

Program EVALPLOT
(Version 2021-1)

by

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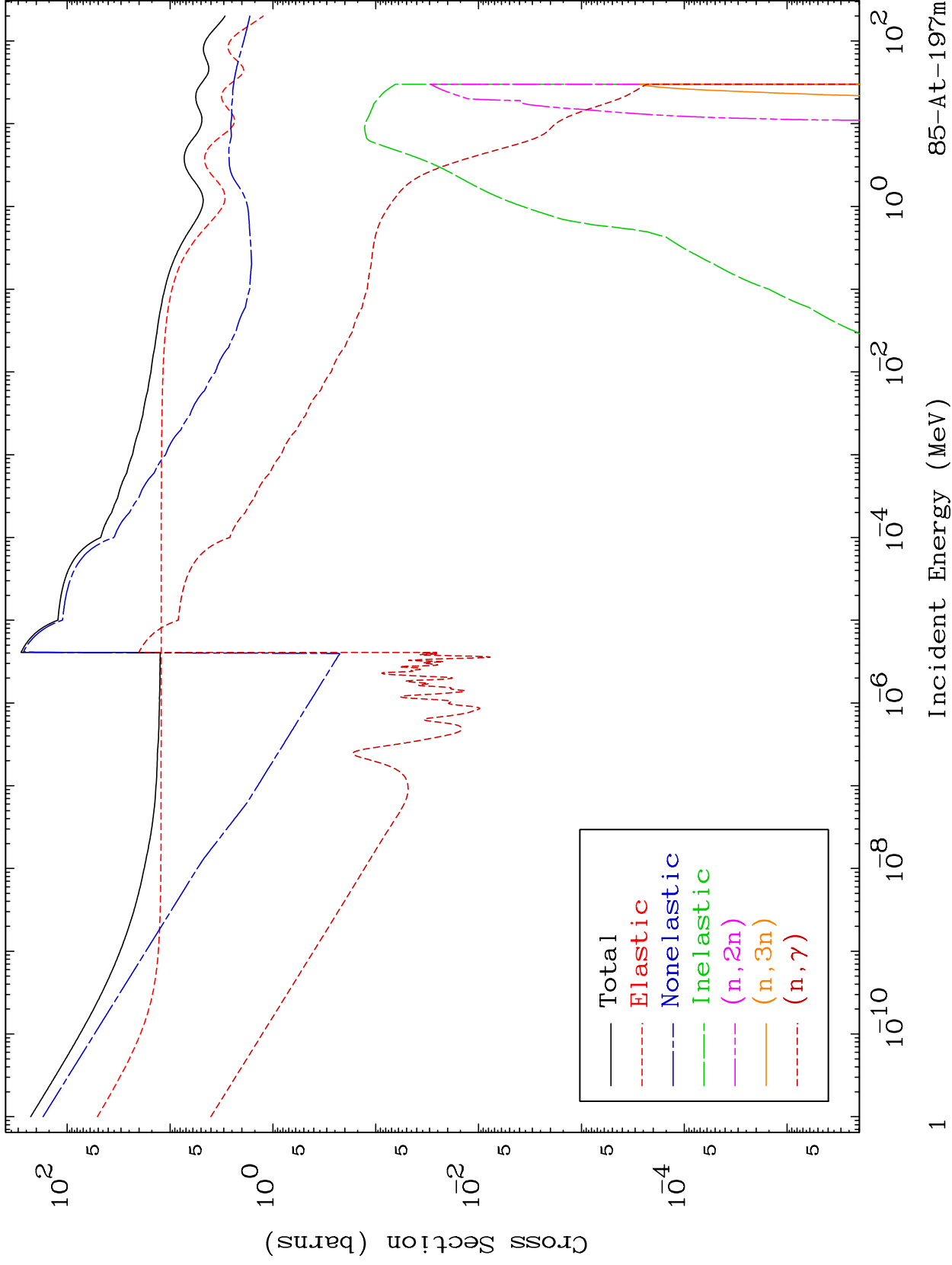
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8508

Neutron Major
293 Kelvin Cross Sections

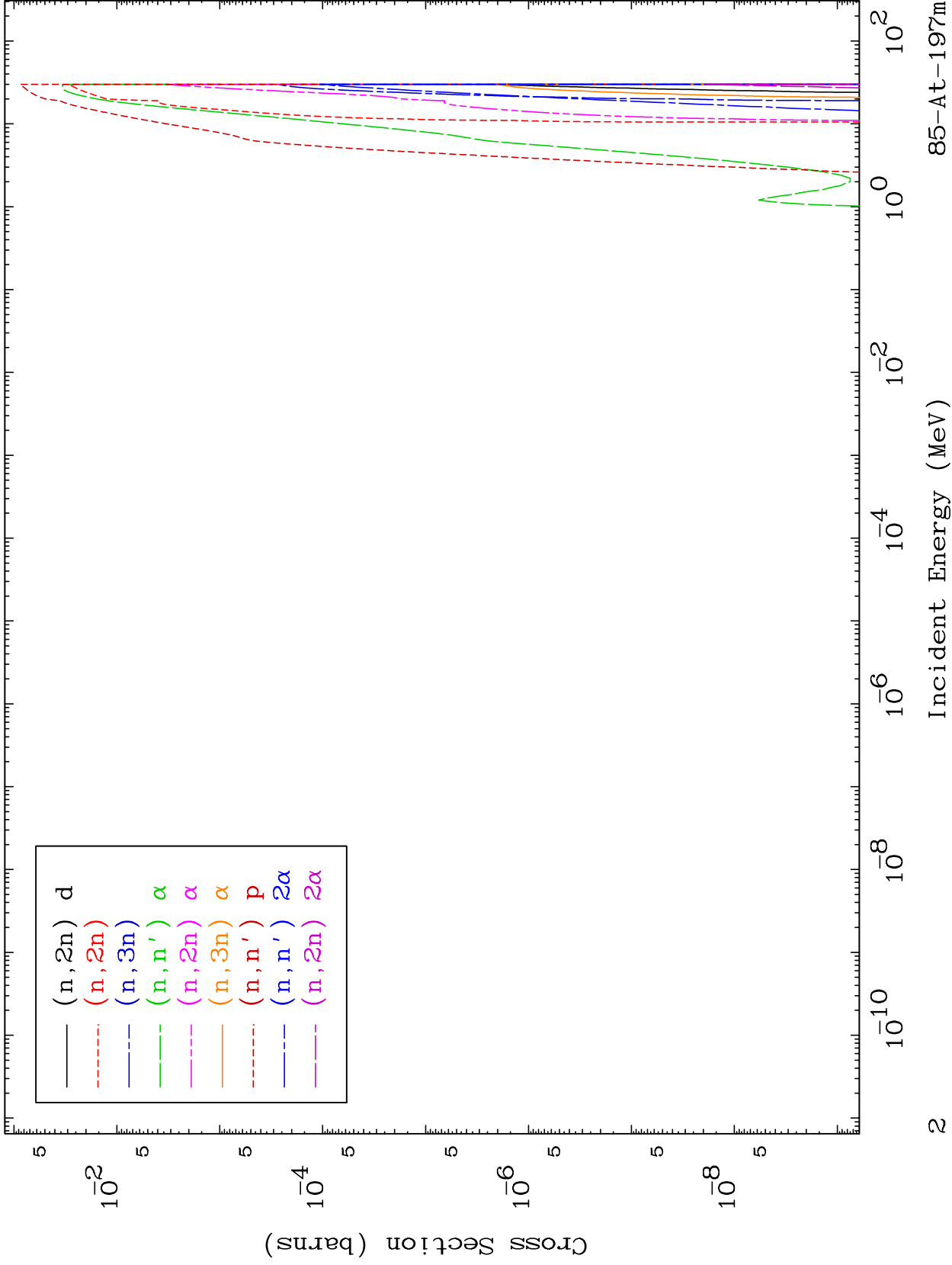
85-At-197m



MAT 8508

Neutron Absorption
293 Kelvin Cross Sections

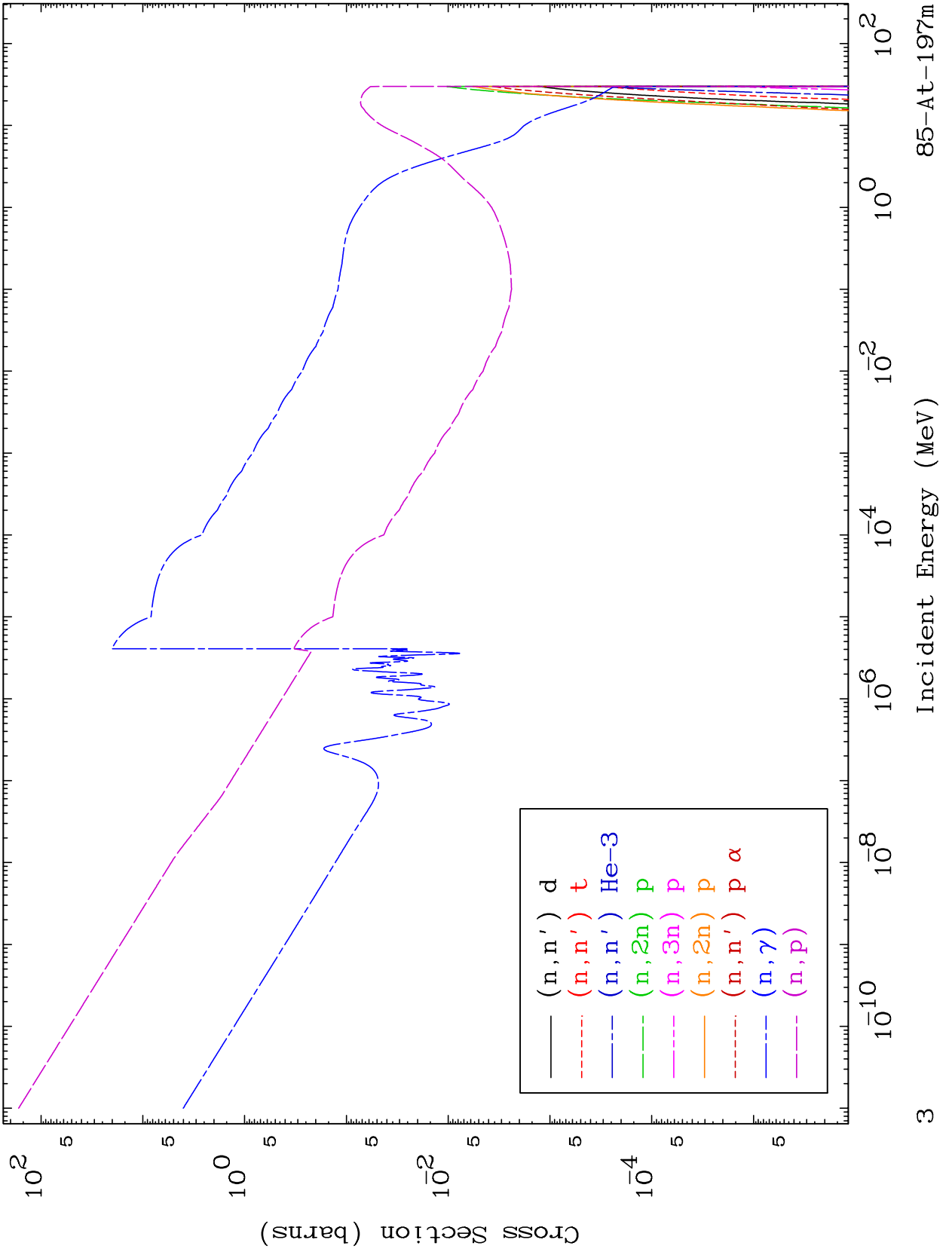
85-At-197m



MAT 8508

Neutron Absorption
293 Kelvin Cross Sections

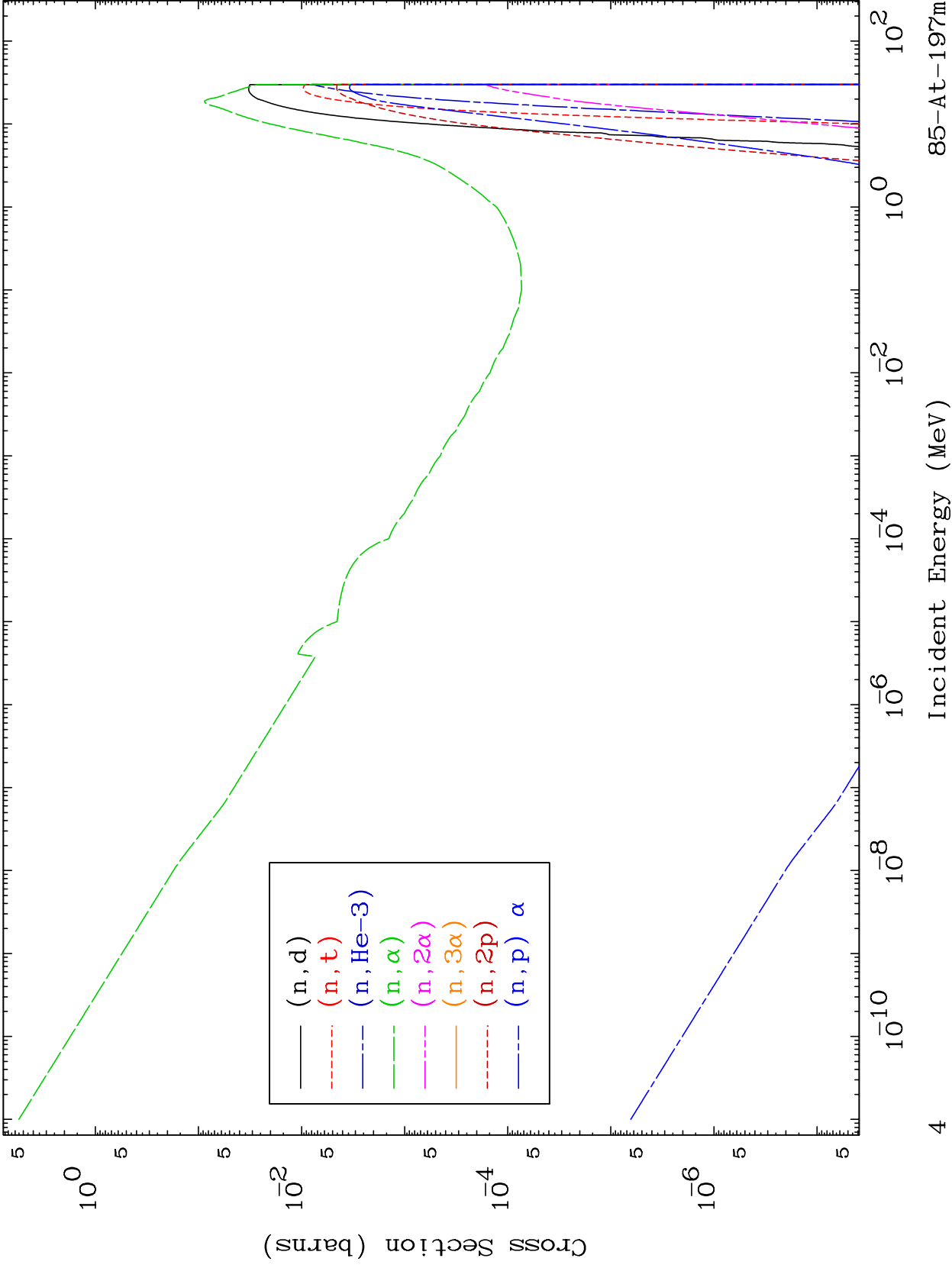
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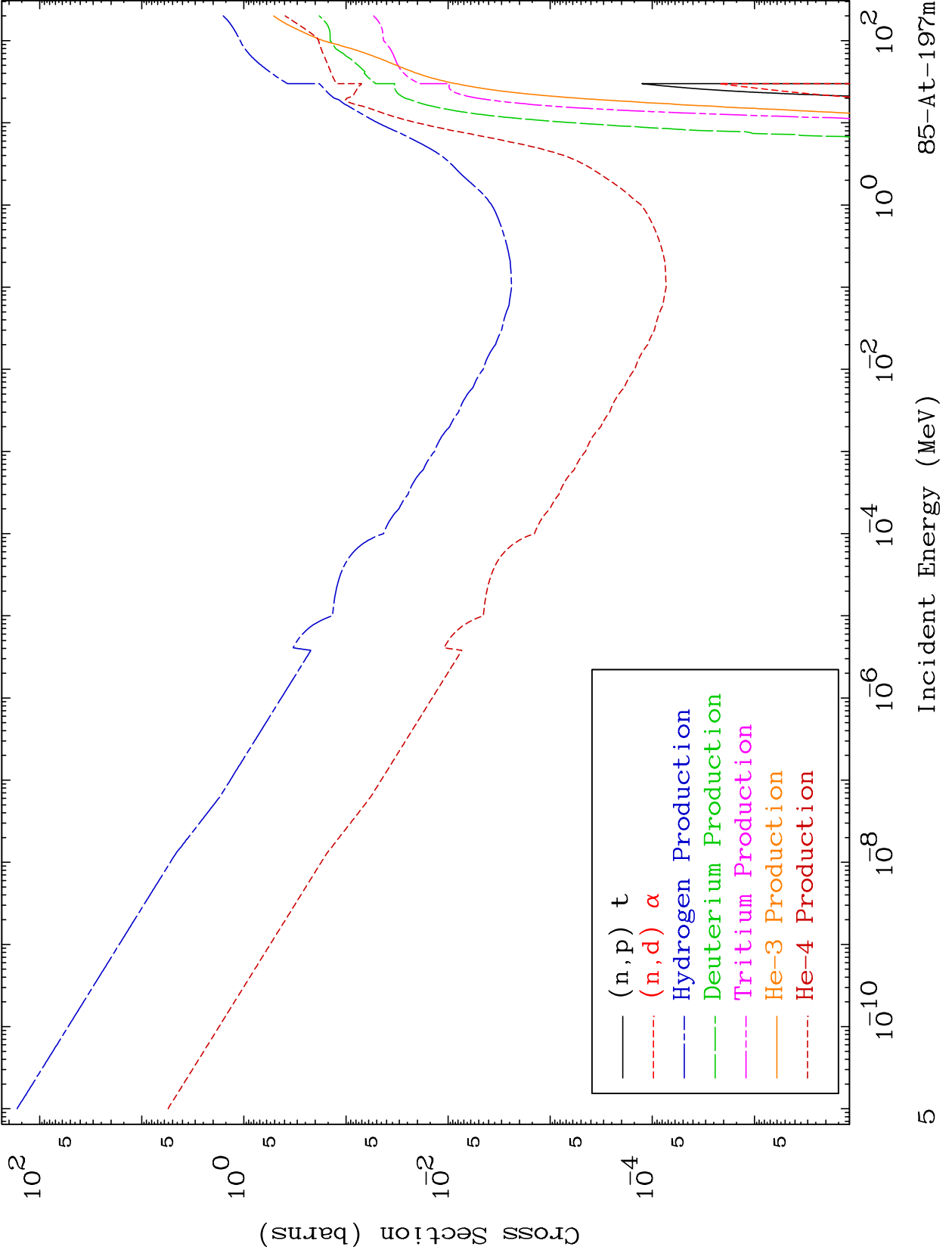


MAT 8508

Neutron Absorption
293 Kelvin Cross Sections

85-At-197m

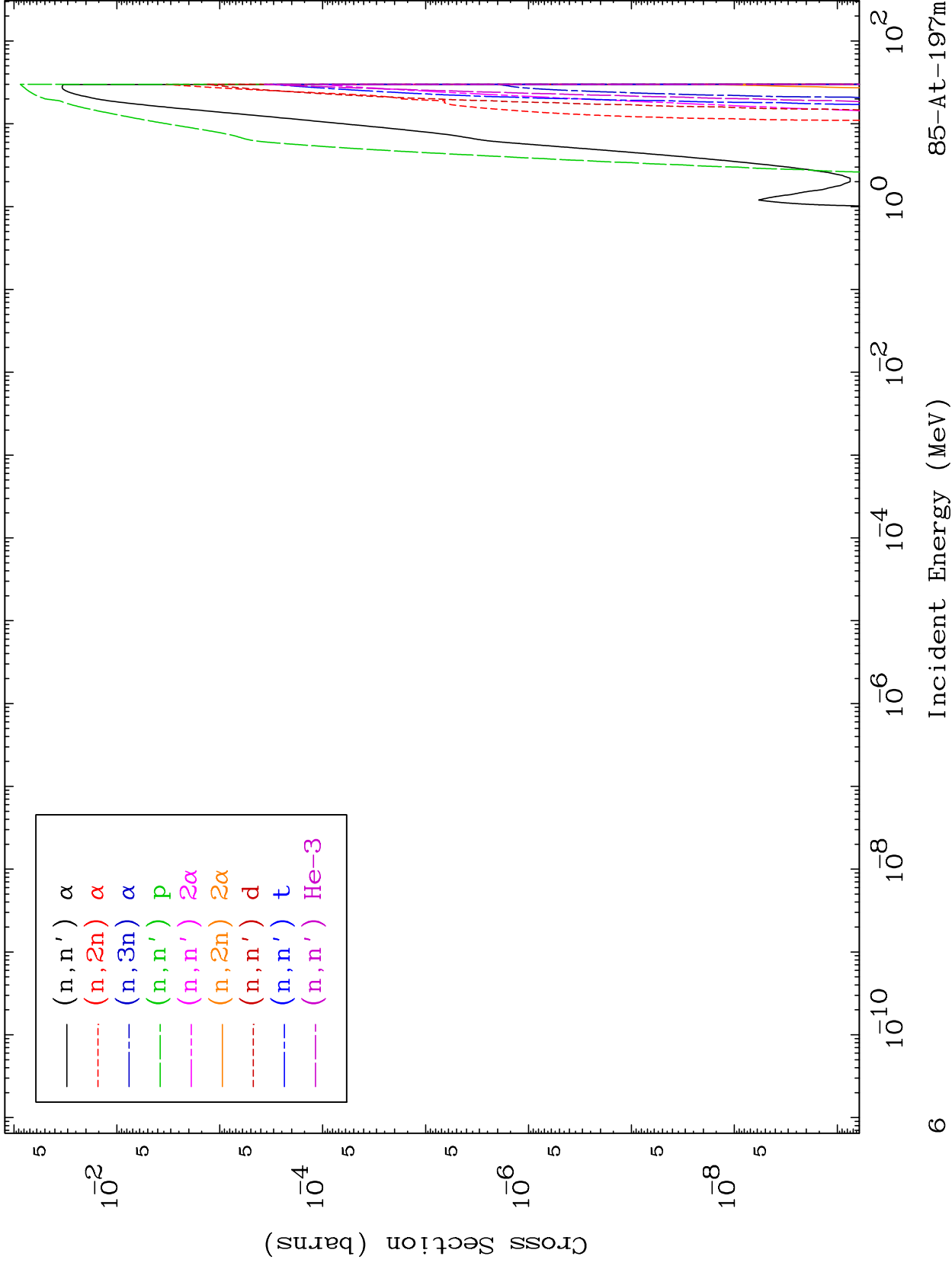




MAT 8508

Charged Particle
293 Kelvin Cross Sections

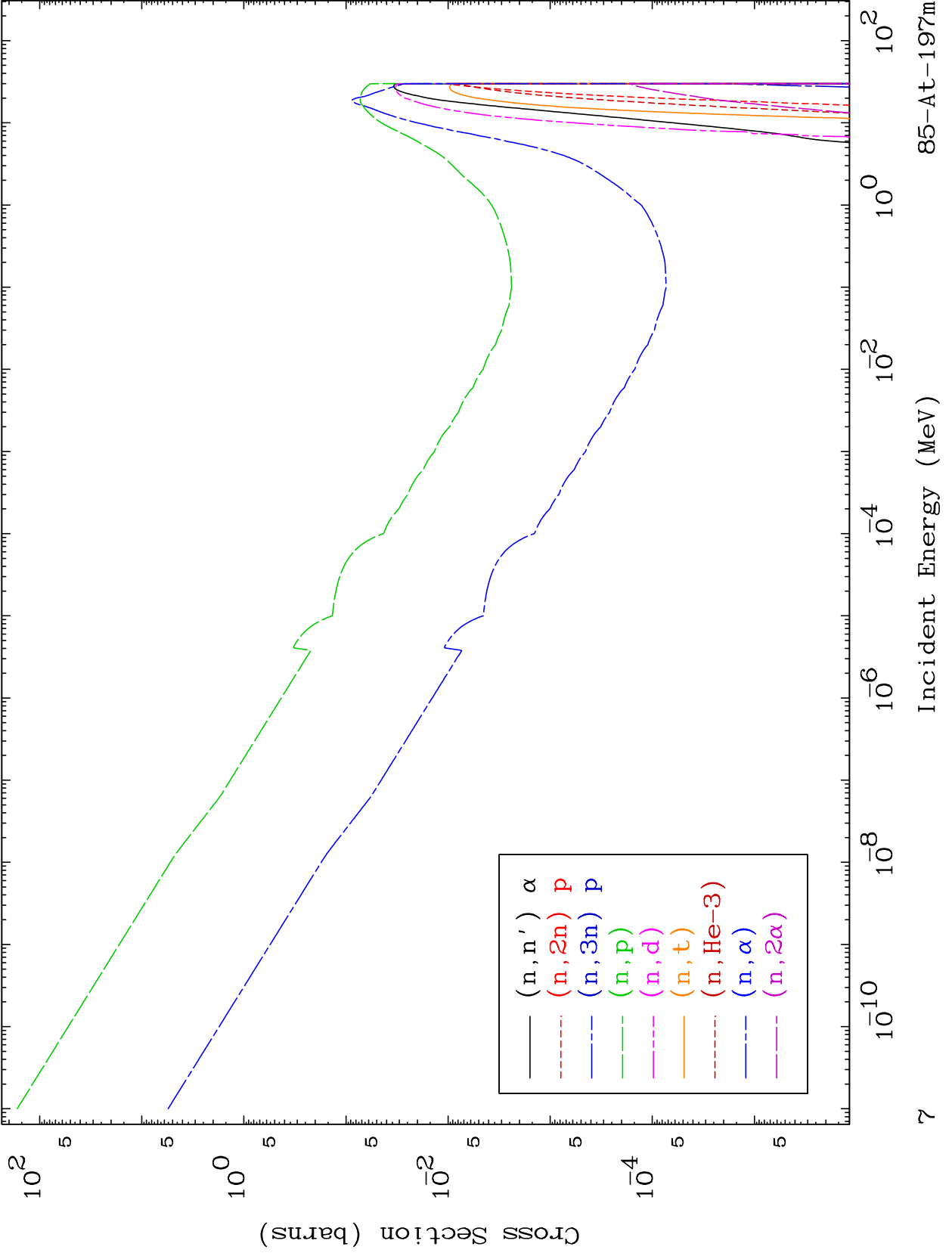
85-At-197m



MAT 8508

Charged Particle
293 Kelvin Cross Sections

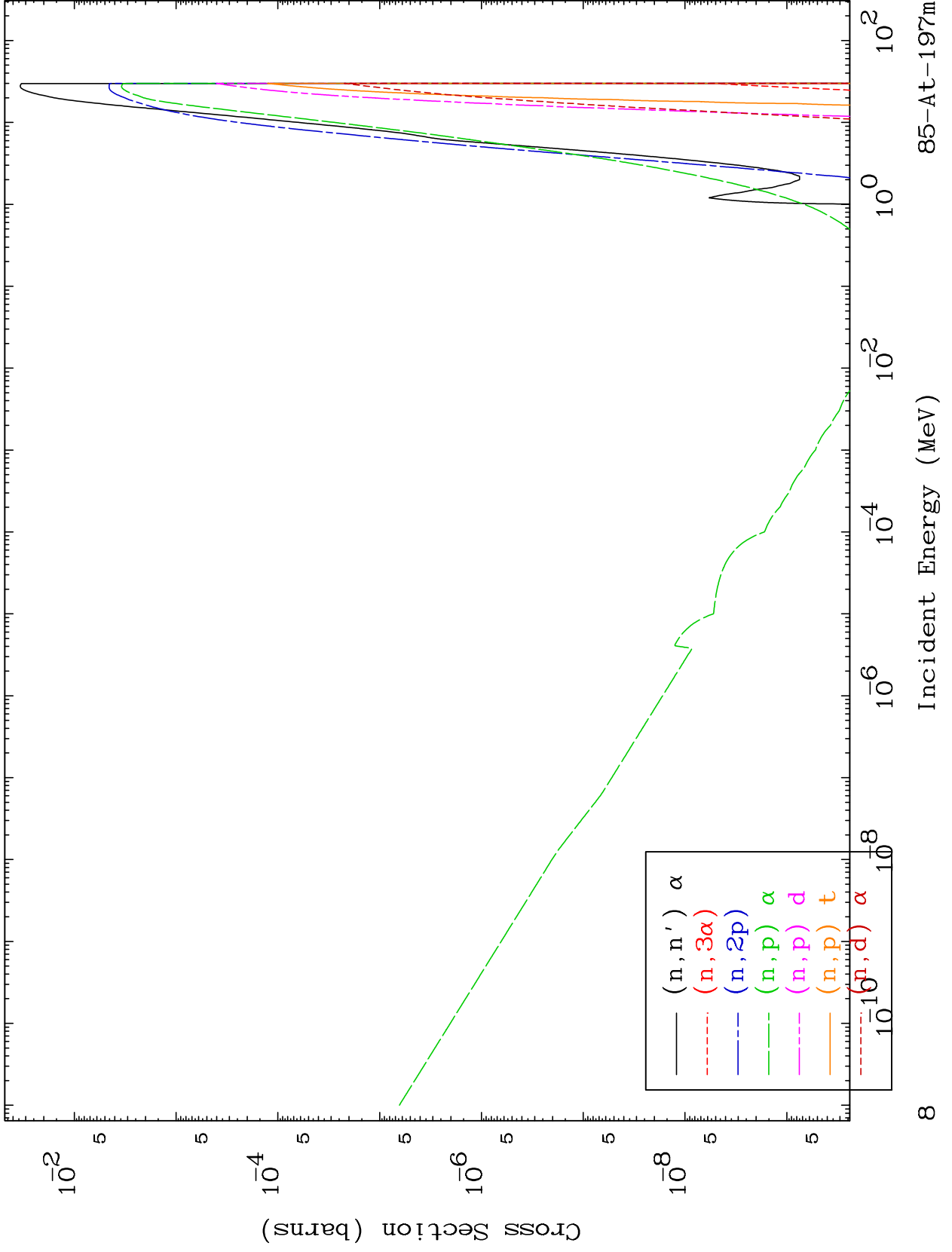
85-At-197m



MAT 8508

Charged Particle
293 Kelvin Cross Sections

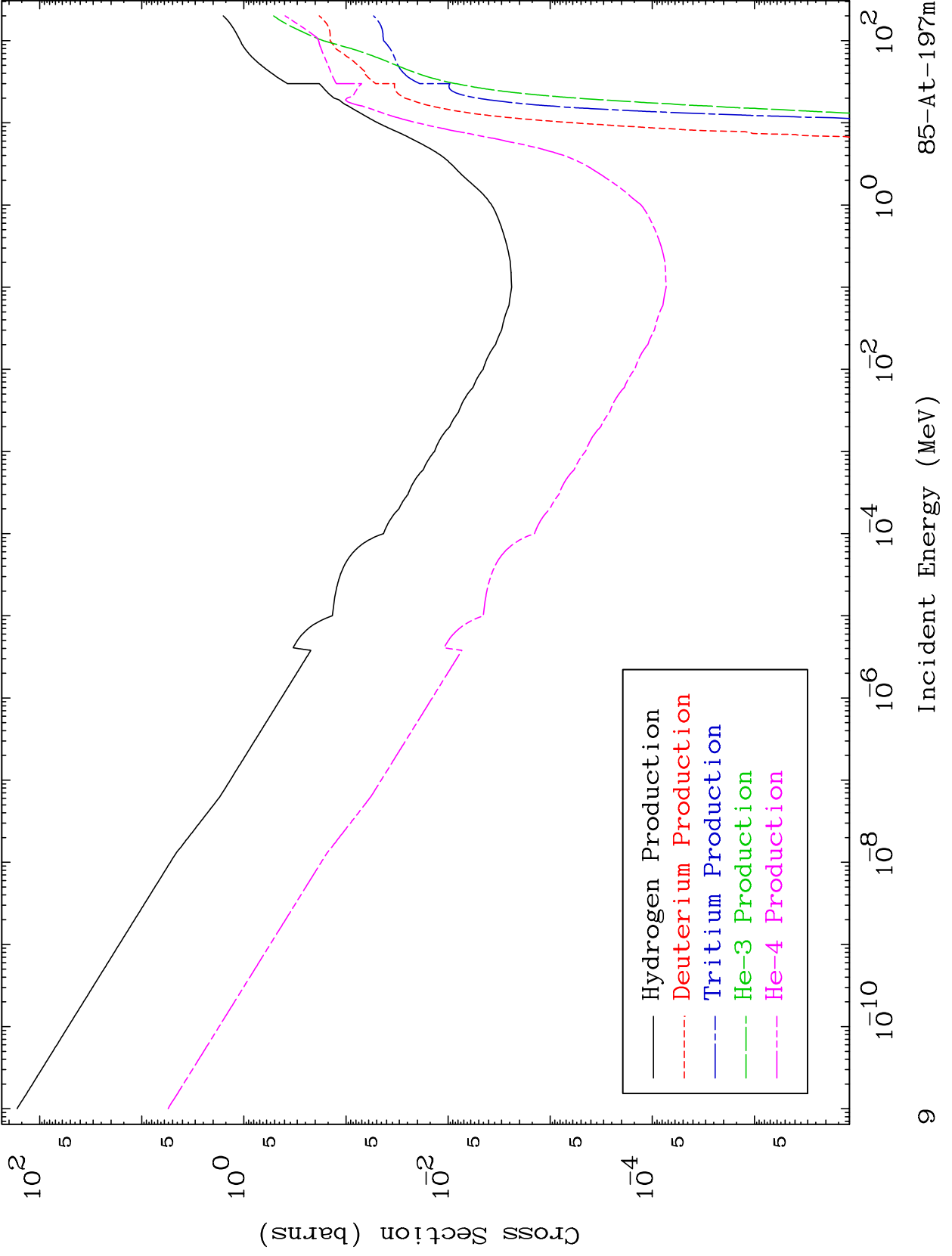
85-At-197m



MAT 8508

Particle Production
293 Kelvin Cross Sections

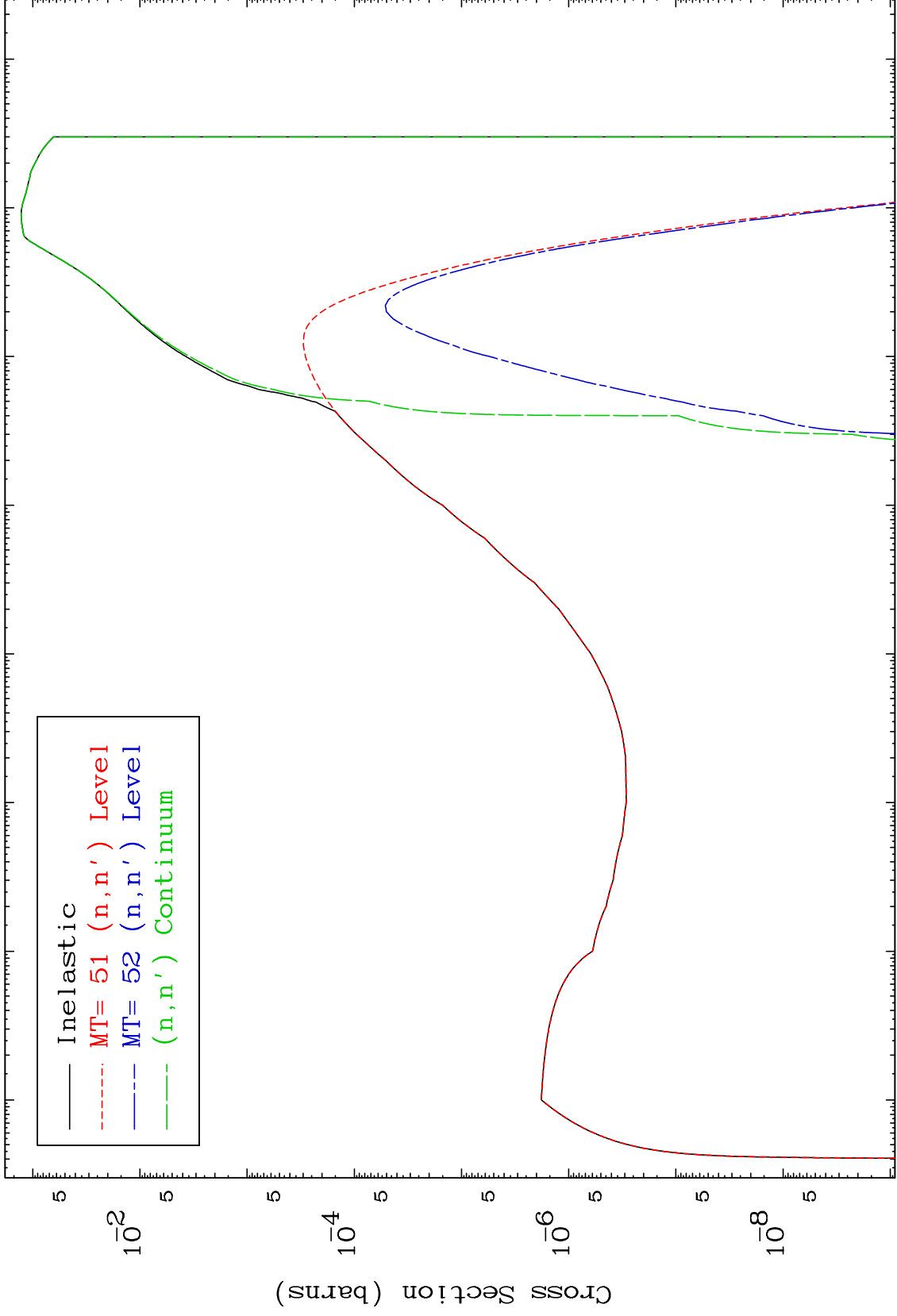
85-At-197m



MAT 8508

(n,n') Levels
293 Kelvin Cross Sections

85-At-197m



Legend:
— Inelastic
- - - MT= 51 (n,n') Level
- . - MT= 52 (n,n') Level
- - - (n,n') Continuum

10

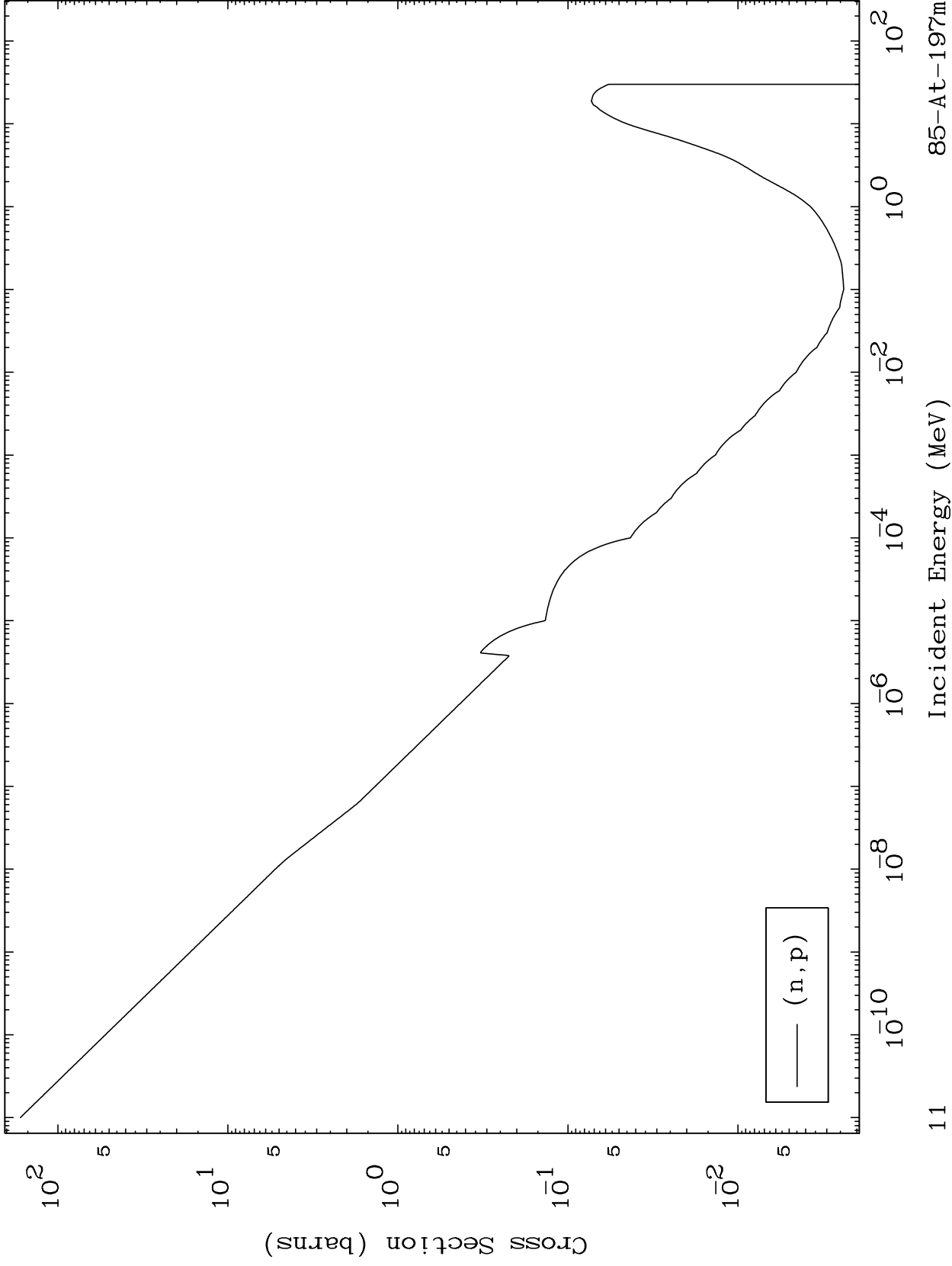
Incident Energy (MeV)

85-At-197m

MAT 8508

(n,p) Levels
293 Kelvin Cross Sections

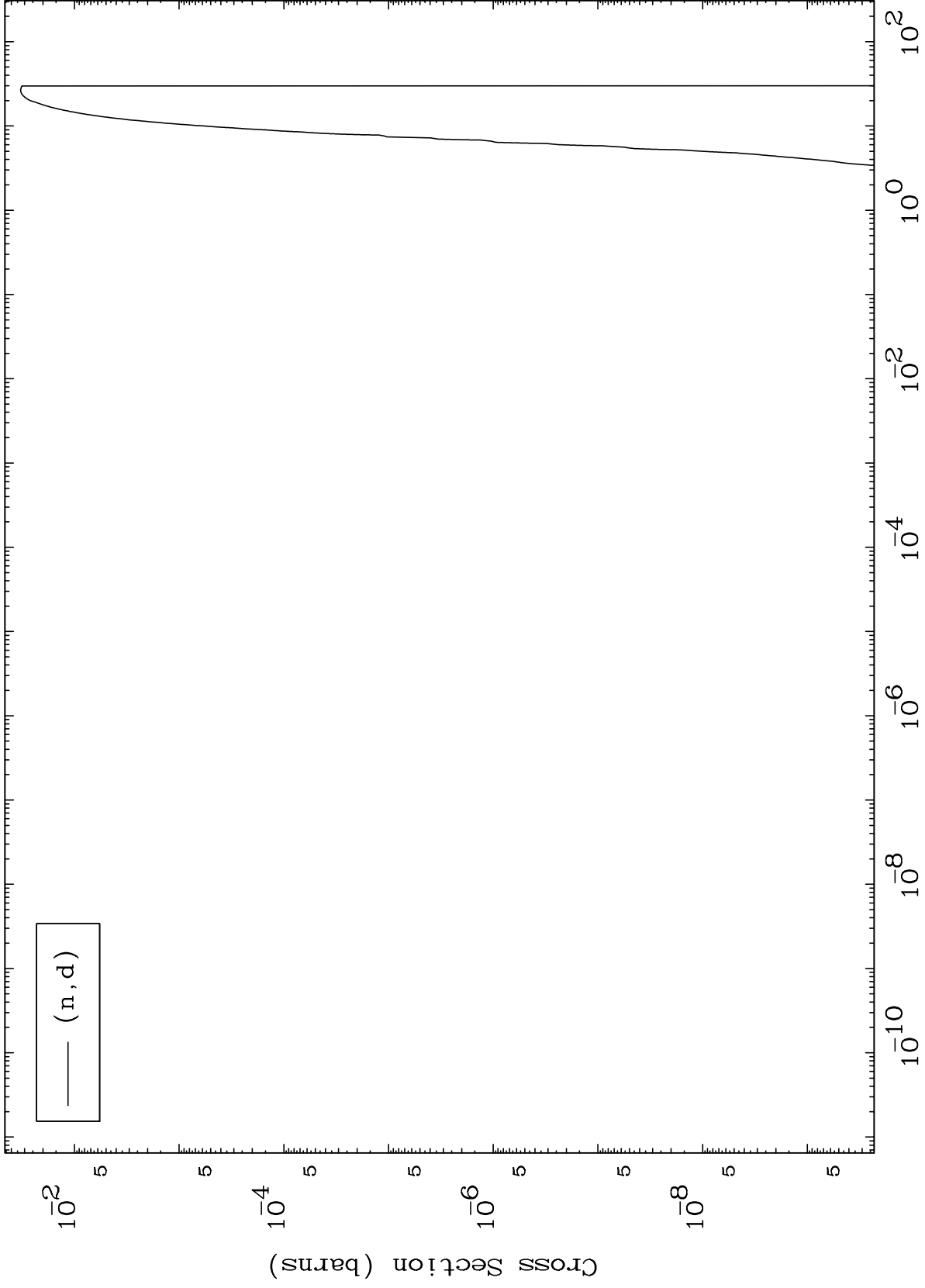
85-At-197m



MAT 8508

(n,d) Levels
293 Kelvin Cross Sections

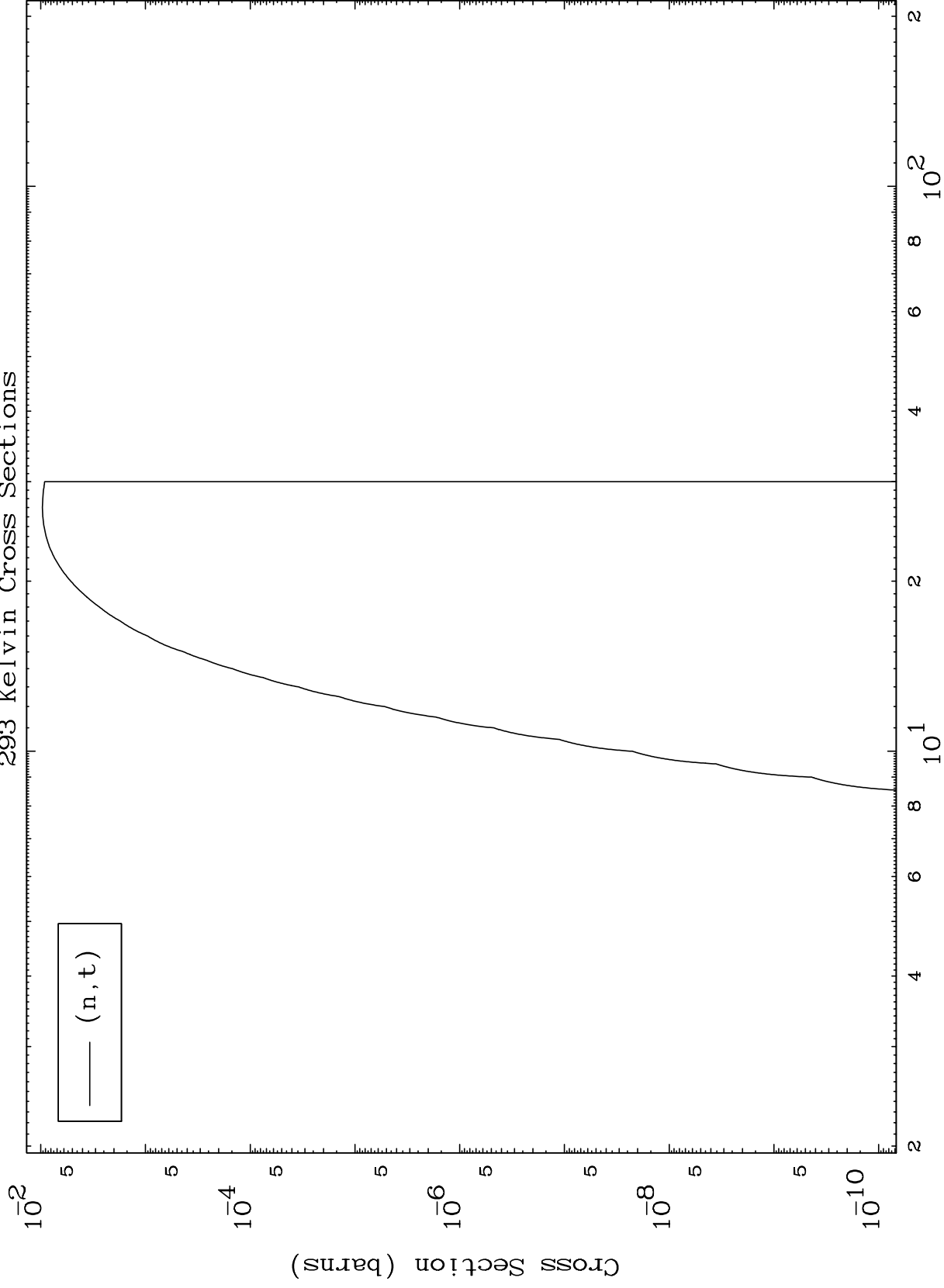
85-At-197m



MAT 8508

(n,t) Levels
293 Kelvin Cross Sections

85-At-197m



13

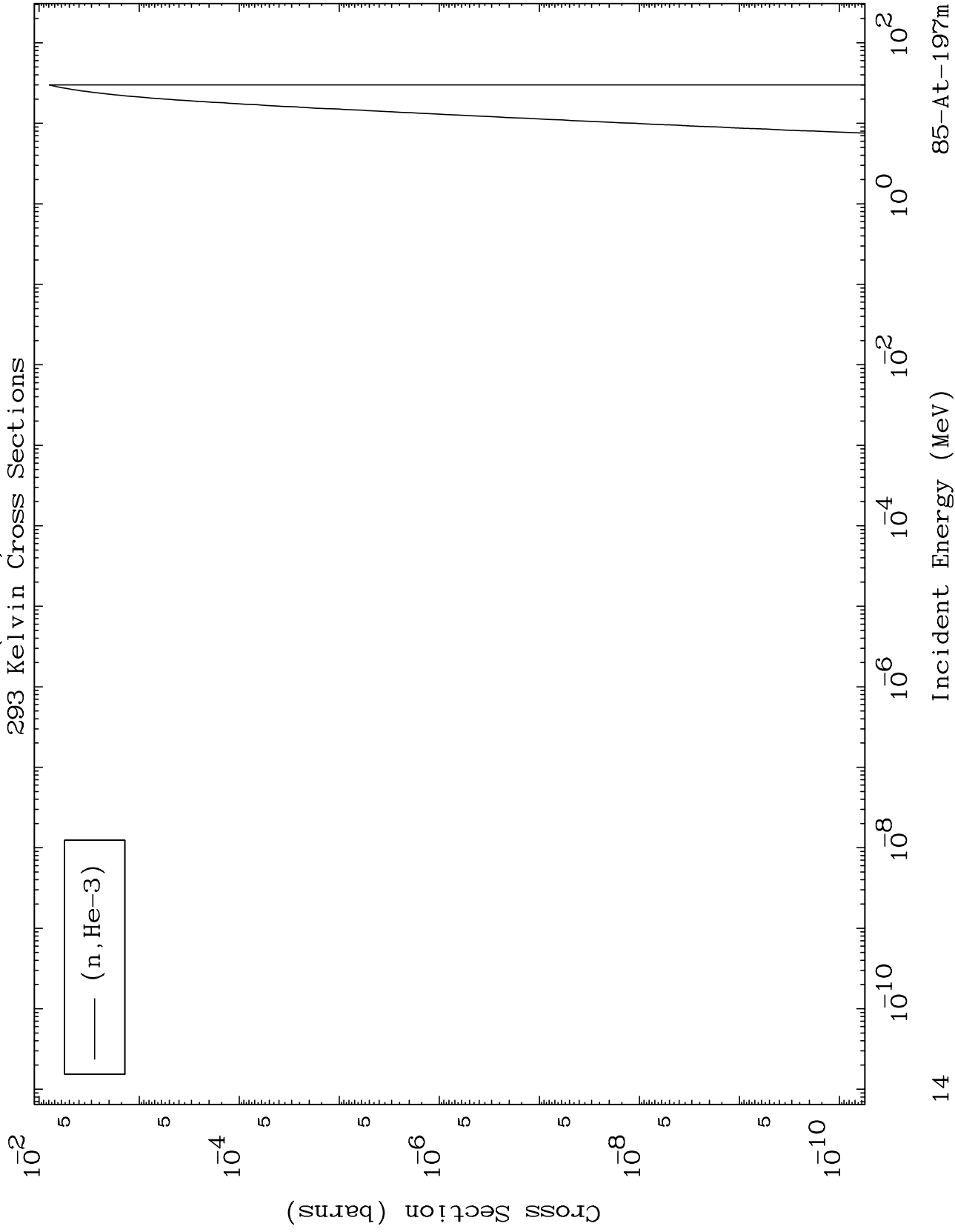
Incident Energy (MeV)

85-At-197m

MAT 8508

(n,He3) Levels
293 Kelvin Cross Sections

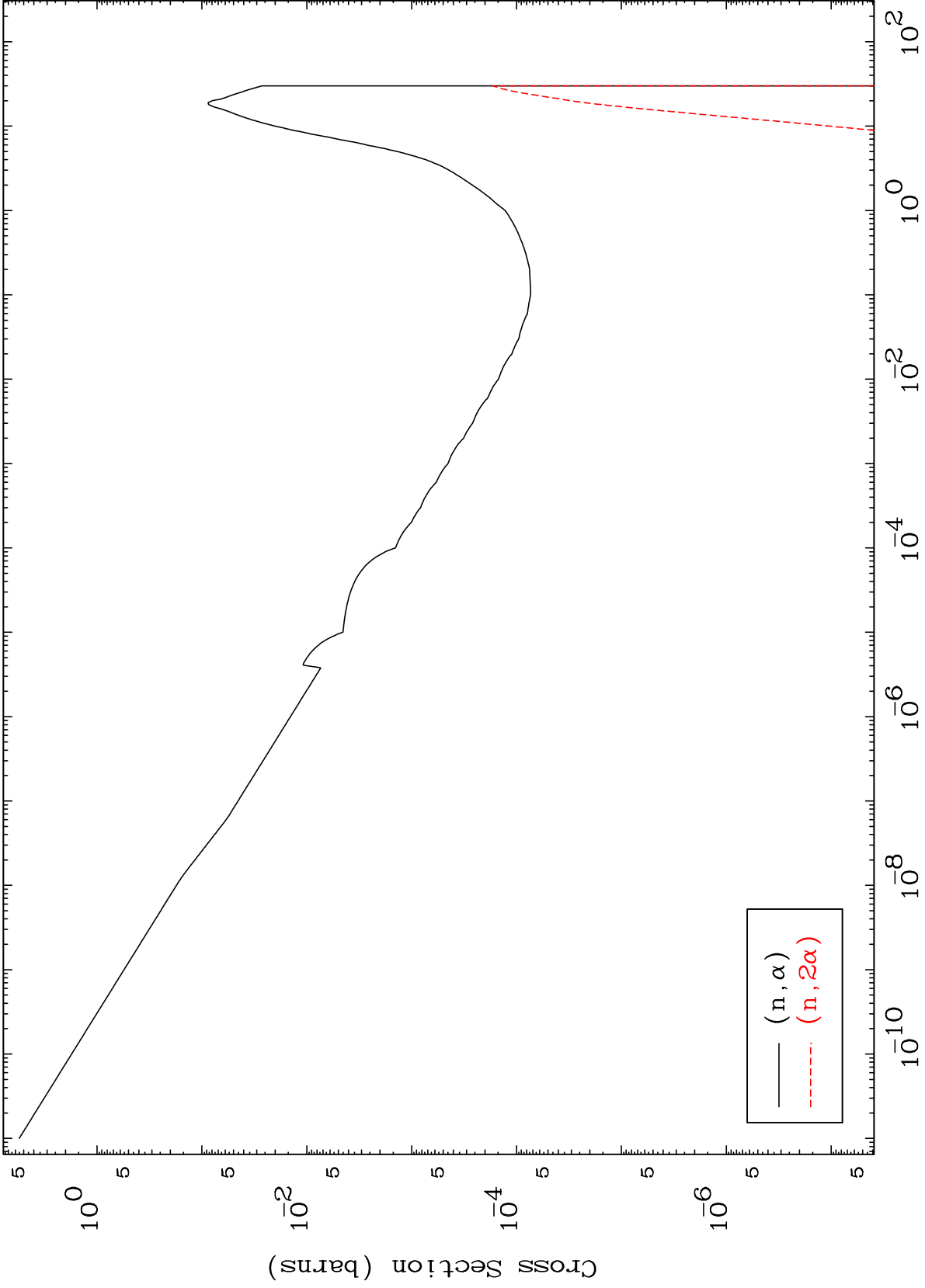
85-At-197m



MAT 8508

(n,α) Levels
293 Kelvin Cross Sections

85-At-197m



15

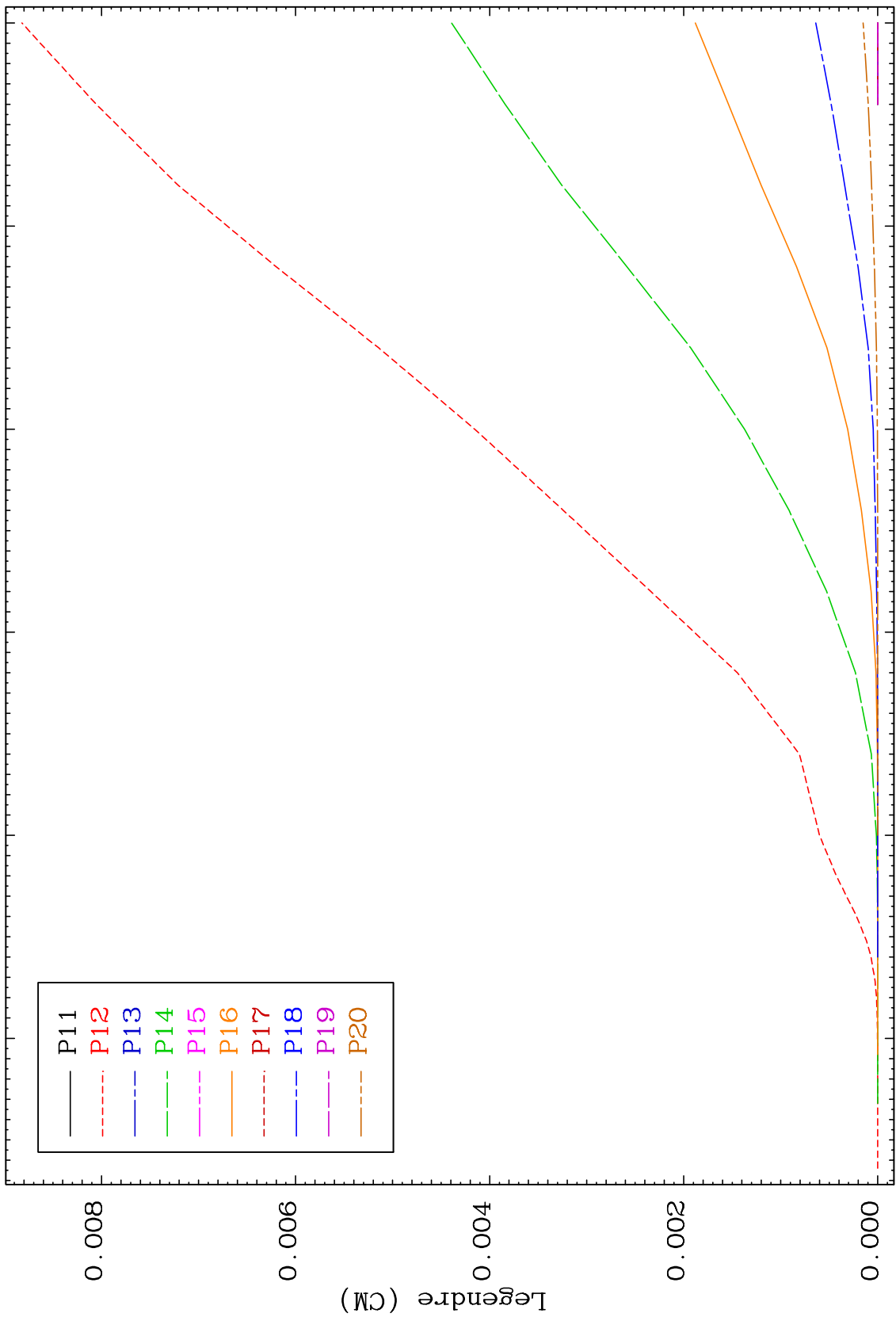
Incident Energy (MeV)

85-At-197m

MAT 8508

Elastic Legendre Coefficients

85-At-197m



17

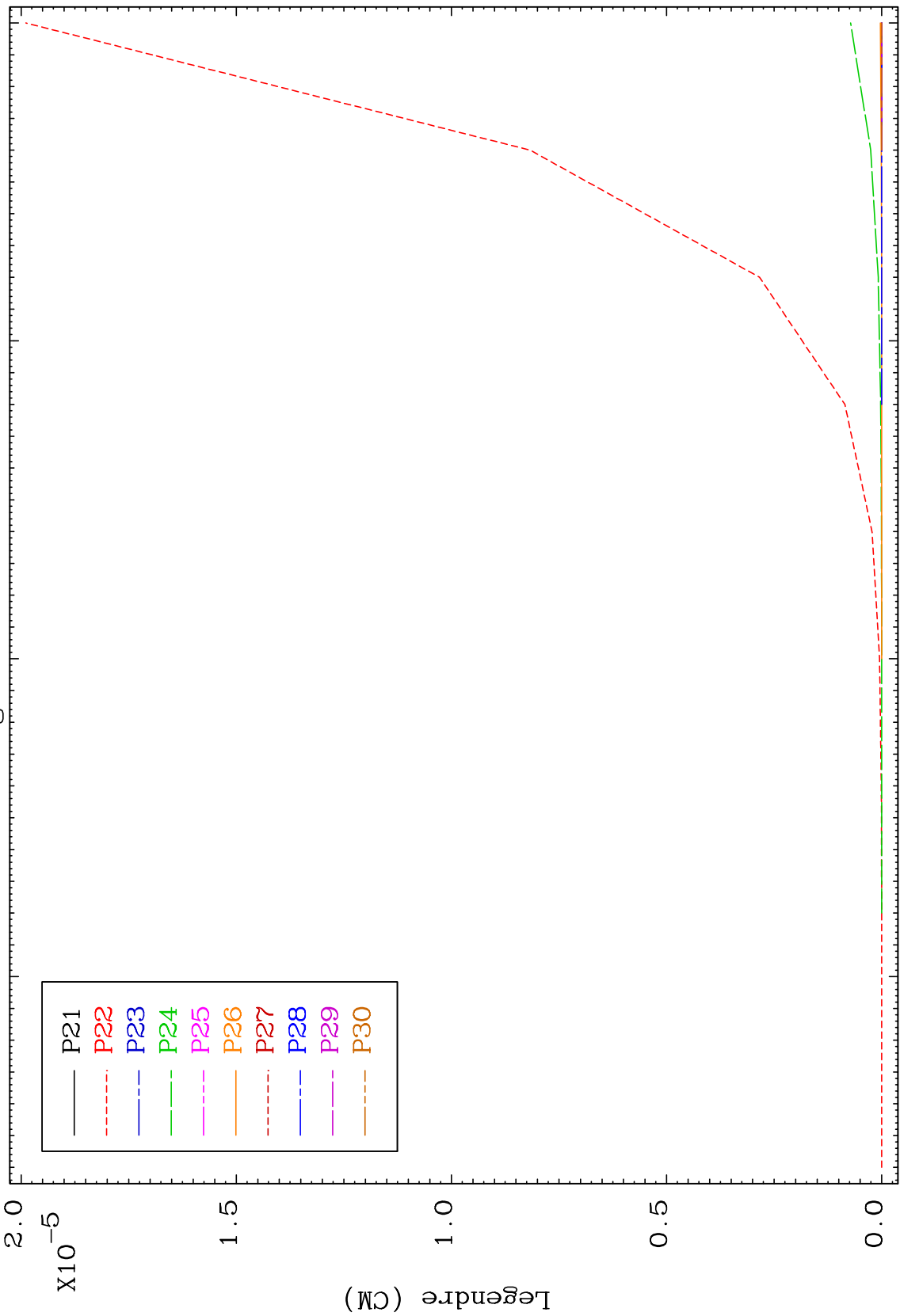
Incident Energy (MeV)

85-At-197m

MAT 8508

Elastic Legendre Coefficients

85-At-197m



18

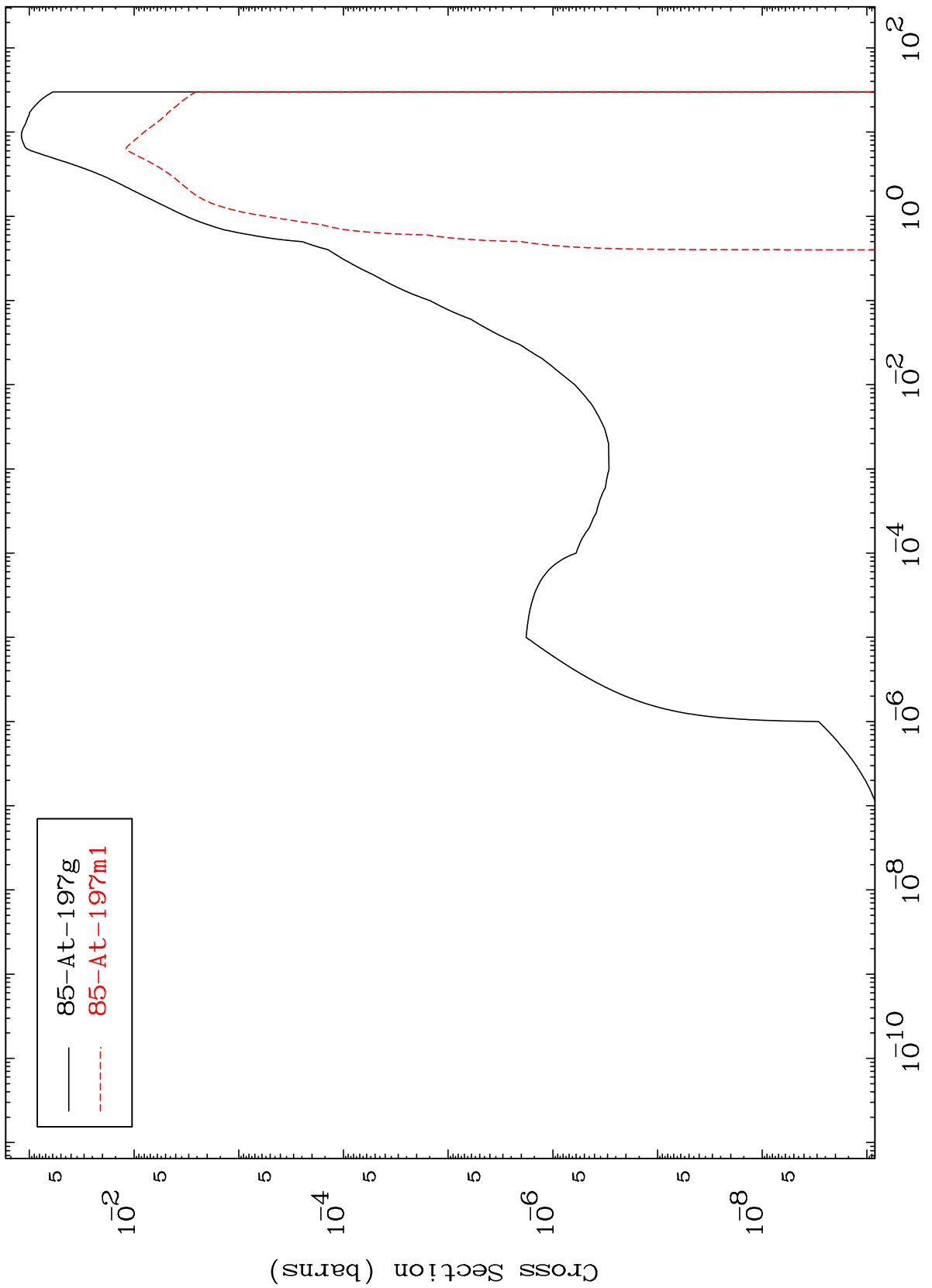
Incident Energy (MeV)

85-At-197m

MAT 8508

Inelastic
Radionuclide Production Cross Section

85-At-197m

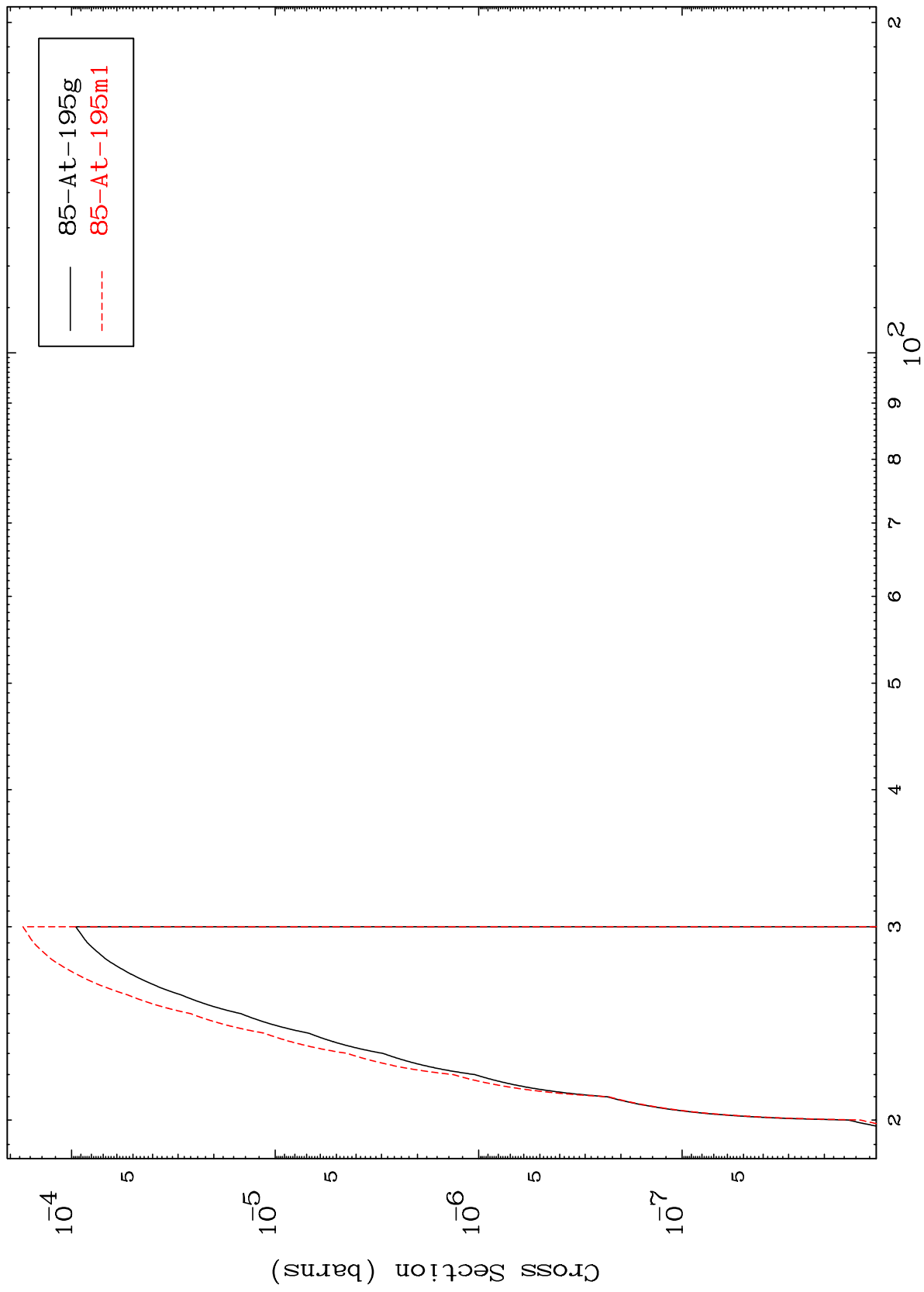


85-At-197g
85-At-197m1

MAT 8508

85-At-197m

(n,3n)
Radionuclide Production Cross Section



20

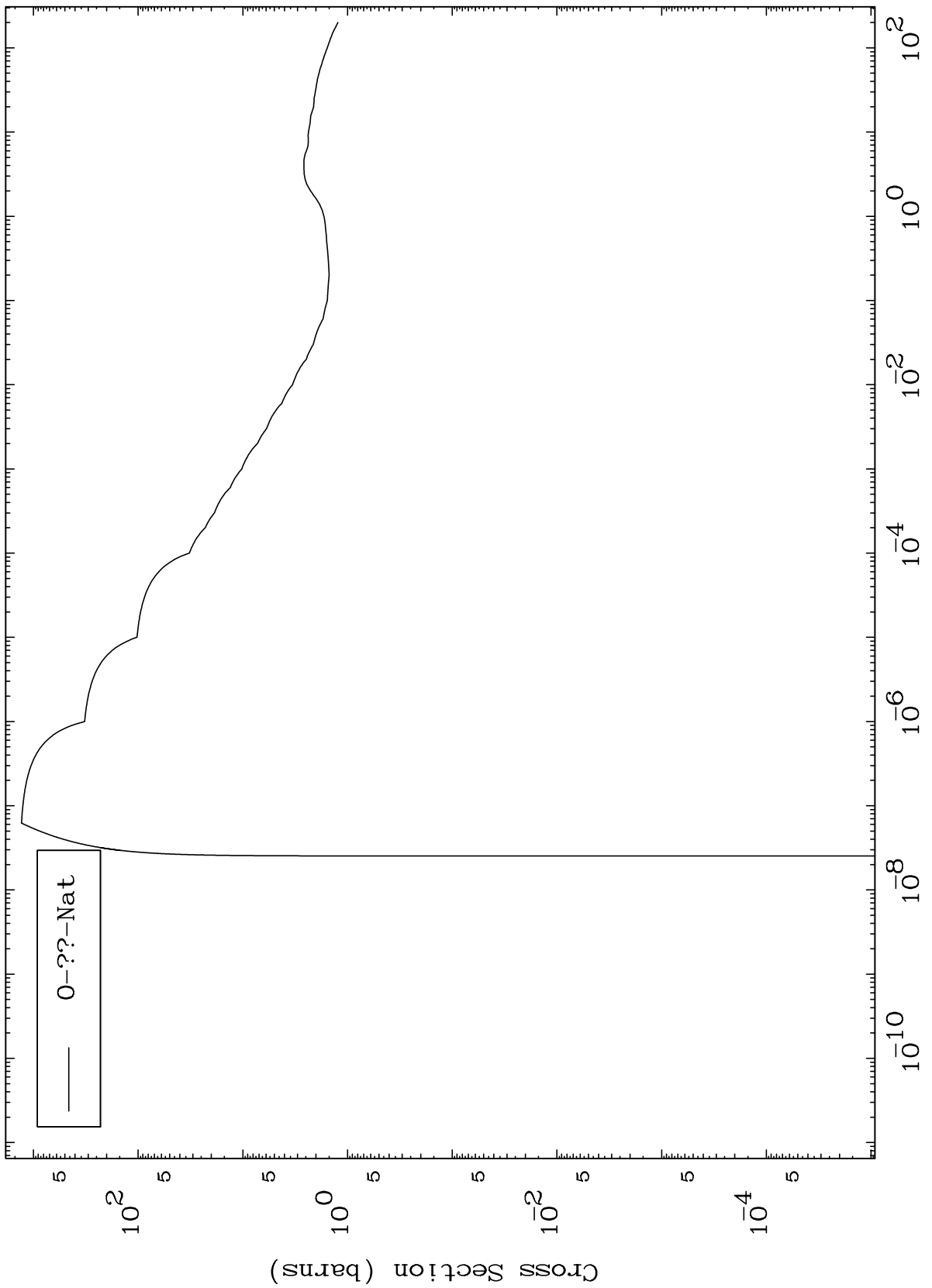
Incident Energy (MeV)

85-At-197m

MAT 8508

85-At-197m

Fission
Radionuclide Production Cross Section

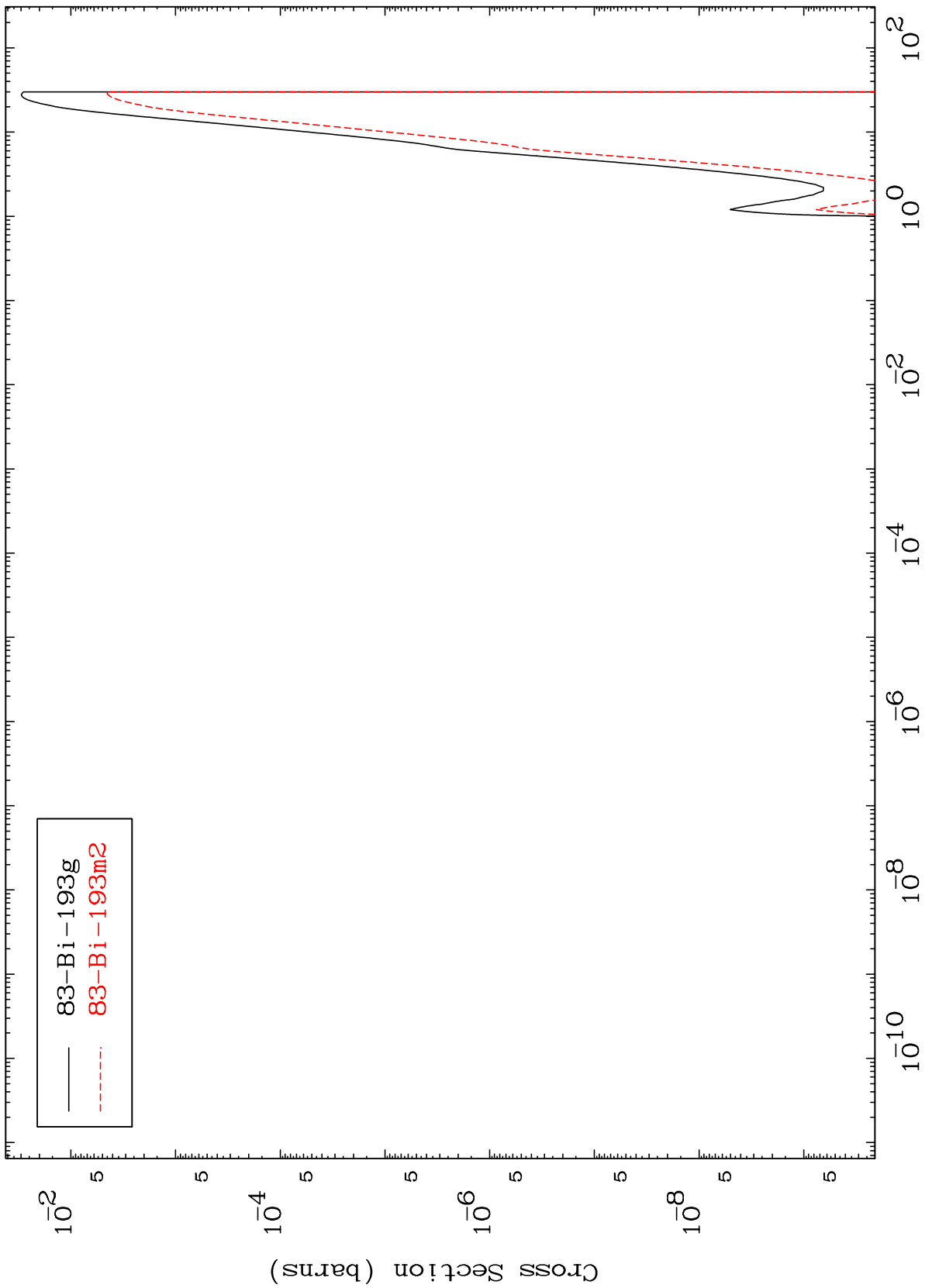


MAT 8508

$(n, n') \alpha$

85-At-197m

Radionuclide Production Cross Section

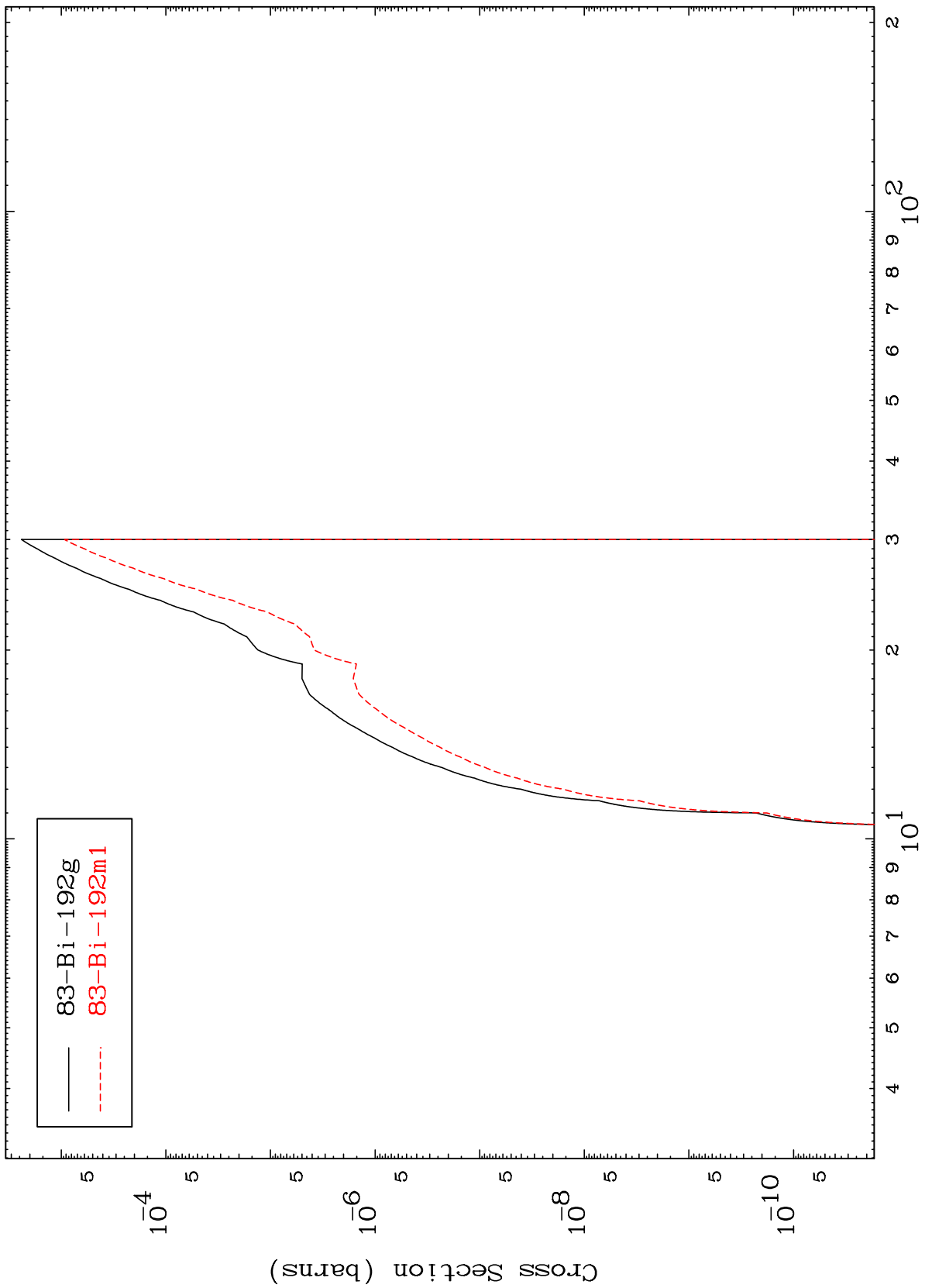


MAT 8508

(n,2n) α

85-At-197m

Radionuclide Production Cross Section



83-Bi-192g
83-Bi-192m1

Incident Energy (MeV)

85-At-197m

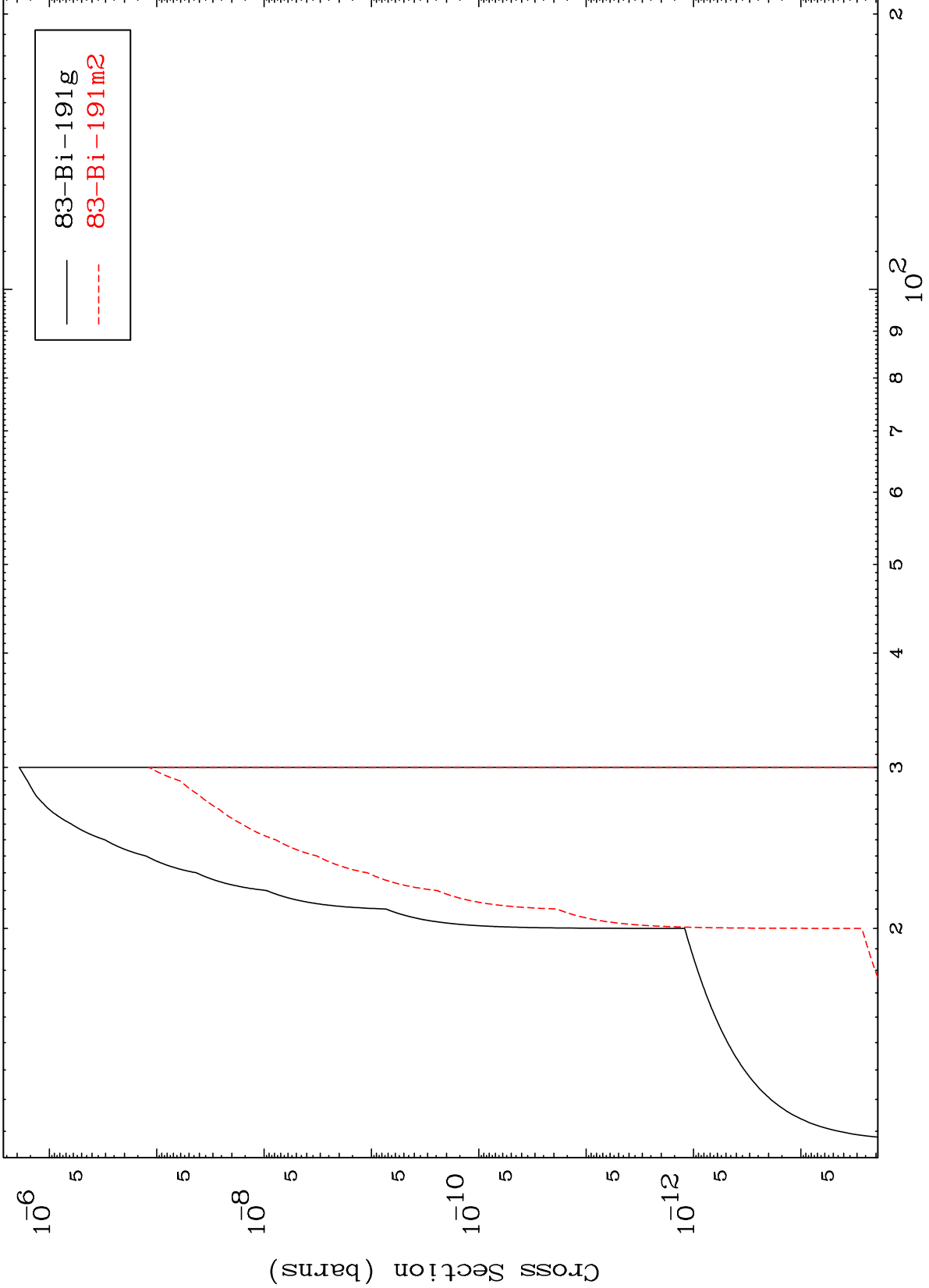
23

MAT 8508

(n,3n) α

85-At-197m

Radionuclide Production Cross Section



24

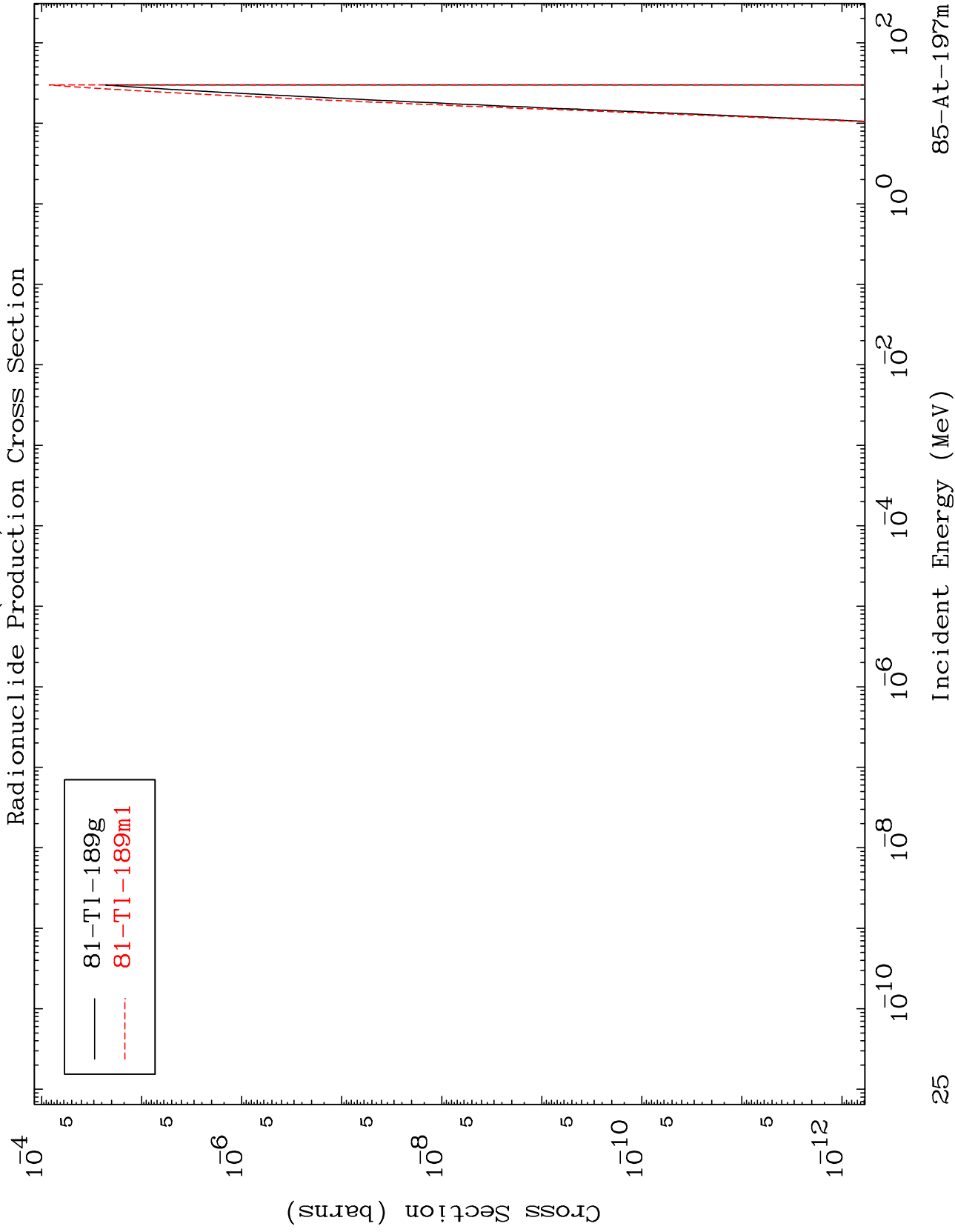
Incident Energy (MeV)

85-At-197m

MAT 8508

(n,n') 2 α

85-At-197m

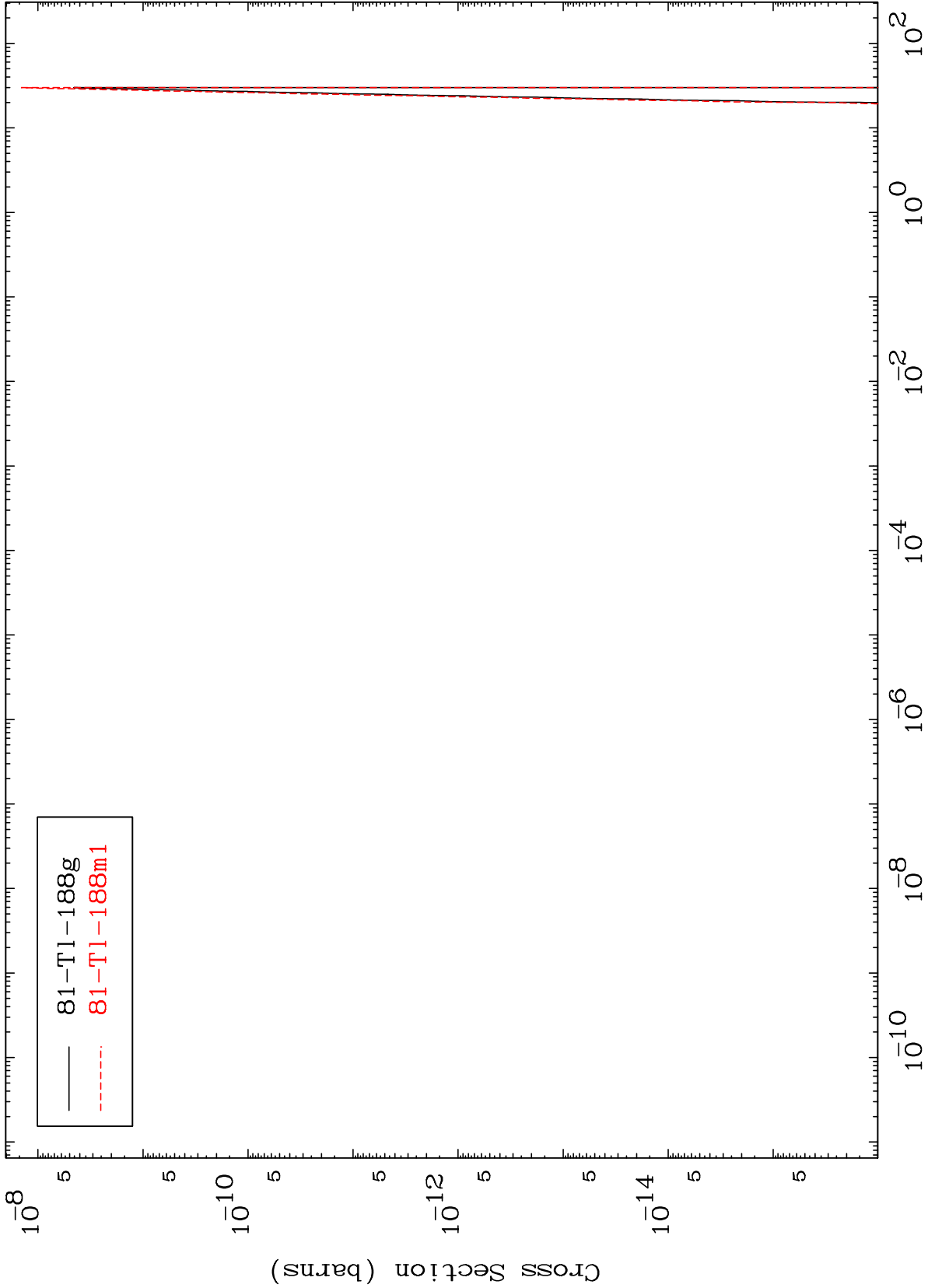


MAT 8508

(n,2n) 2α

85-At-197m

Radionuclide Production Cross Section



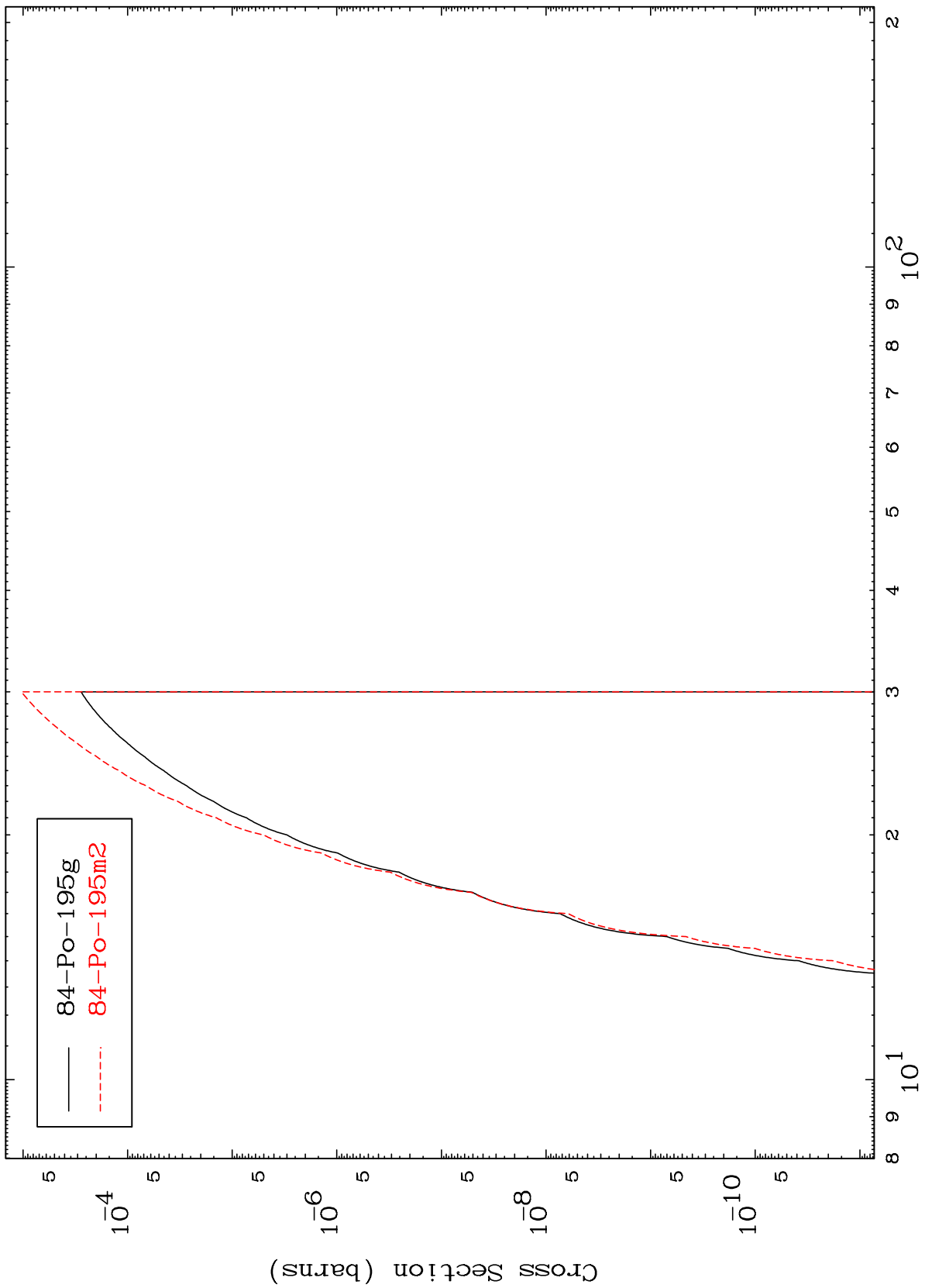
81-Tl-188g
81-Tl-188m1

MAT 8508

(n,n') d

85-At-197m

Radionuclide Production Cross Section



27

Incident Energy (MeV)

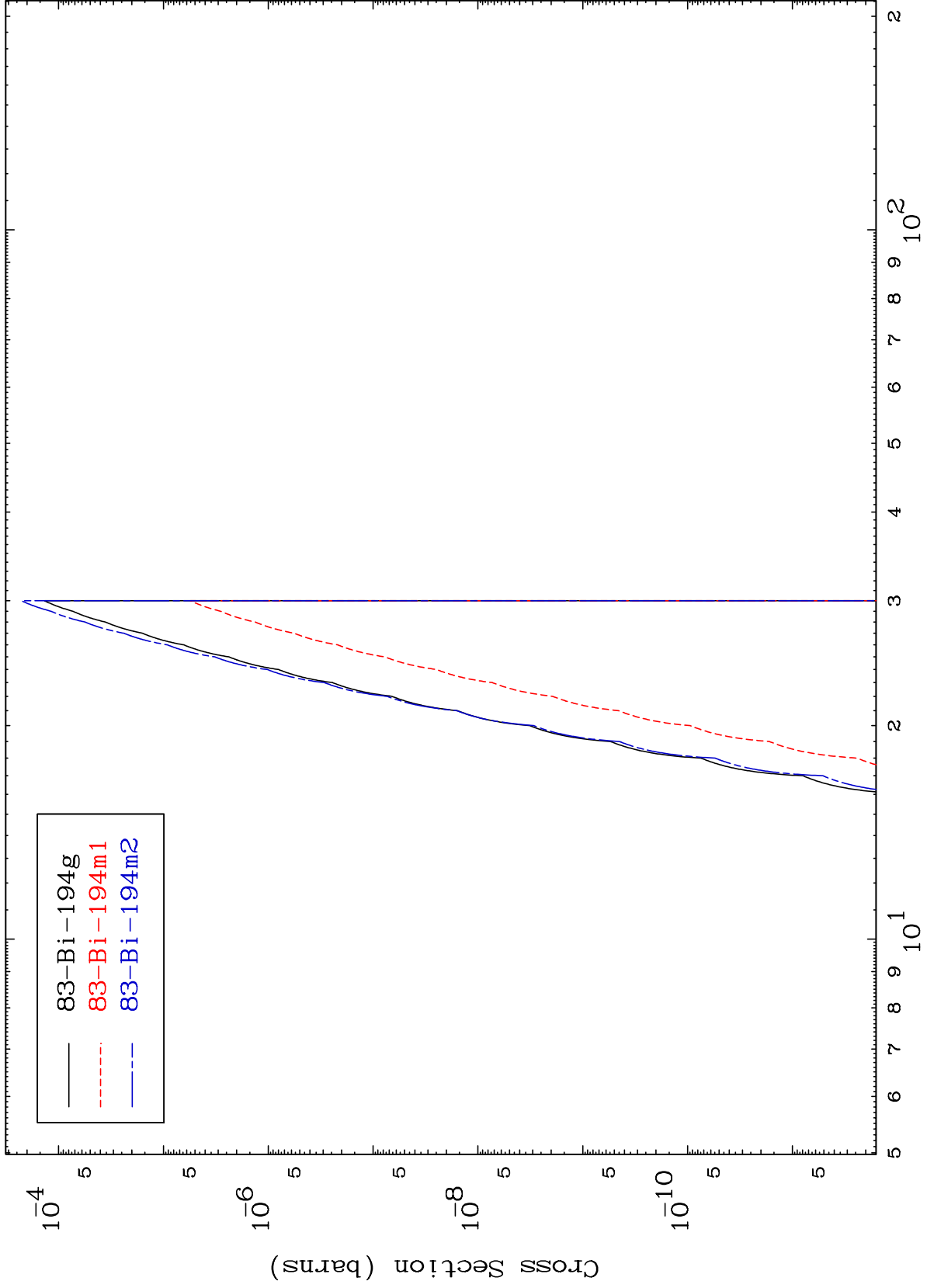
85-At-197m

MAT 8508

(n,n') He-3

85-At-197m

Radionuclide Production Cross Section



28

Incident Energy (MeV)

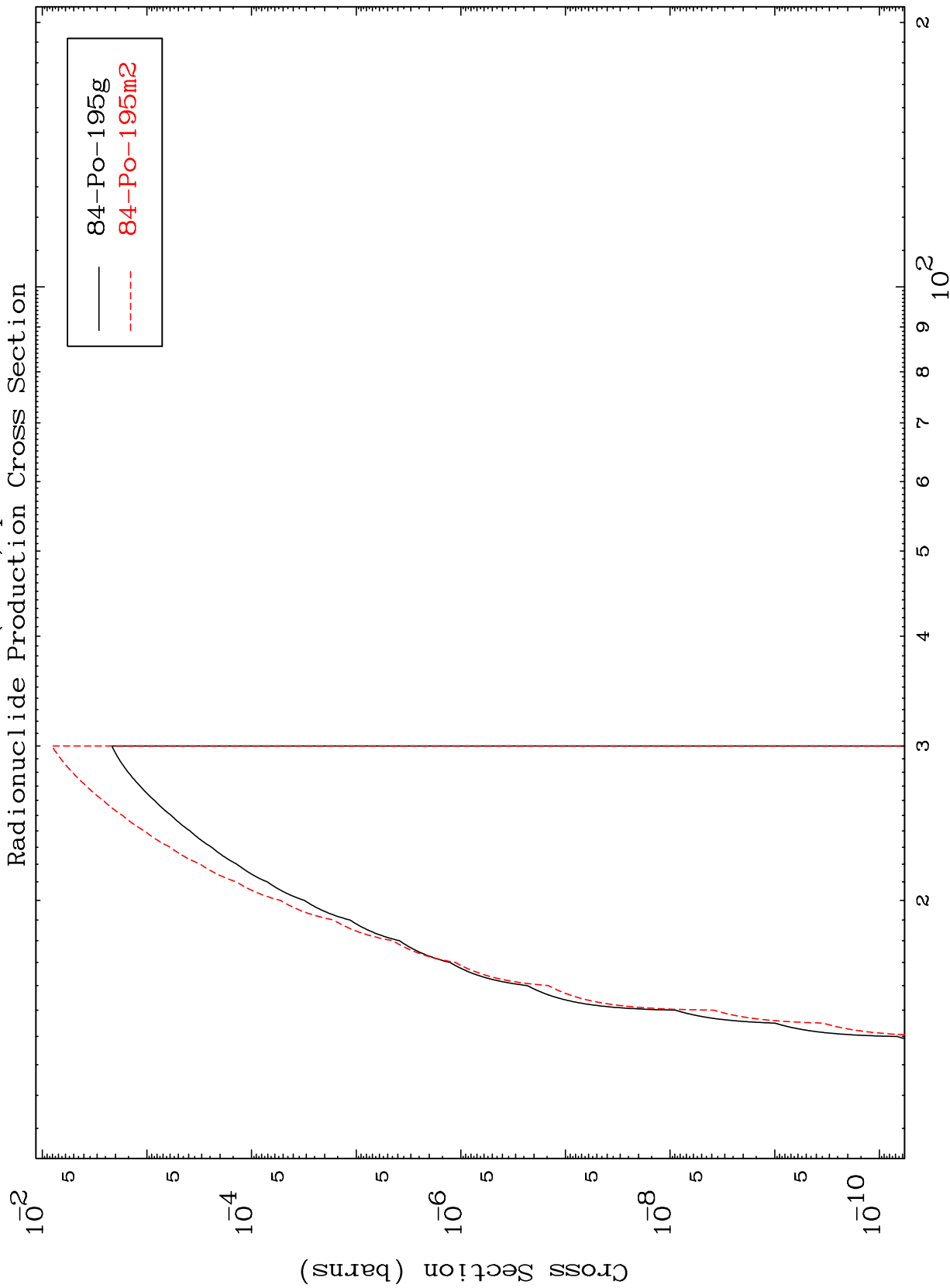
85-At-197m

MAT 8508

85-At-197m

(n,2n) p

Radionuclide Production Cross Section



Incident Energy (MeV)

85-At-197m

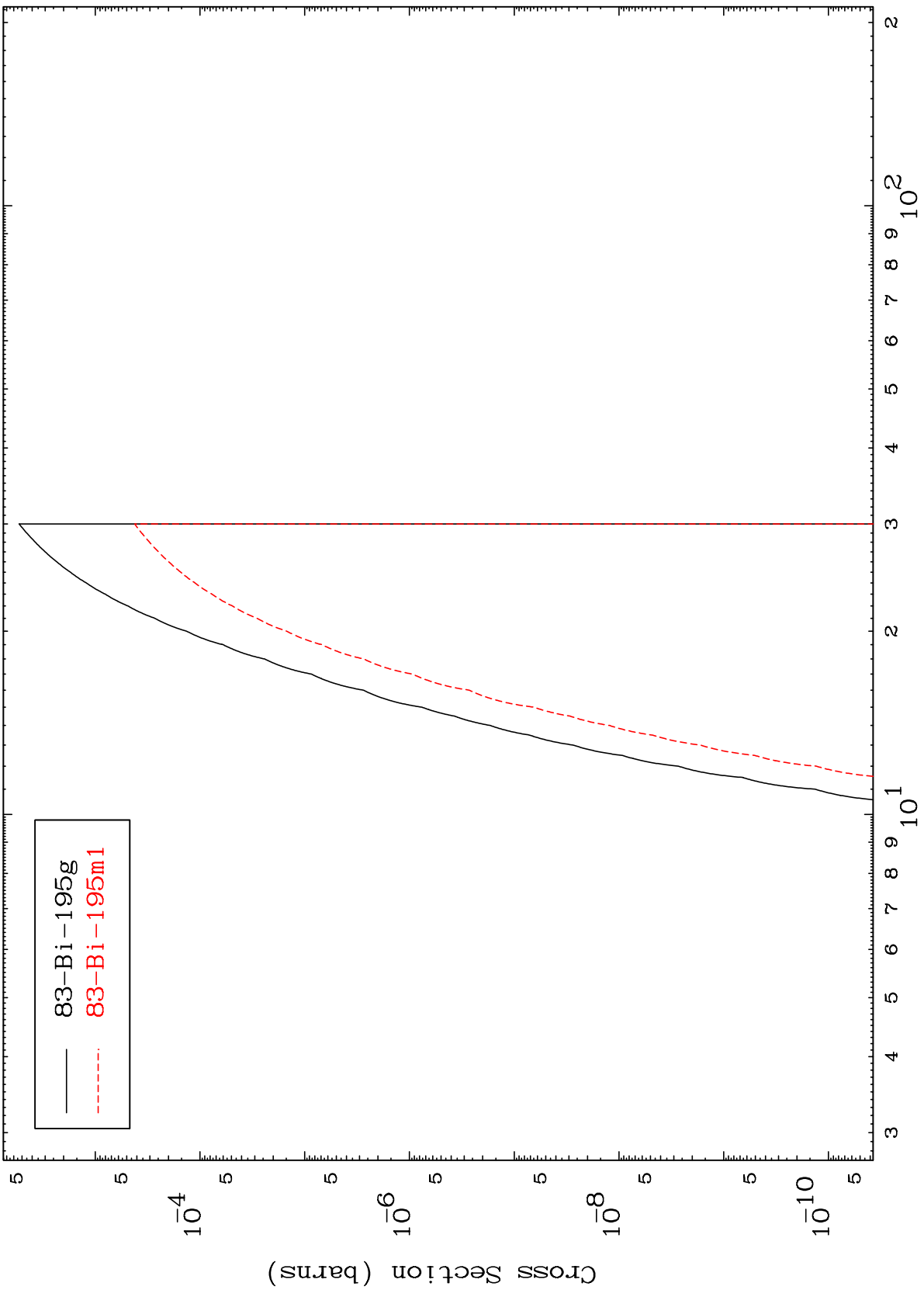
29

MAT 8508

(n,2n) p

85-At-197m

Radionuclide Production Cross Section



83-Bi-195g
83-Bi-195m1

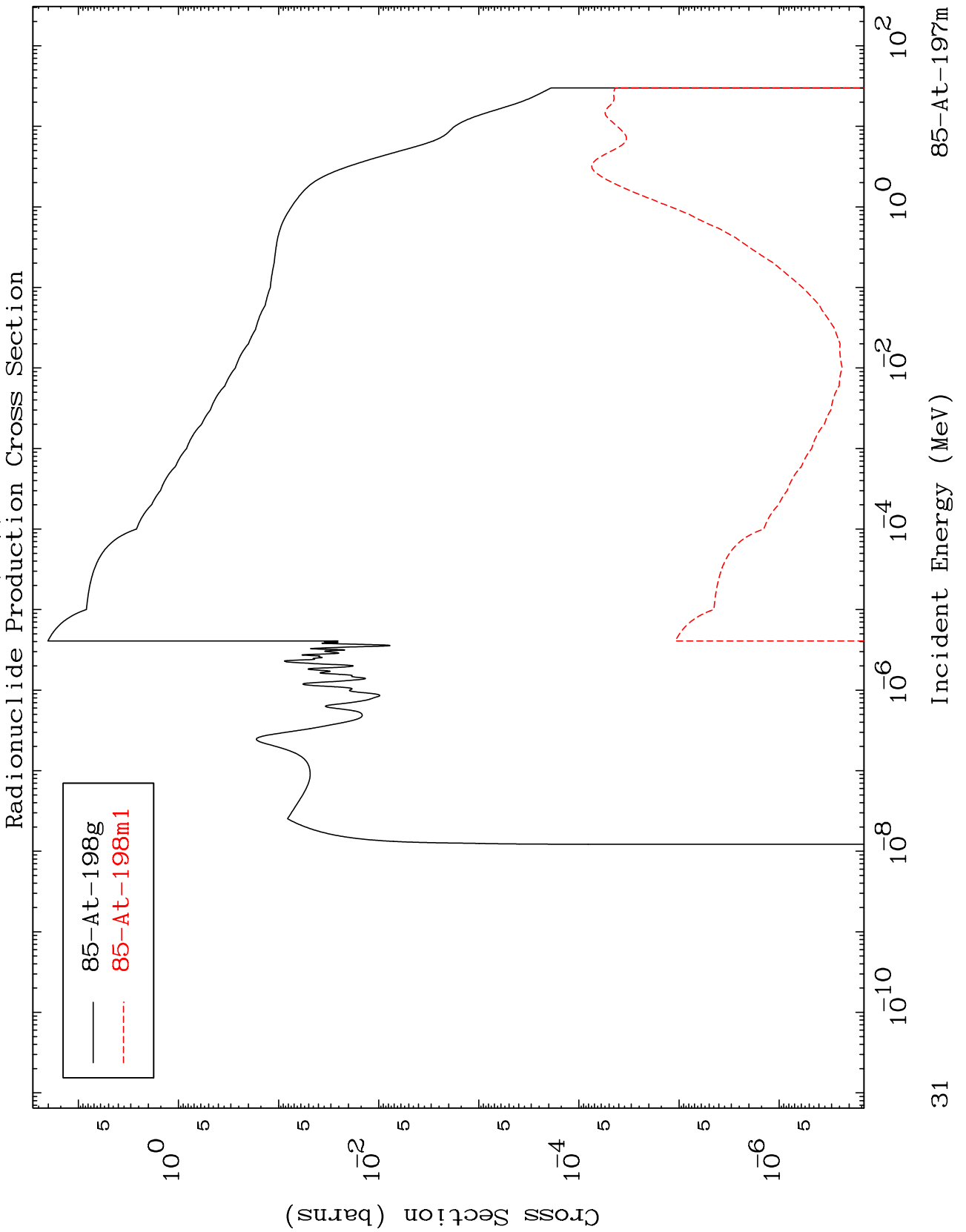
Incident Energy (MeV)

85-At-197m

30

MAT 8508

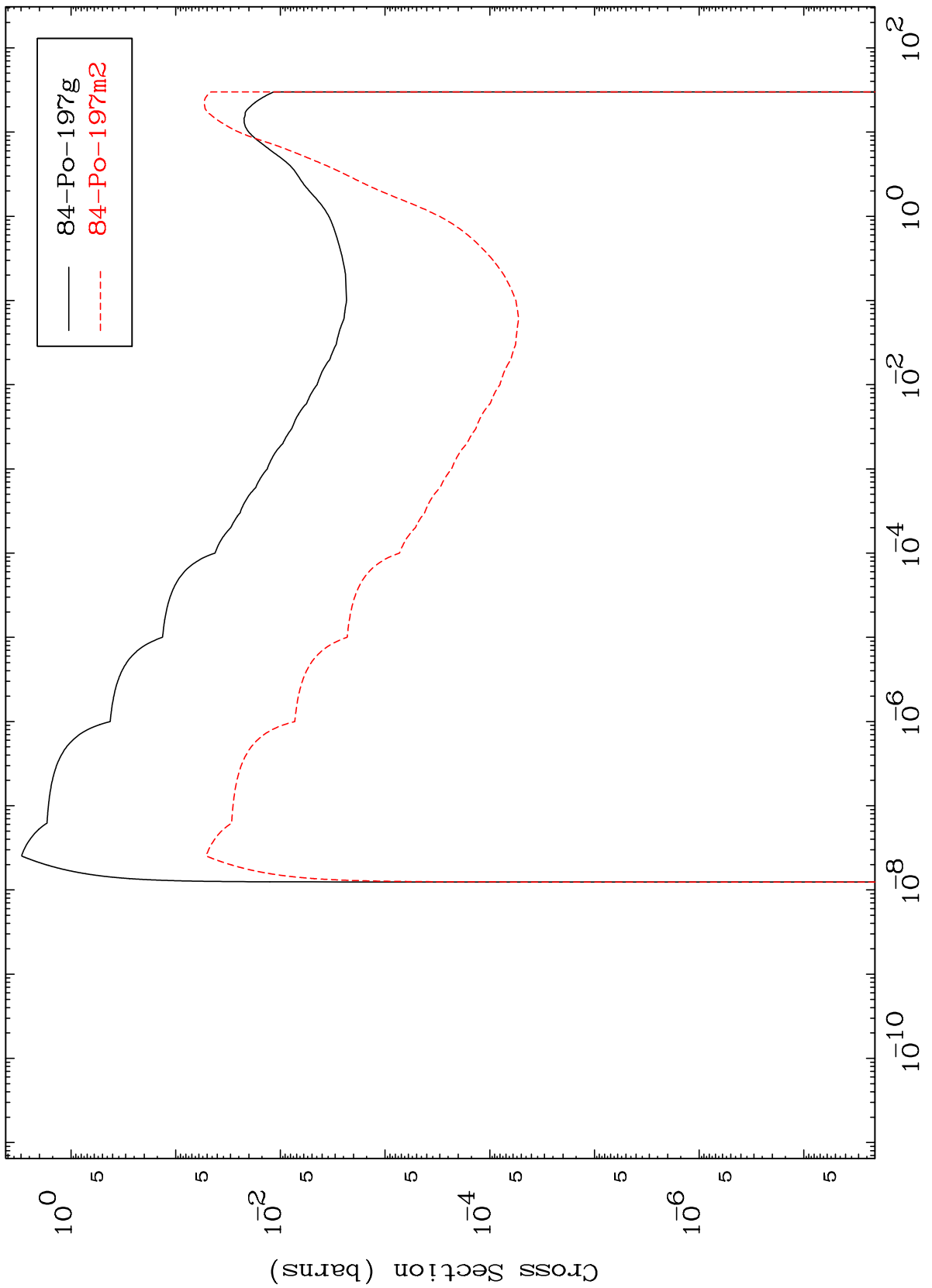
85-At-197m



MAT 8508

85-At-197m

(n,p)
Radionuclide Production Cross Section



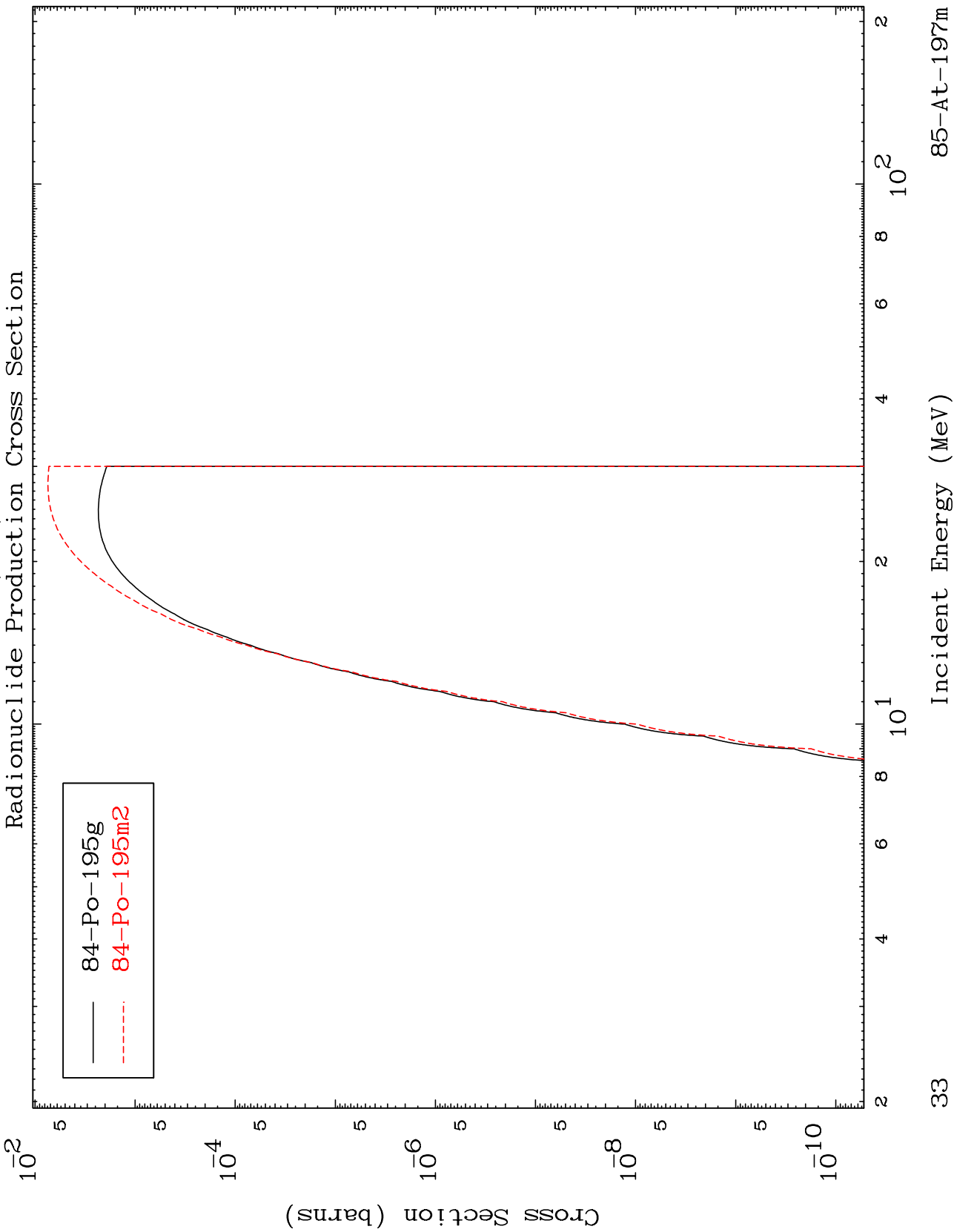
32

85-At-197m

Incident Energy (MeV)

MAT 8508

85-At-197m

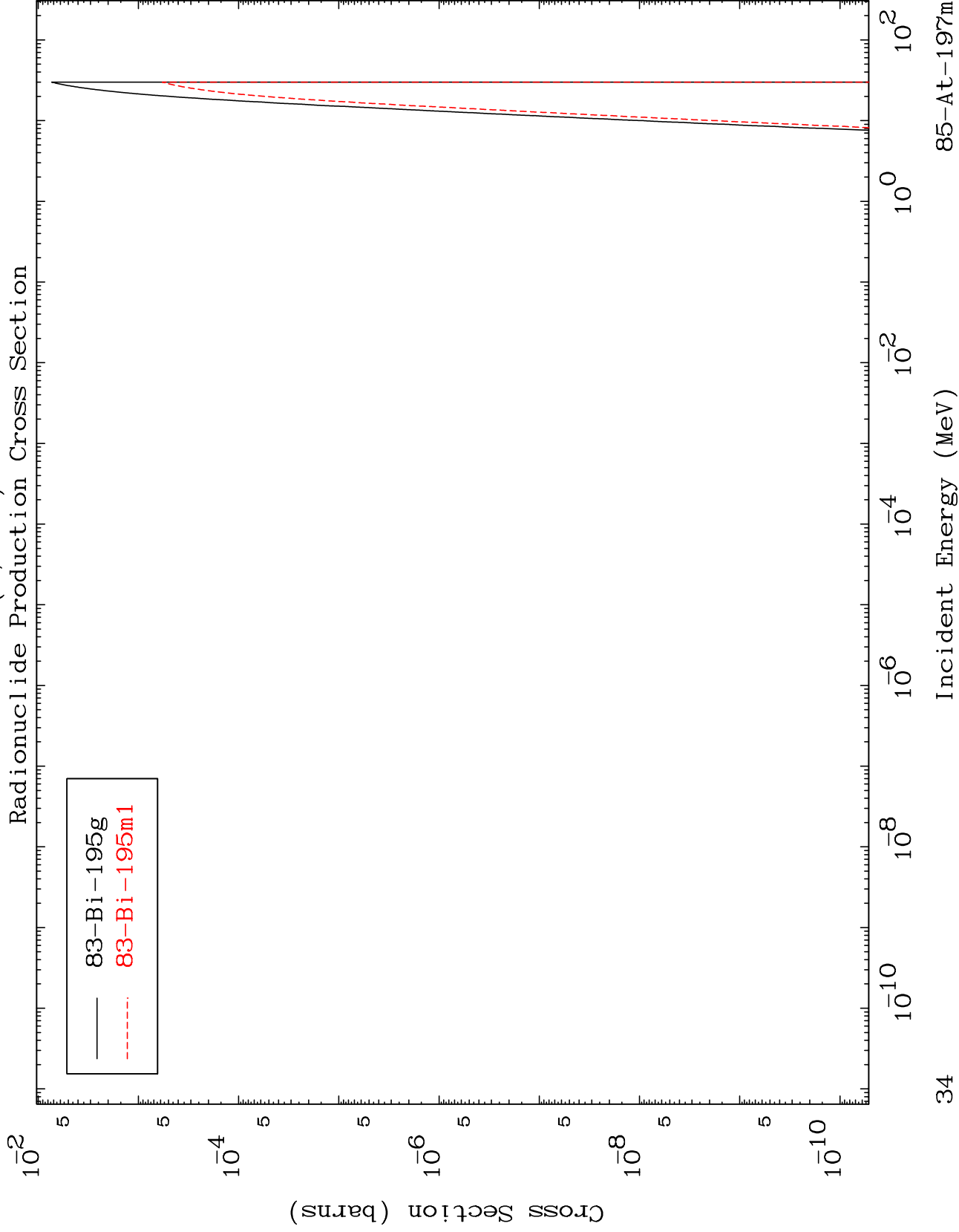


MAT 8508

(n,He-3)

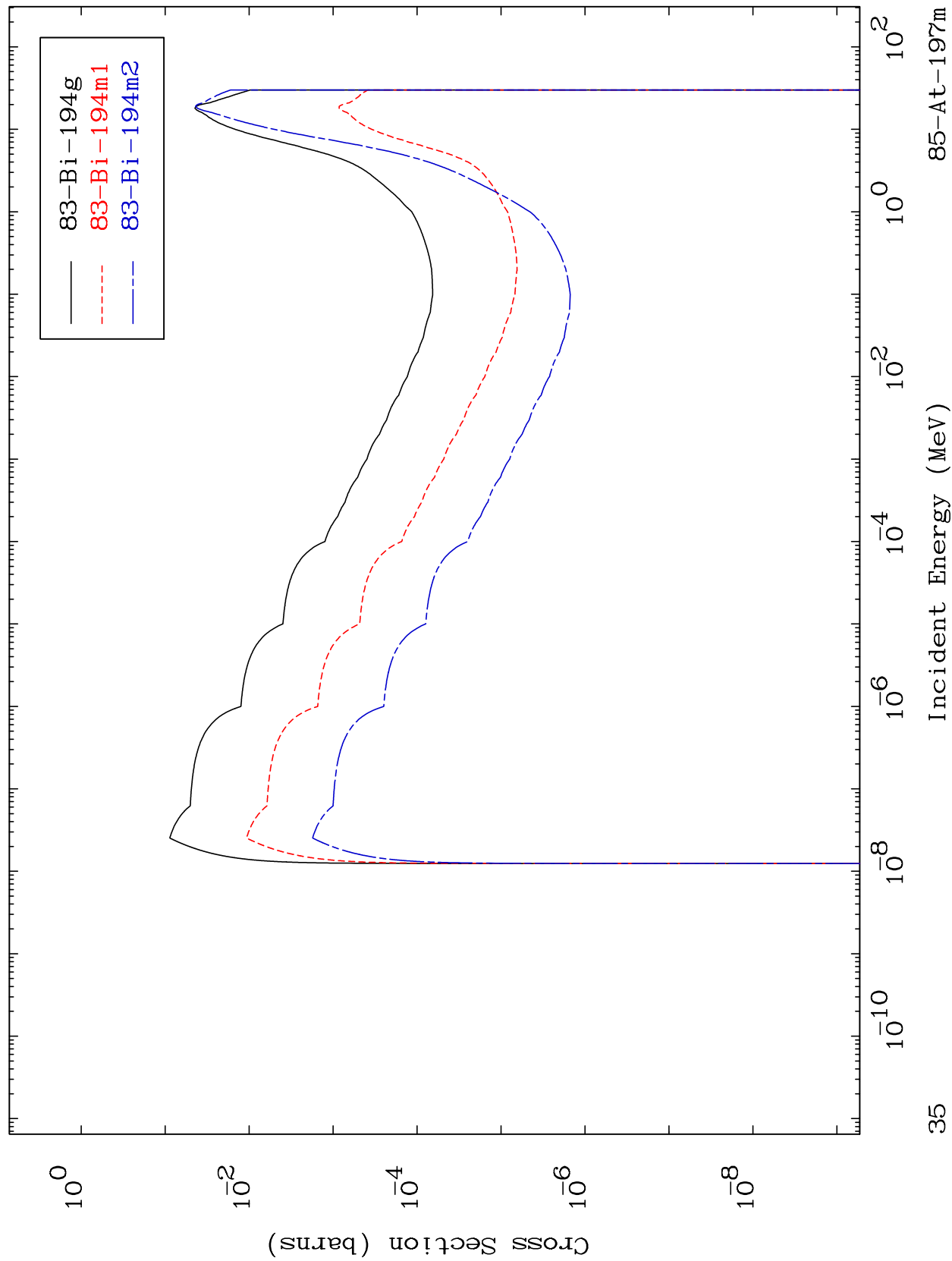
85-At-197m

Radionuclide Production Cross Section



83-Bi-195g
83-Bi-195m1

Radionuclide Production Cross Section
(n, α)

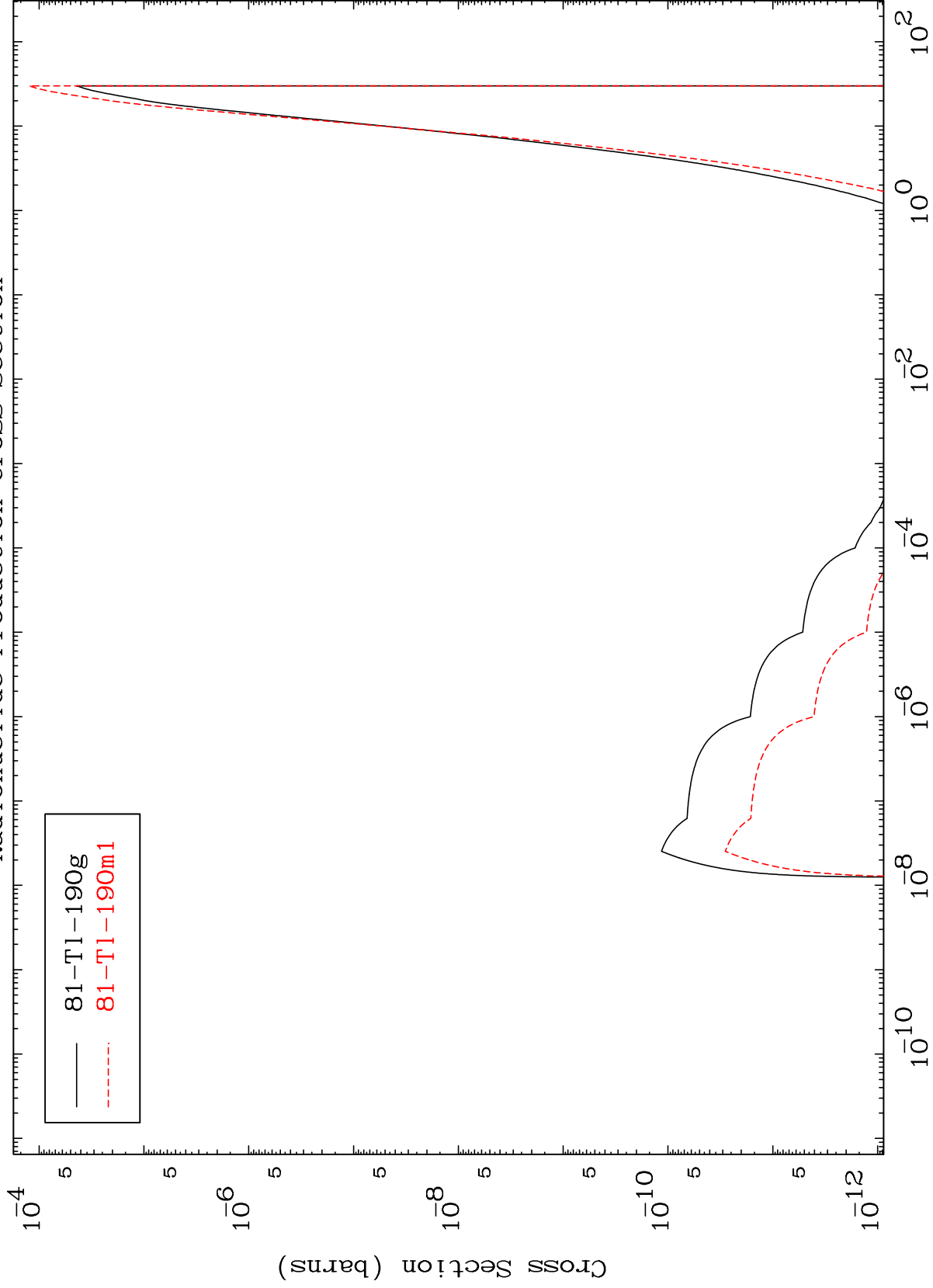


MAT 8508

(n,2α)

85-At-197m

Radionuclide Production Cross Section



36

Incident Energy (MeV)

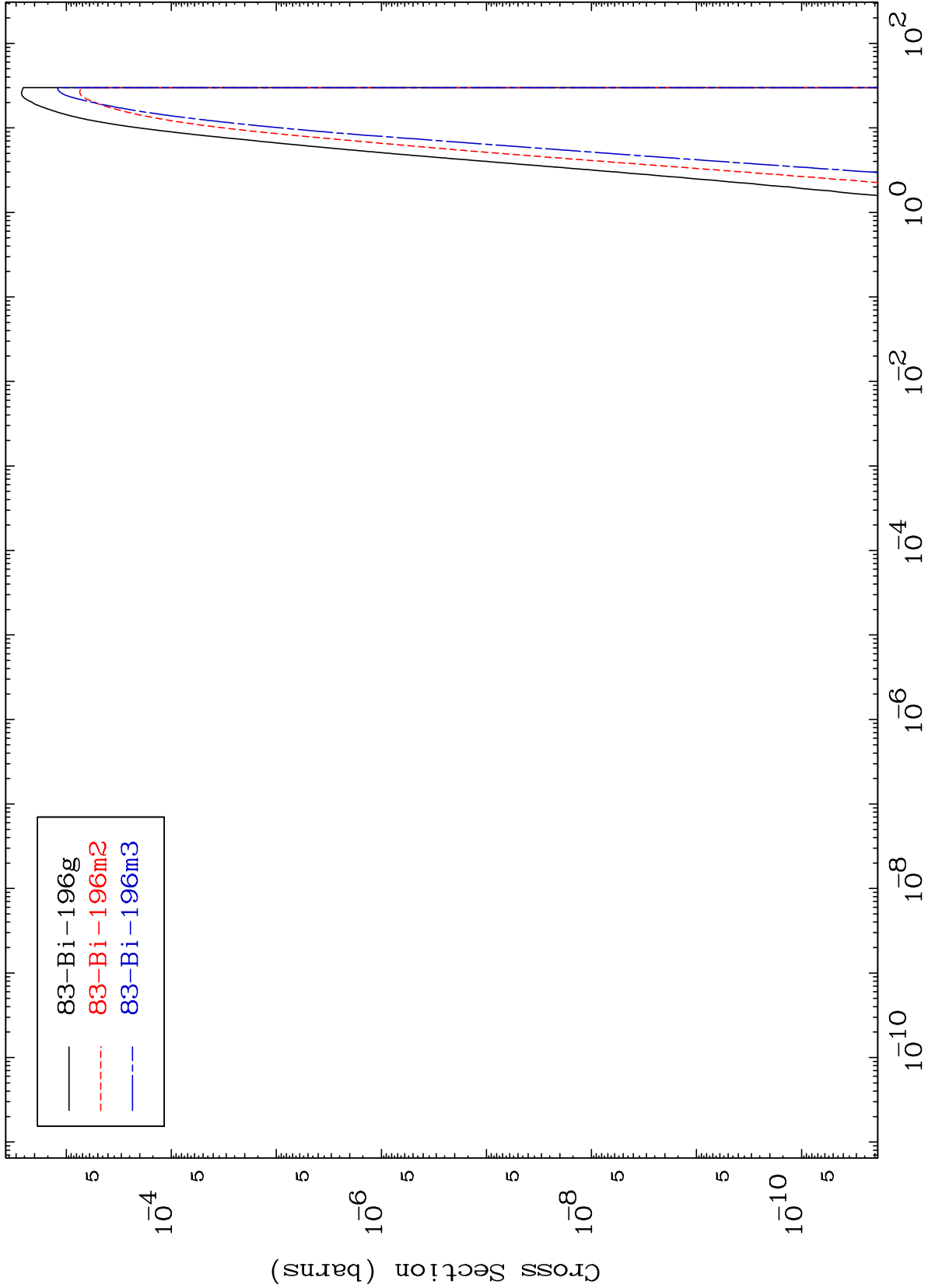
85-At-197m

MAT 8508

(n,2p)

85-At-197m

Radionuclide Production Cross Section

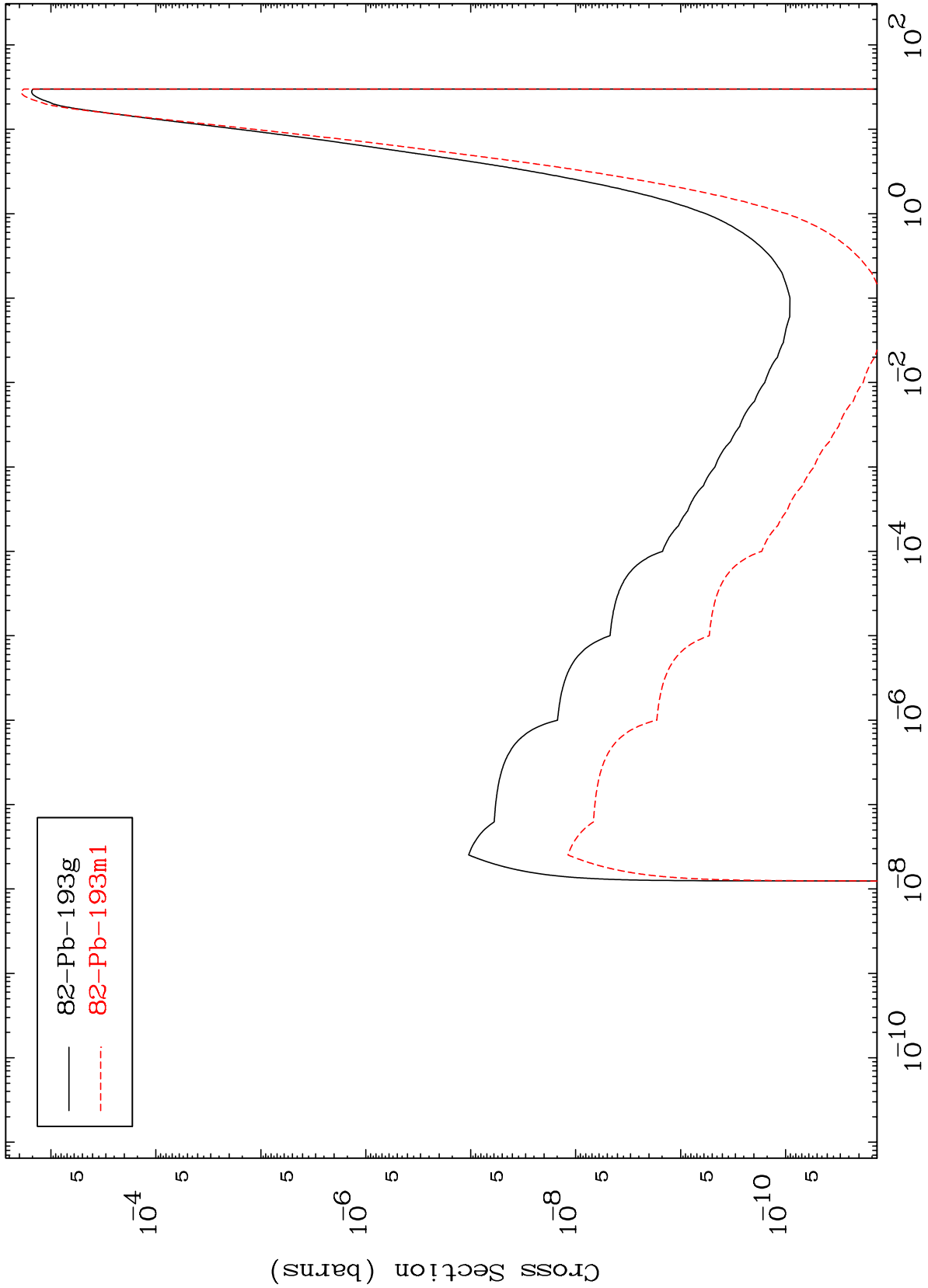
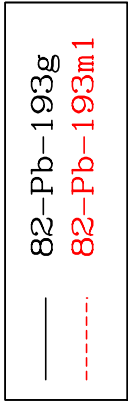


MAT 8508

(n,p) α

85-At-197m

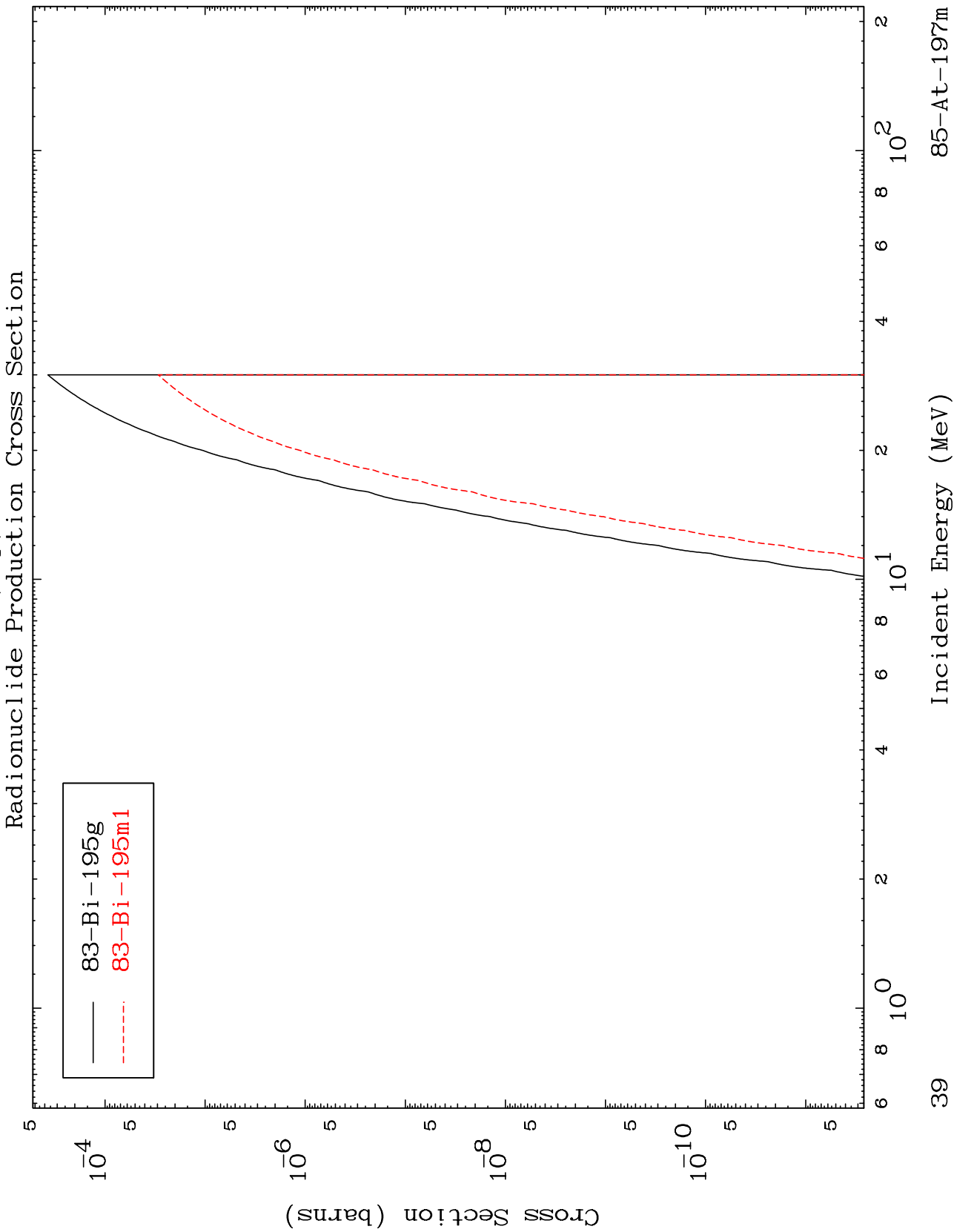
Radionuclide Production Cross Section



MAT 8508

(n,p) d

85-At-197m



39

85-At-197m

MAT 8508

(n,p) t

85-At-197m

