

Program EVALPLOT
(Version 2021-1)

by

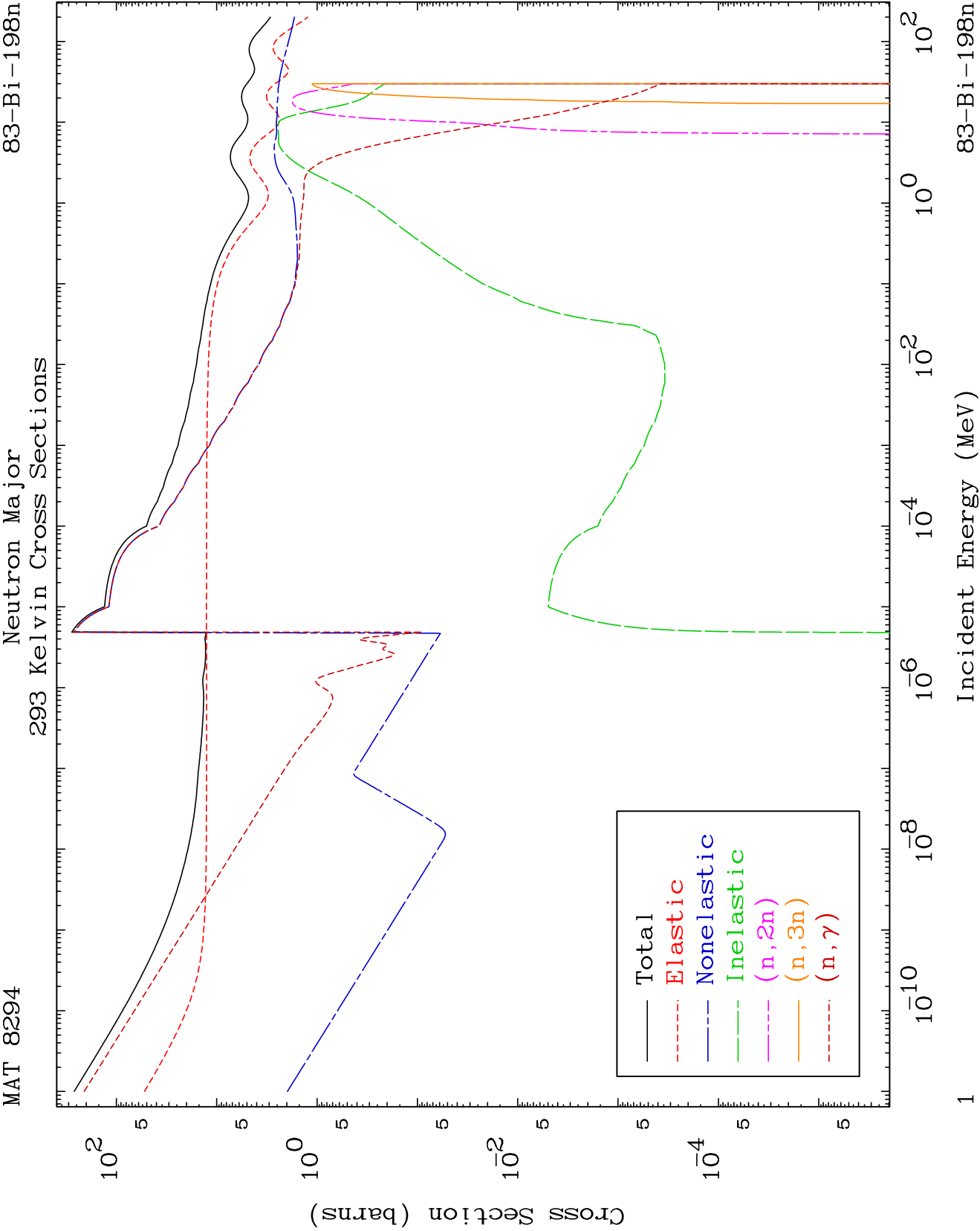
Dermott E. Cullen
(Present Contact Information)

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E.Mail:redcullen1@comcast.net
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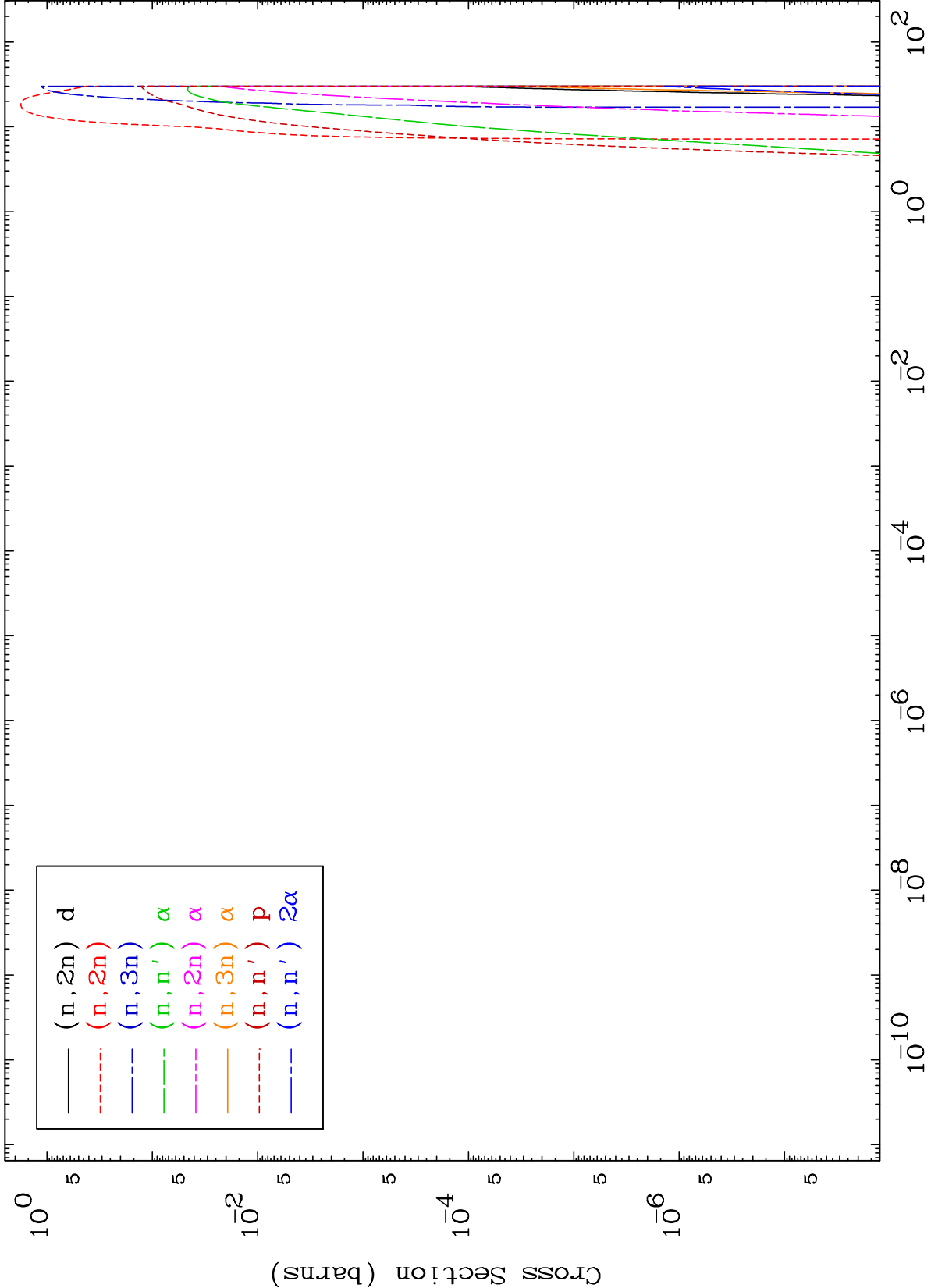
Press Mouse Button to Start

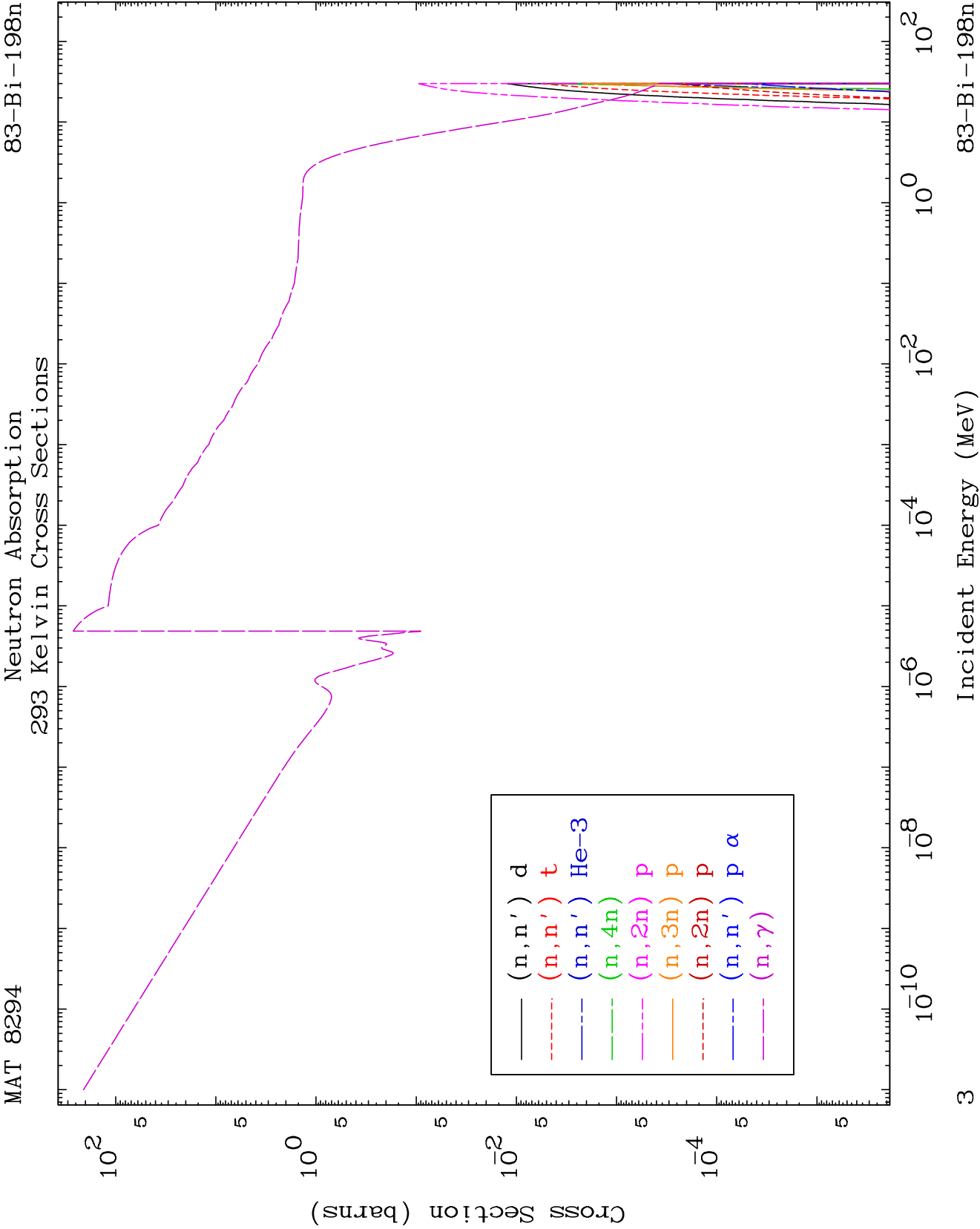


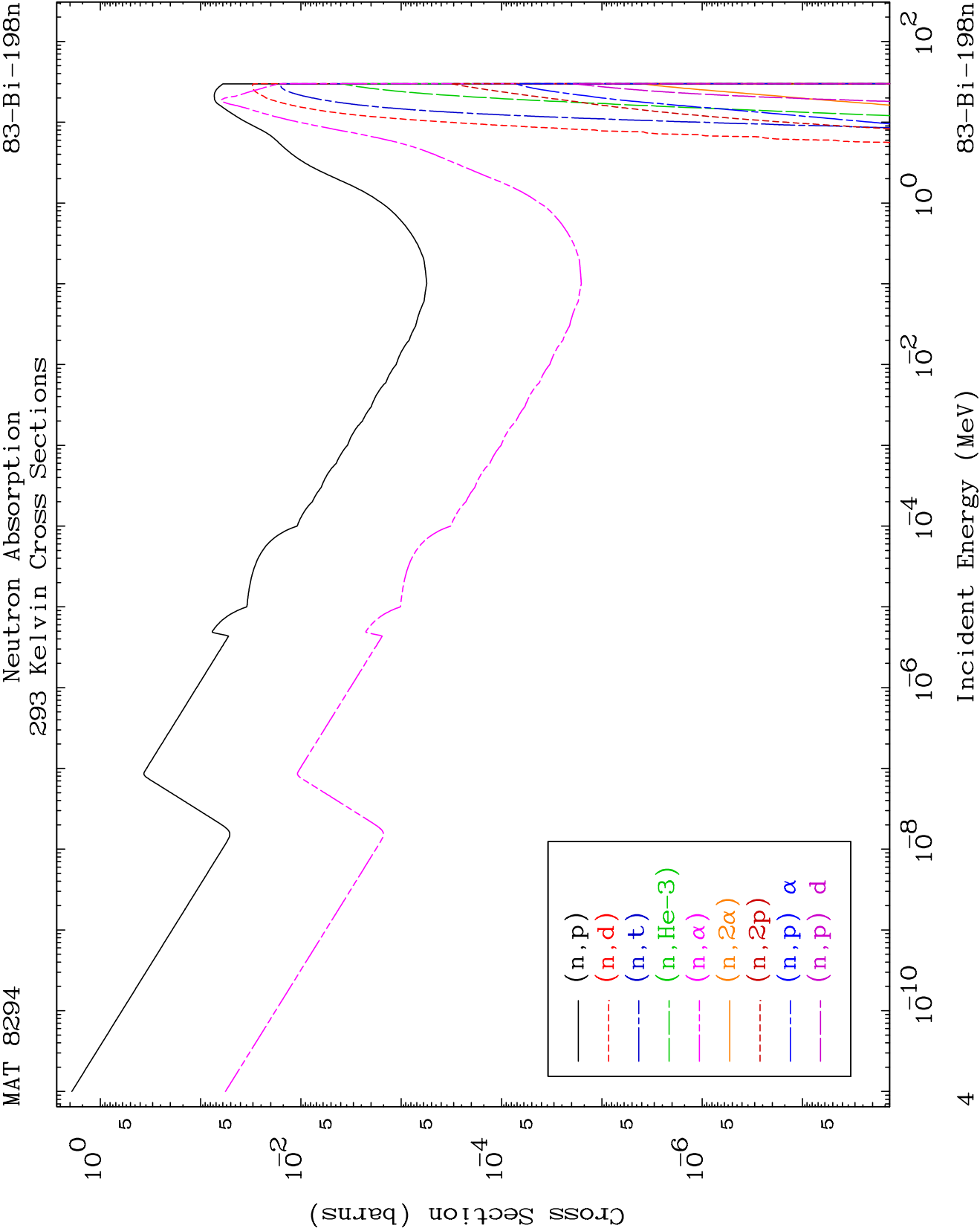
MAT 8294

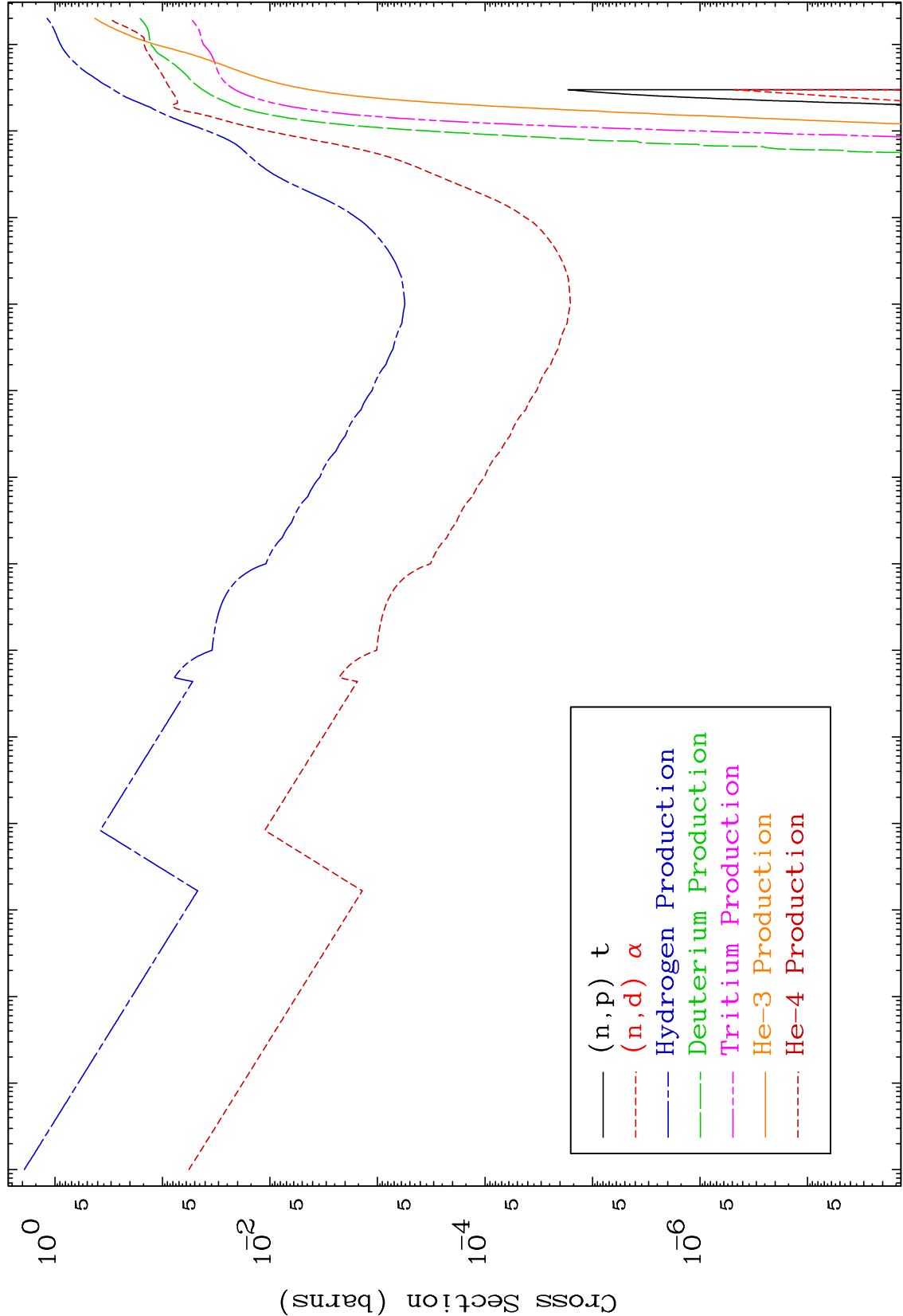
Neutron Absorption
293 Kelvin Cross Sections

83-Bi-198n





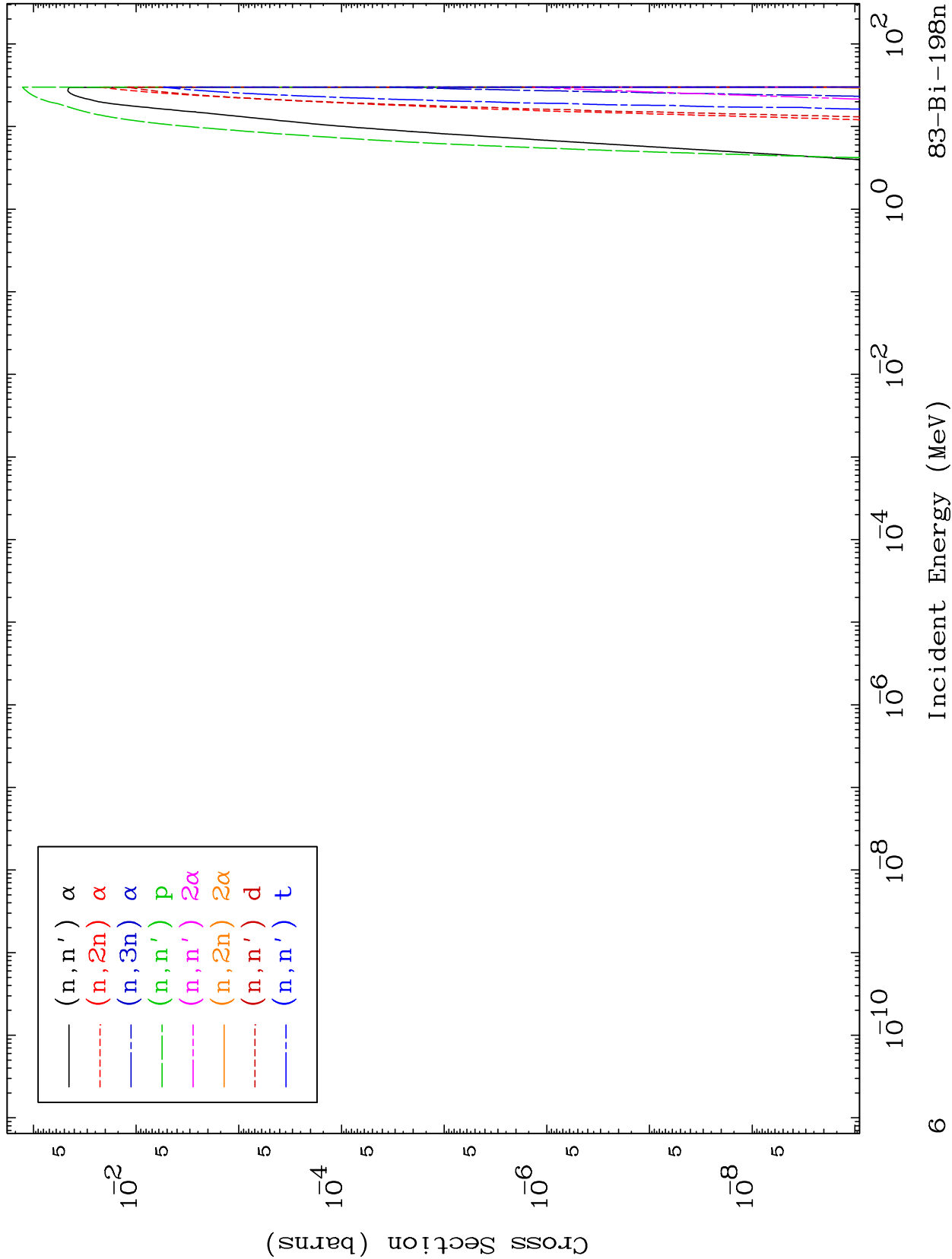


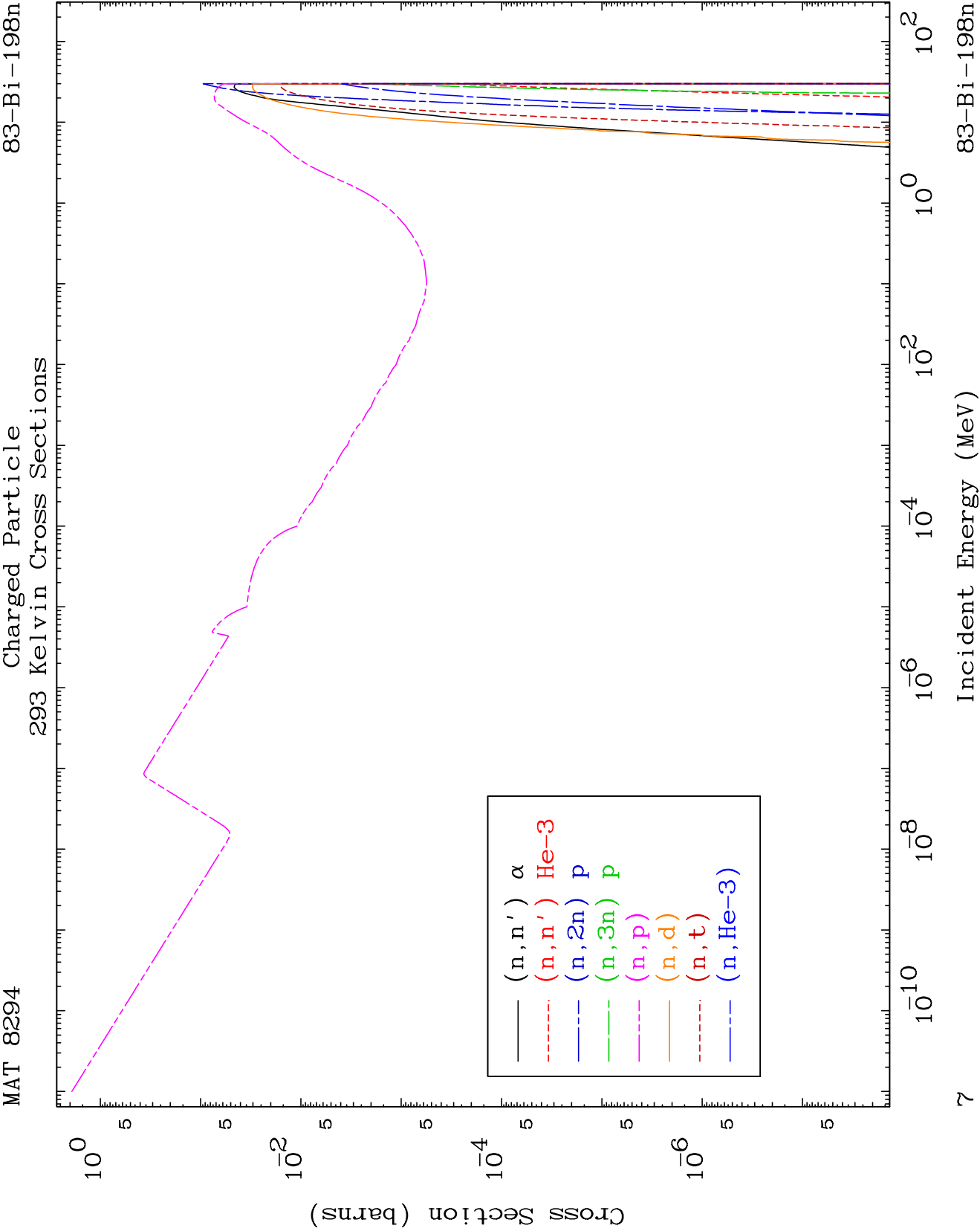


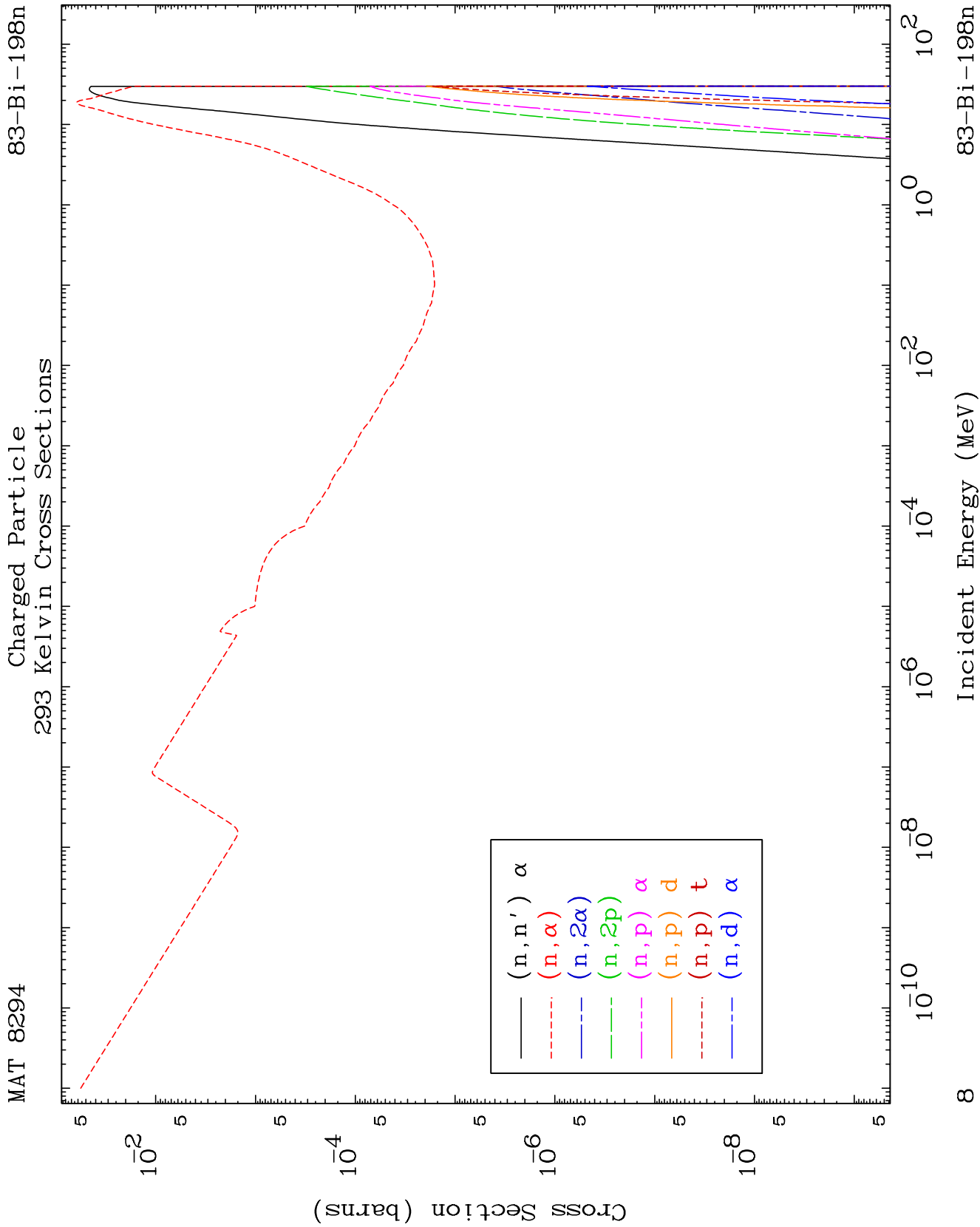
MAT 8294

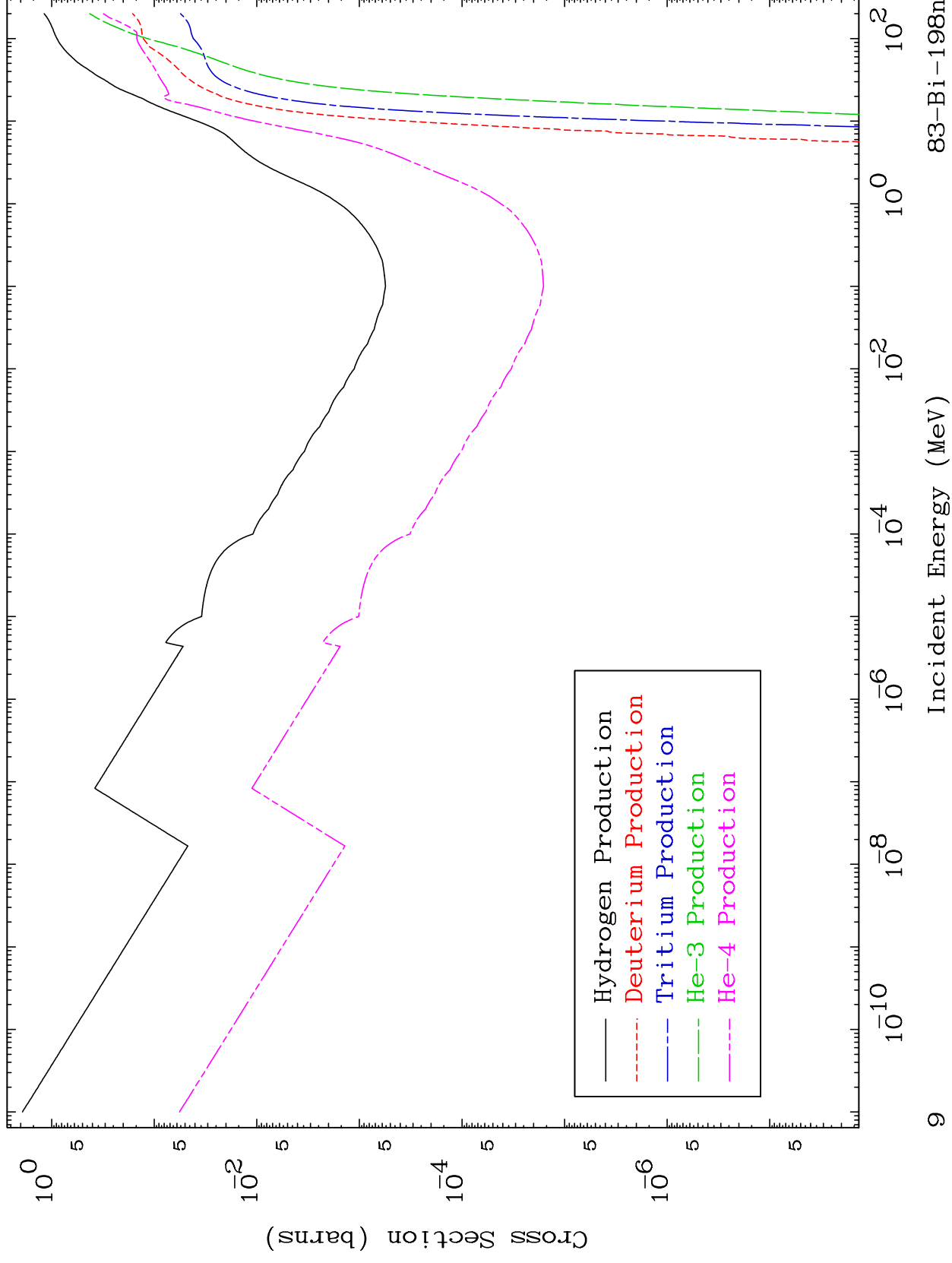
Charged Particle
293 Kelvin Cross Sections

83-Bi-198n





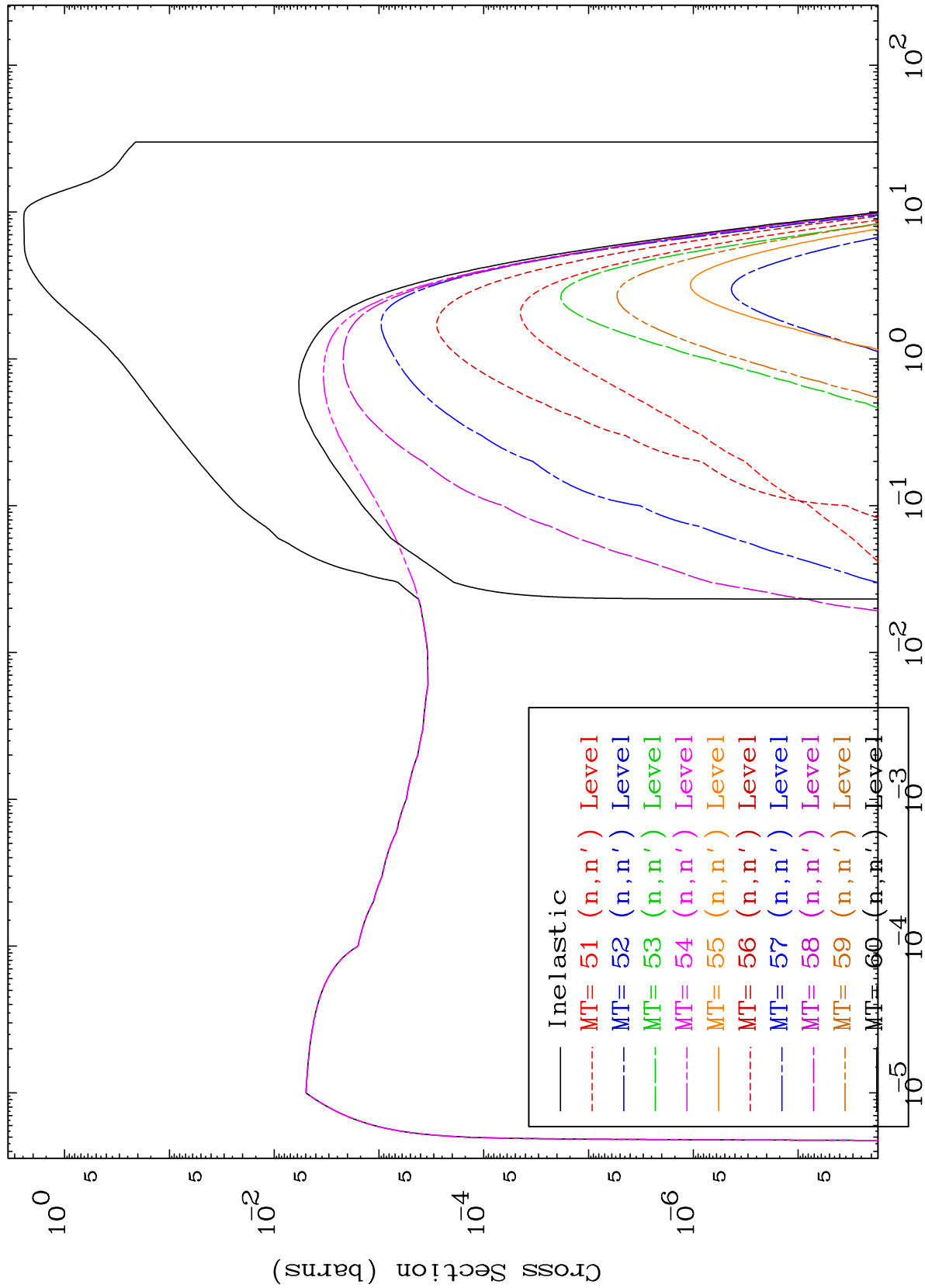




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83-Bi-198n

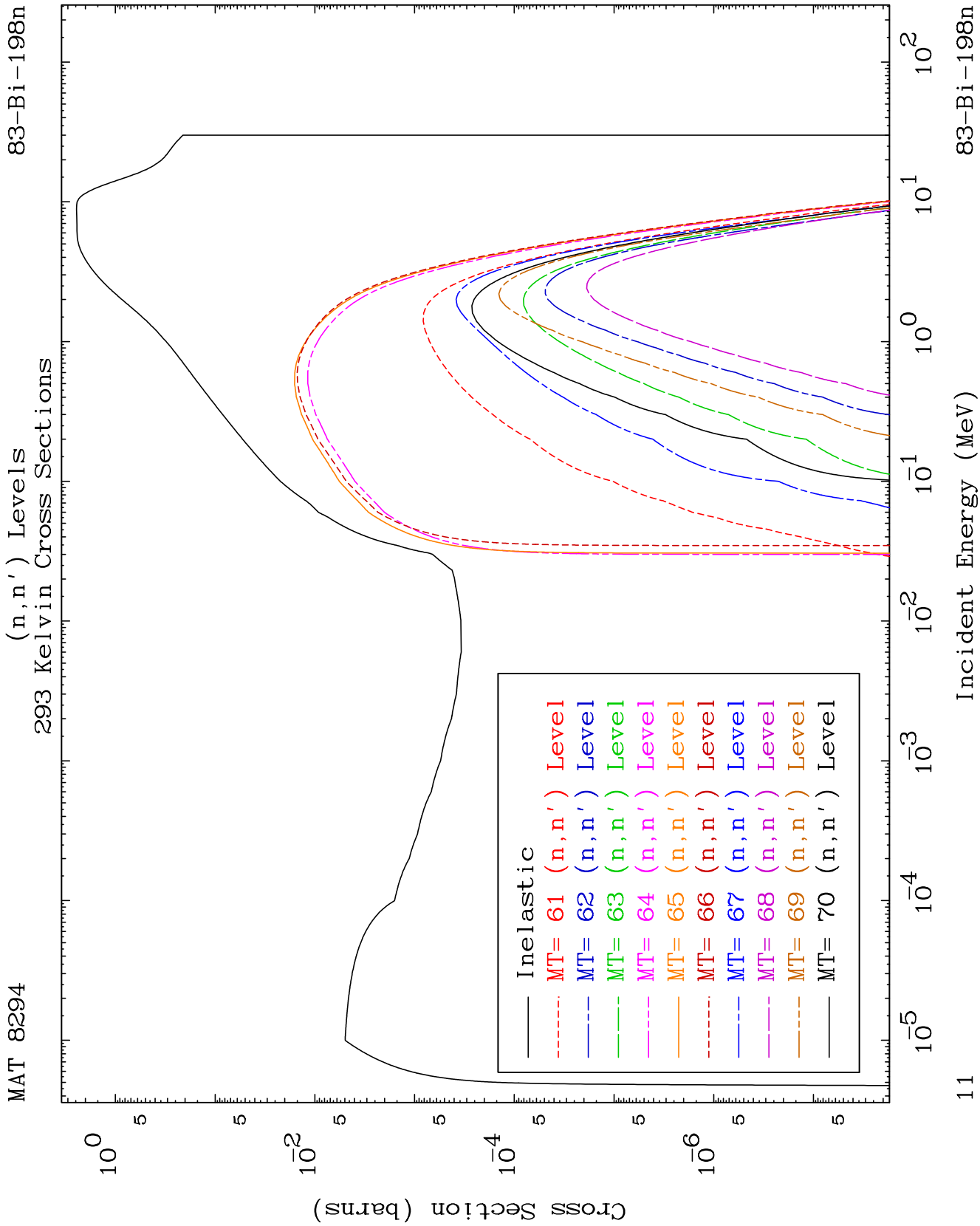
(n,n') Levels
293 Kelvin Cross Sections



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Incident Energy (MeV)

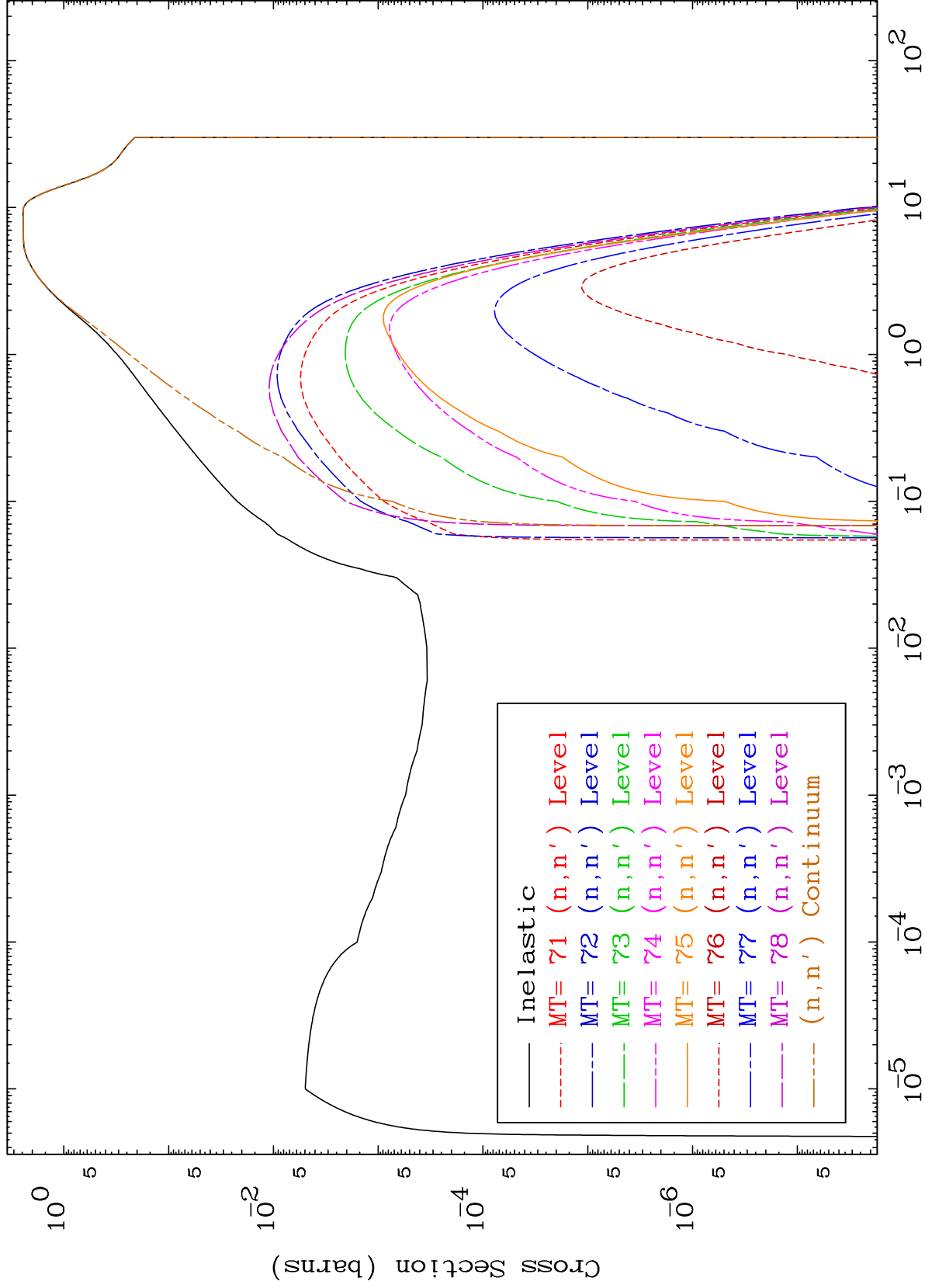
83-Bi-198n



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83-Bi-198n

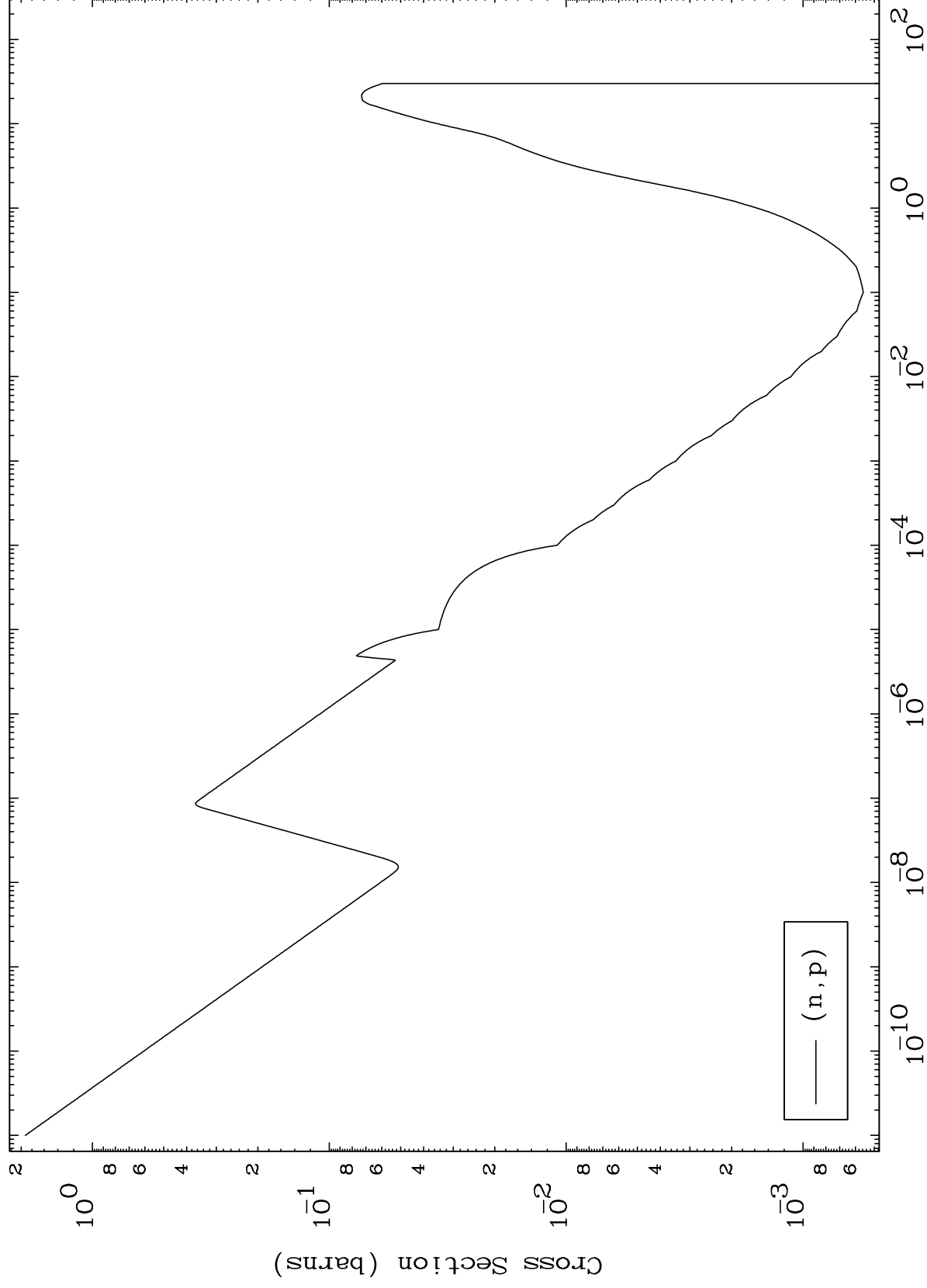
(n,n') Levels
293 Kelvin Cross Sections



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(n,p) Levels
293 Kelvin Cross Sections

83-Bi-198n



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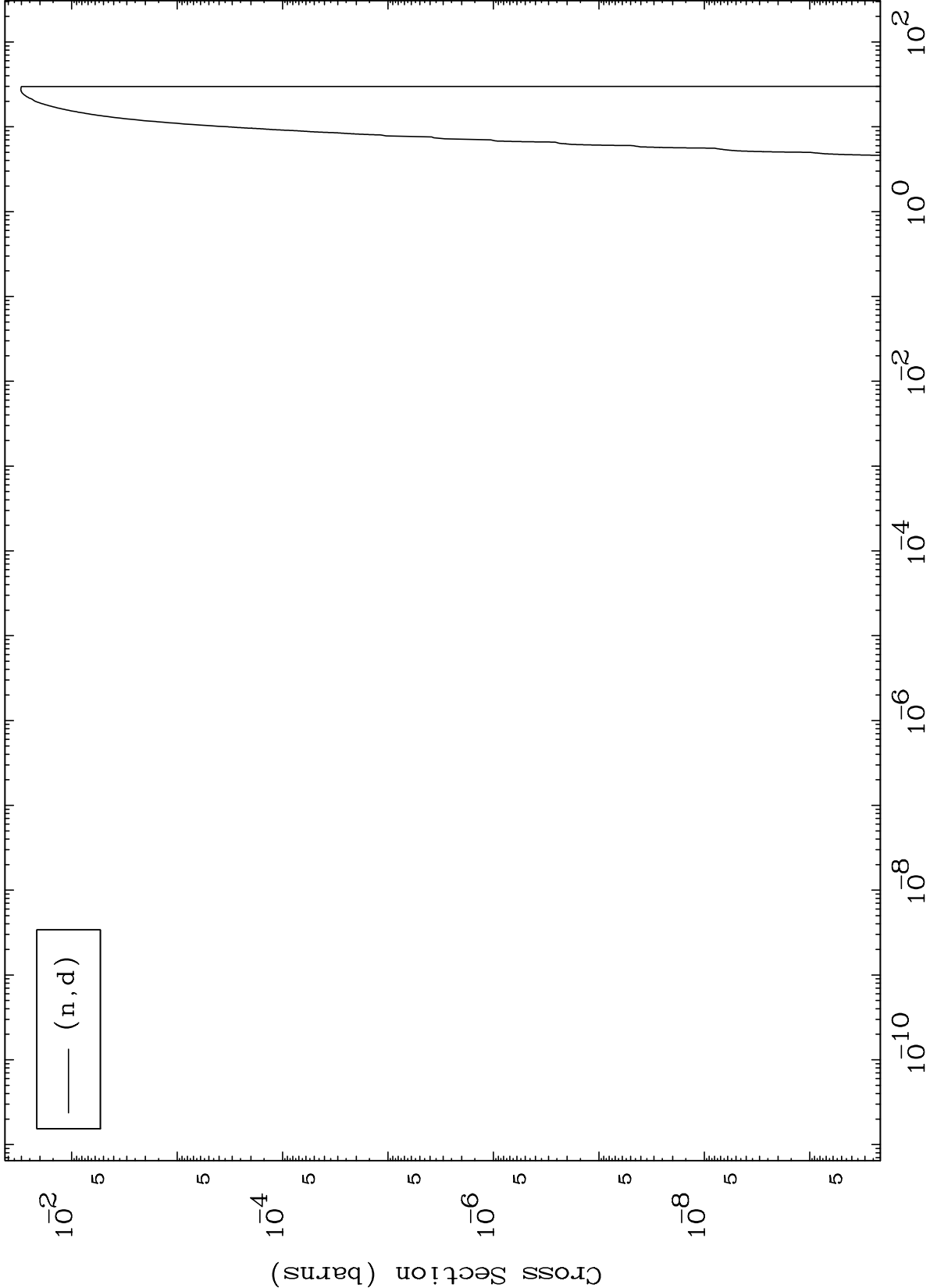
Incident Energy (MeV)

83-Bi-198n

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(n,d) Levels
293 Kelvin Cross Sections

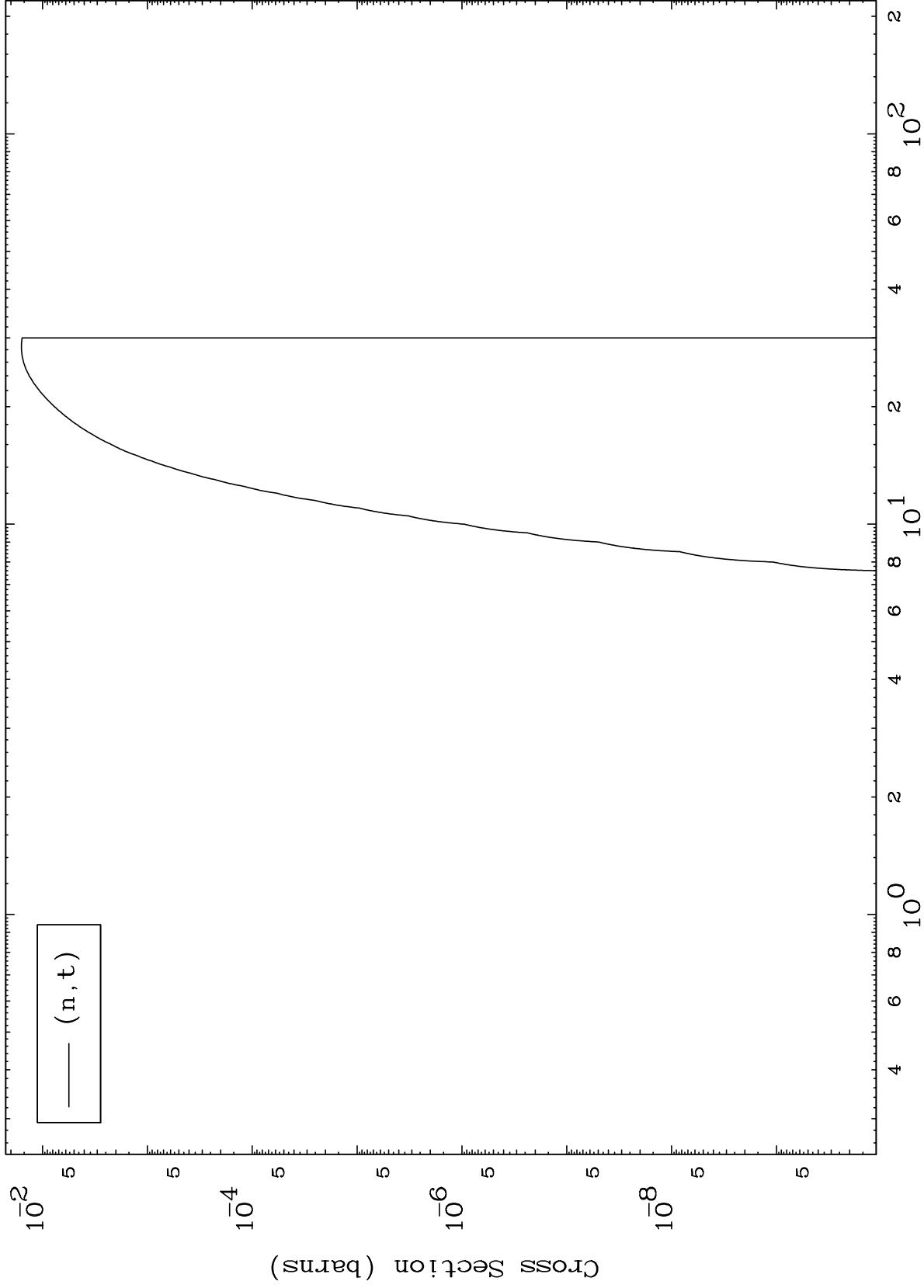
83-Bi-198n



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83-Bi-198n

(n,t) Levels
293 Kelvin Cross Sections



15

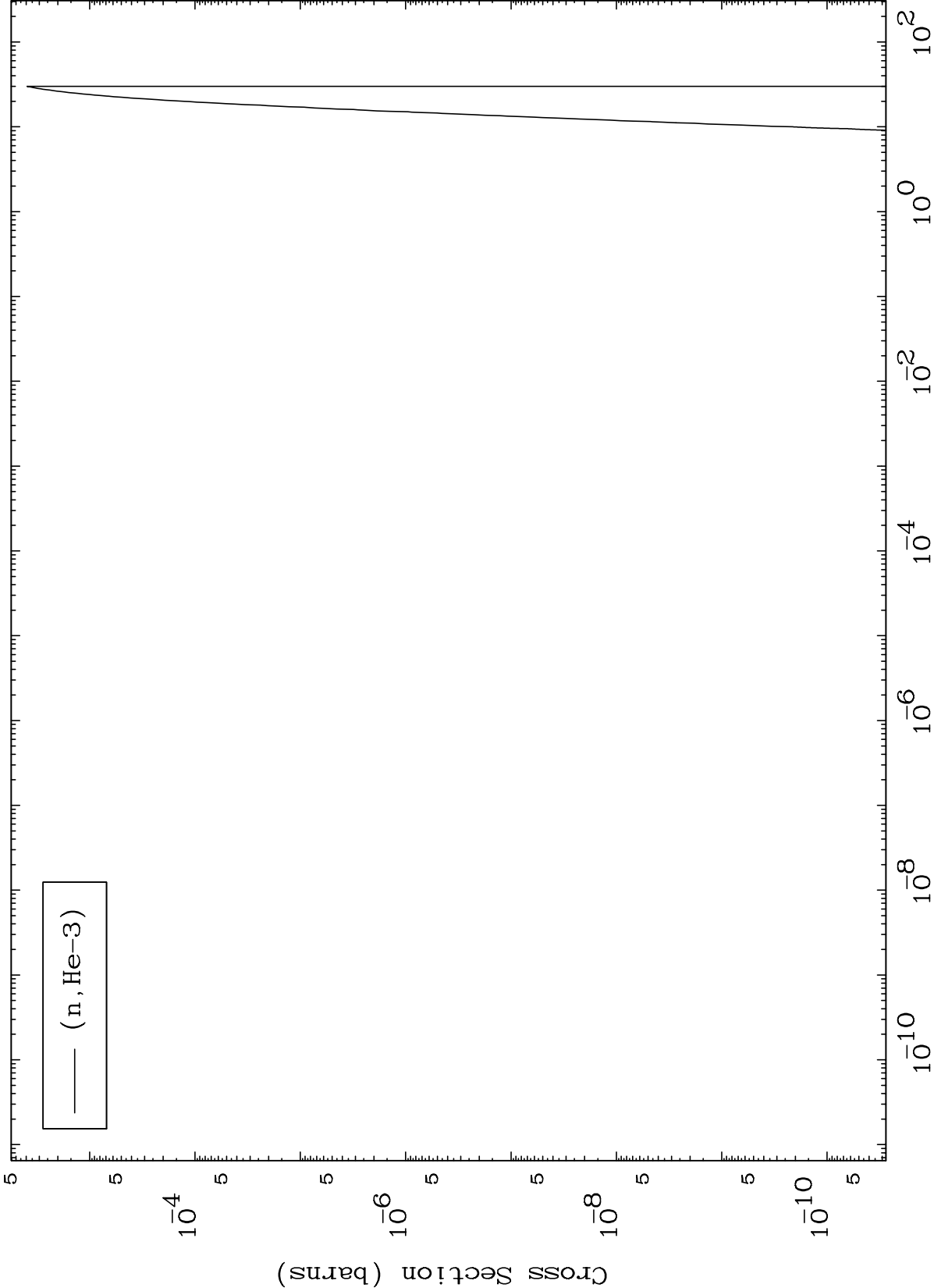
Incident Energy (MeV)

83-Bi-198n

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(n,He3) Levels
293 Kelvin Cross Sections

83-Bi-198n



16

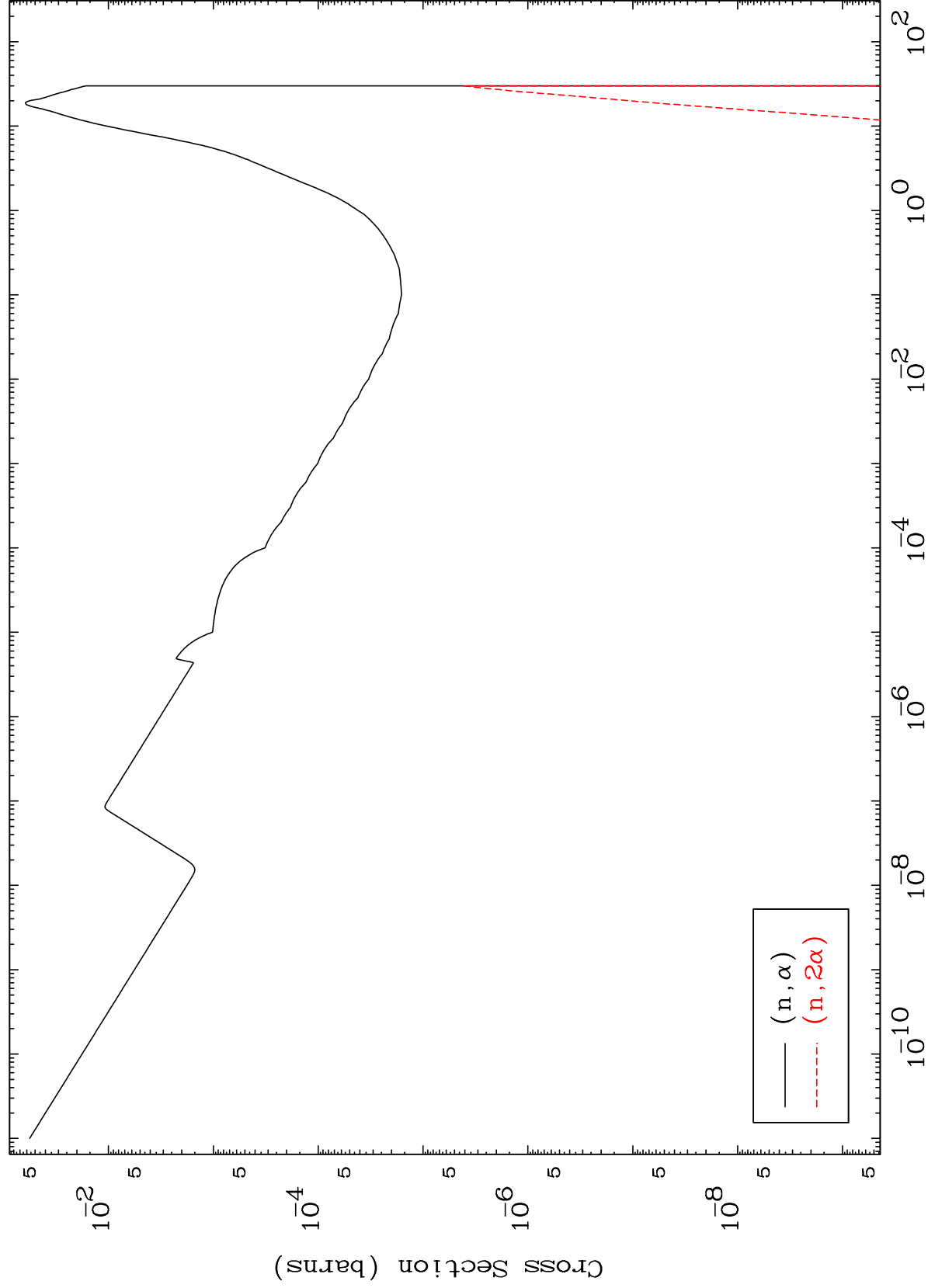
Incident Energy (MeV)

83-Bi-198n

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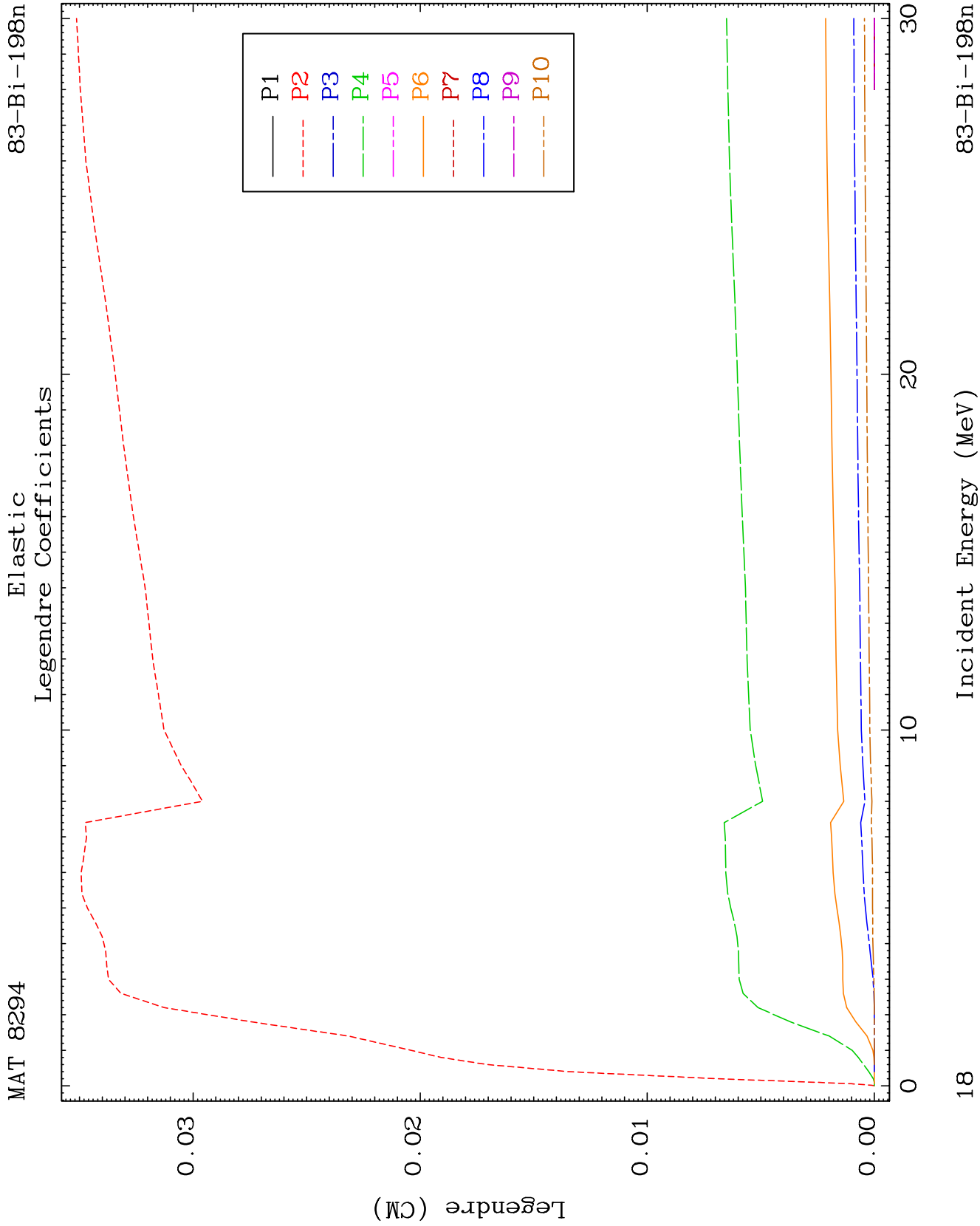
(n, α) Levels
293 Kelvin Cross Sections

83-Bi-198n



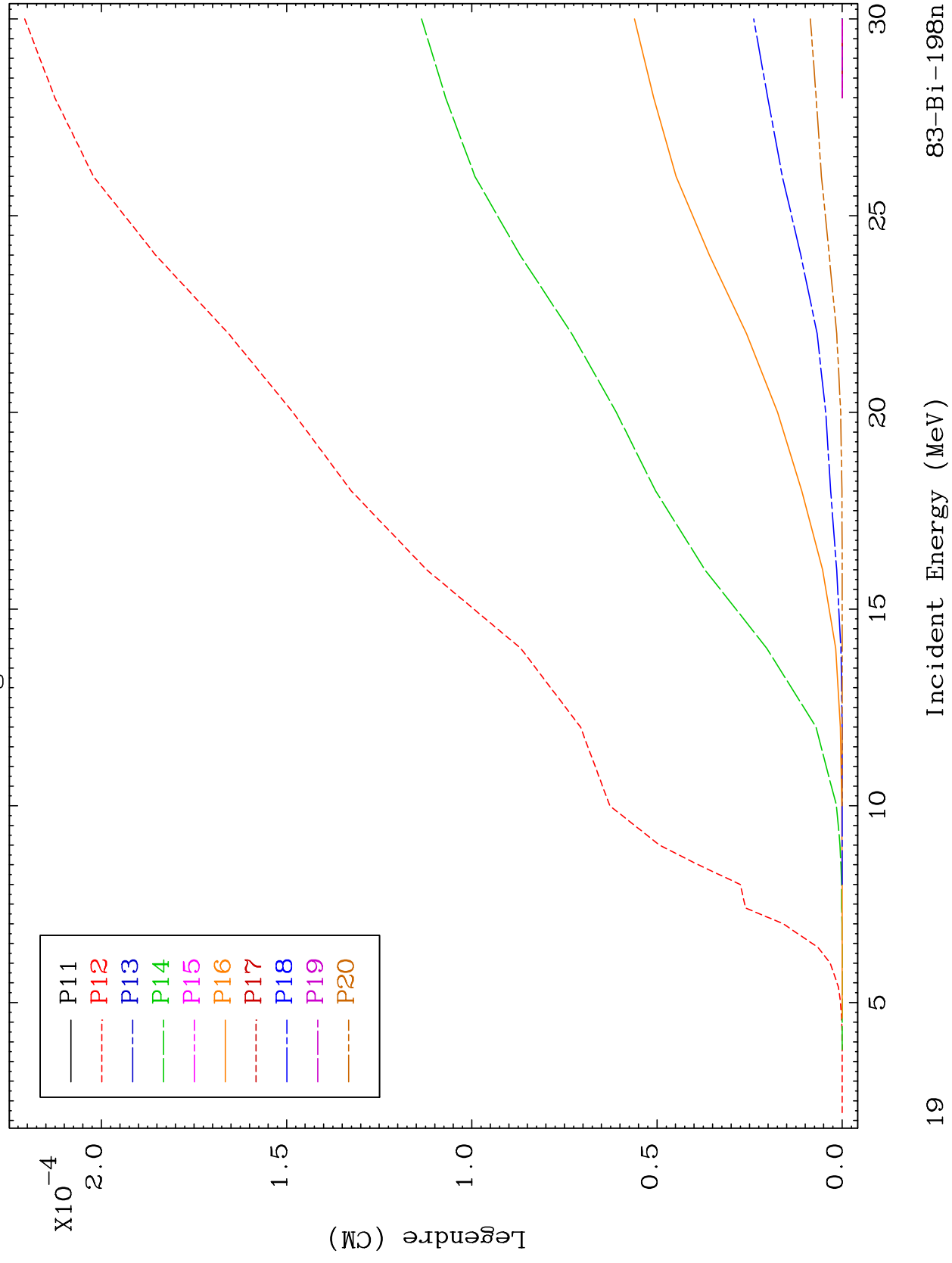
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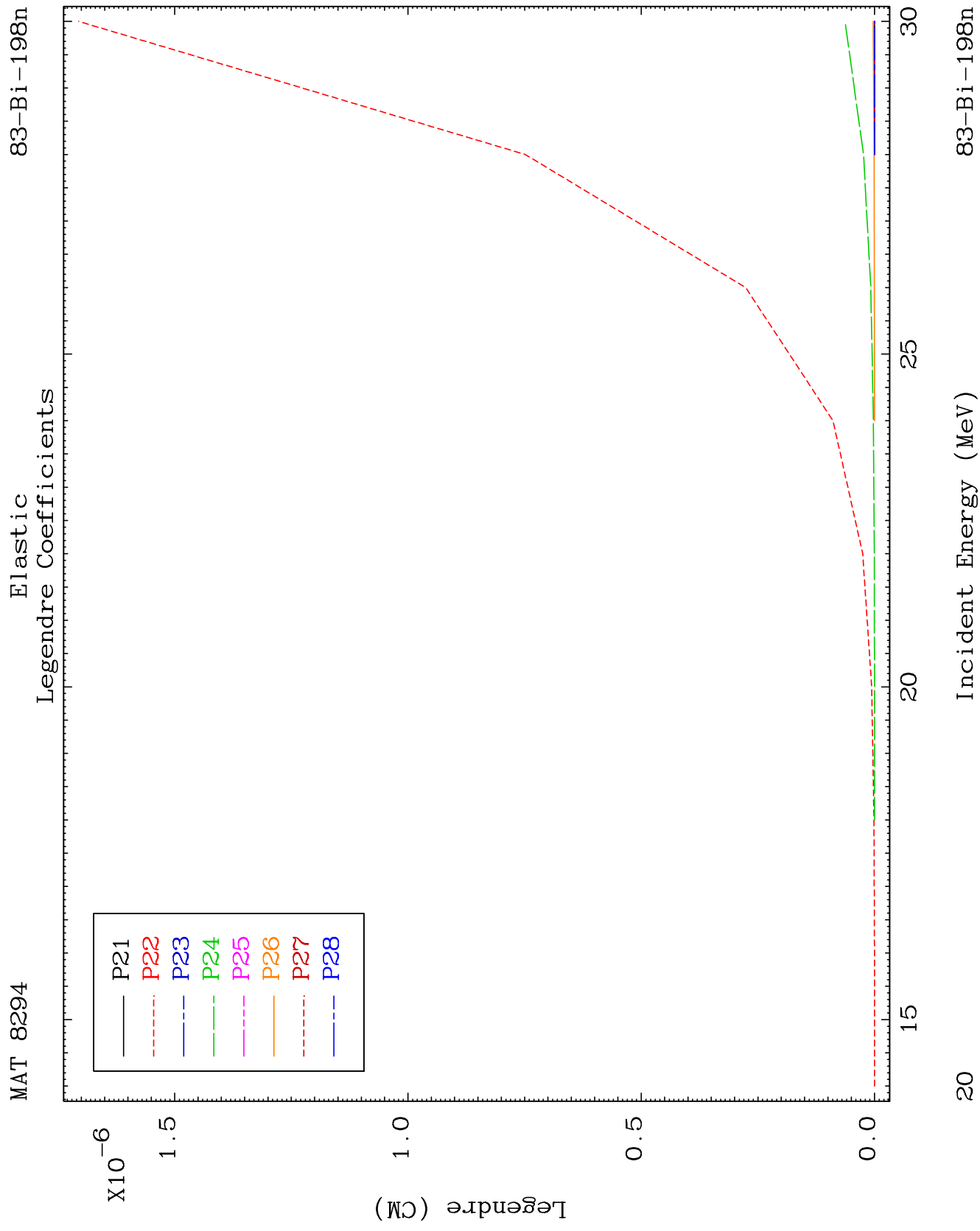
83-Bi-198n



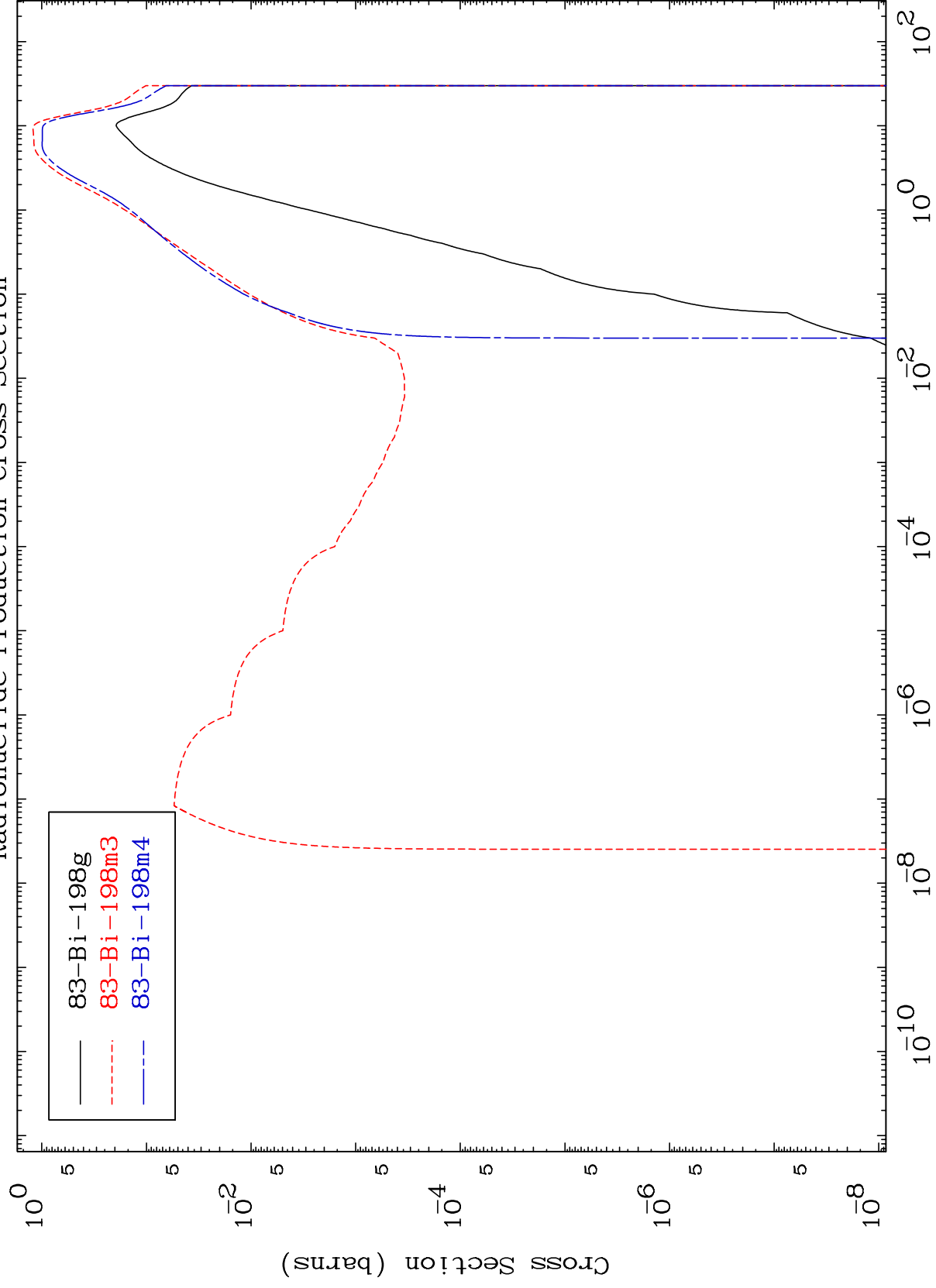
83-Bi-198n

83-Bi-198n

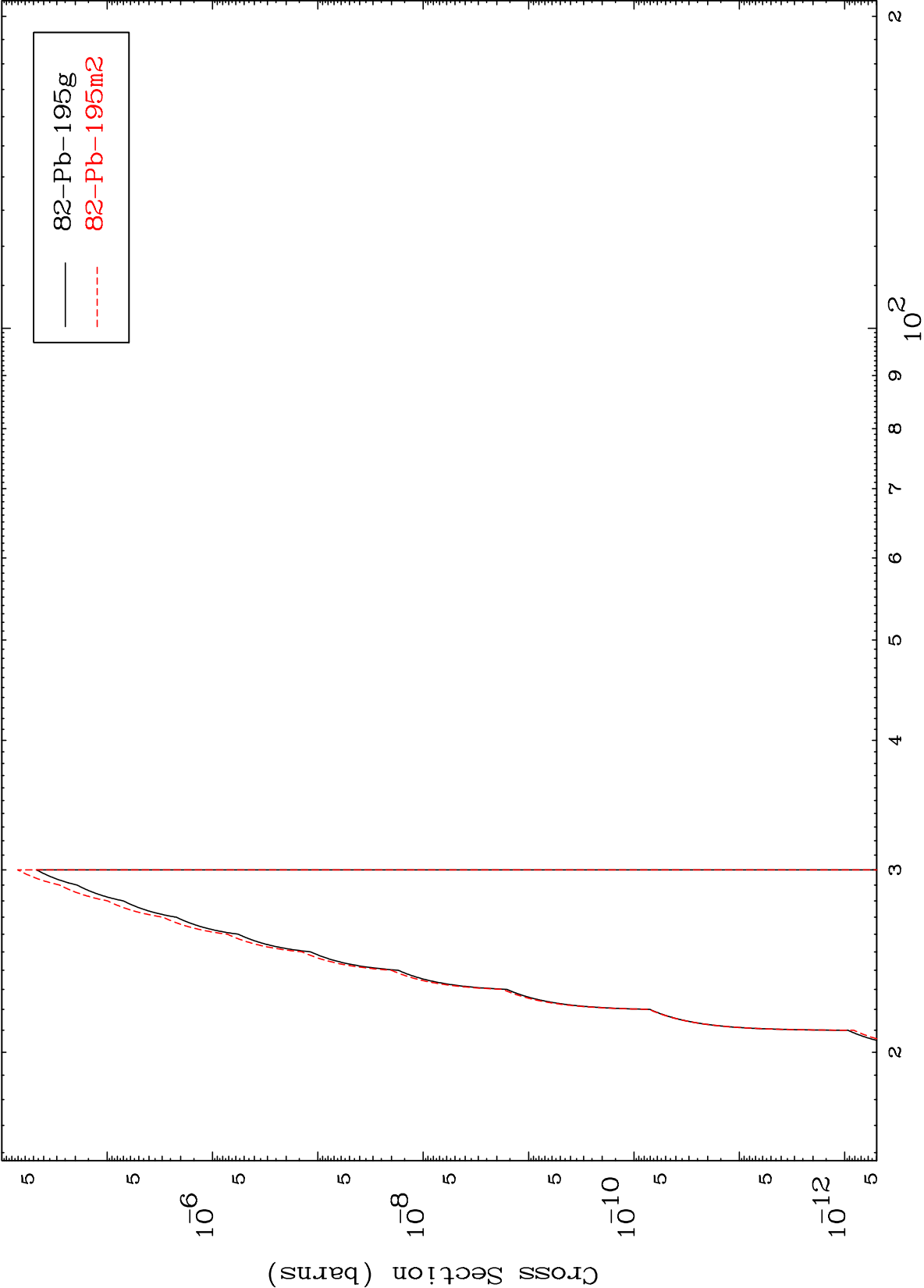




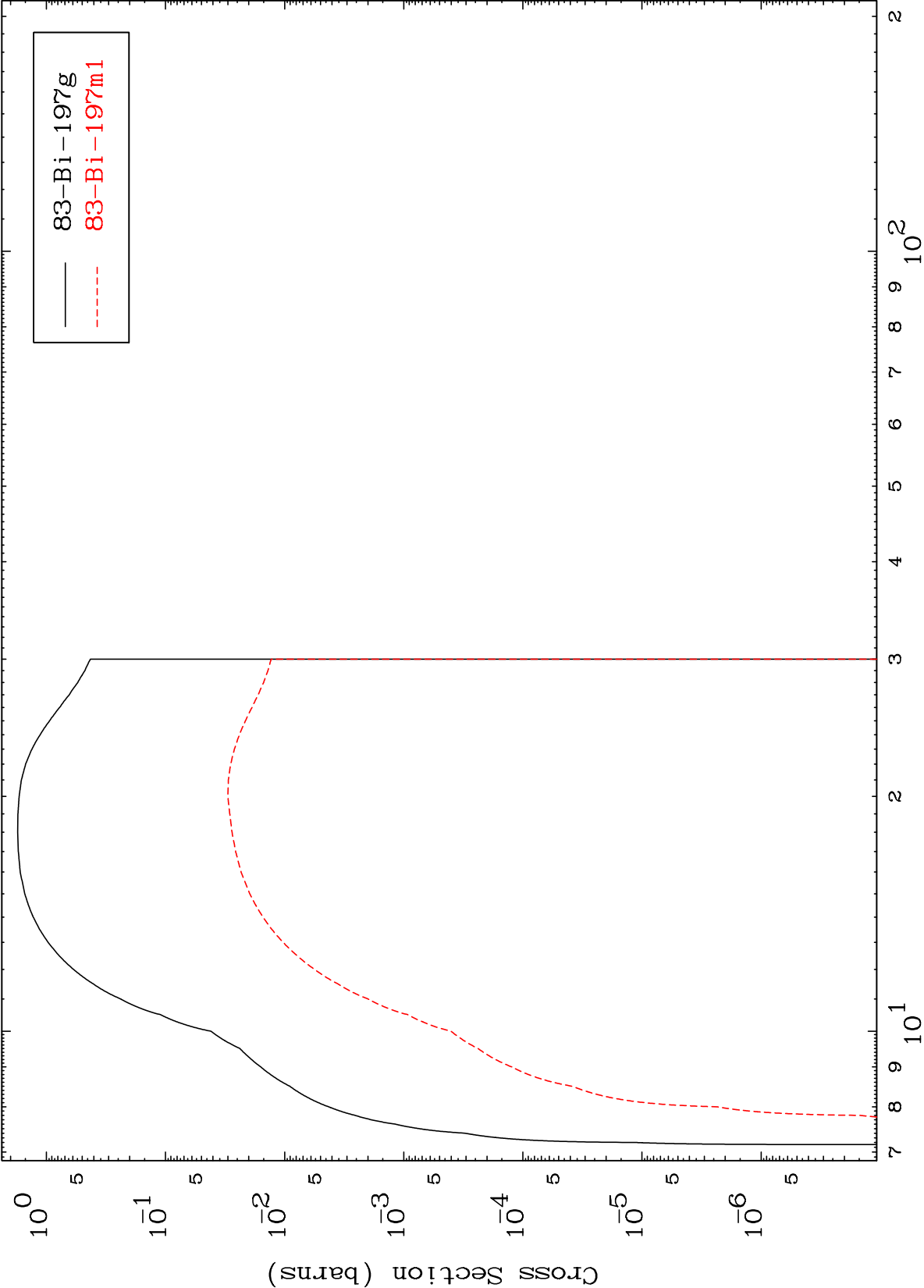
Radionuclide Production Cross Section



Radionuclide Production Cross Section



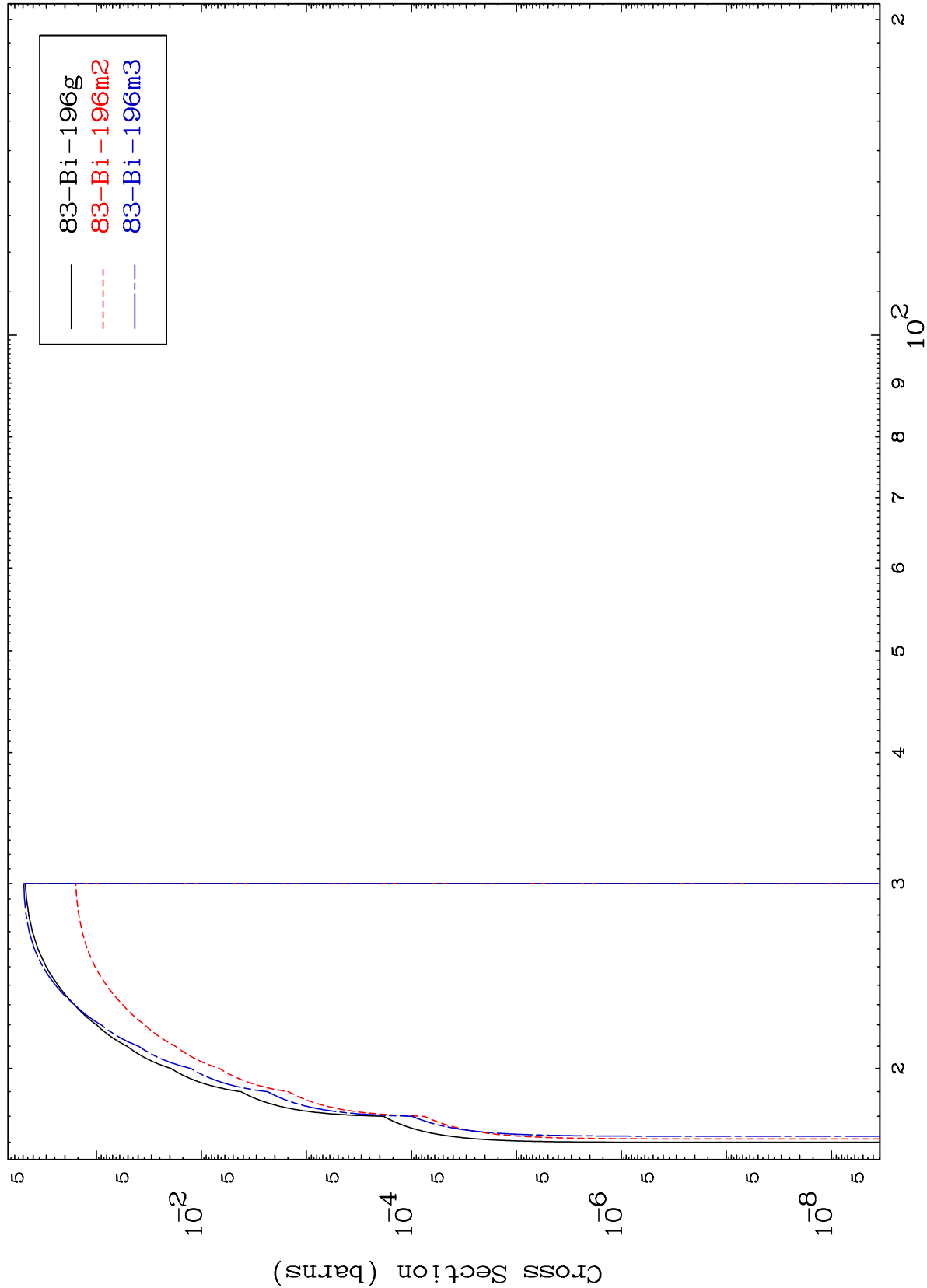
(n,2n)
Radionuclide Production Cross Section



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$^{83}\text{Bi}-198\text{n}$

$(\text{n},3\text{n})$
Radionuclide Production Cross Section



$^{83}\text{Bi}-198\text{n}$

Incident Energy (MeV)

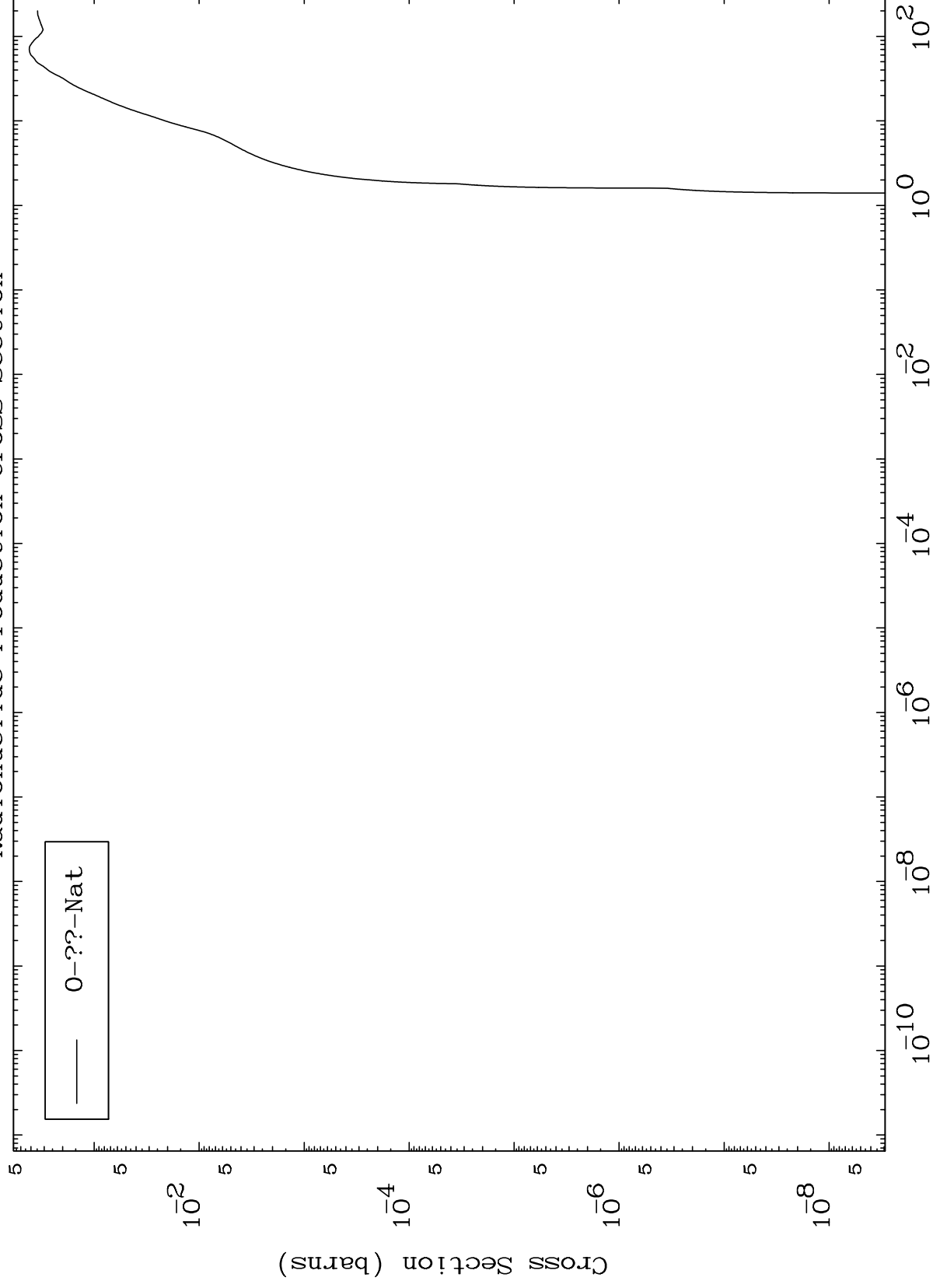
24

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Fission

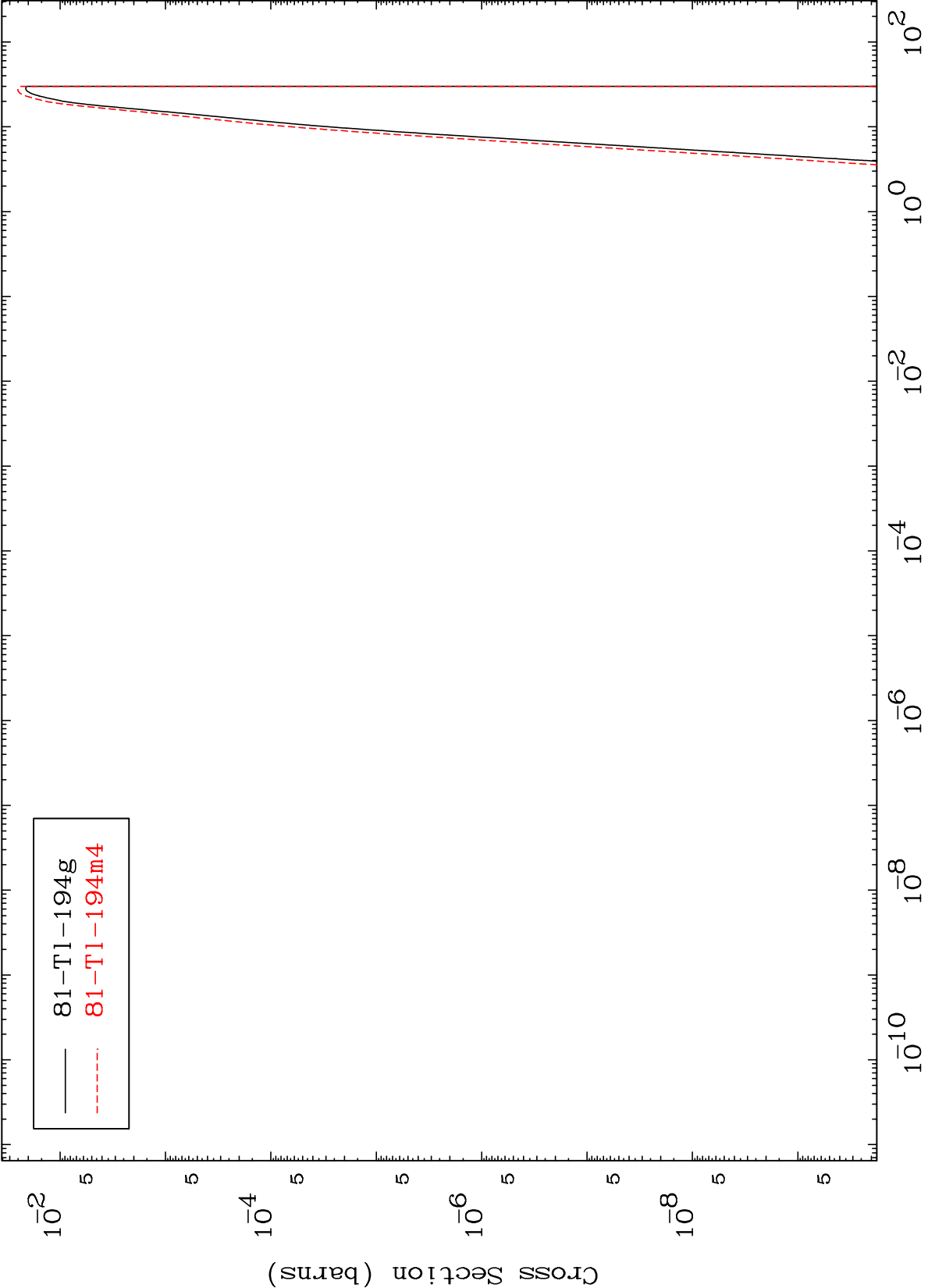
$^{83}\text{Bi}-198\text{n}$

Radionuclide Production Cross Section



0-??-Nat

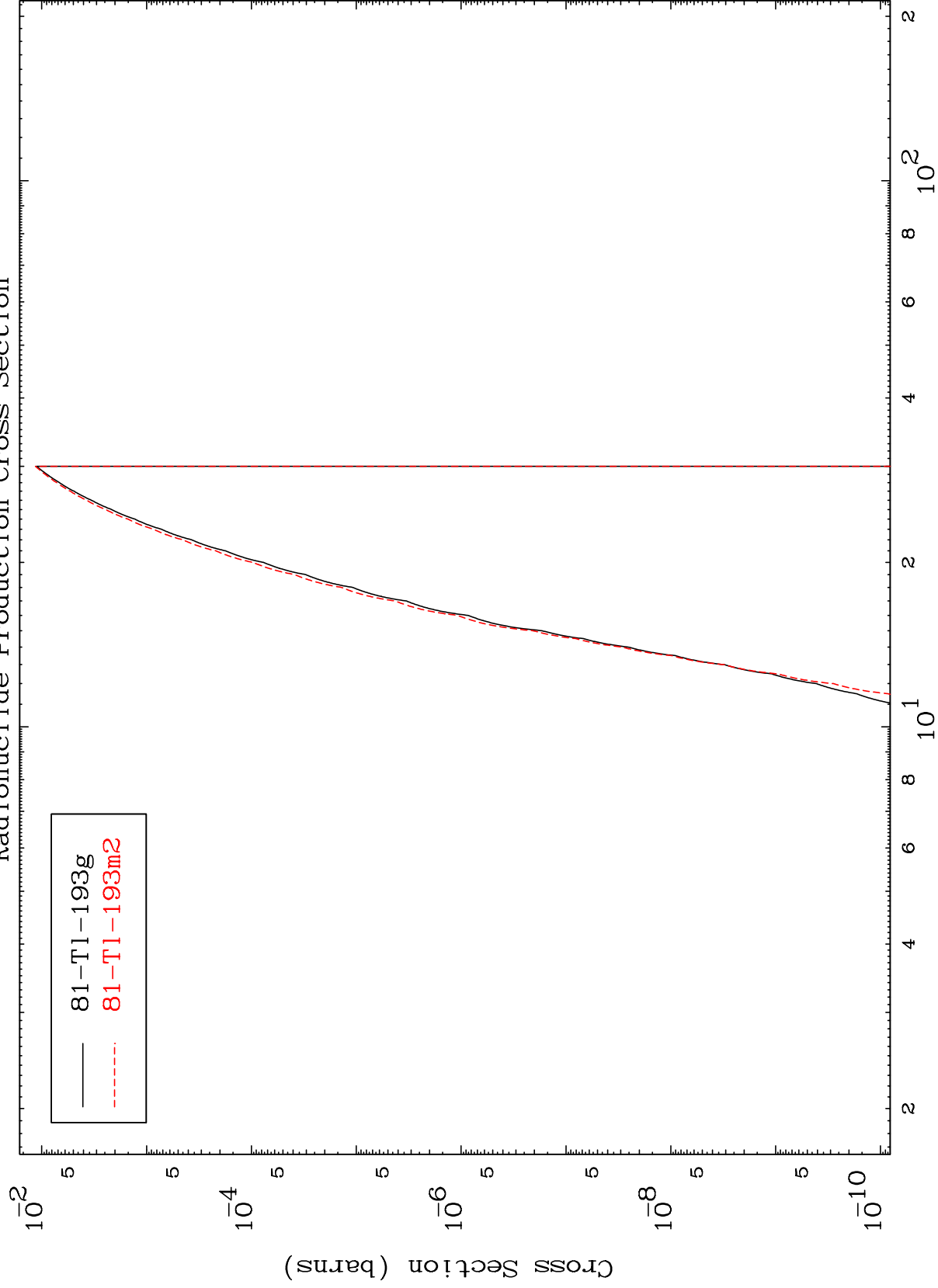
Radionuclide Production Cross Section



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83-Bi-198n

(n,2n) α
Radionuclide Production Cross Section

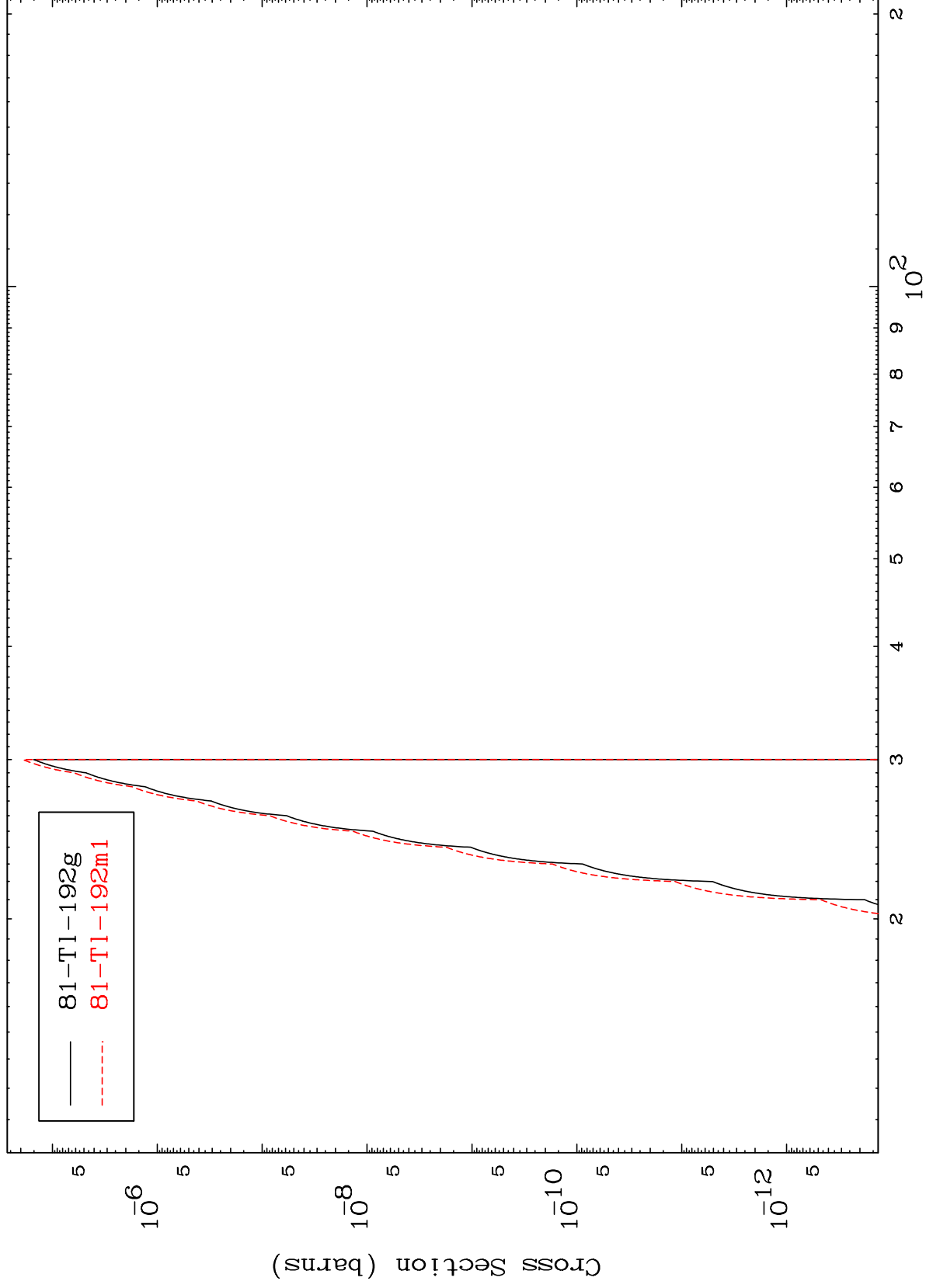


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Incident Energy (MeV)

83-Bi-198n

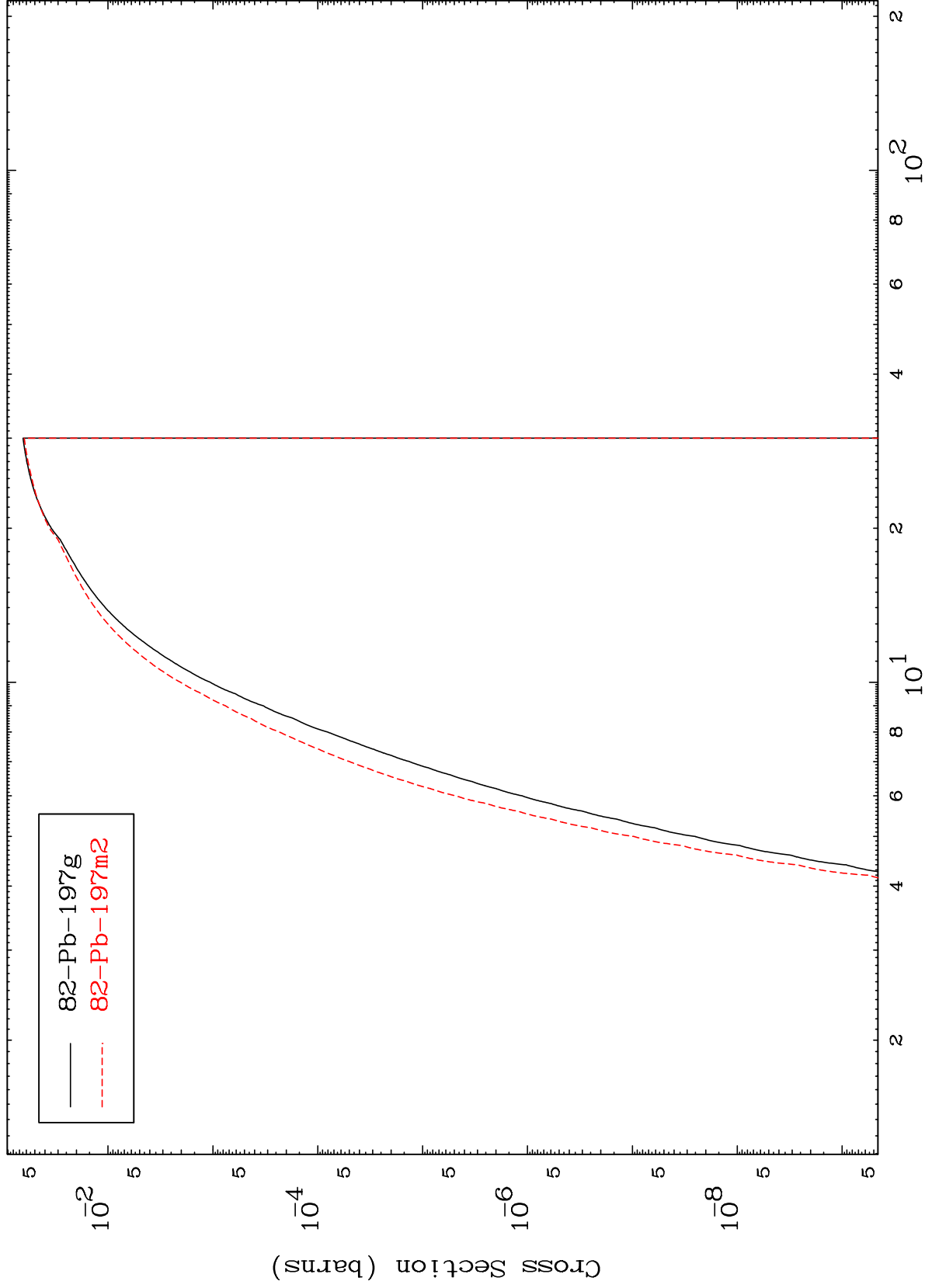
(n,3n) α
Radionuclide Production Cross Section



MAT 8294

83-Bi-198n

(n,n') p
Radionuclide Production Cross Section



29

Incident Energy (MeV)

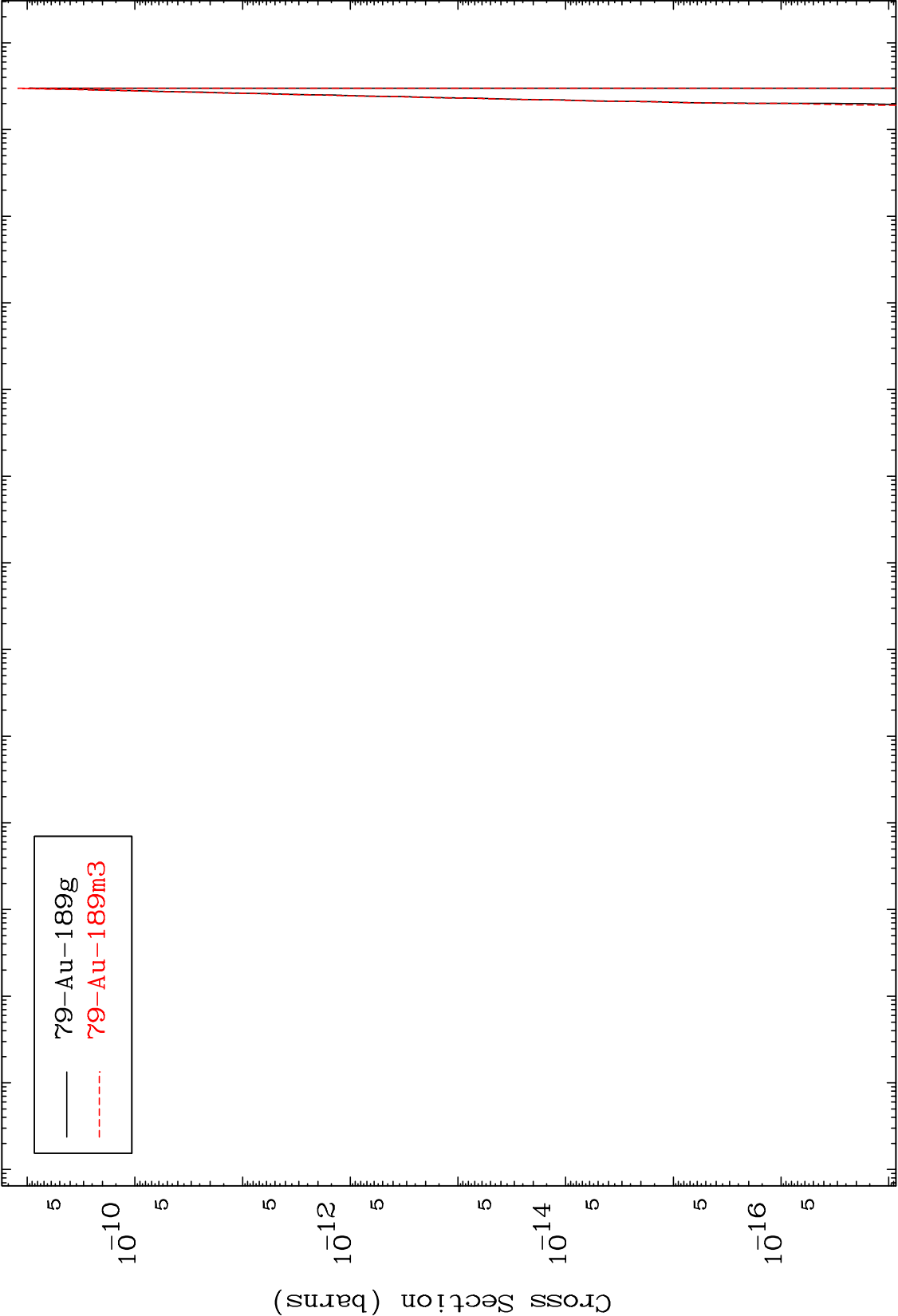
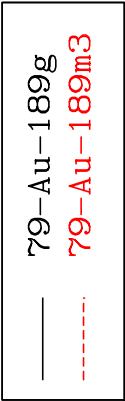
83-Bi-198n

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(n,2n) 2 α

83-Bi-198n

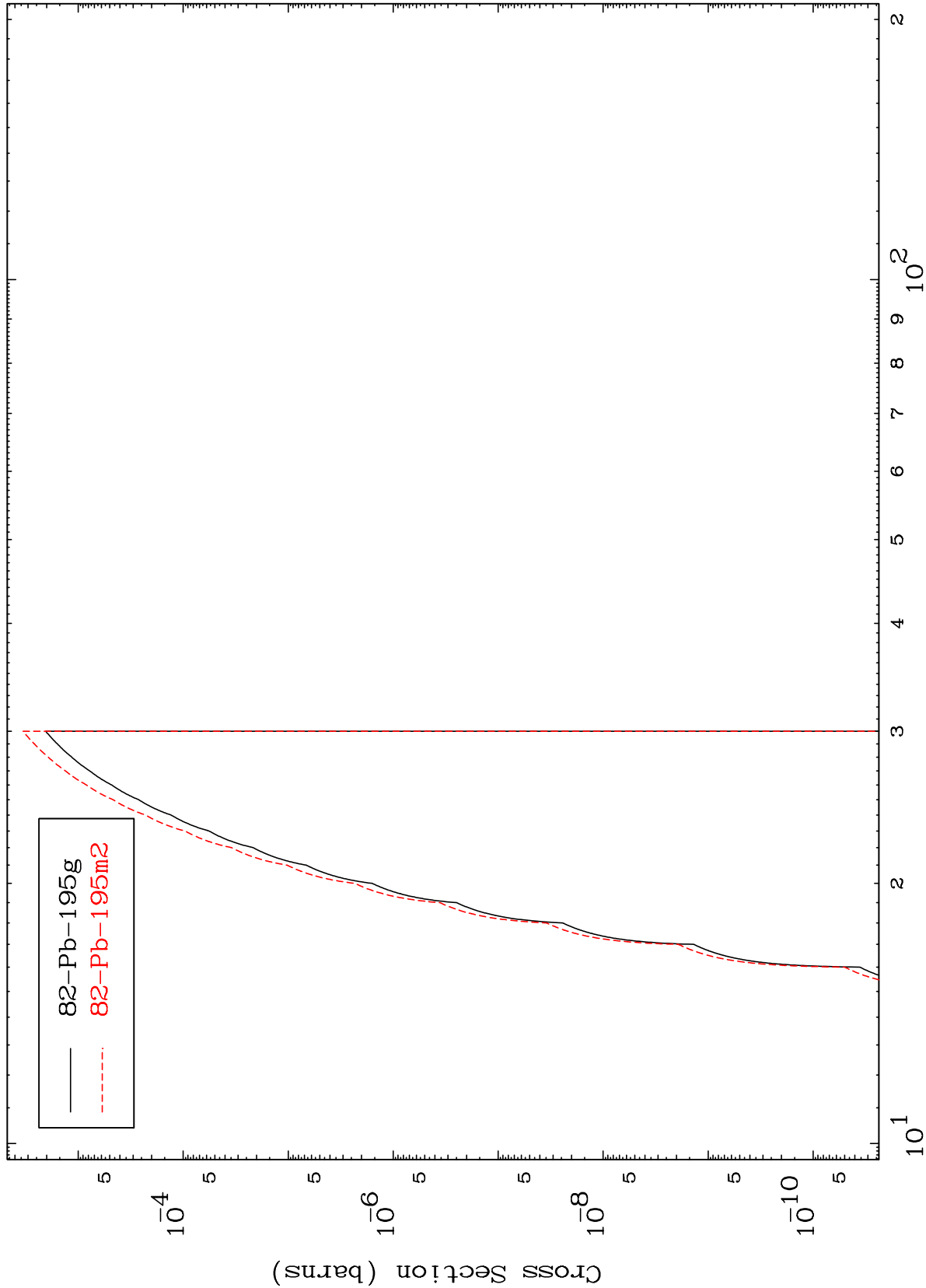
Radionuclide Production Cross Section



MAT 8294

83-Bi-198n

(n,n') t
Radionuclide Production Cross Section



83-Bi-198n

Incident Energy (MeV)

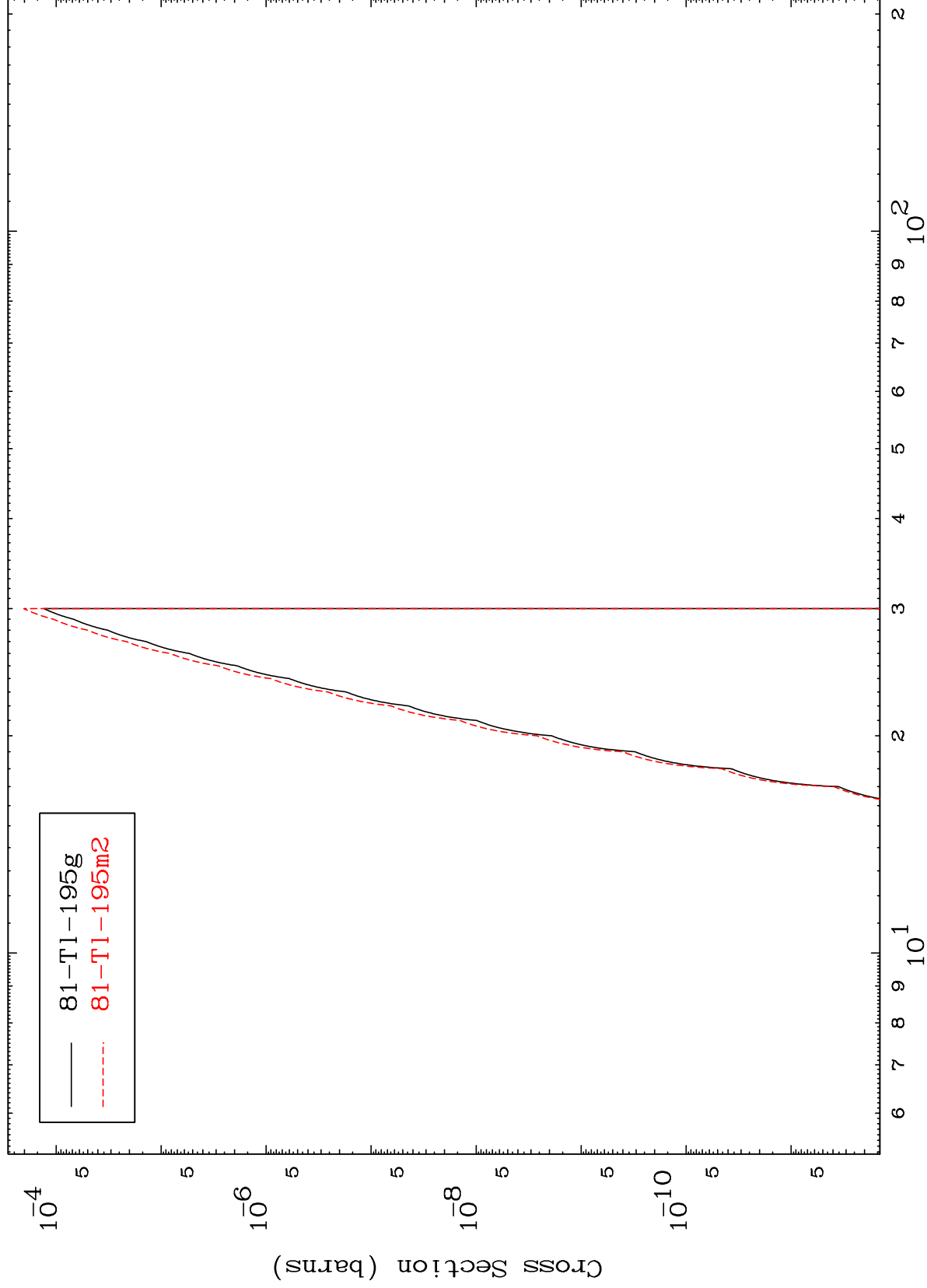
31

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(n,n') He-3

83-Bi-198n

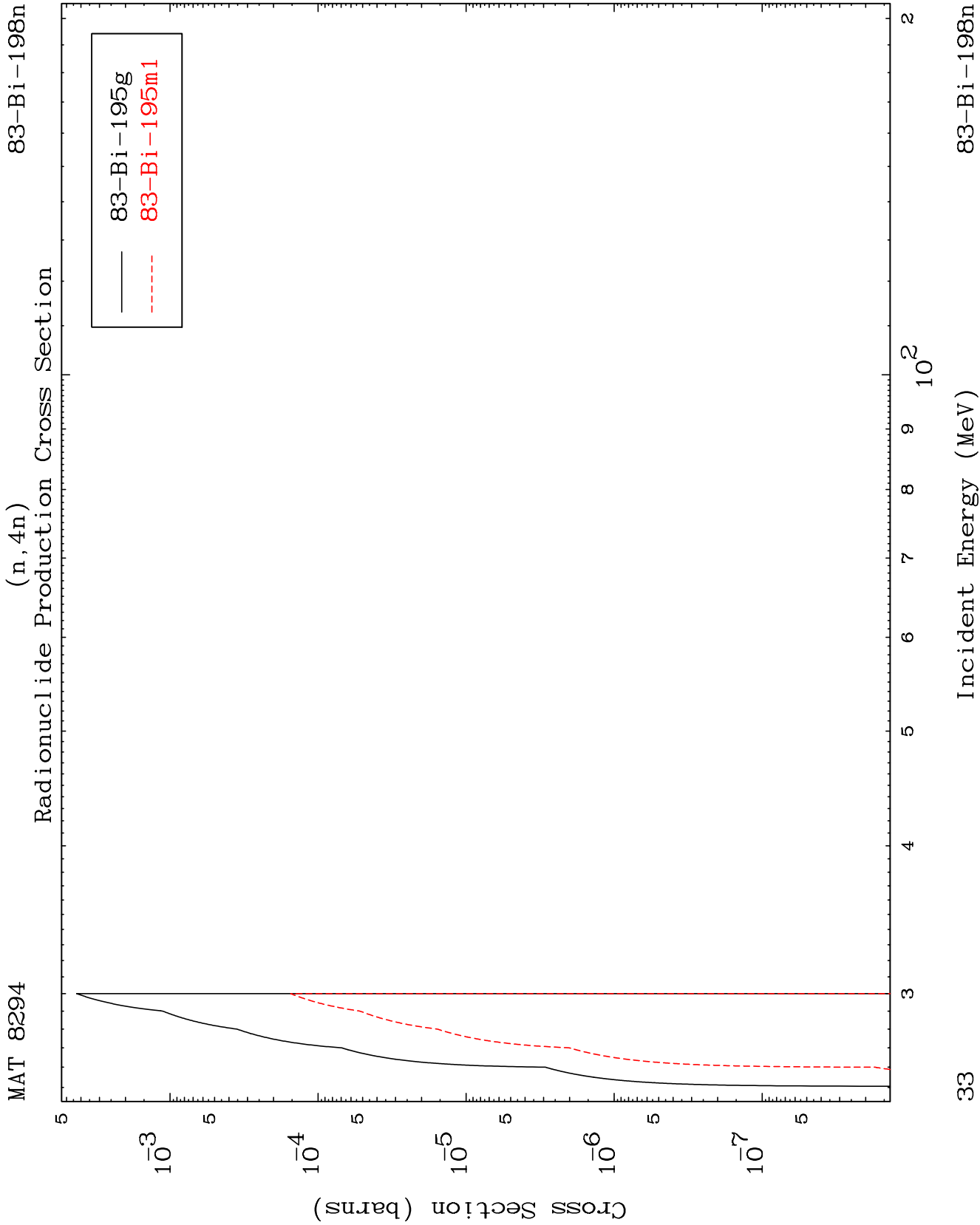
Radionuclide Production Cross Section



32

Incident Energy (MeV)

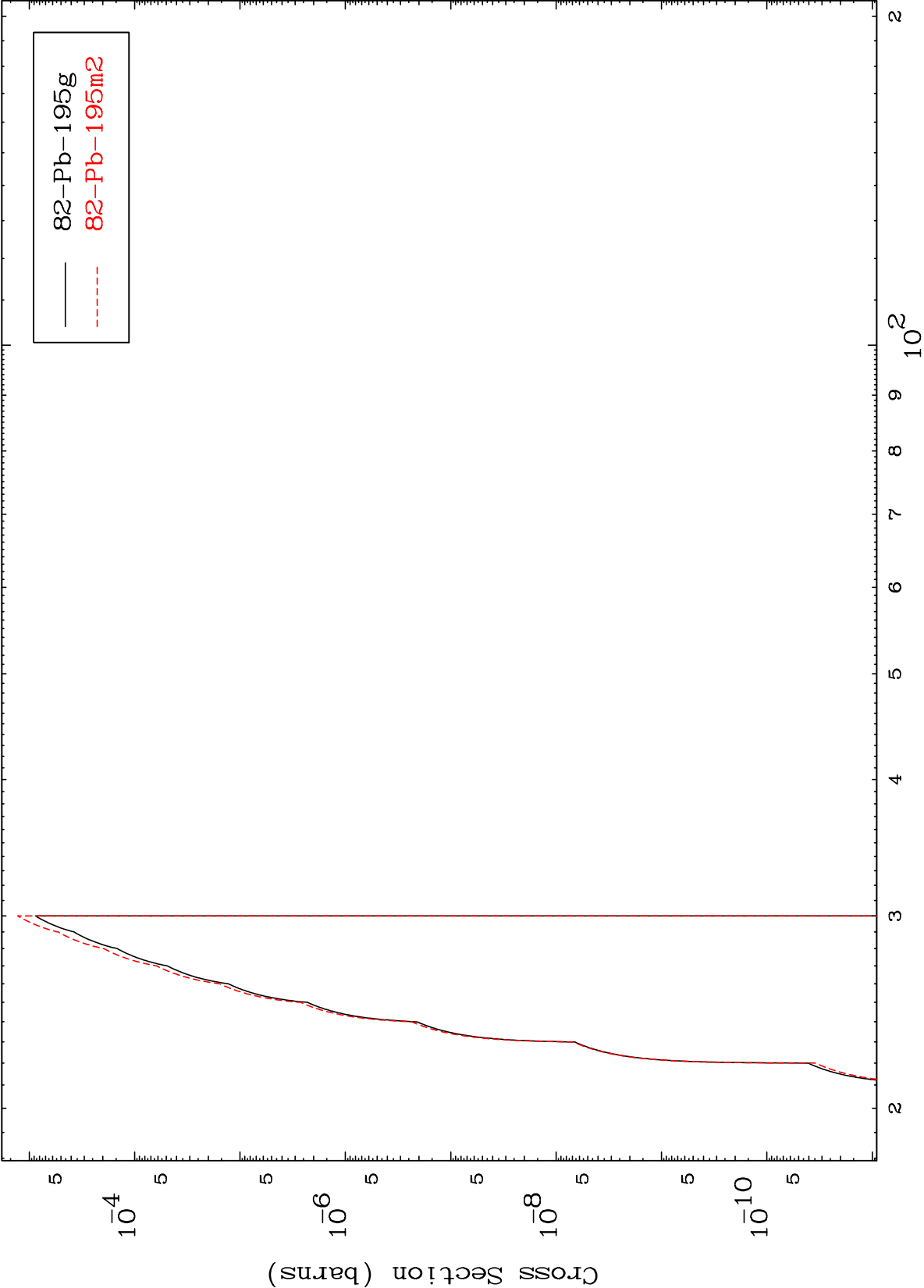
83-Bi-198n



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83-Bi-198n

(n,3n) p
Radionuclide Production Cross Section



34

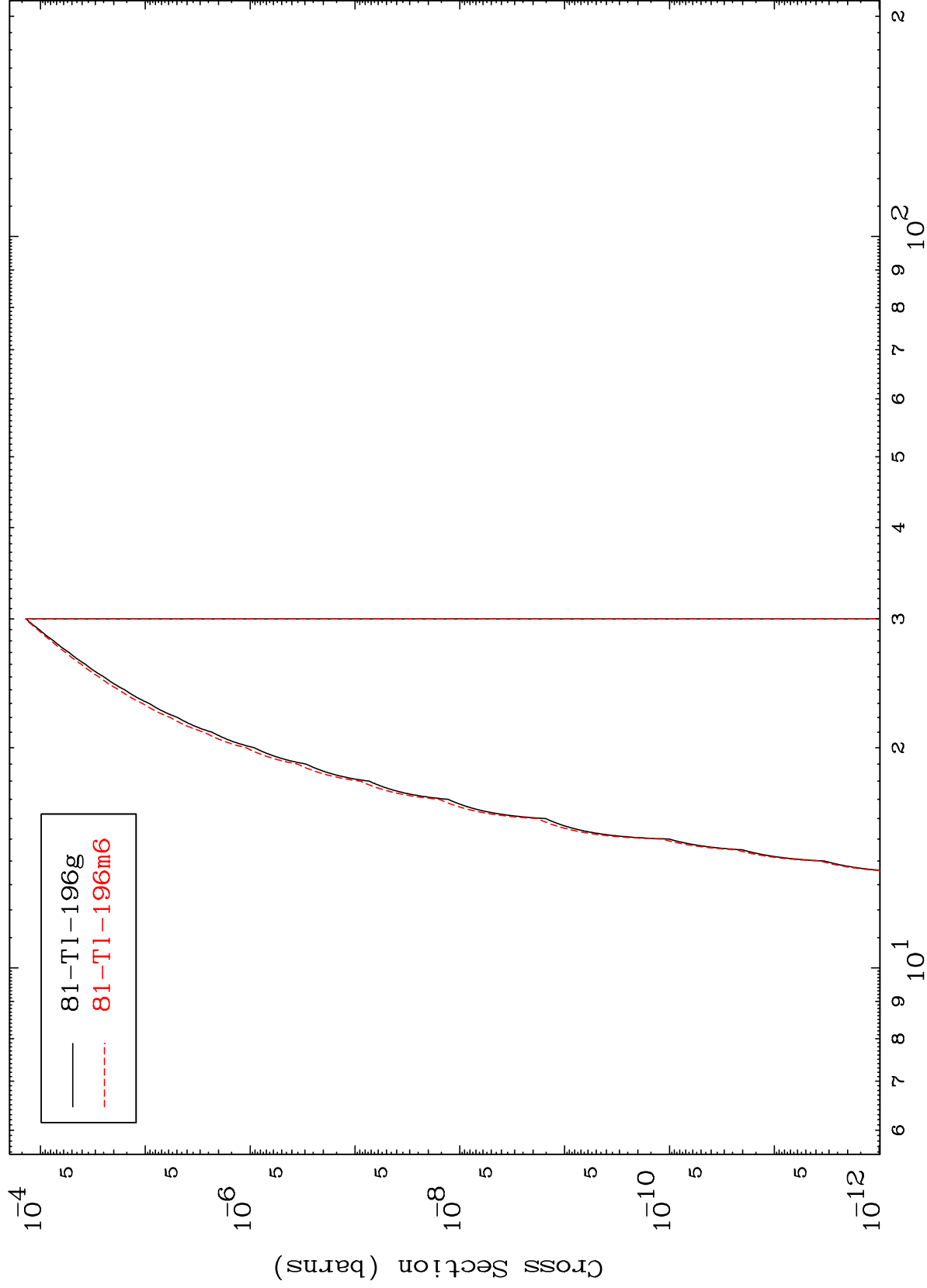
Incident Energy (MeV)

83-Bi-198n

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83-Bi-198n

(n,2n) p
Radionuclide Production Cross Section

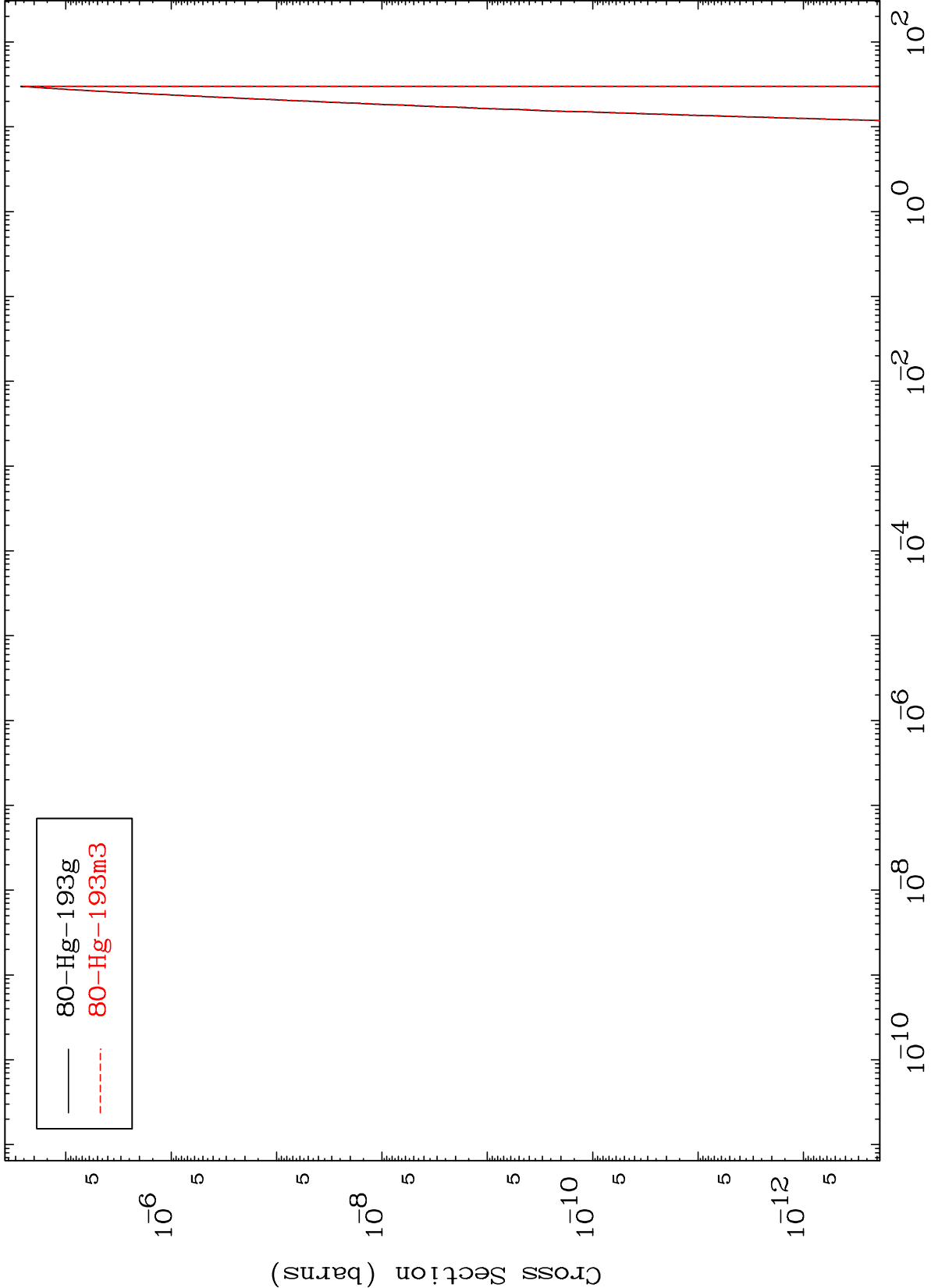


35

Incident Energy (MeV)

83-Bi-198n

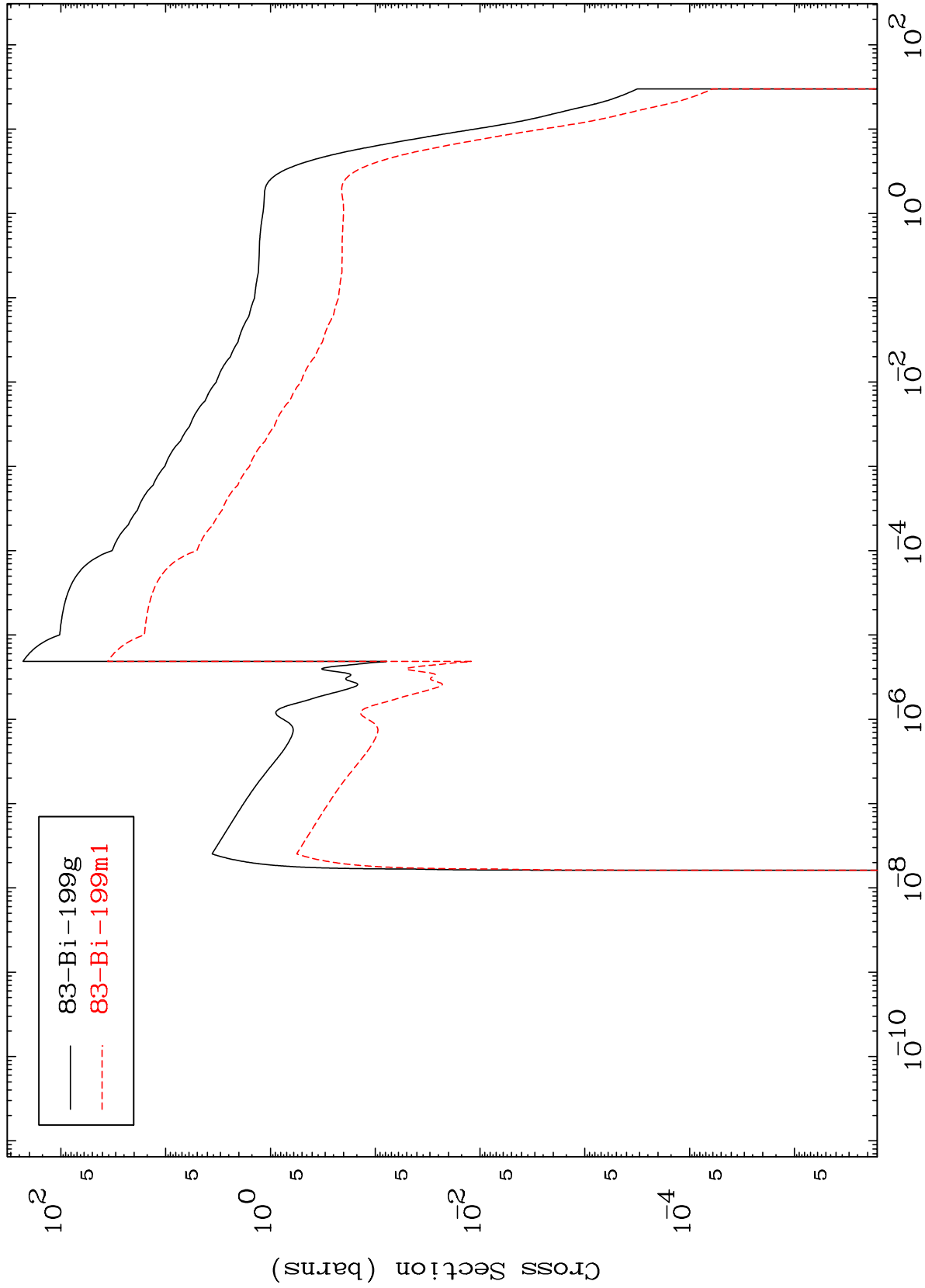
(n,n') p α
Radionuclide Production Cross Section



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83-Bi-198n

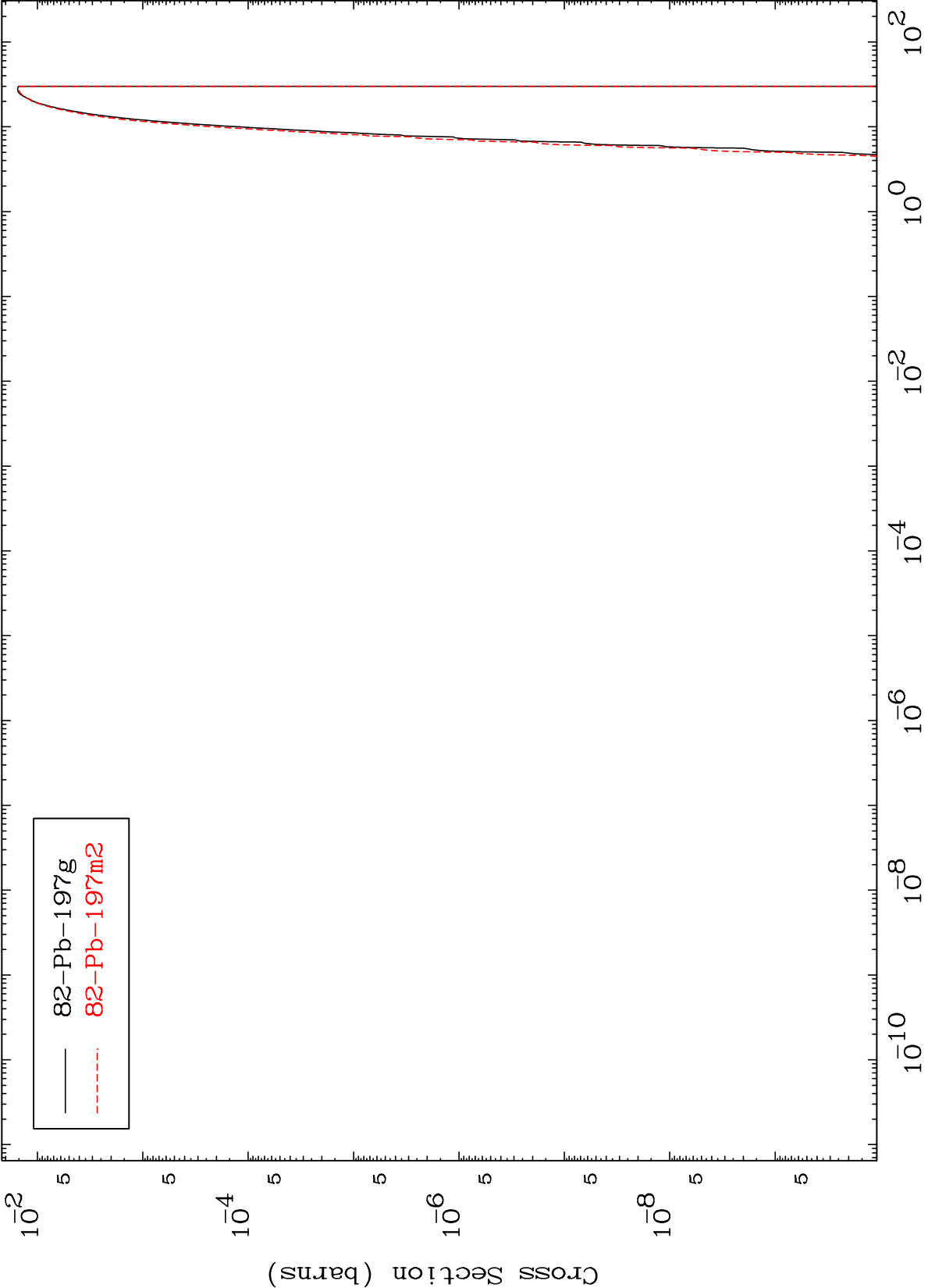
Radionuclide Production Cross Section



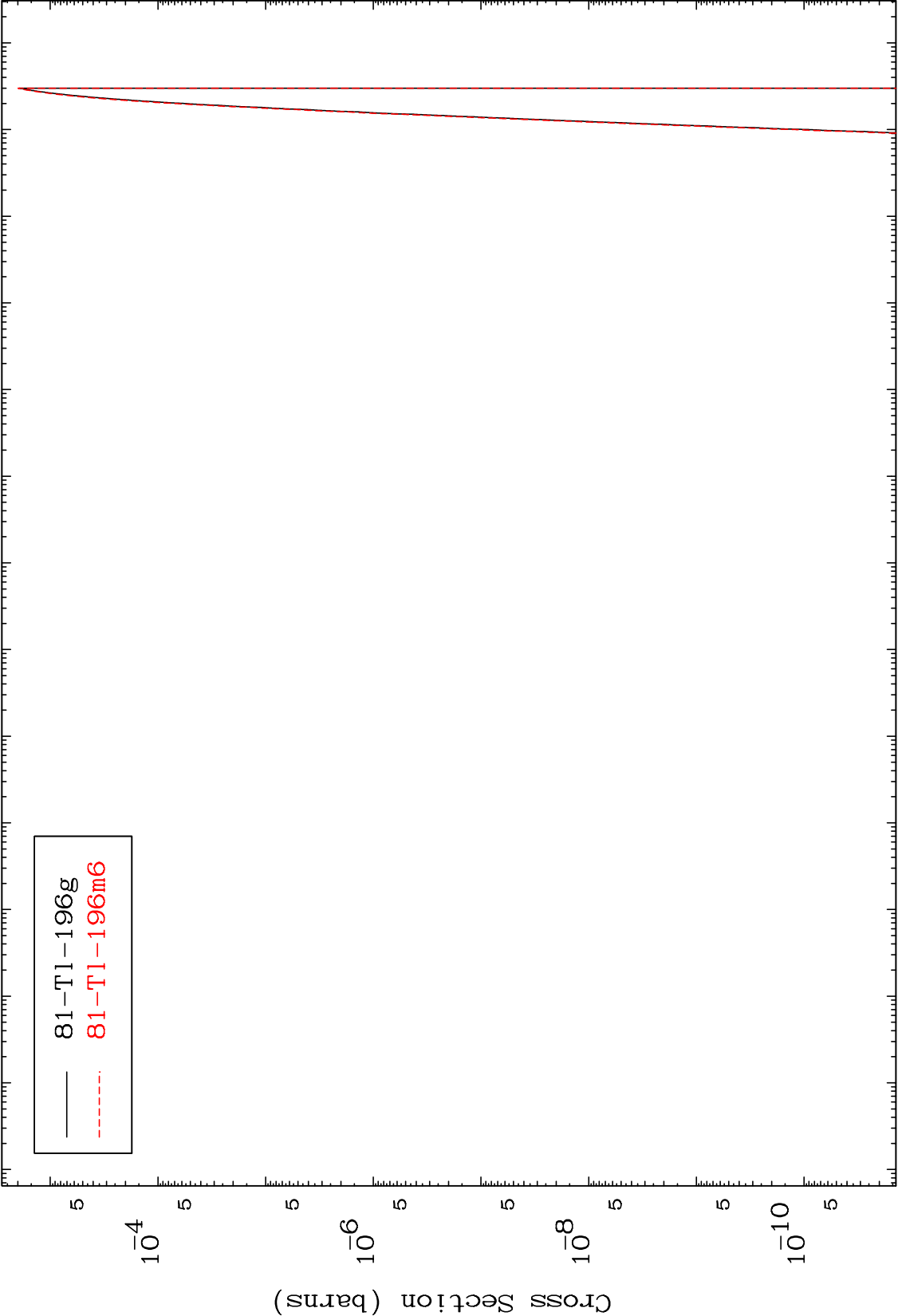
37

83-Bi-198n

Radionuclide Production Cross Section



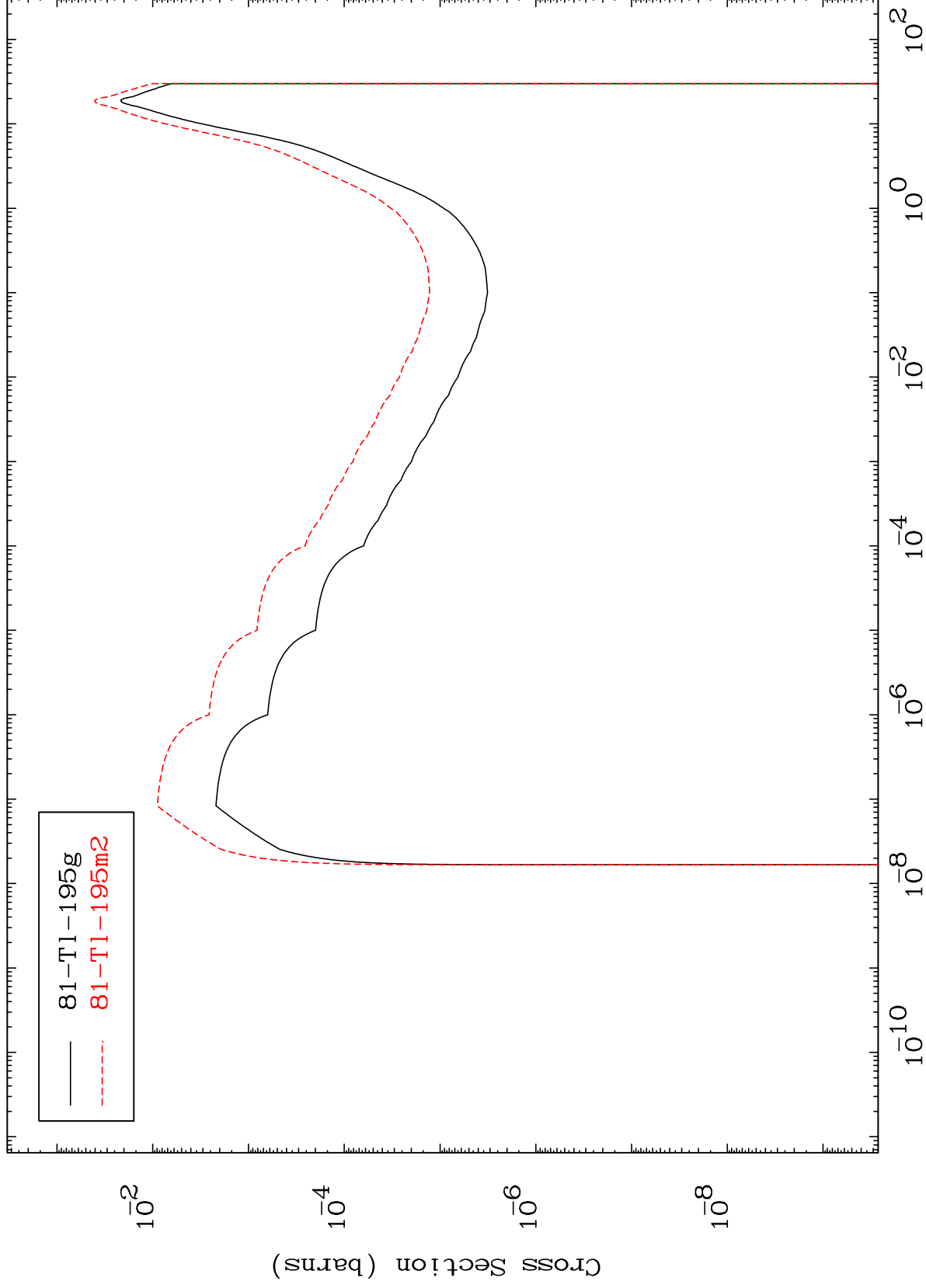
Radionuclide Production Cross Section



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83-Bi-198n

Radionuclide Production Cross Section

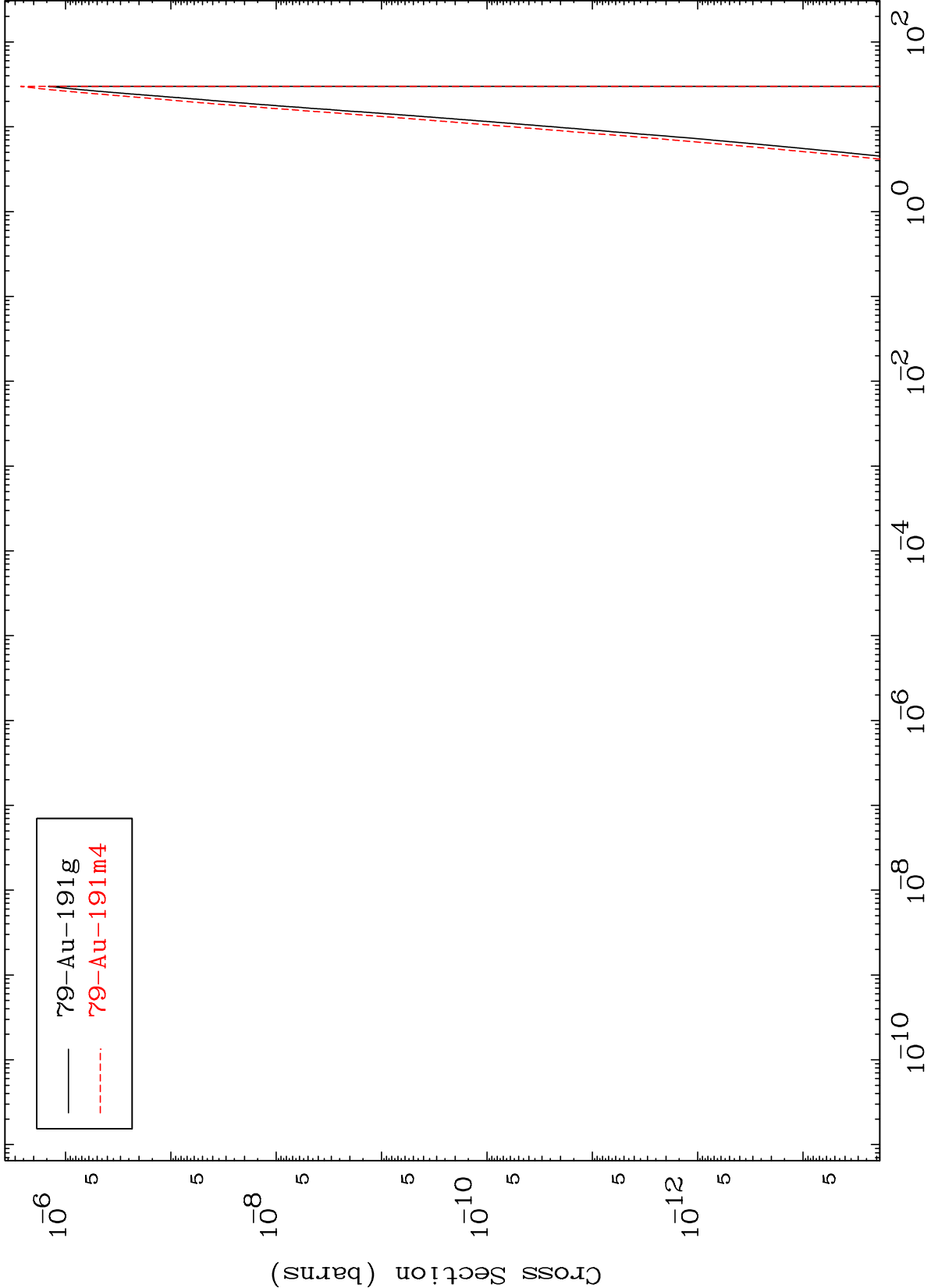


83-Bi-198n

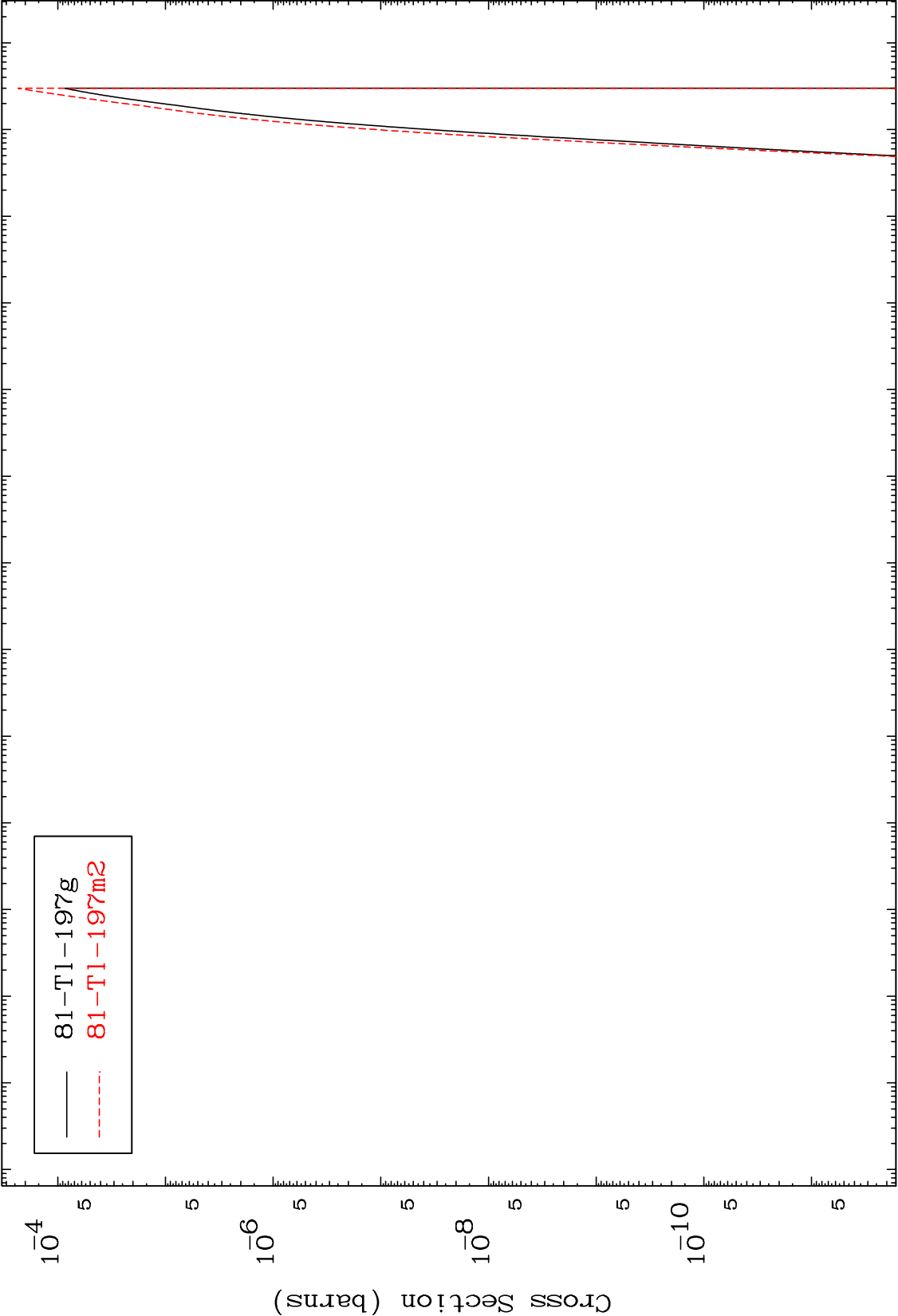
Incident Energy (MeV)

40

Radionuclide Production Cross Section
(n,2α)



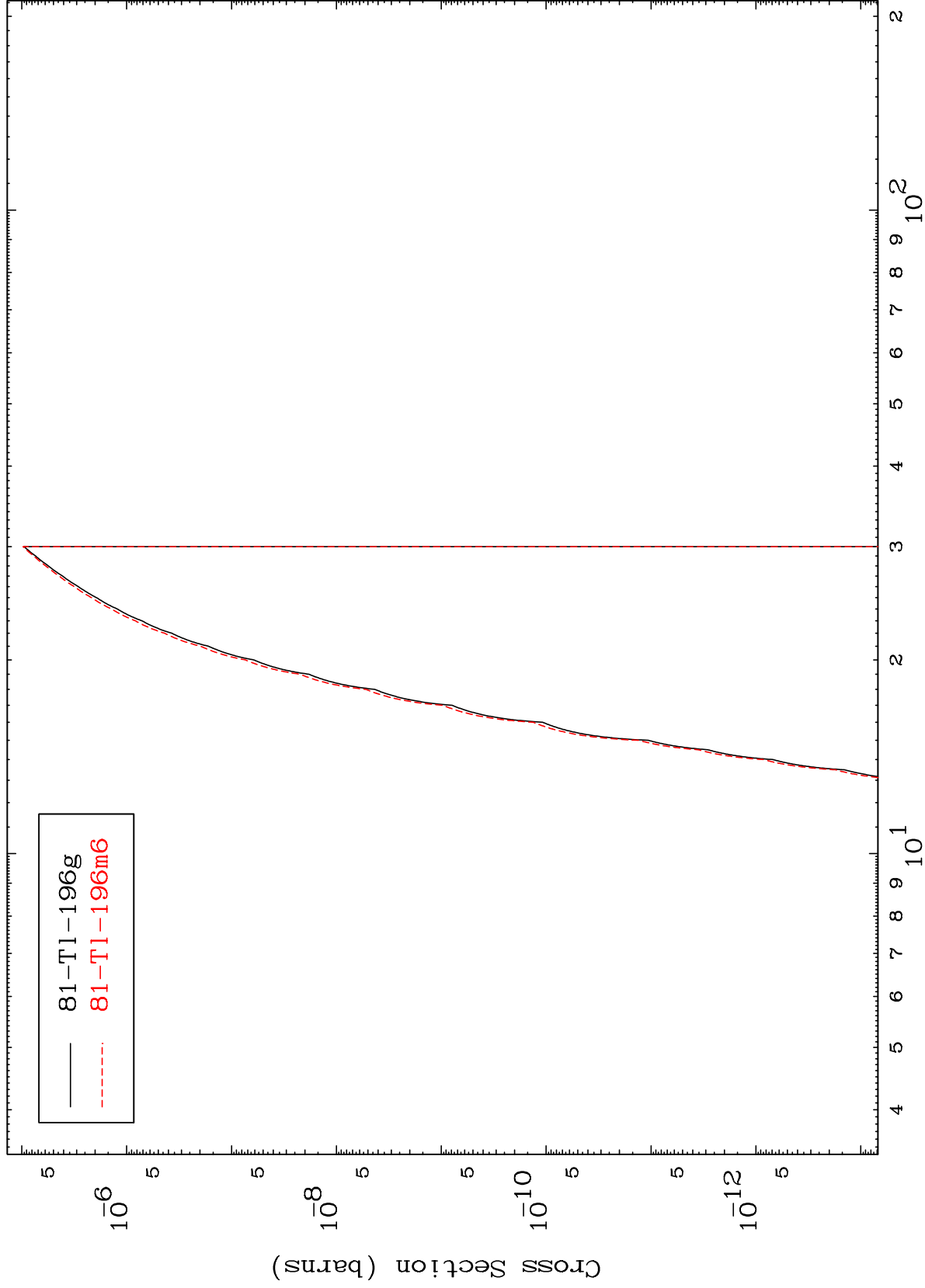
Radionuclide Production Cross Section



MAT 8294

83-Bi-198n

(n,p) d
Radionuclide Production Cross Section



43

Incident Energy (MeV)

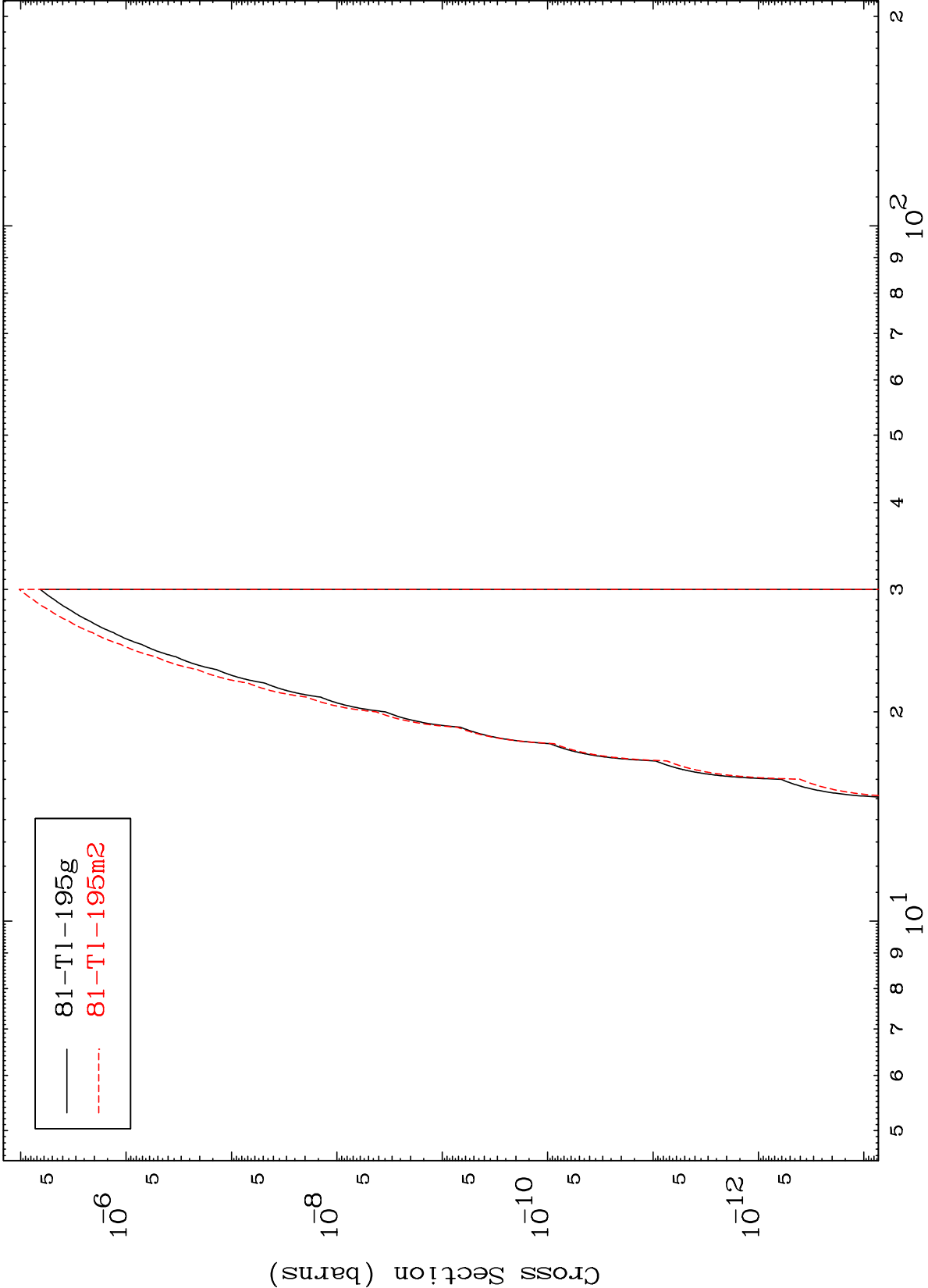
83-Bi-198n

MAT 8294

(n,p) t

83-Bi-198n

Radionuclide Production Cross Section



44

Incident Energy (MeV)

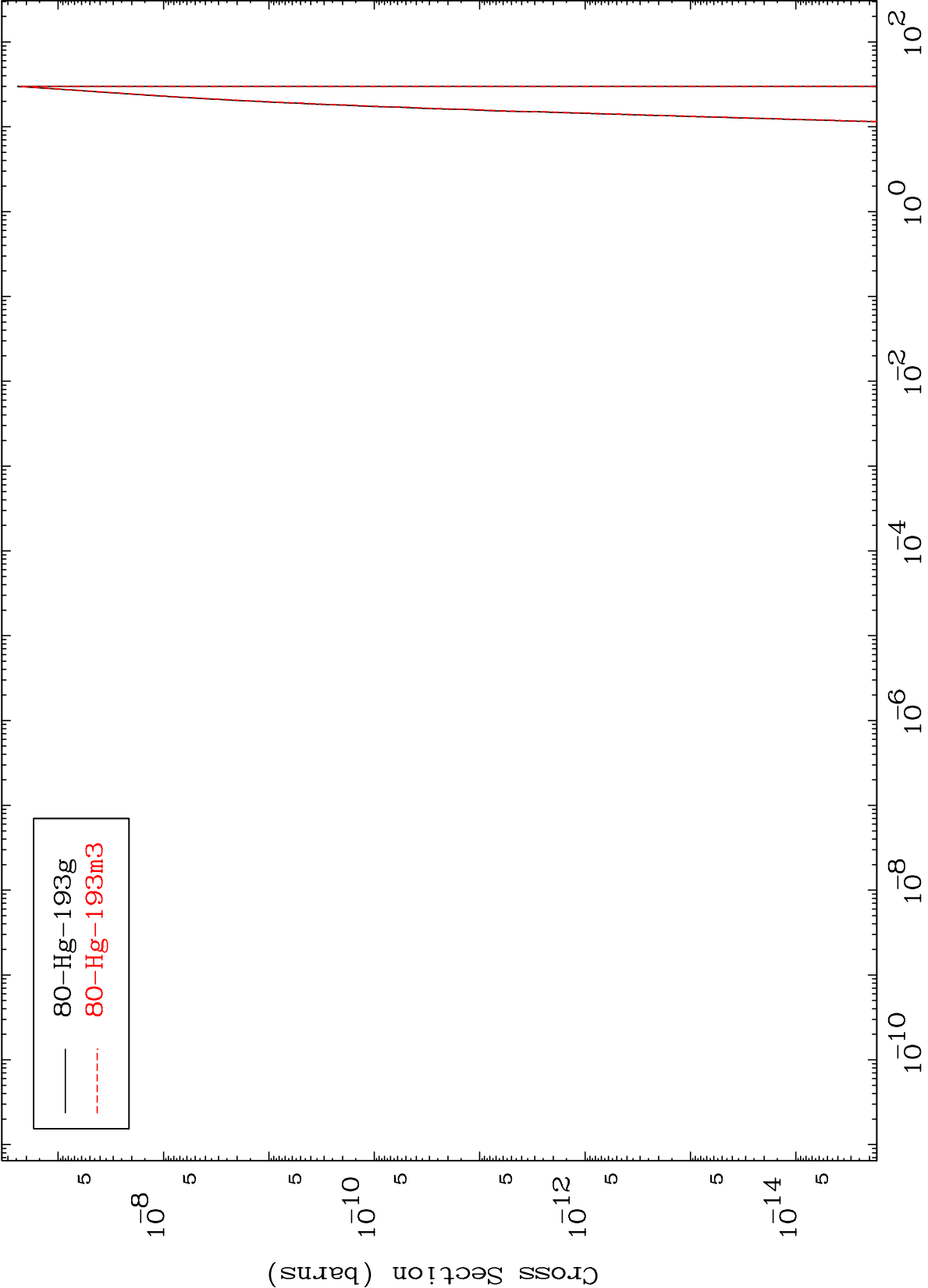
83-Bi-198n

MAT 8294

(n,d) α

83-Bi-198n

Radionuclide Production Cross Section



45

Incident Energy (MeV)

83-Bi-198n