

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

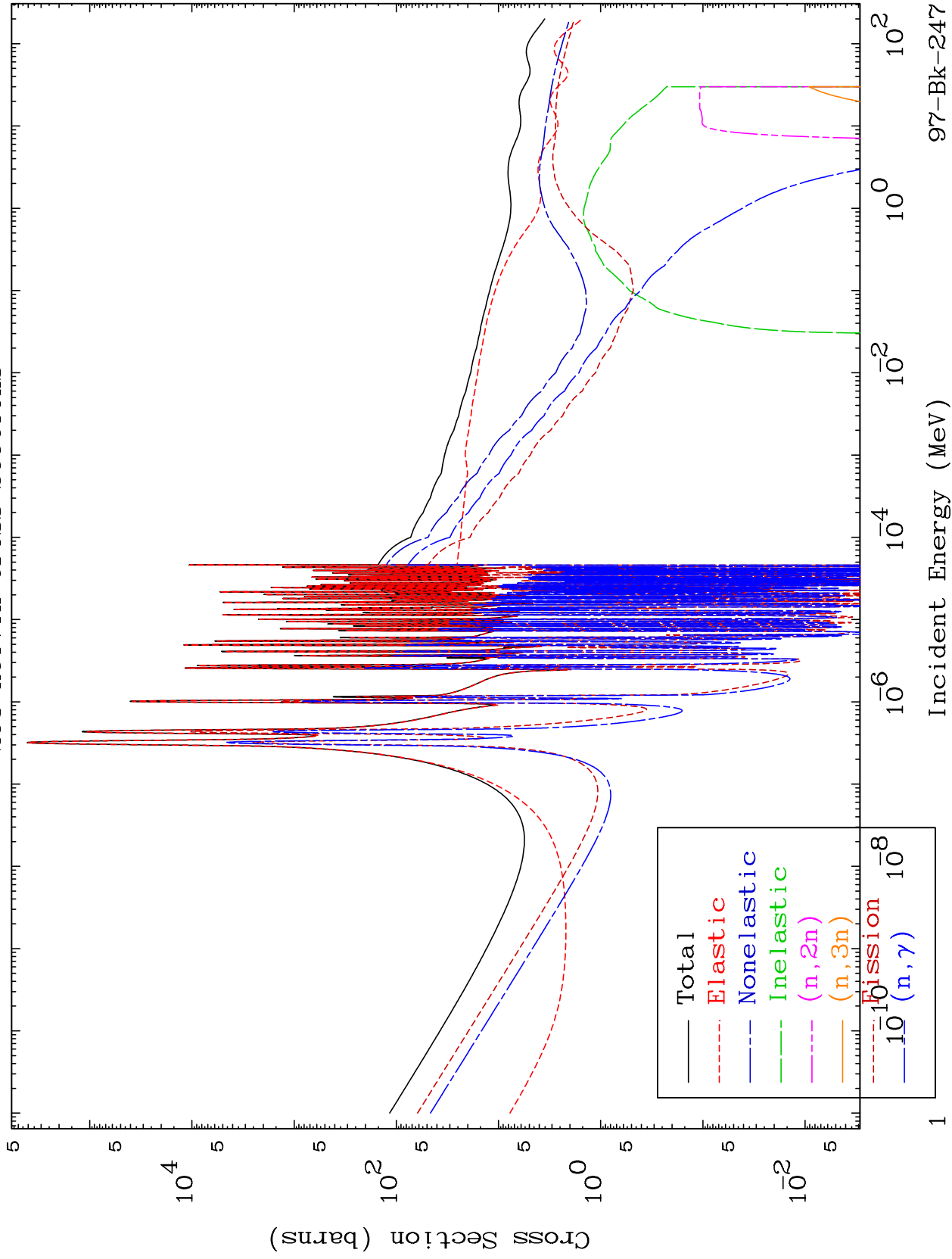
Web:redcullen1.net/HOMEPAGE.NEW

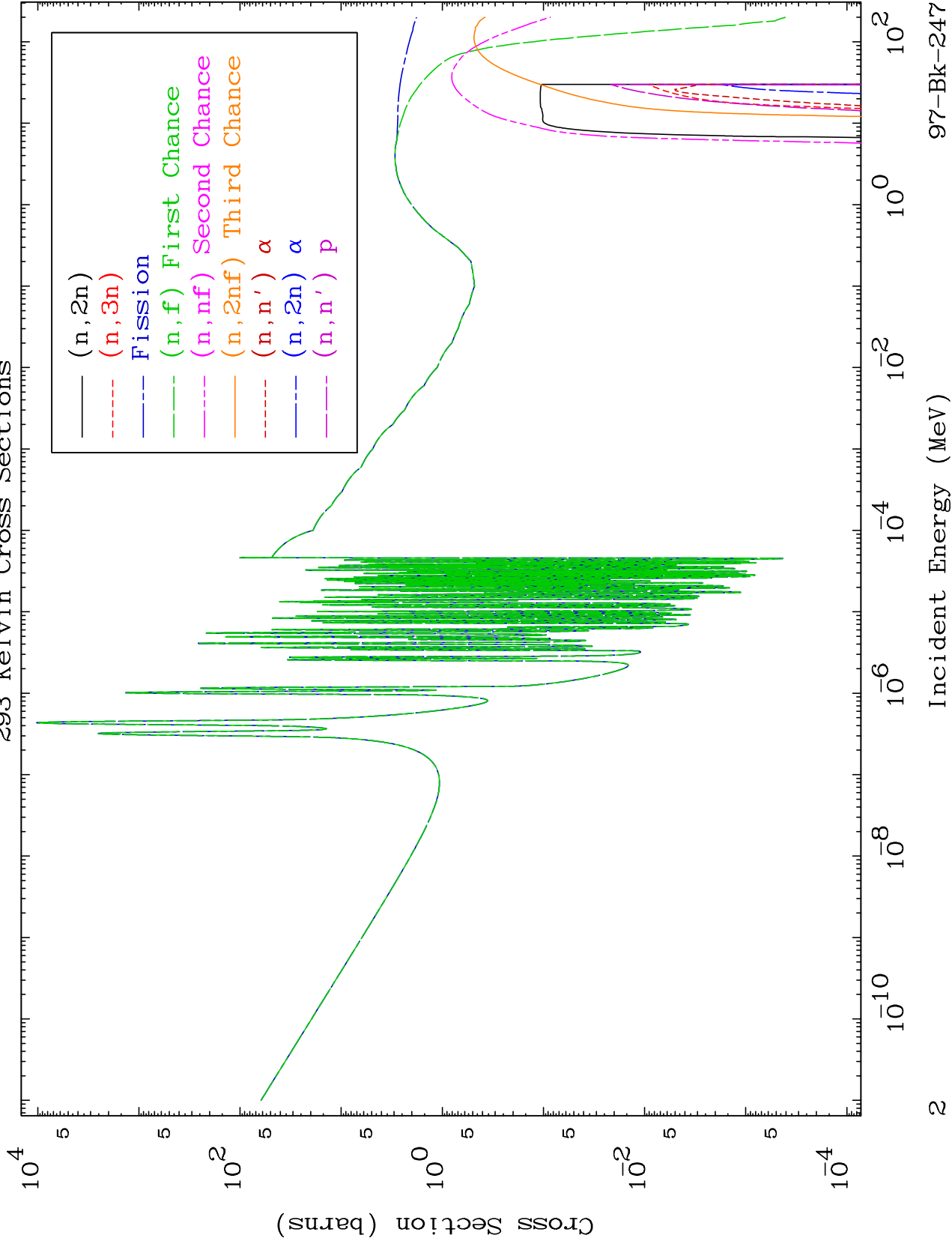
Press Mouse Button to Start

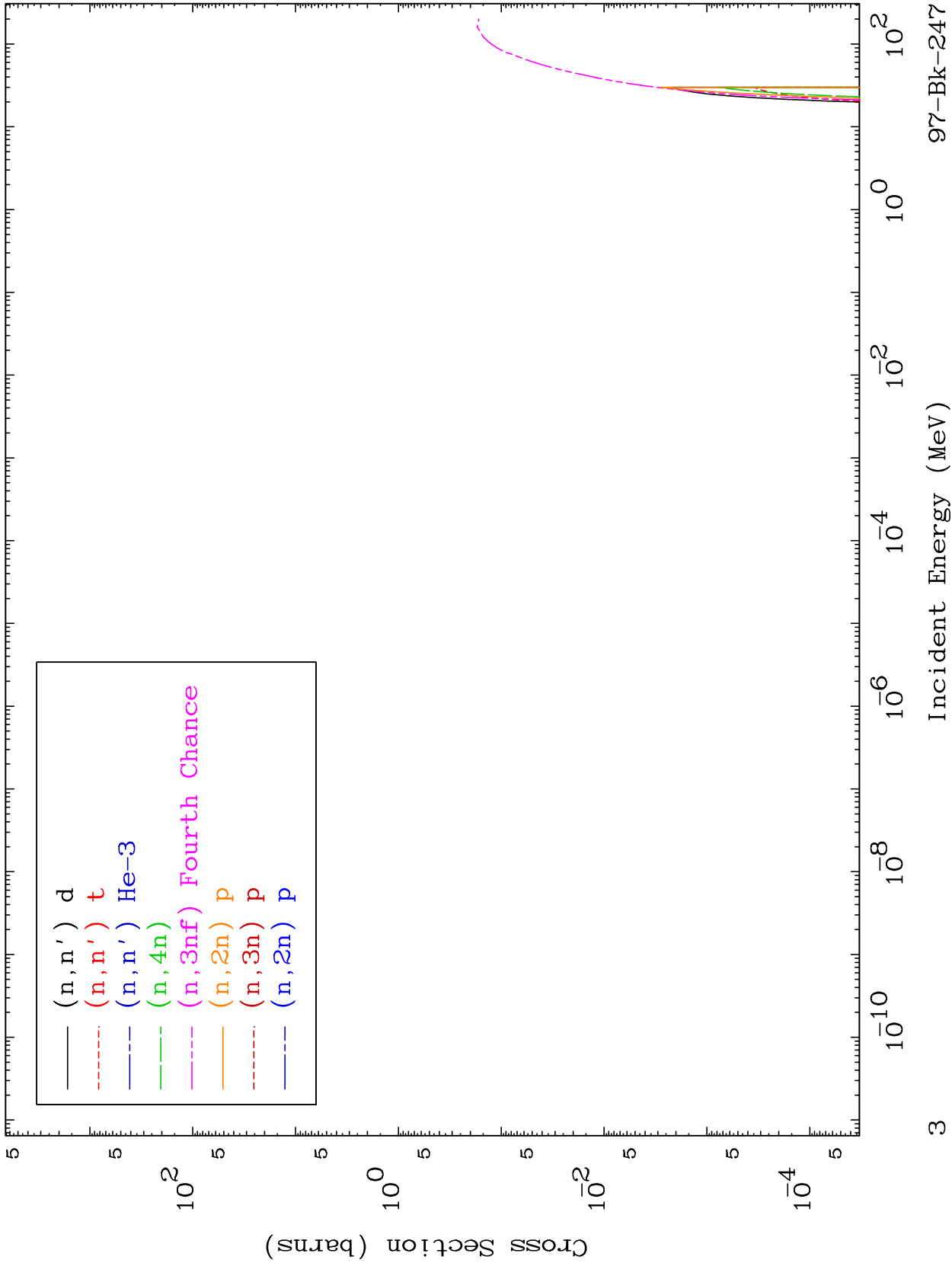
MAT 9746

Neutron Major  
293 Kelvin Cross Sections

97-Bk-247



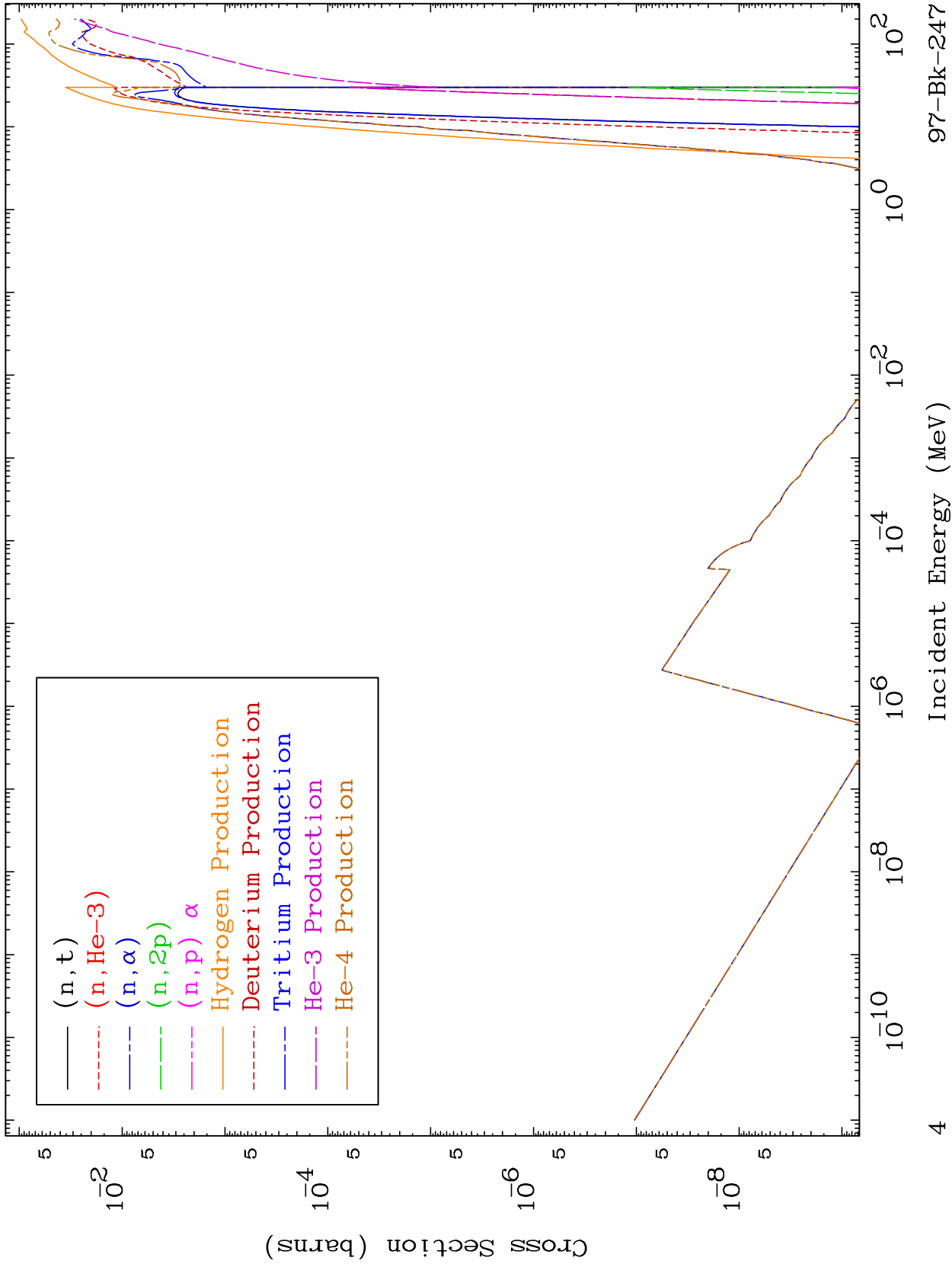




MAT 9746

Neutron Absorption  
293 Kelvin Cross Sections

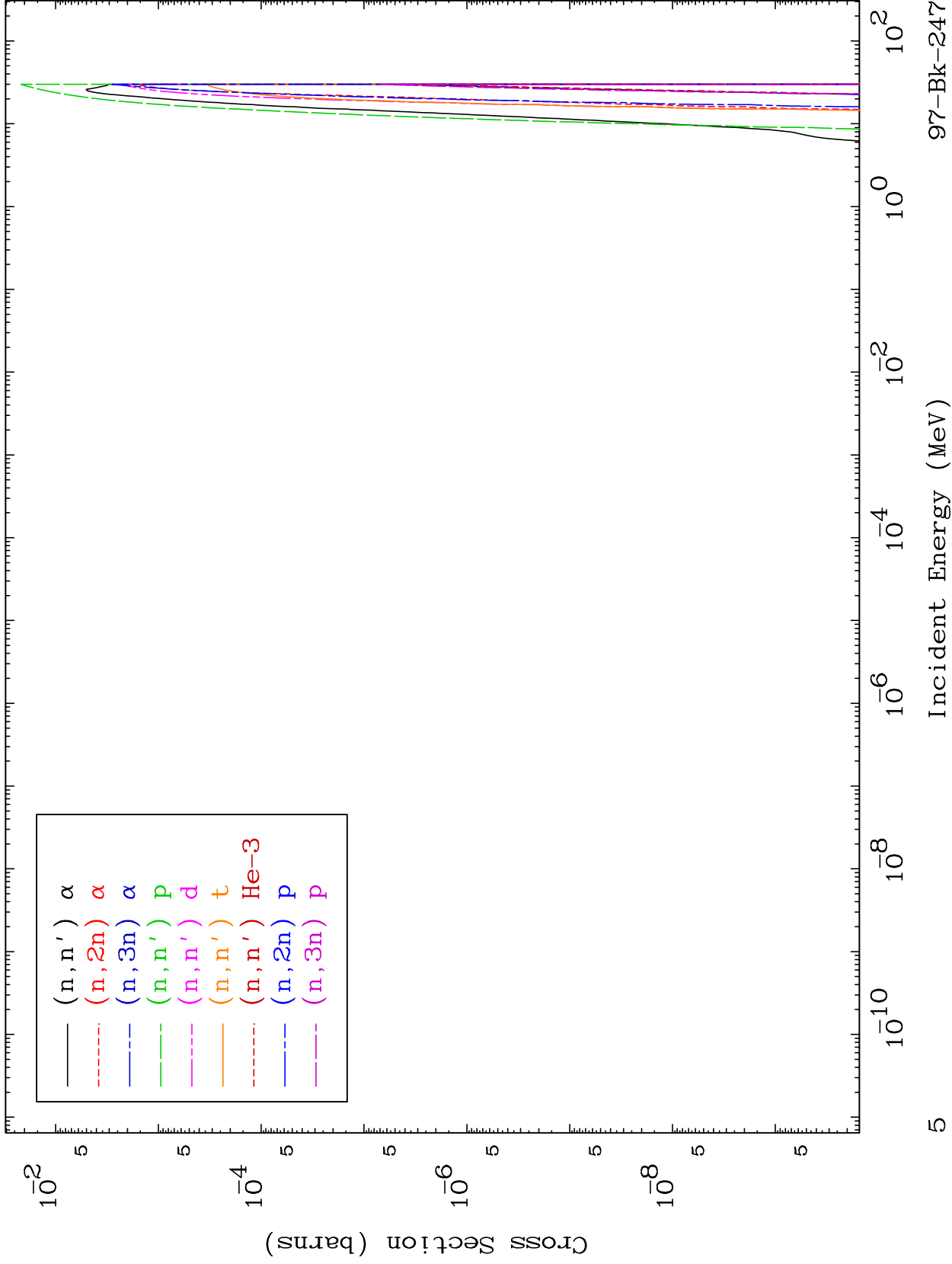
97-Bk-247



MAT 9746

Charged Particle  
293 Kelvin Cross Sections

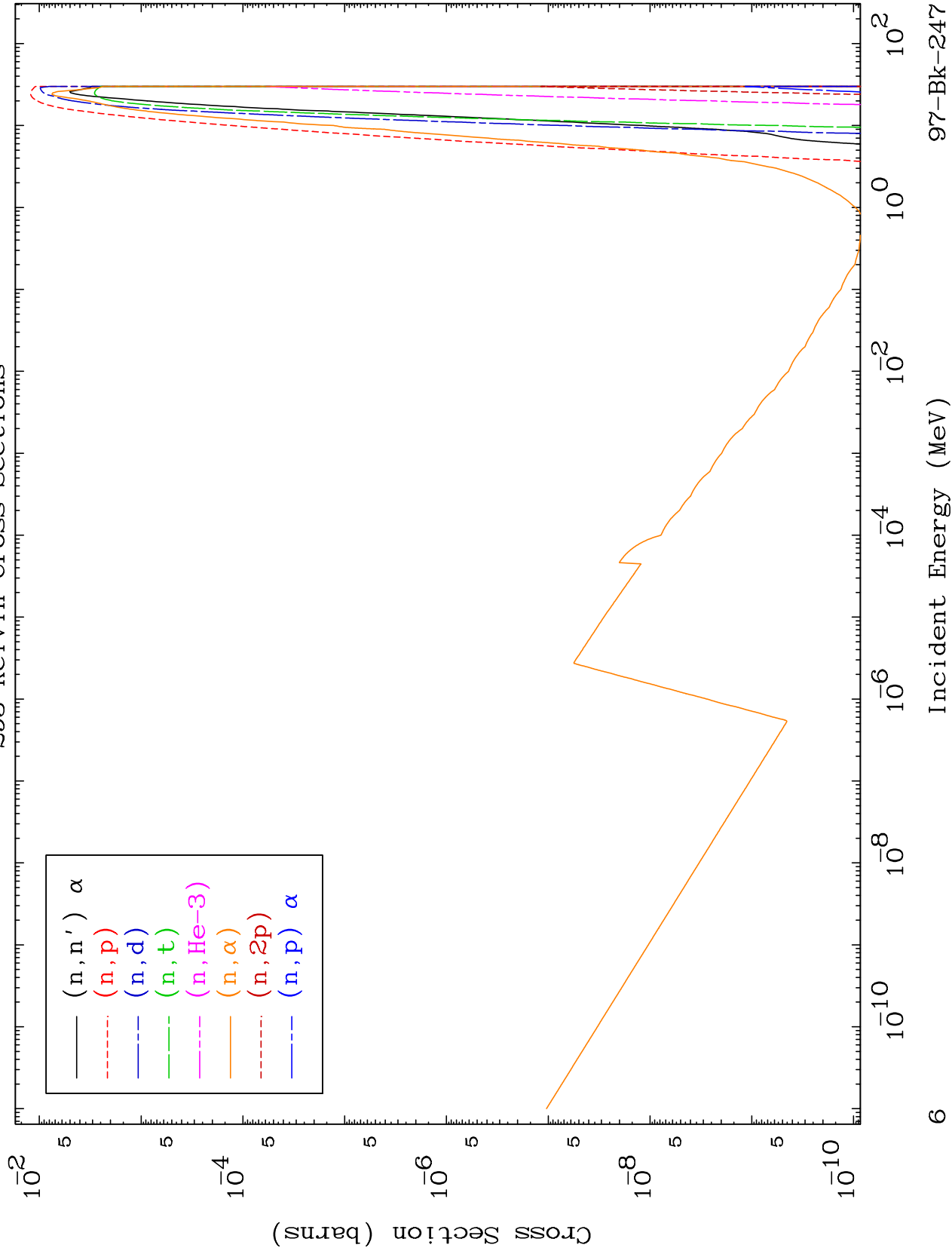
97-Bk-247



MAT 9746

Charged Particle  
293 Kelvin Cross Sections

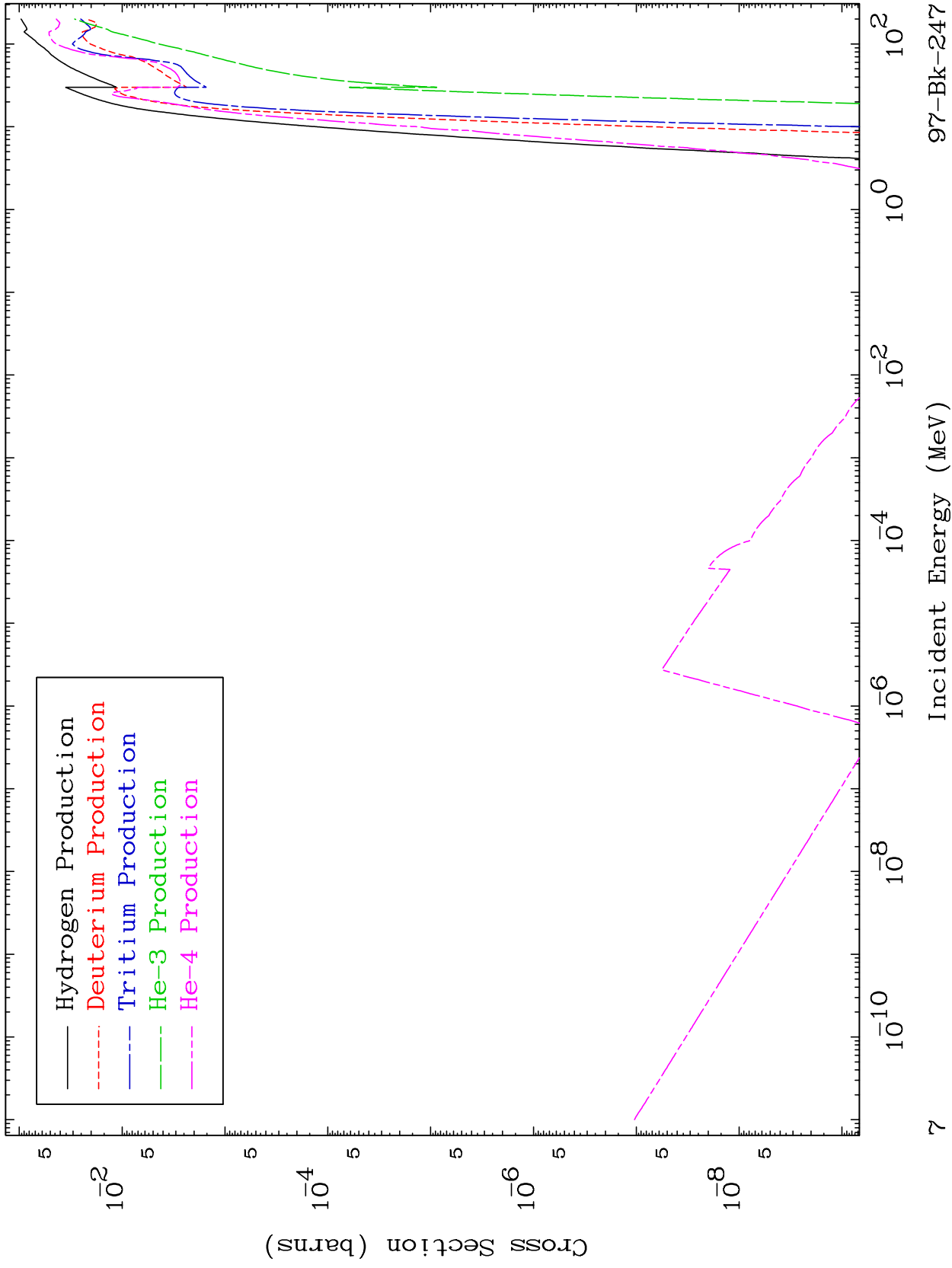
97-Bk-247



MAT 9746

Particle Production  
293 Kelvin Cross Sections

97-Bk-247

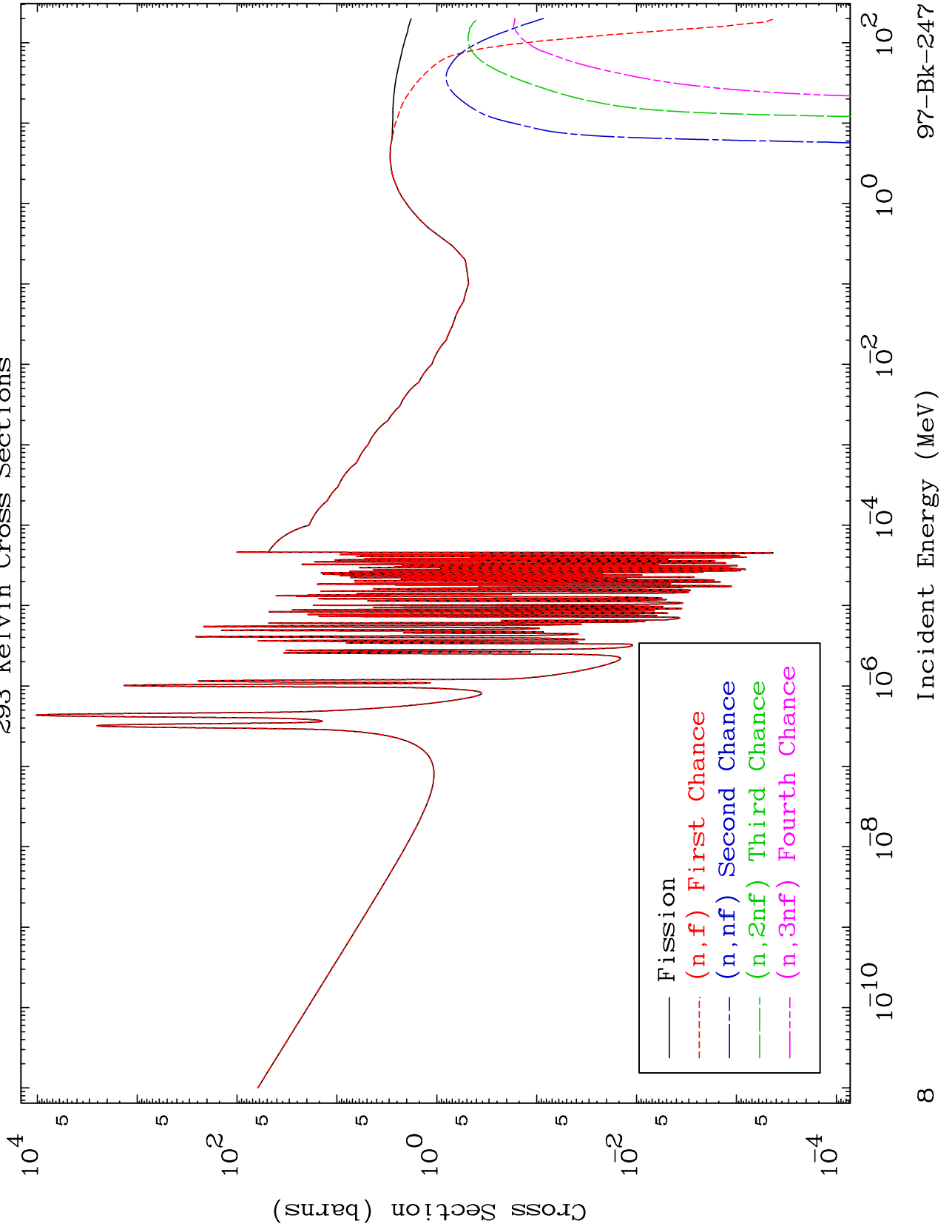


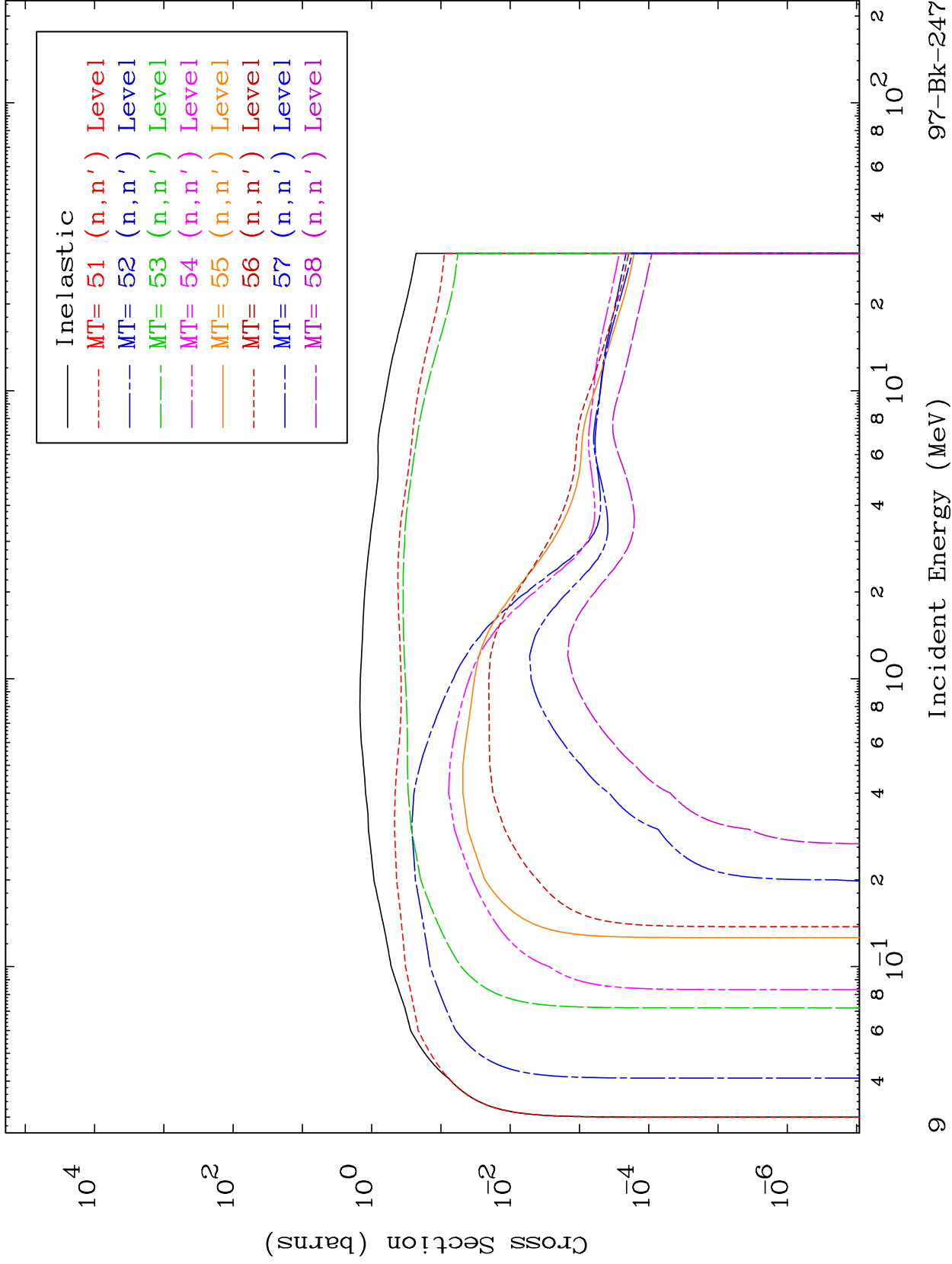


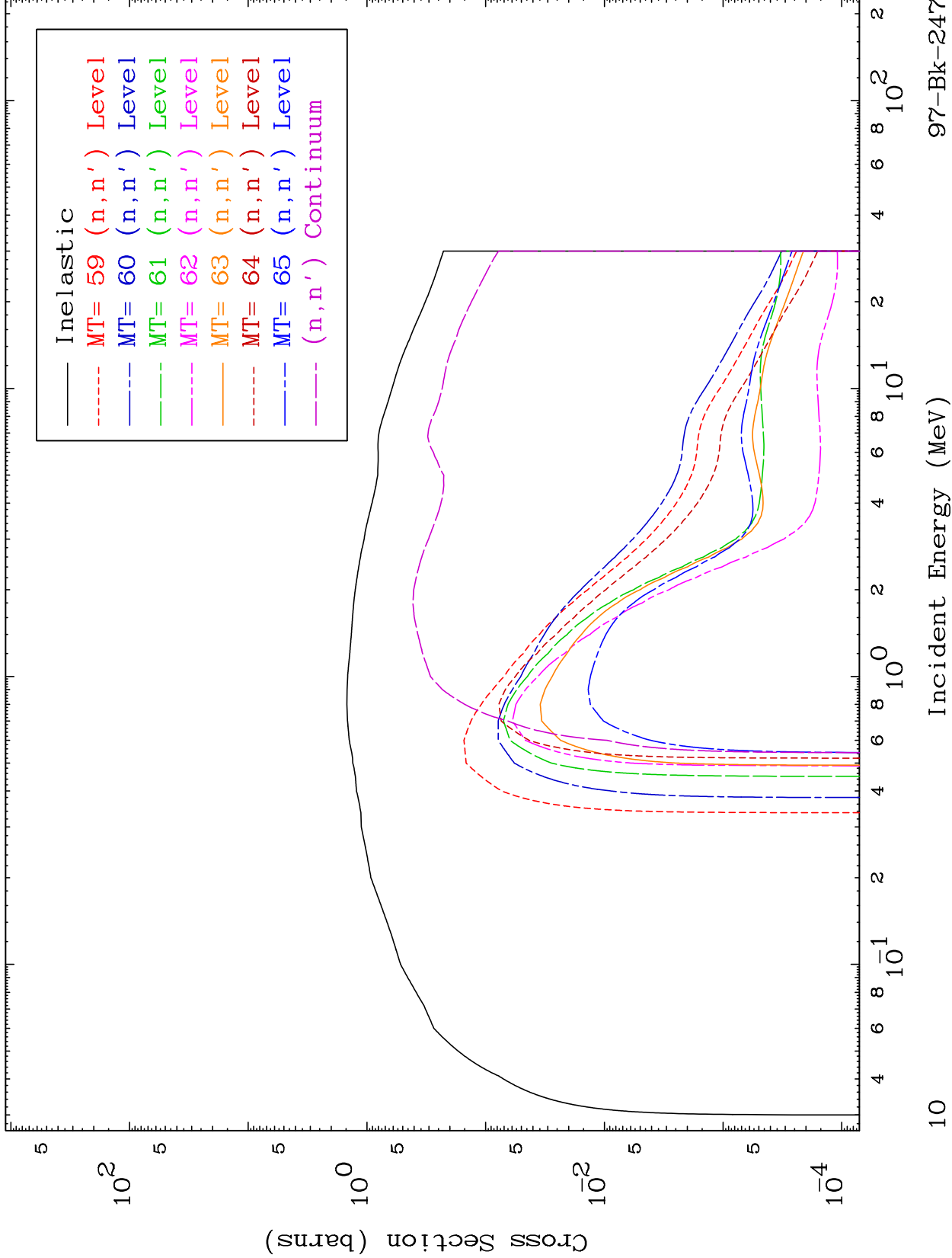
MAT 9746

Fission  
293 Kelvin Cross Sections

97-Bk-247



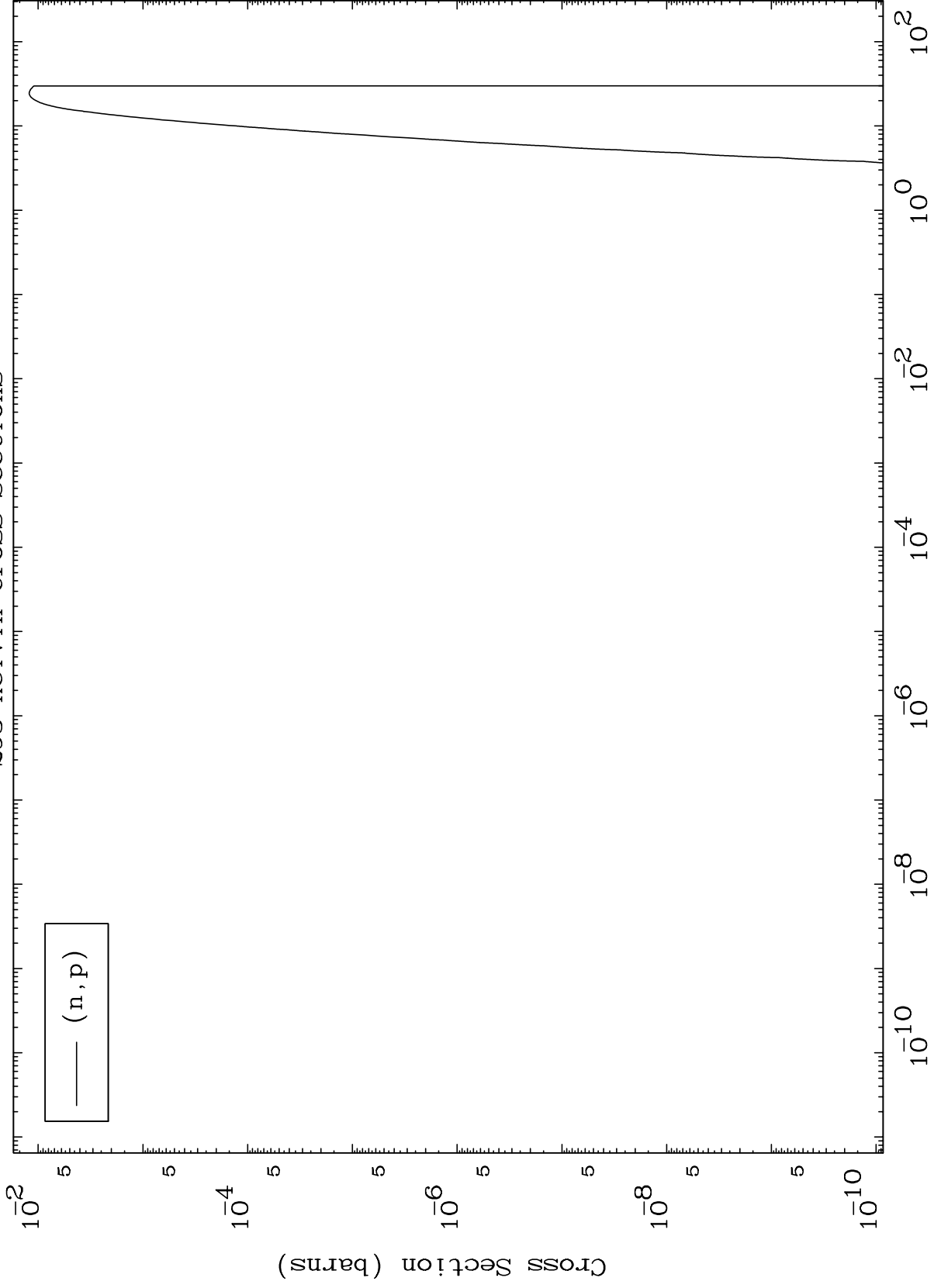




MAT 9746

(n,p) Levels  
293 Kelvin Cross Sections

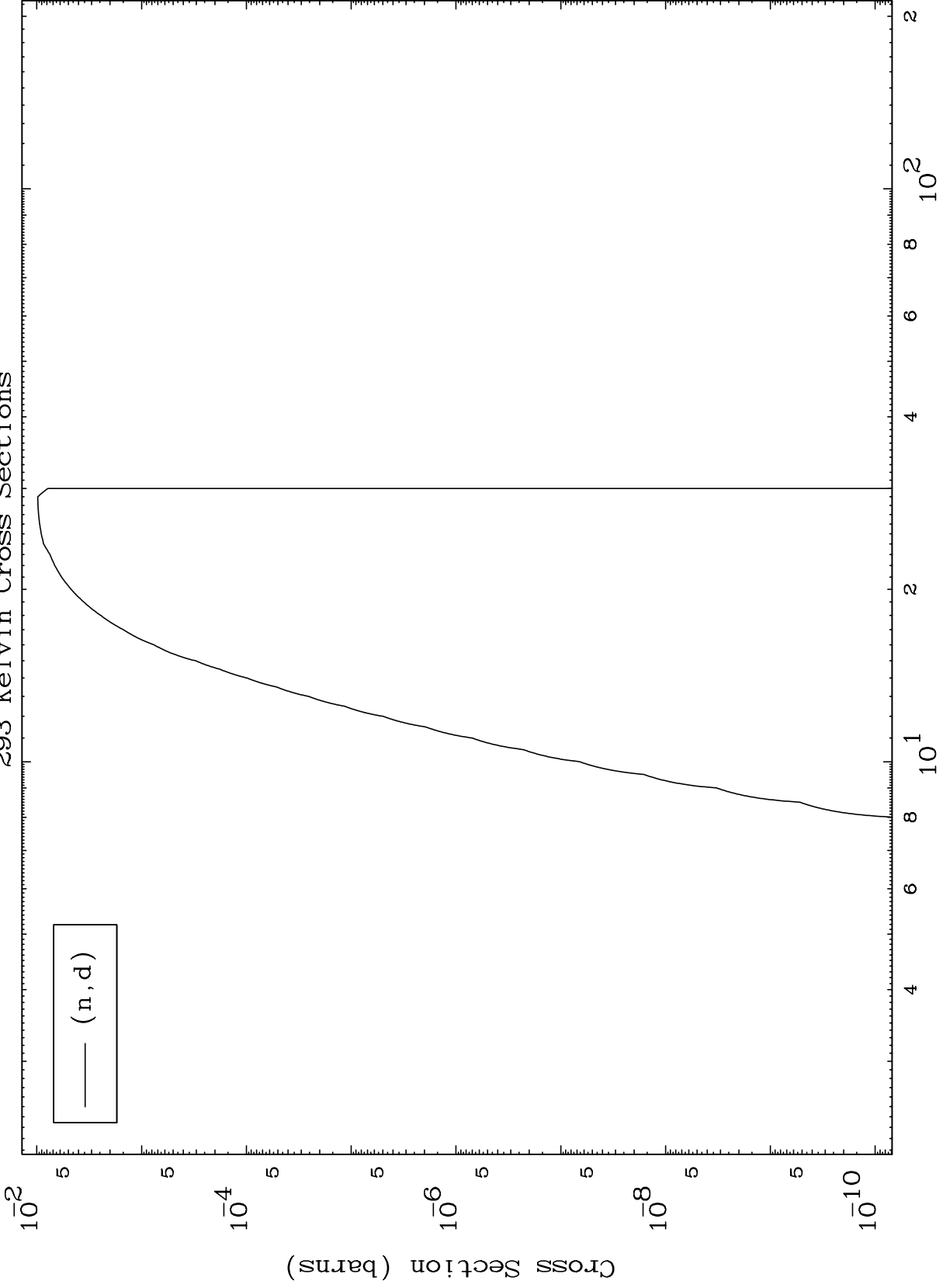
97-Bk-247



MAT 9746

(n,d) Levels  
293 Kelvin Cross Sections

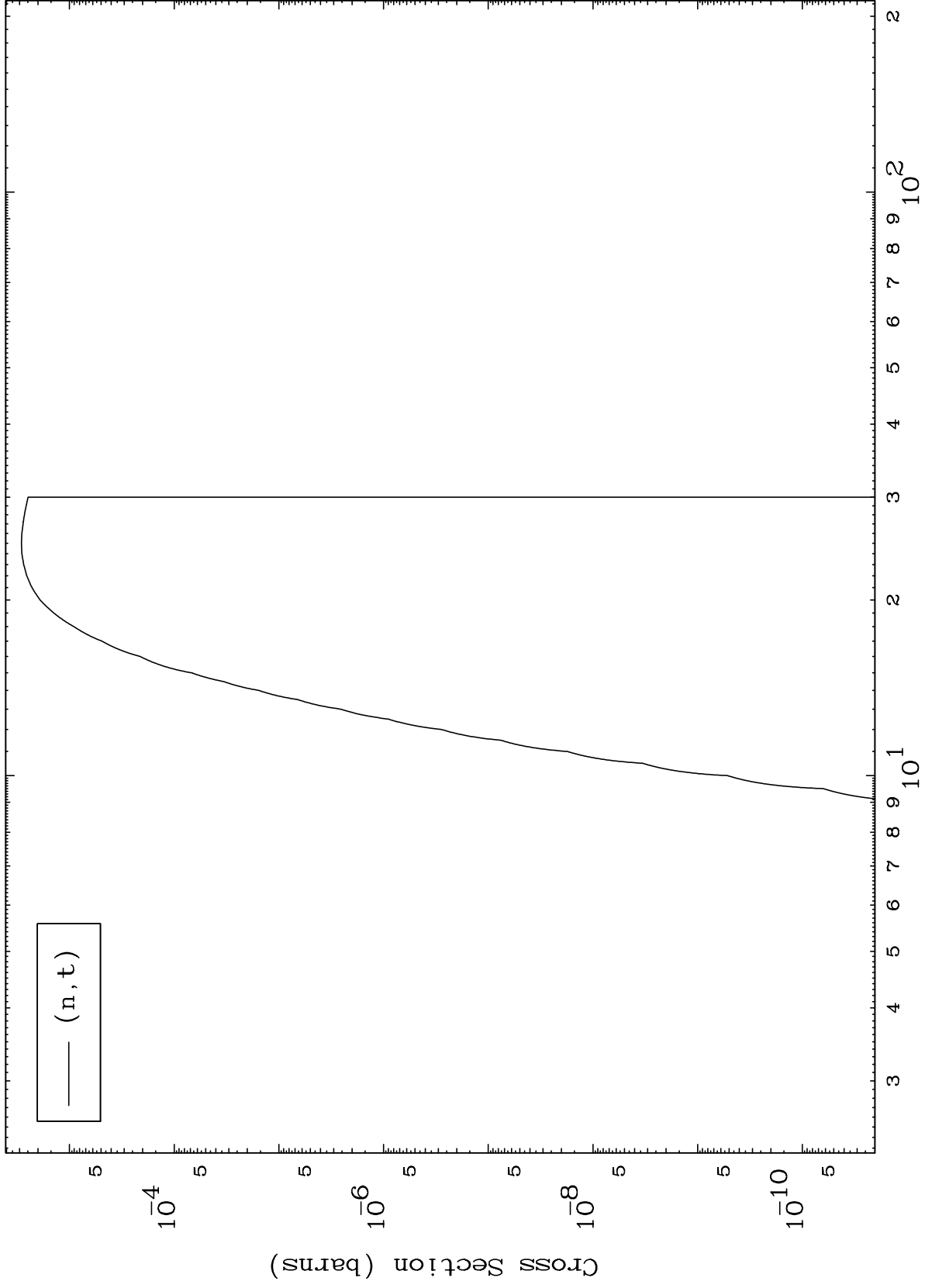
97-Bk-247



MAT 9746

(n,t) Levels  
293 Kelvin Cross Sections

97-Bk-247



13

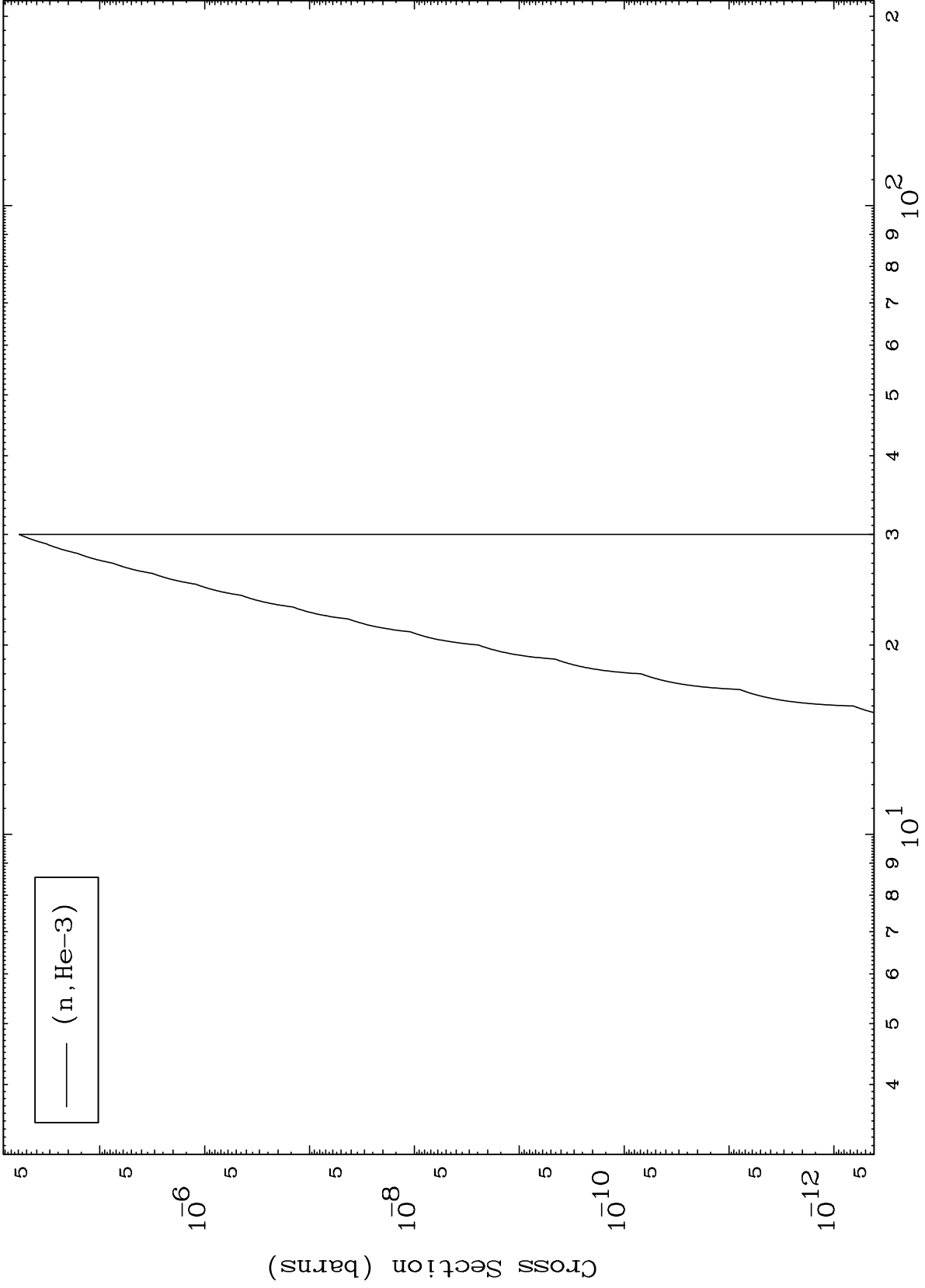
Incident Energy (MeV)

97-Bk-247

MAT 9746

(n,He3) Levels  
293 Kelvin Cross Sections

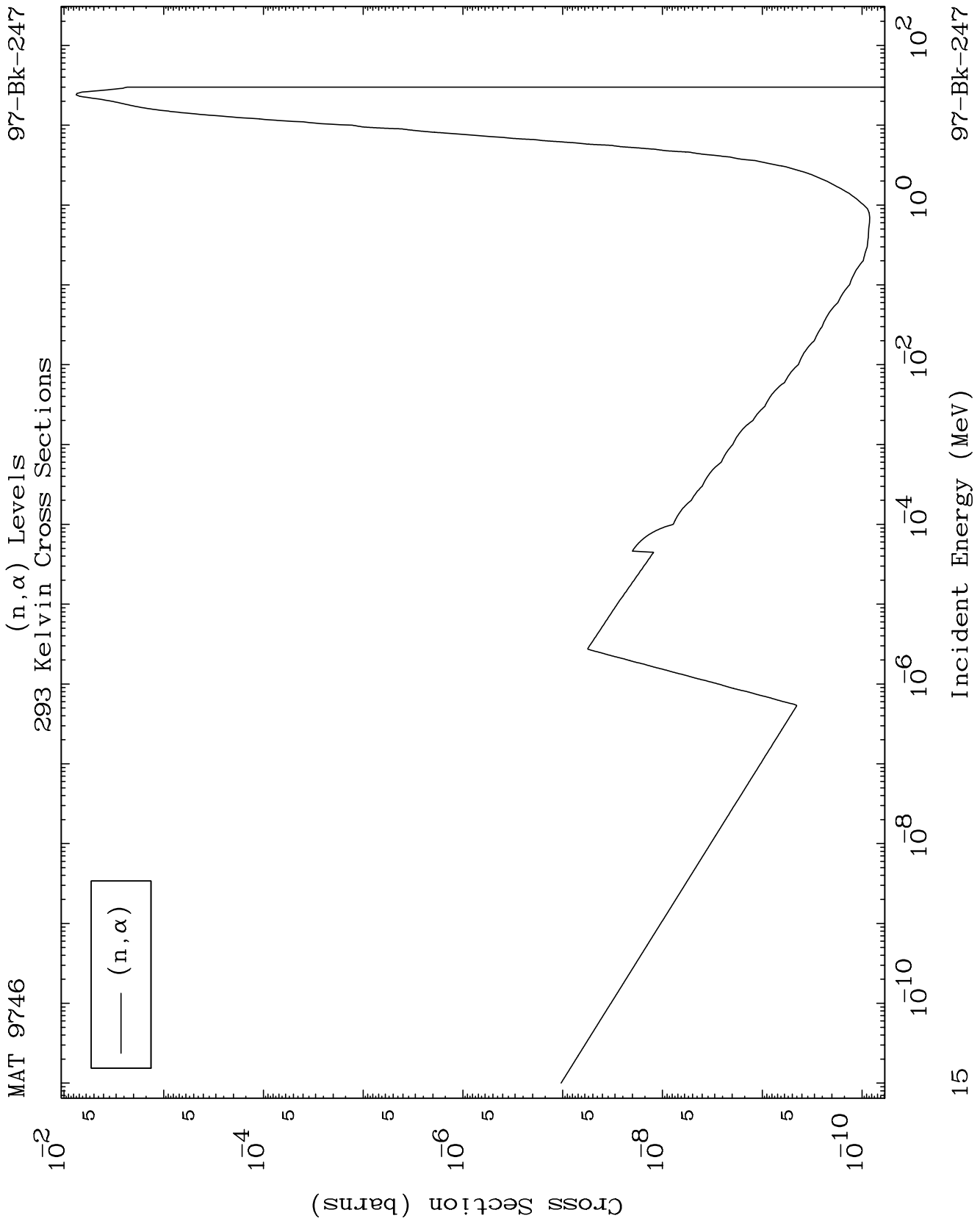
97-Bk-247



14

Incident Energy (MeV)

97-Bk-247

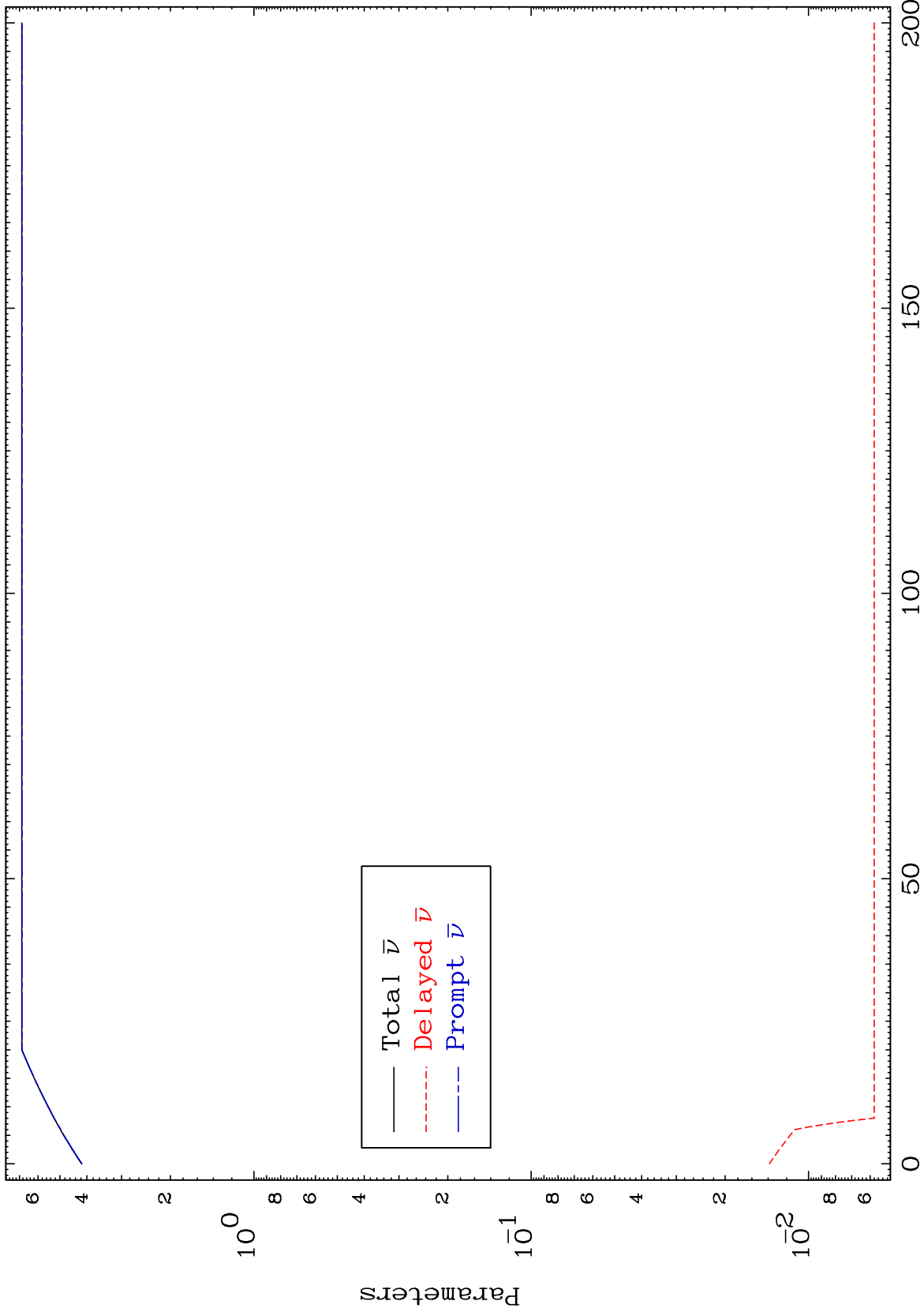




MAT 9746

Energy Release  
Parameters

97-Bk-247



16

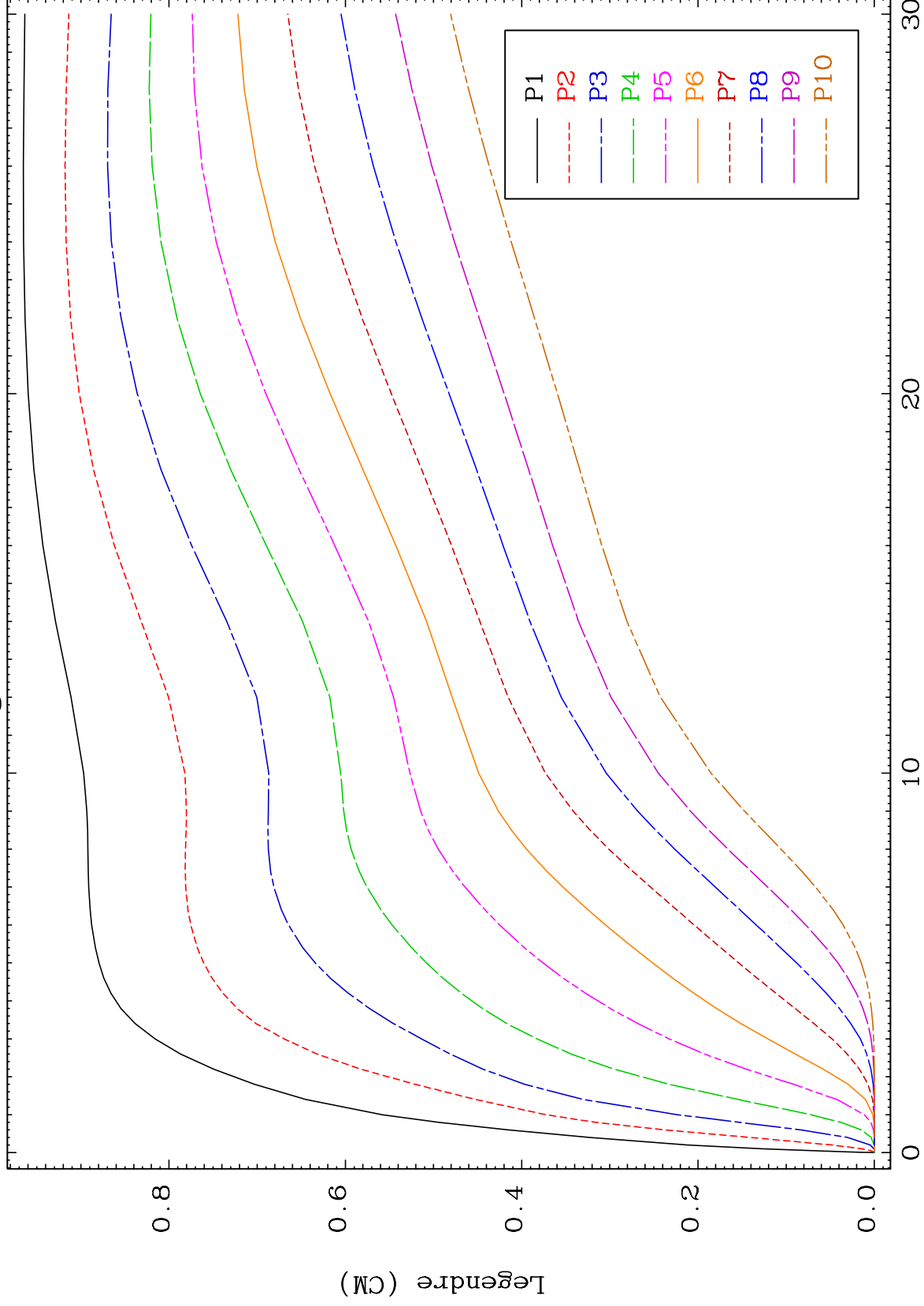
Incident Energy (MeV)

97-Bk-247

MAT 9746

Elastic Legendre Coefficients

97-Bk-247



17

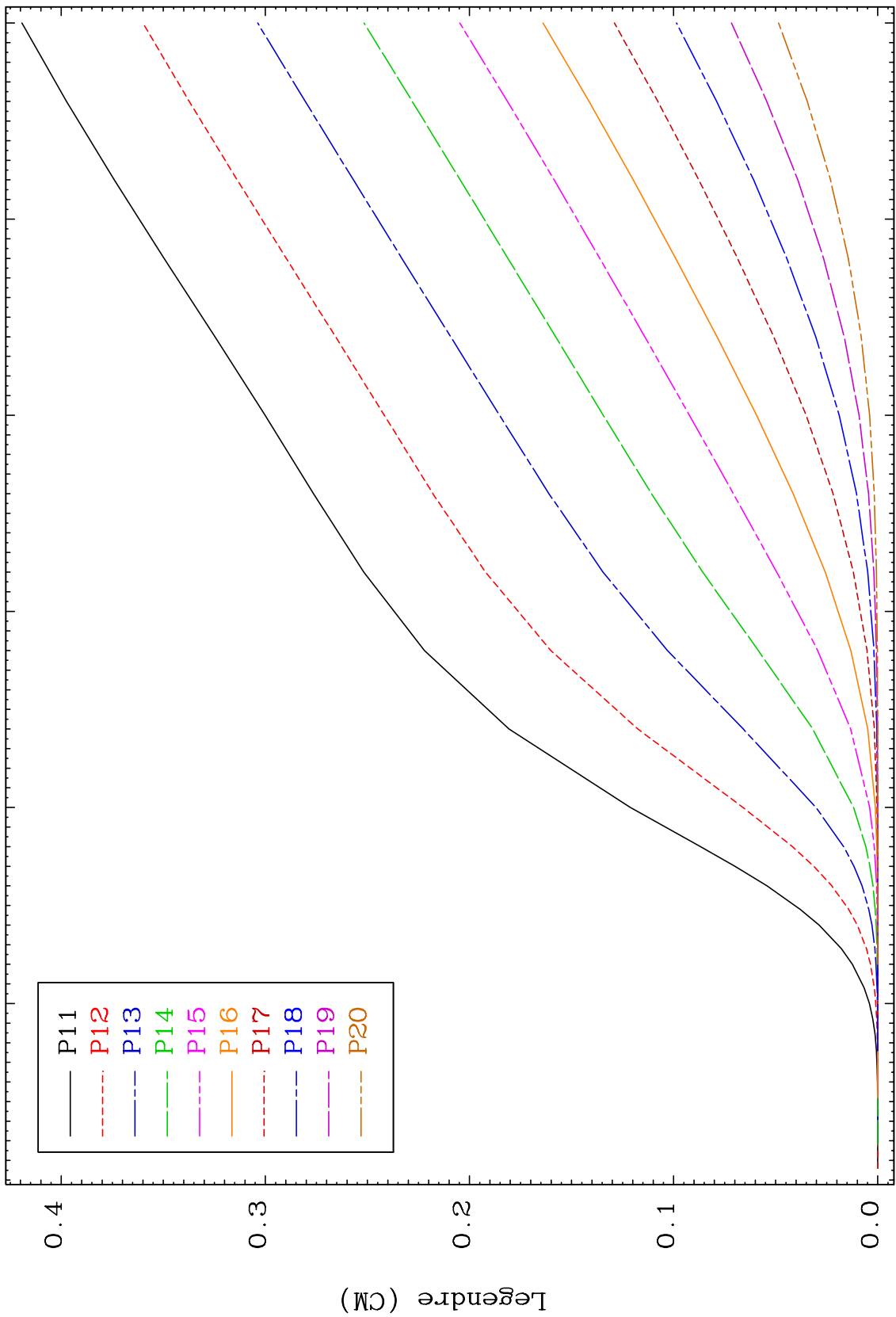
Incident Energy (MeV)

97-Bk-247

MAT 9746

Elastic Legendre Coefficients

97-Bk-247



18

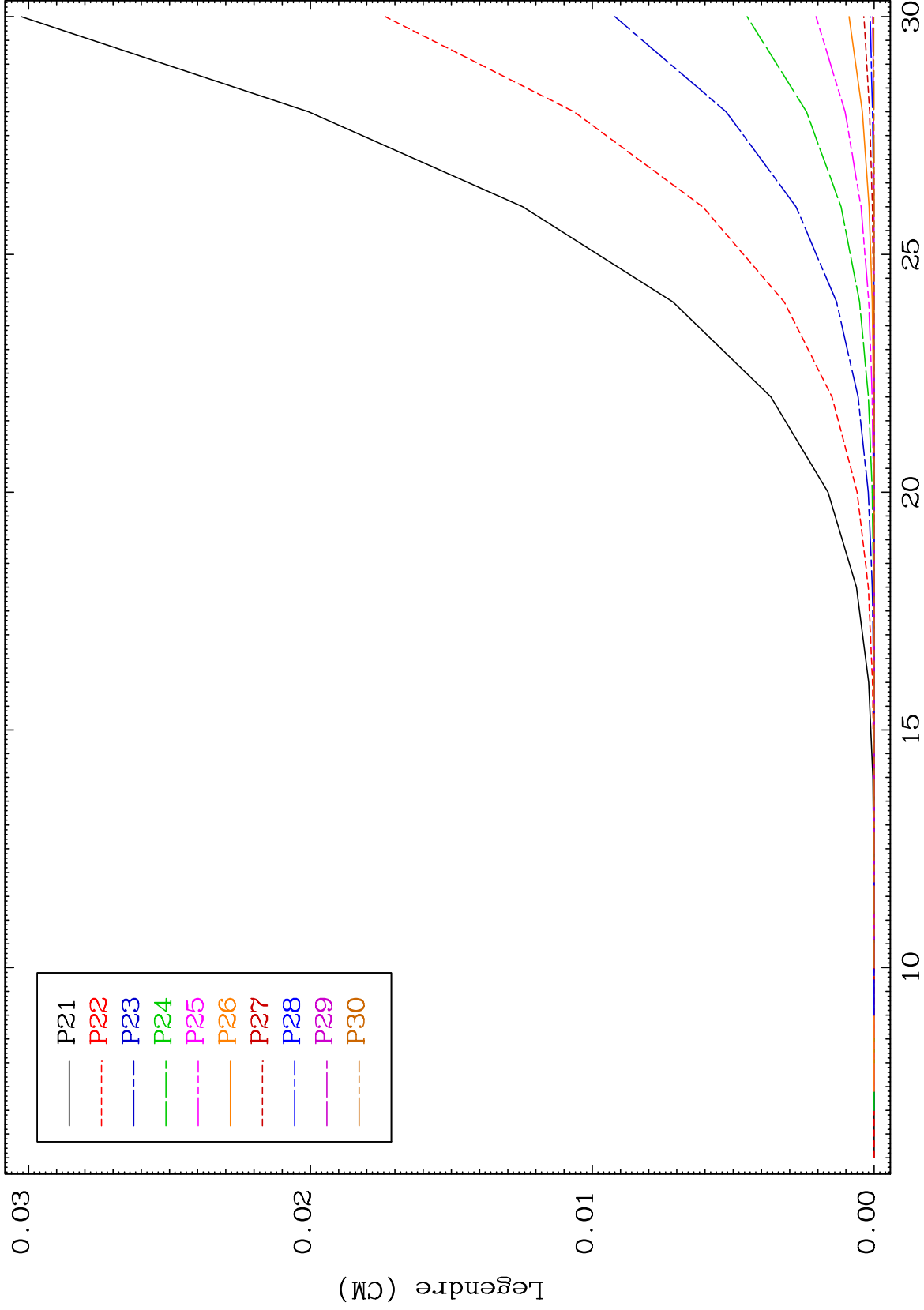
Incident Energy (MeV)

97-Bk-247

MAT 9746

### Elastic Legendre Coefficients

97-Bk-247



19

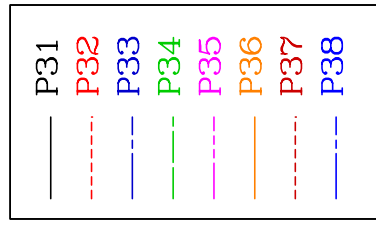
Incident Energy (MeV)

97-Bk-247

MAT 9746

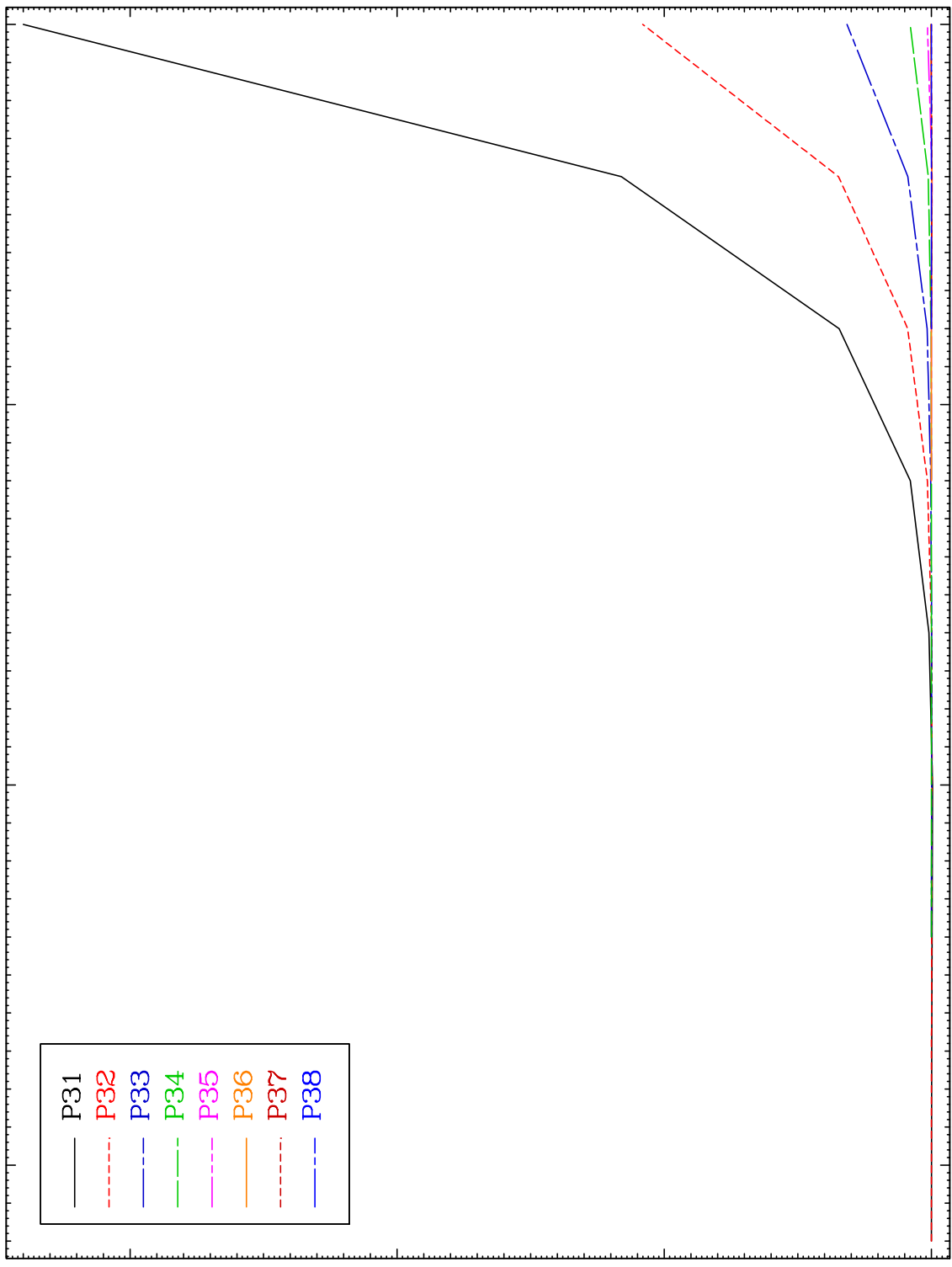
Elastic Legendre Coefficients

97-Bk-247



$\times 10^{-6}$

Legendre (CM)

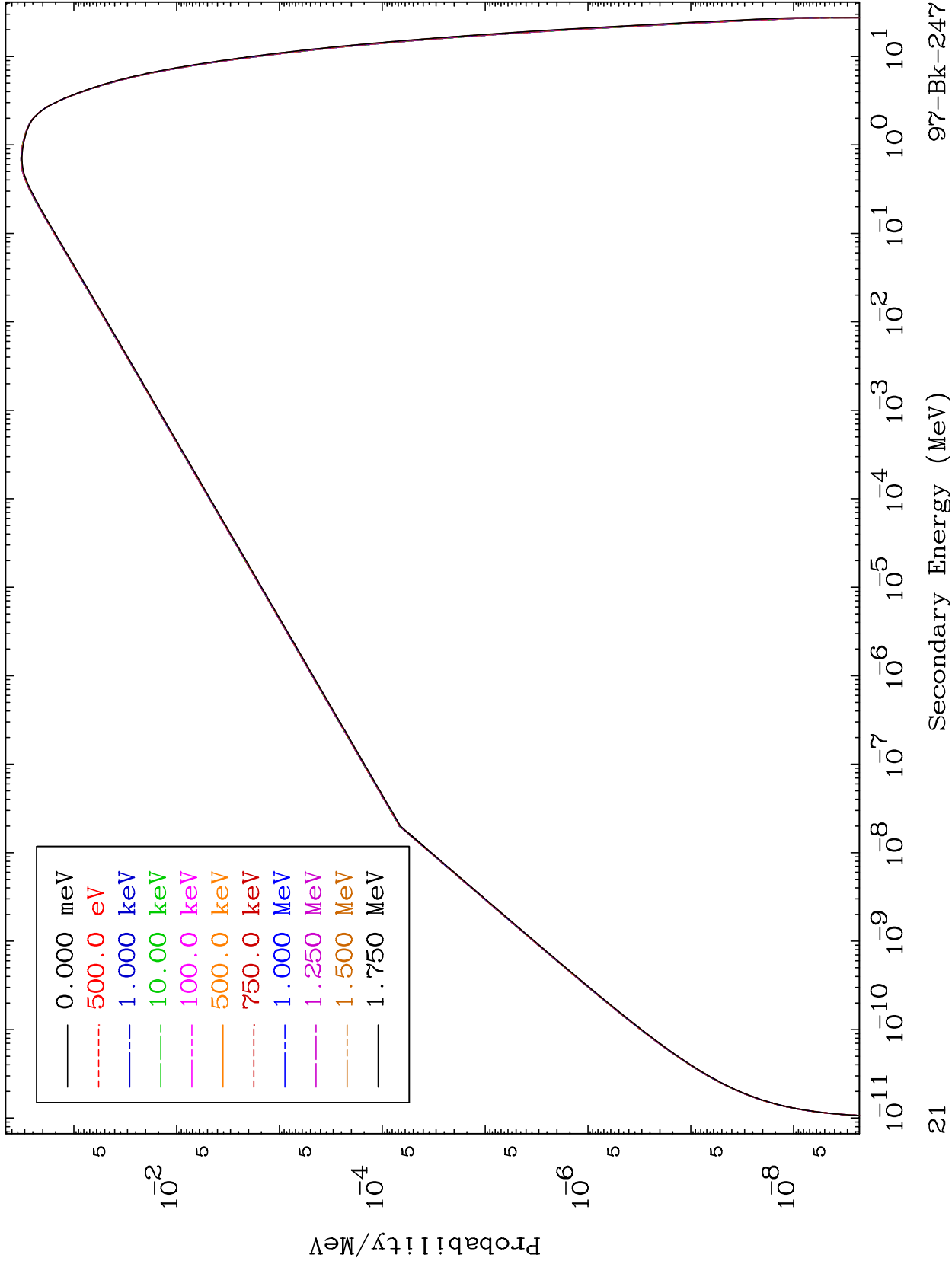


20

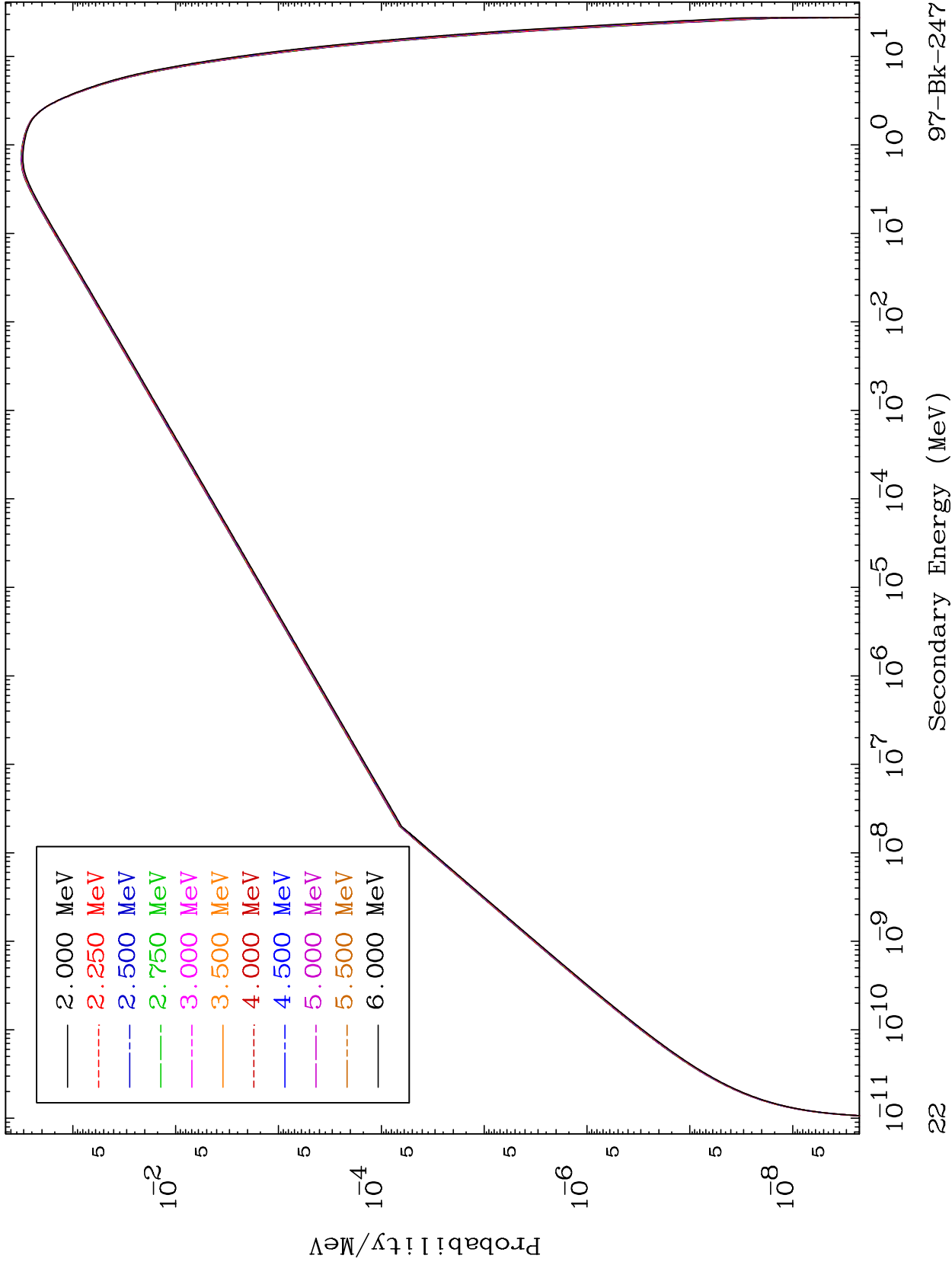
Incident Energy (MeV)

97-Bk-247

Fission Energy Distributions



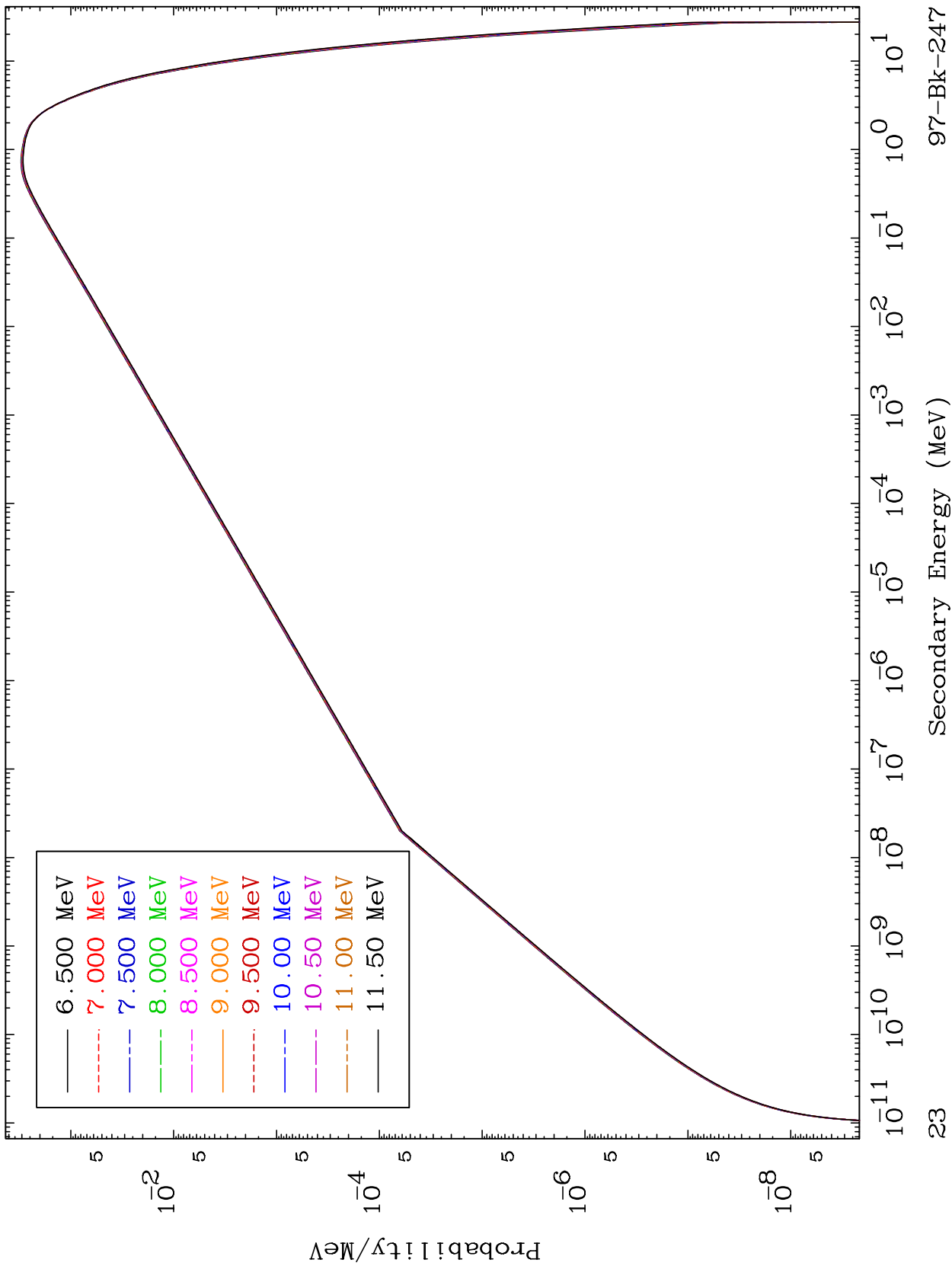
Fission Energy Distributions



MAT 9746

### Fission Energy Distributions

97-Bk-247



97-Bk-247

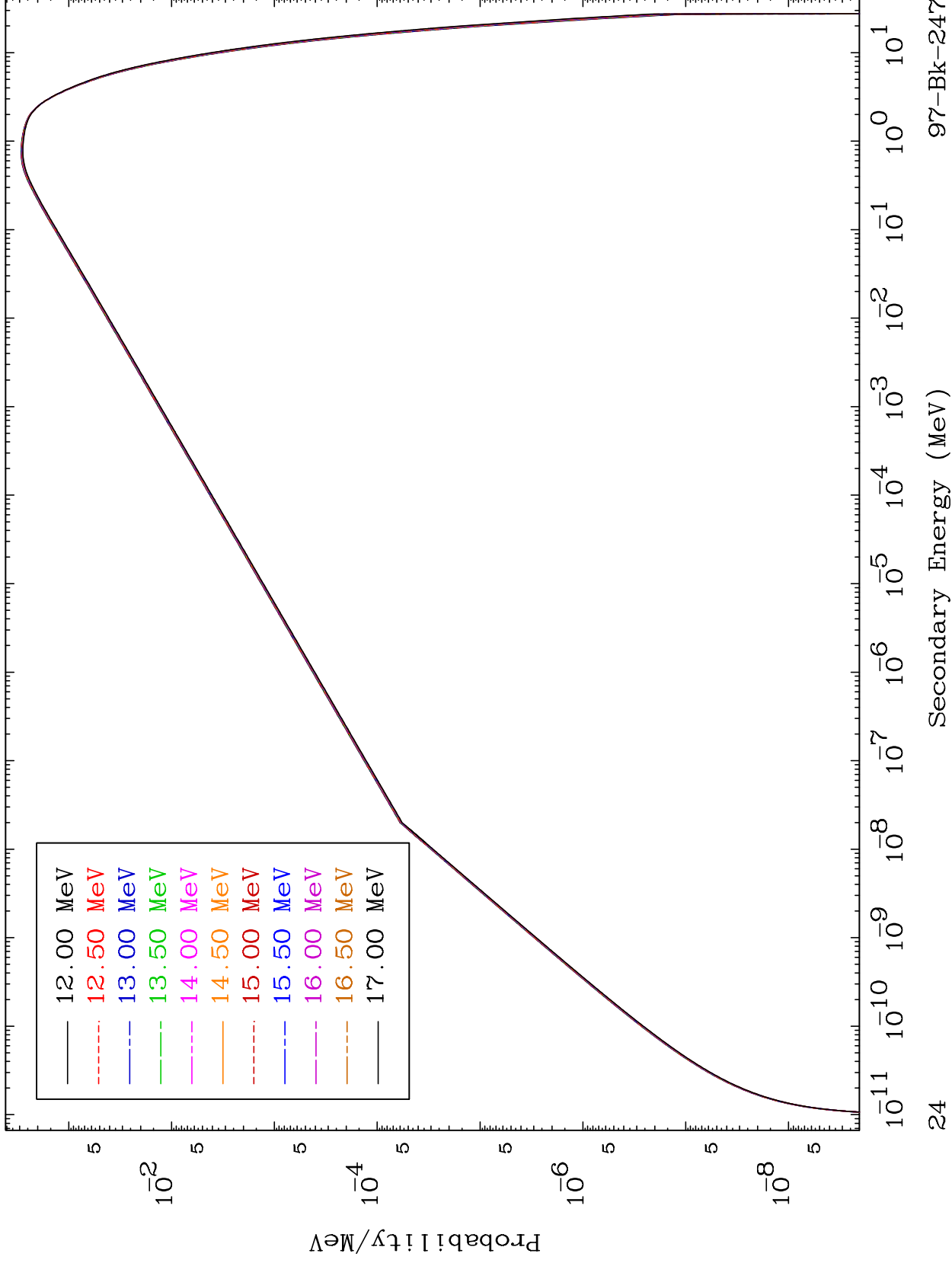
Secondary Energy (MeV)



MAT 9746

### Fission Energy Distributions

97-Bk-247



97-Bk-247

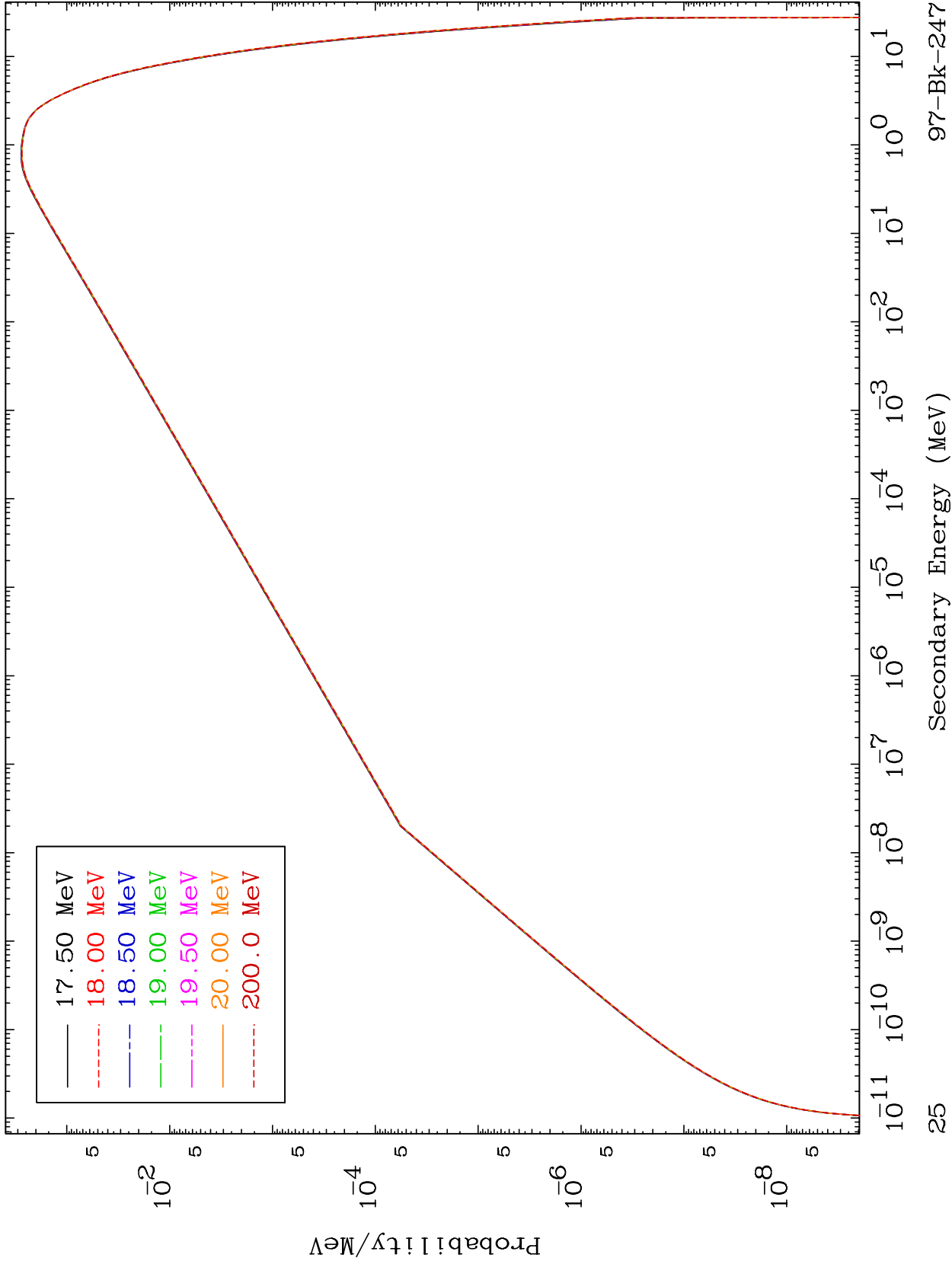
Secondary Energy (MeV)

24

MAT 9746

### Fission Energy Distributions

97-Bk-247



97-Bk-247

Secondary Energy (MeV)

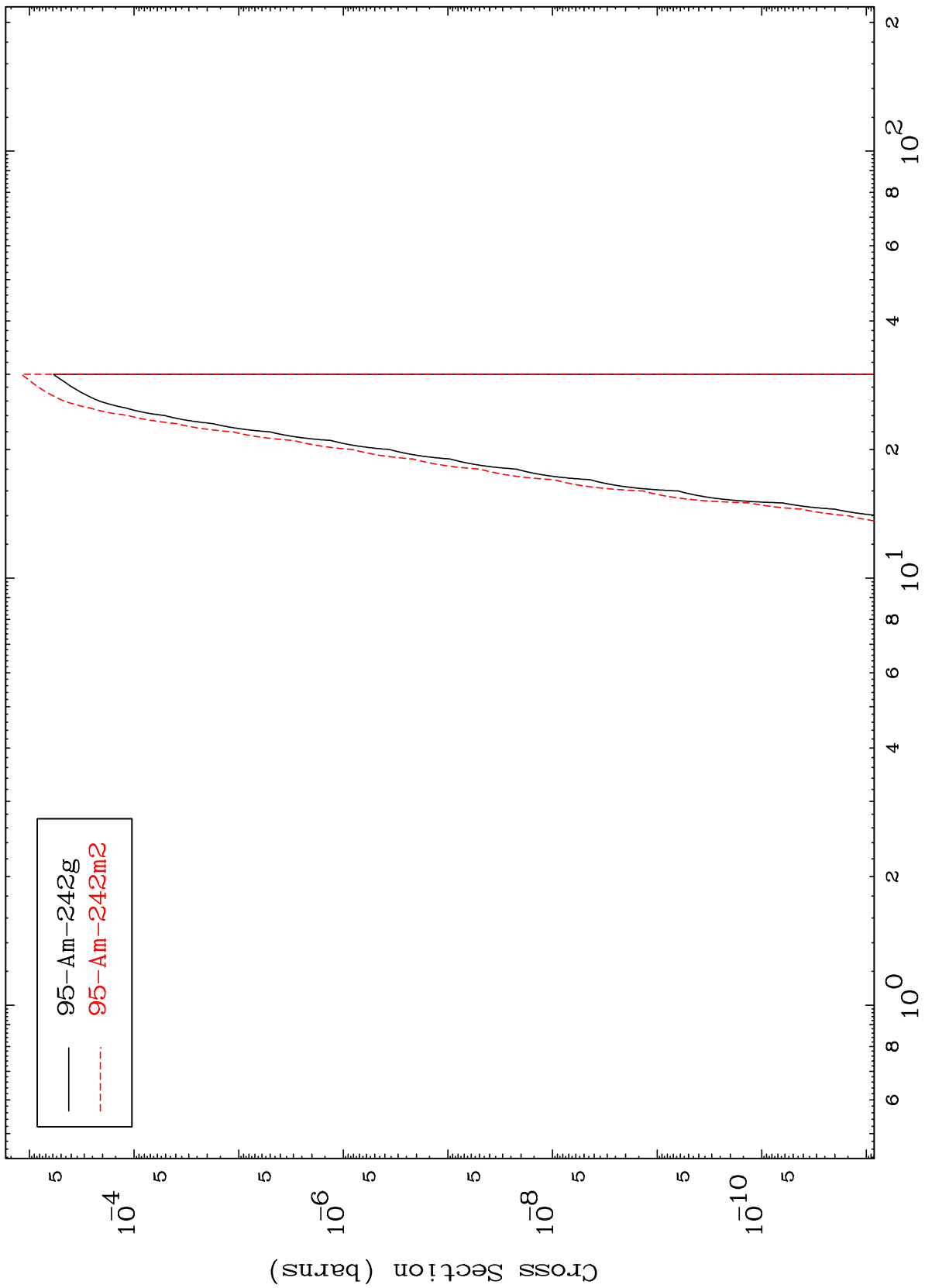
25

MAT 9746

(n,2n)  $\alpha$

97-Bk-247

Radionuclide Production Cross Section



95-Am-242g  
95-Am-242m2

26

Incident Energy (MeV)

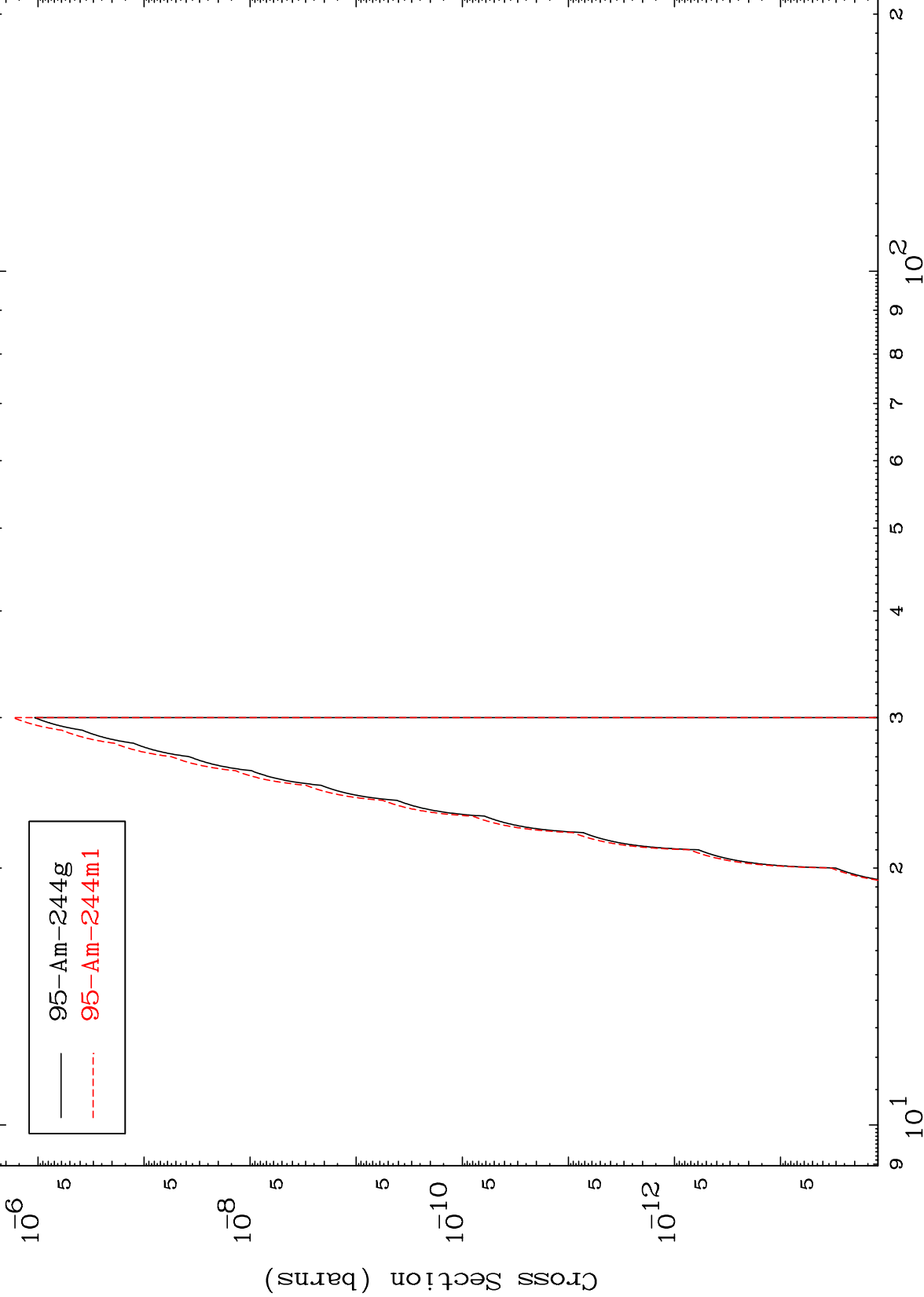
97-Bk-247

MAT 9746

(n,n') He-3

97-Bk-247

Radionuclide Production Cross Section



95-Am-244g  
95-Am-244m1

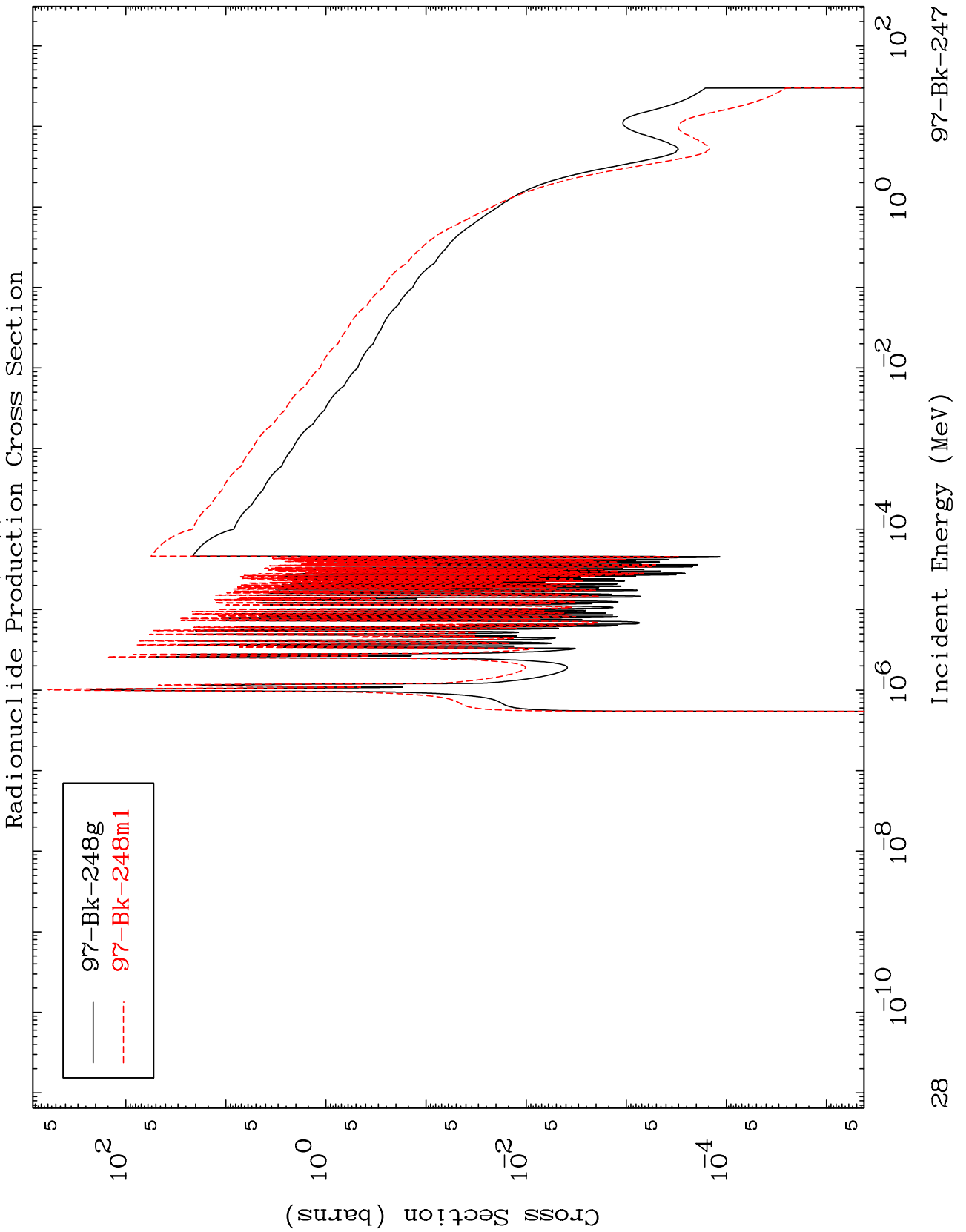
27

Incident Energy (MeV)

97-Bk-247

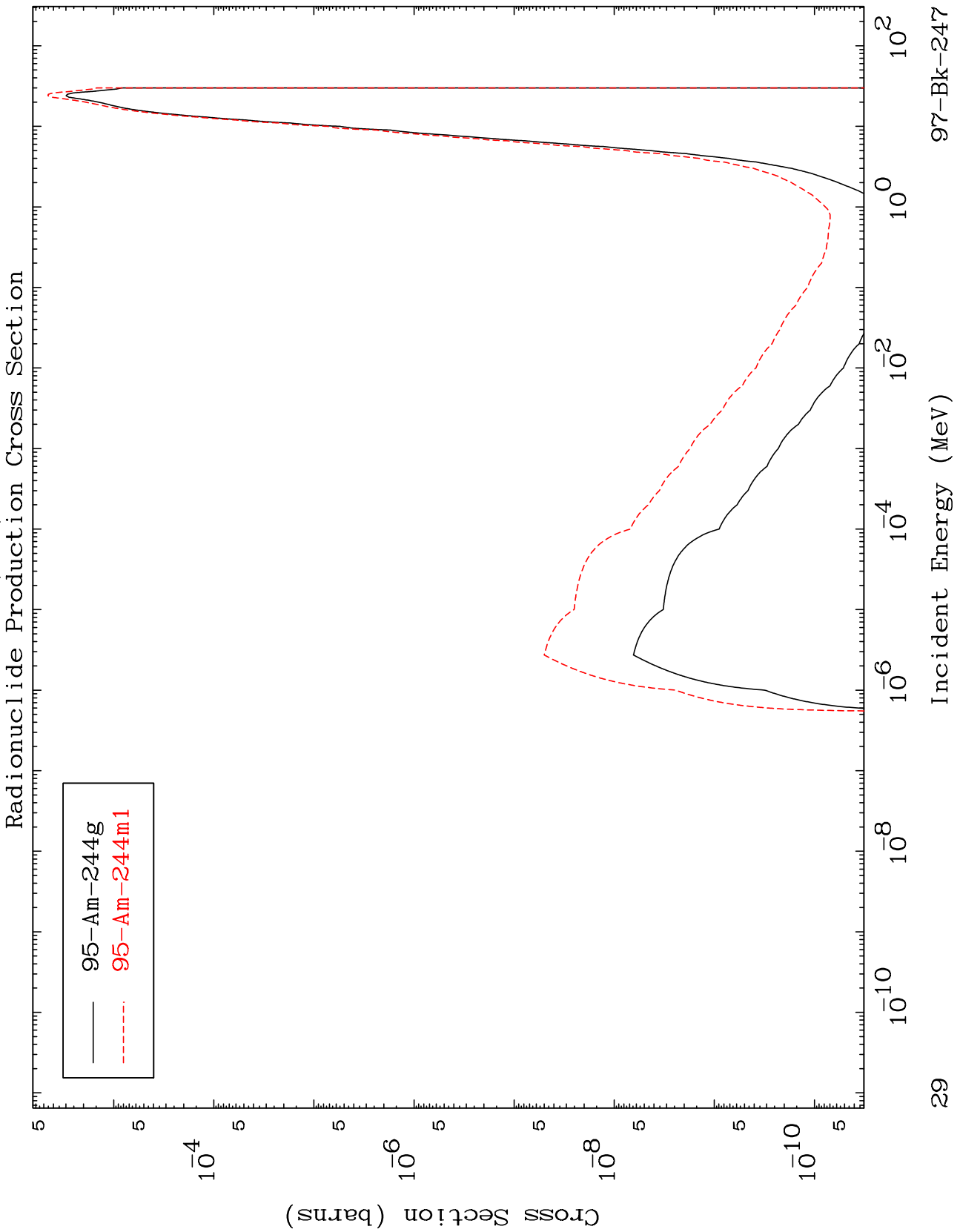
MAT 9746

97-Bk-247



MAT 9746

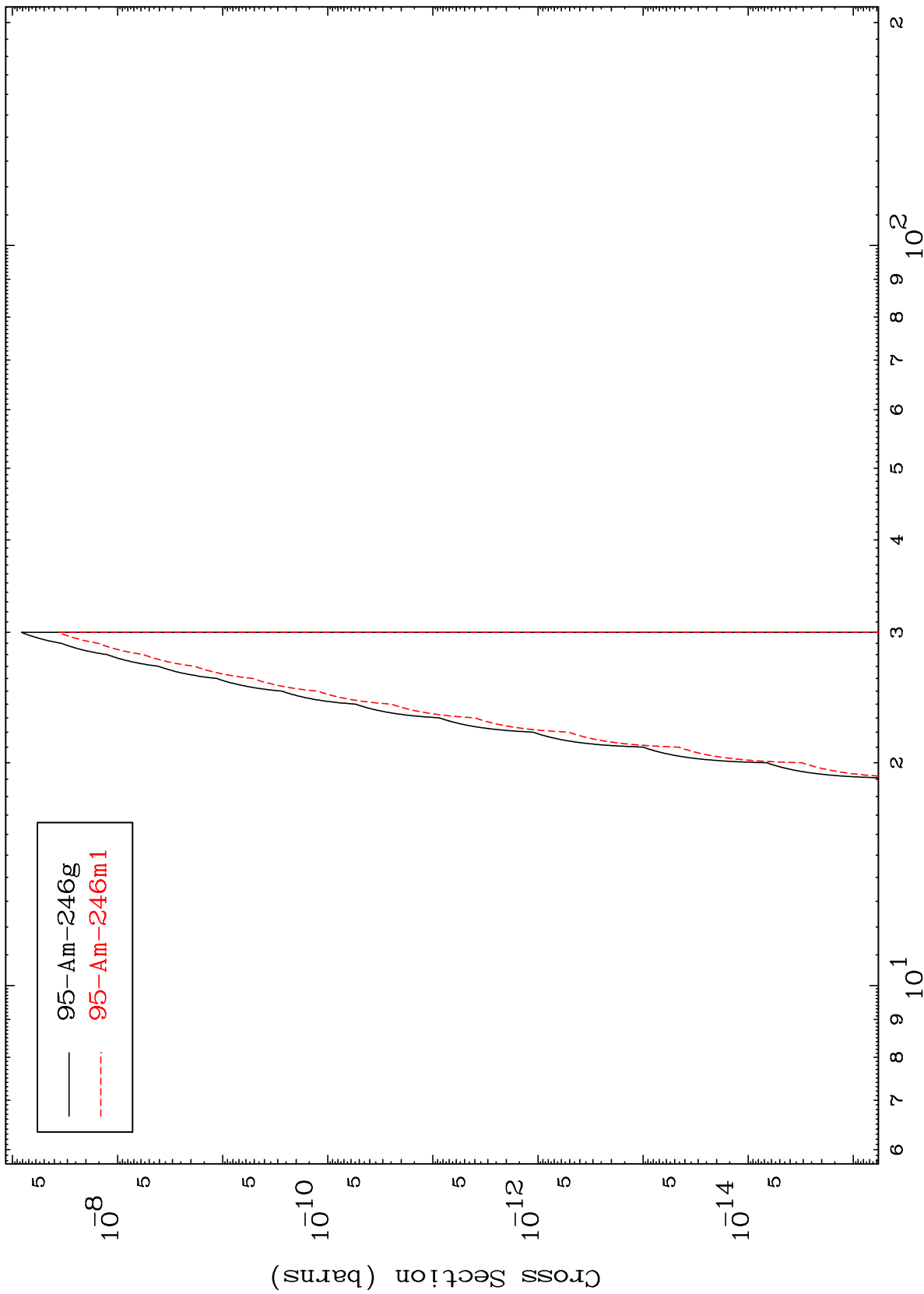
97-Bk-247



MAT 9746

97-Bk-247

(n,2p)  
Radionuclide Production Cross Section



30

Incident Energy (MeV)

97-Bk-247