

Program EVALPLOT
(Version 2021-1)

by

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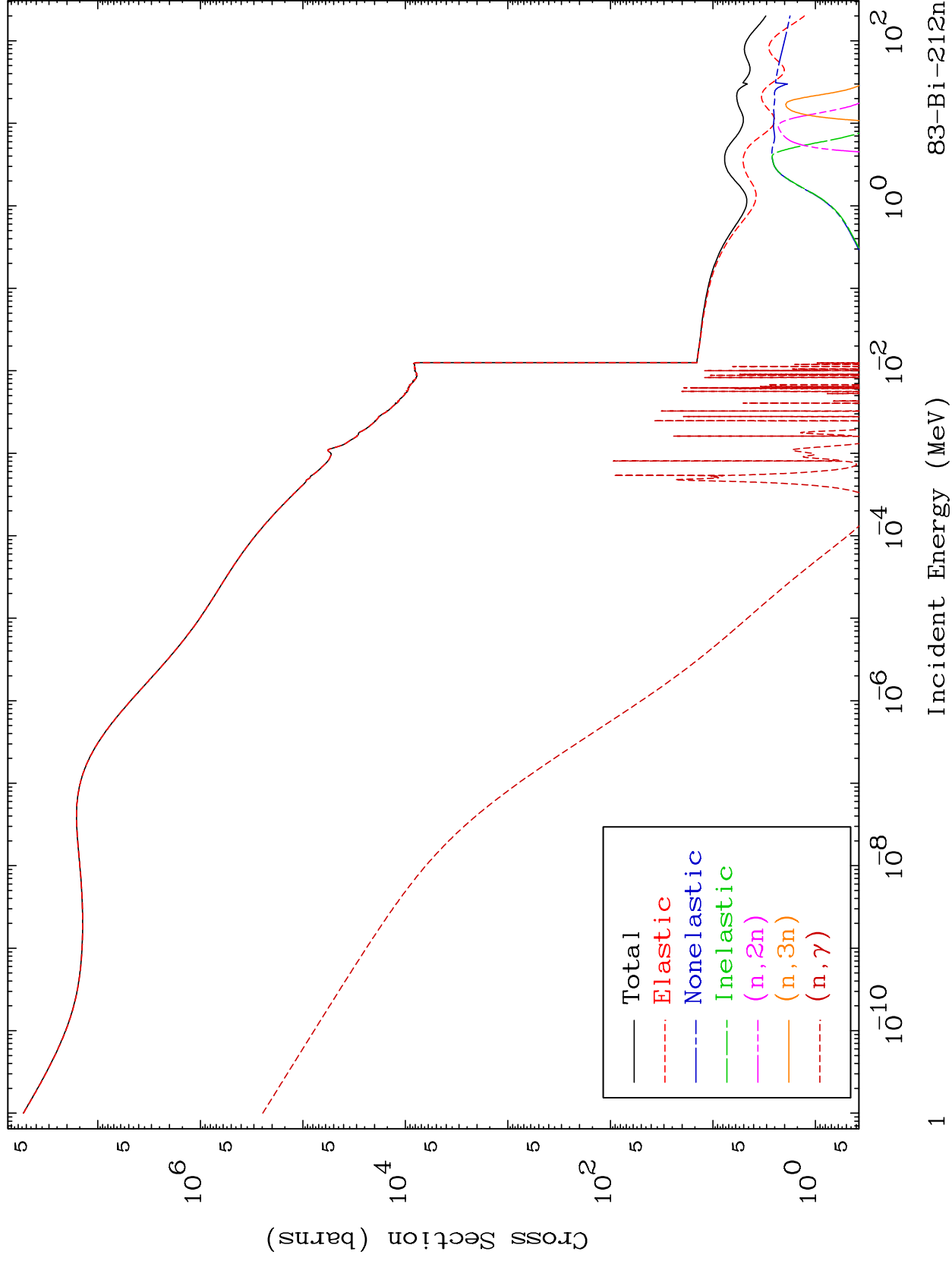
Press Mouse Button to Start

MAT 8336

Neutron Major

 $^{83}\text{Bi}-^{212}\text{n}$

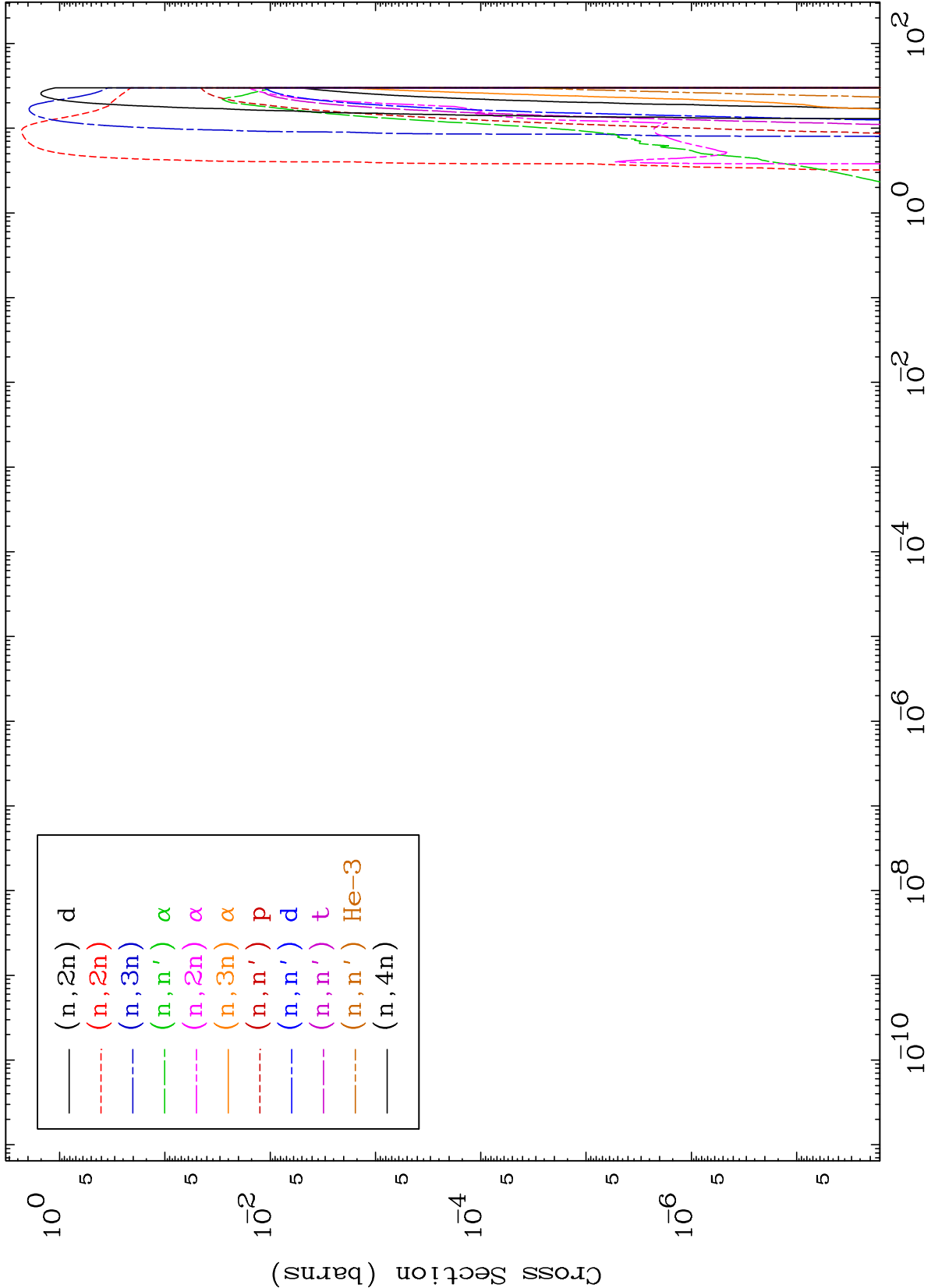
293 Kelvin Cross Sections

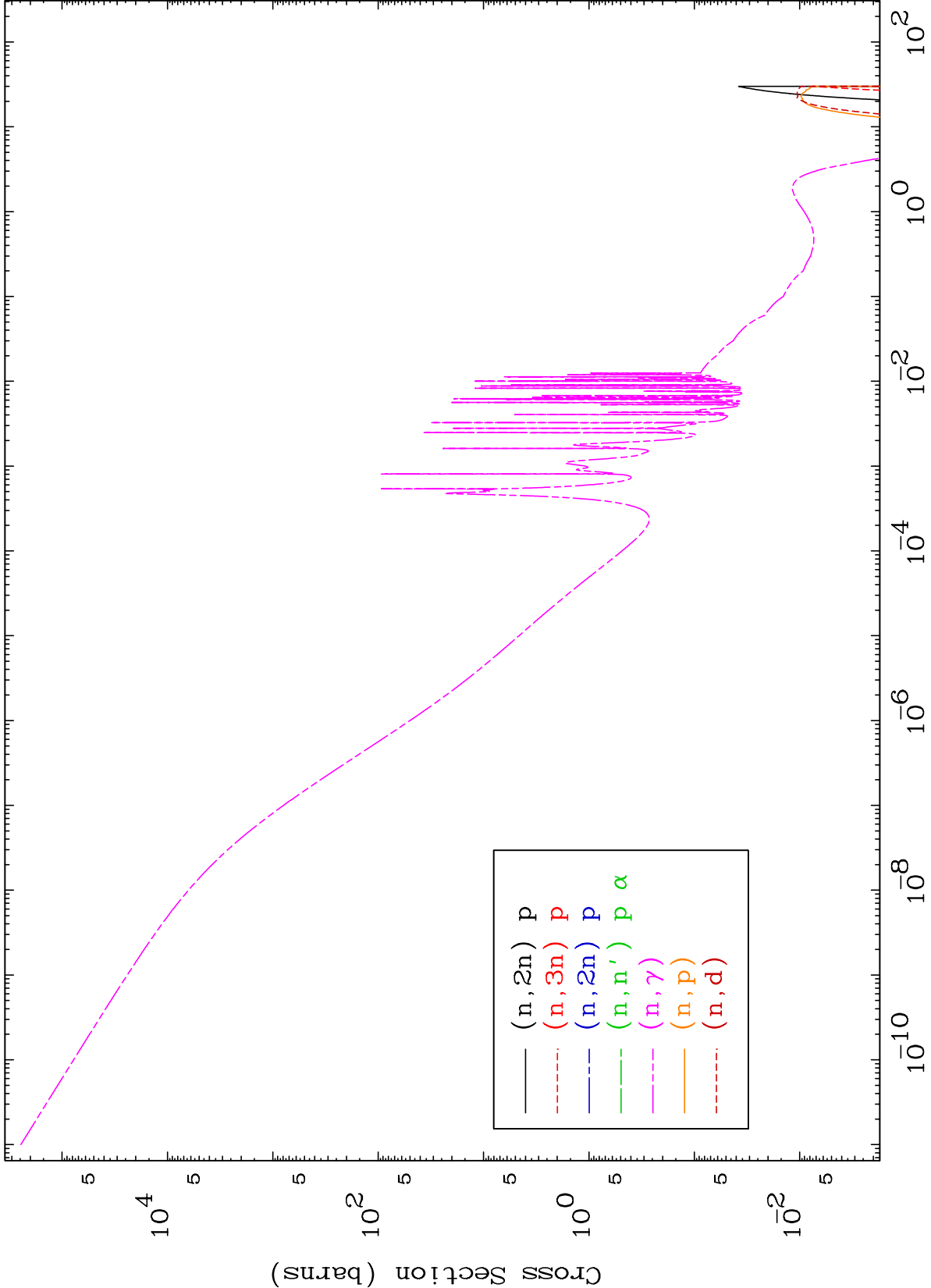


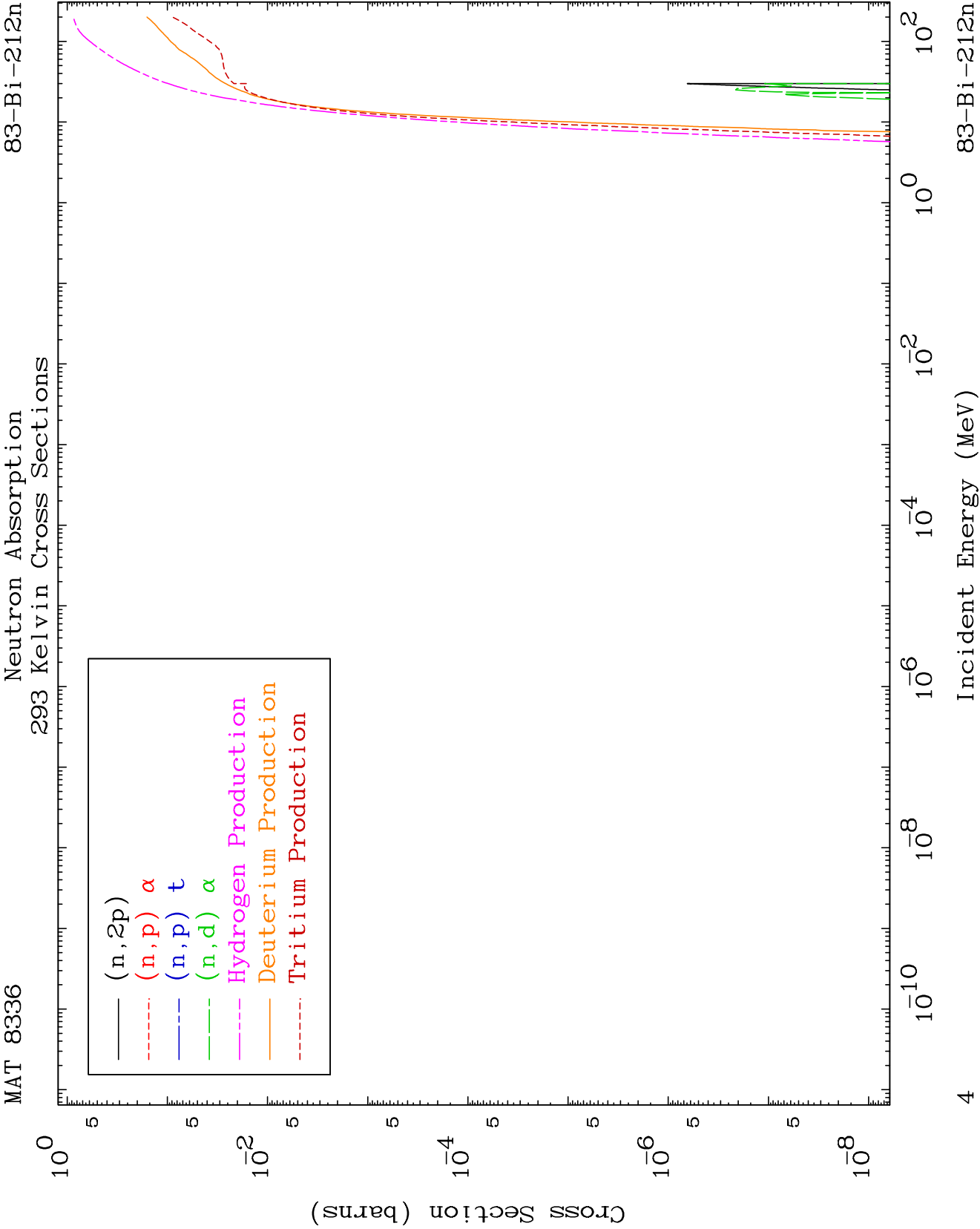
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Neutron Absorption
293 Kelvin Cross Sections

83-Bi-212n



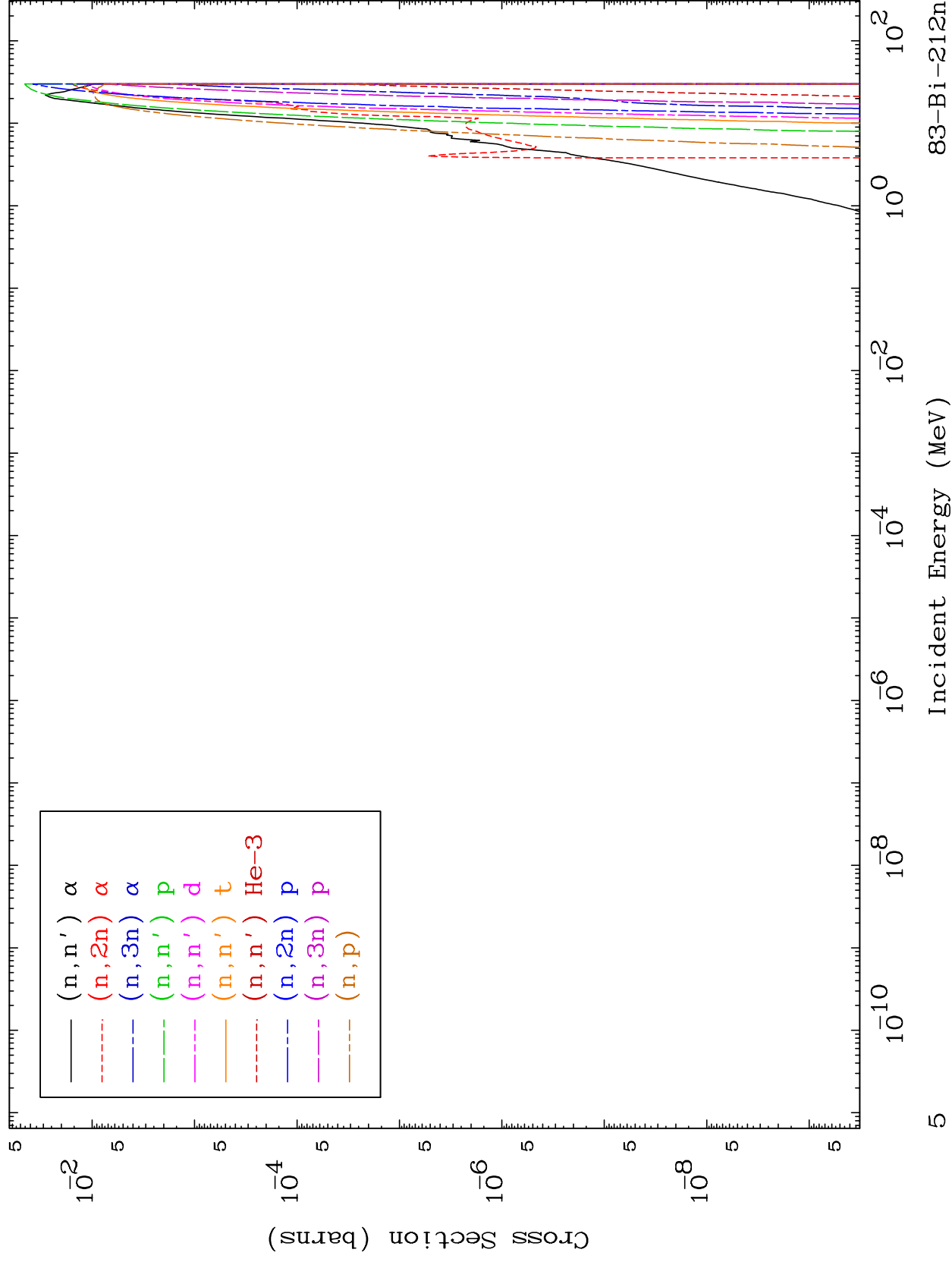




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Charged Particle
293 Kelvin Cross Sections

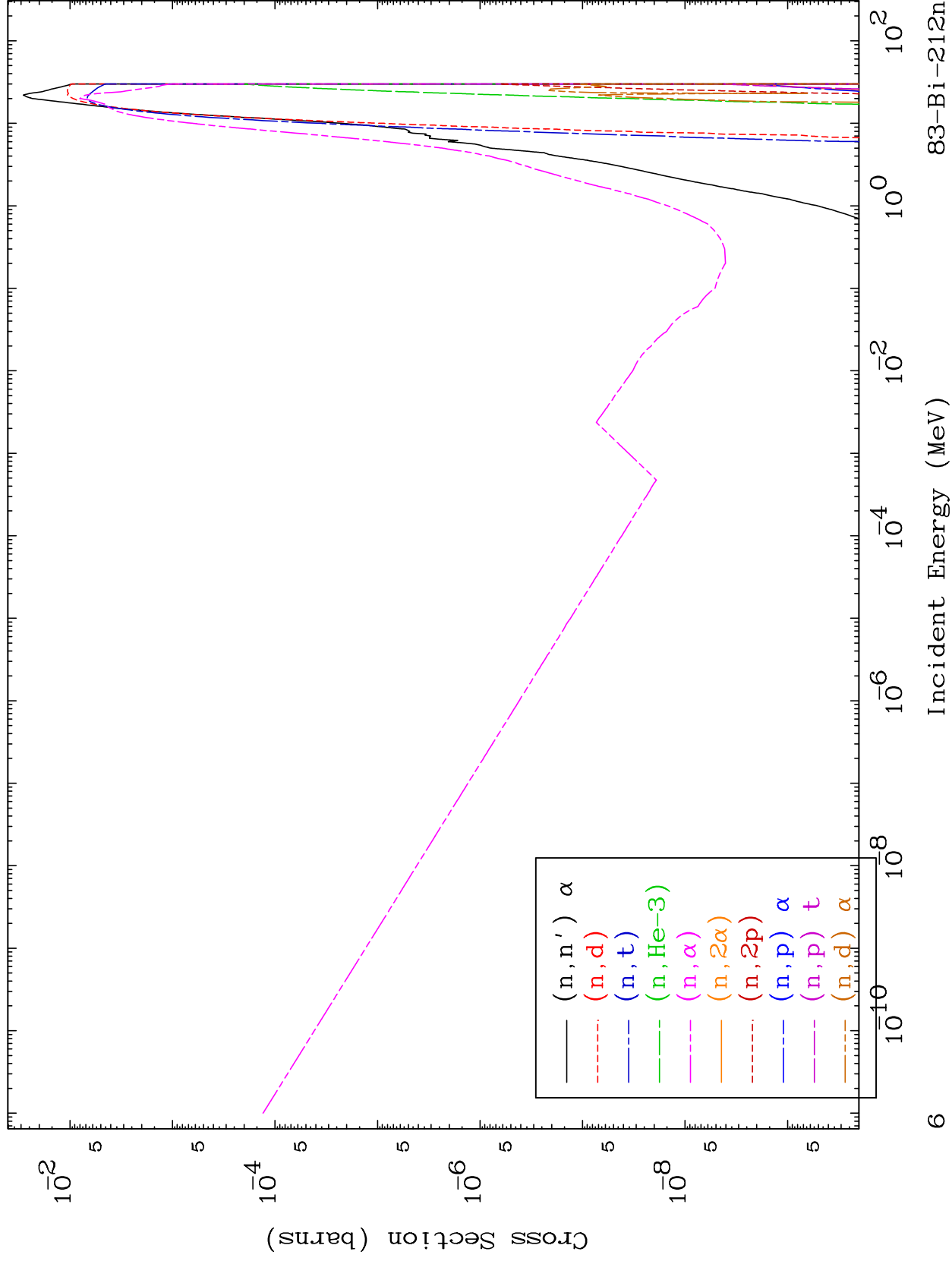
83-Bi-212n

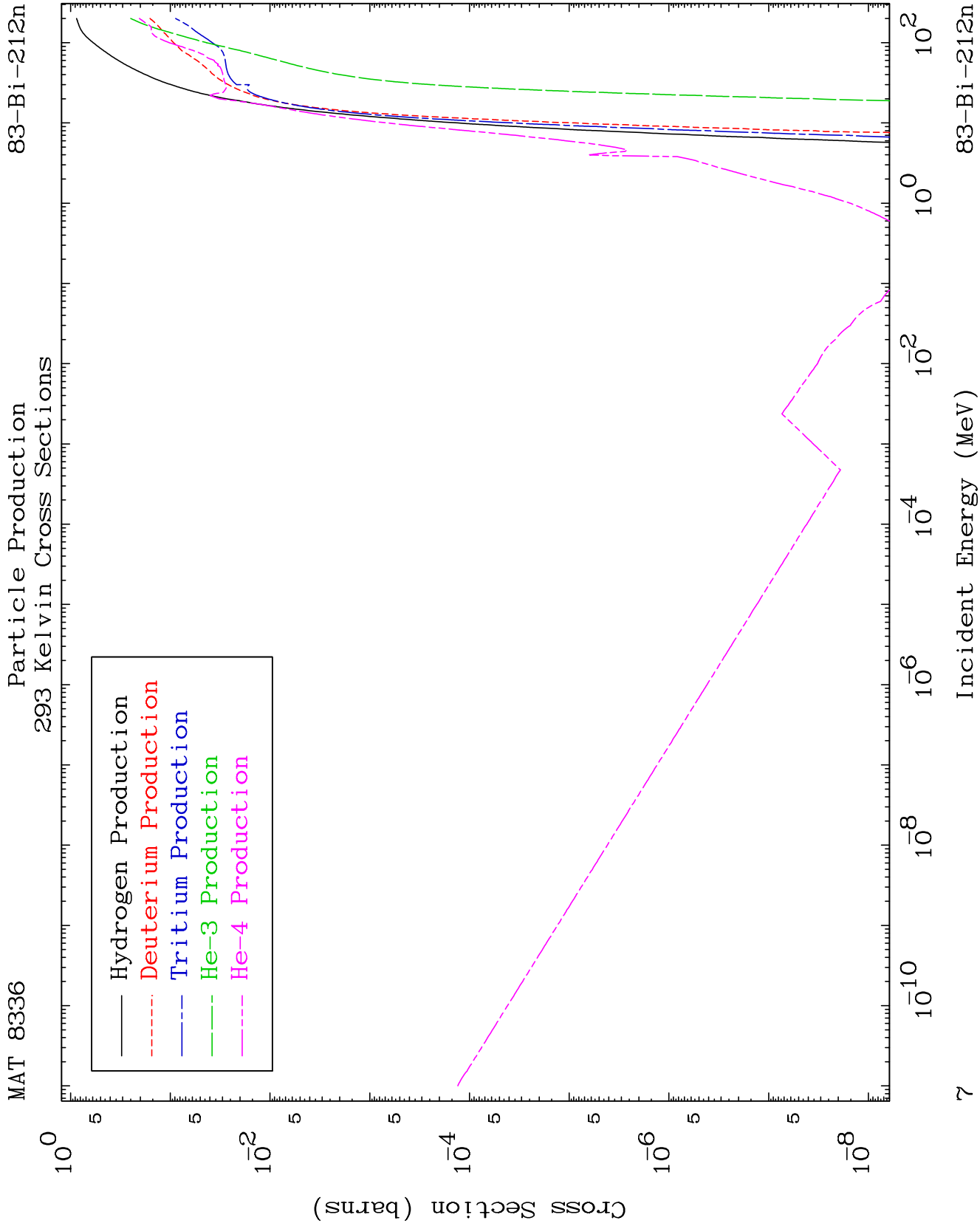


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Charged Particle
293 Kelvin Cross Sections

83-Bi-212n

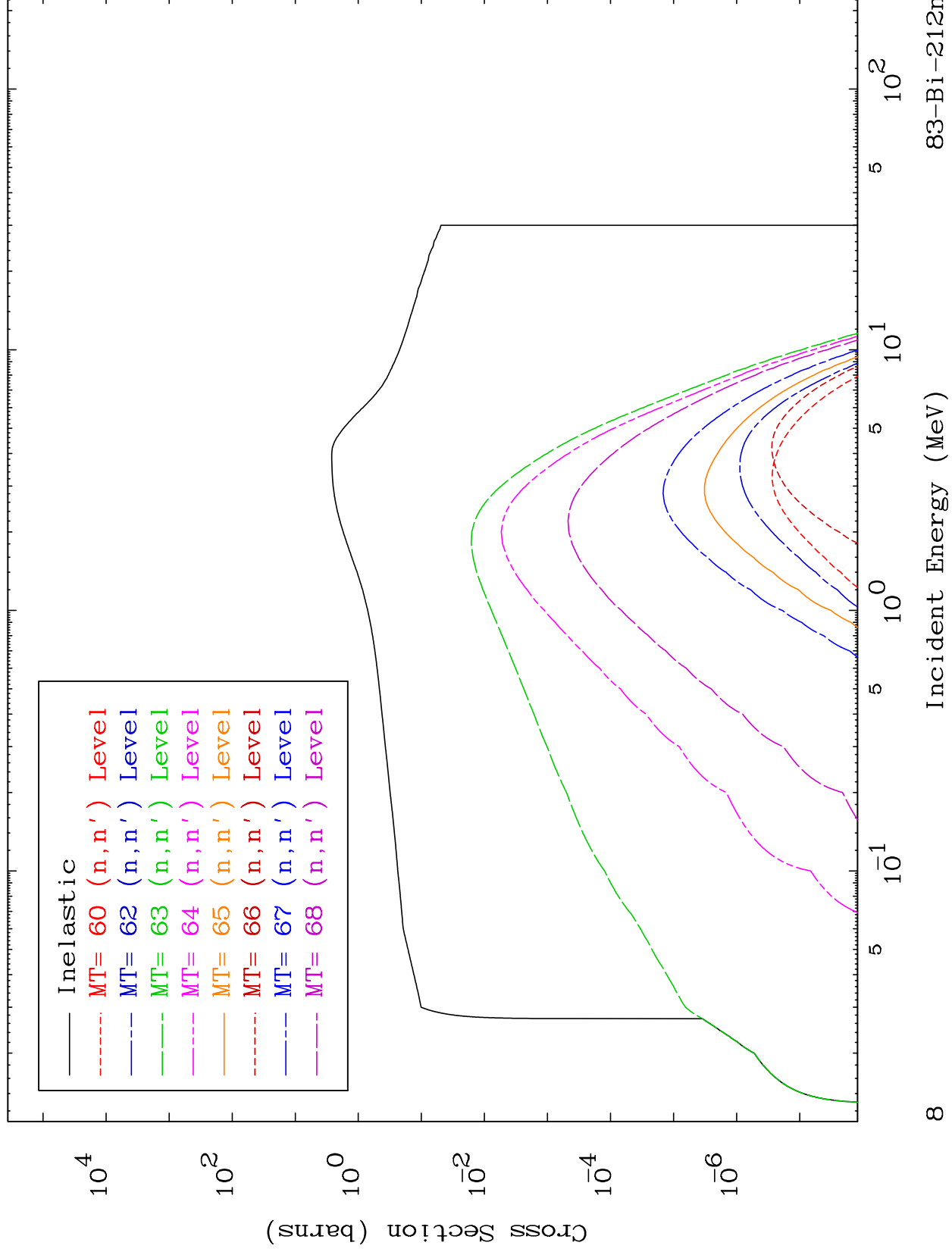




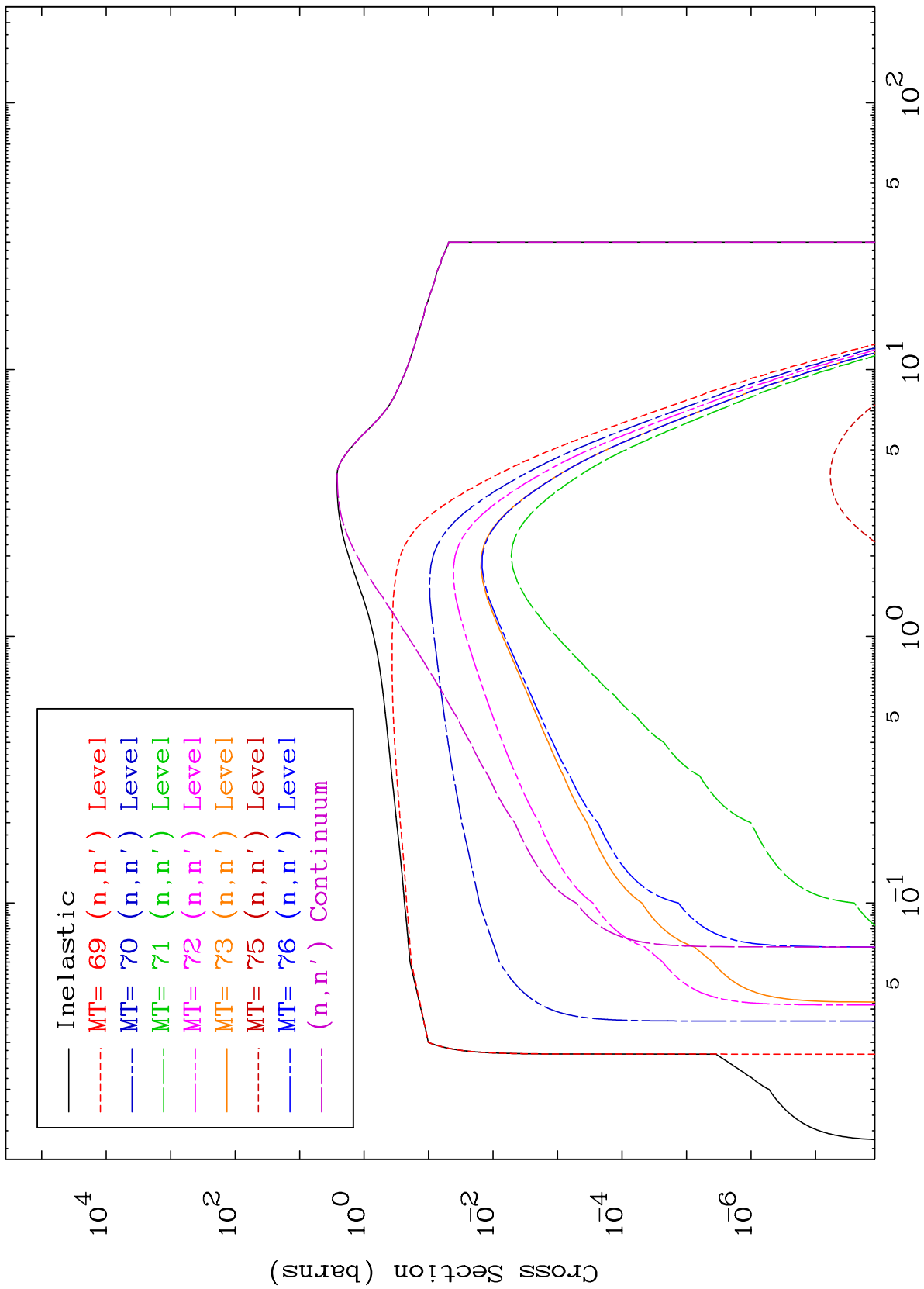
MAT 8336

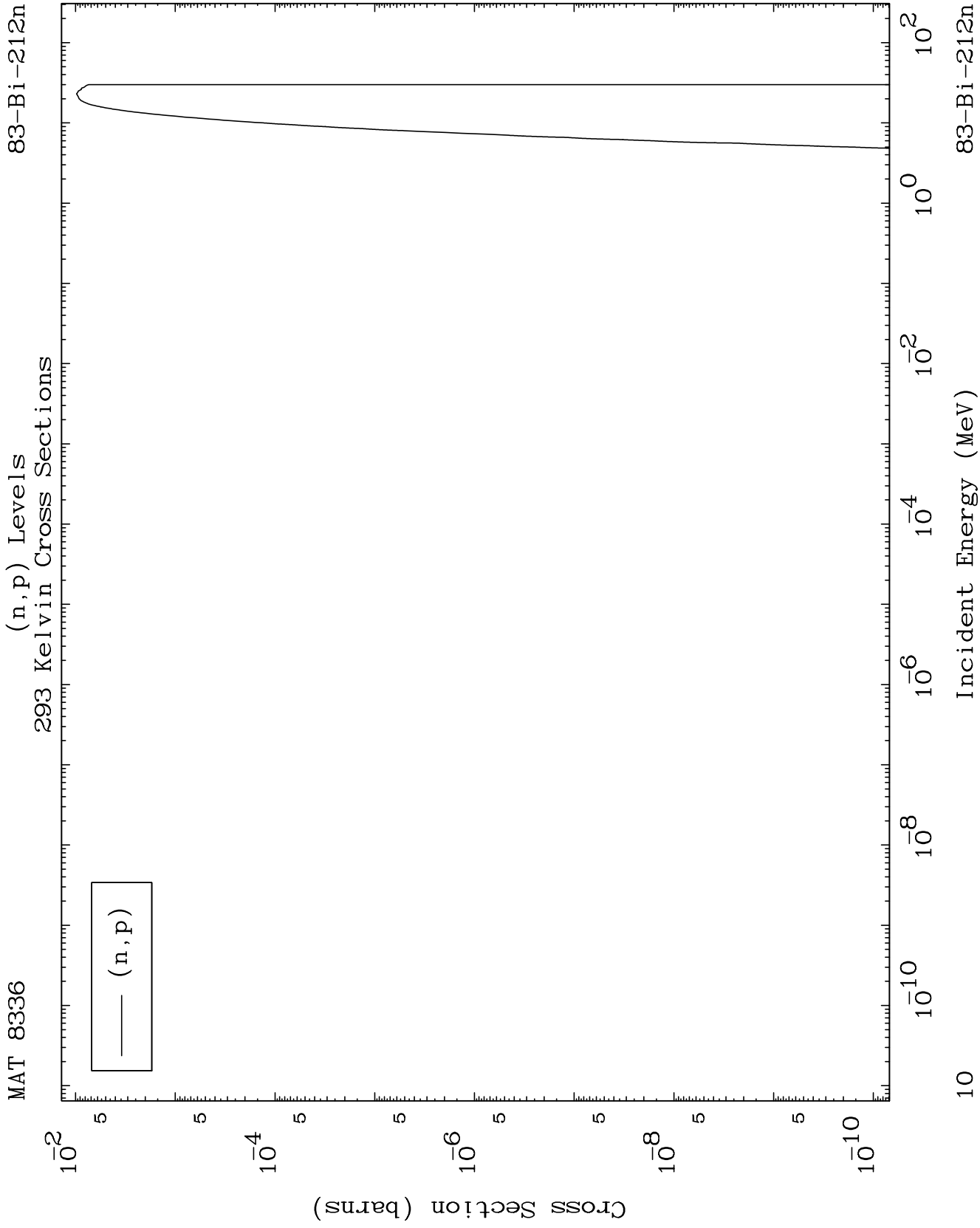
(n,n') Levels
293 Kelvin Cross Sections

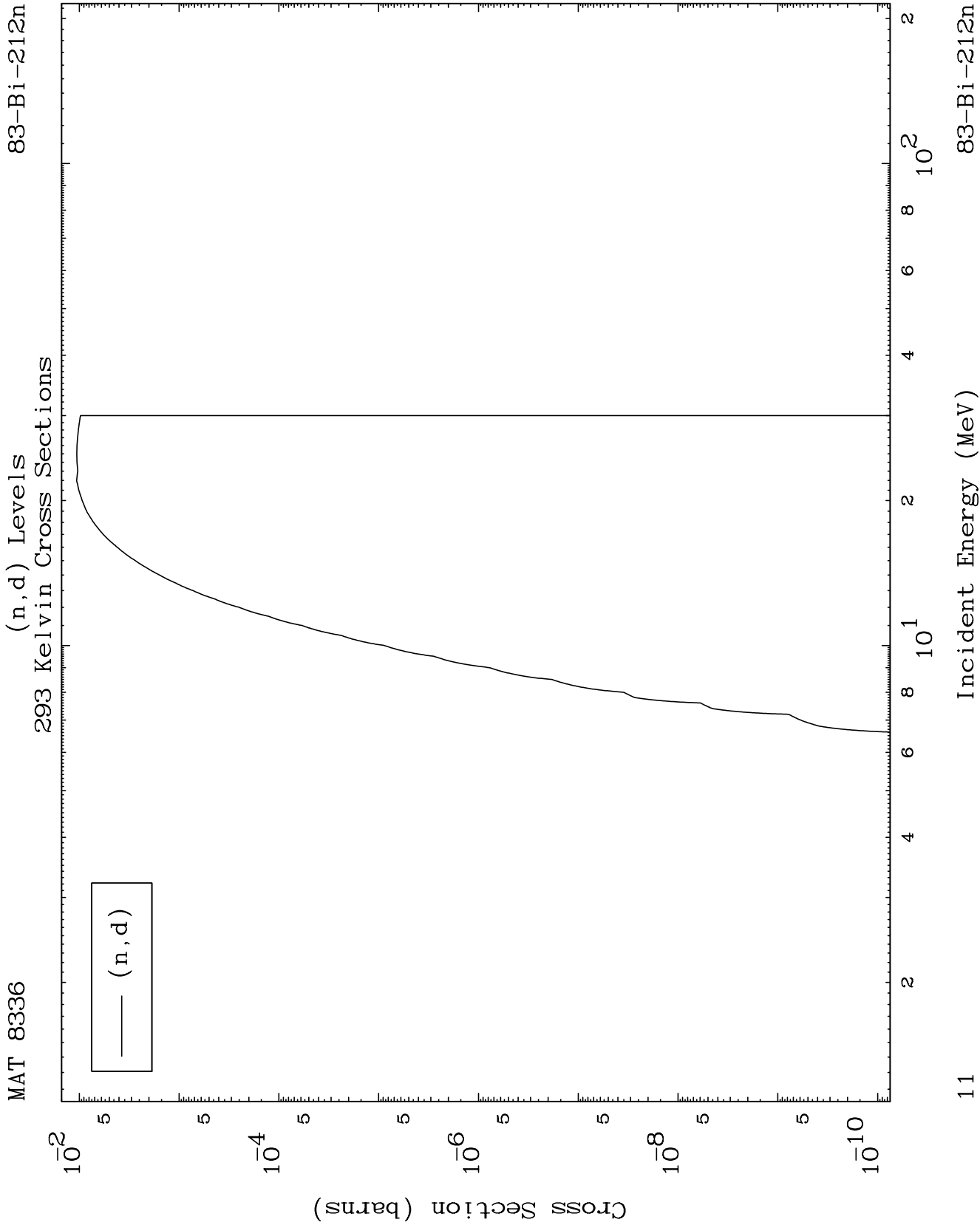
83-Bi-212n



(n,n') Levels
293 Kelvin Cross Sections



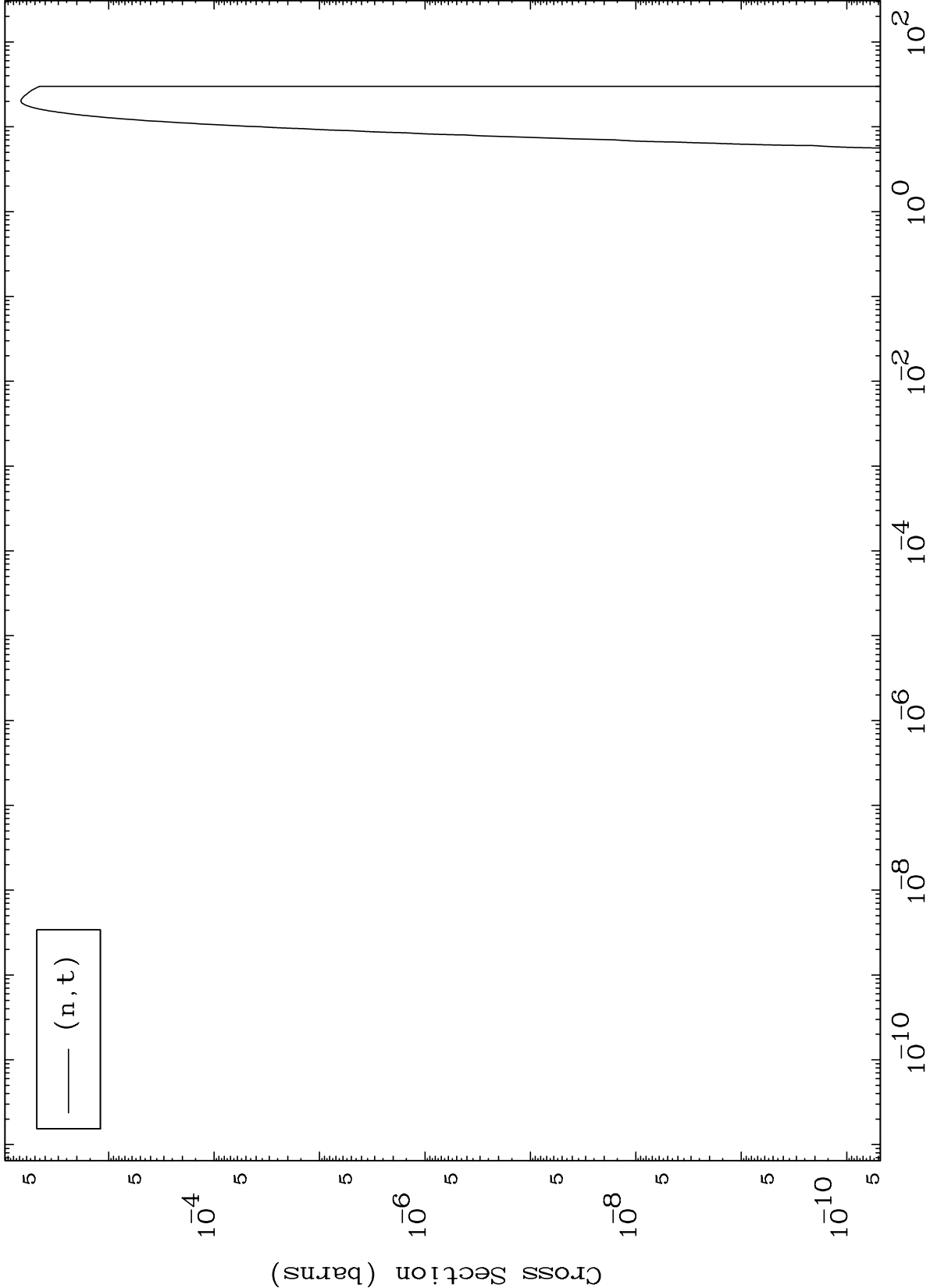




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(n,t) Levels
293 Kelvin Cross Sections

83-Bi-212n



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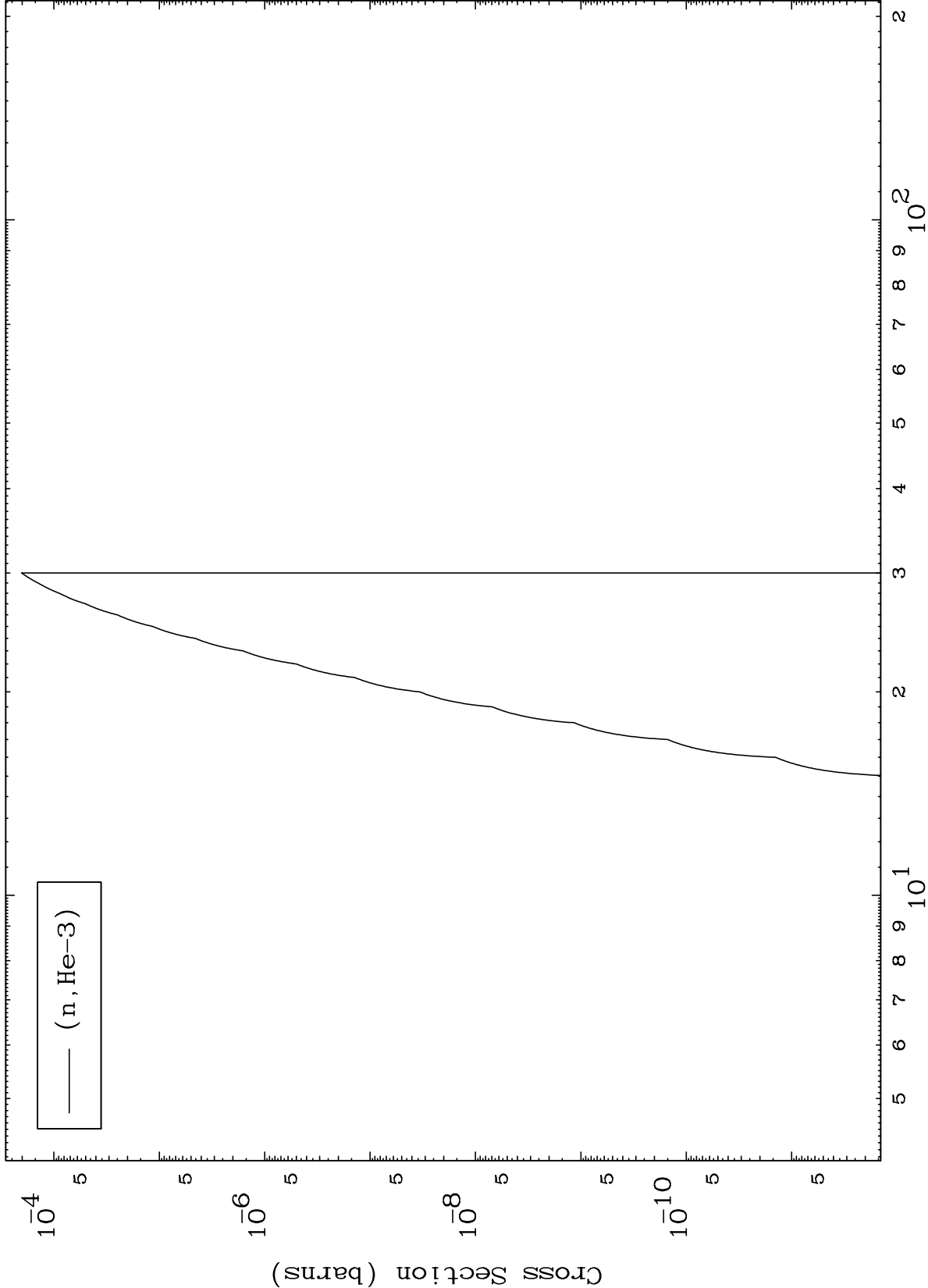
Incident Energy (MeV)

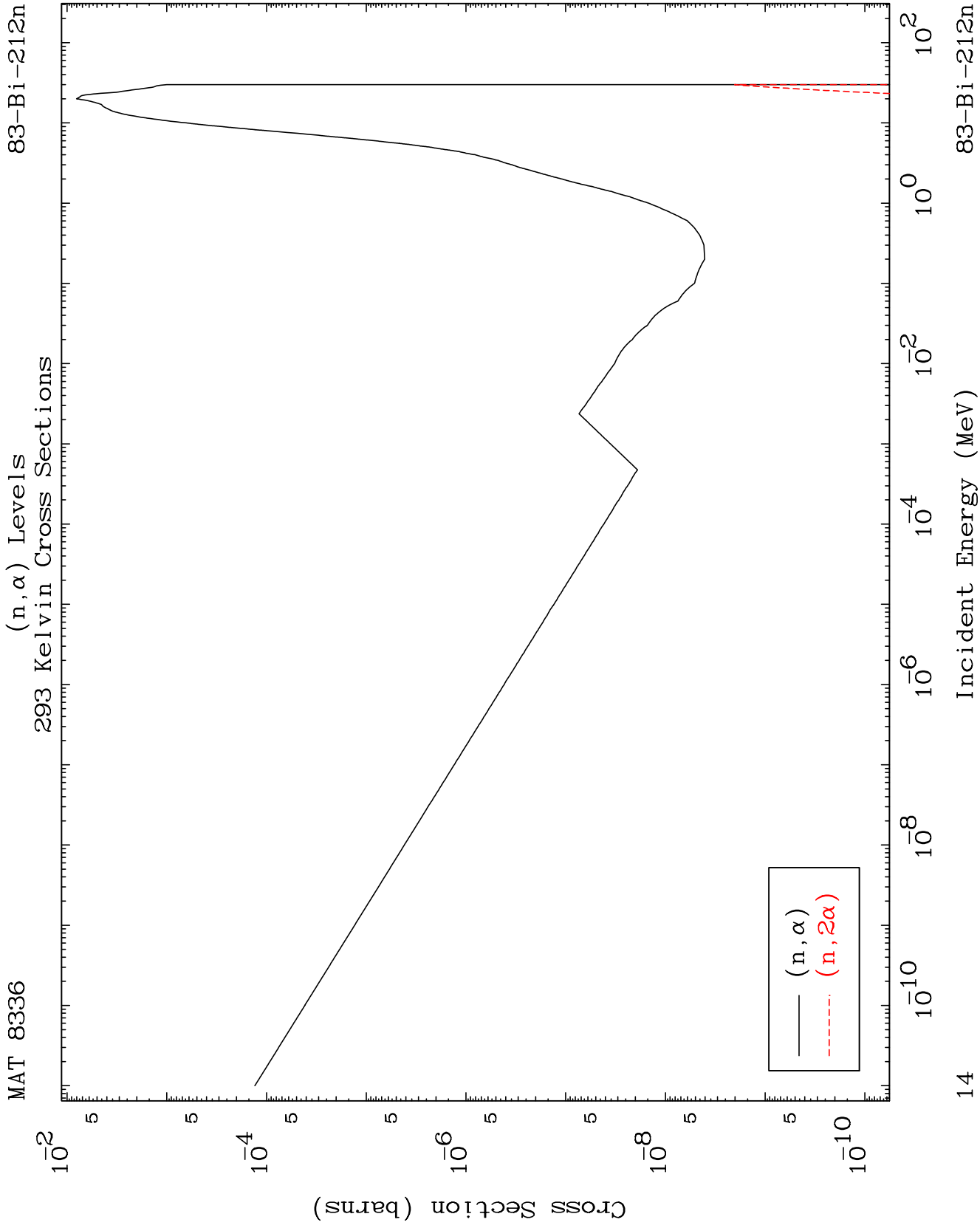
83-Bi-212n

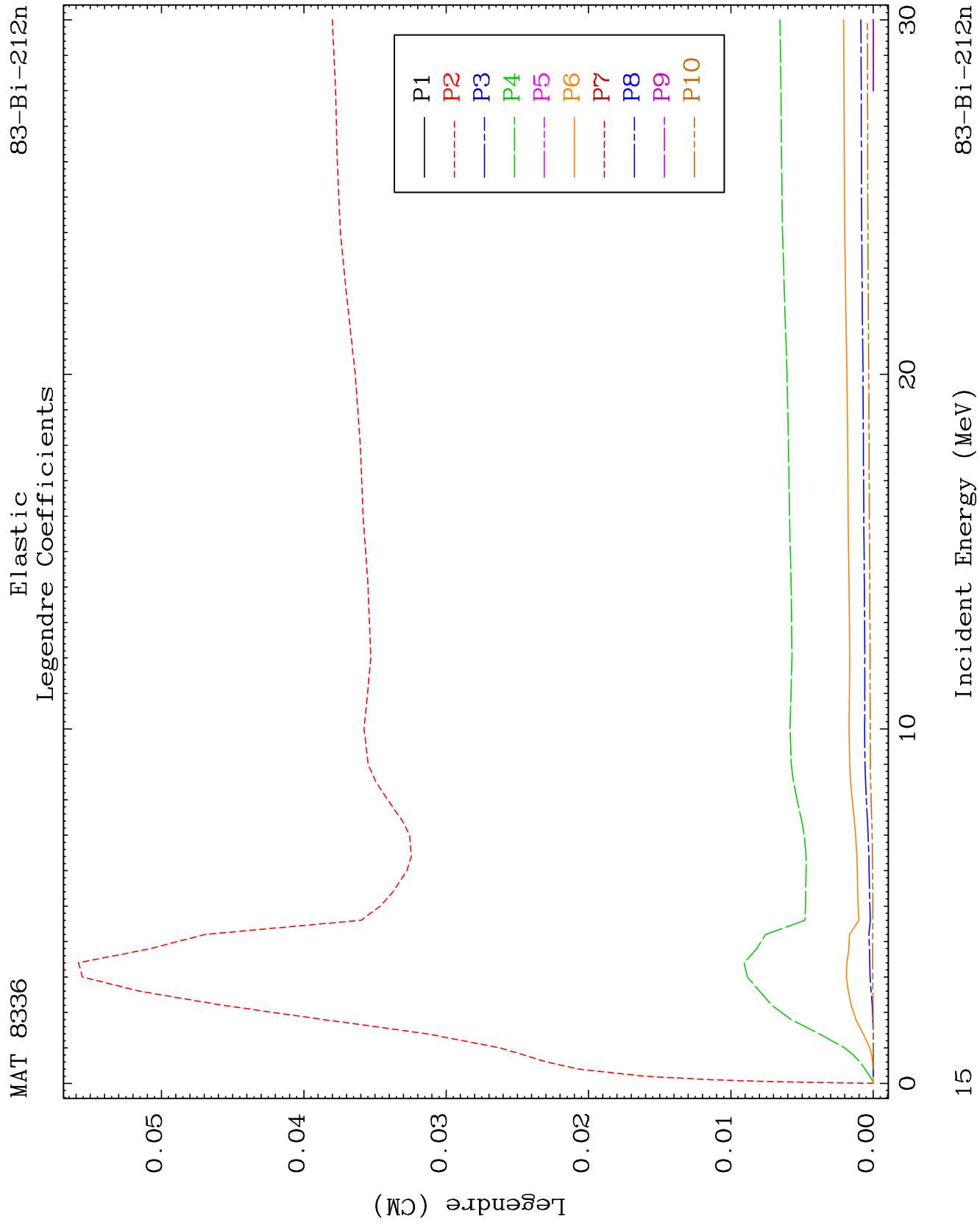
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(n,He3) Levels
293 Kelvin Cross Sections

83-Bi-212n

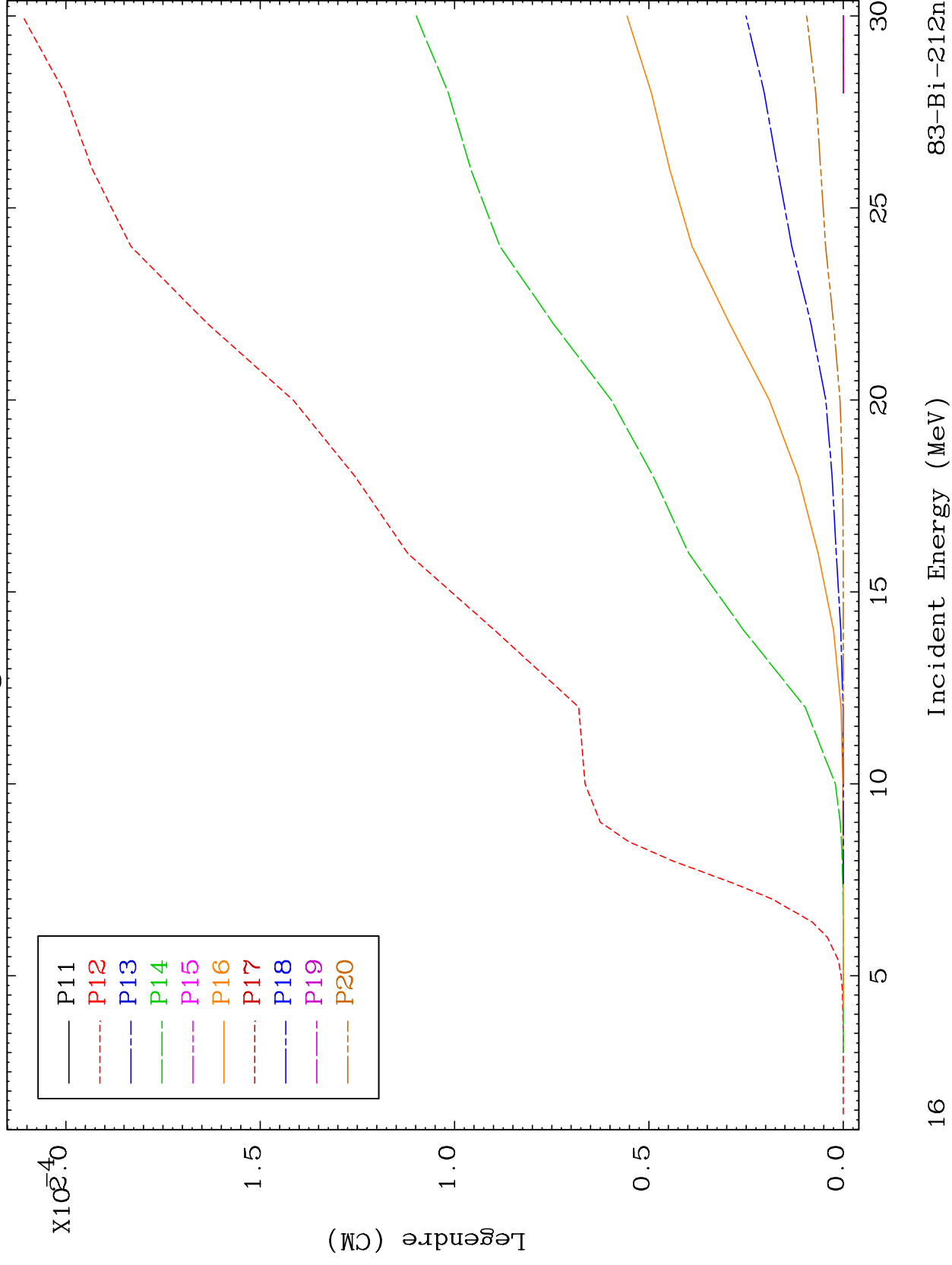


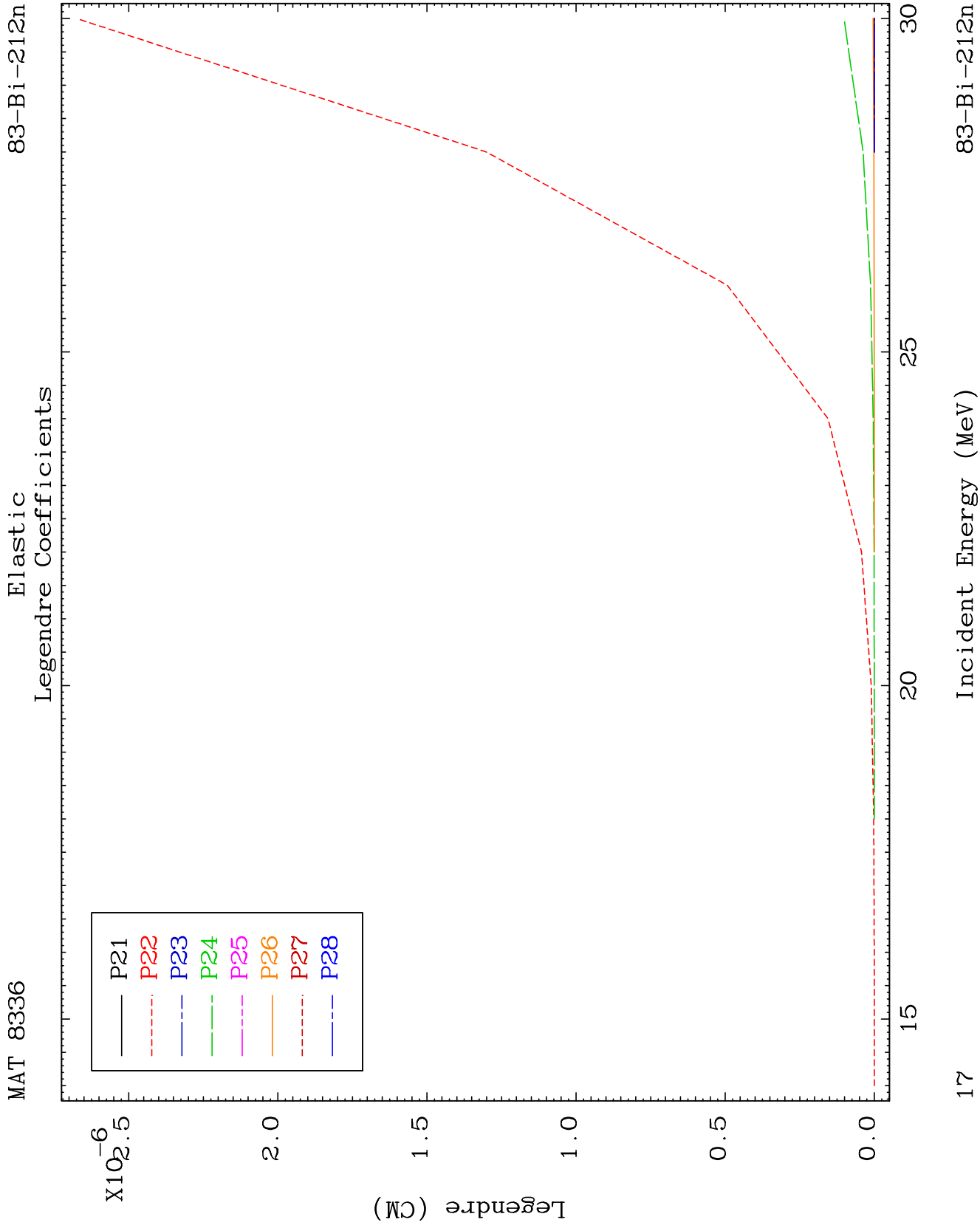




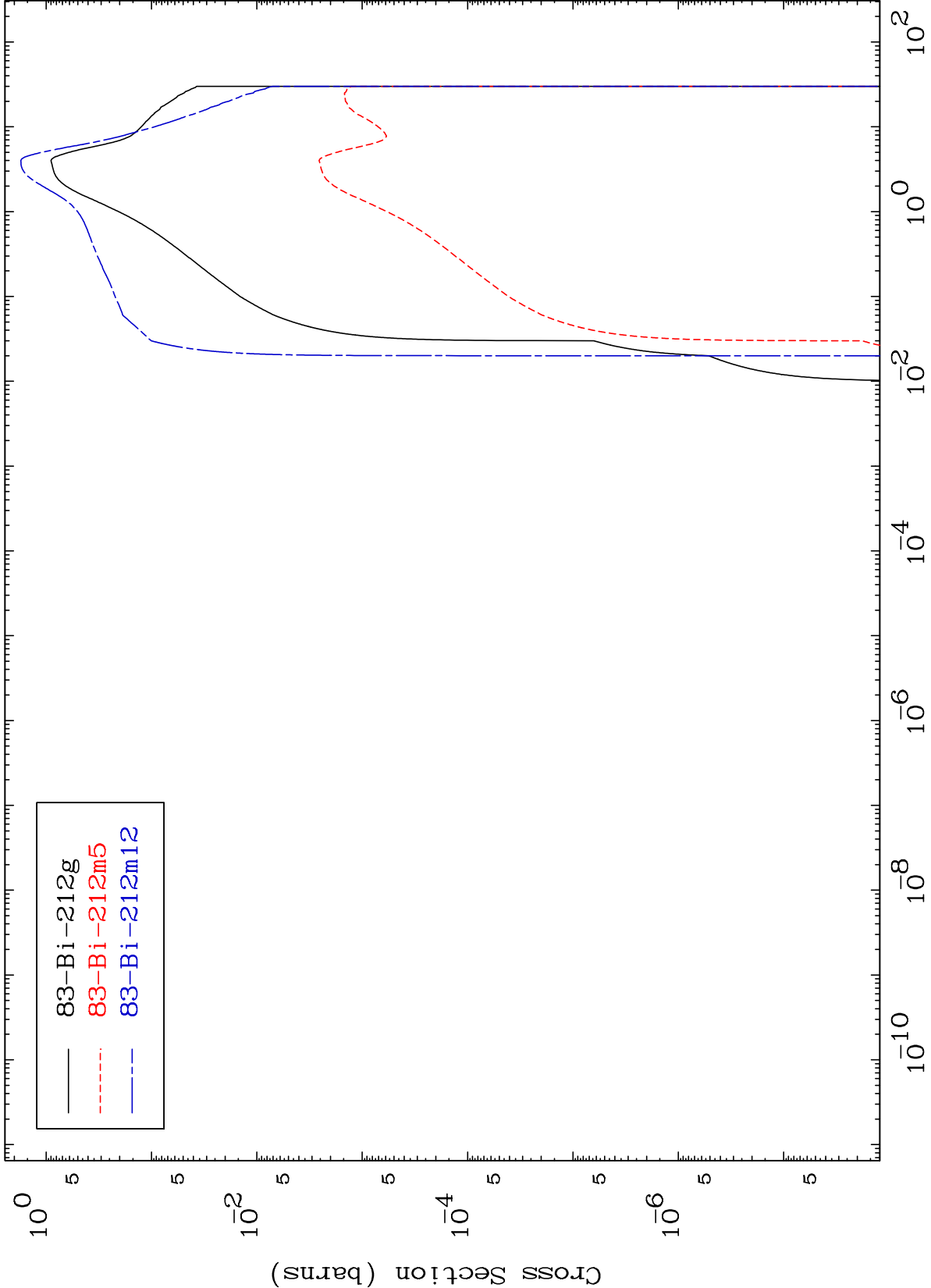
MAT 8336

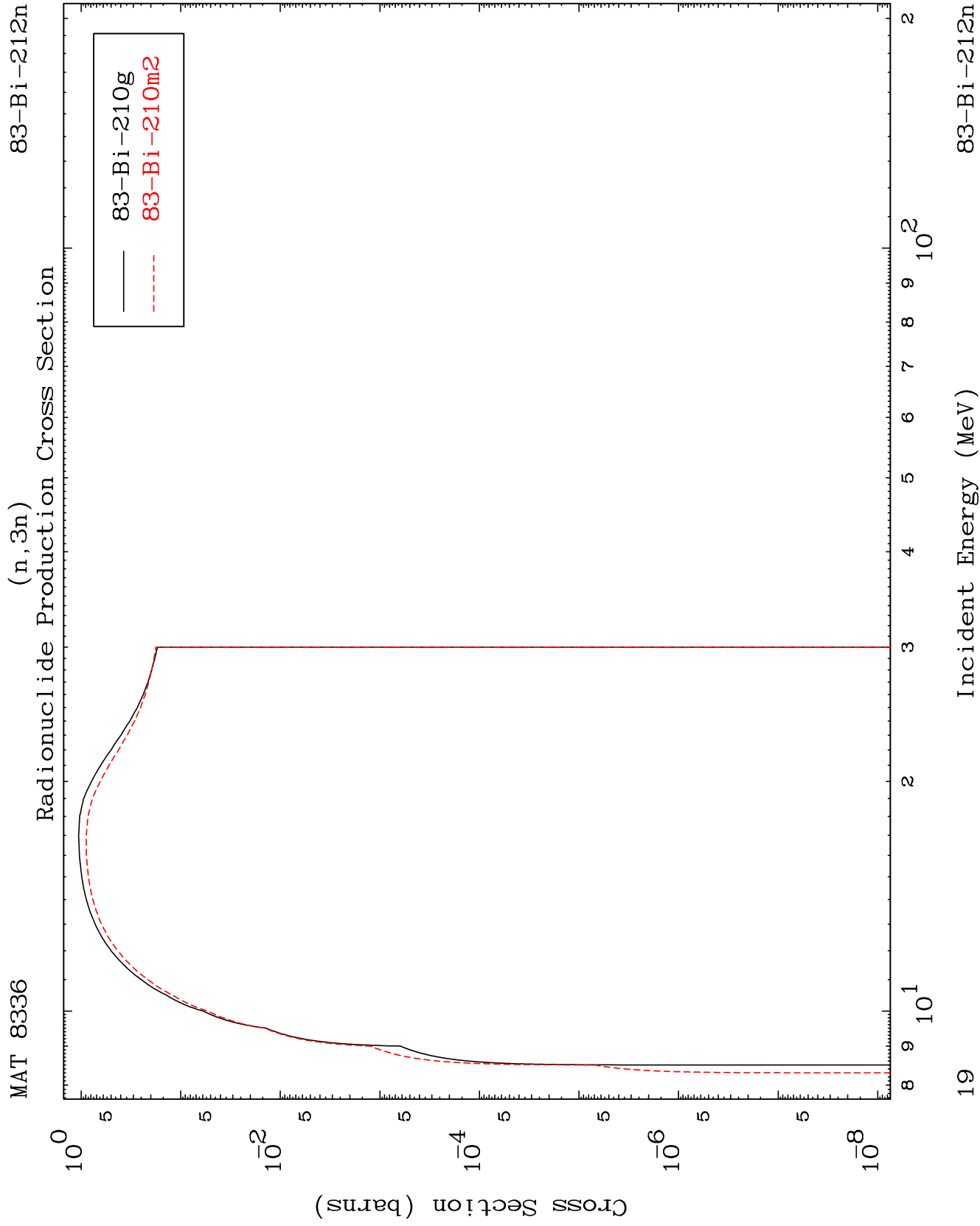
Elastic Legendre Coefficients

 $^{83}\text{Bi}-^{212}\text{n}$ 



Inelastic
Radionuclide Production Cross Section





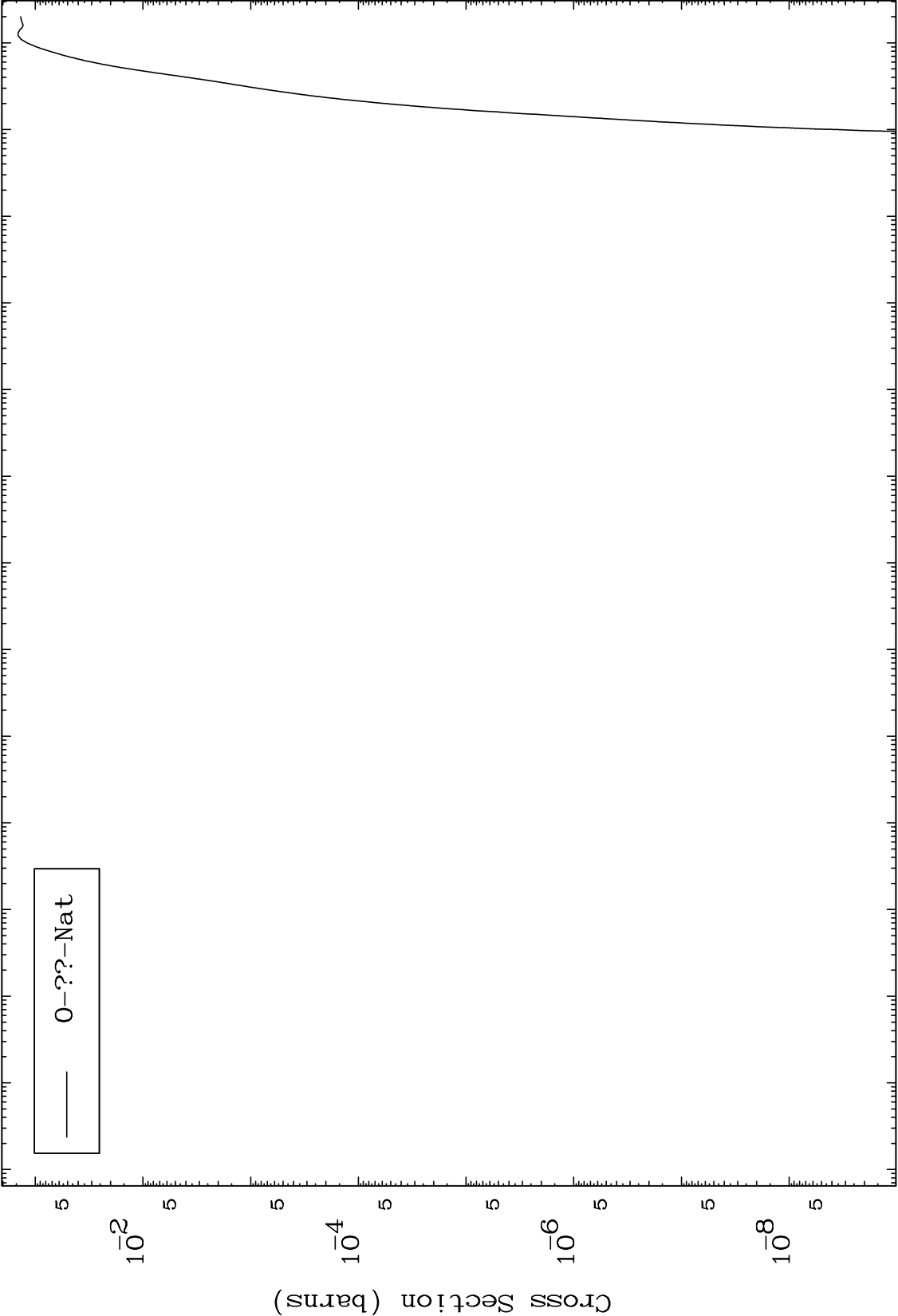
MAT 8336

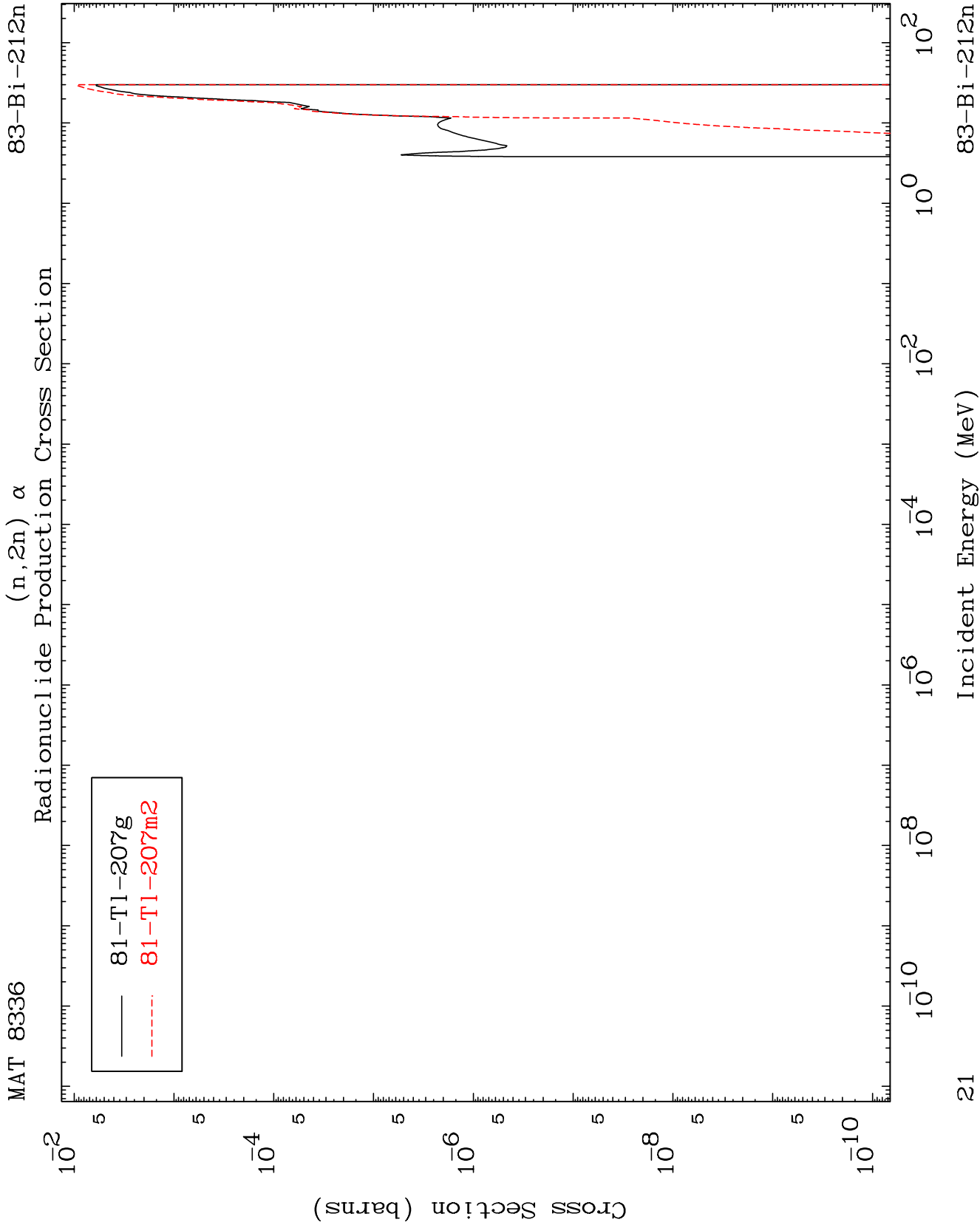
Fission

83-Bi-212n

Radionuclide Production Cross Section

— 0-??-Nat



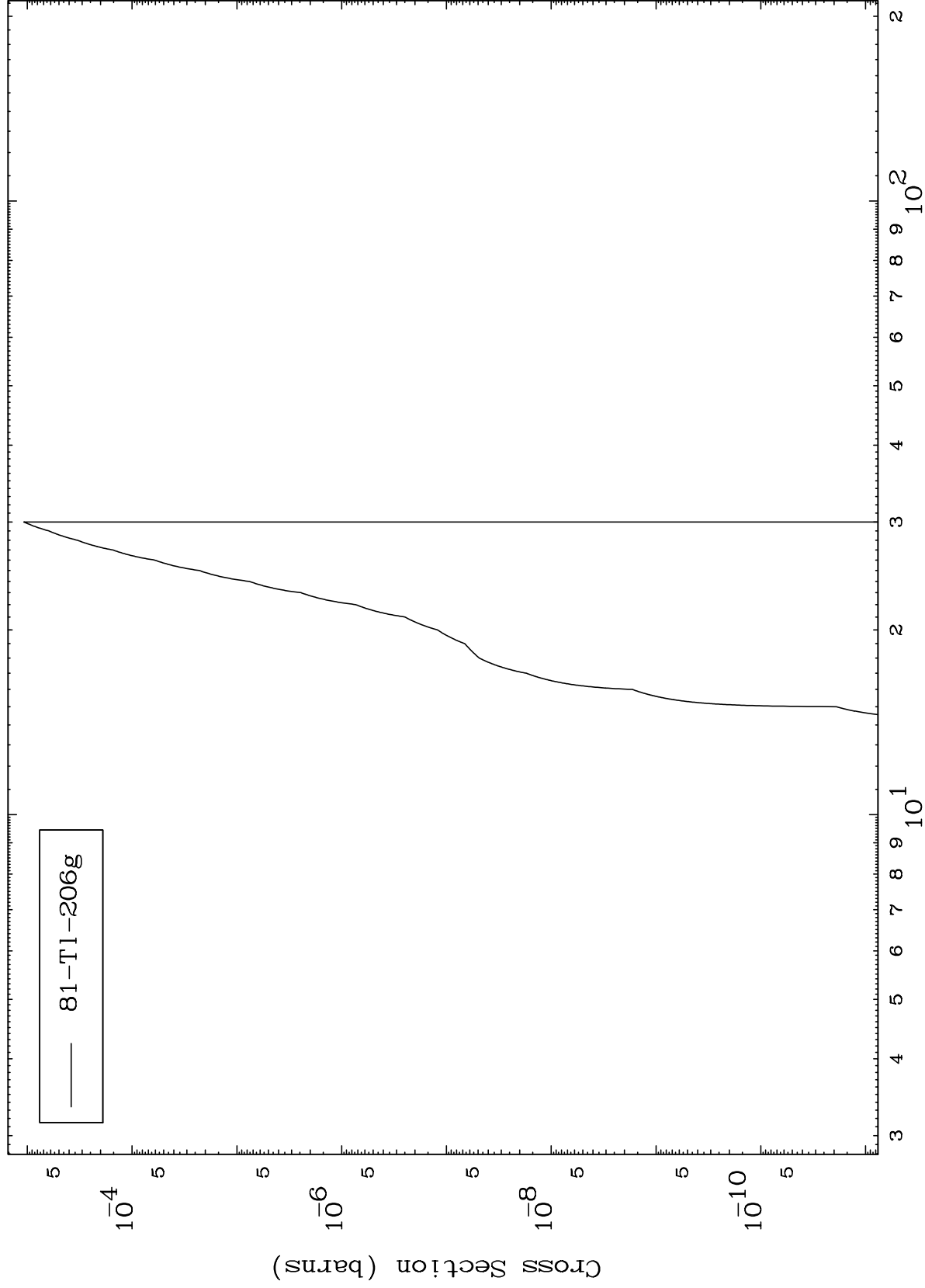


MAT 8336

83-Bi-212n

(n,3n) α

Radionuclide Production Cross Section



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Incident Energy (MeV)

83-Bi-212n

MAT 8336

(n,2α)

83-Bi-212n

Radionuclide Production Cross Section

