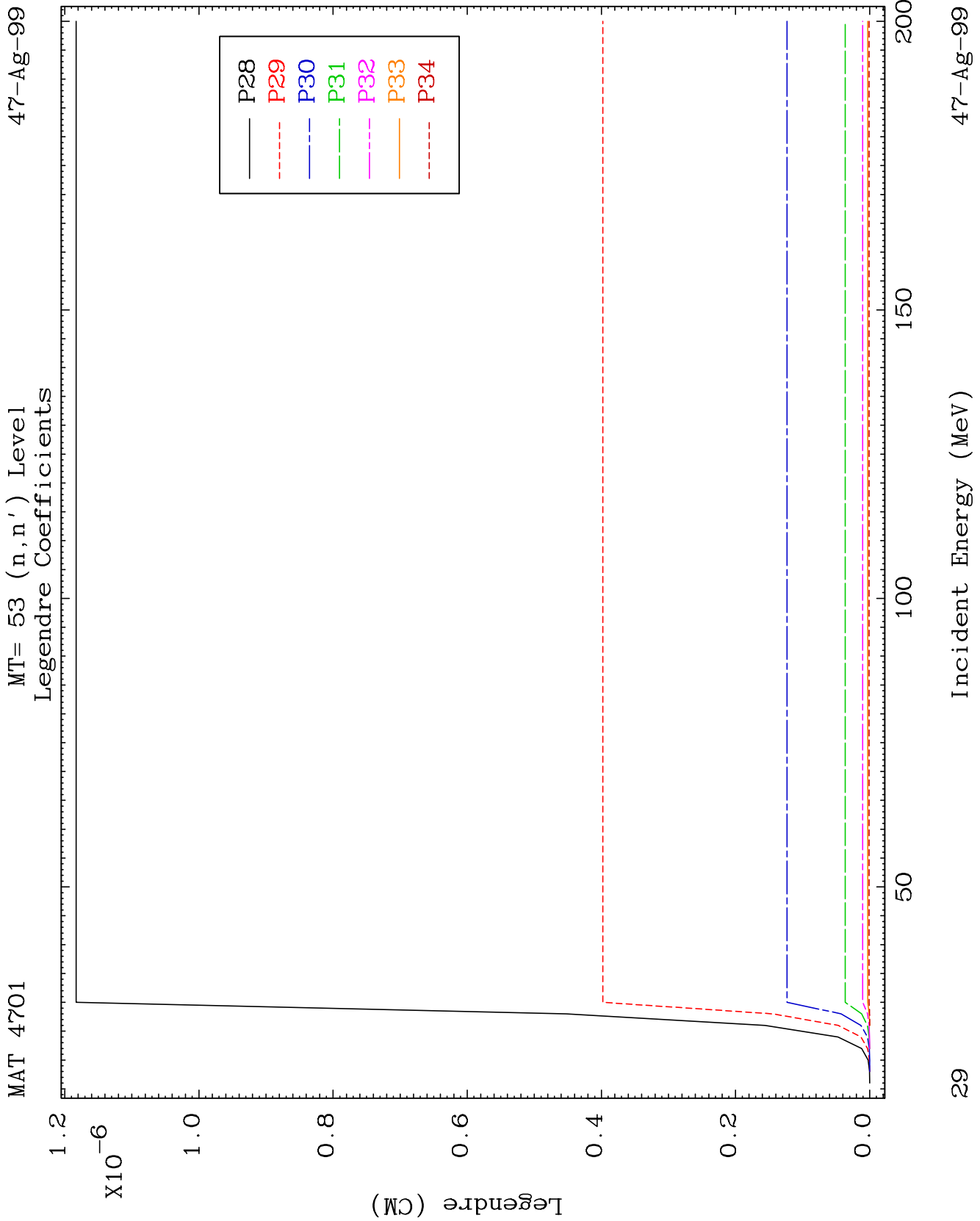


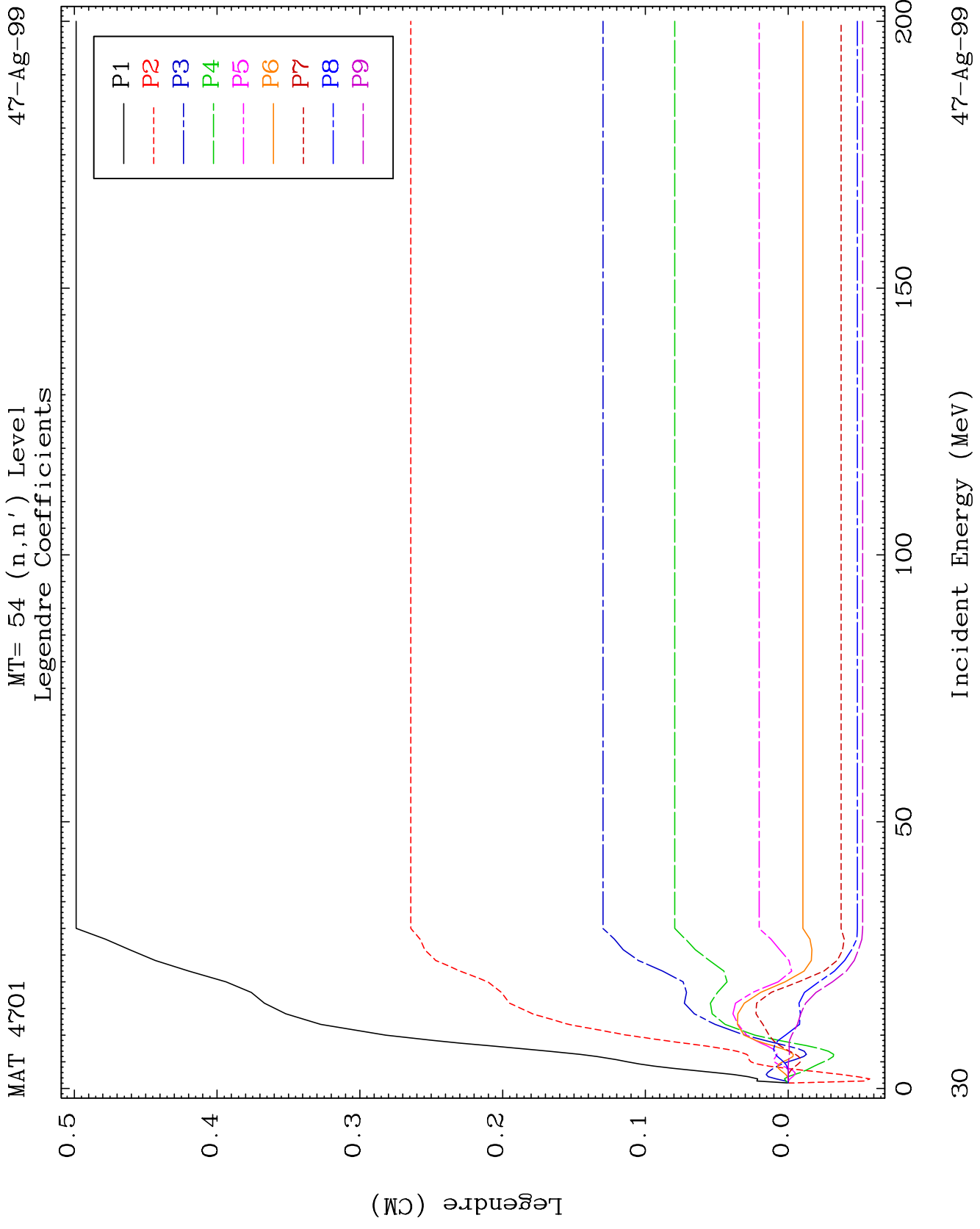


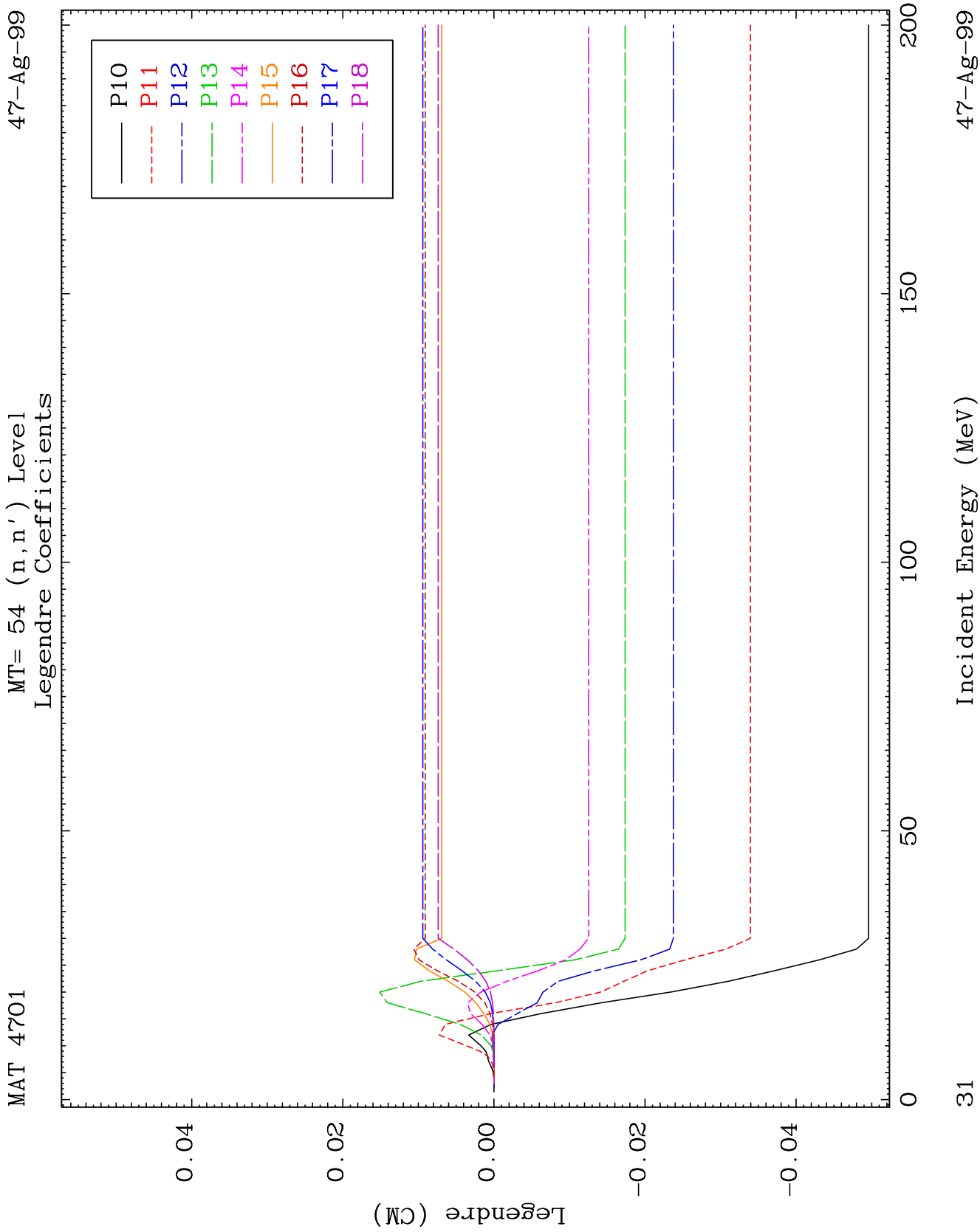


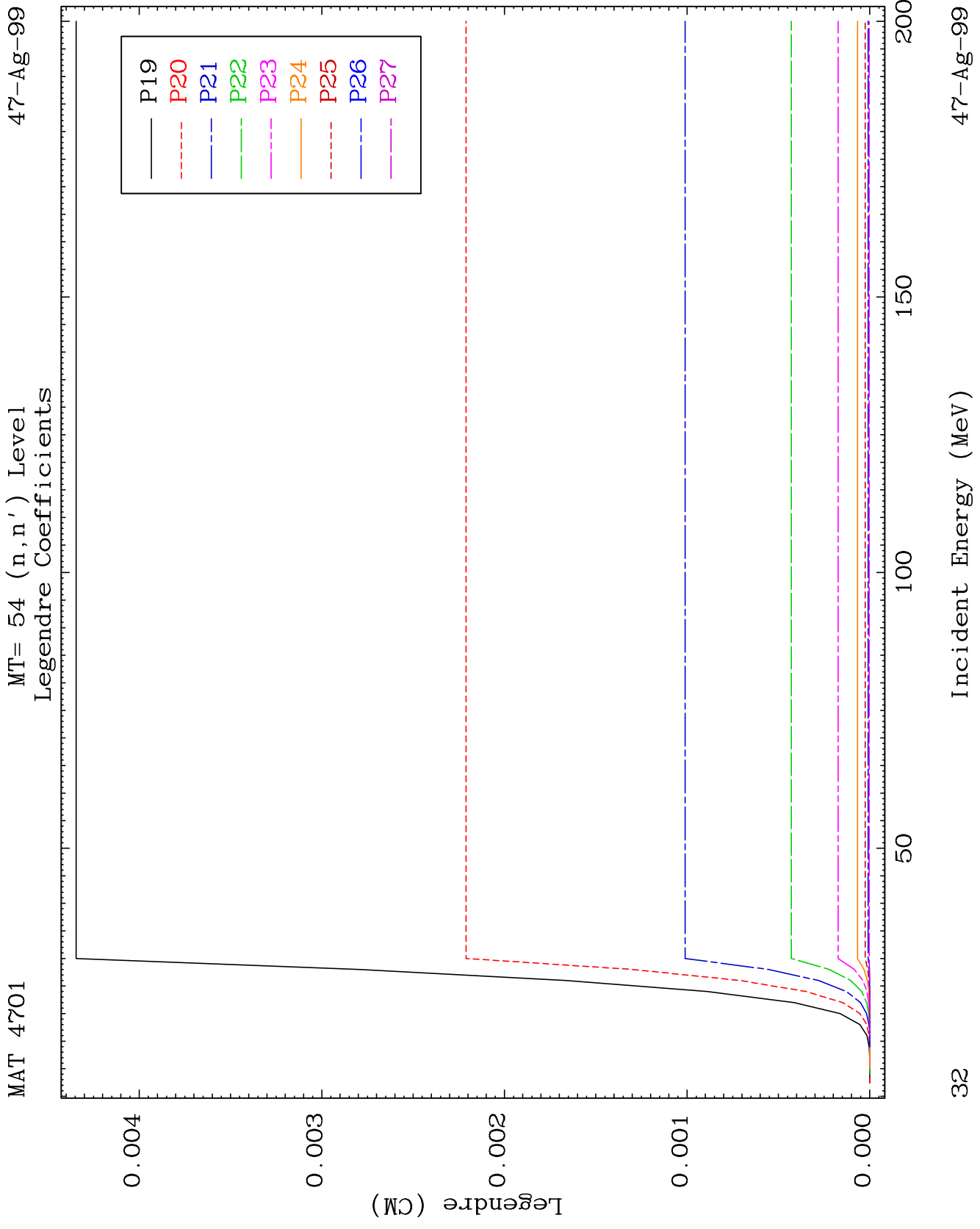


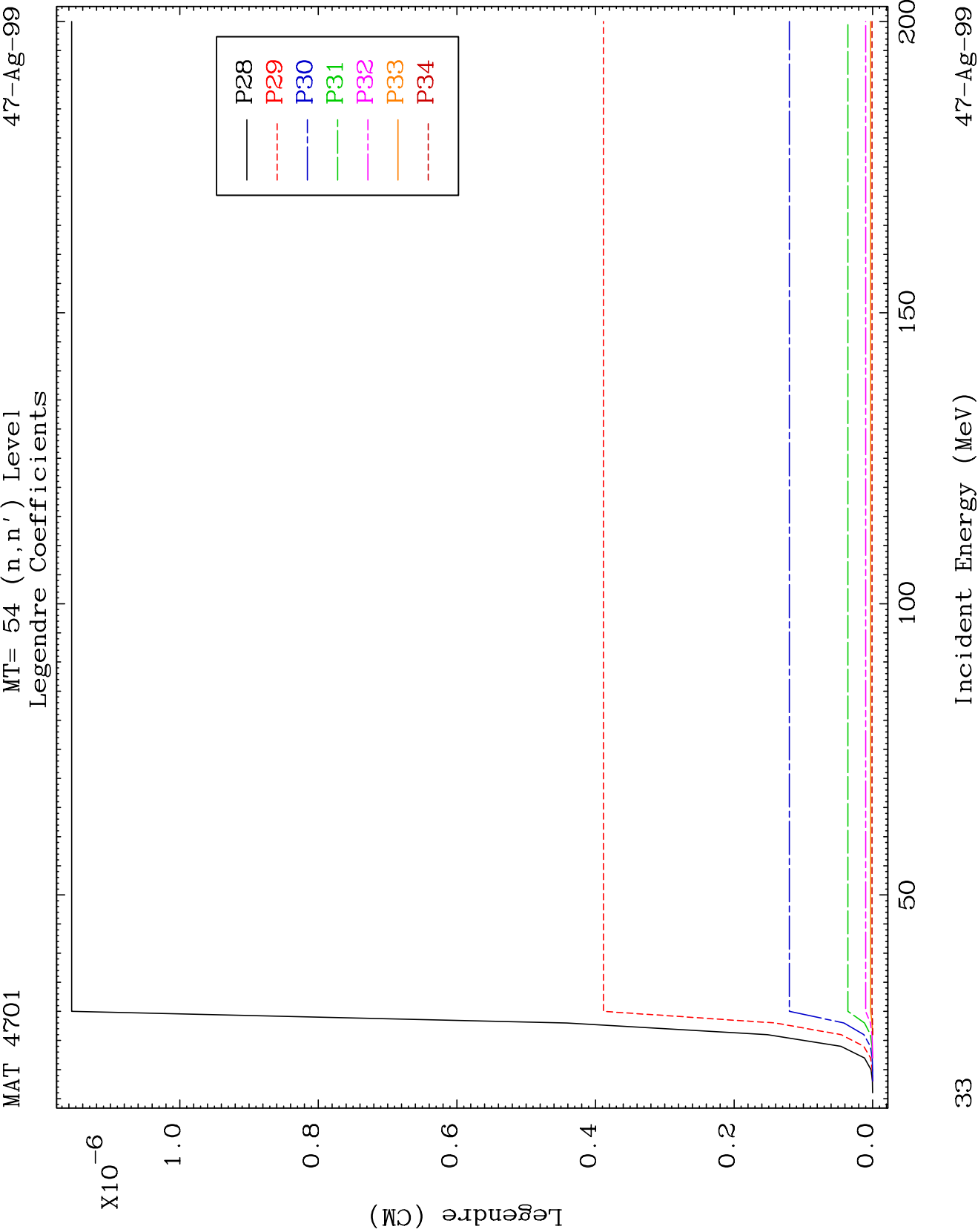








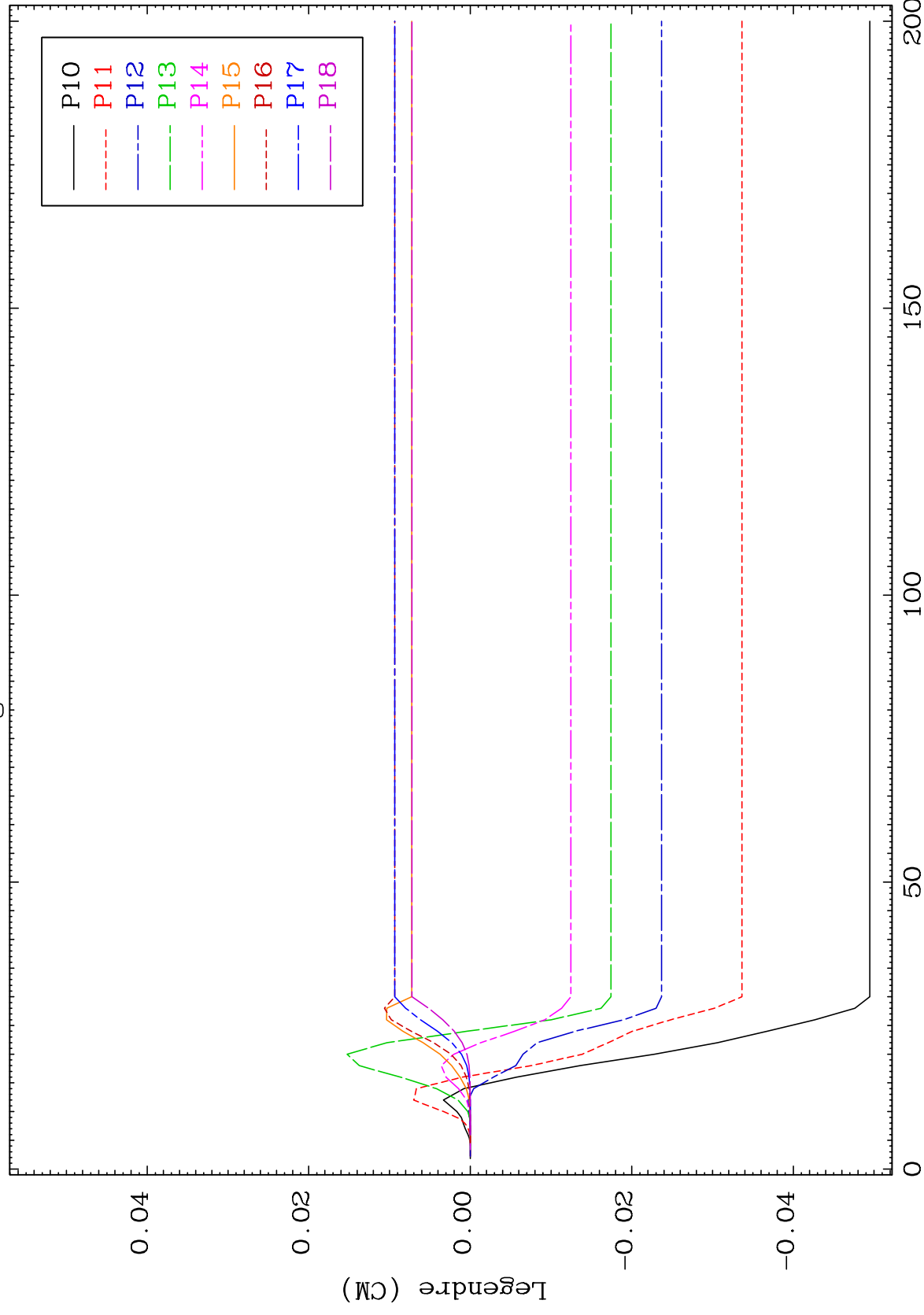




MAT 4701

MT= 55 (n, n') Level
Legendre Coefficients

47-Ag-99



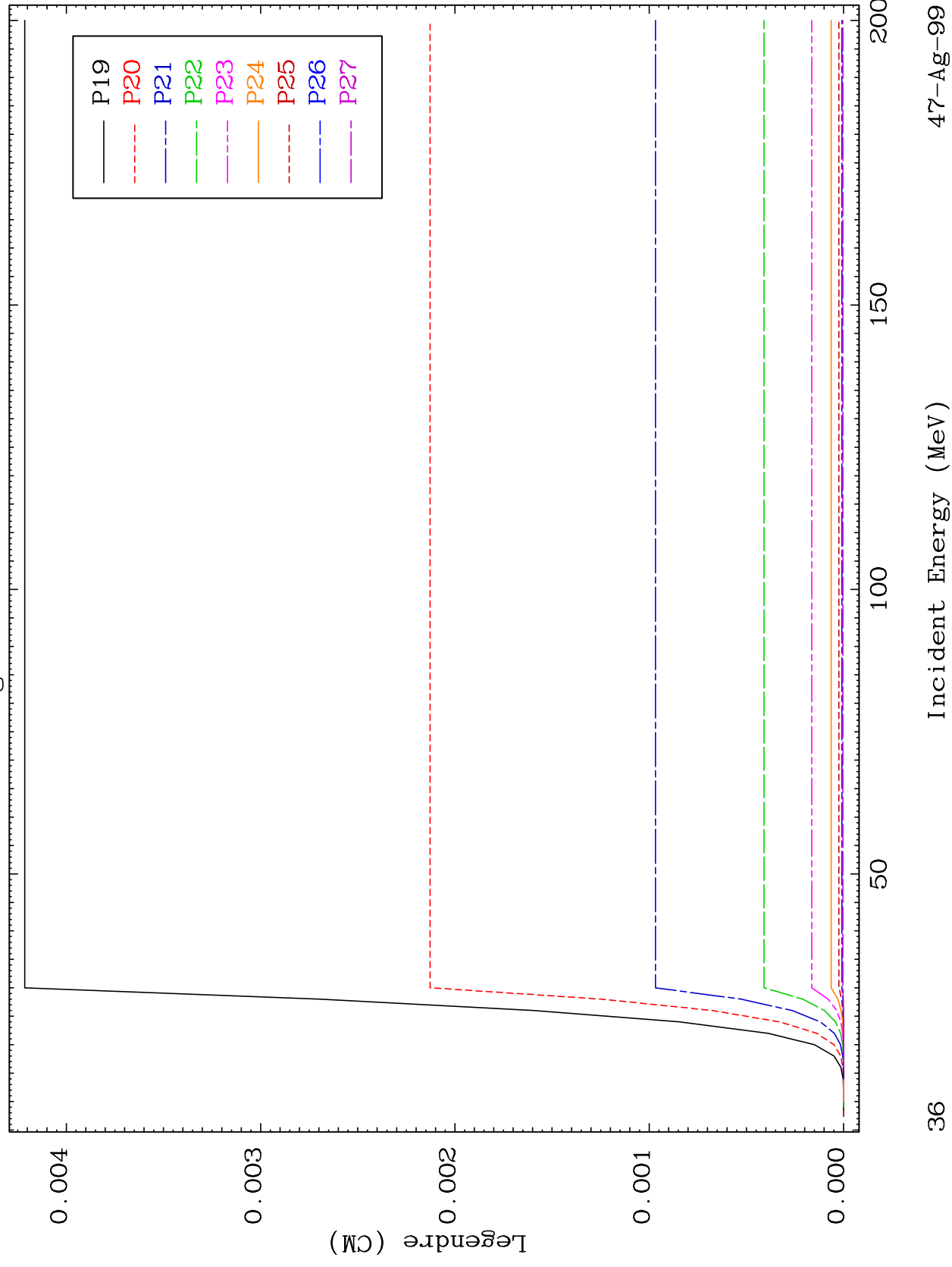
47-A8-99

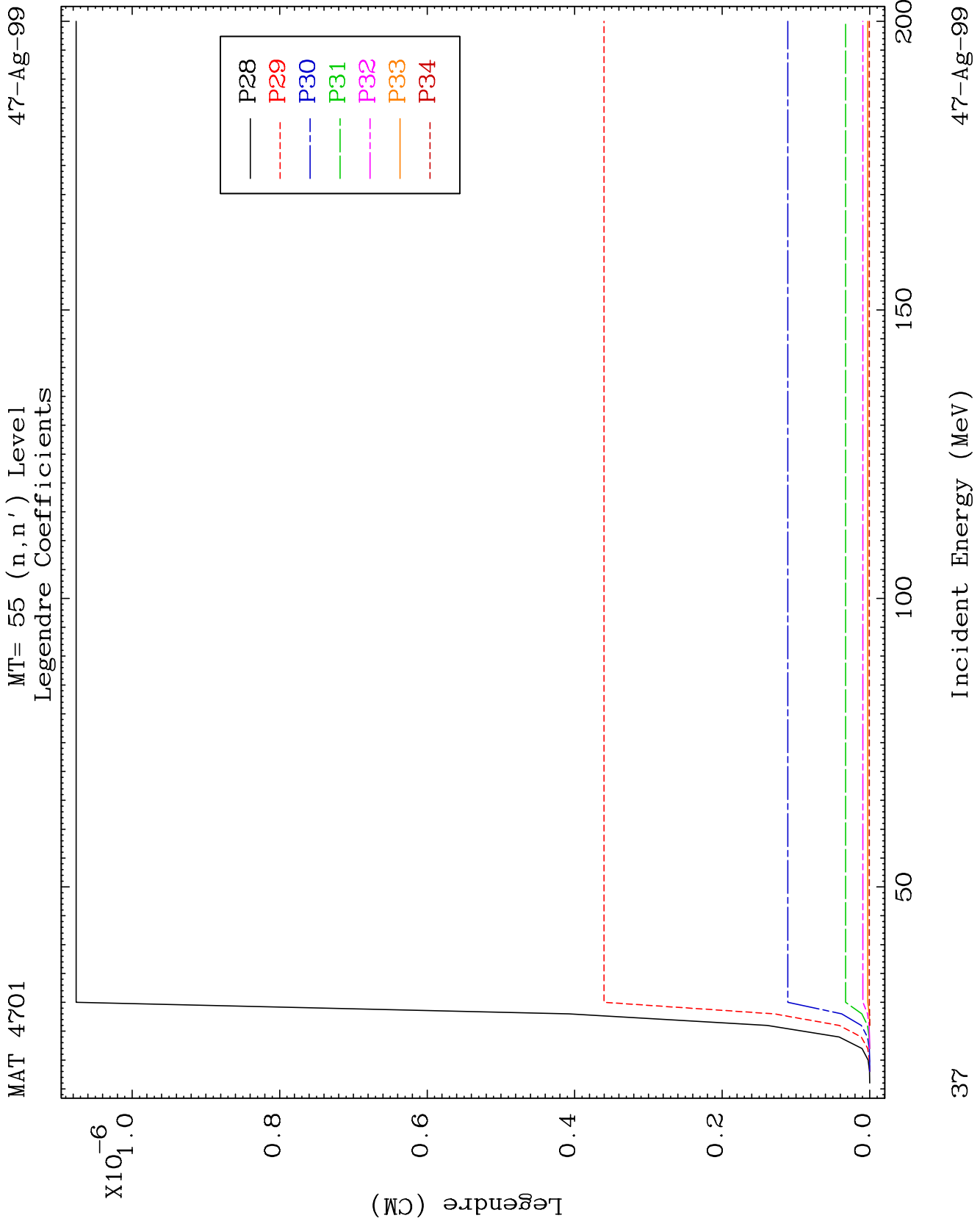
Incident Energy (MeV)

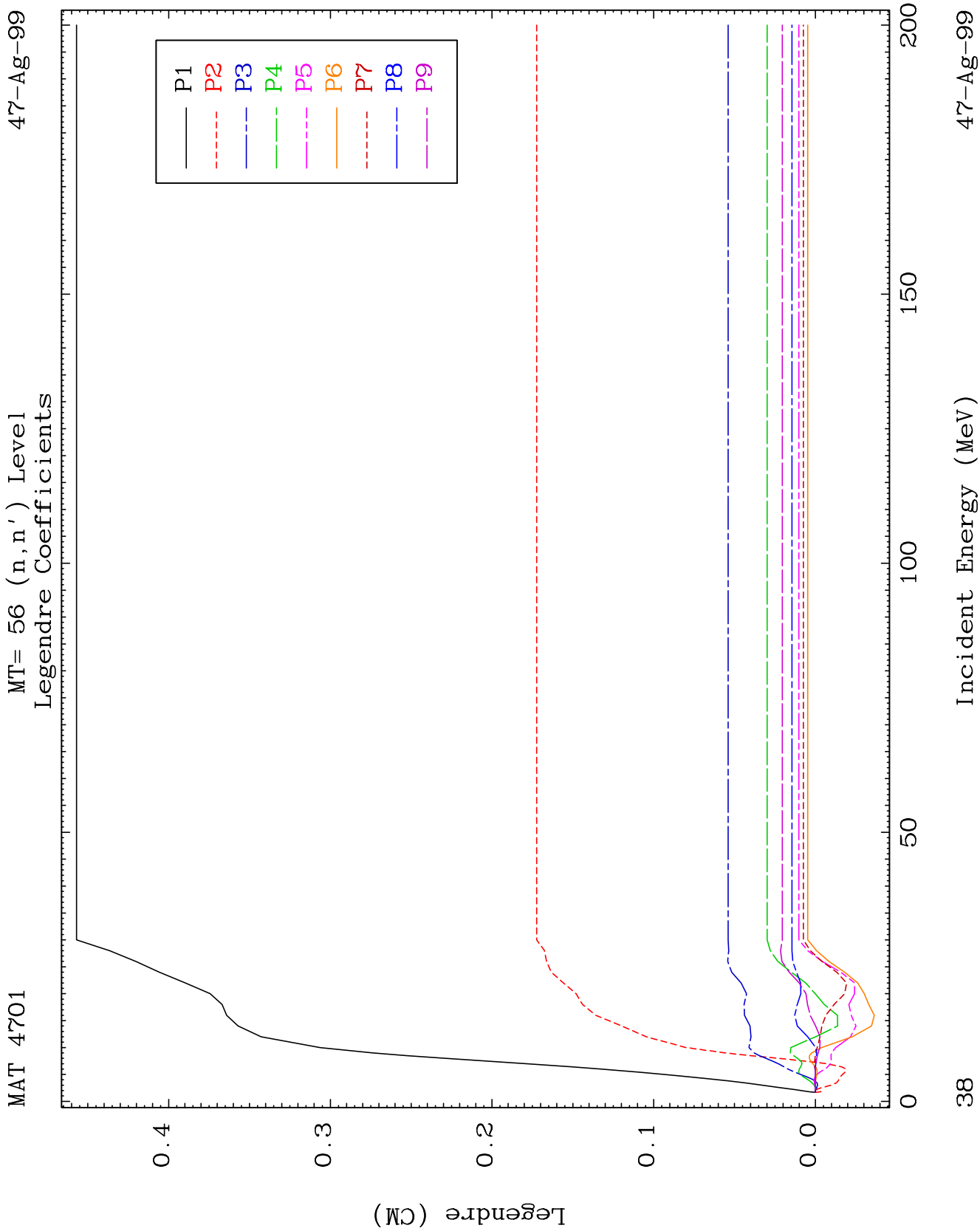
MAT 4701

MT= 55 (n, n') Level
Legendre Coefficients

47-Ag-99



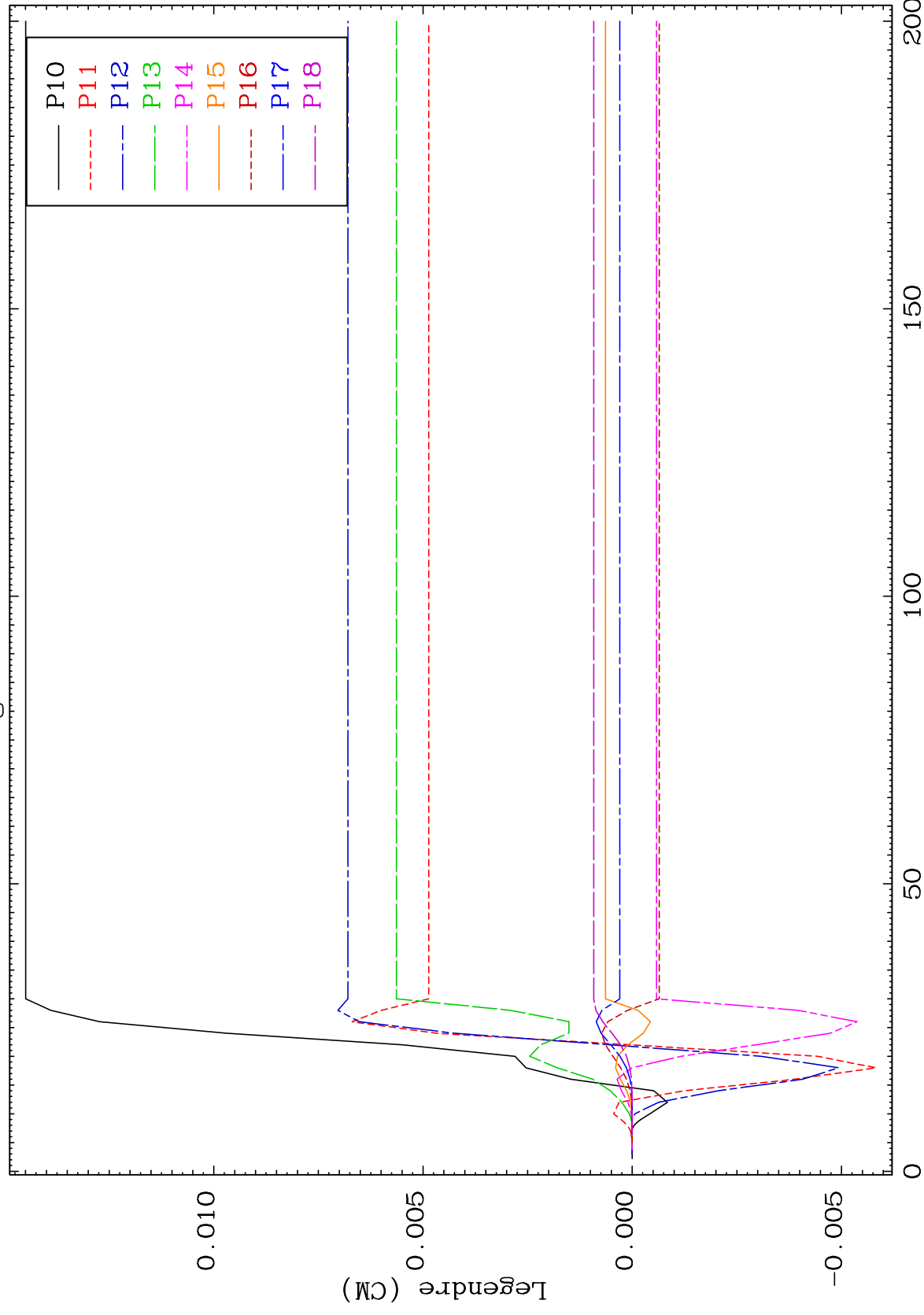




MAT 4701

MT= 56 (n, n') Level
Legendre Coefficients

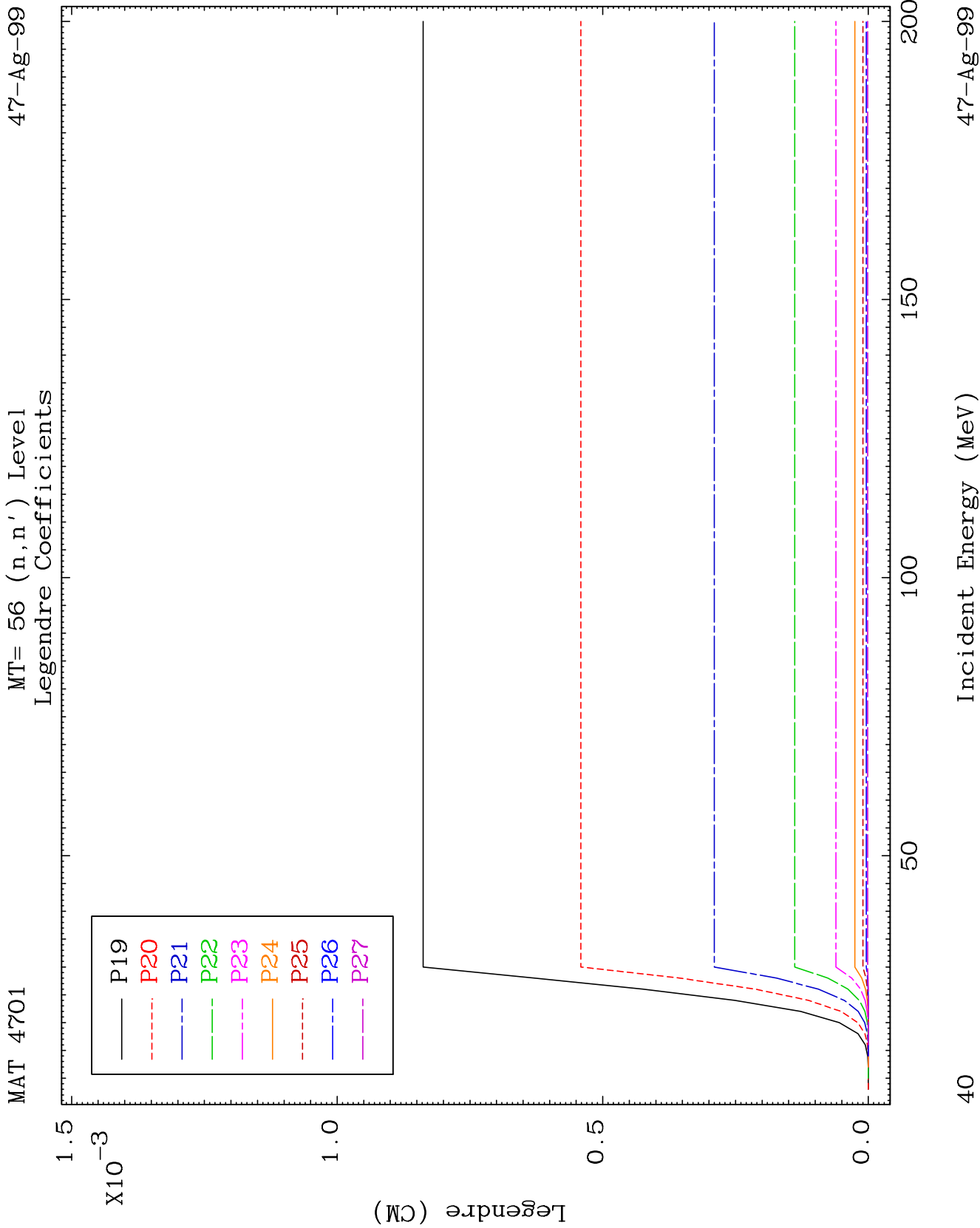
47-Ag-99

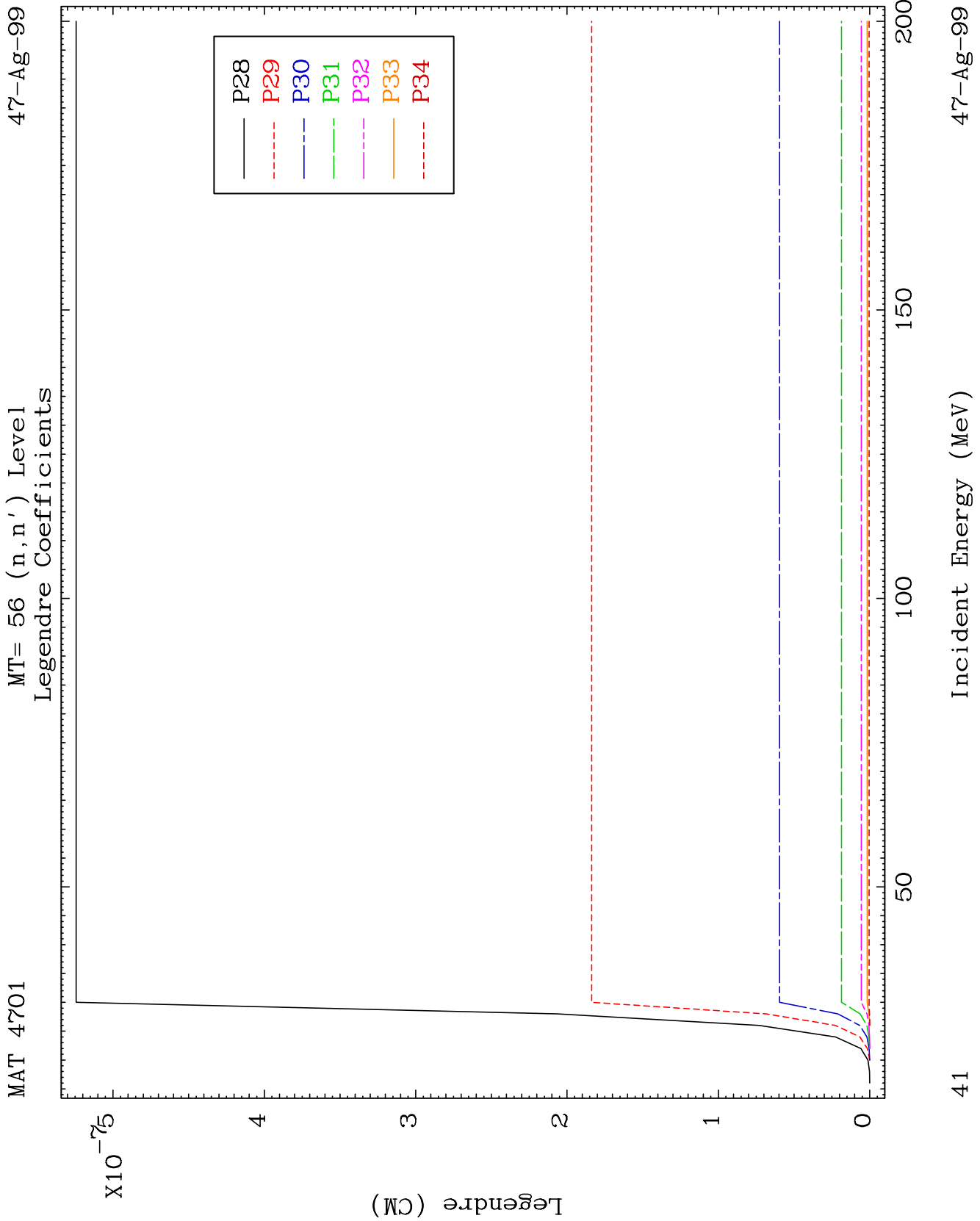


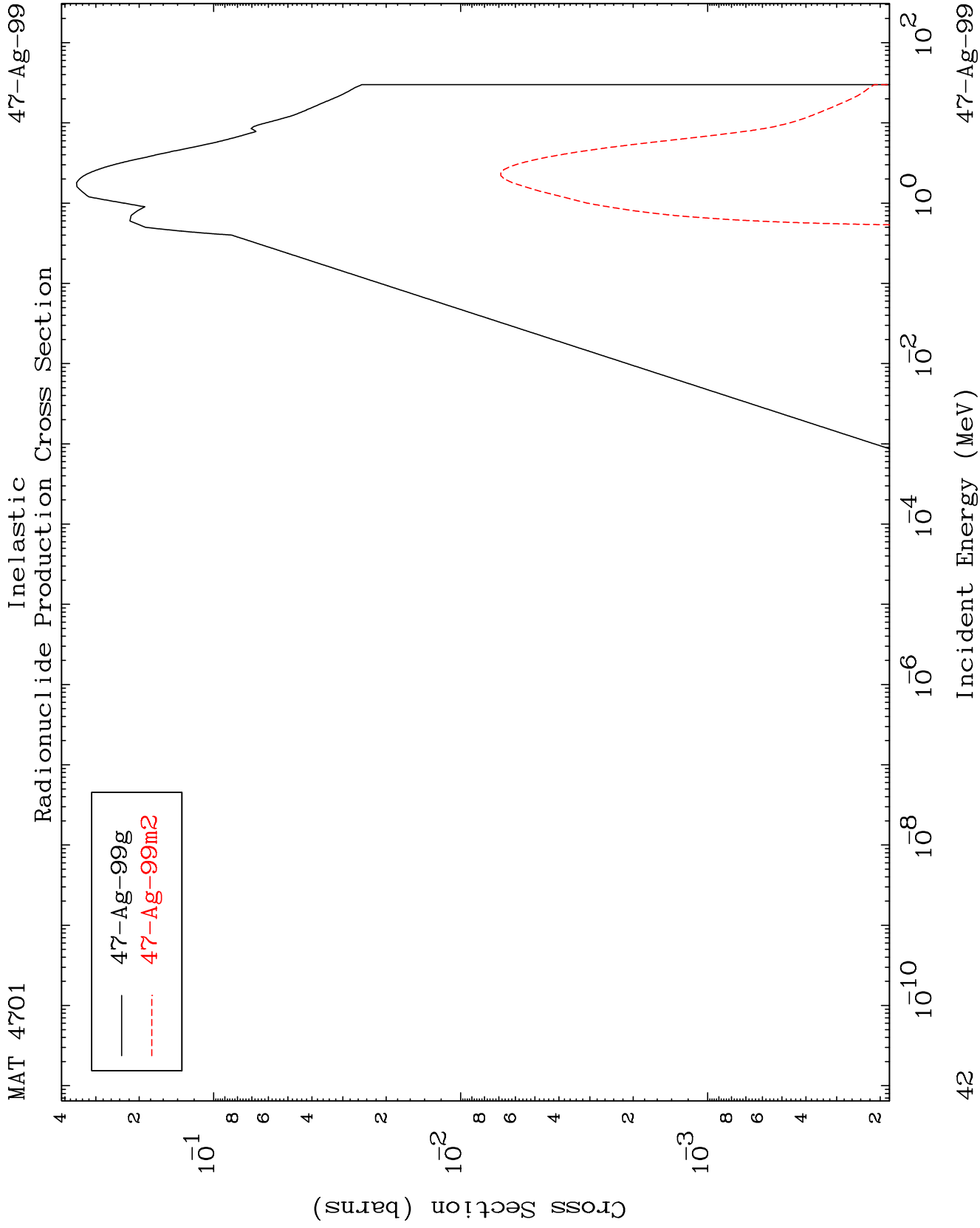
39

Incident Energy (MeV)

47-A8-99



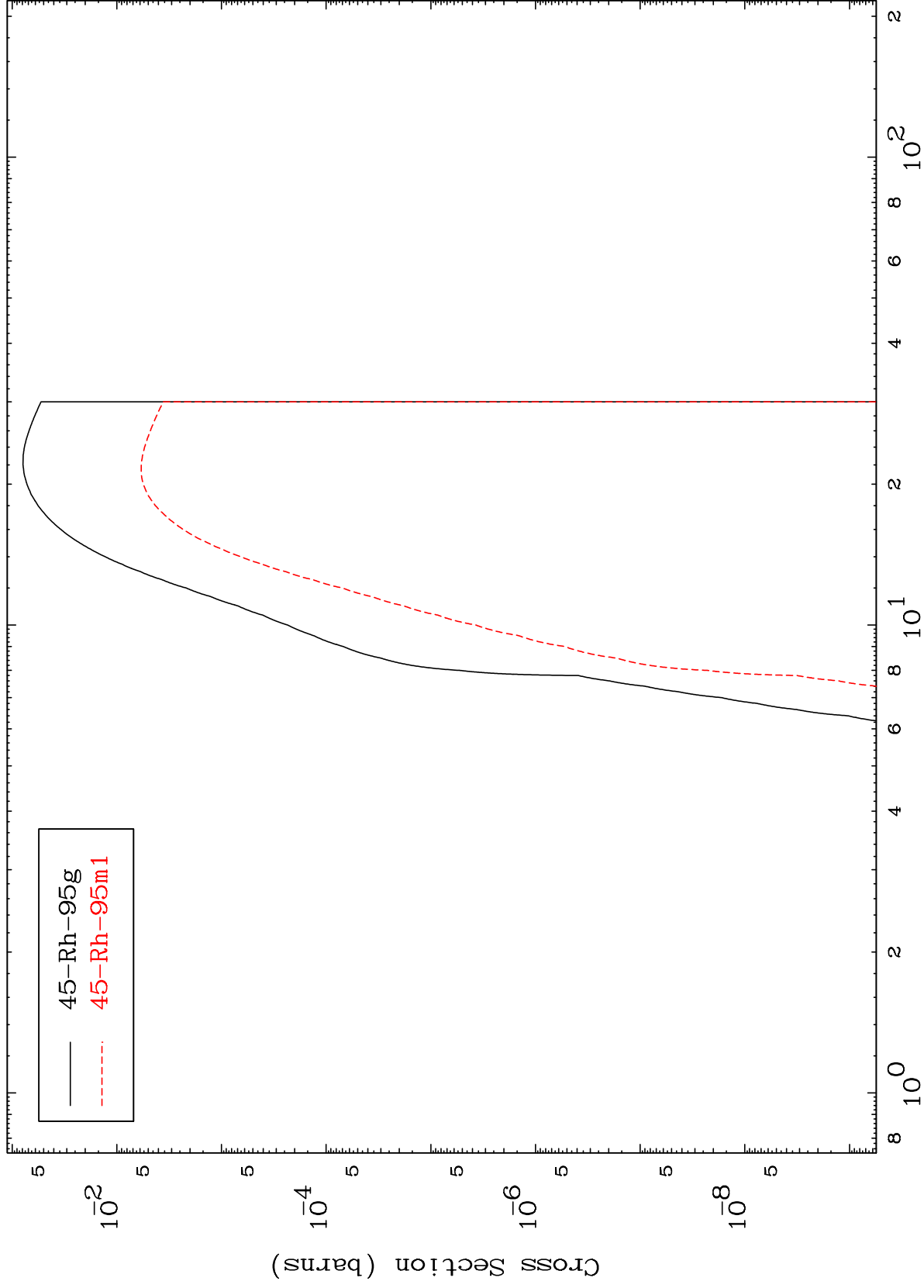




MAT 4701

47-Ag-99

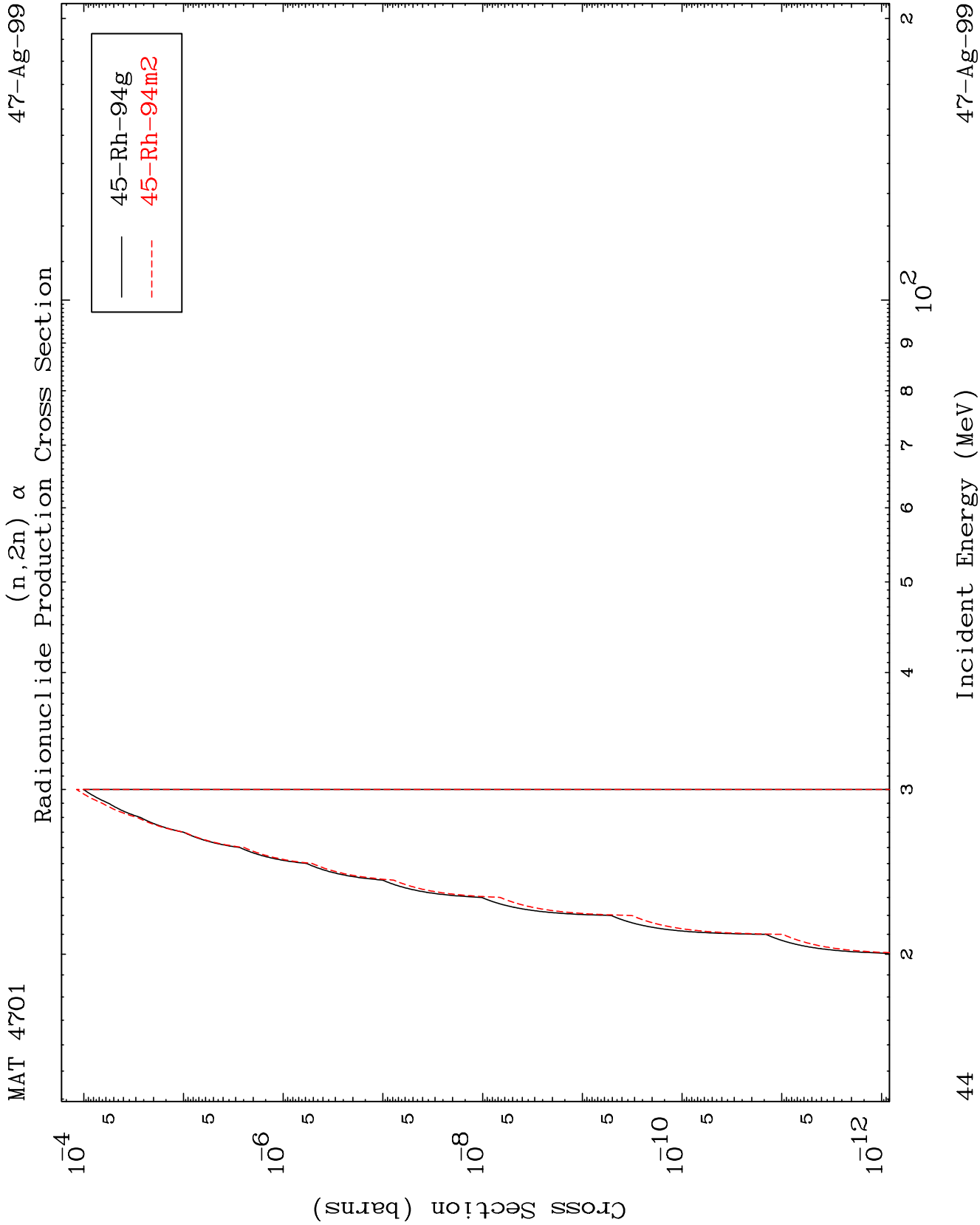
$(n,n') \alpha$
Radionuclide Production Cross Section



43

Incident Energy (MeV)

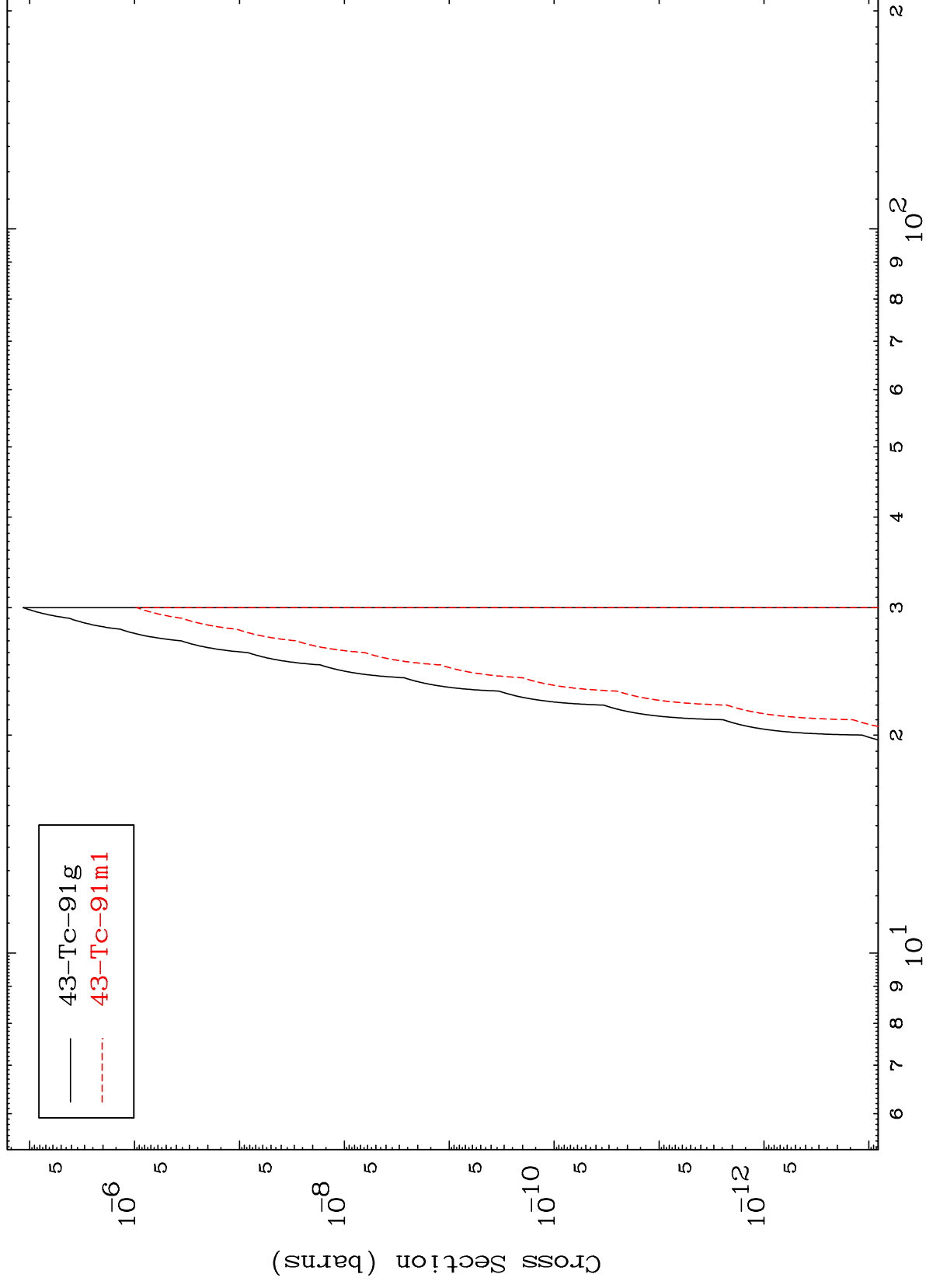
47-Ag-99



MAT 4701

47-Ag-99

$(n,n')\ 2\alpha$
Radionuclide Production Cross Section



45

Incident Energy (MeV)

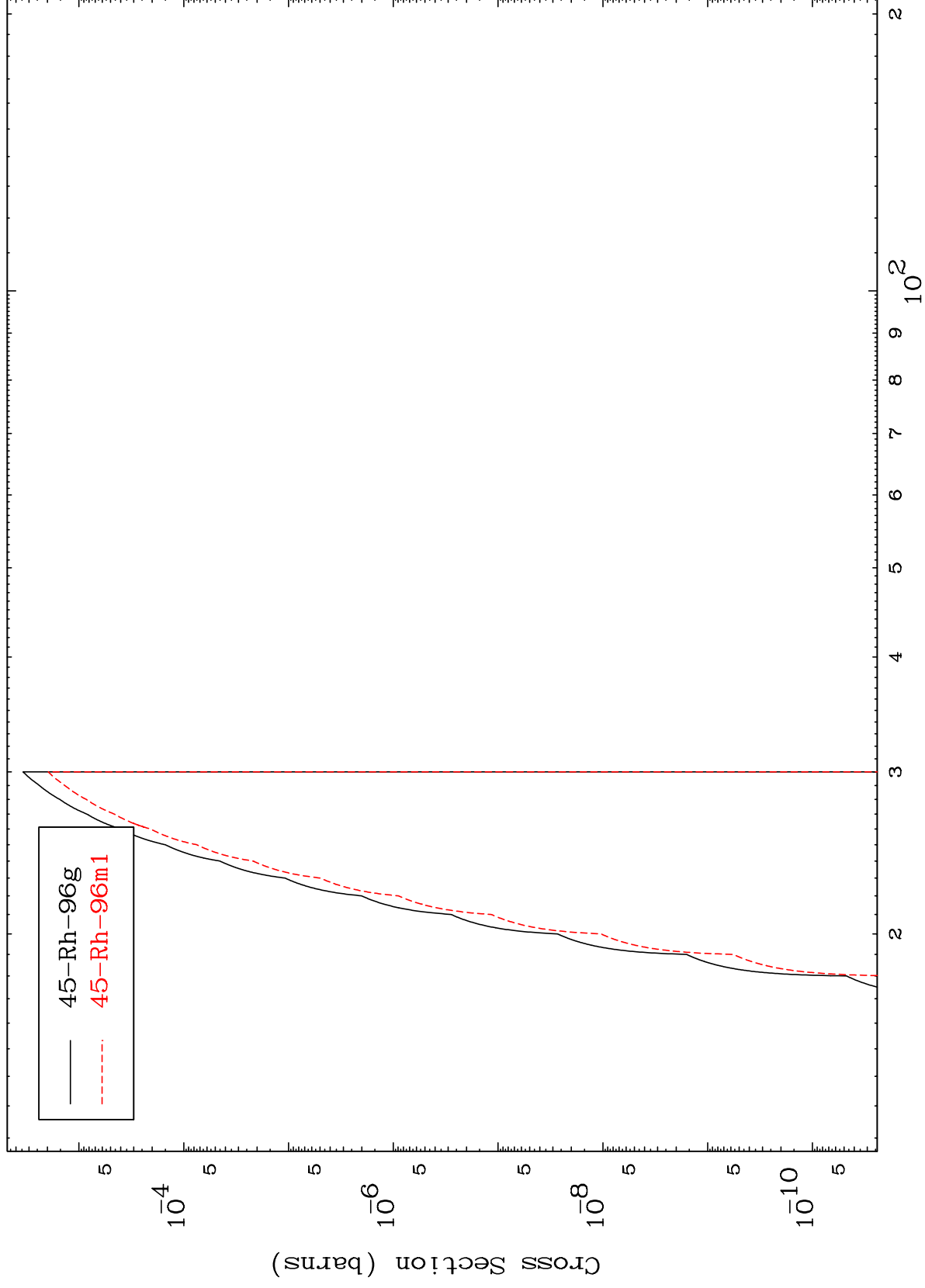
47-Ag-99

MAT 4701

(n,n') He-3

47-Ag-99

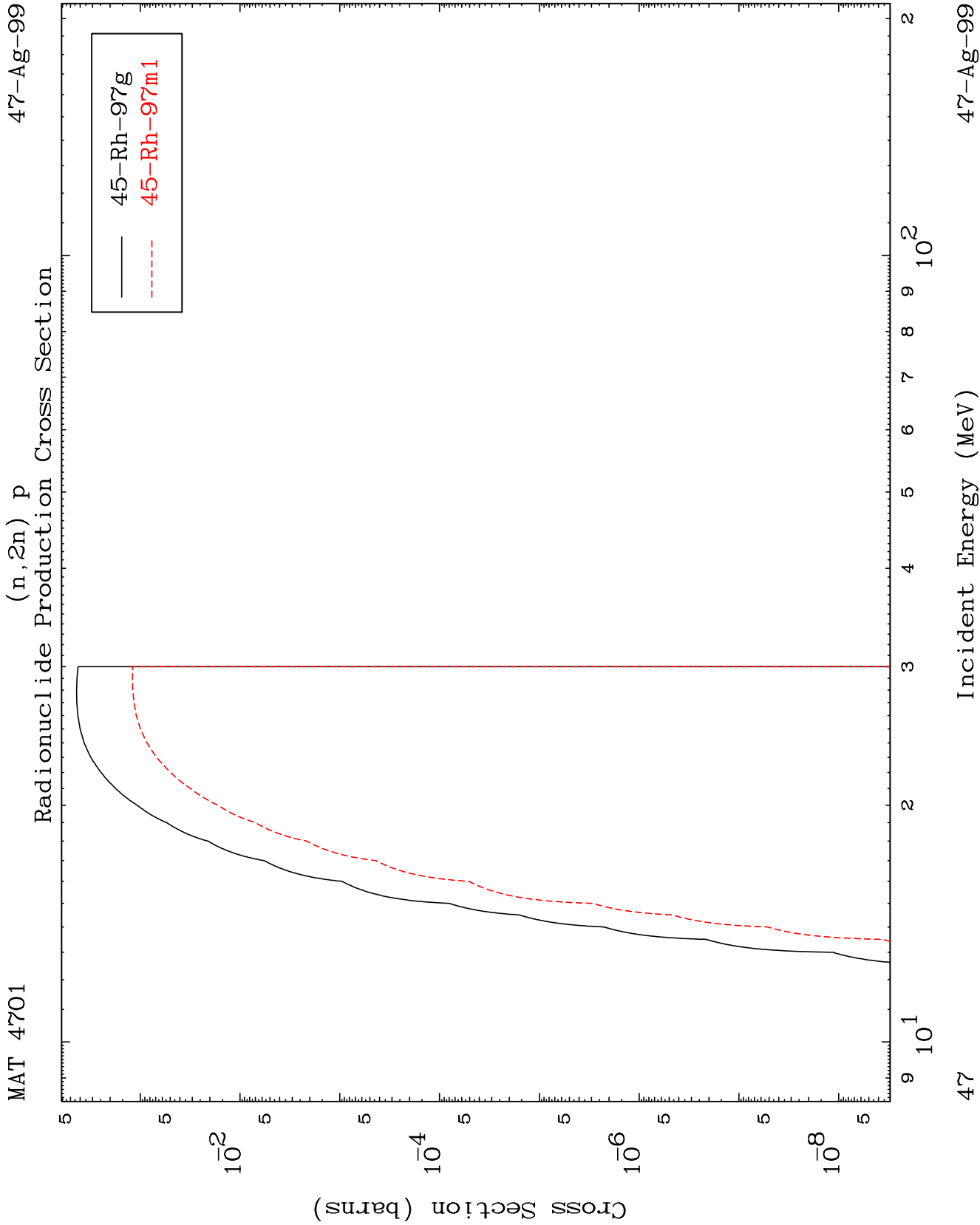
Radionuclide Production Cross Section



46

Incident Energy (MeV)

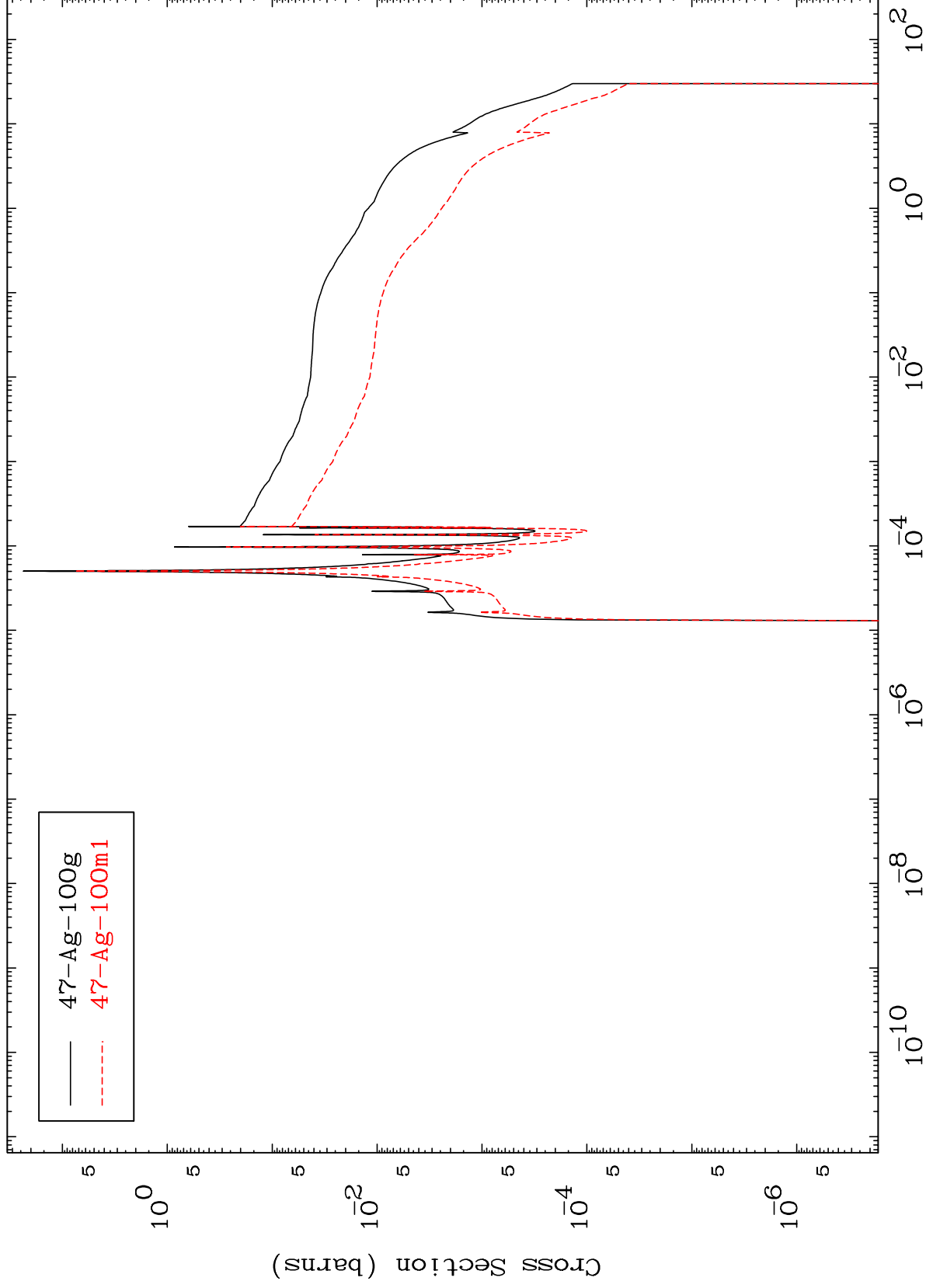
47-Ag-99



MAT 4701

47-Ag-99

Radionuclide Production Cross Section



48

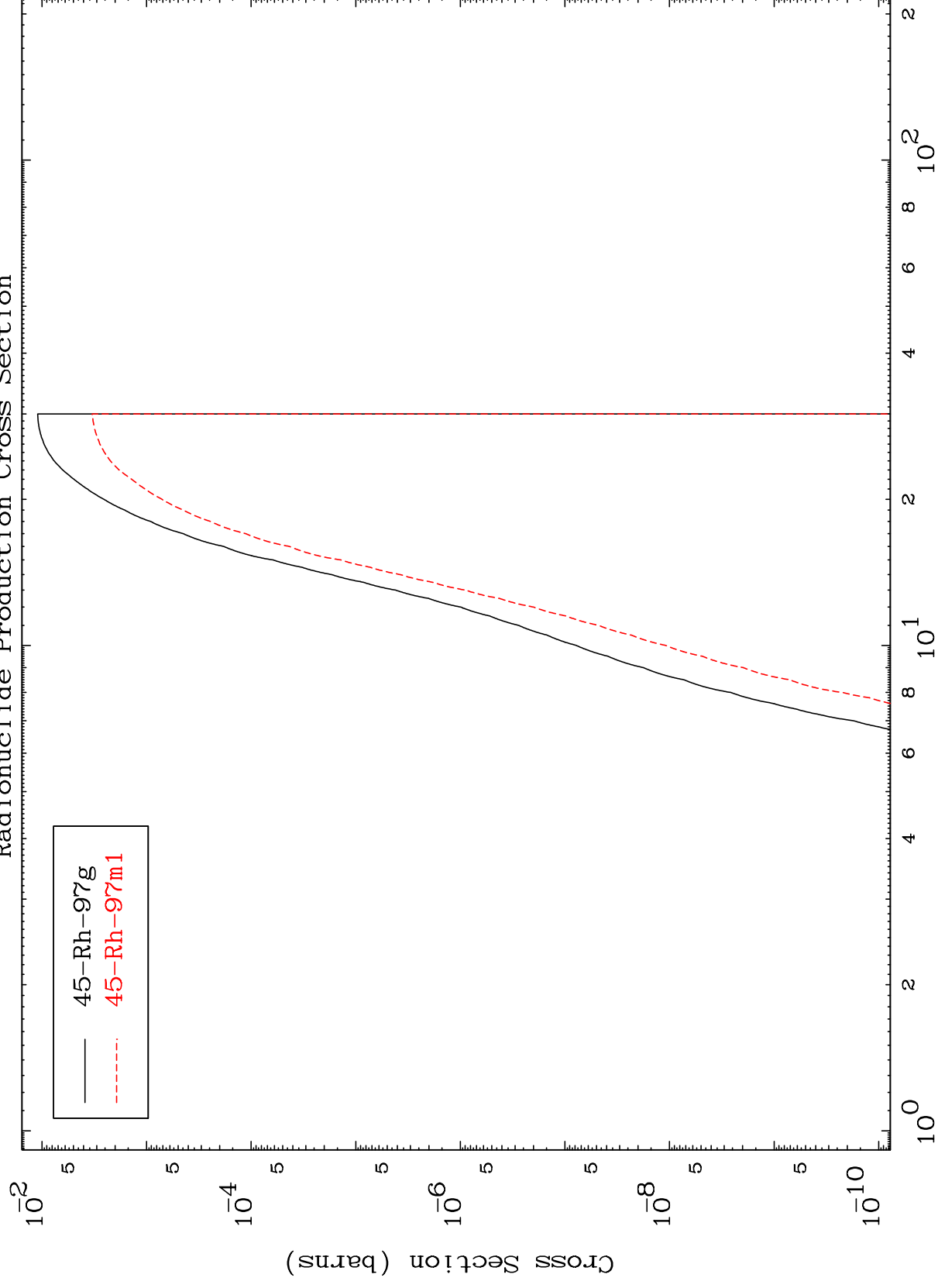
47-Ag-99

MAT 4701

 $(n, \text{He-3})$

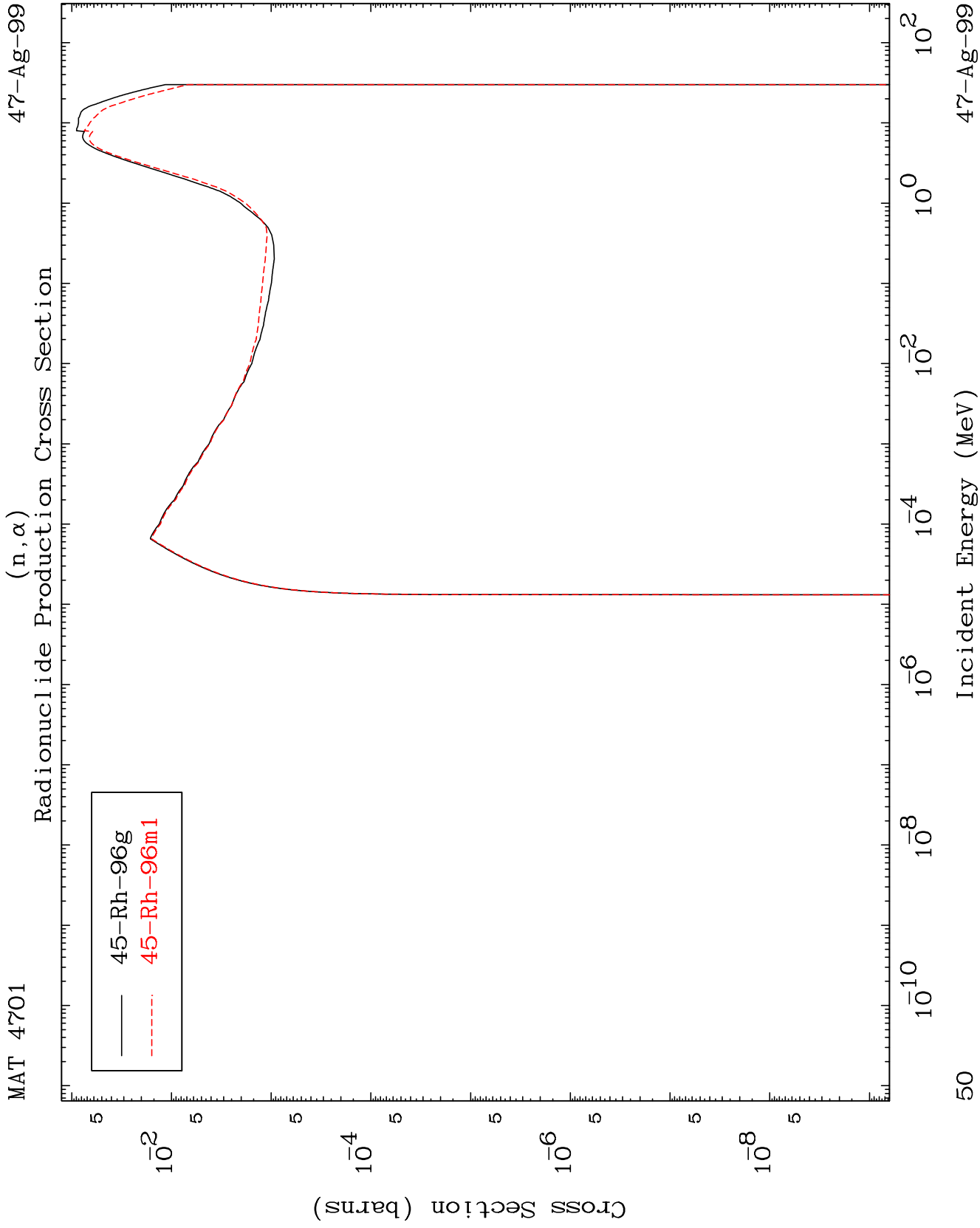
47-A8-99

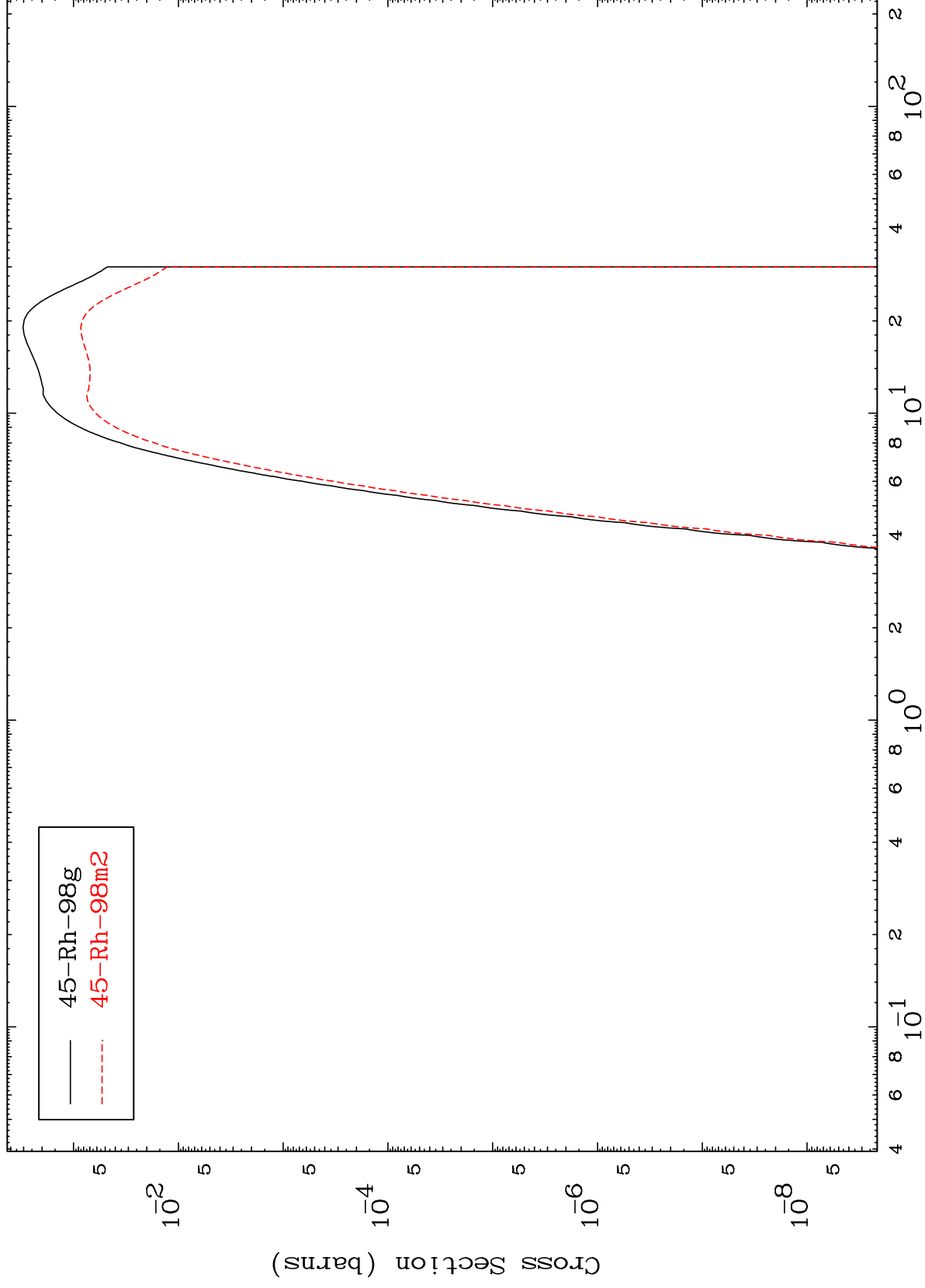
Radionuclide Production Cross Section



Incident Energy (MeV)

47-A8-99



Radionuclide Production Cross Section
(n,2p)

(n,p) d
Radionuclide Production Cross Section

