

Program EVALPLOT
(Version 2021-1)

by

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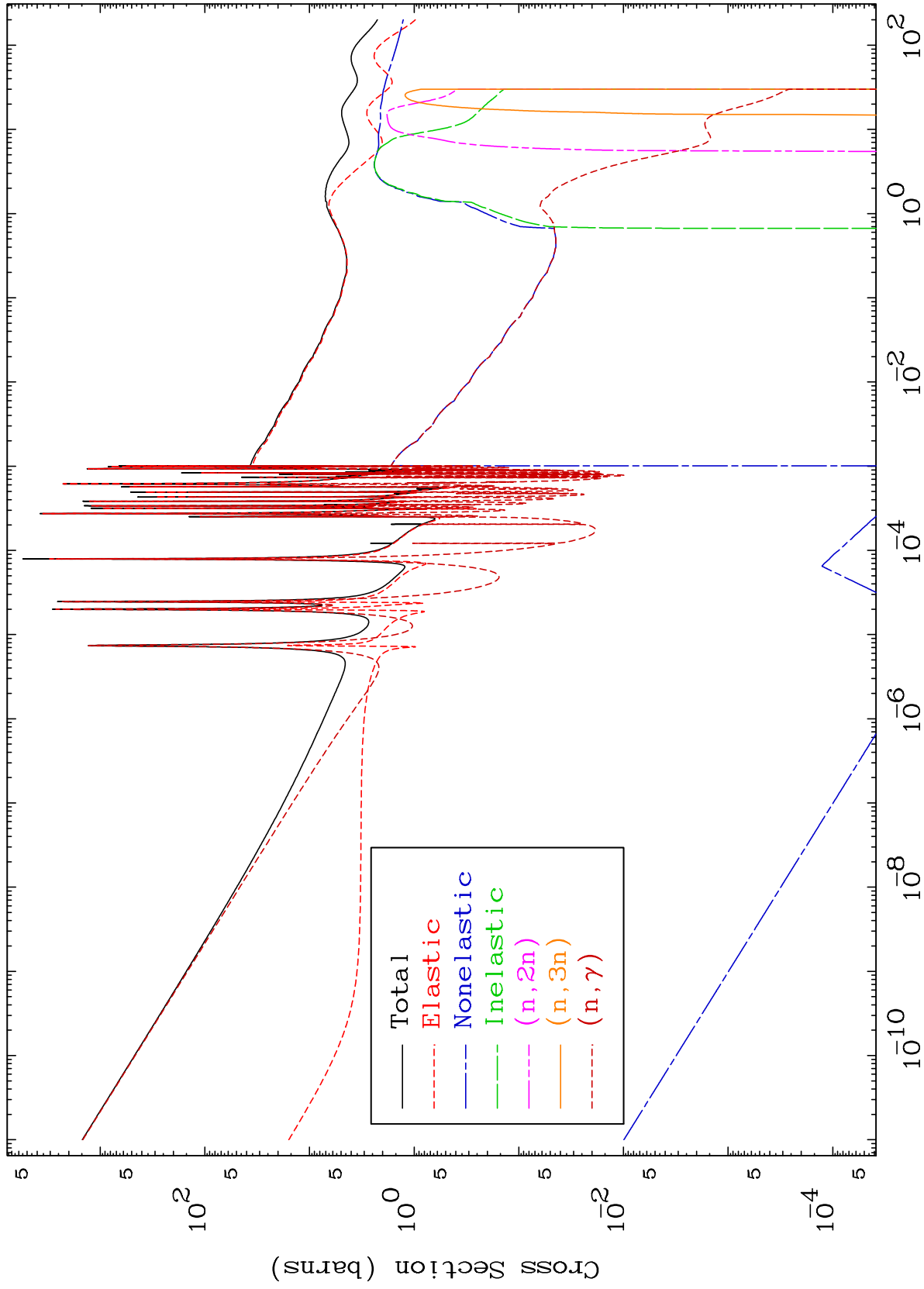
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 5840

Neutron Major
293 Kelvin Cross Sections

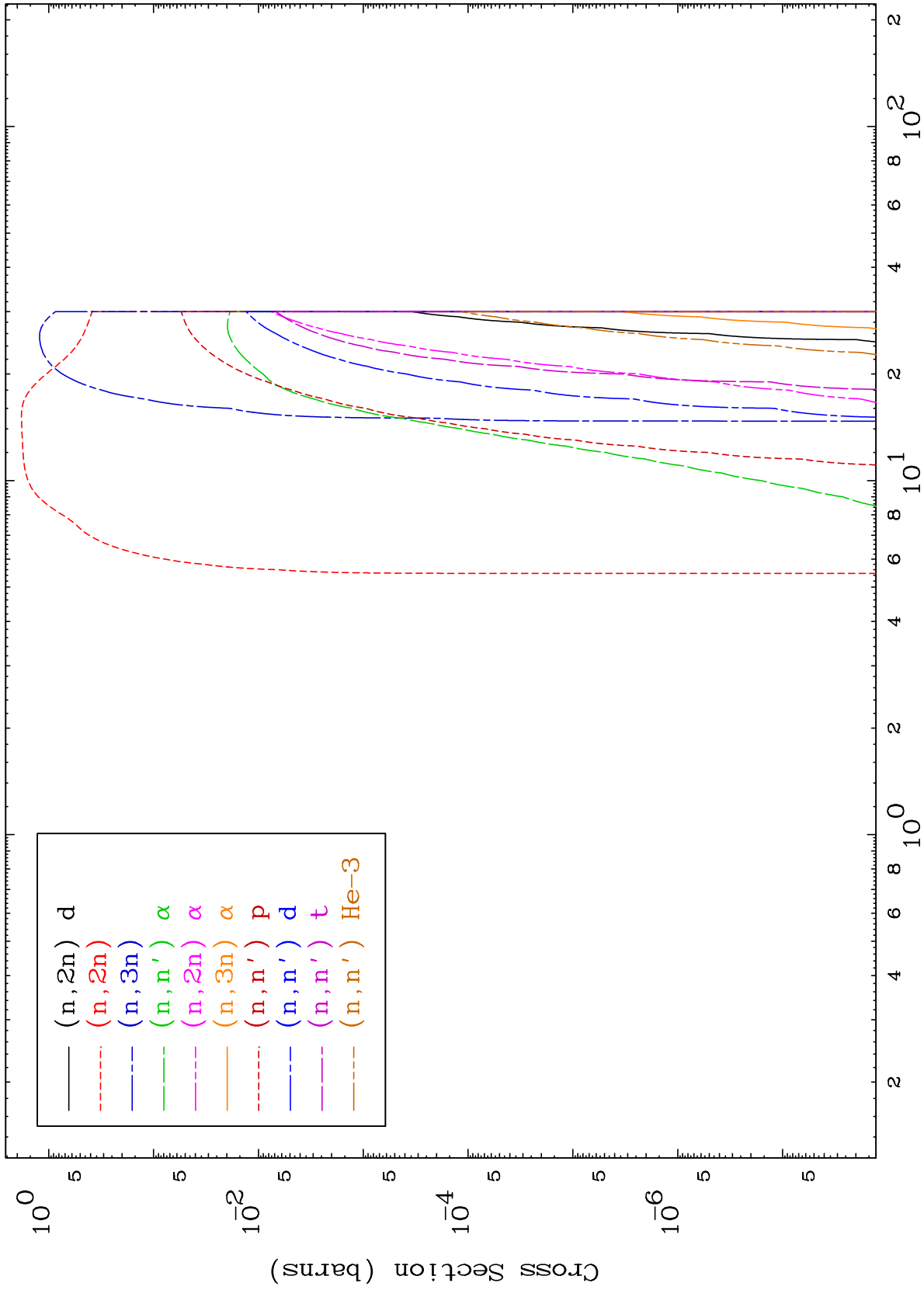
58-Ce-141



MAT 5840

Neutron Absorption
293 Kelvin Cross Sections

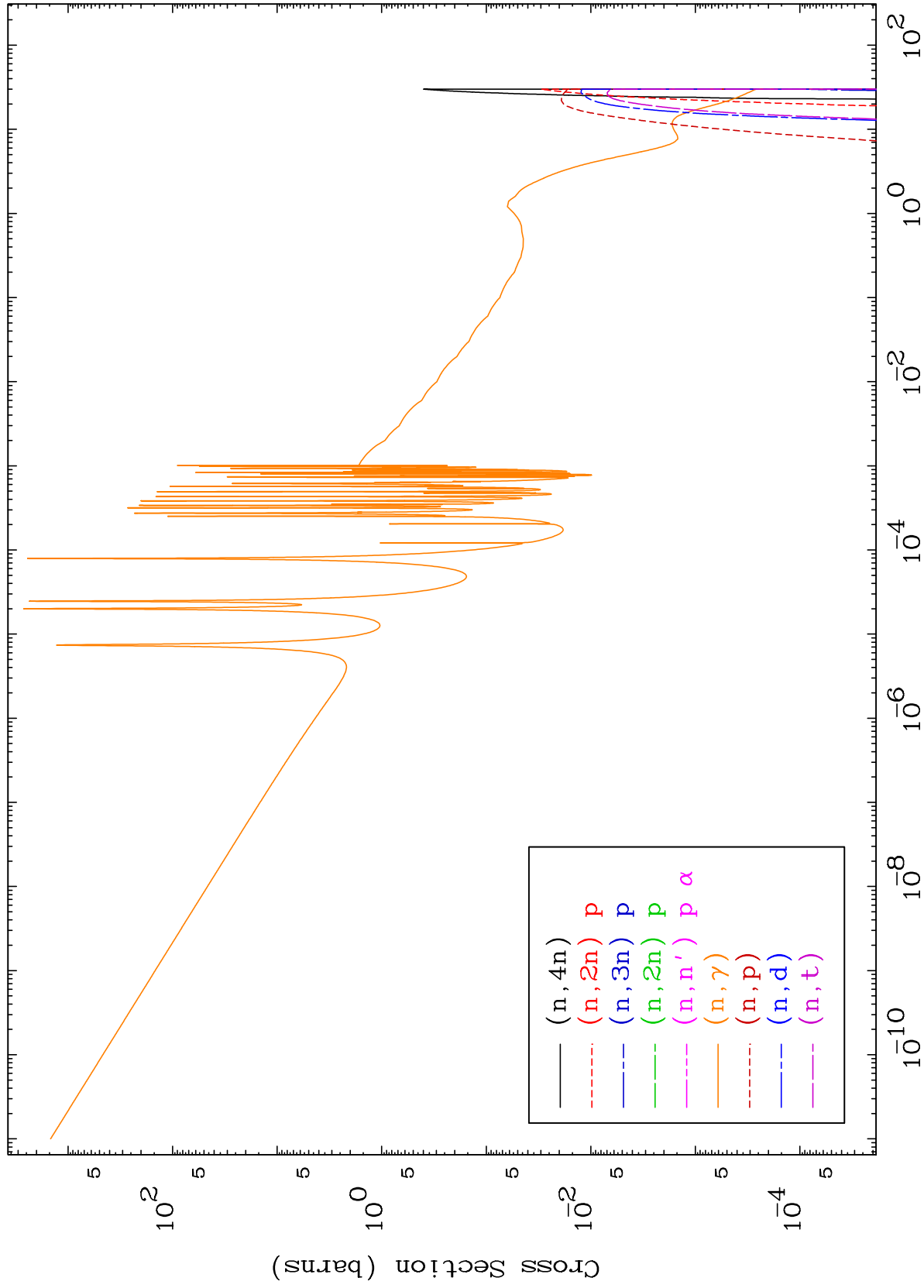
58-Ce-141



MAT 5840

Neutron Absorption
293 Kelvin Cross Sections

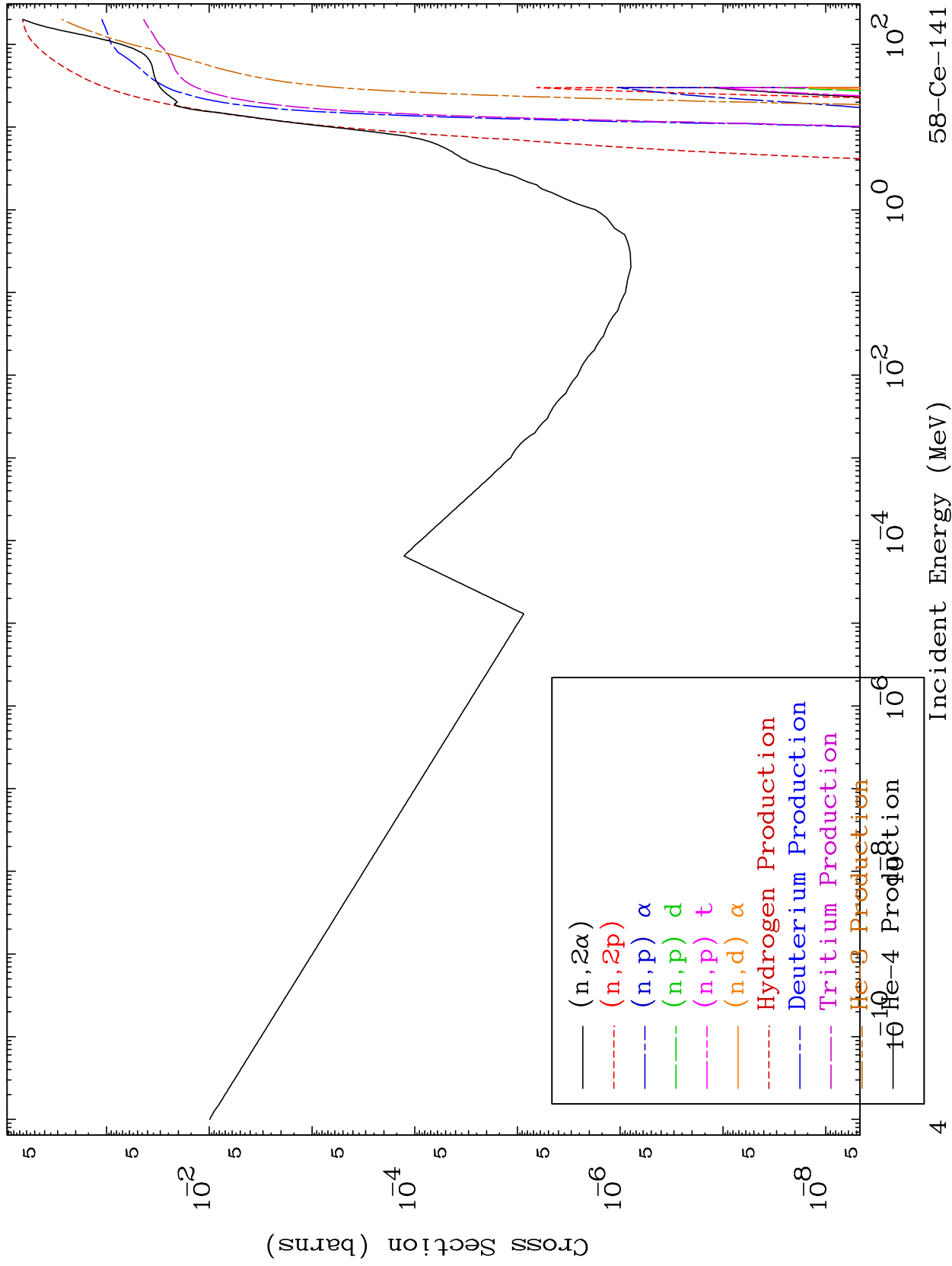
58-Ce-141

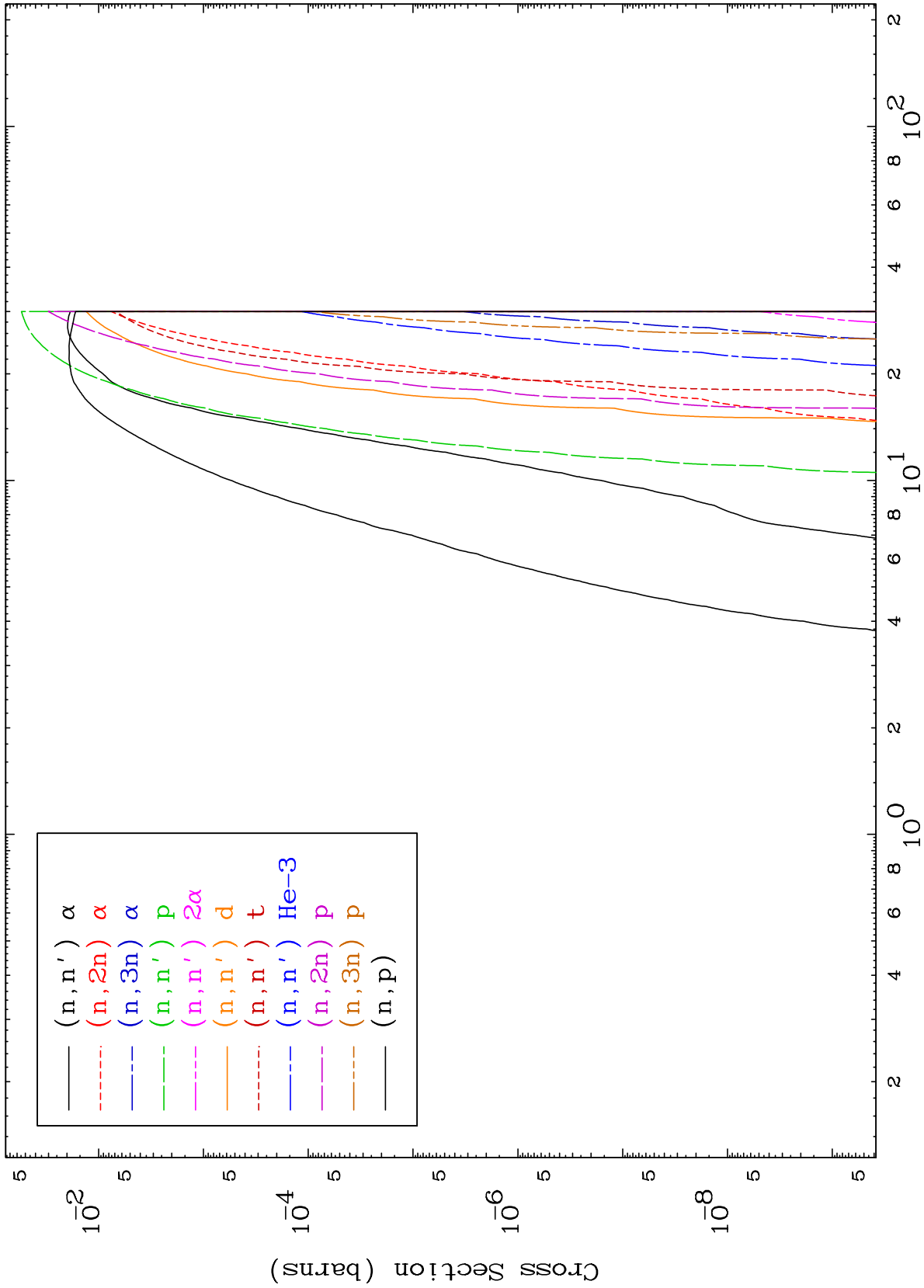


MAT 5840

Neutron Absorption
293 Kelvin Cross Sections

58-Ce-141

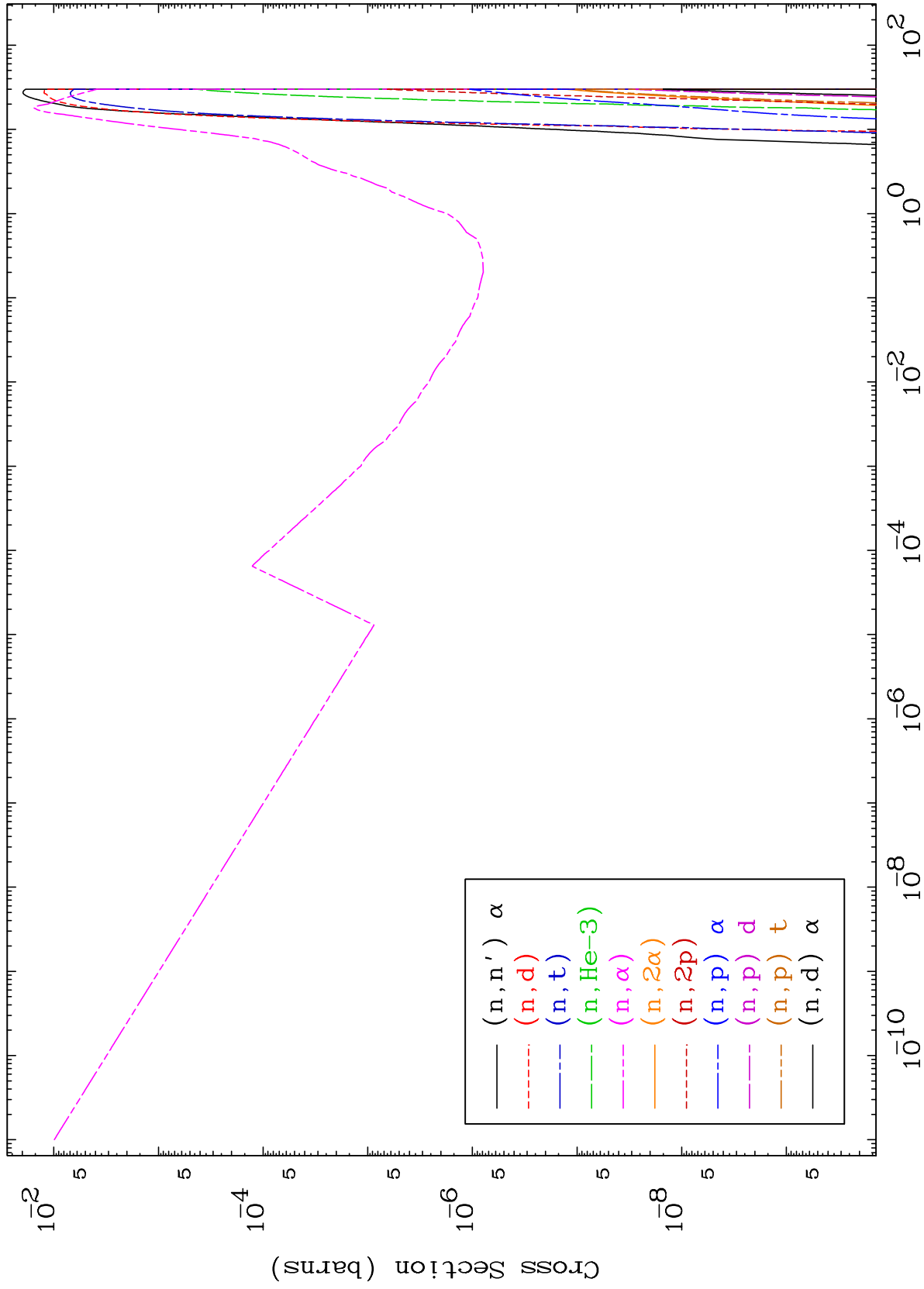


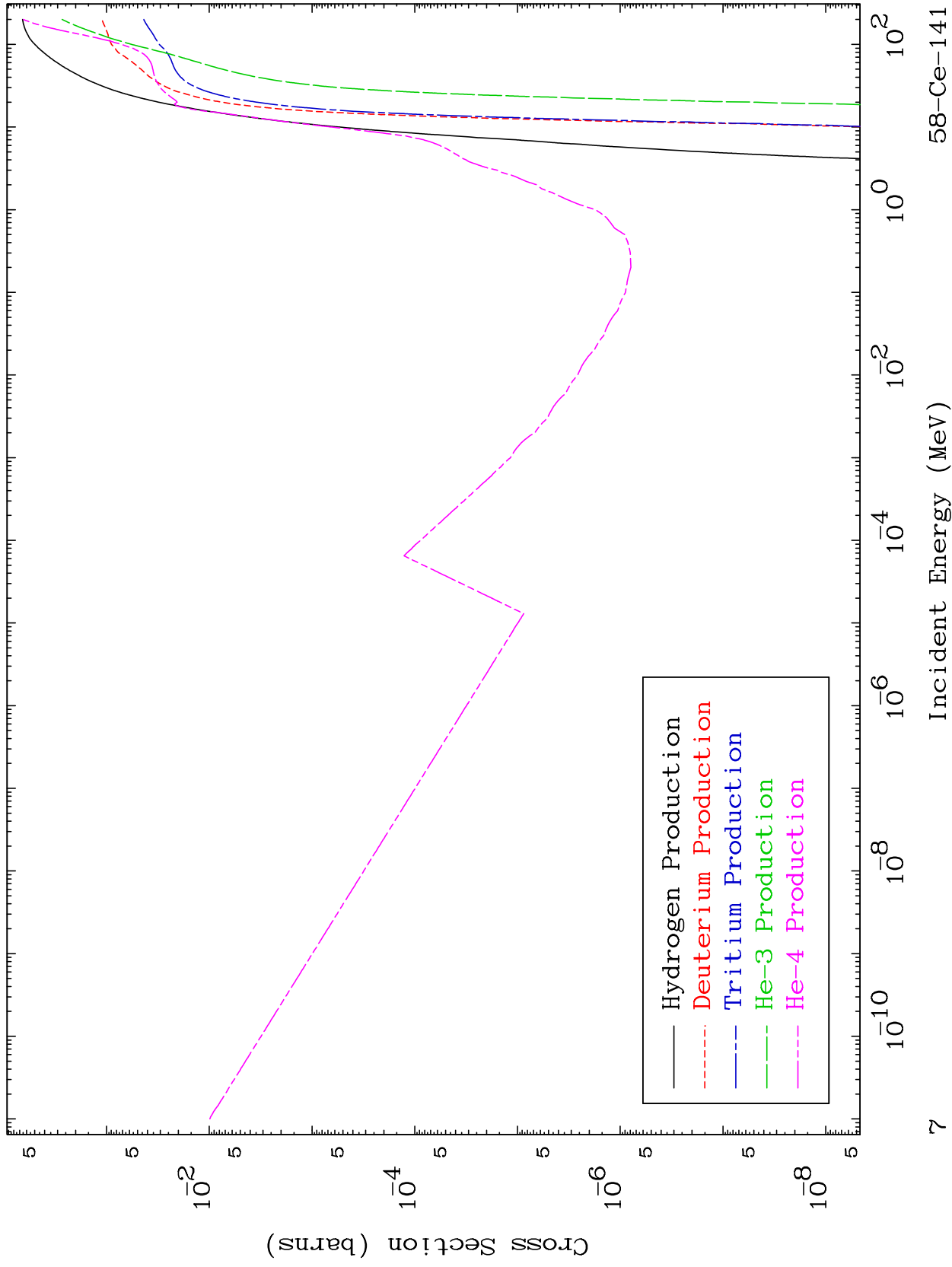


MAT 5840

Charged Particle
293 Kelvin Cross Sections

58-Ce-141



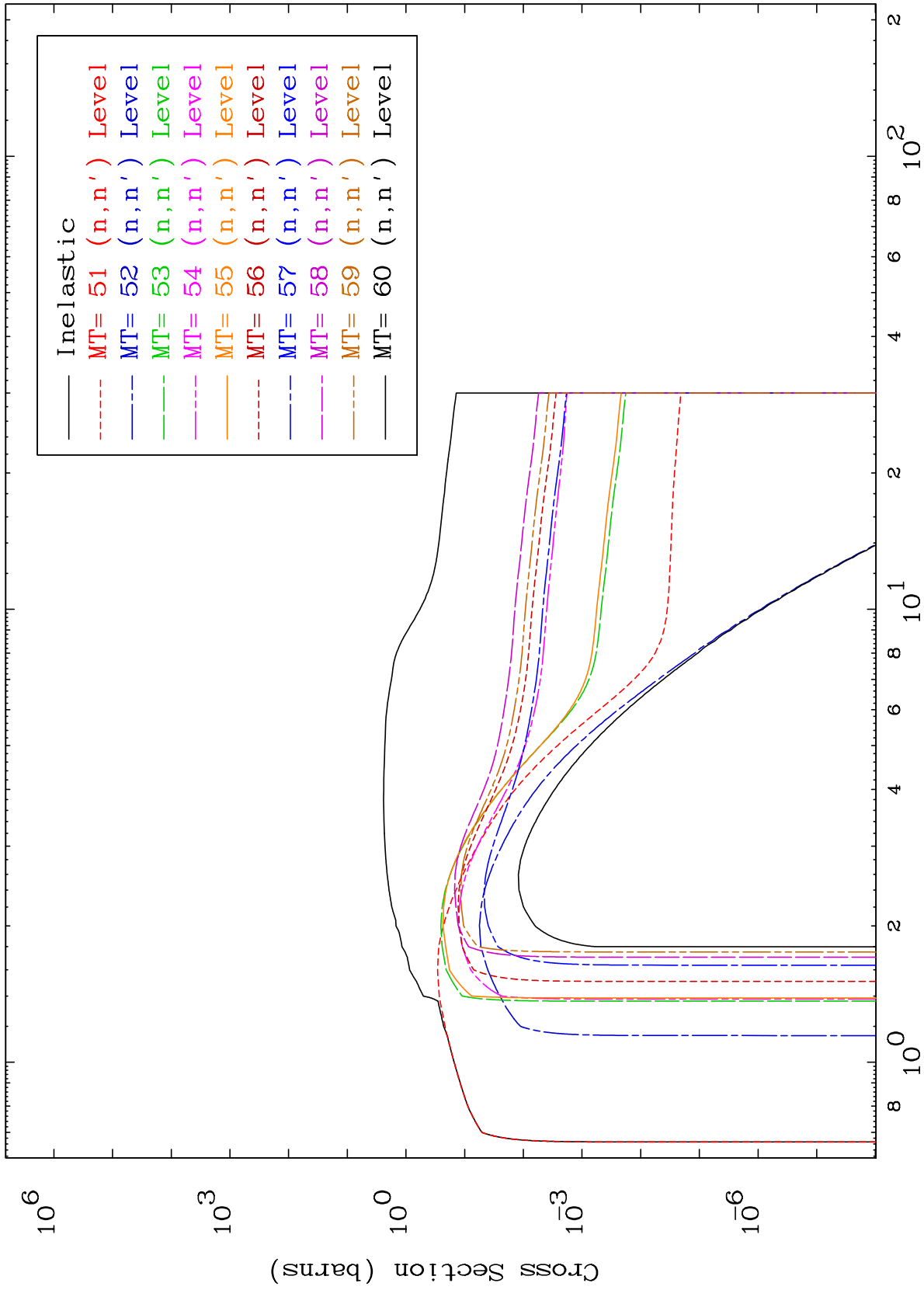


MAT 5840

(n,n') Levels

58-Ce-141

293 Kelvin Cross Sections



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Incident Energy (MeV)

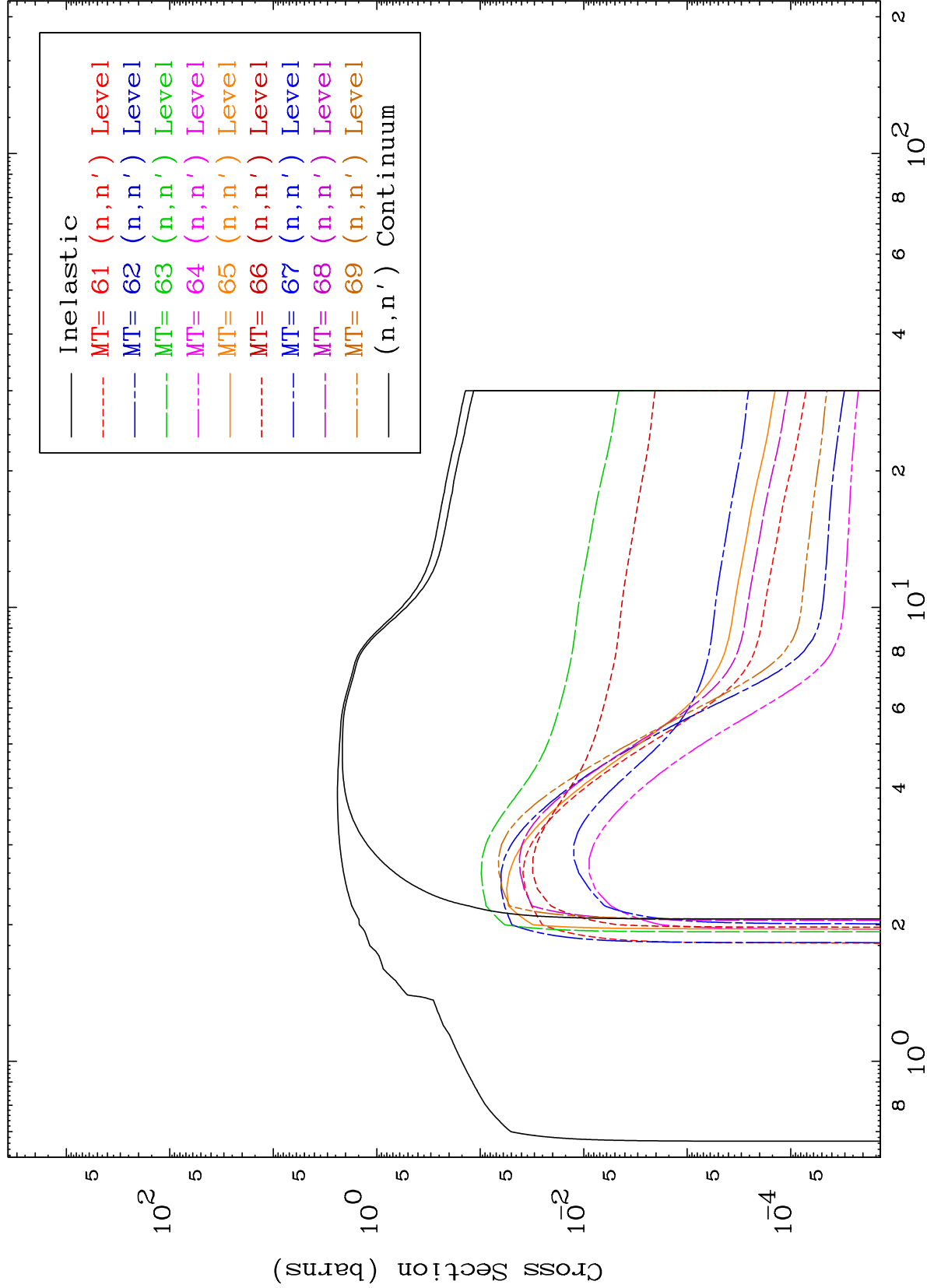
58-Ce-141

MAT 5840

(n,n') Levels

58-Ce-141

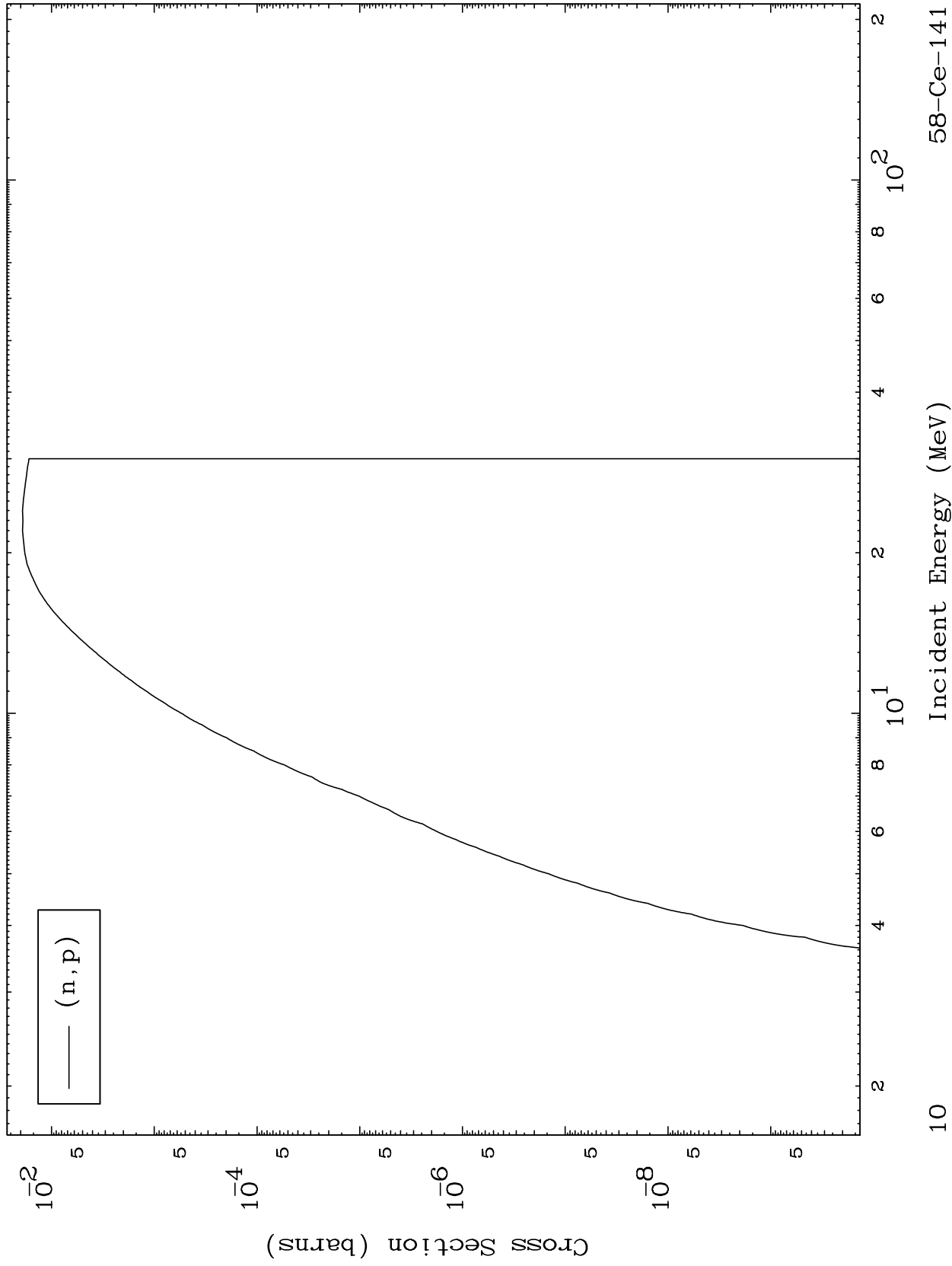
293 Kelvin Cross Sections



MAT 5840

(n,p) Levels
293 Kelvin Cross Sections

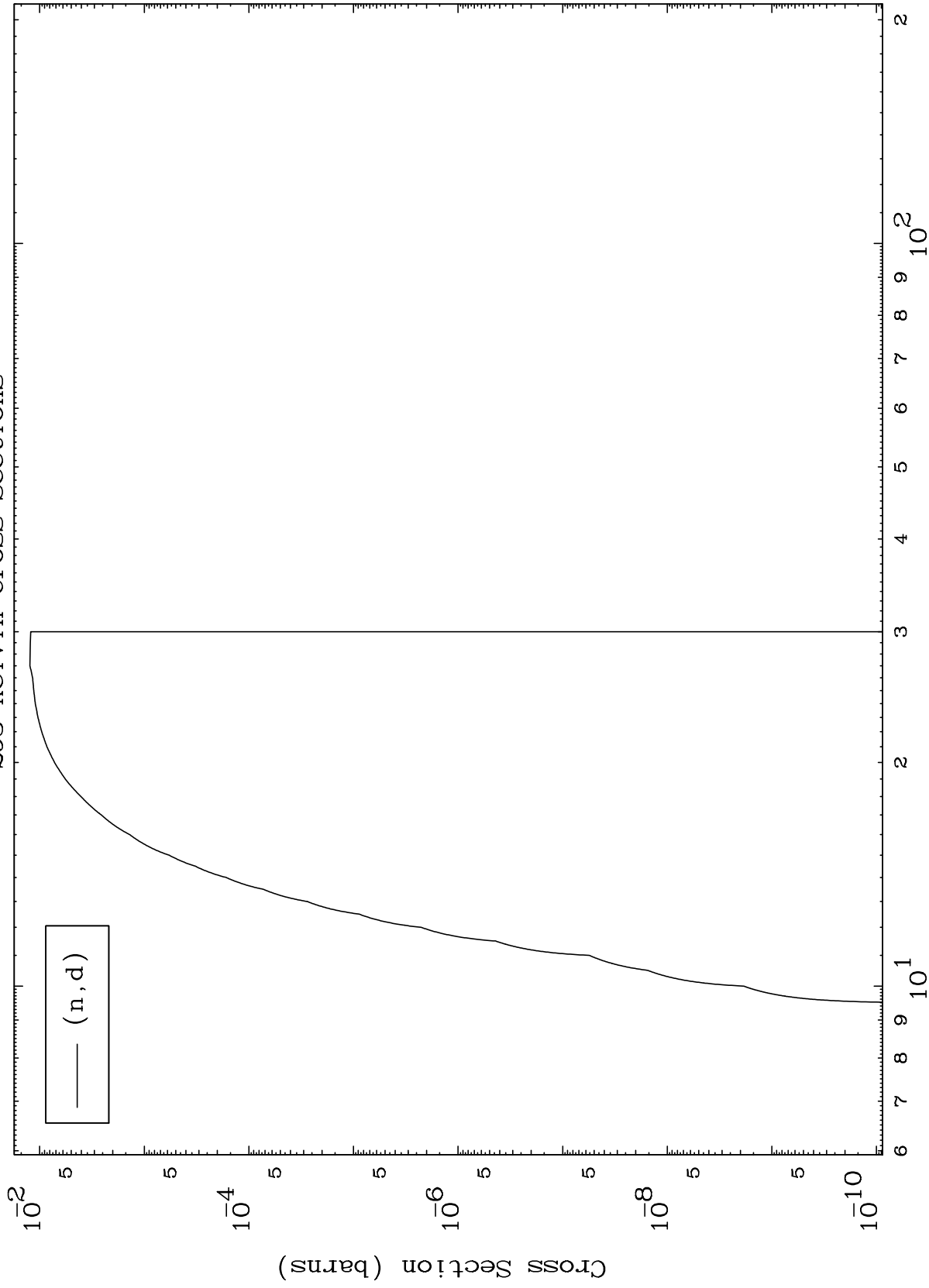
58-Ce-141



MAT 5840

58-Ce-141

(n,d) Levels
293 Kelvin Cross Sections



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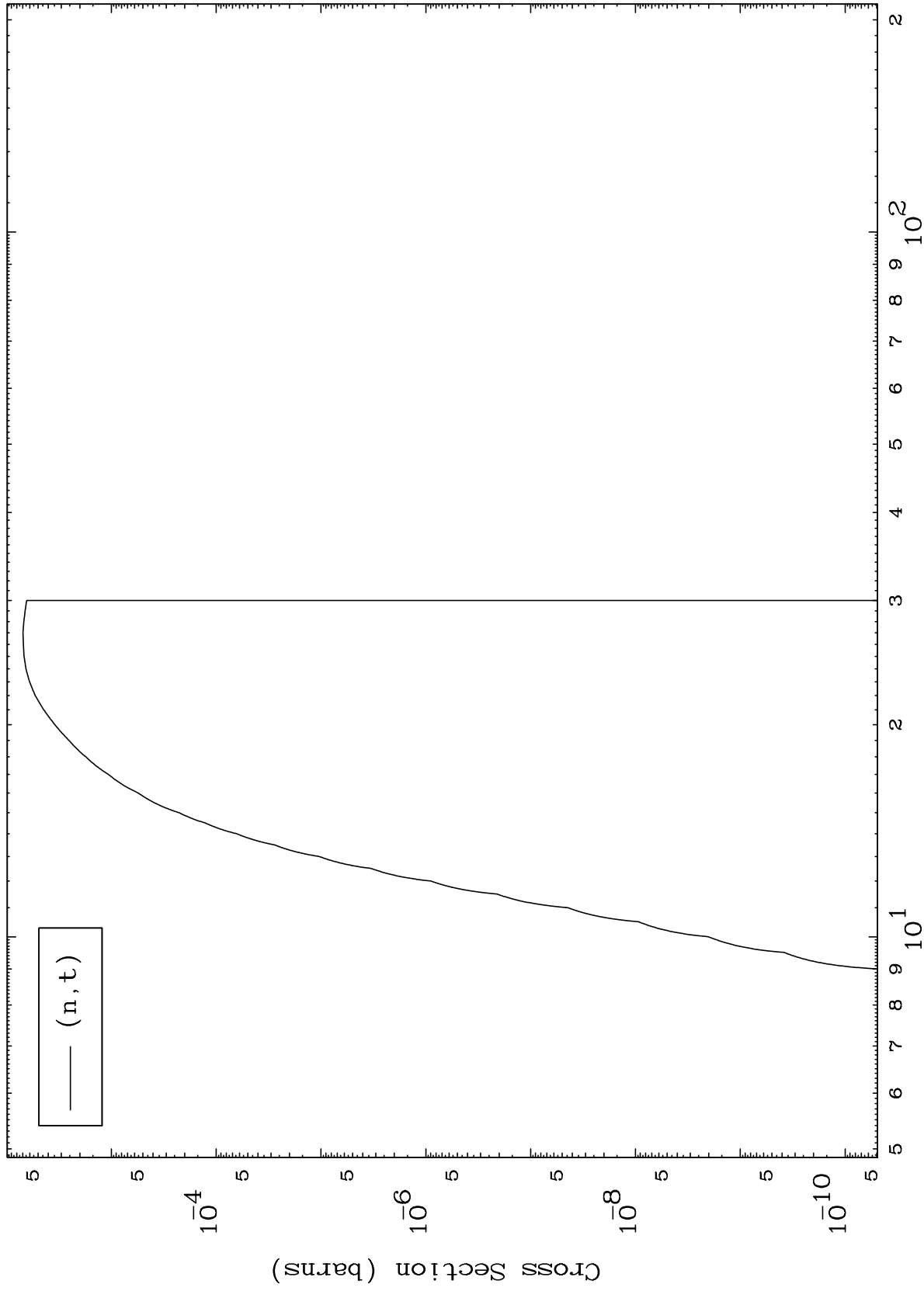
Incident Energy (MeV)

58-Ce-141

MAT 5840

58-Ce-141

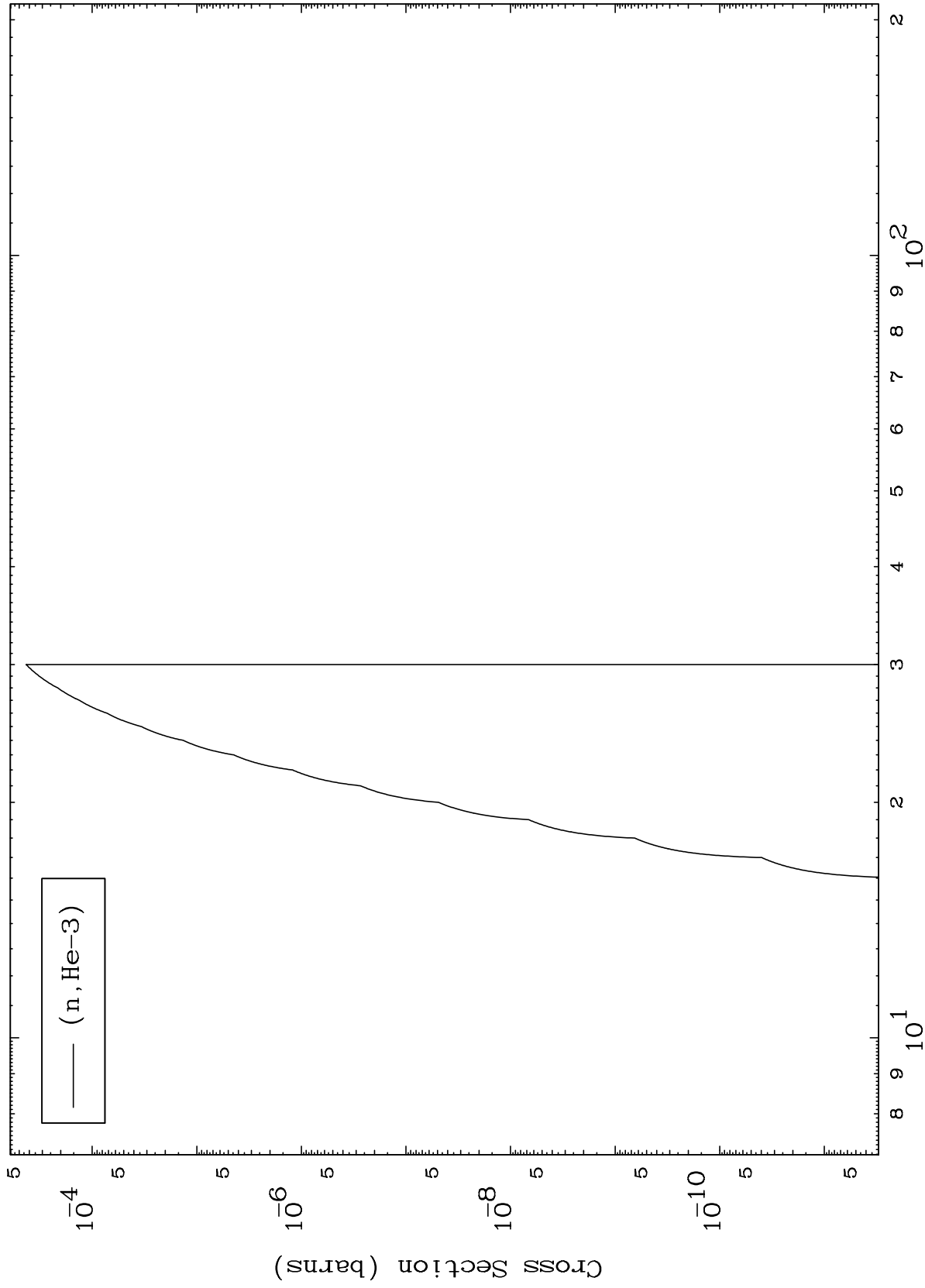
(n,t) Levels
293 Kelvin Cross Sections



MAT 5840

58-Ce-141

(n,He3) Levels
293 Kelvin Cross Sections



13

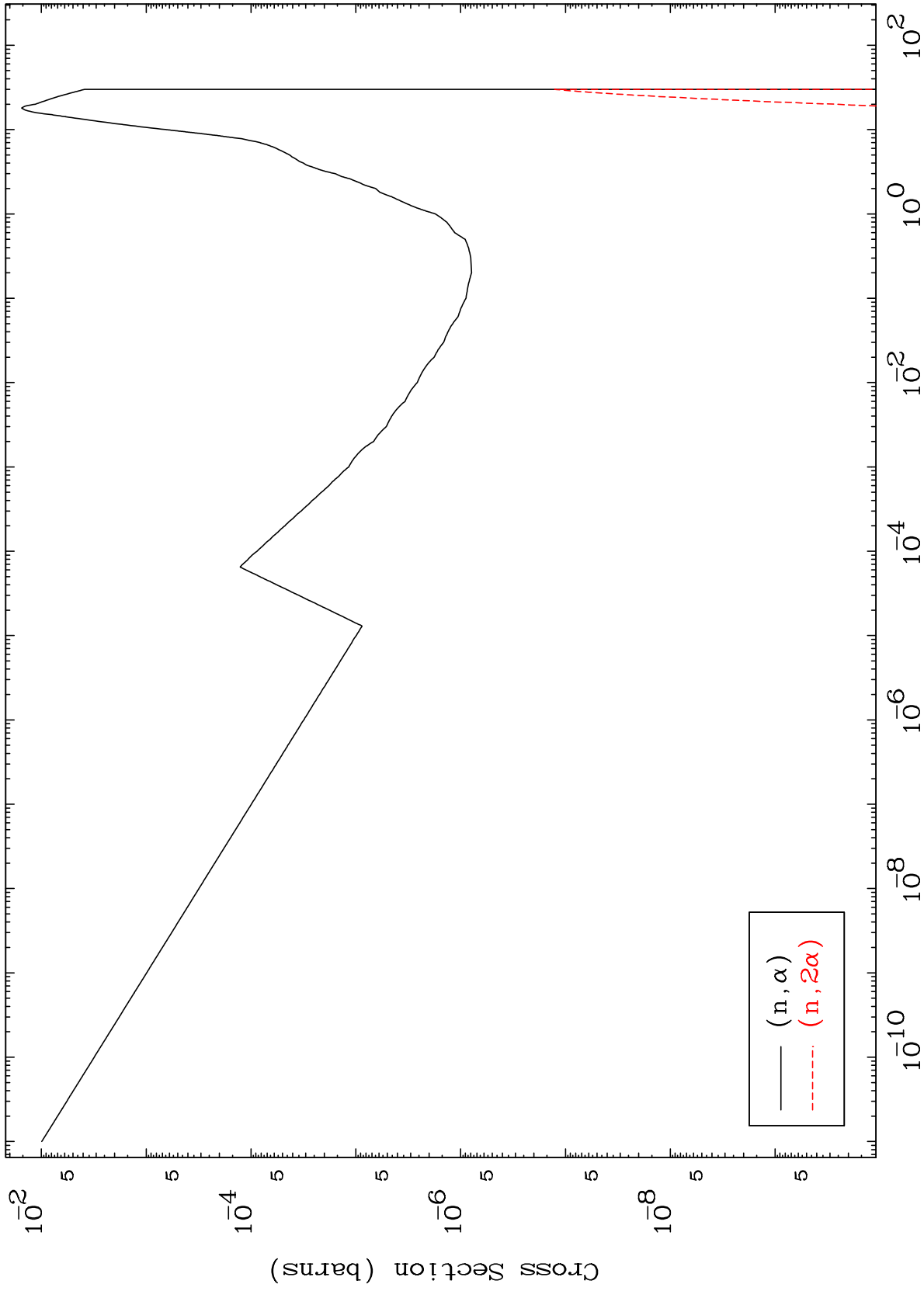
Incident Energy (MeV)

58-Ce-141

MAT 5840

(n,α) Levels
293 Kelvin Cross Sections

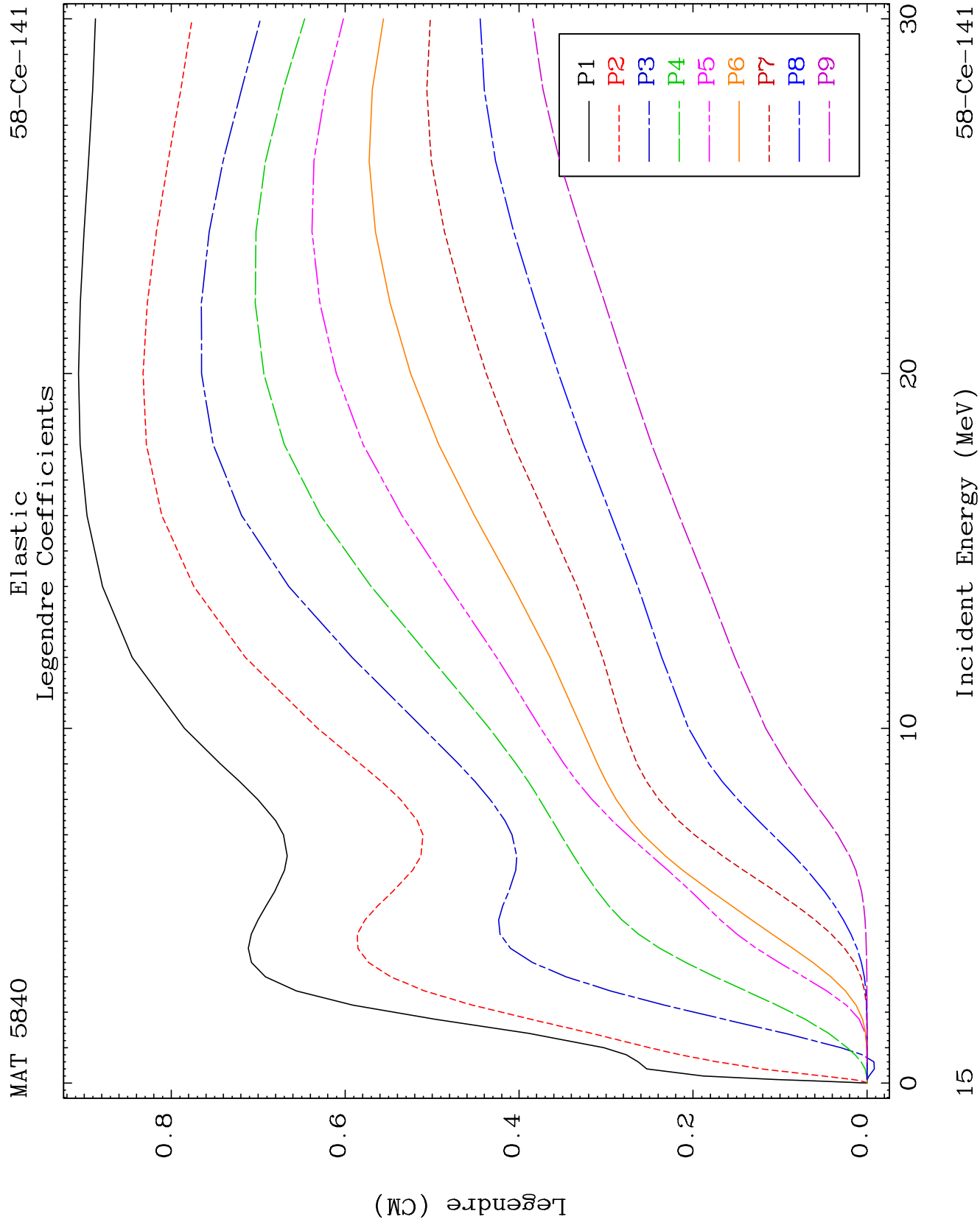
58-Ce-141

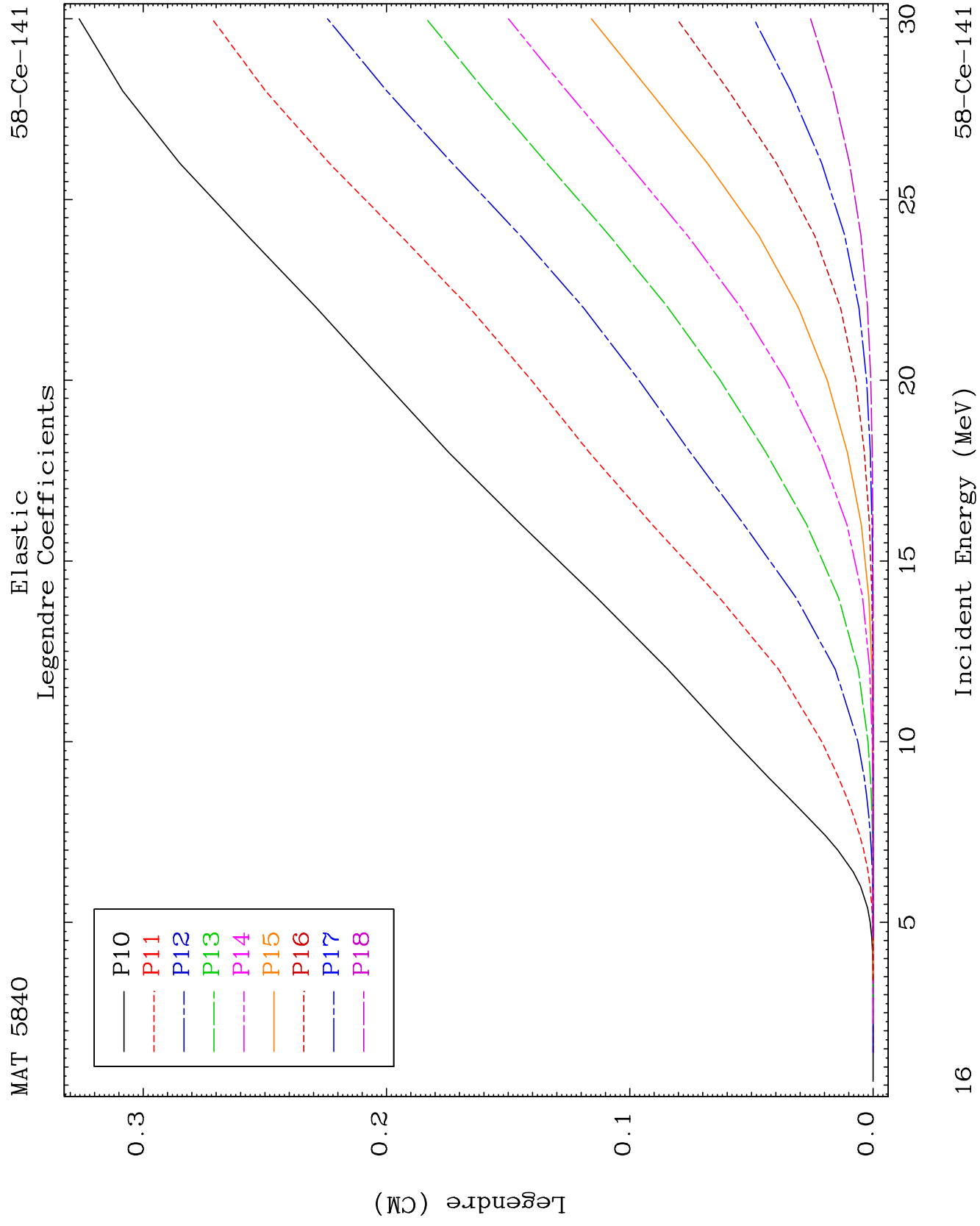


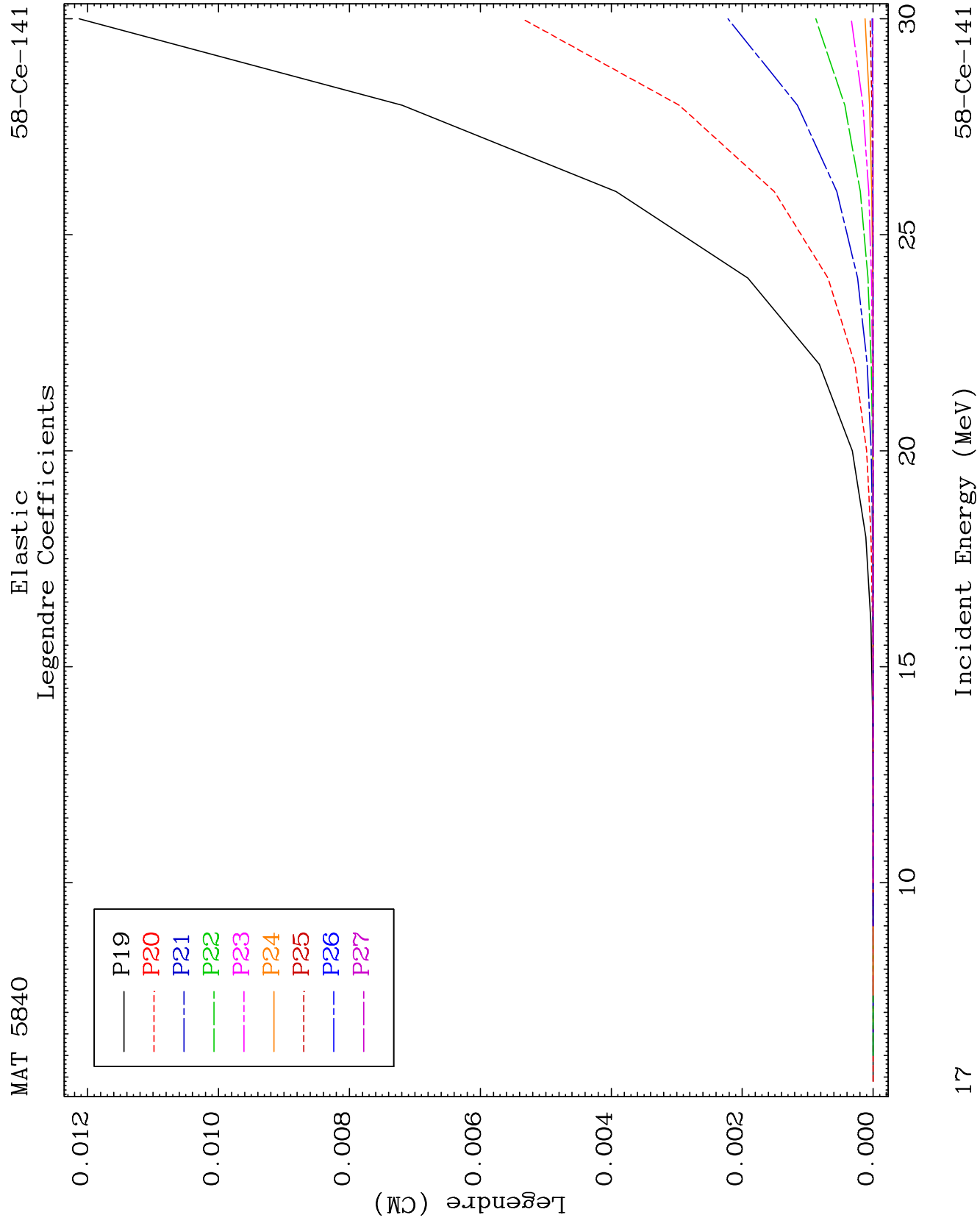
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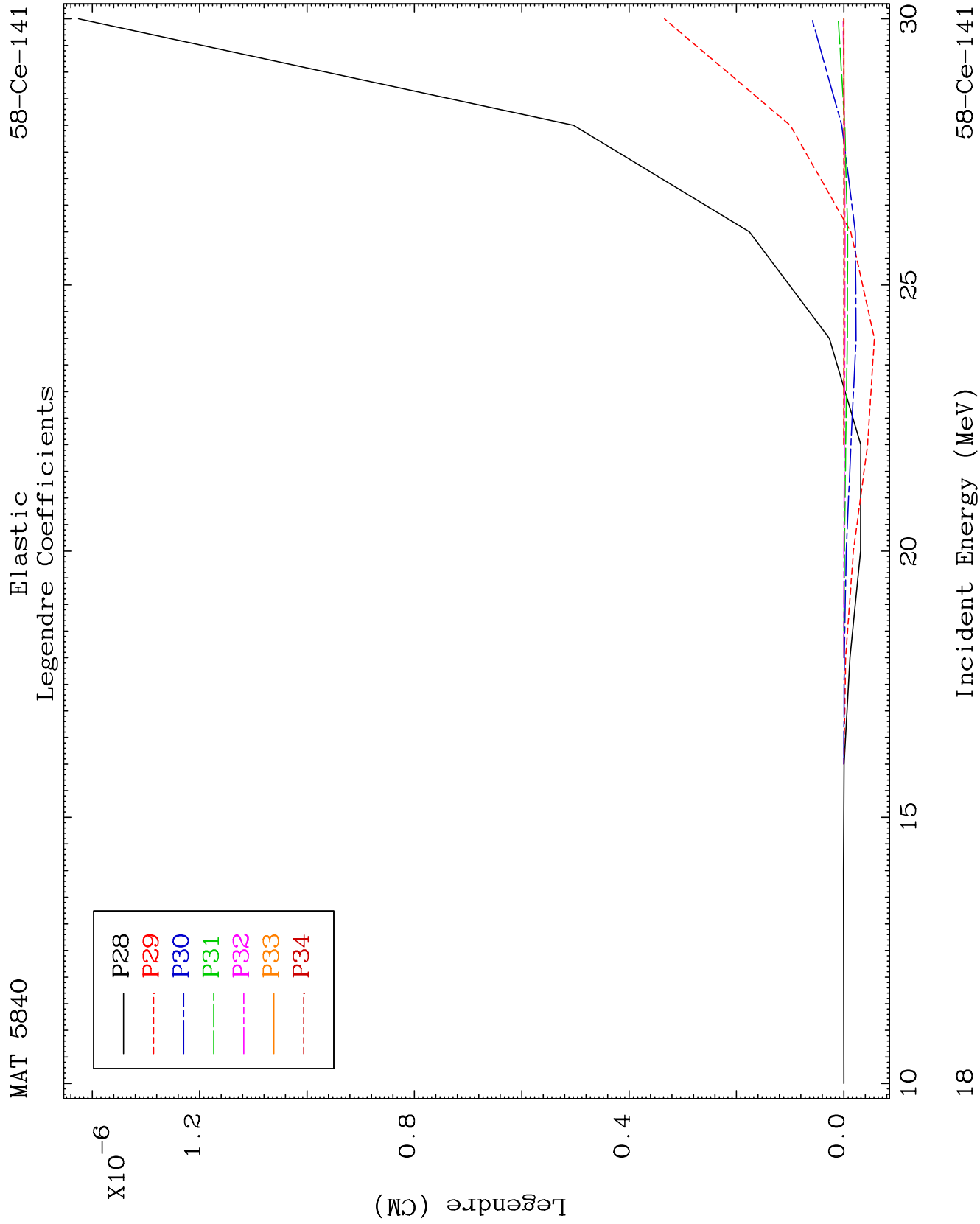
Incident Energy (MeV)

58-Ce-141





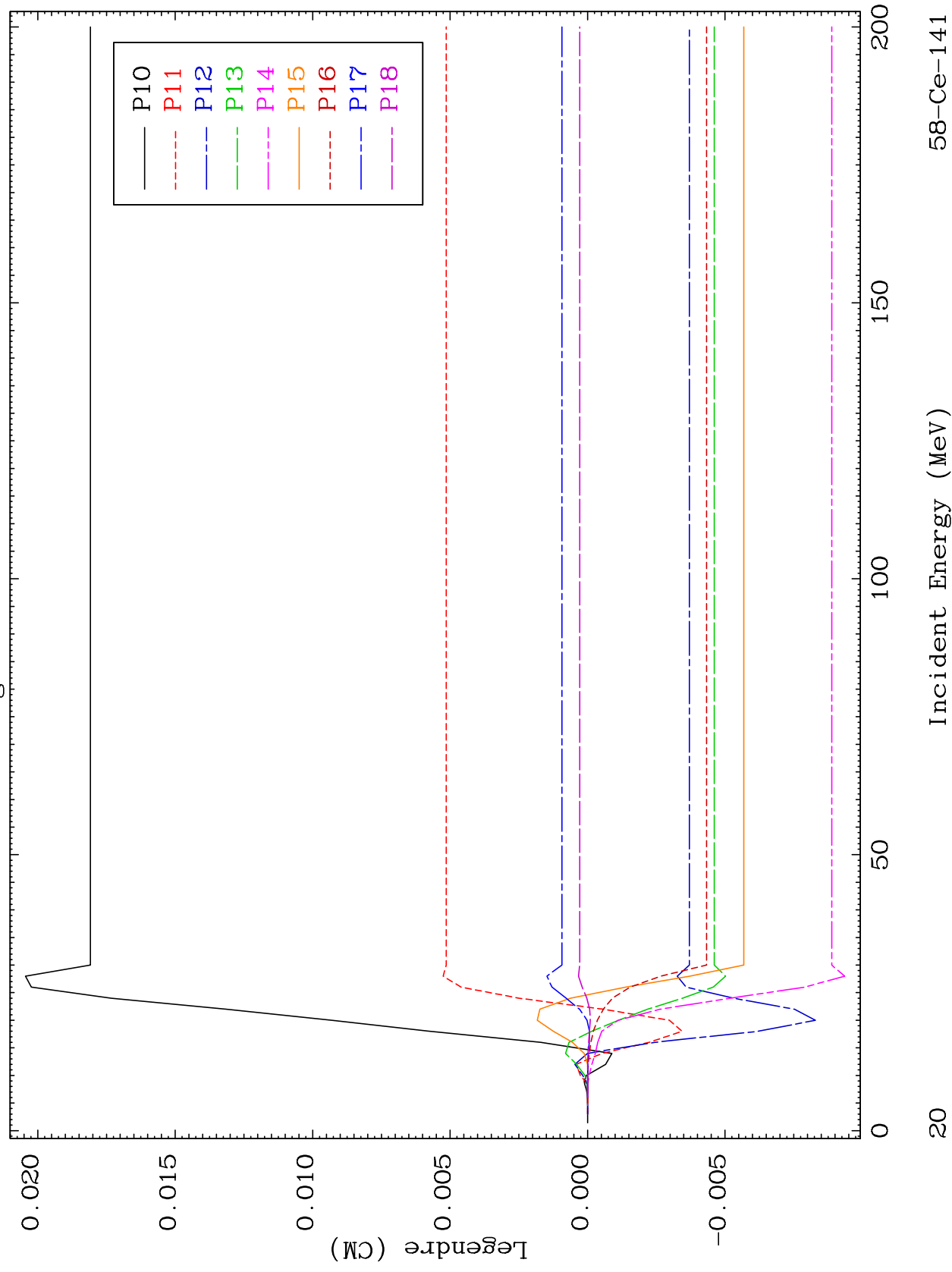




MAT 5840

MT= 51 (n, n') Level
Legendre Coefficients

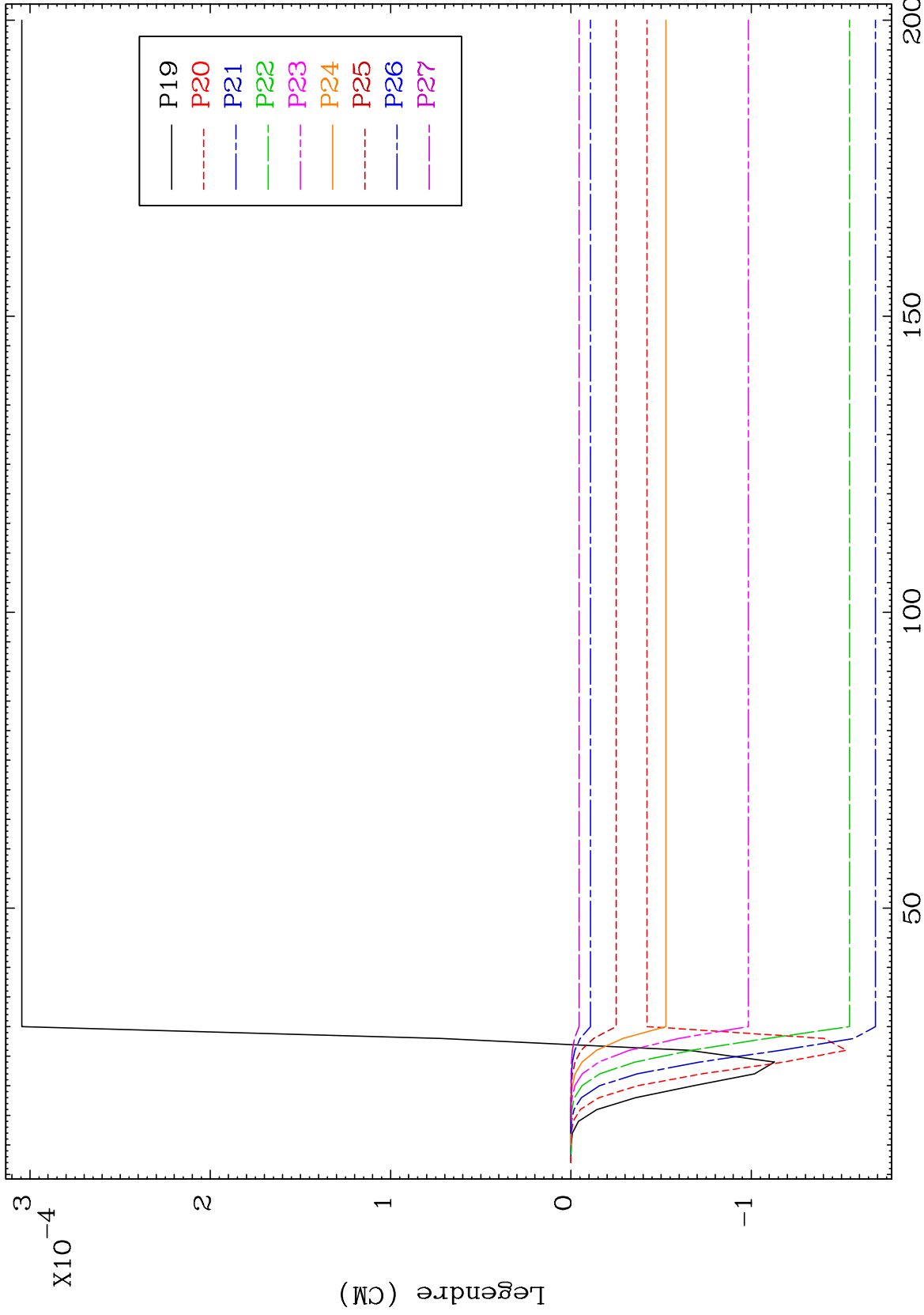
58-Ce-141



MAT 5840

MT= 51 (n,n') Level
Legendre Coefficients

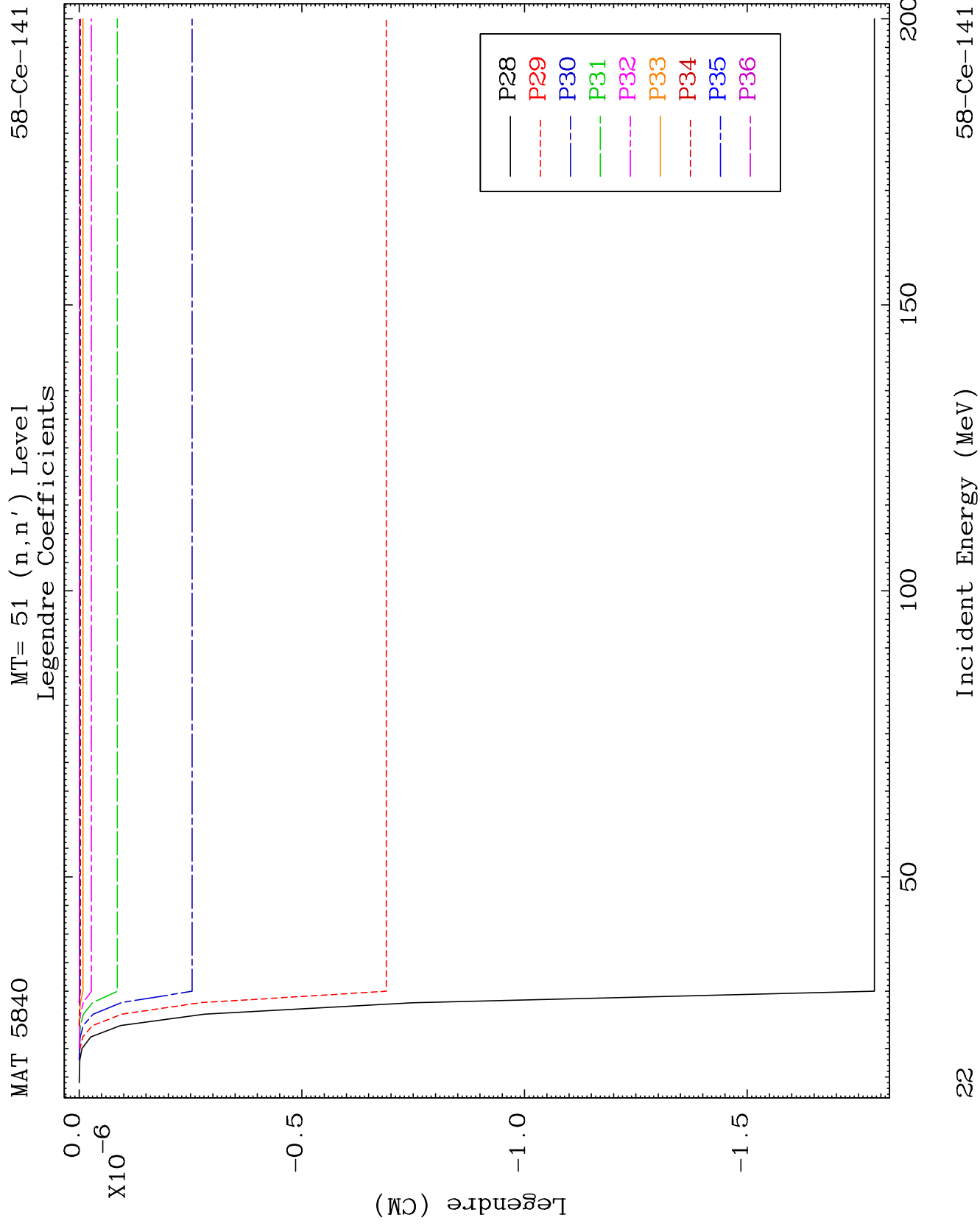
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Incident Energy (MeV)

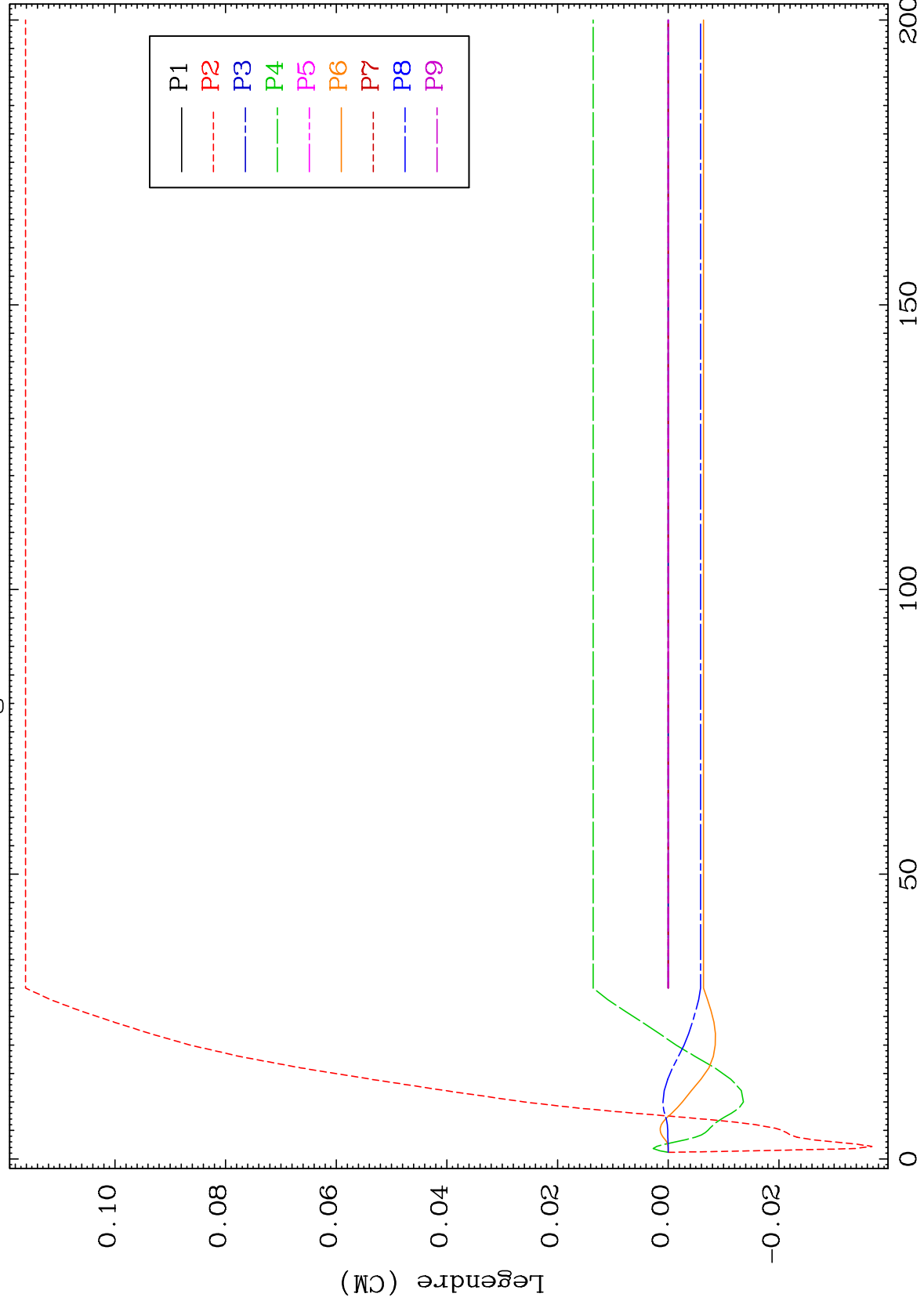
58-Ce-141



MAT 5840

MT= 52 (n, n') Level
Legendre Coefficients

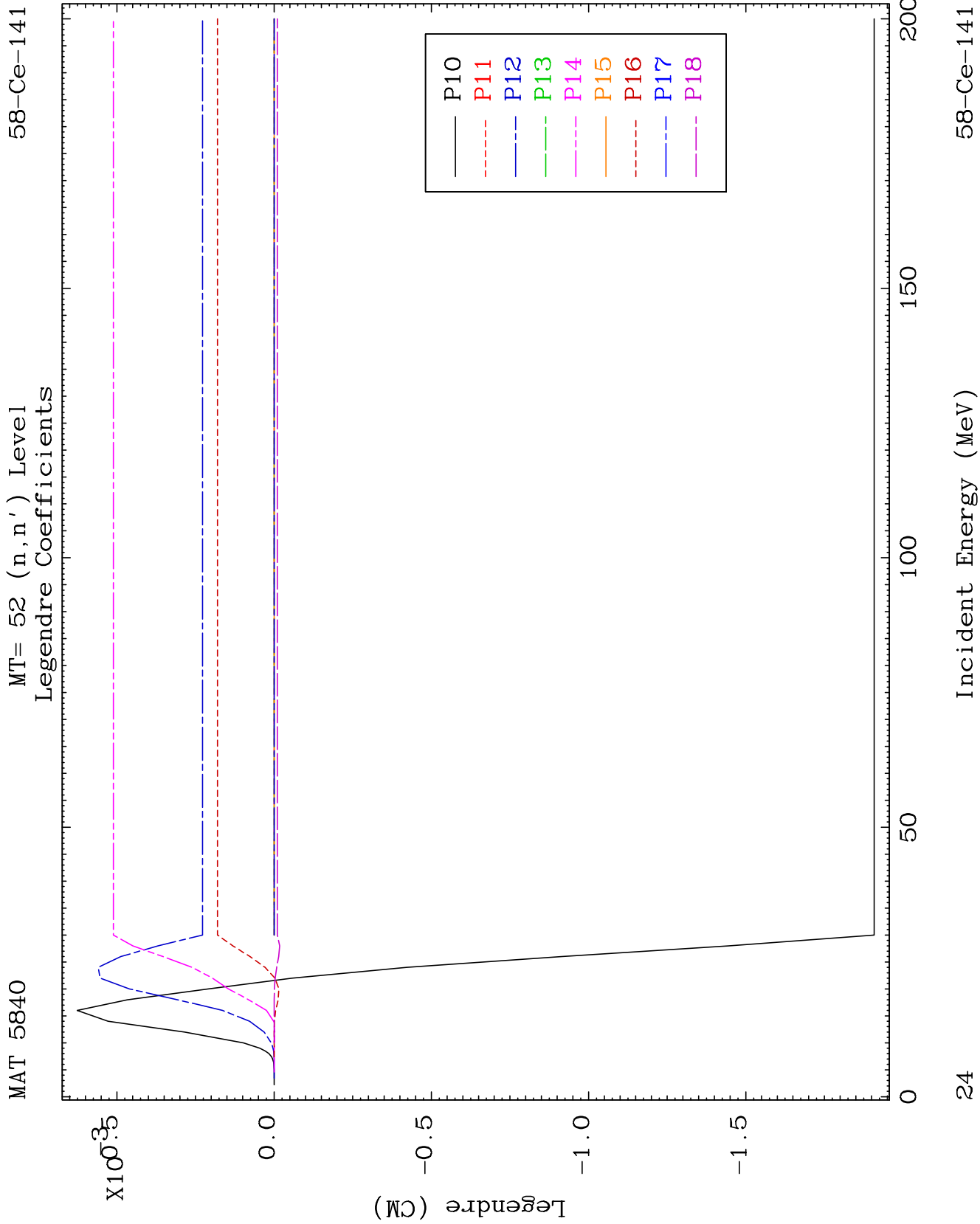
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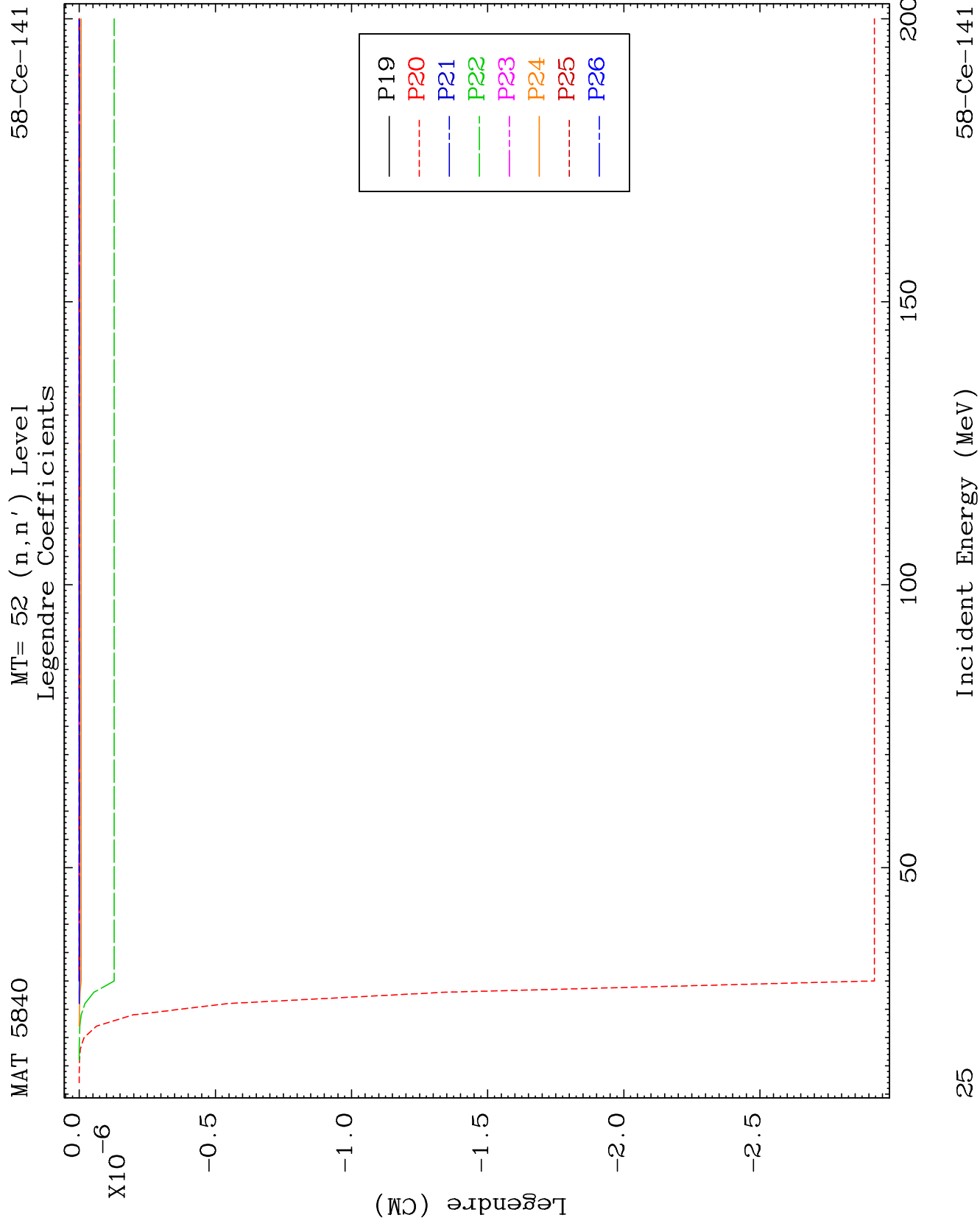


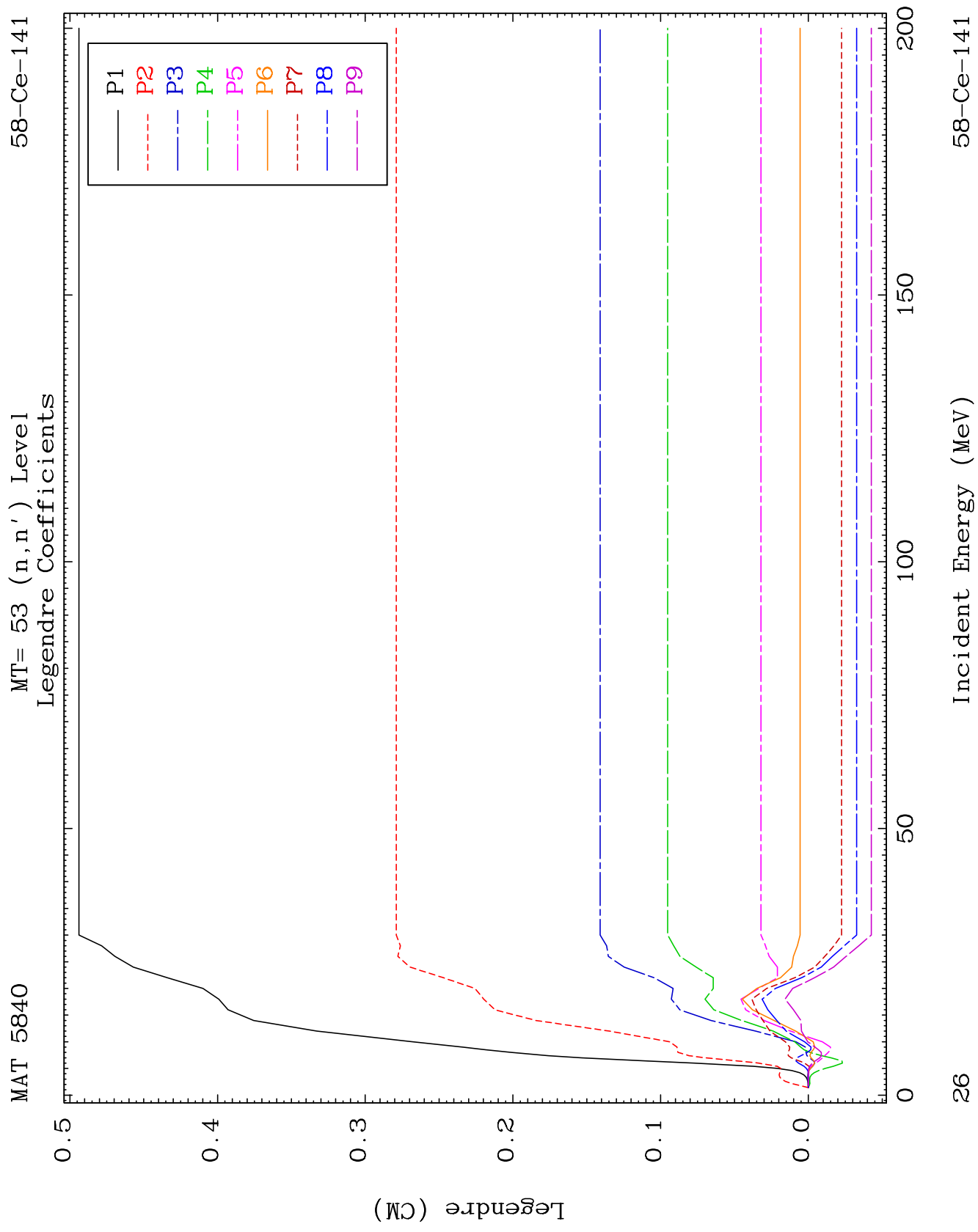
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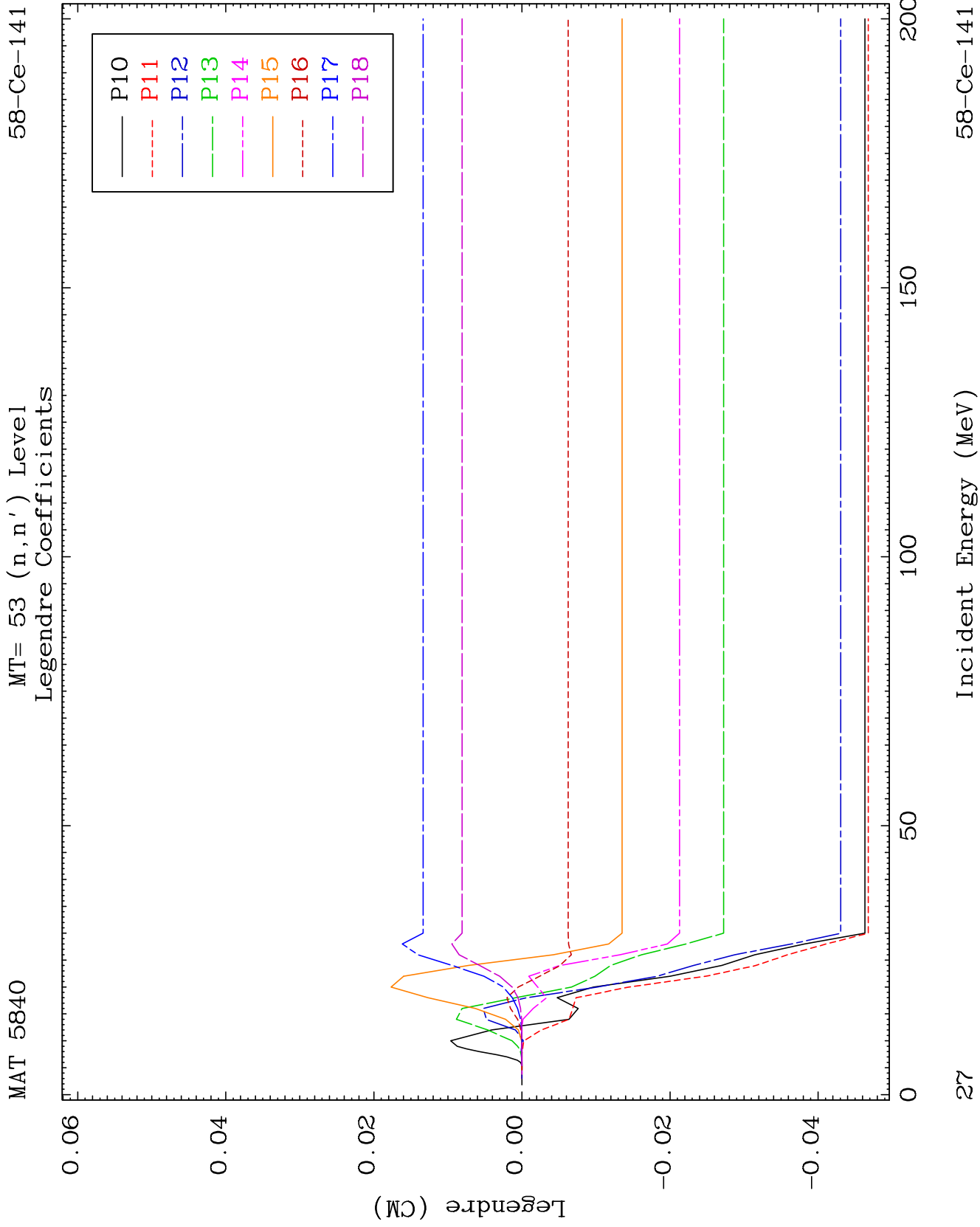
Incident Energy (MeV)

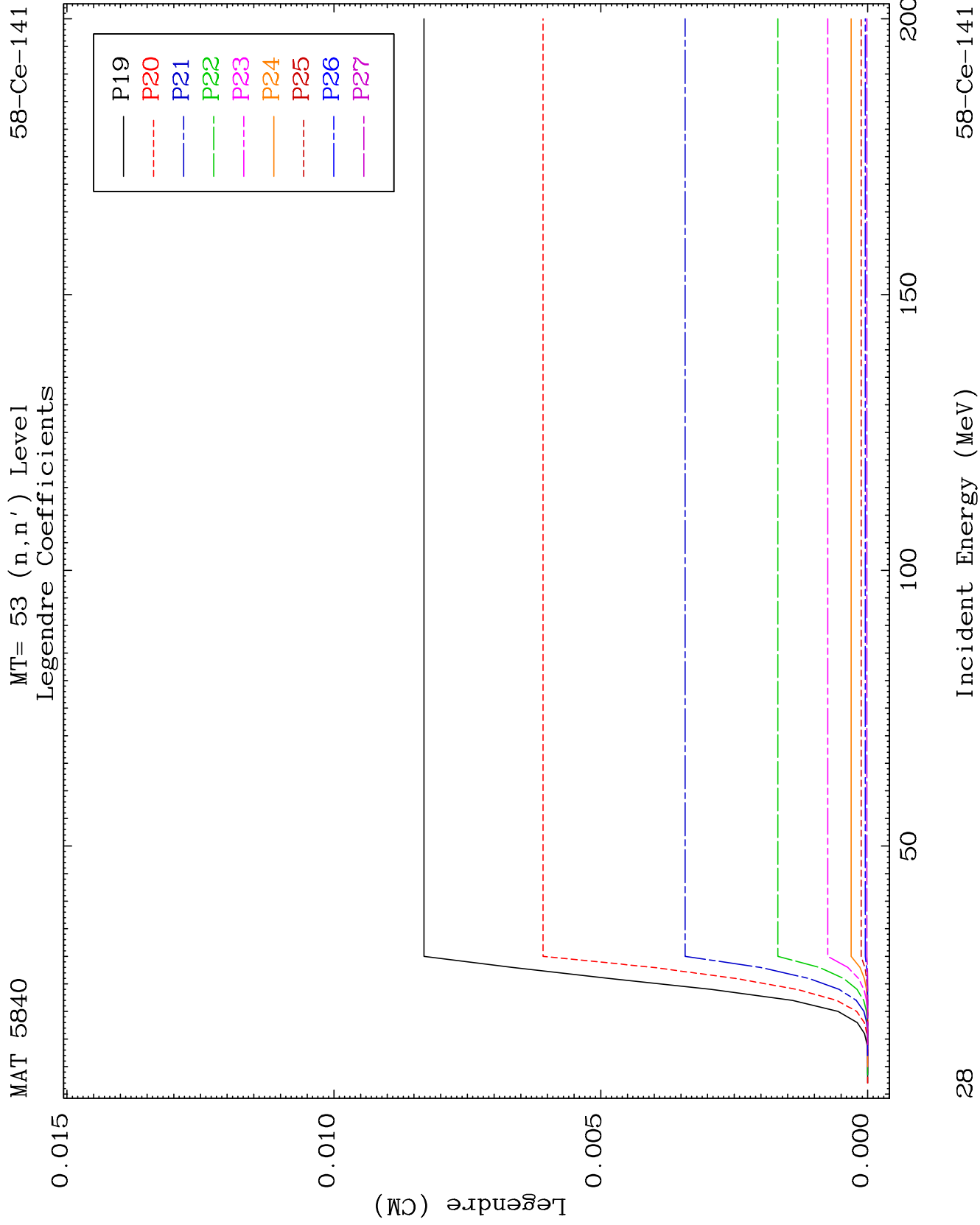
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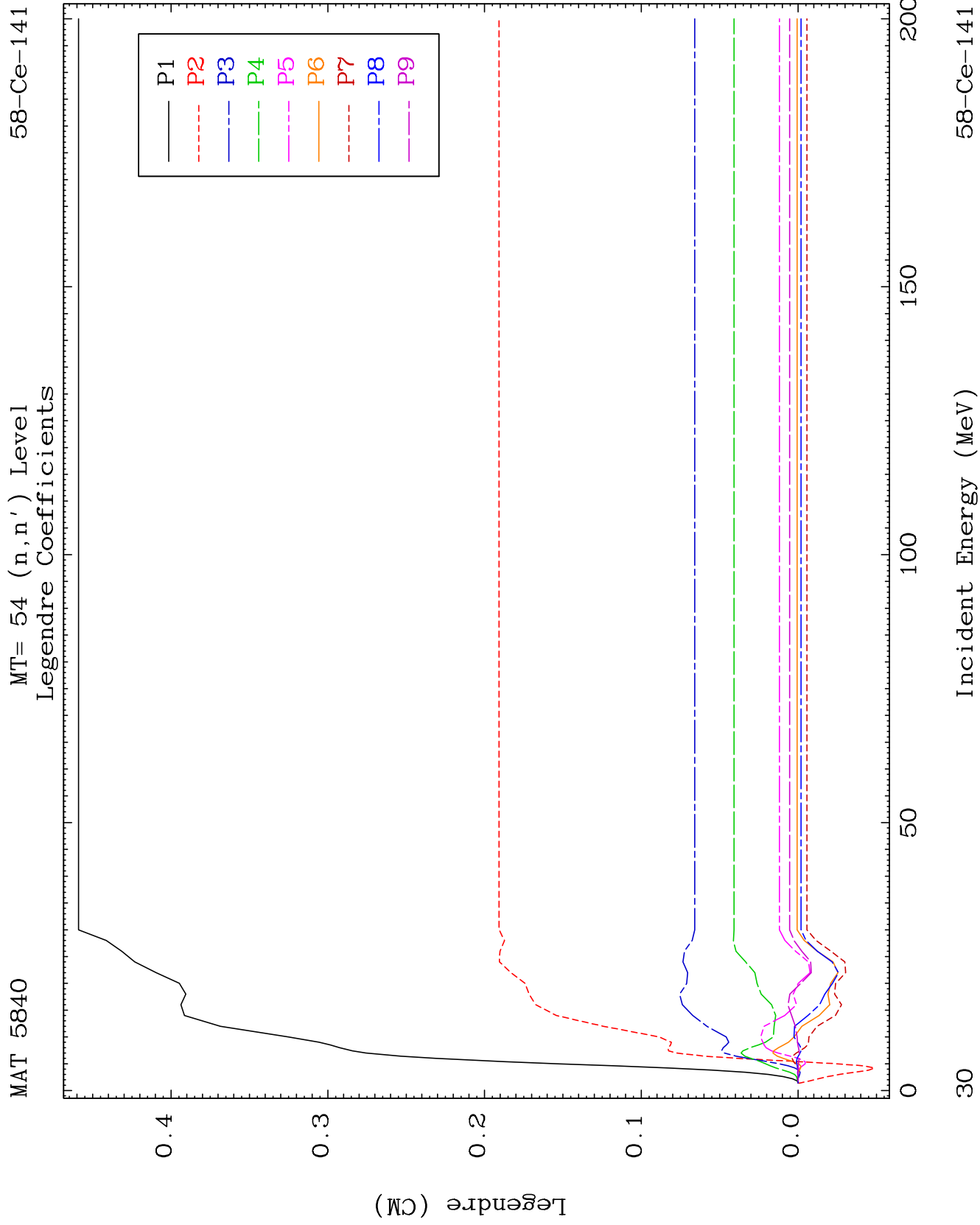








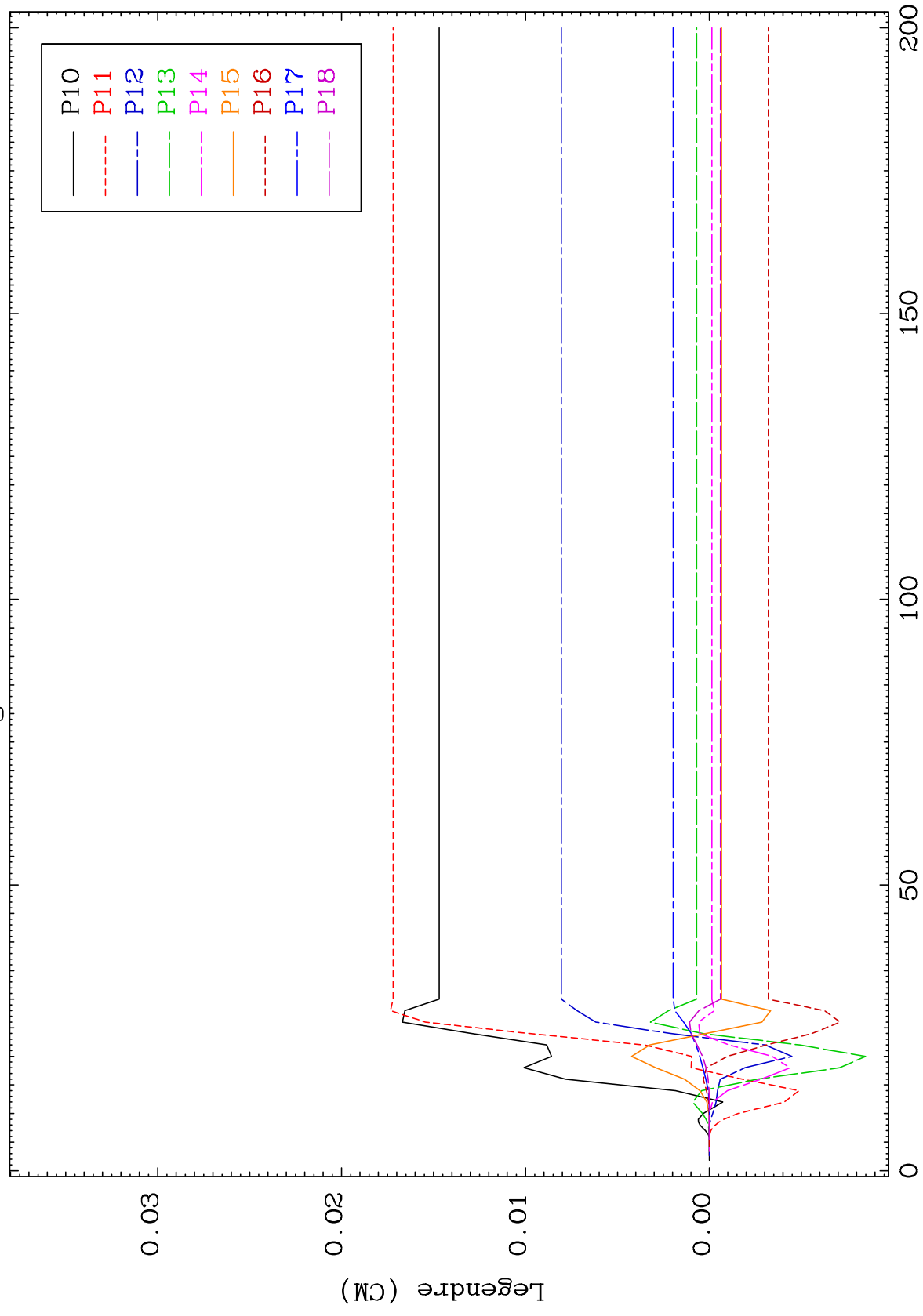




MAT 5840

MT= 54 (n, n') Level
Legendre Coefficients

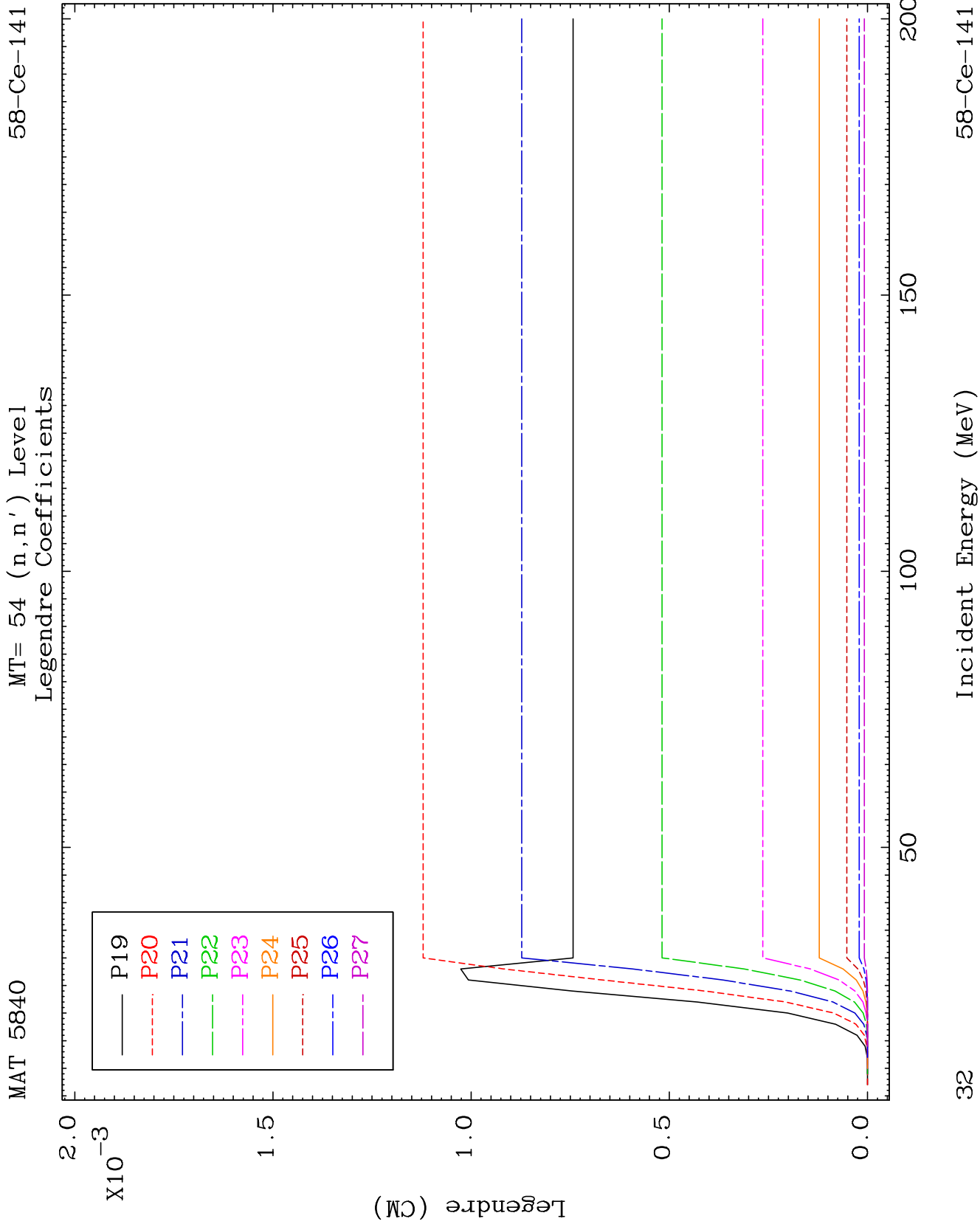
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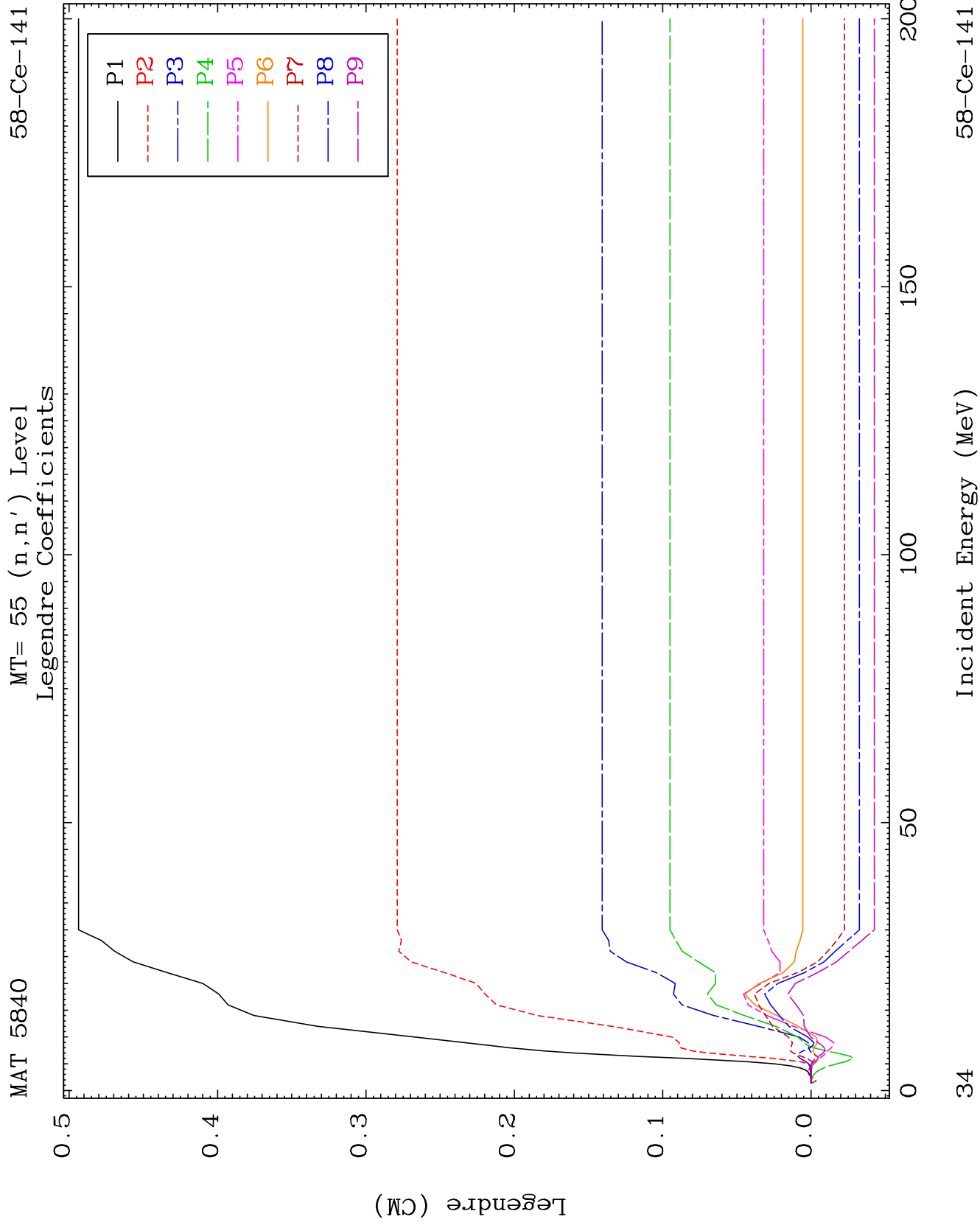


31

Incident Energy (MeV)

58-Ce-141

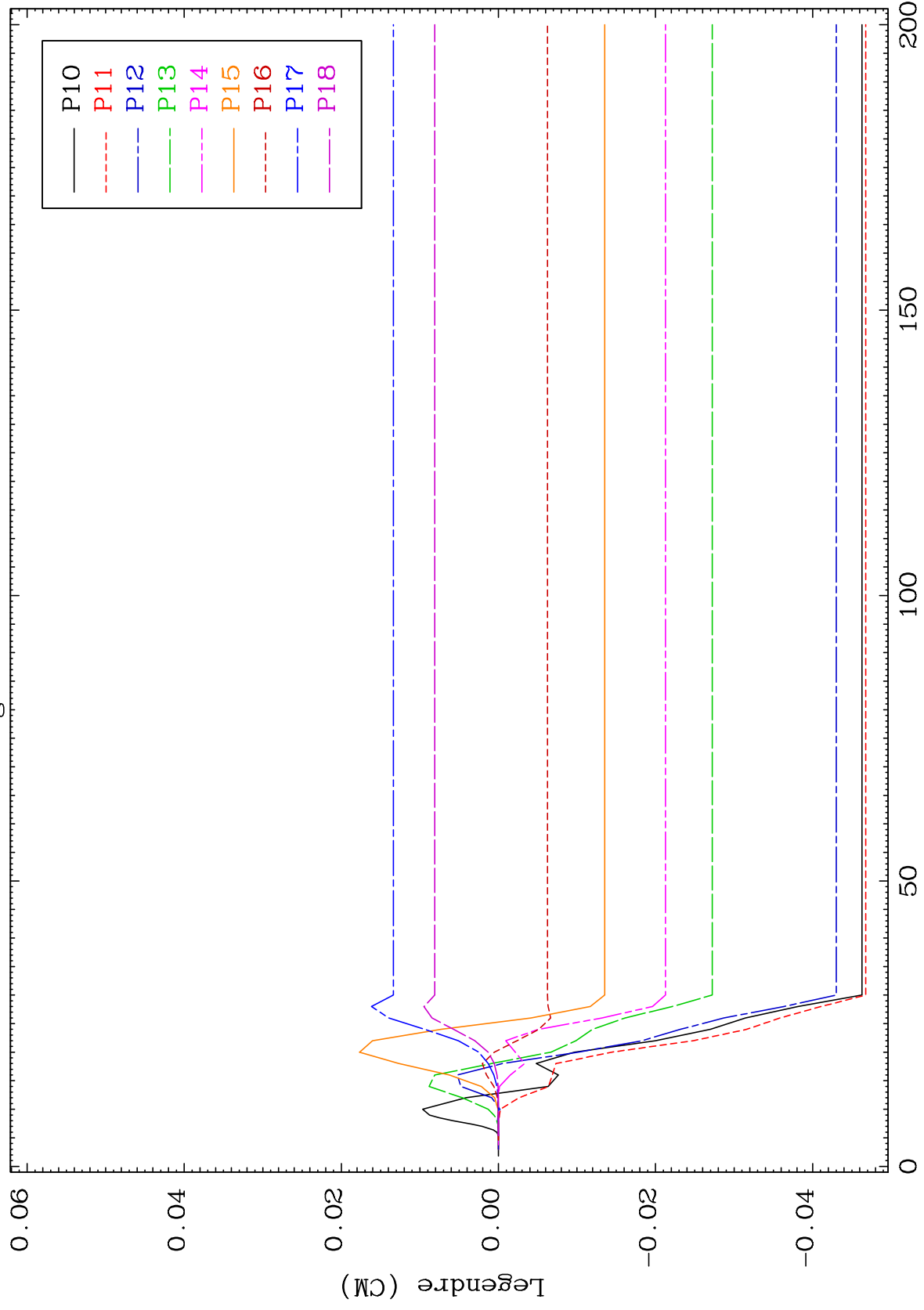


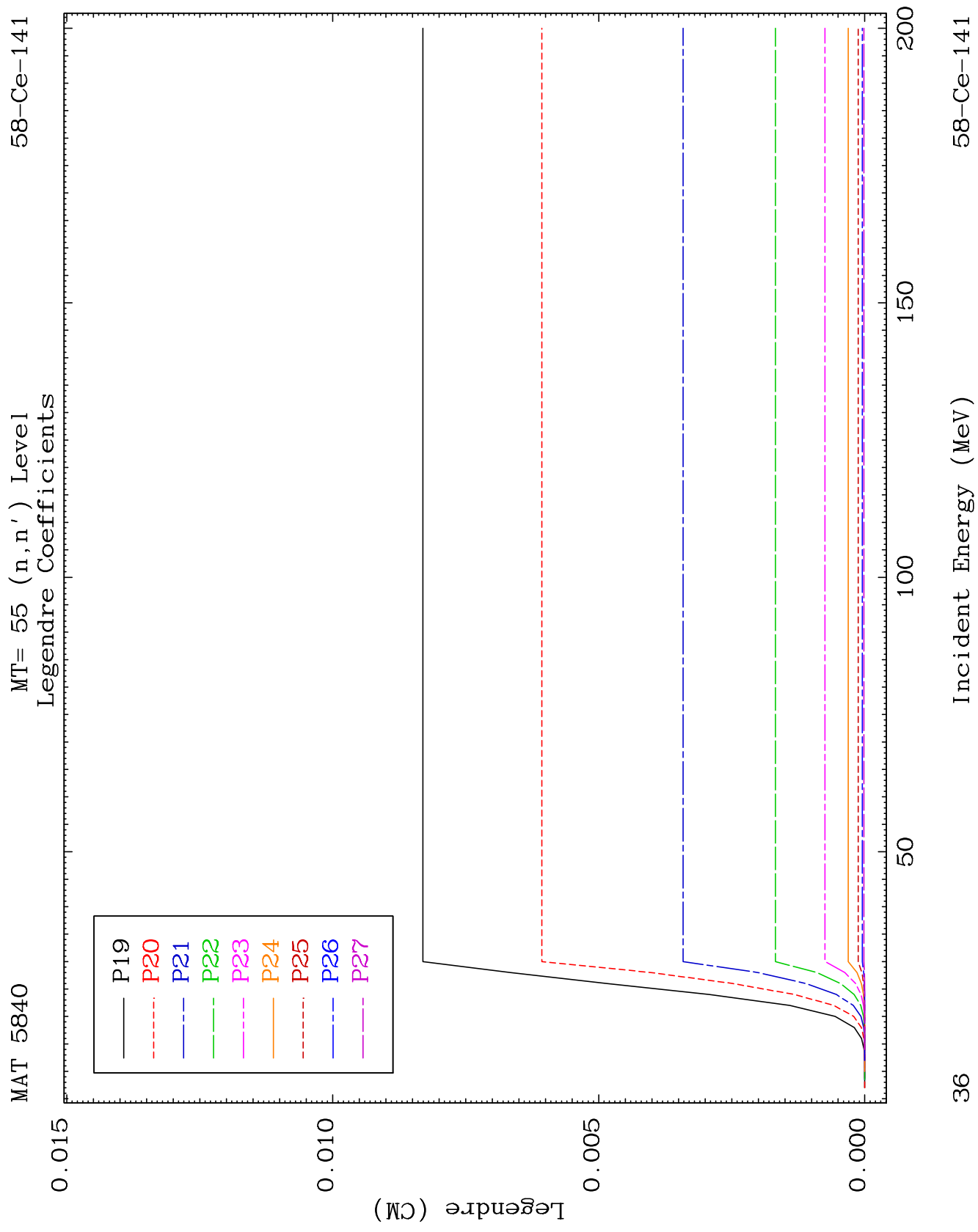


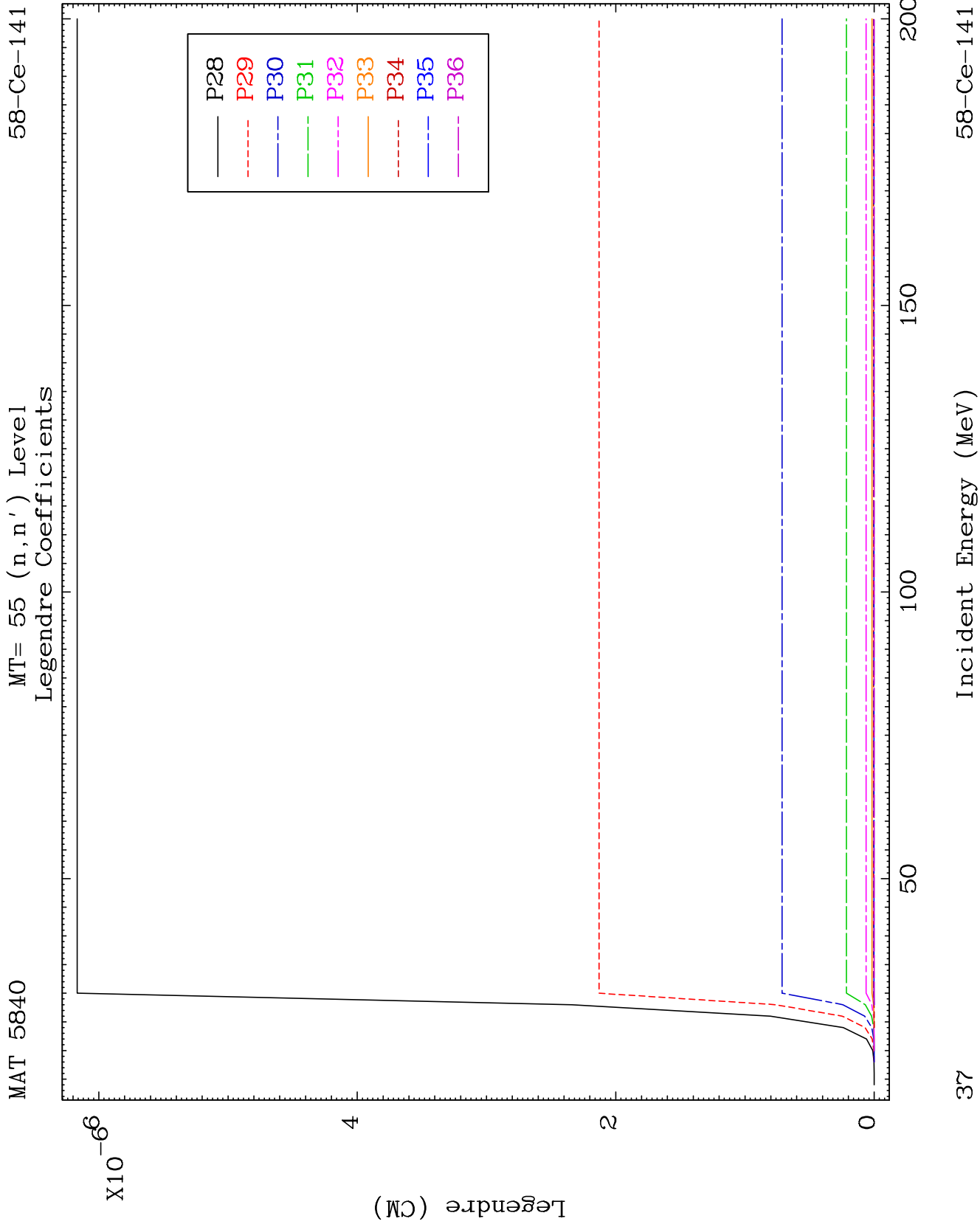
MAT 5840

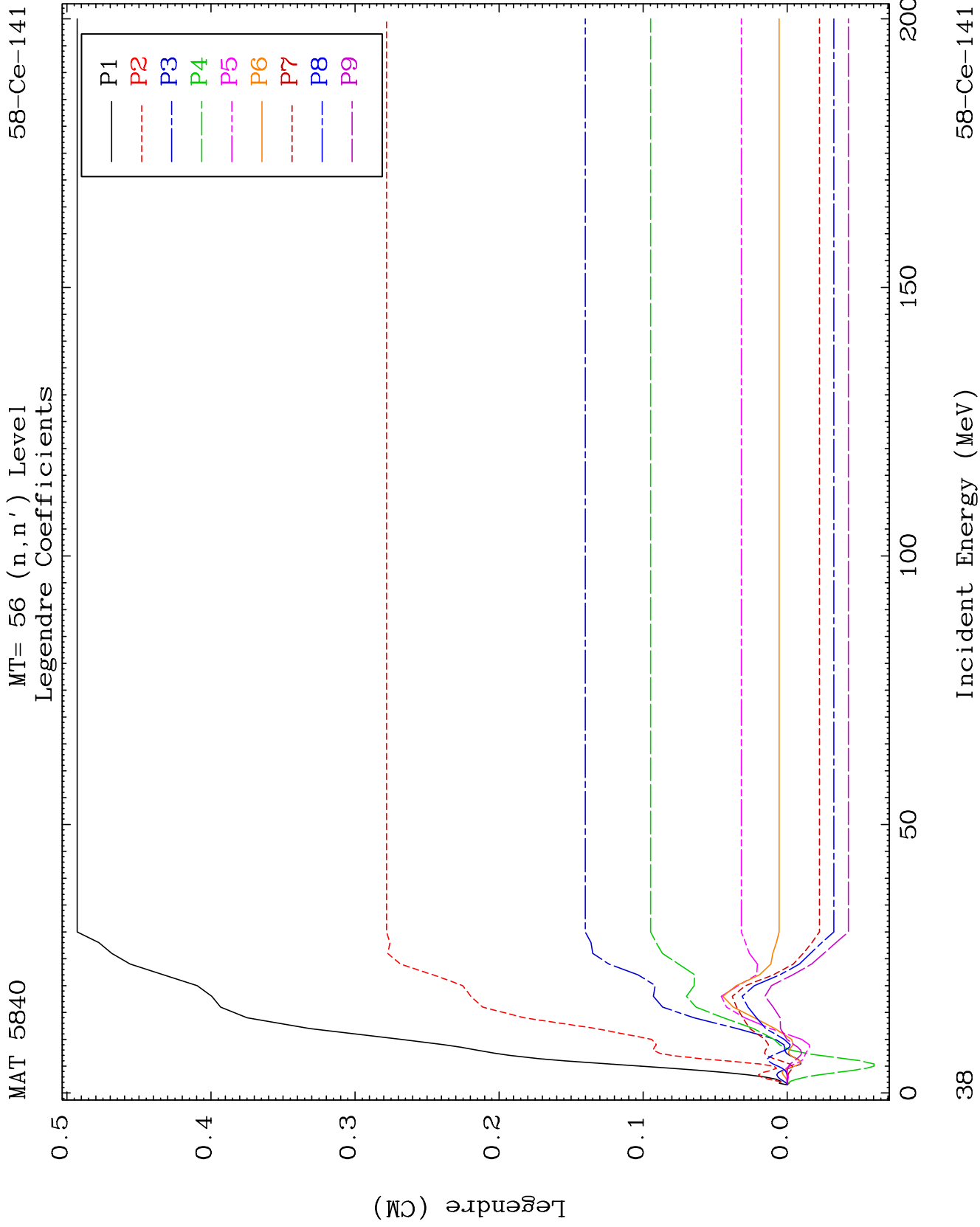
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Legendre Coefficients

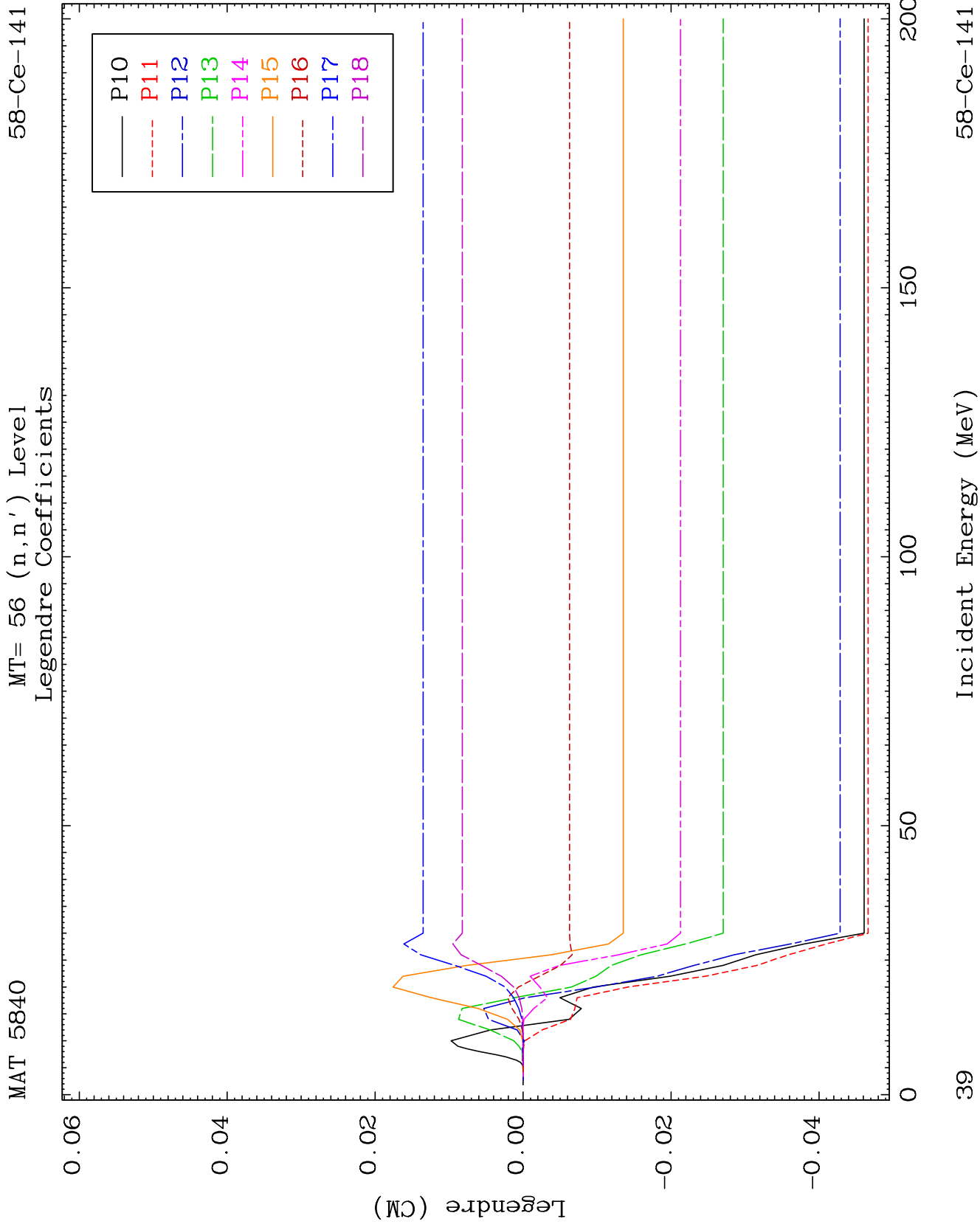
58-Ce-141







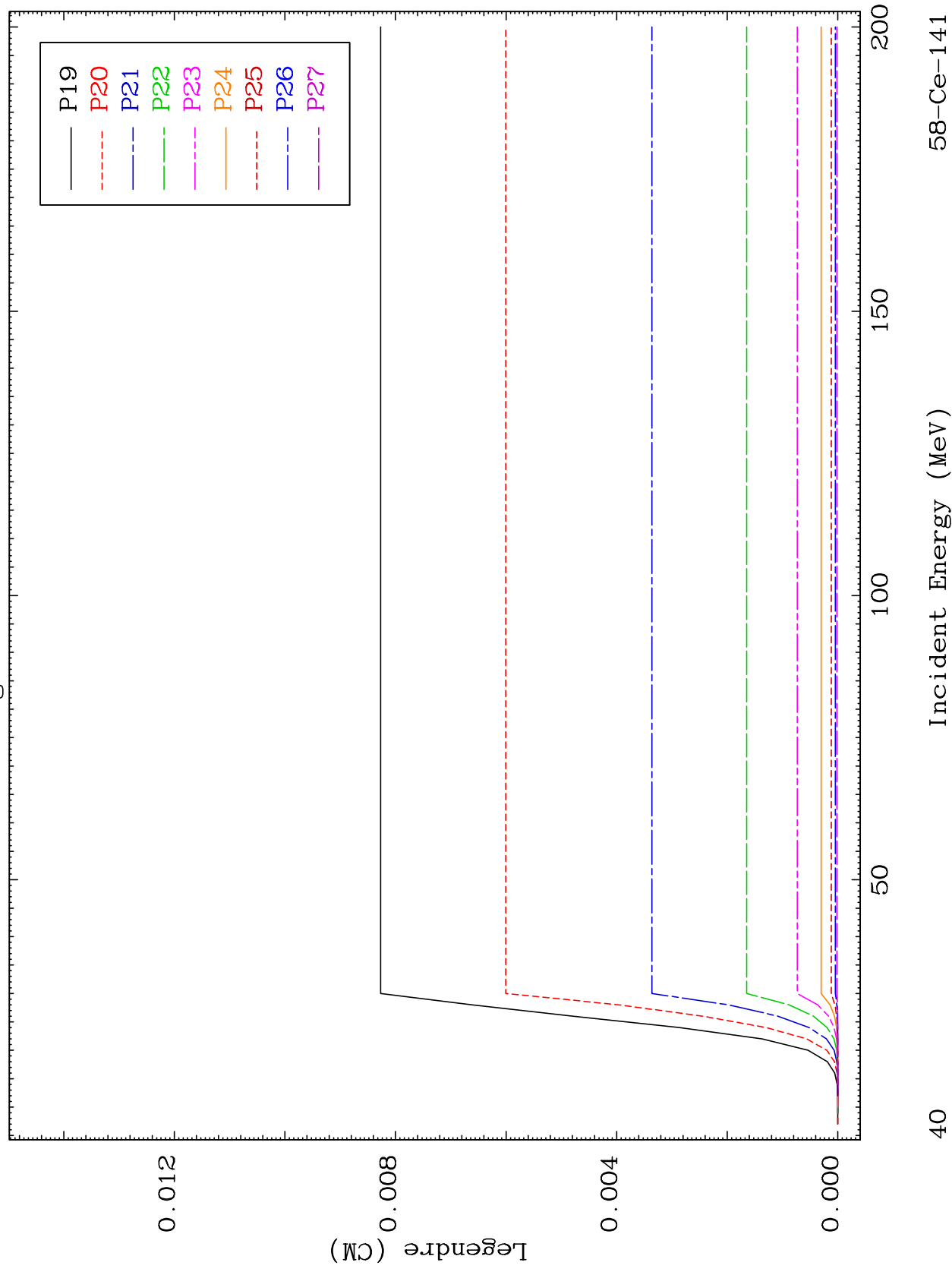


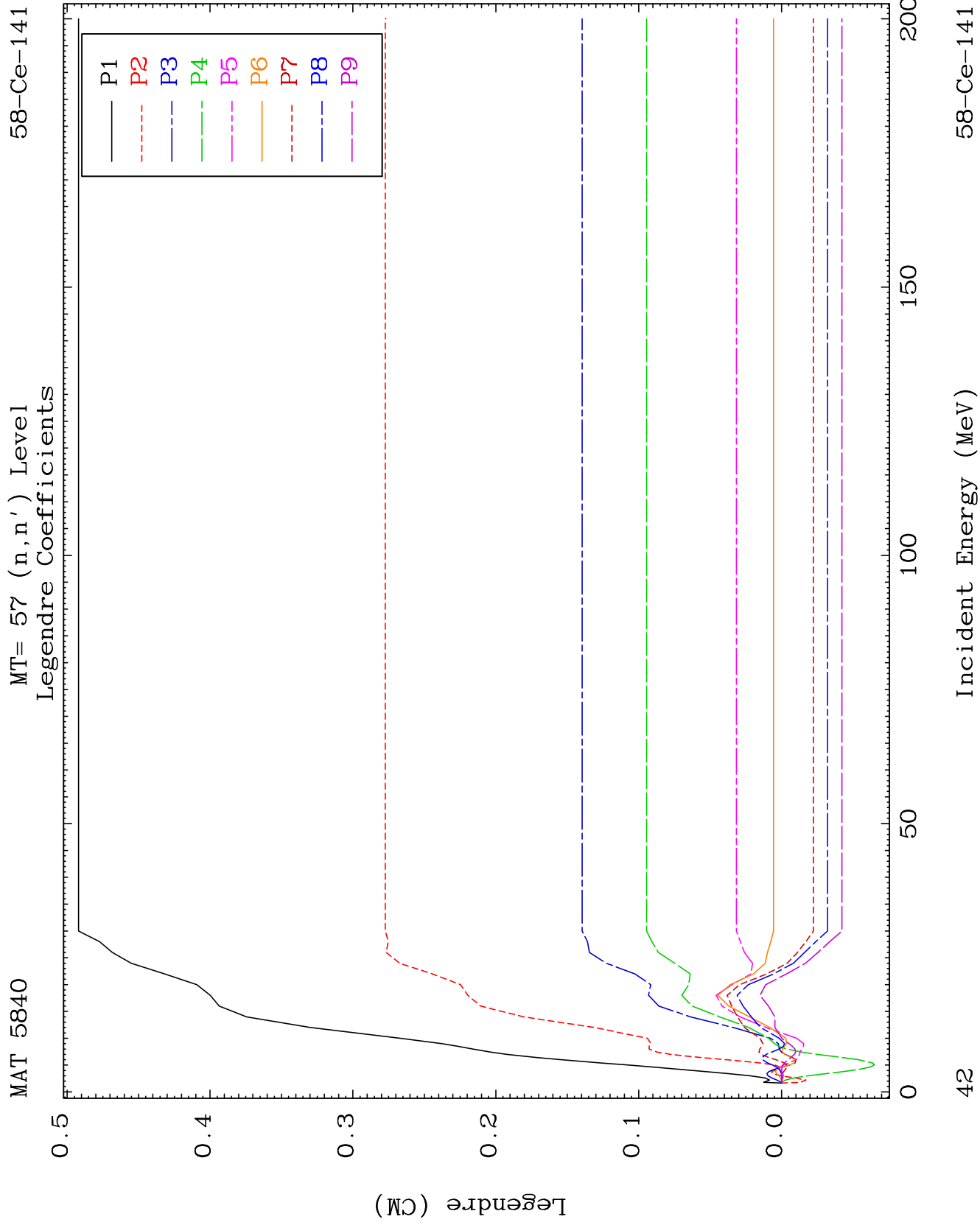


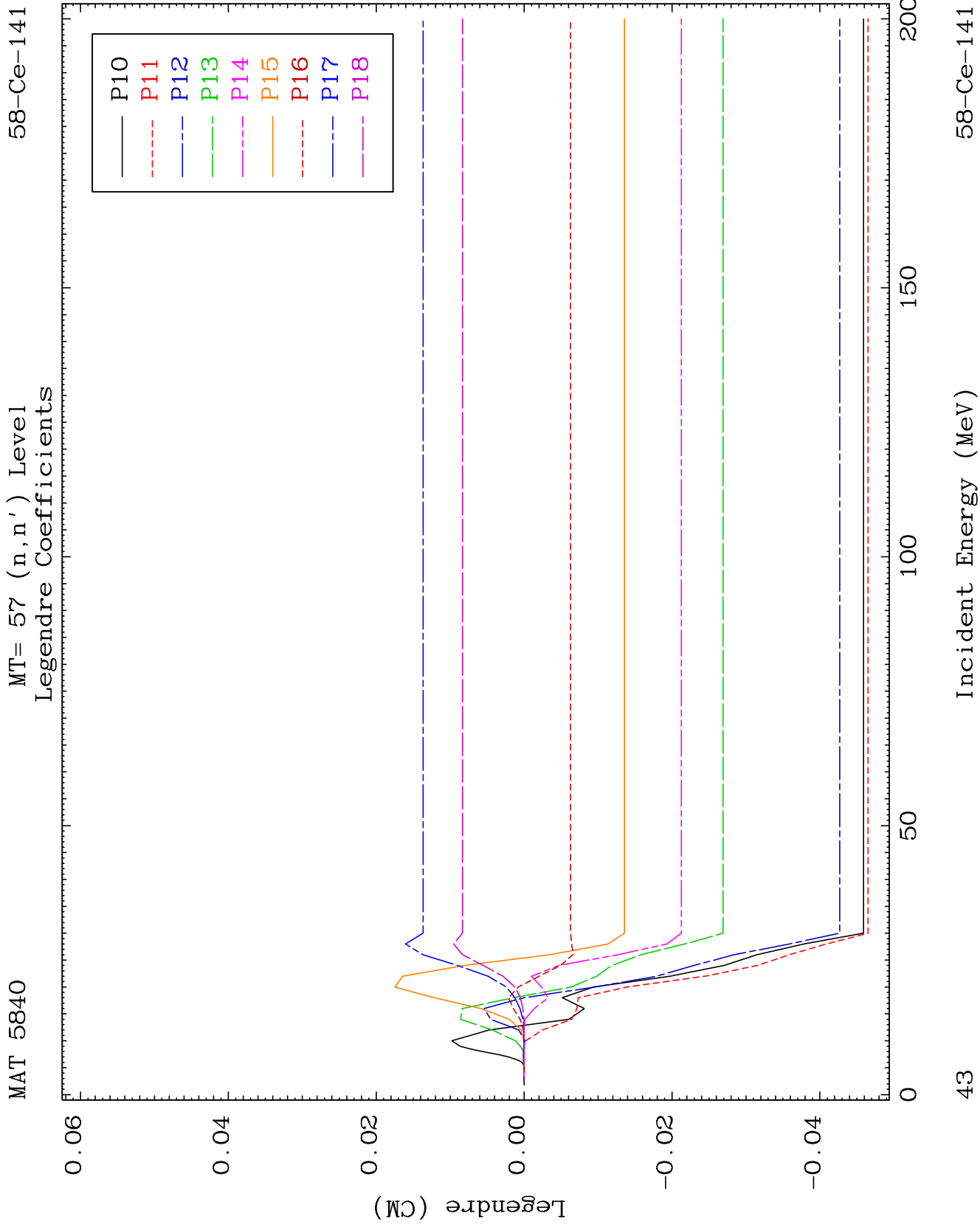
MAT 5840

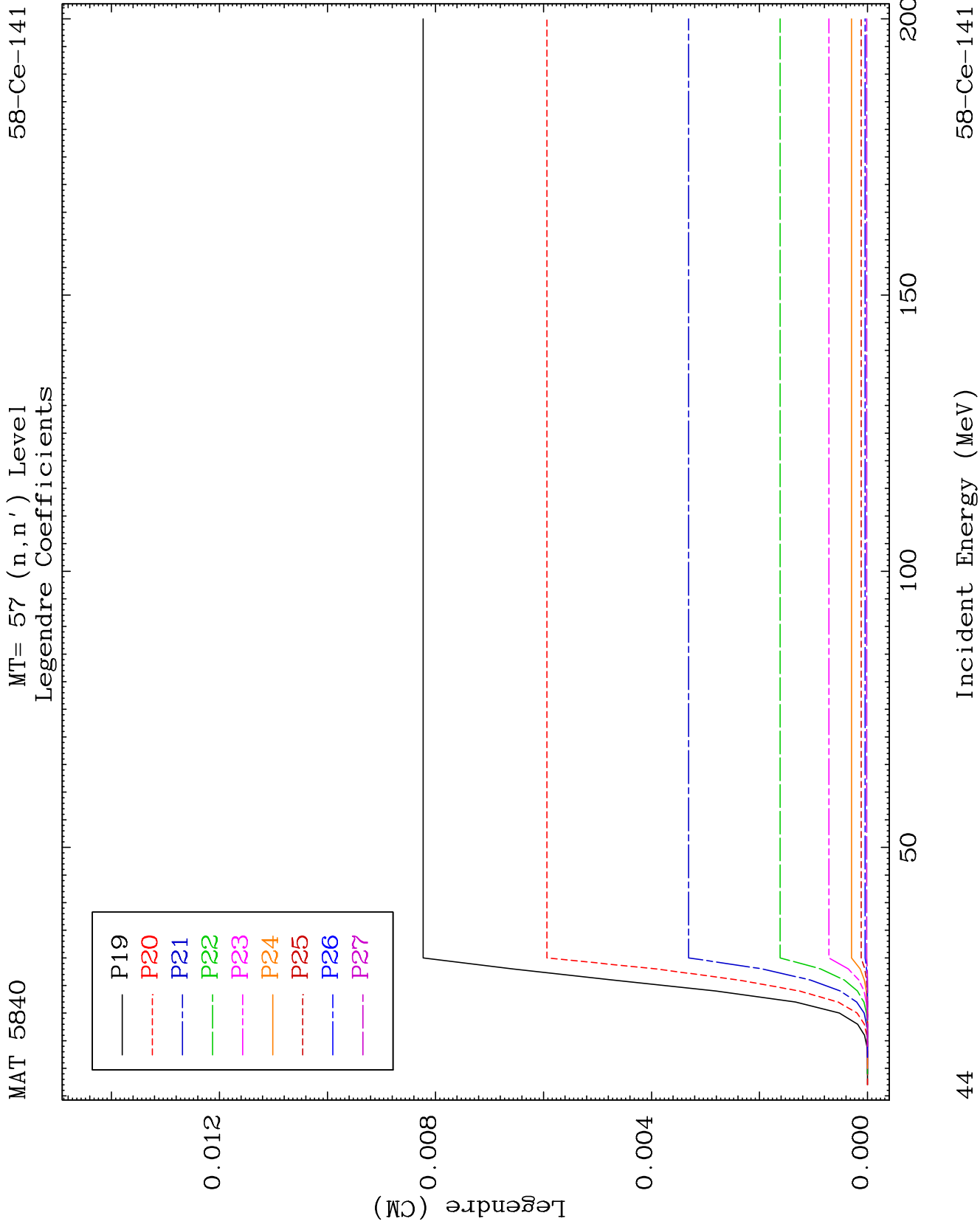
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Legendre Coefficients

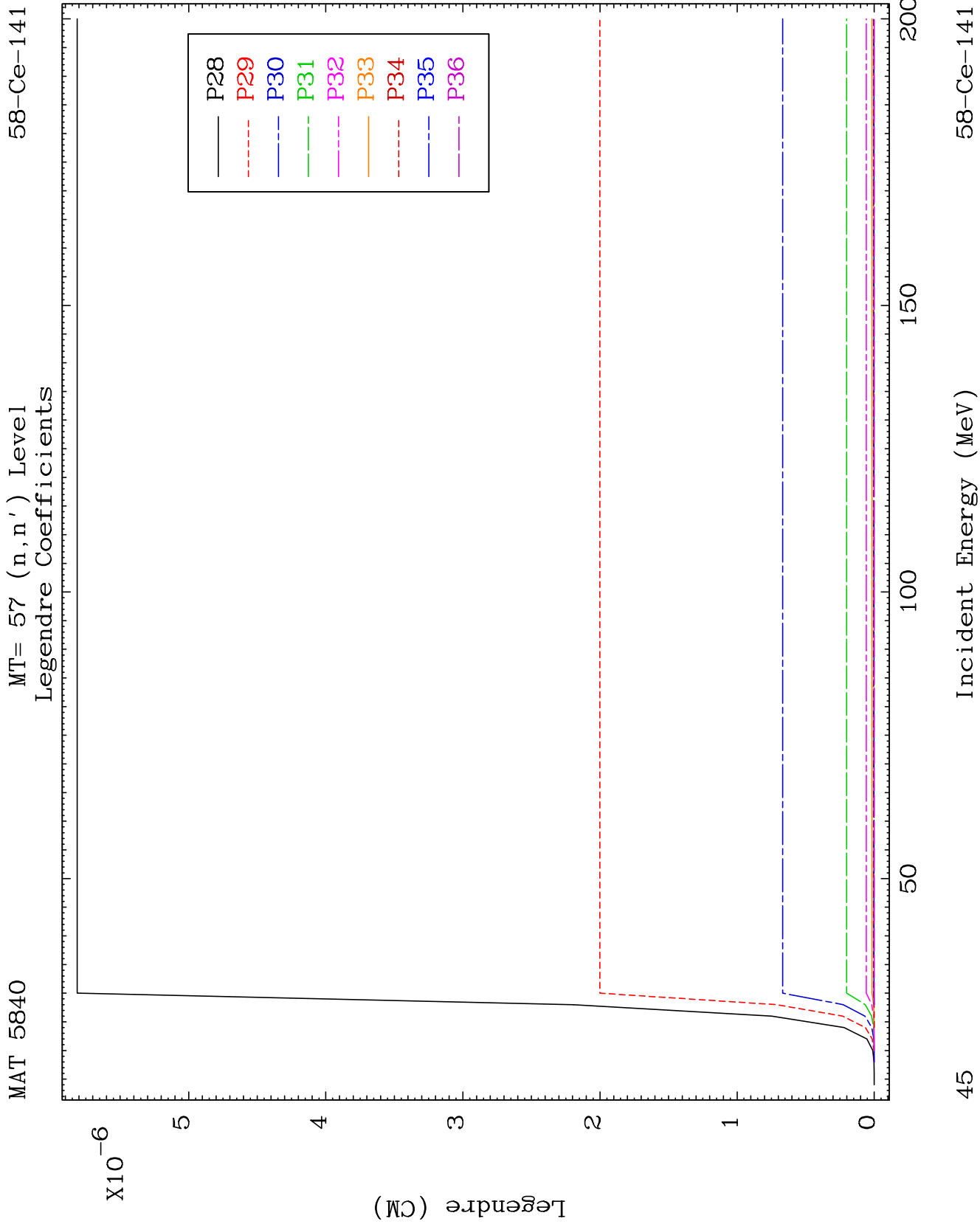
58-Ce-141

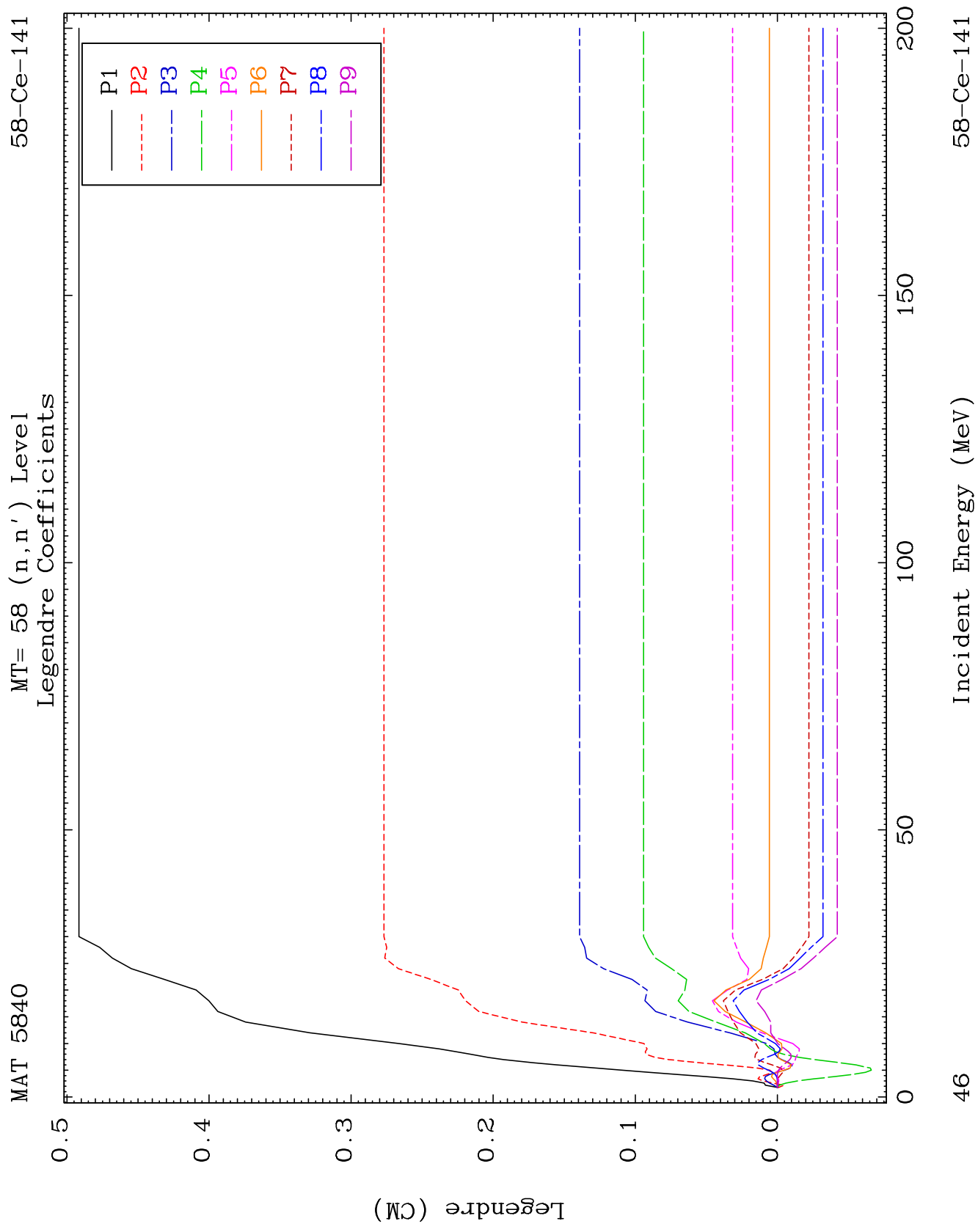


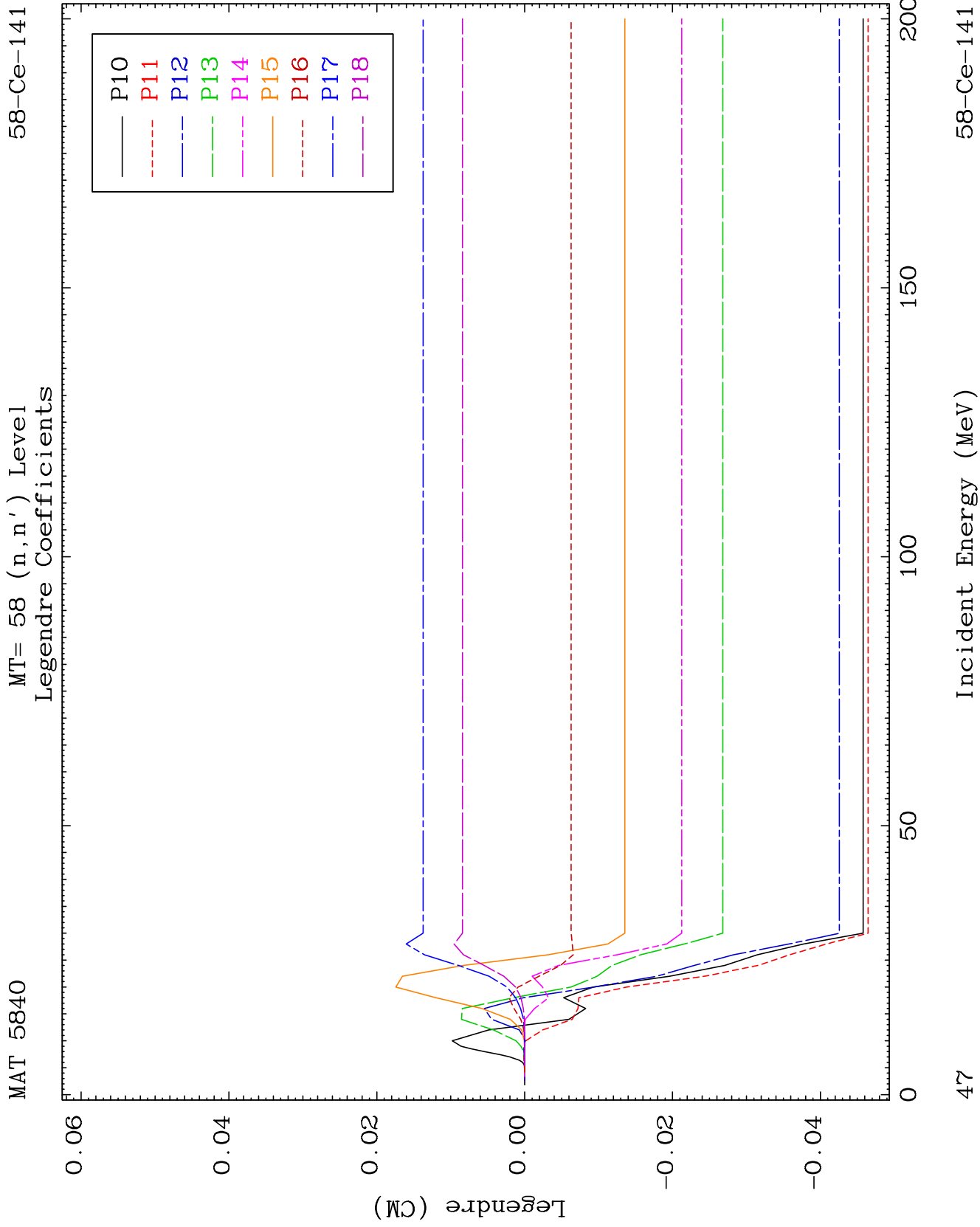








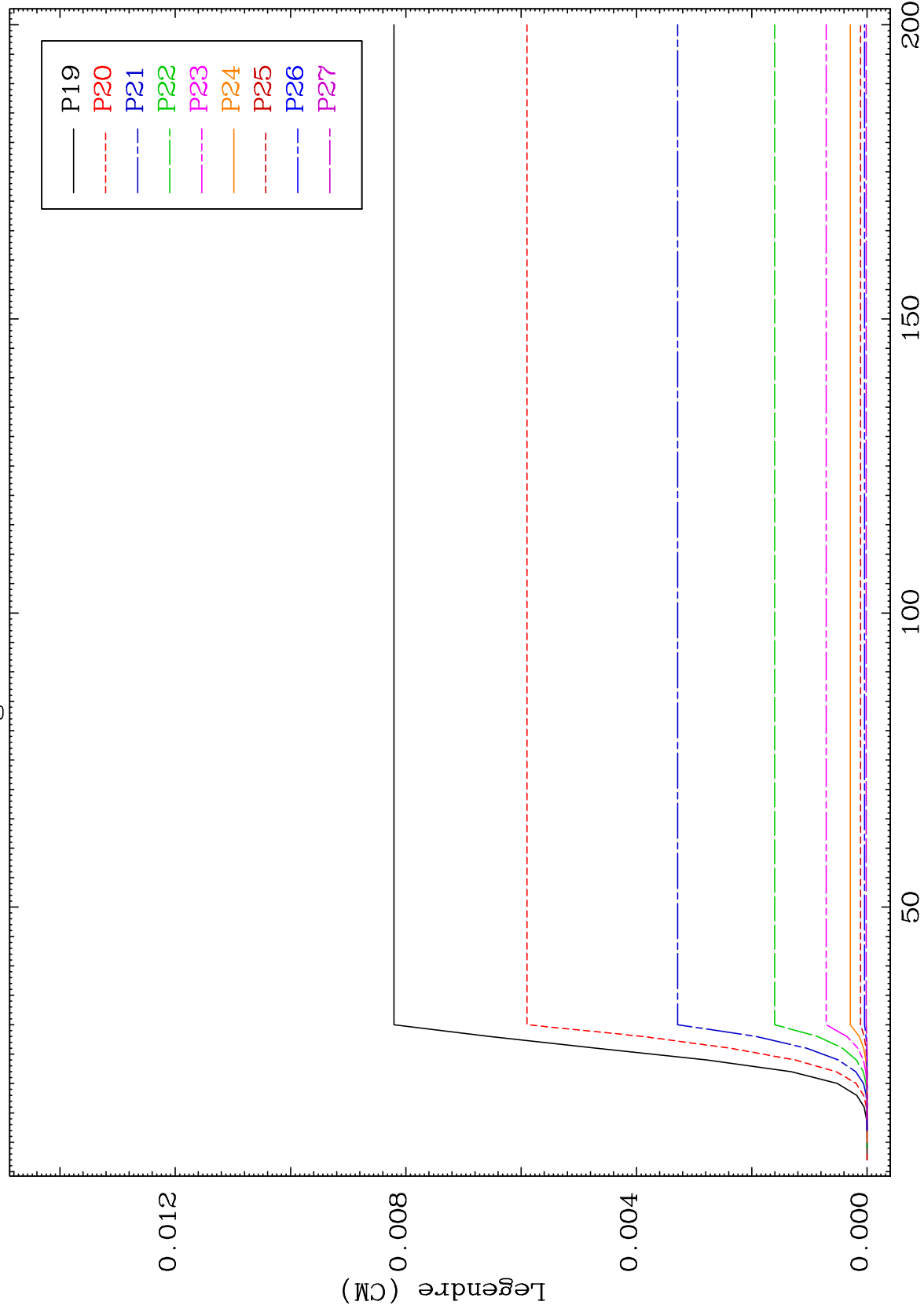




MAT 5840

MT= 58 (n, n') Level
Legendre Coefficients

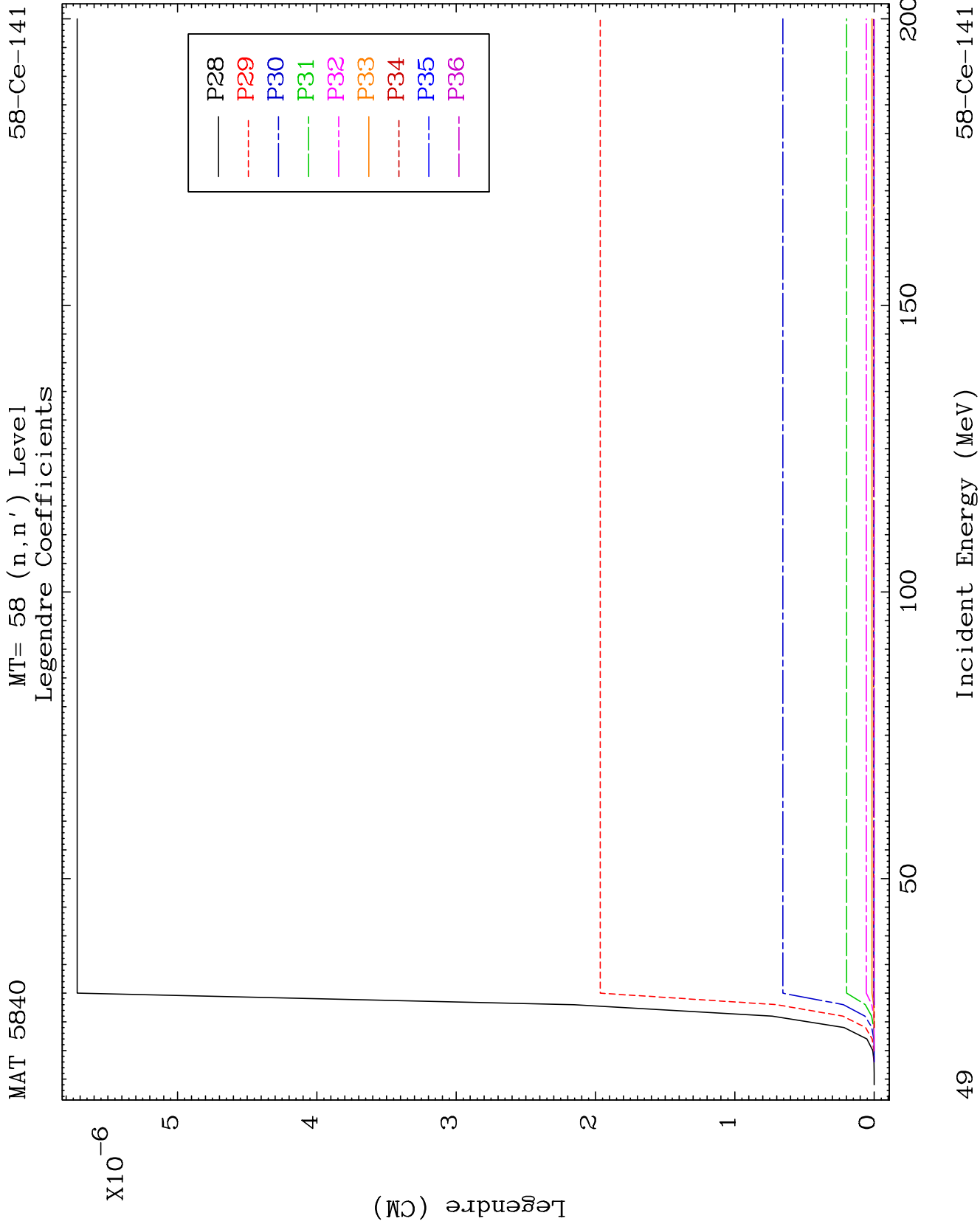
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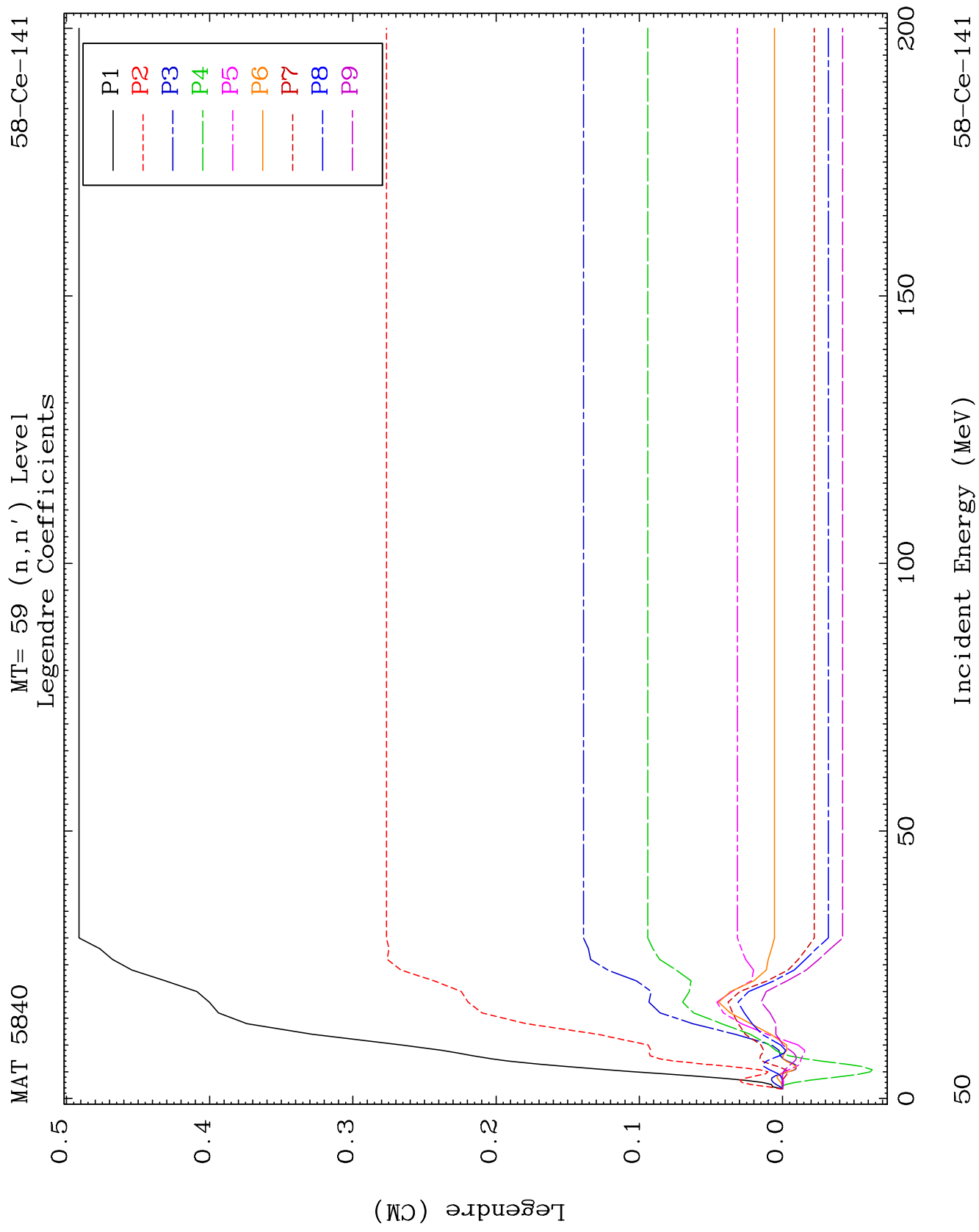


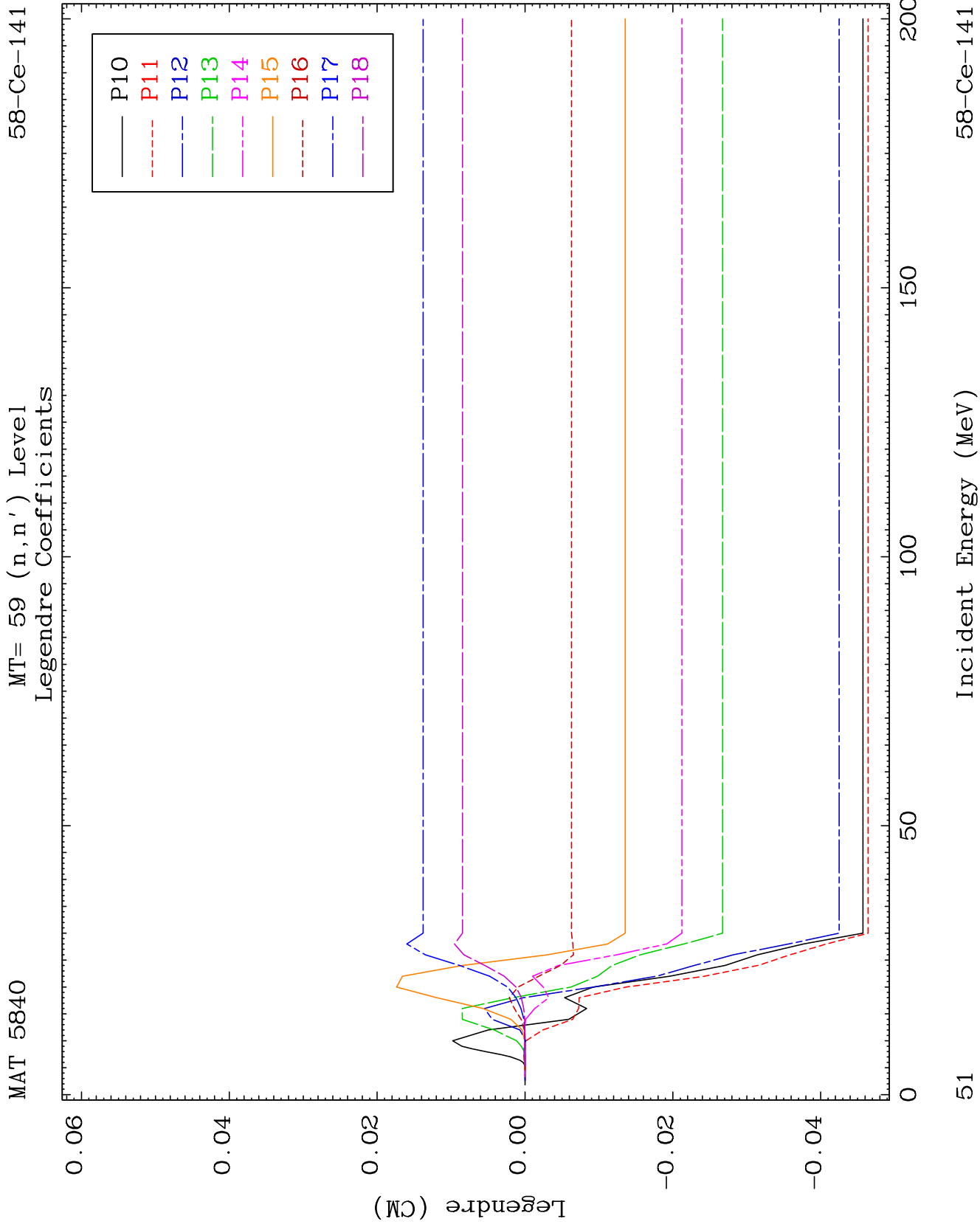
48

Incident Energy (MeV)

58-Ce-141



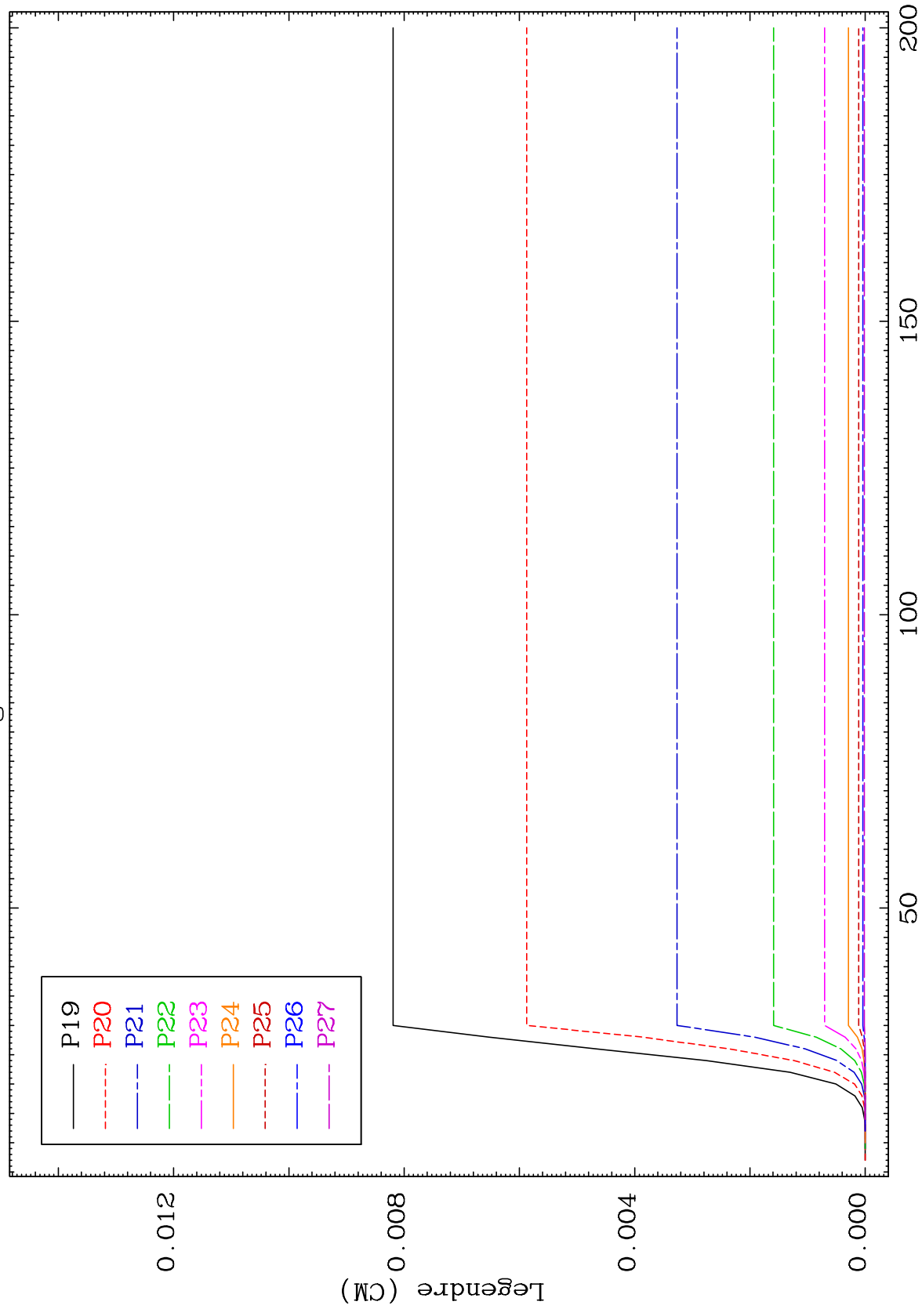




MAT 5840

MT= 59 (n, n') Level
Legendre Coefficients

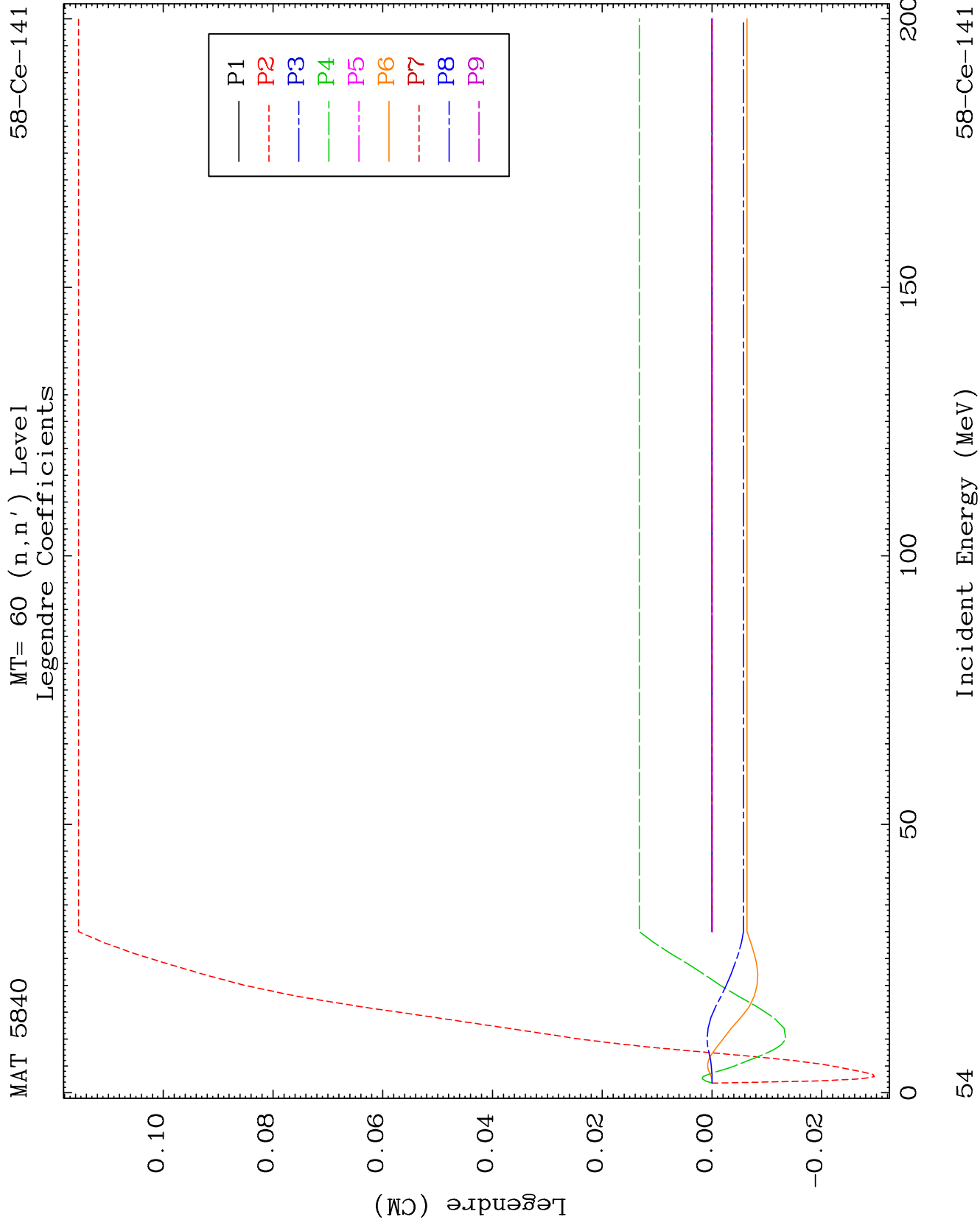
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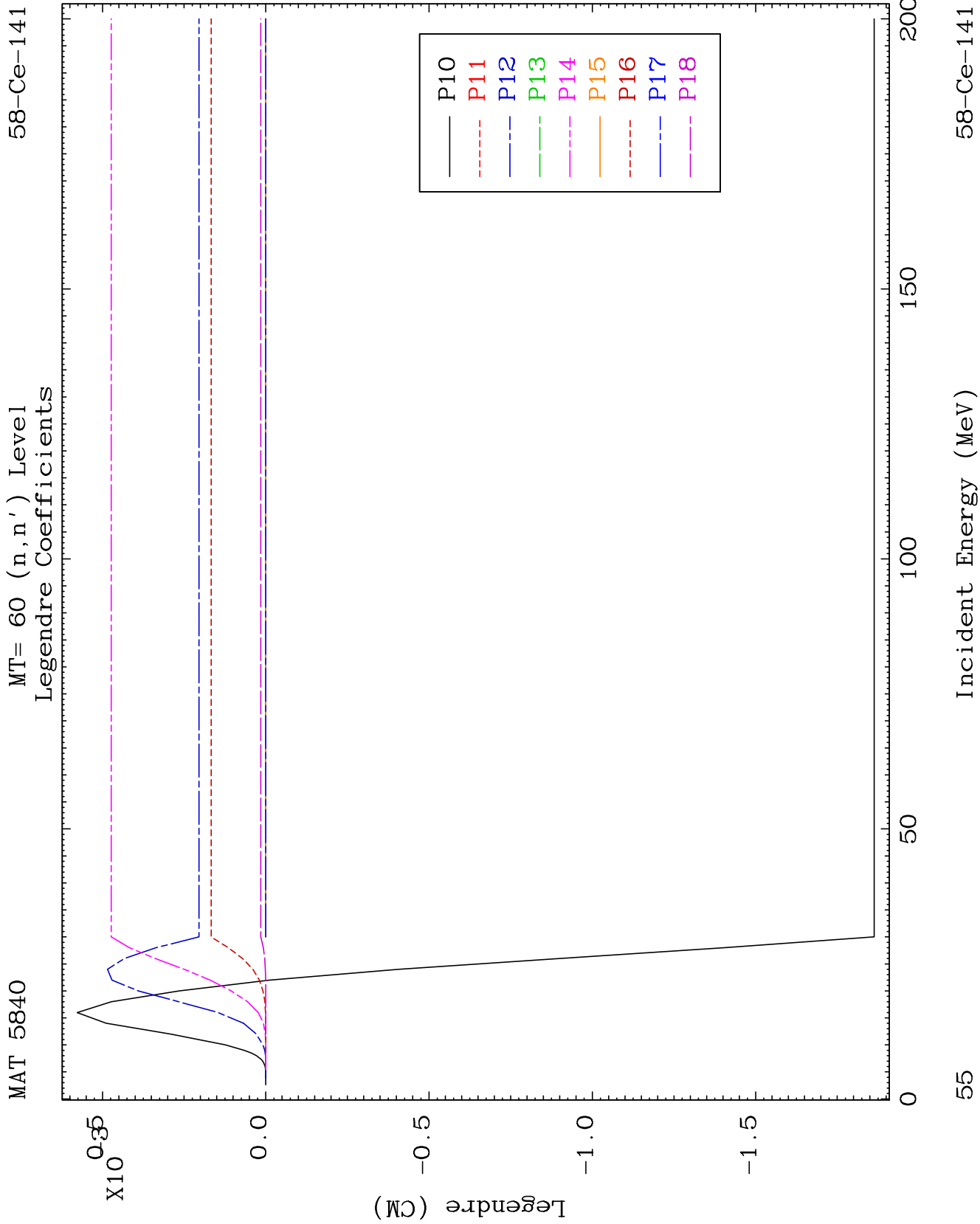


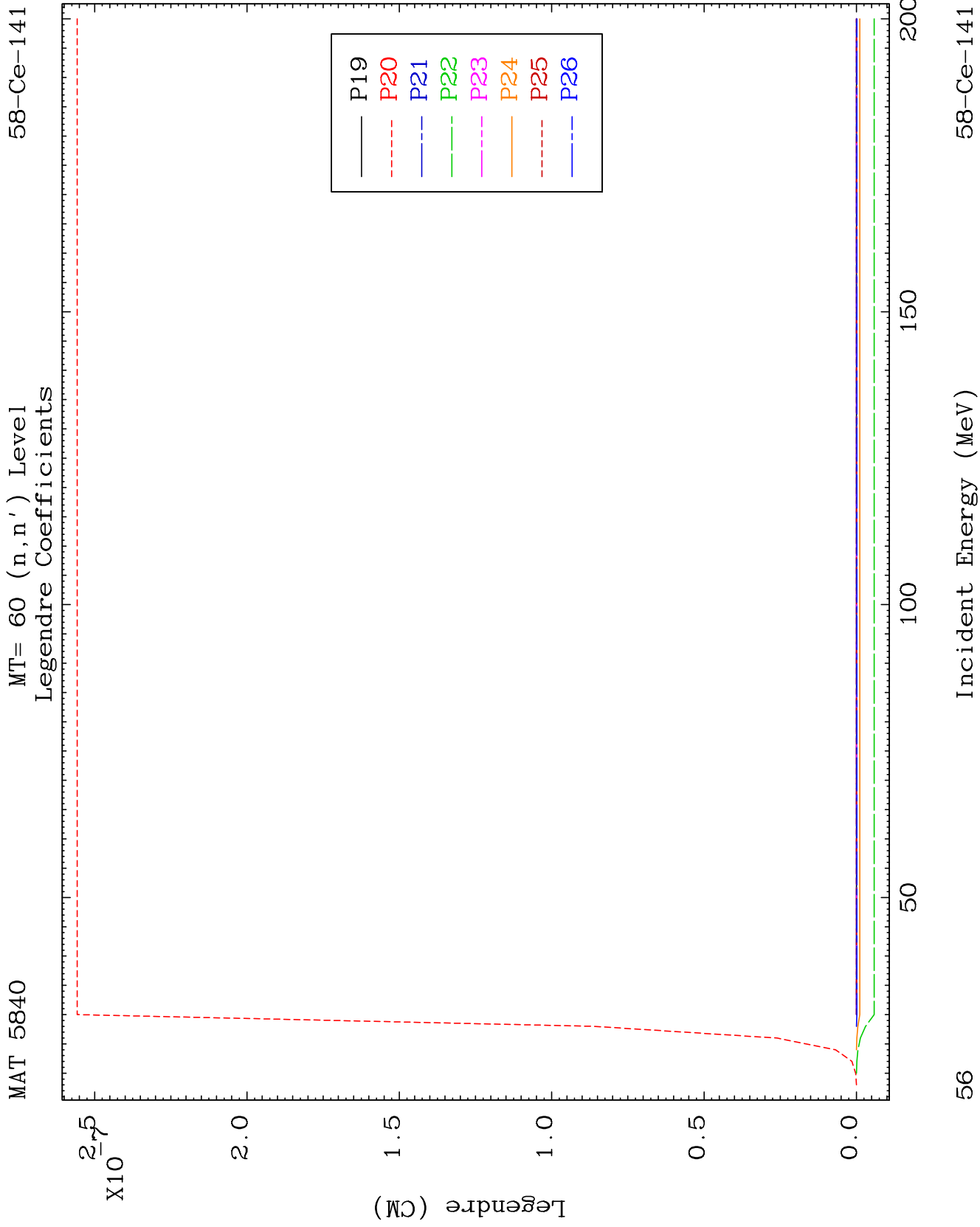
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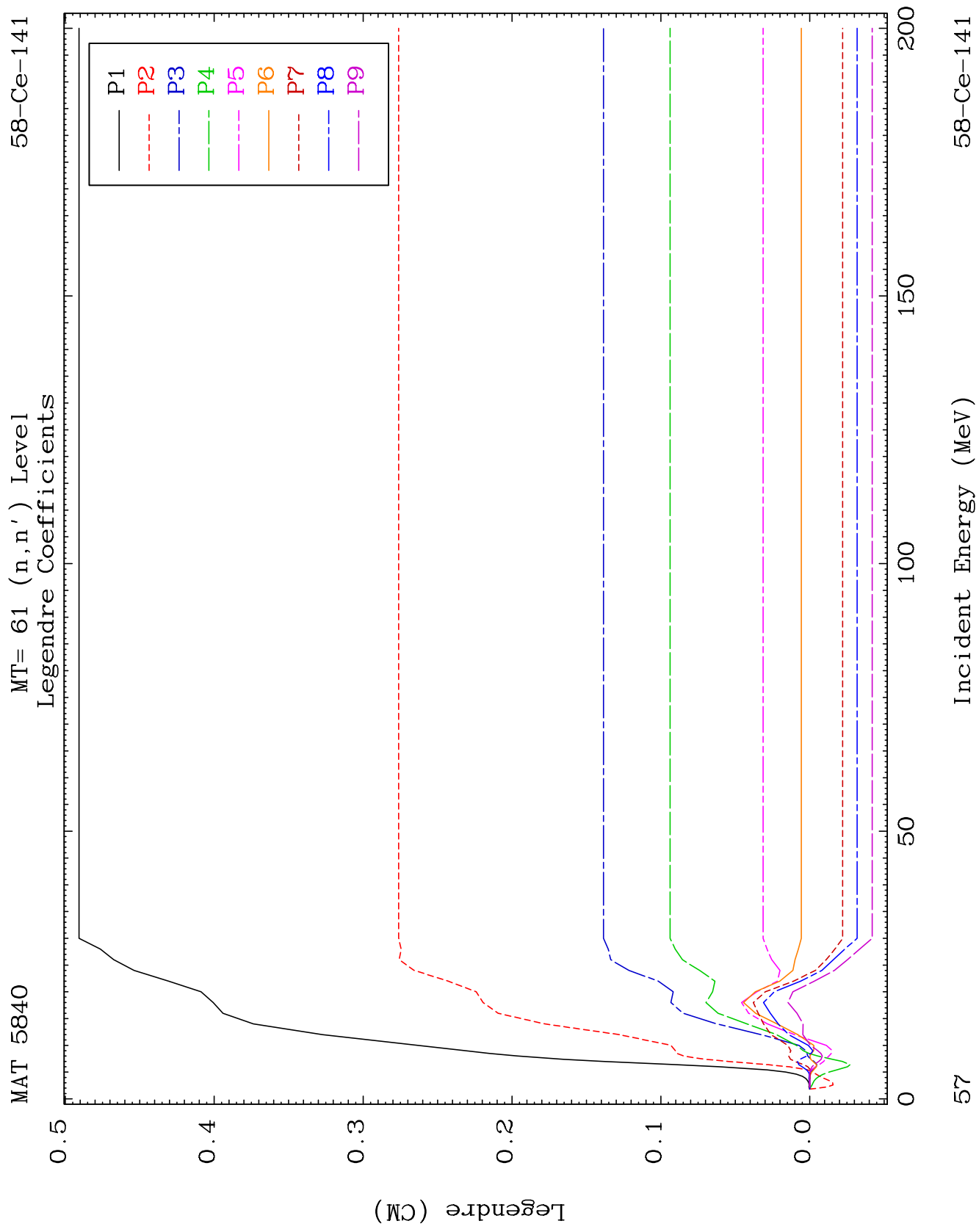
Incident Energy (MeV)

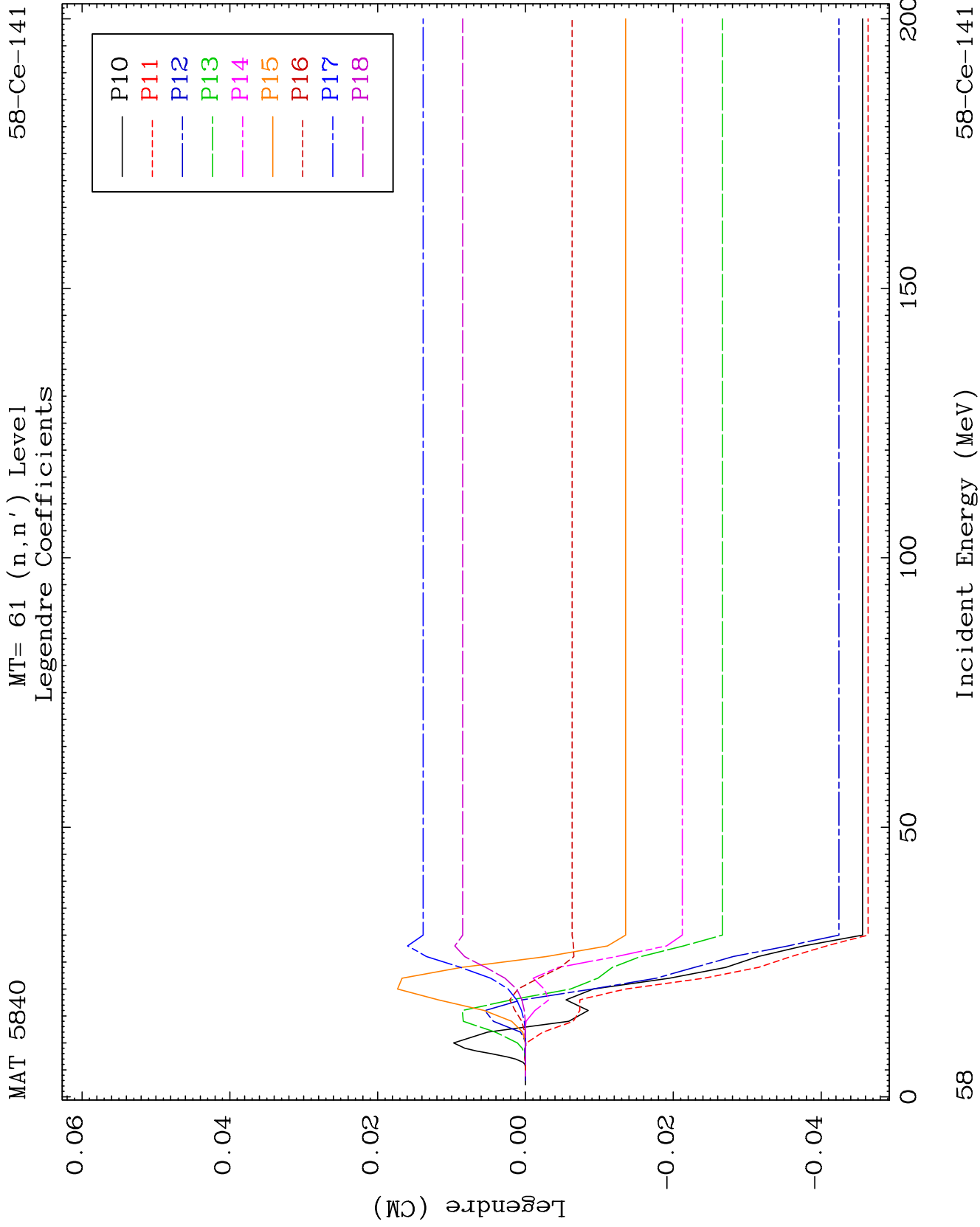
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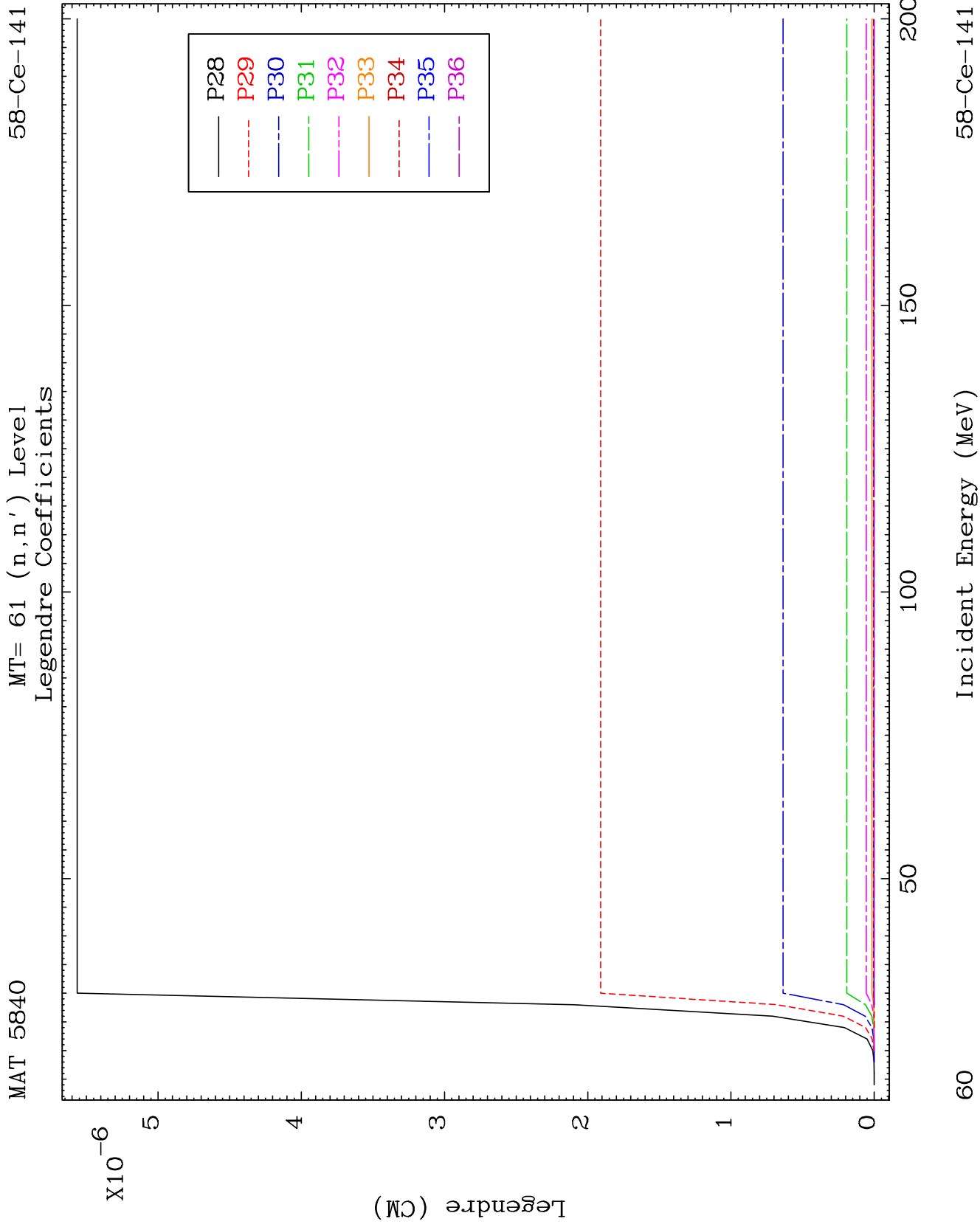


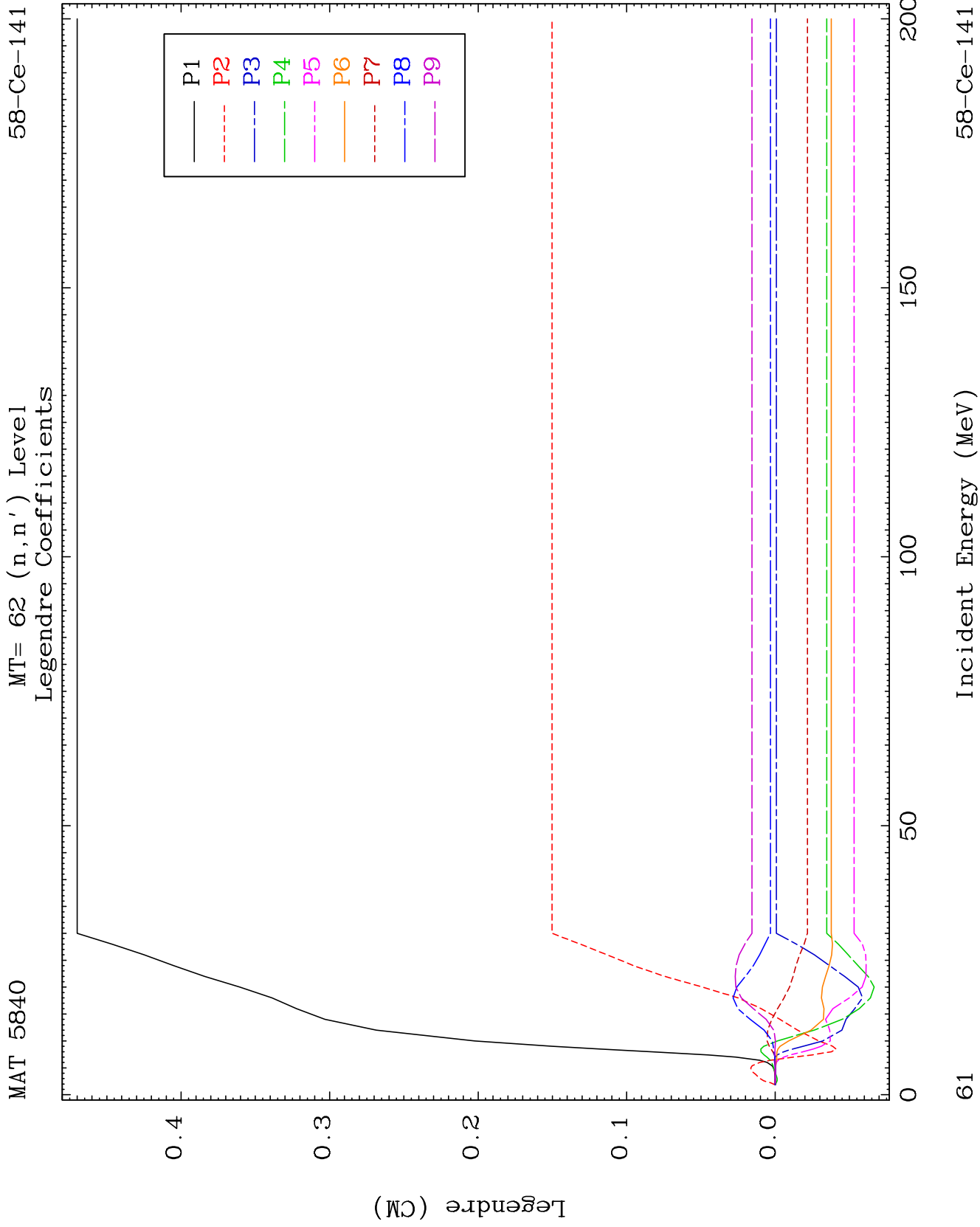


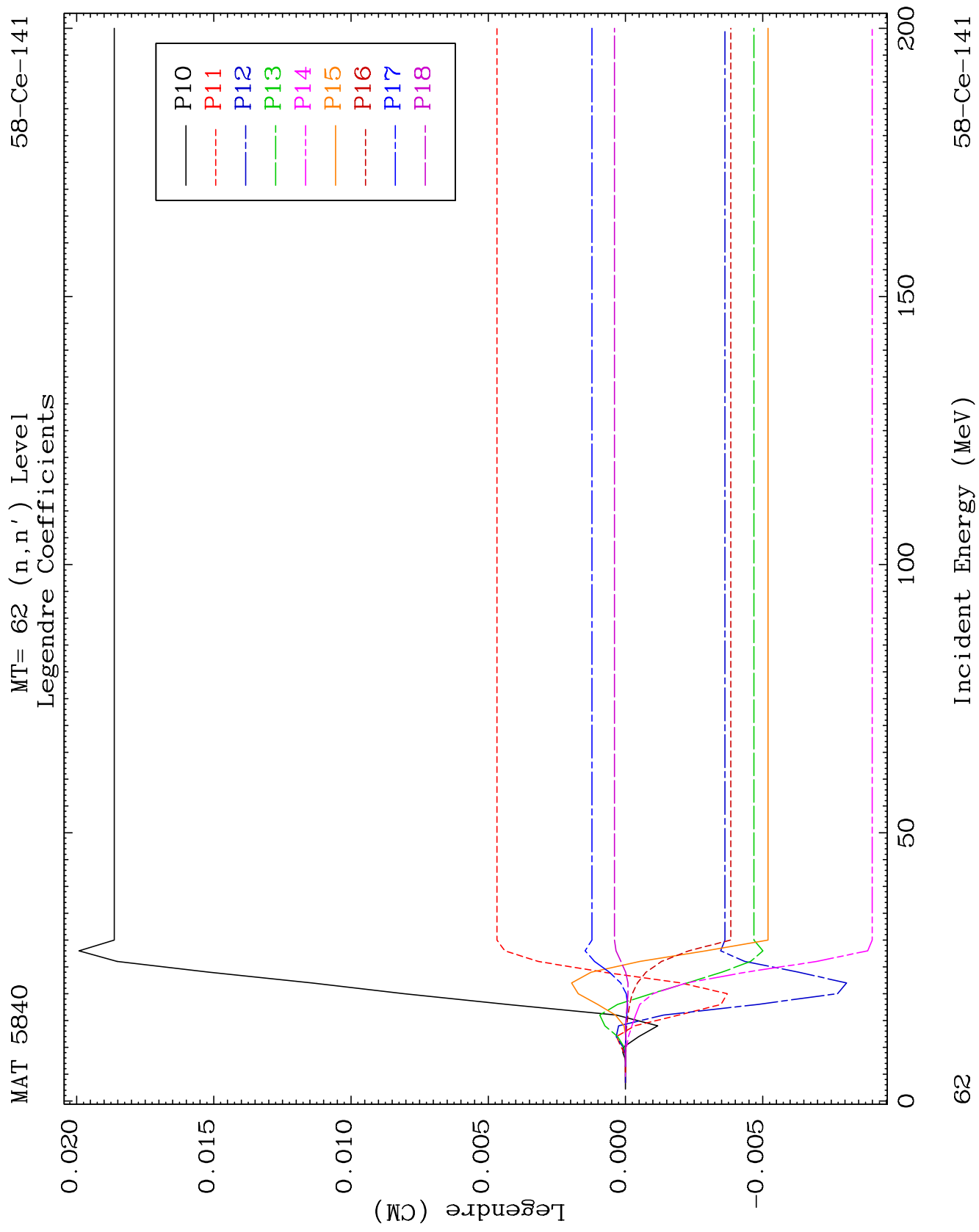








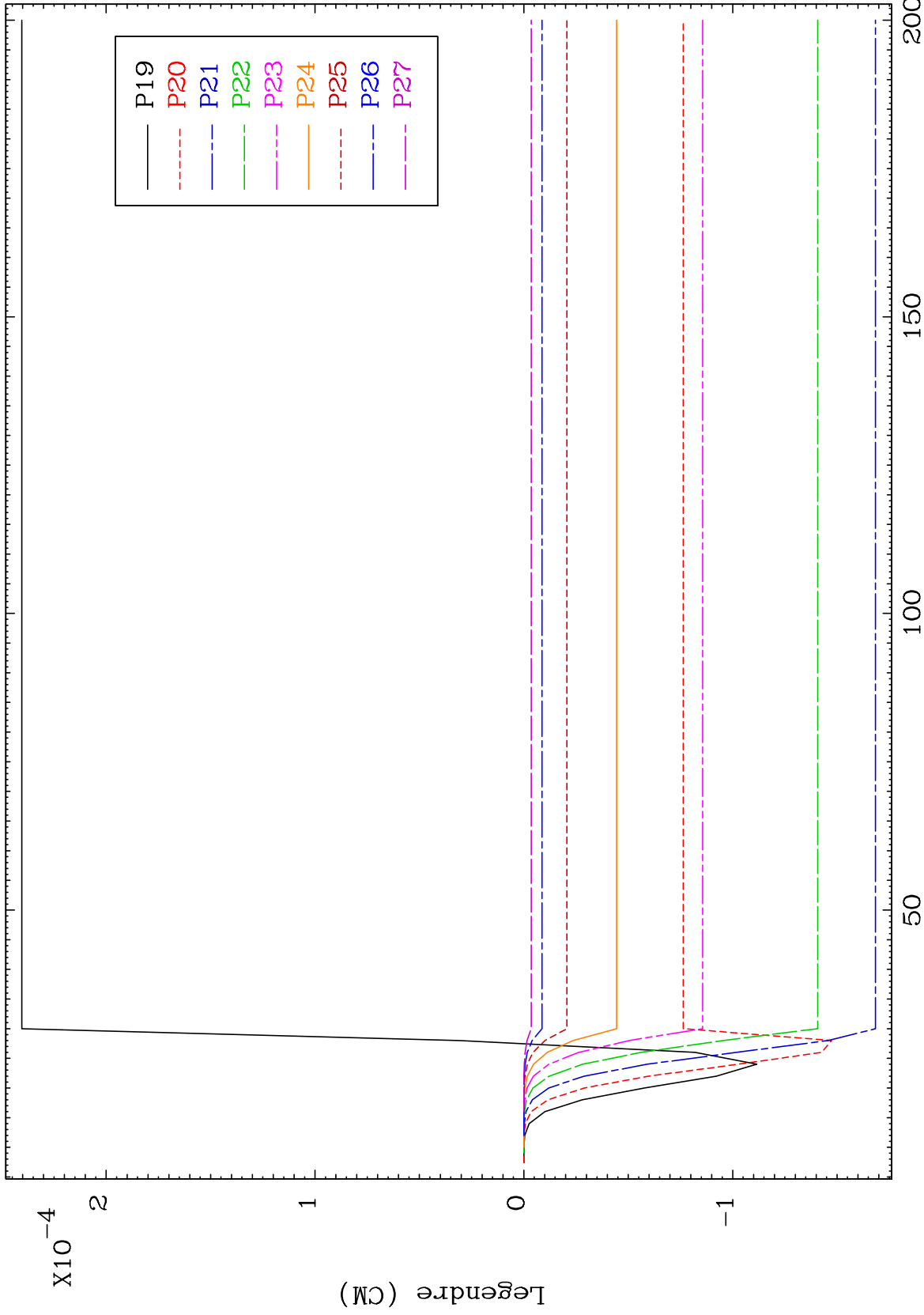




MAT 5840

MT= 62 (n,n') Level
Legendre Coefficients

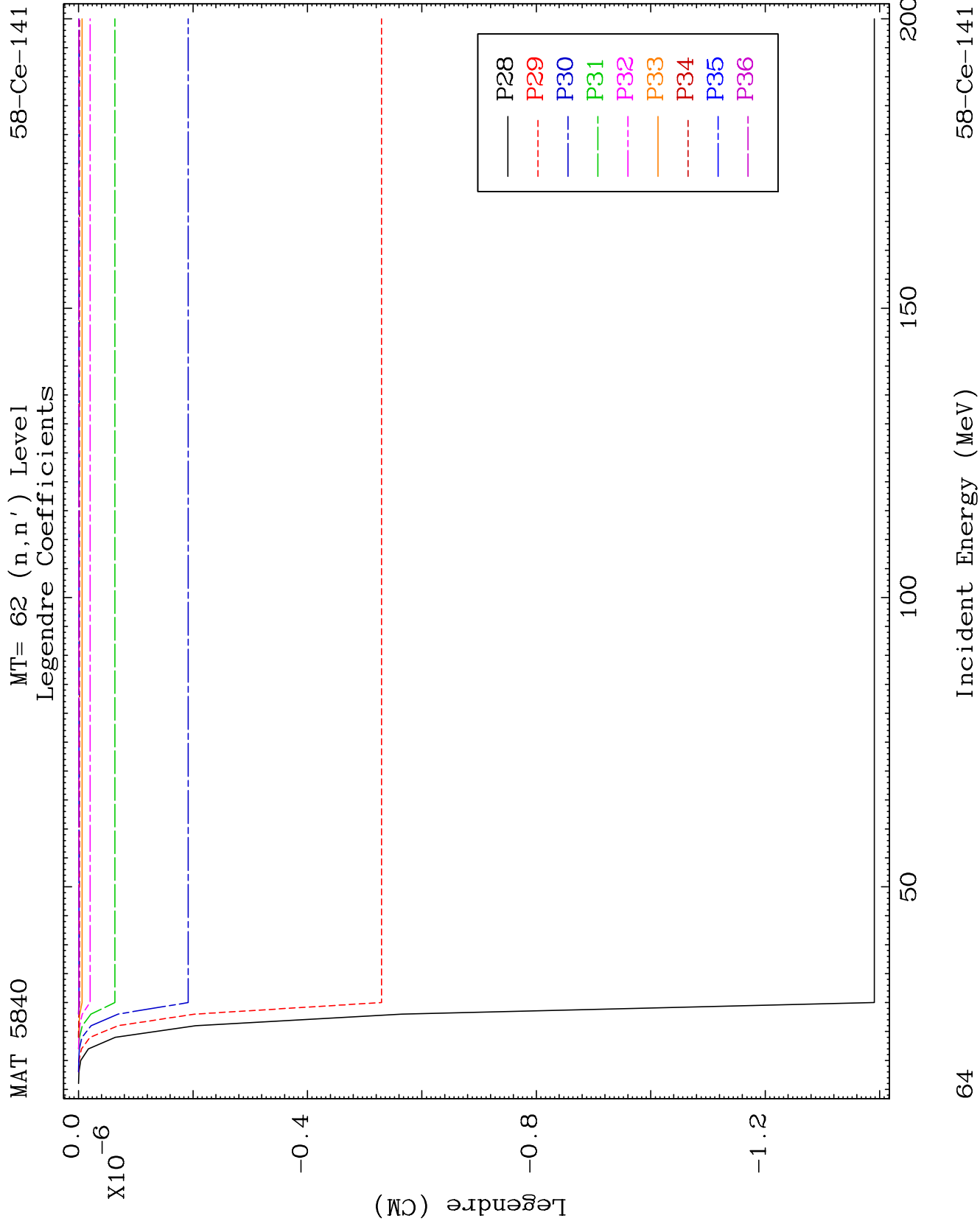
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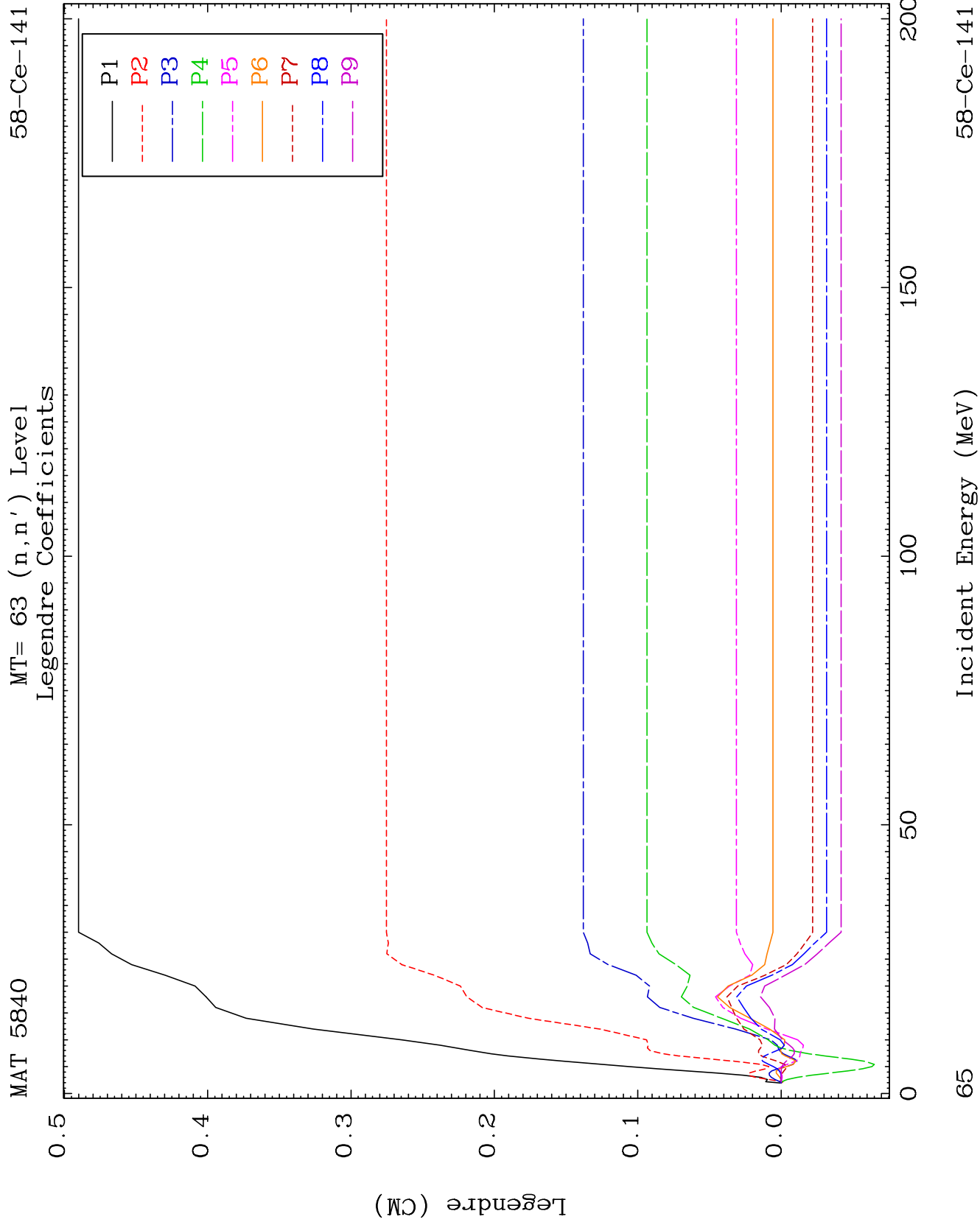


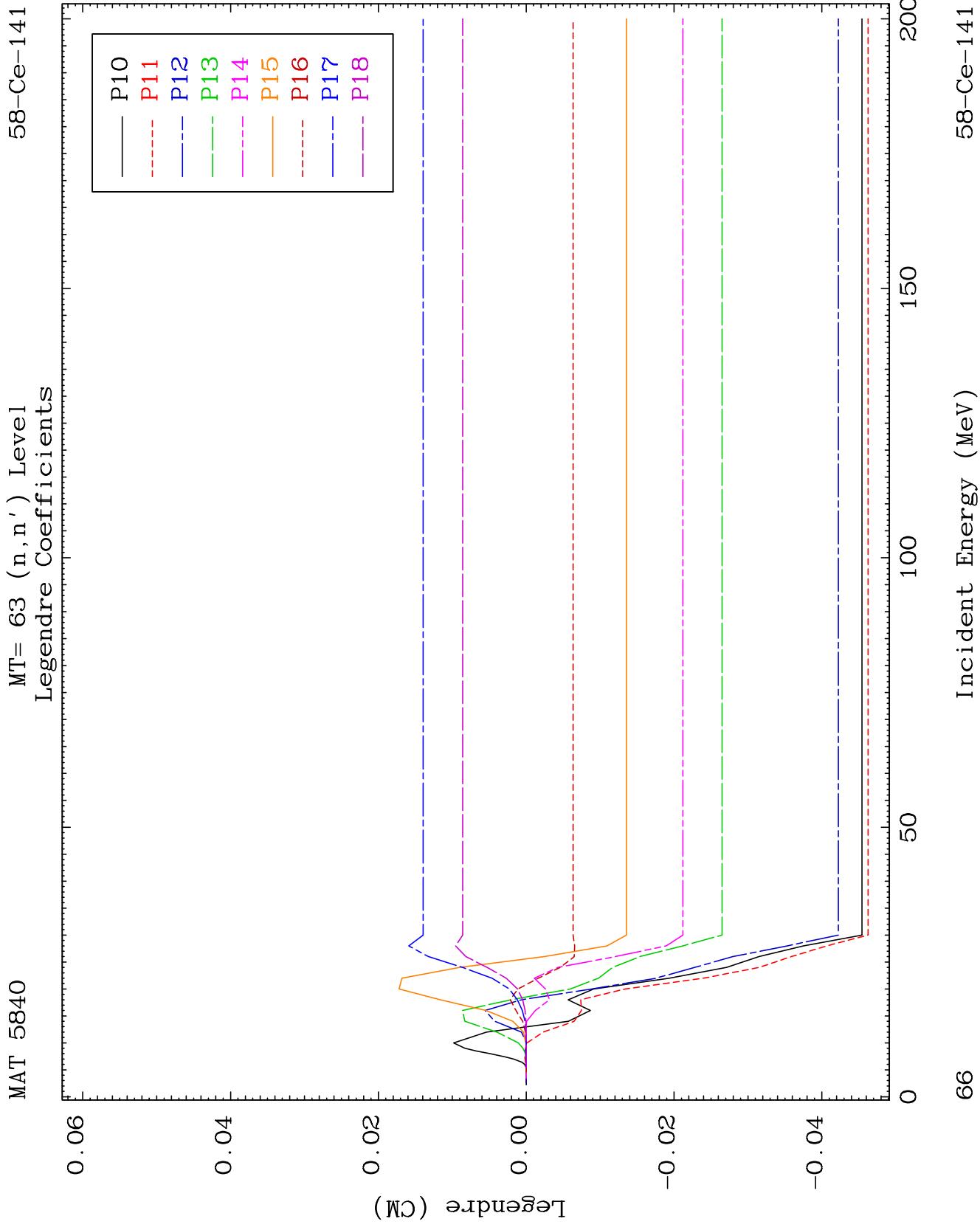
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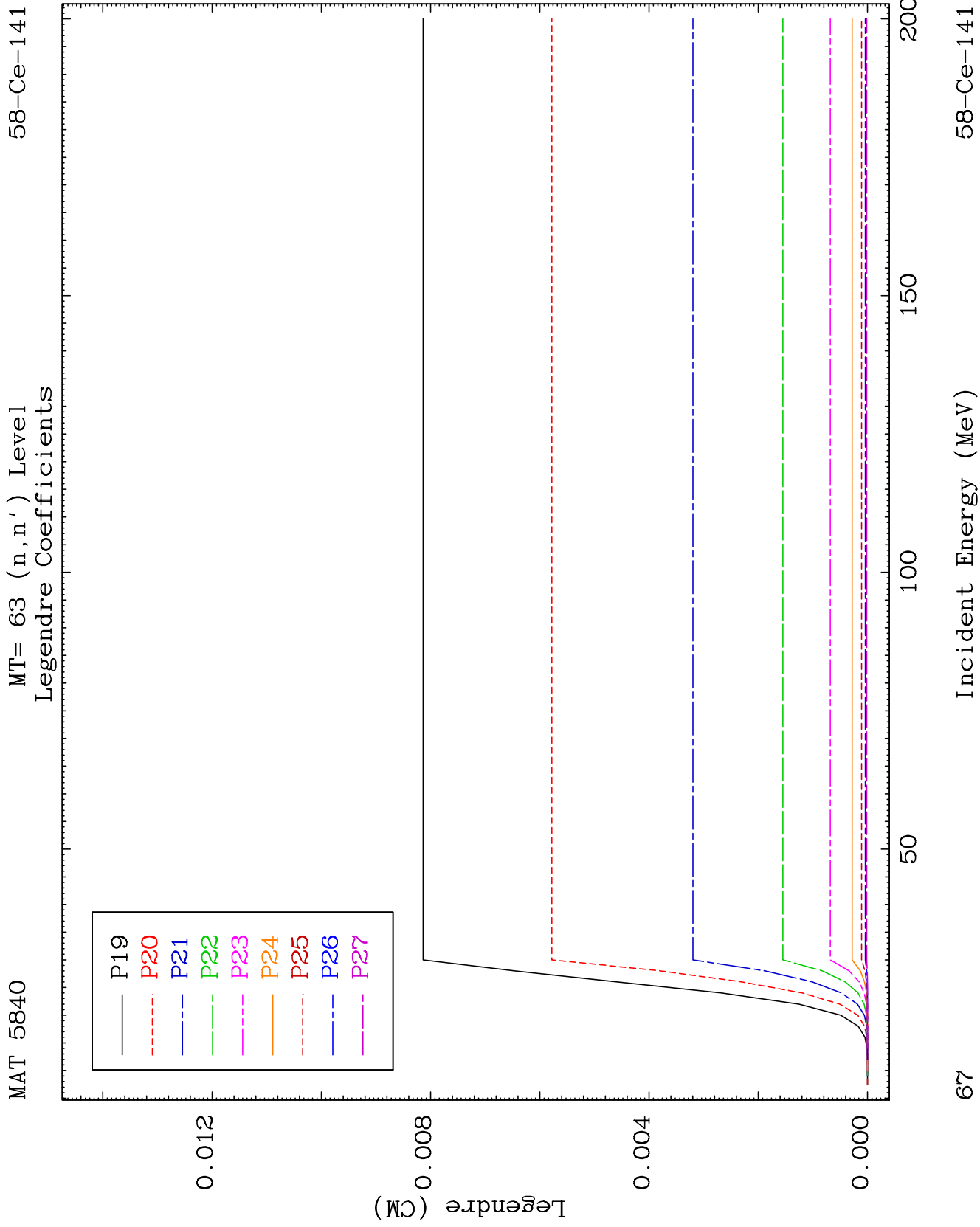
Incident Energy (MeV)

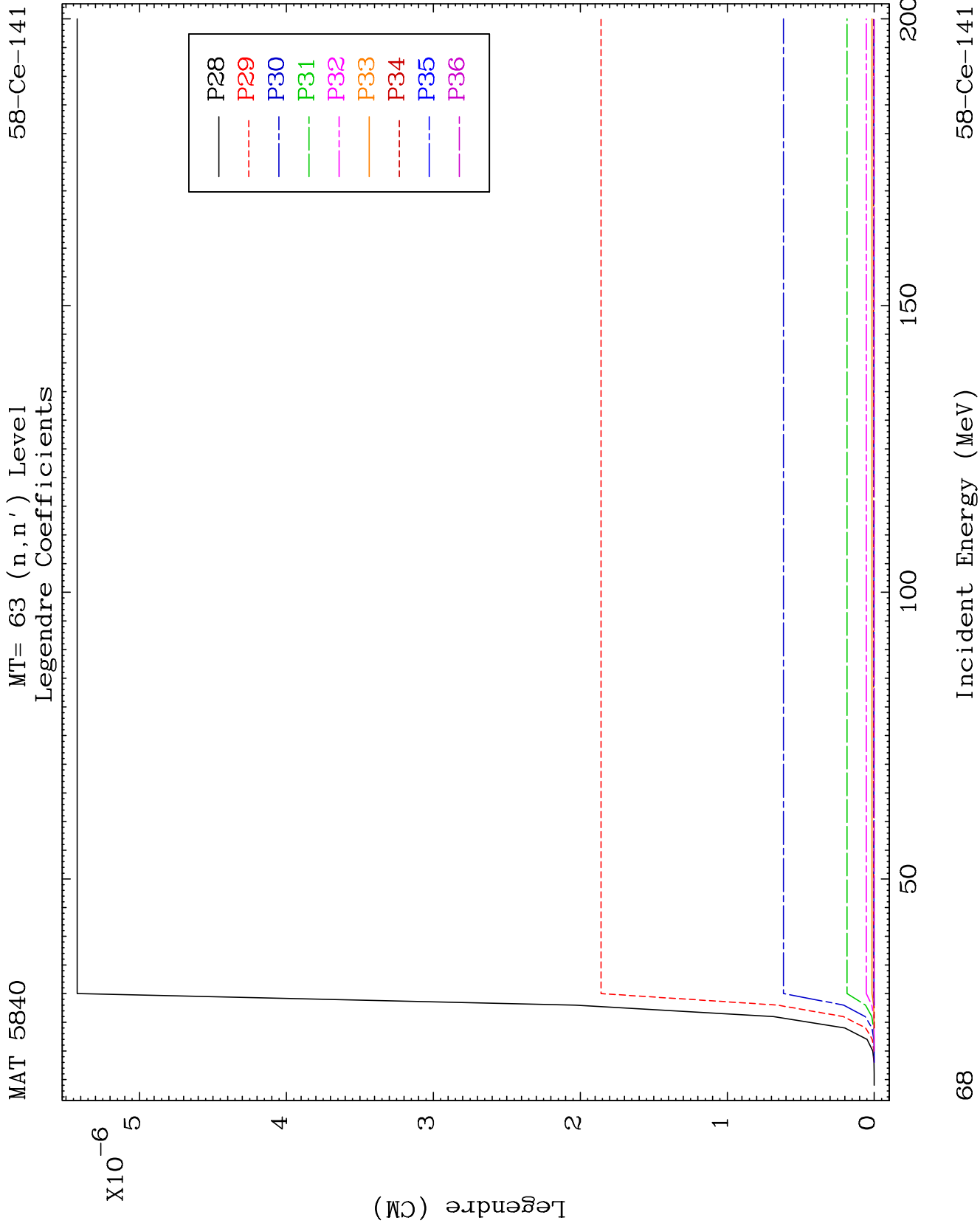
58-Ce-141







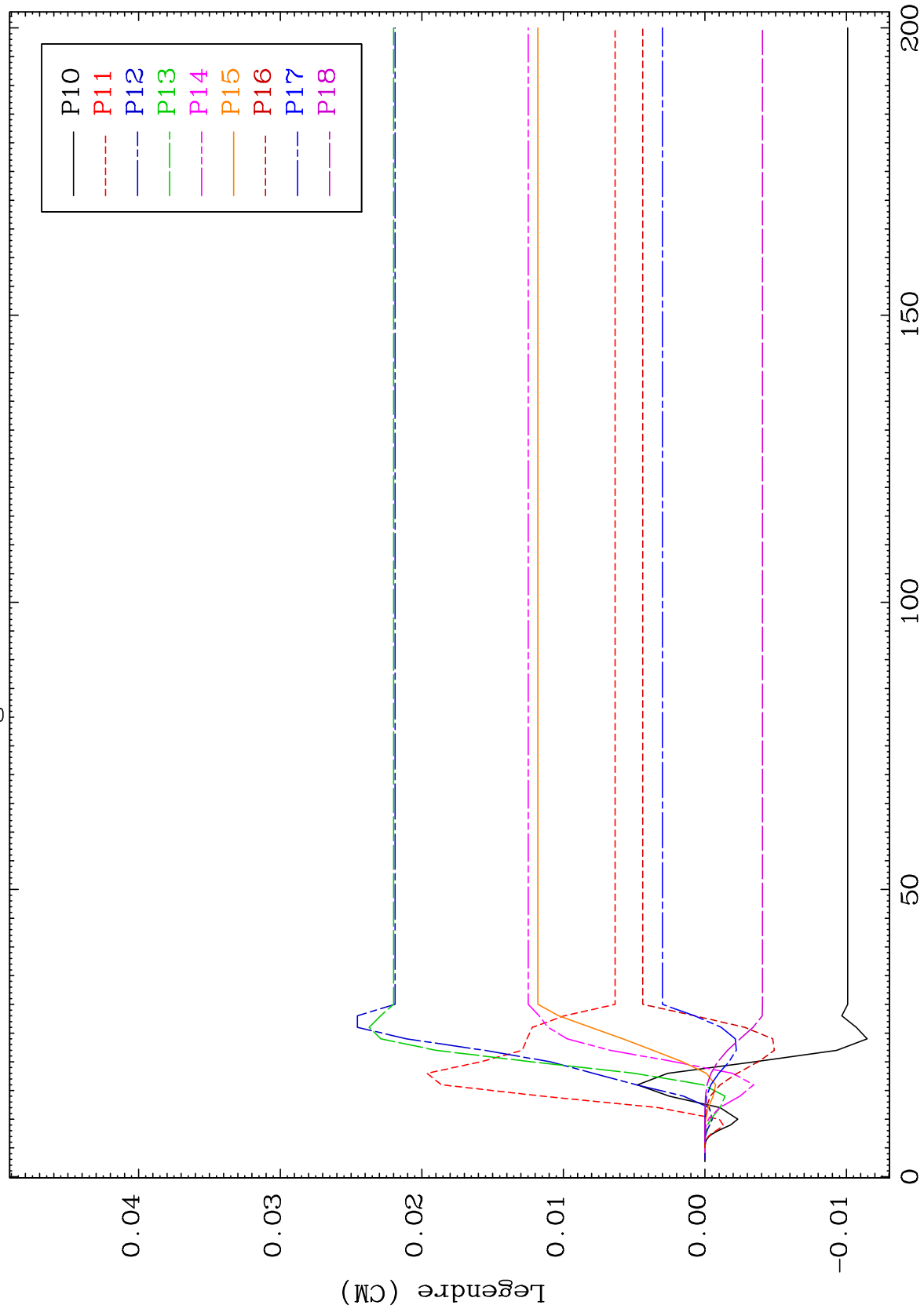




MAT 5840

MT= 64 (n, n') Level
Legendre Coefficients

58-Ce-141



20

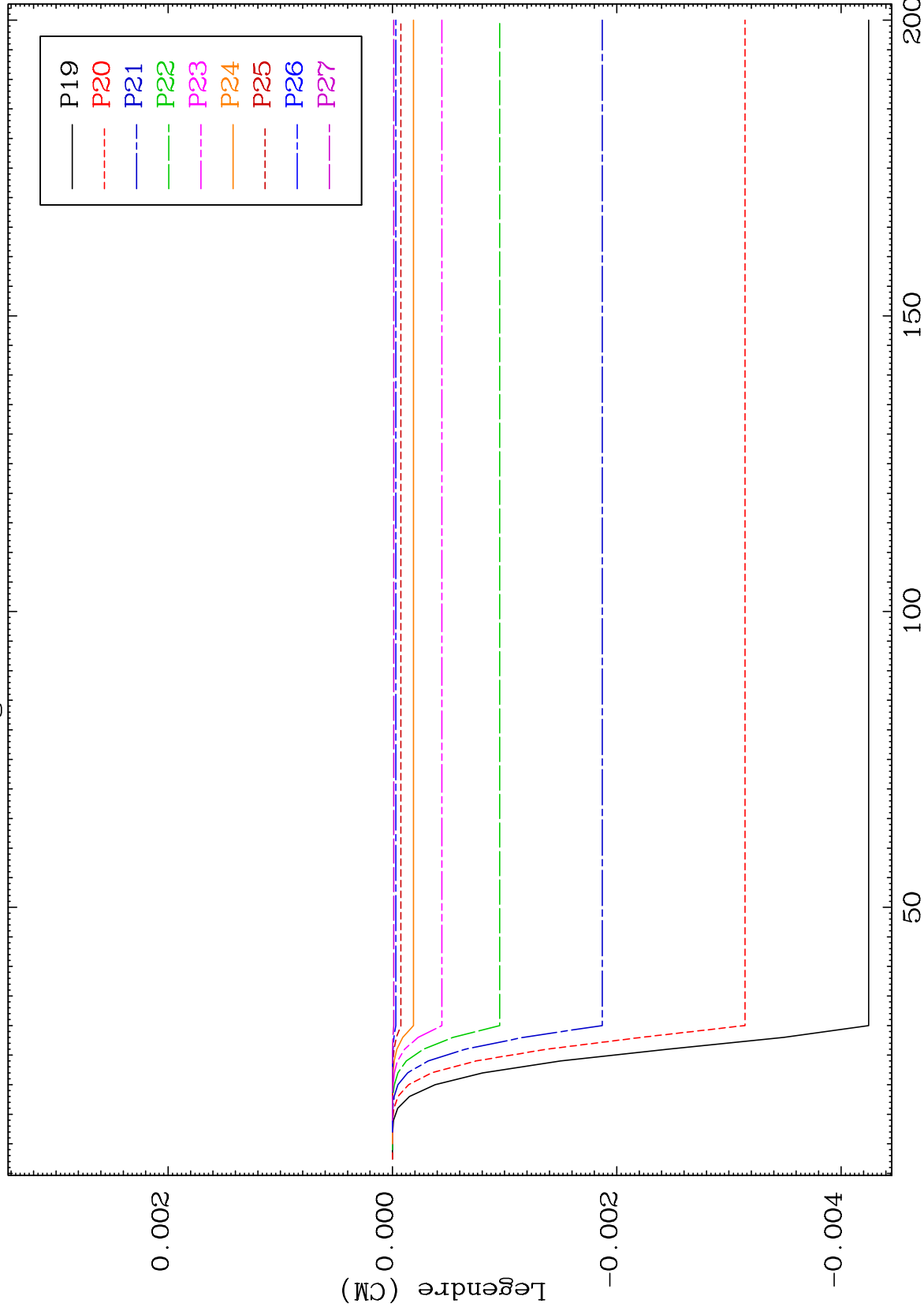
Incident Energy (MeV)

58-Ce-141

MAT 5840

MT= 64 (n,n') Level
Legendre Coefficients

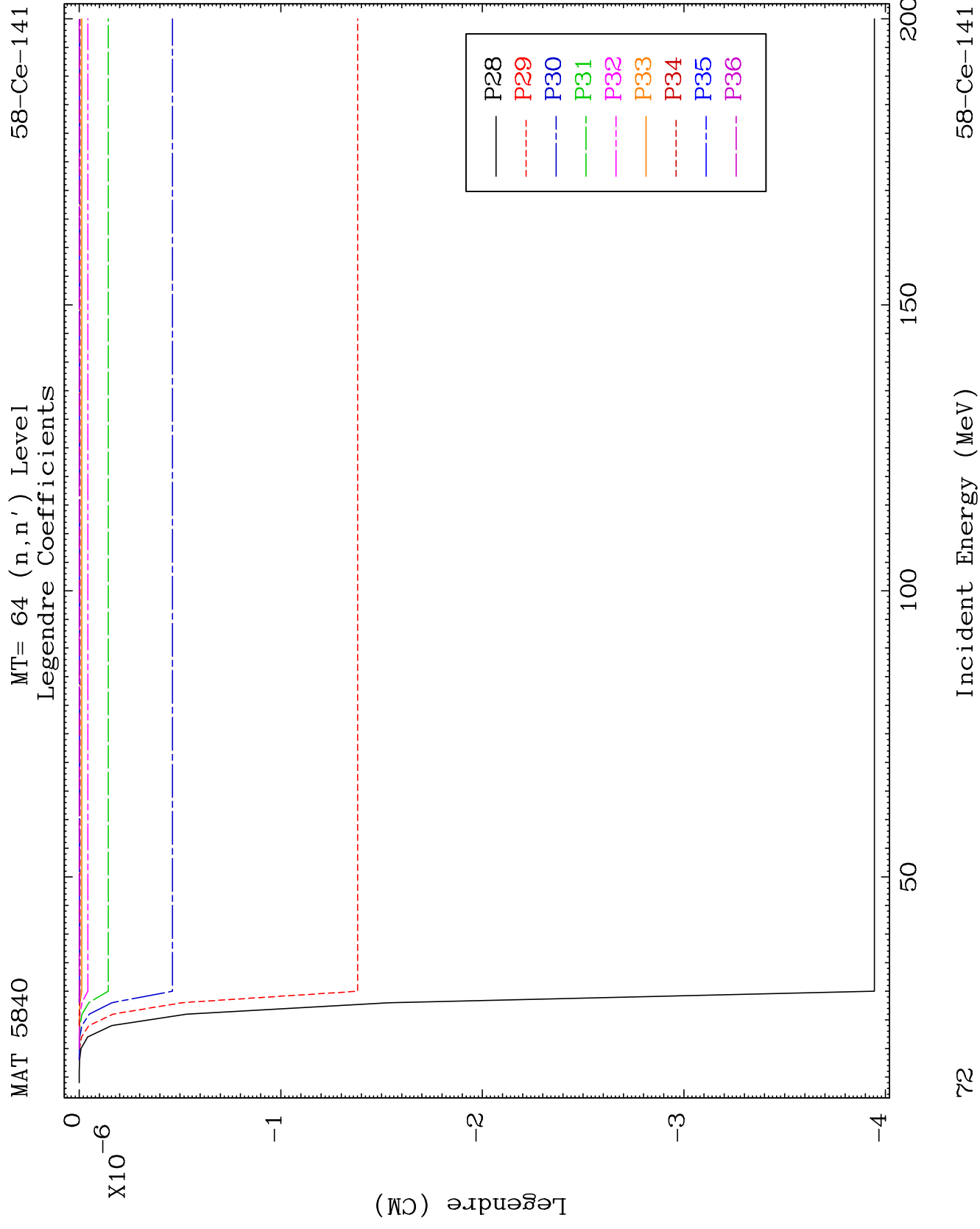
58-Ce-141



71

Incident Energy (MeV)

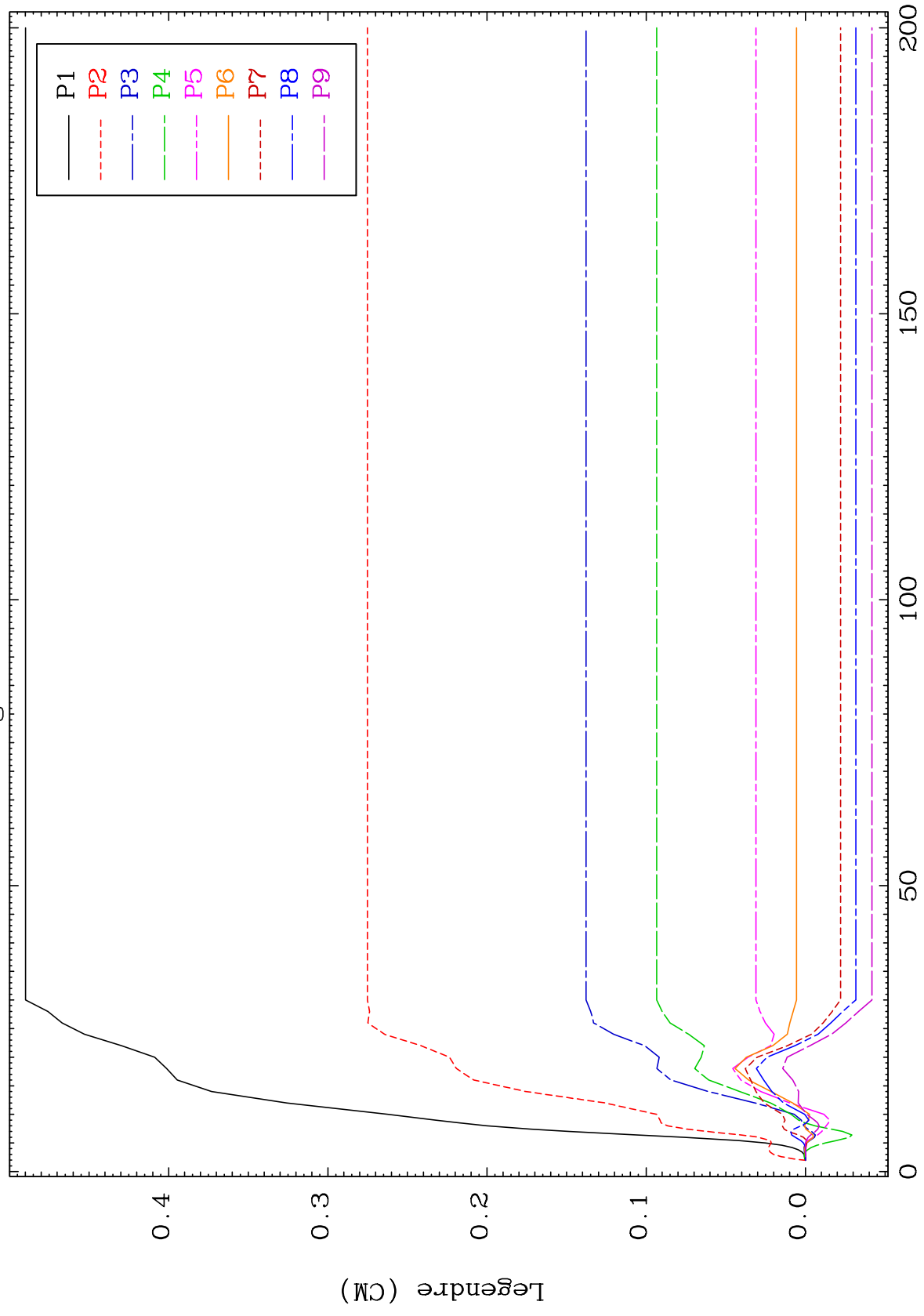
58-Ce-141



MAT 5840

MT= 65 (n, n') Level
Legendre Coefficients

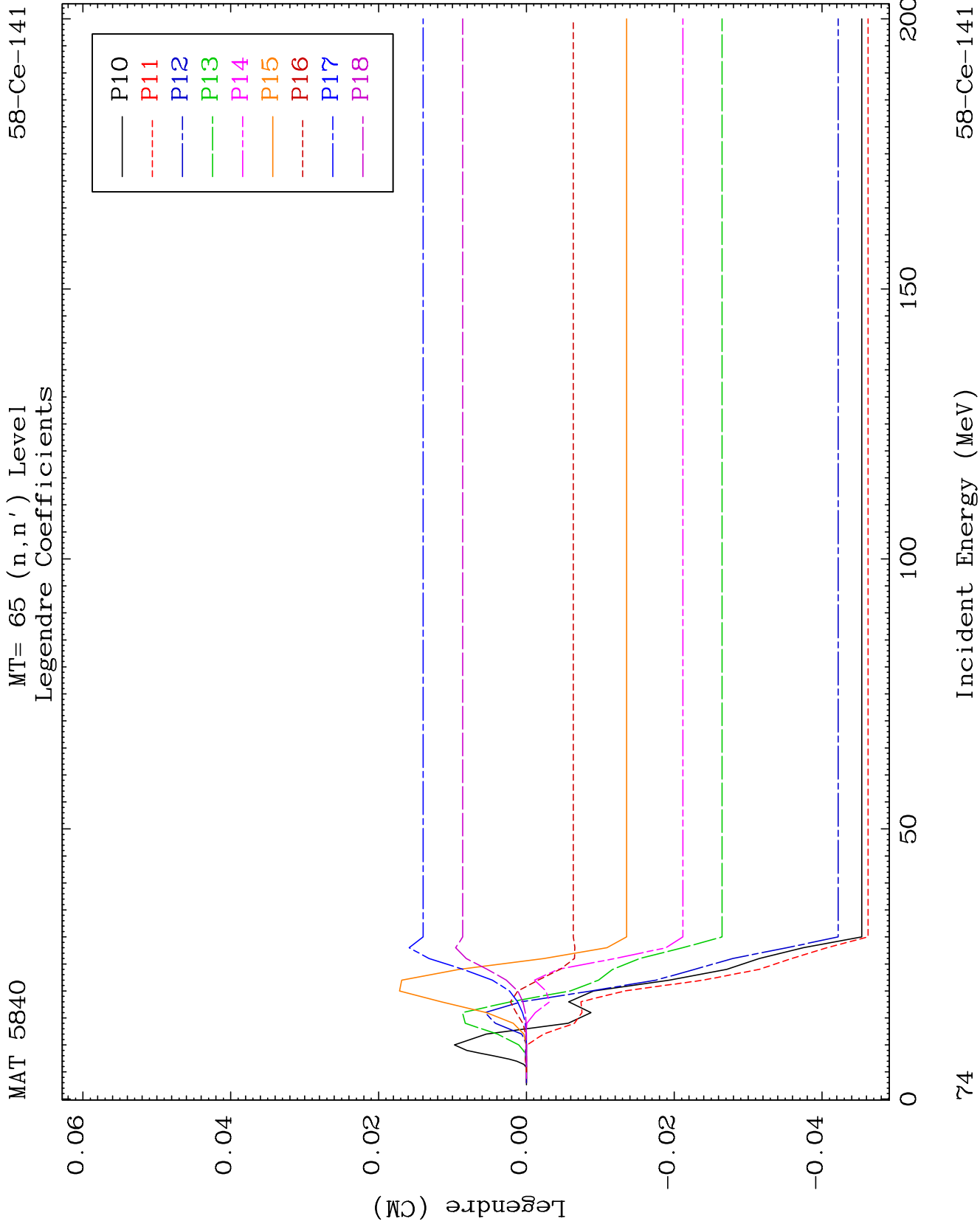
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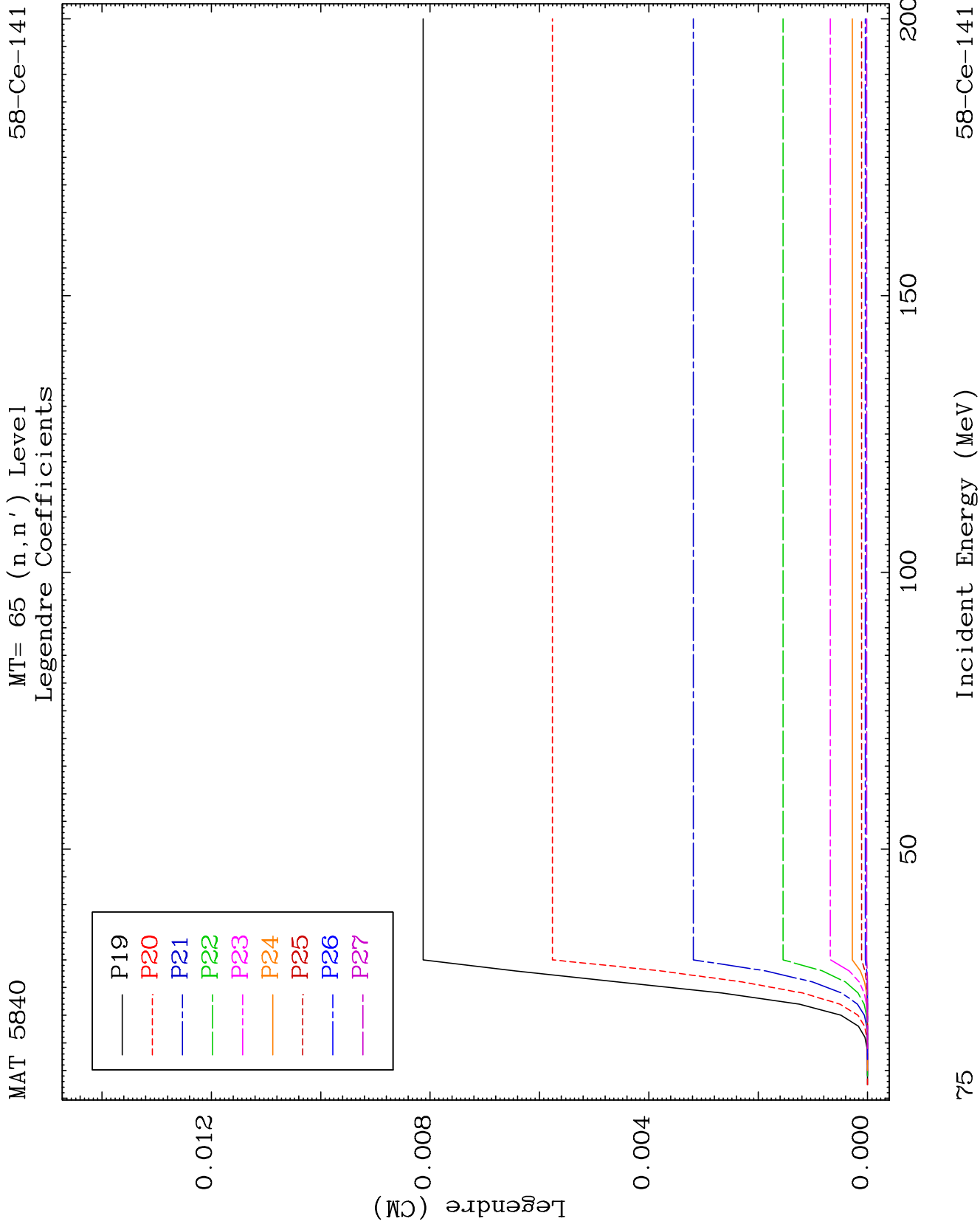


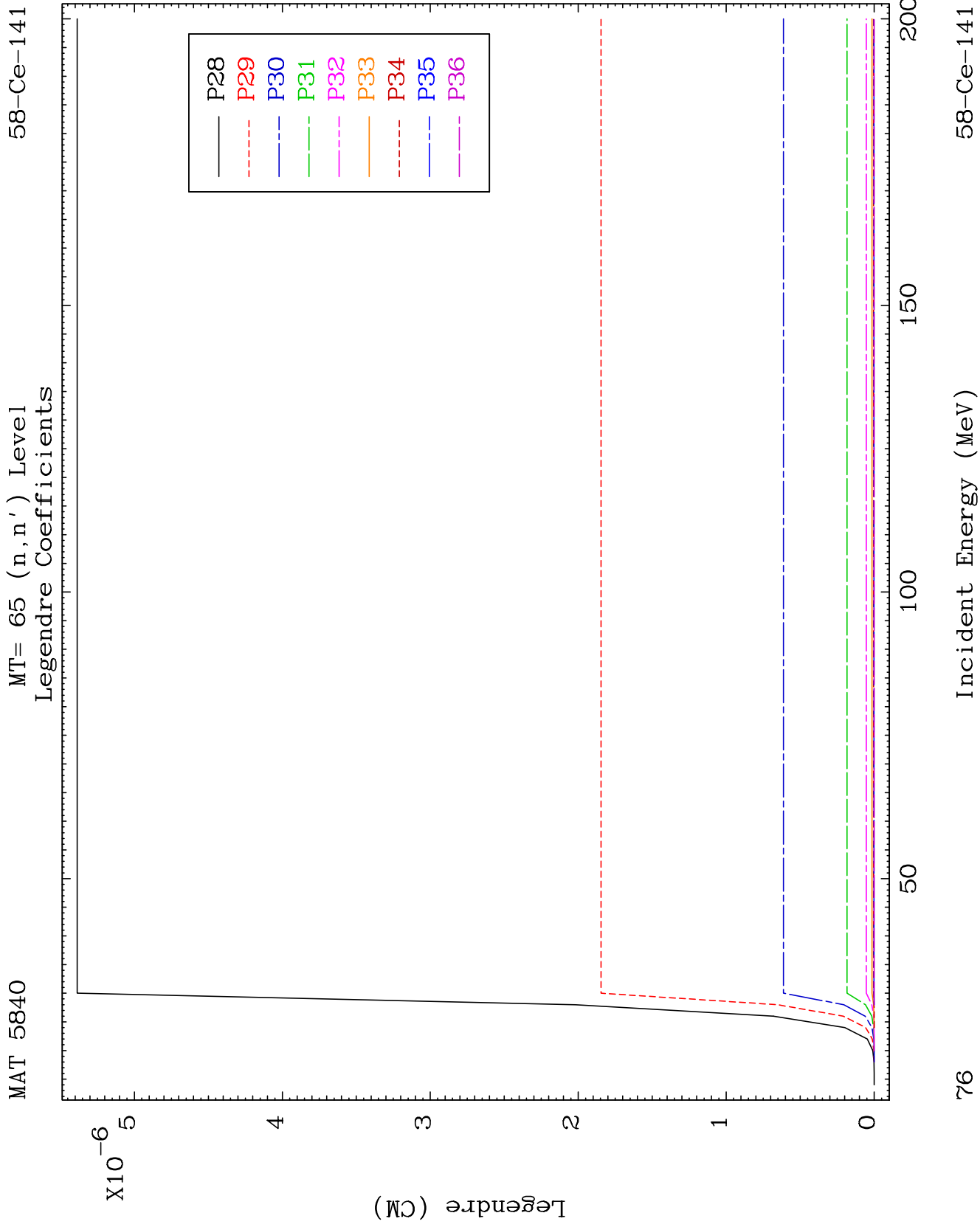
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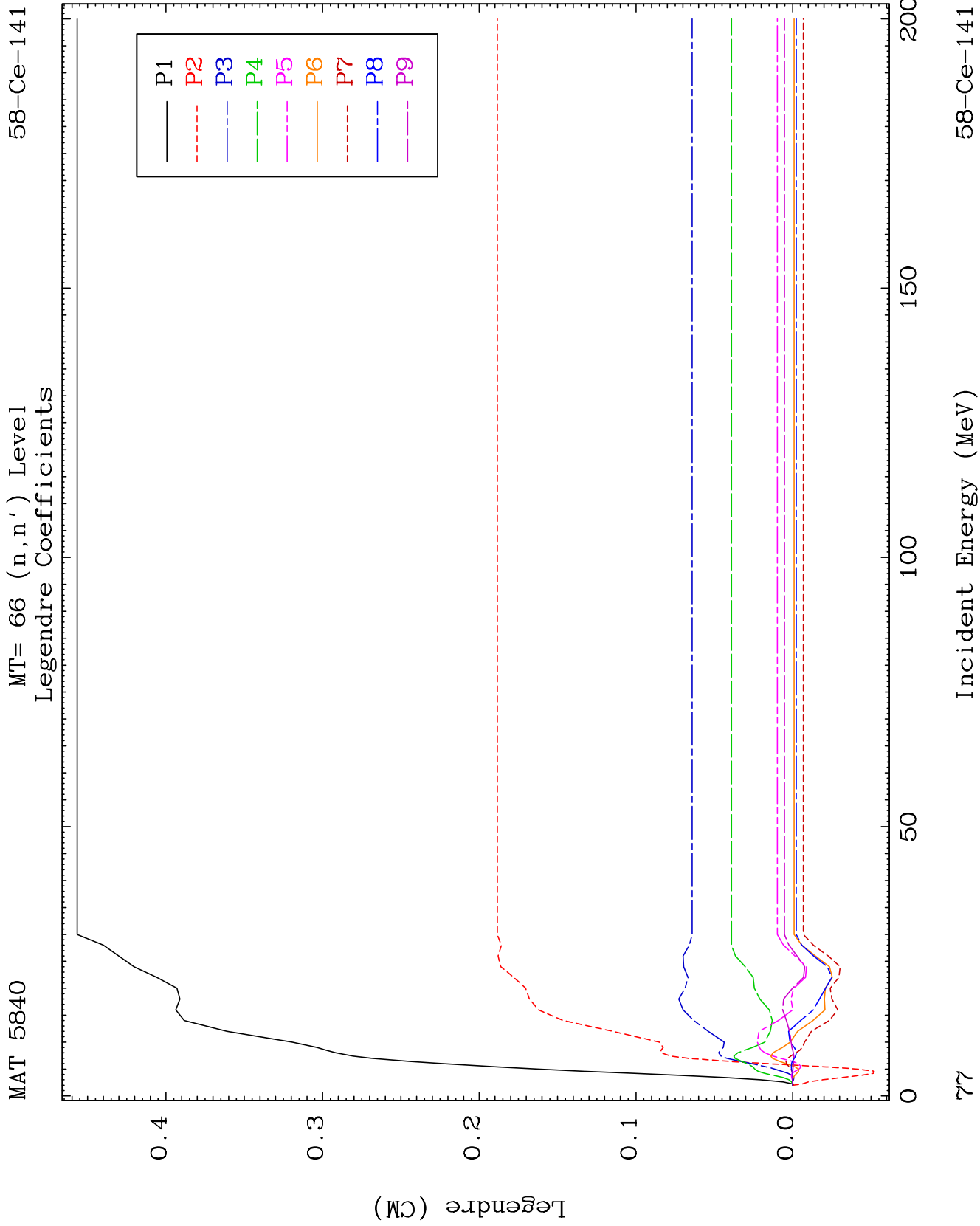
Incident Energy (MeV)

58-Ce-141





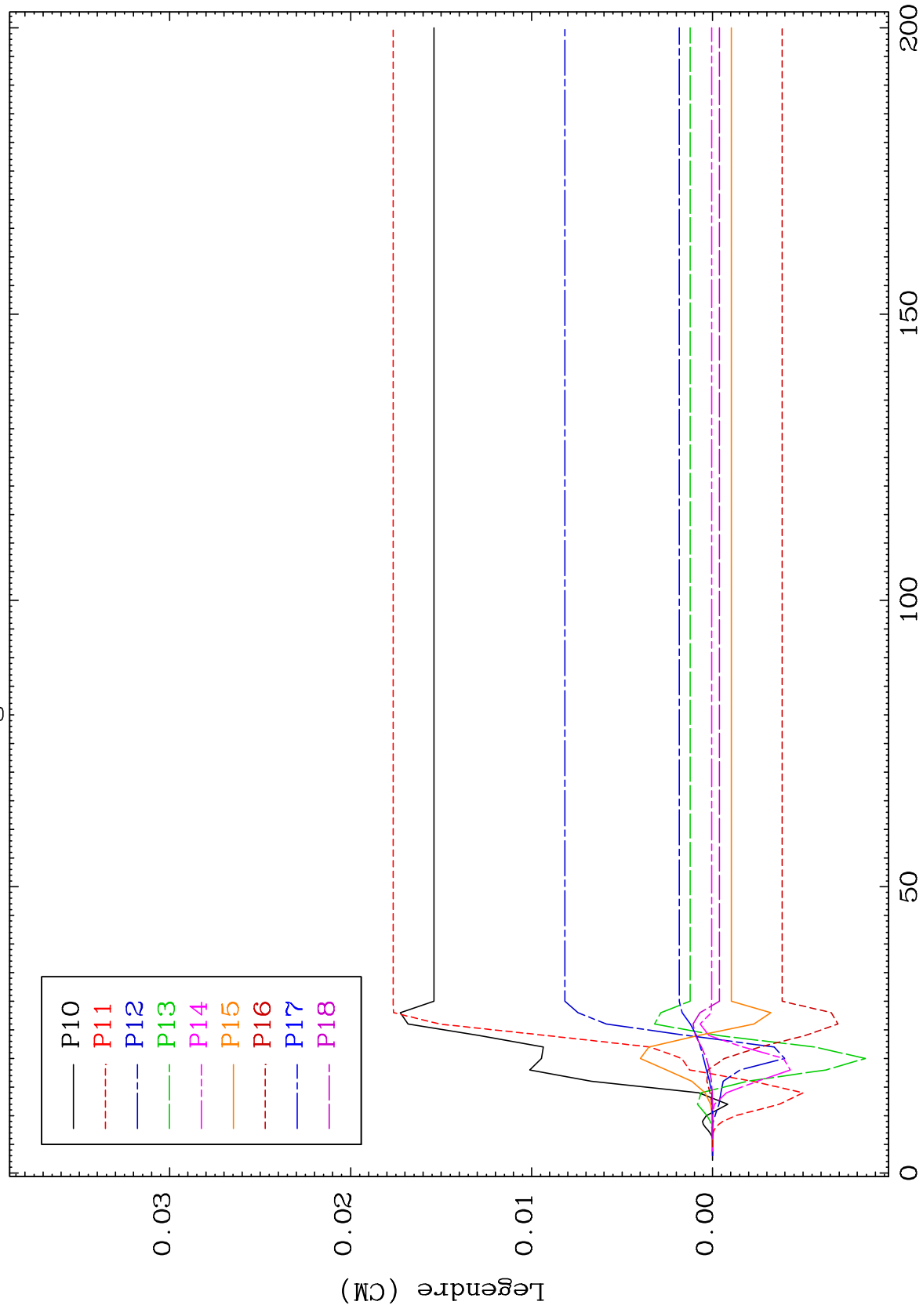




MAT 5840

MT= 66 (n, n') Level
Legendre Coefficients

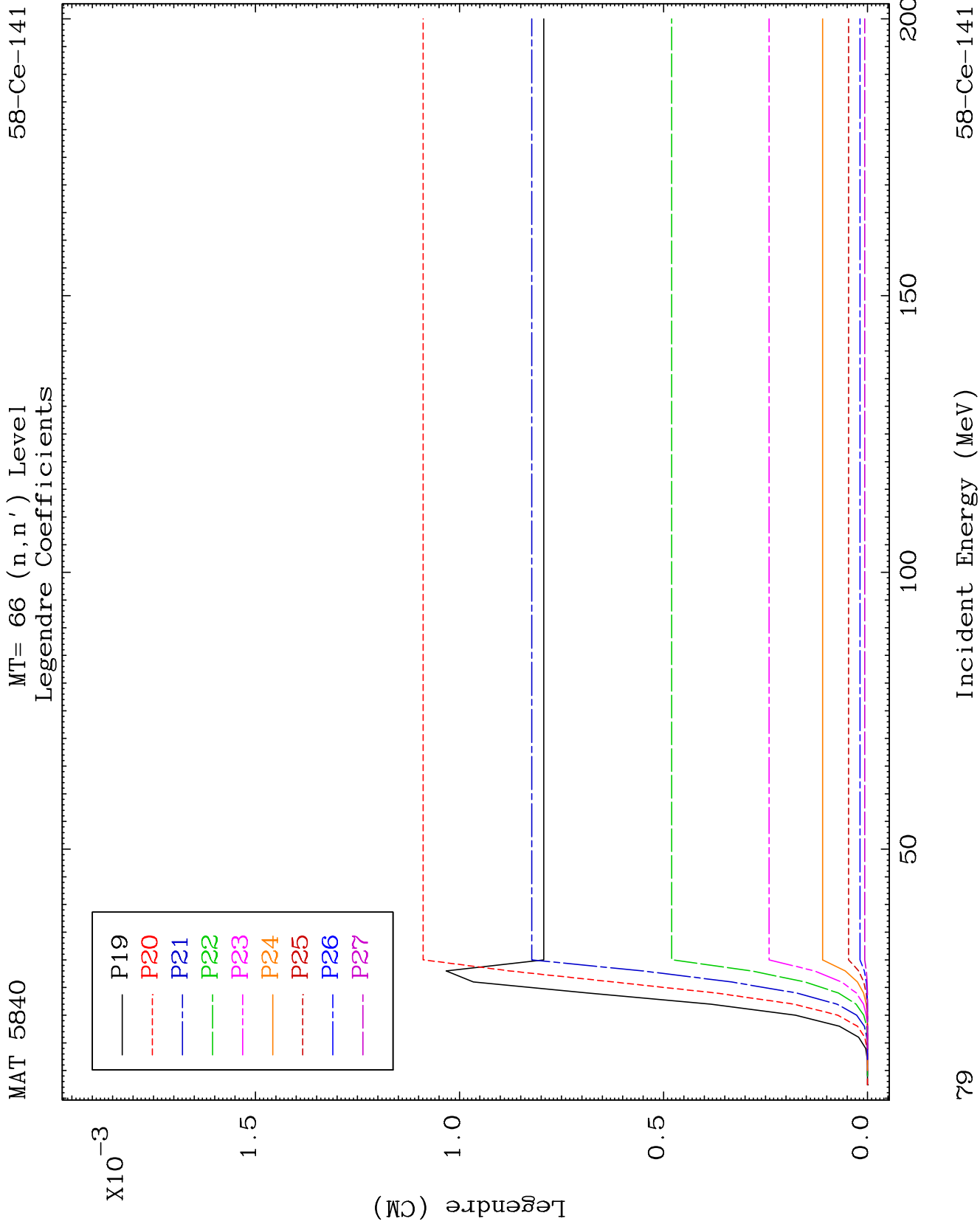
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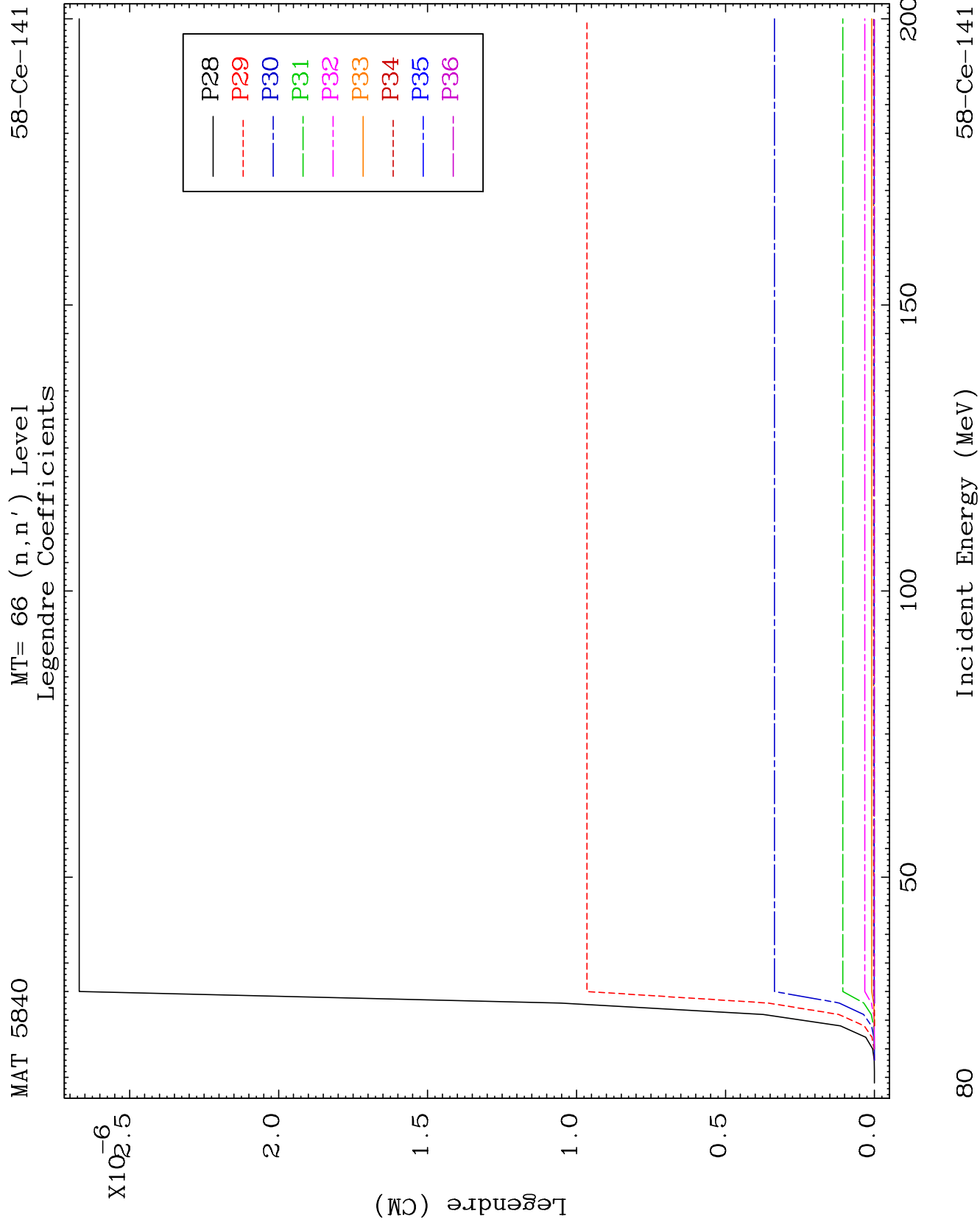


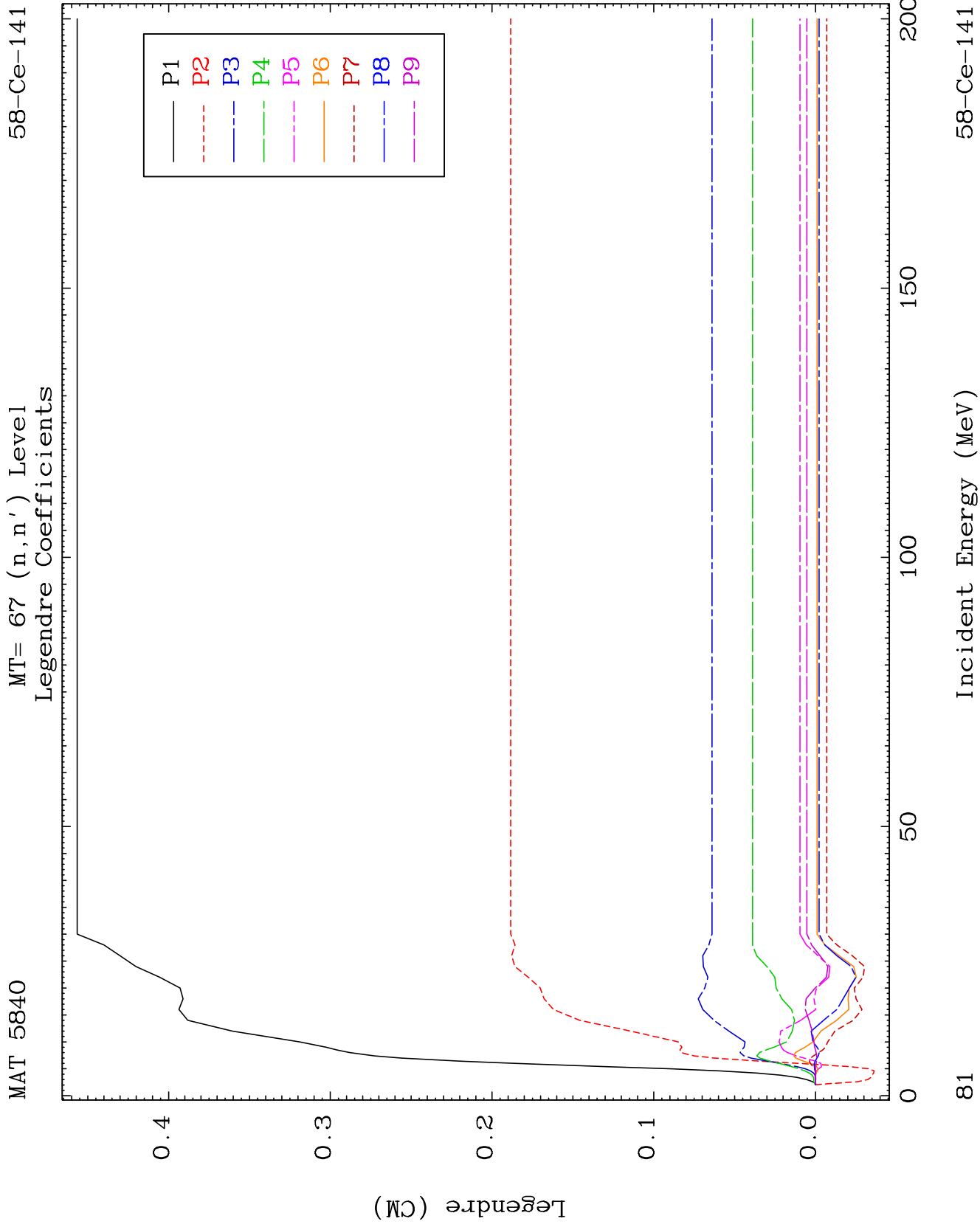
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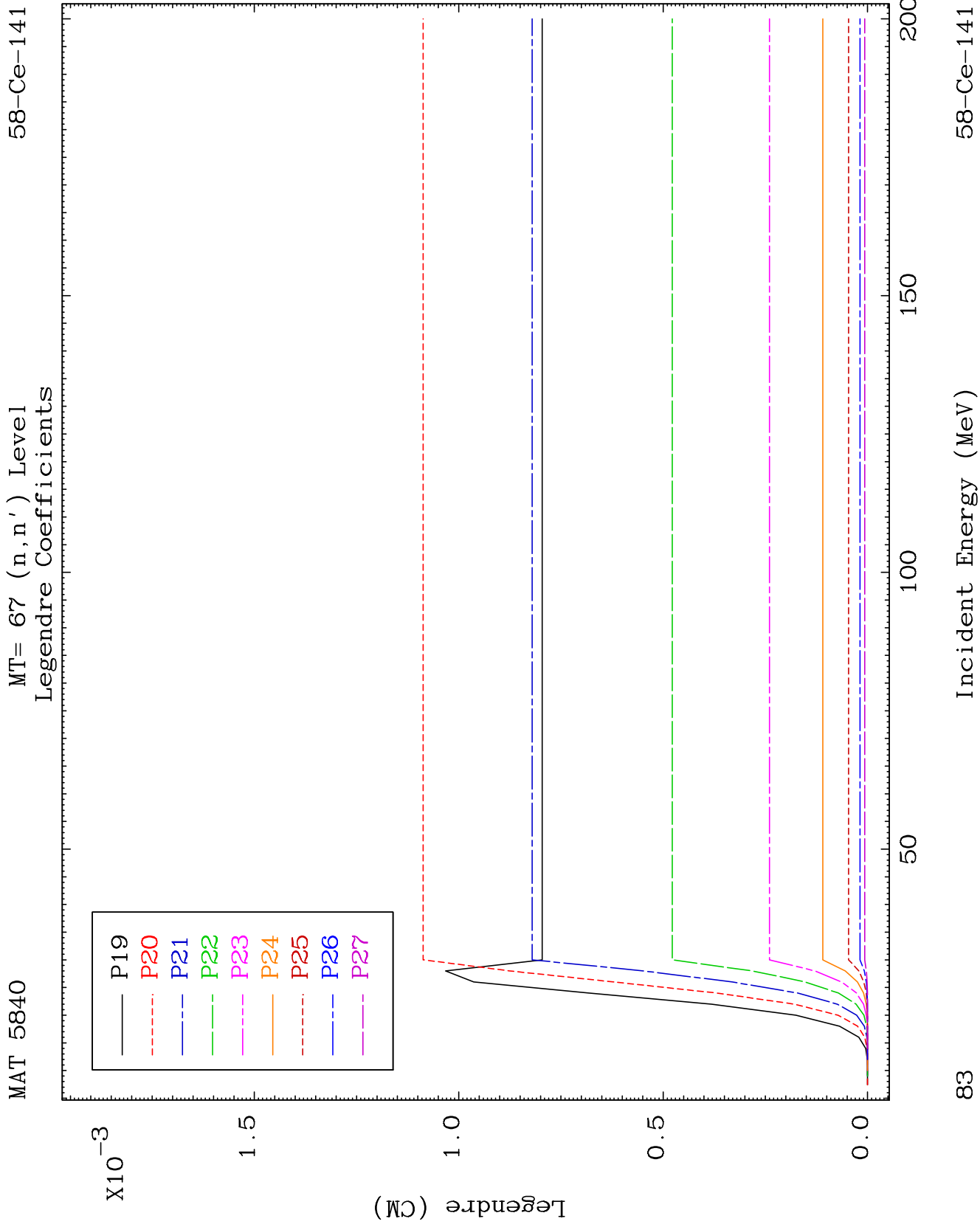
Incident Energy (MeV)

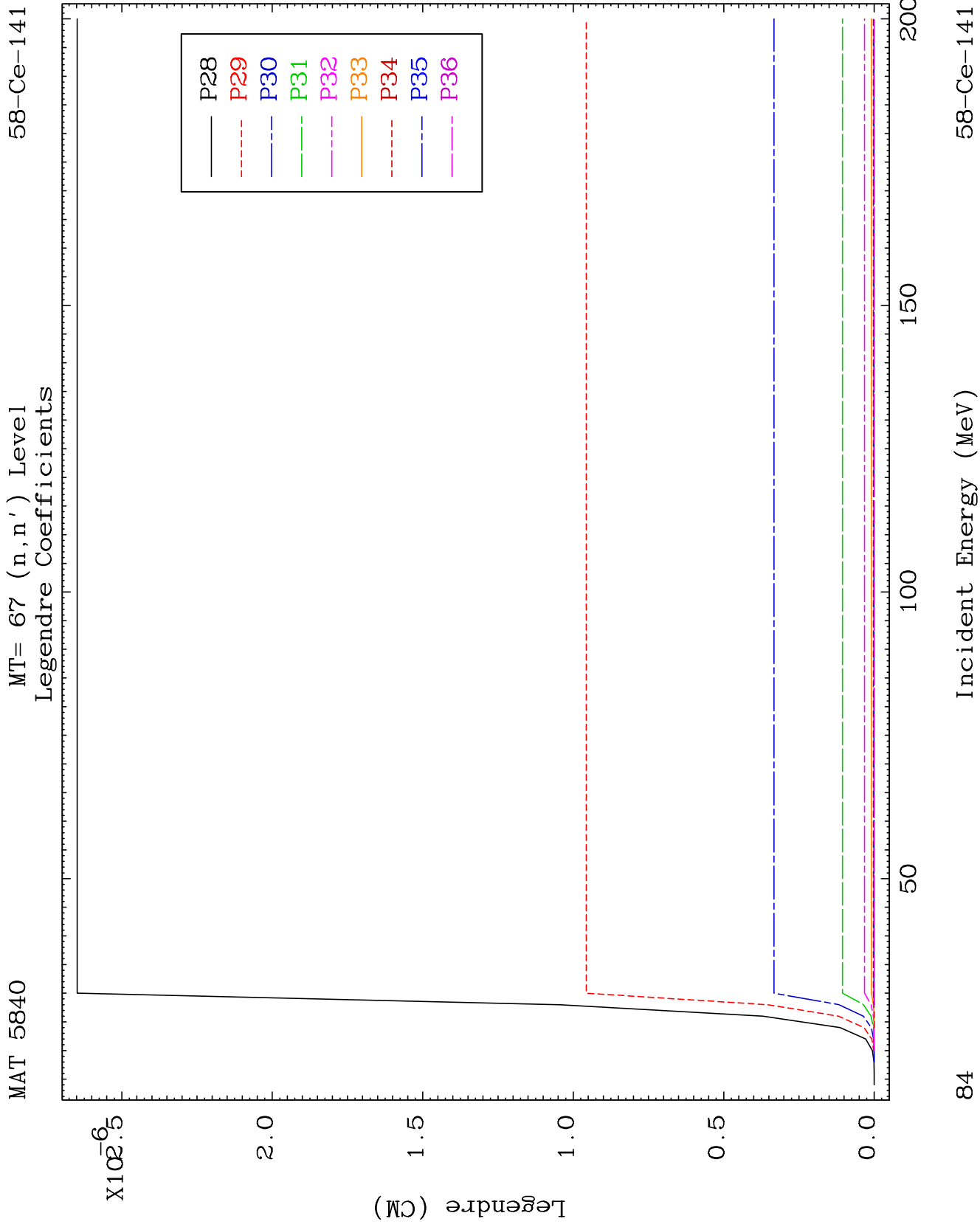
58-Ce-141

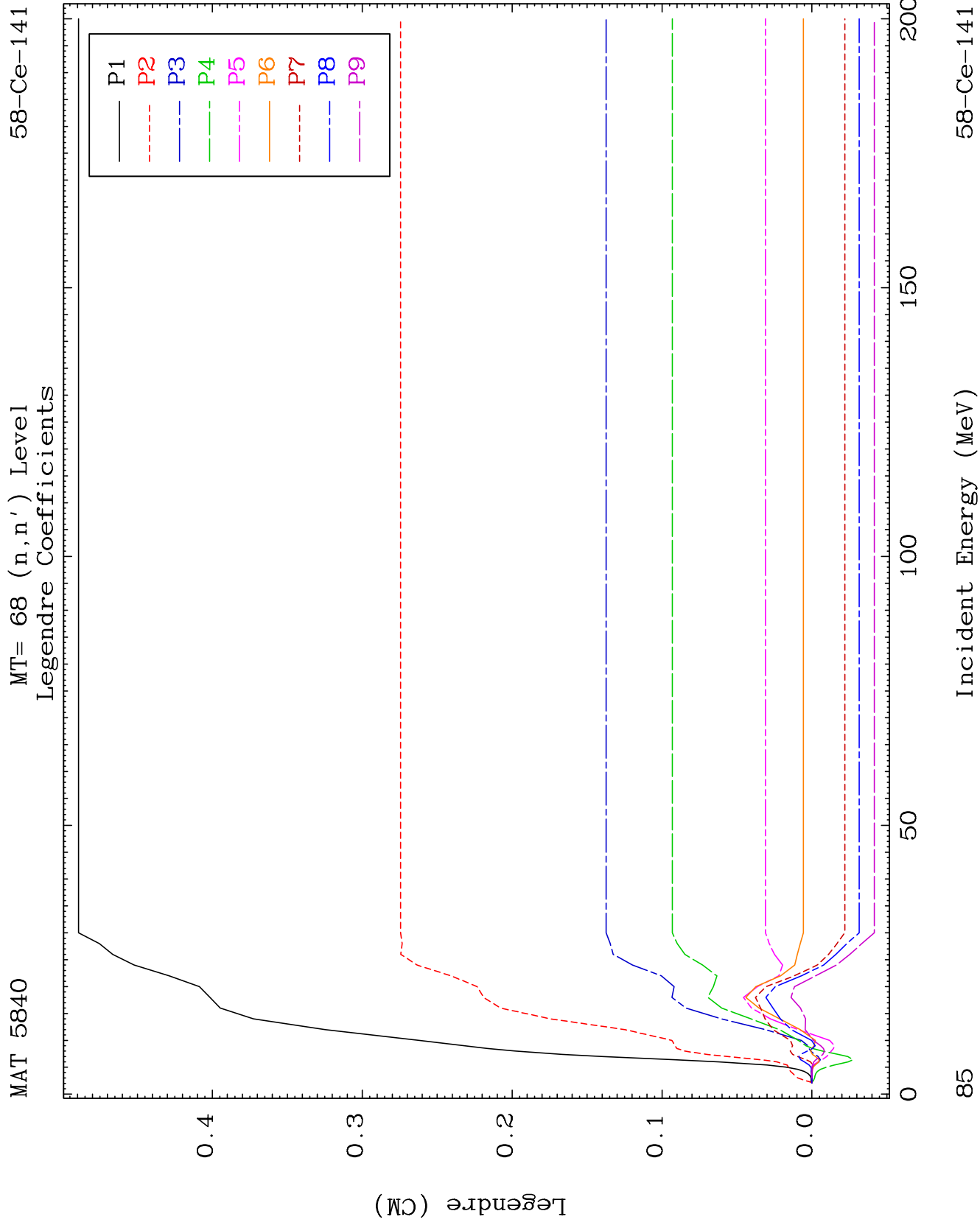


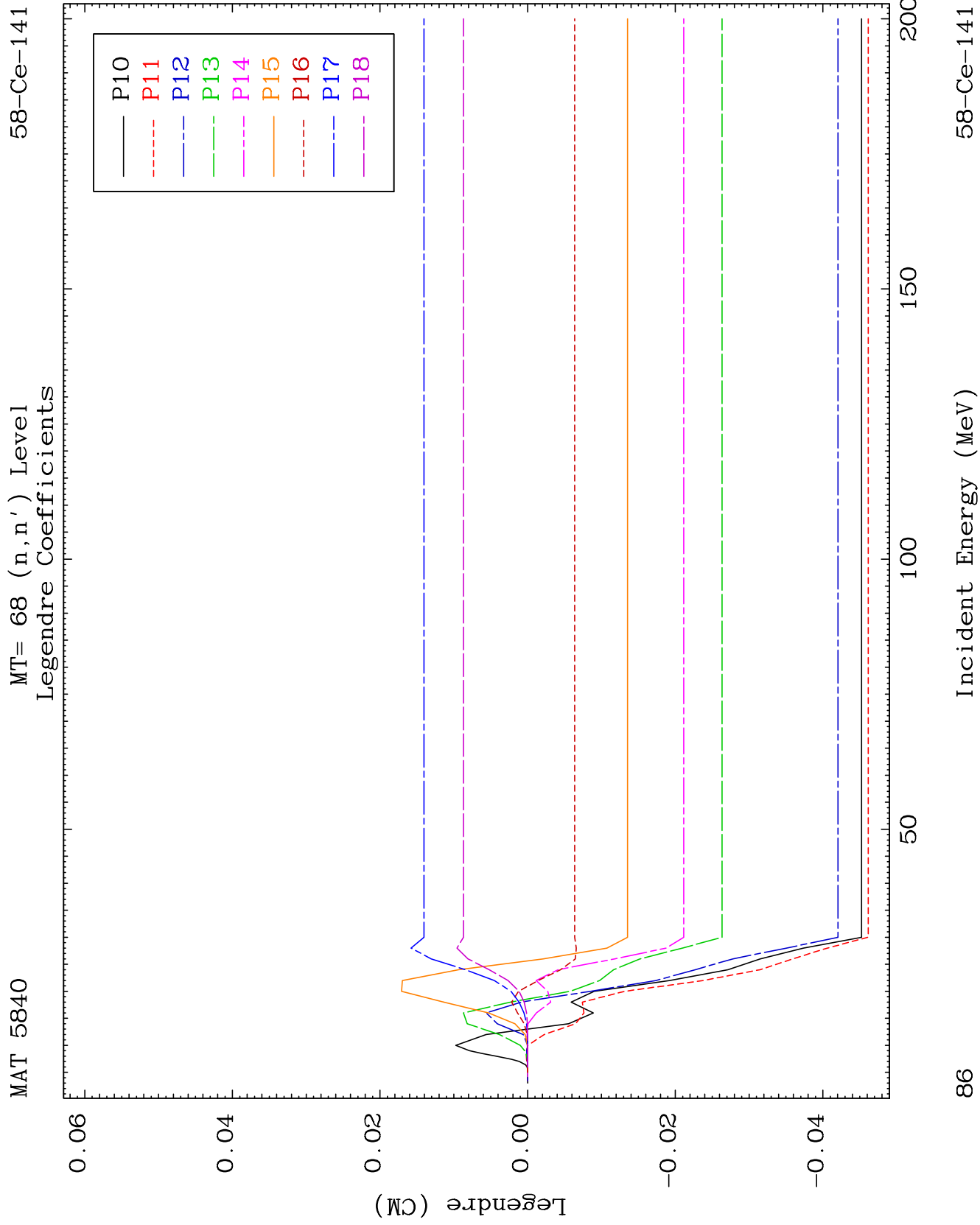








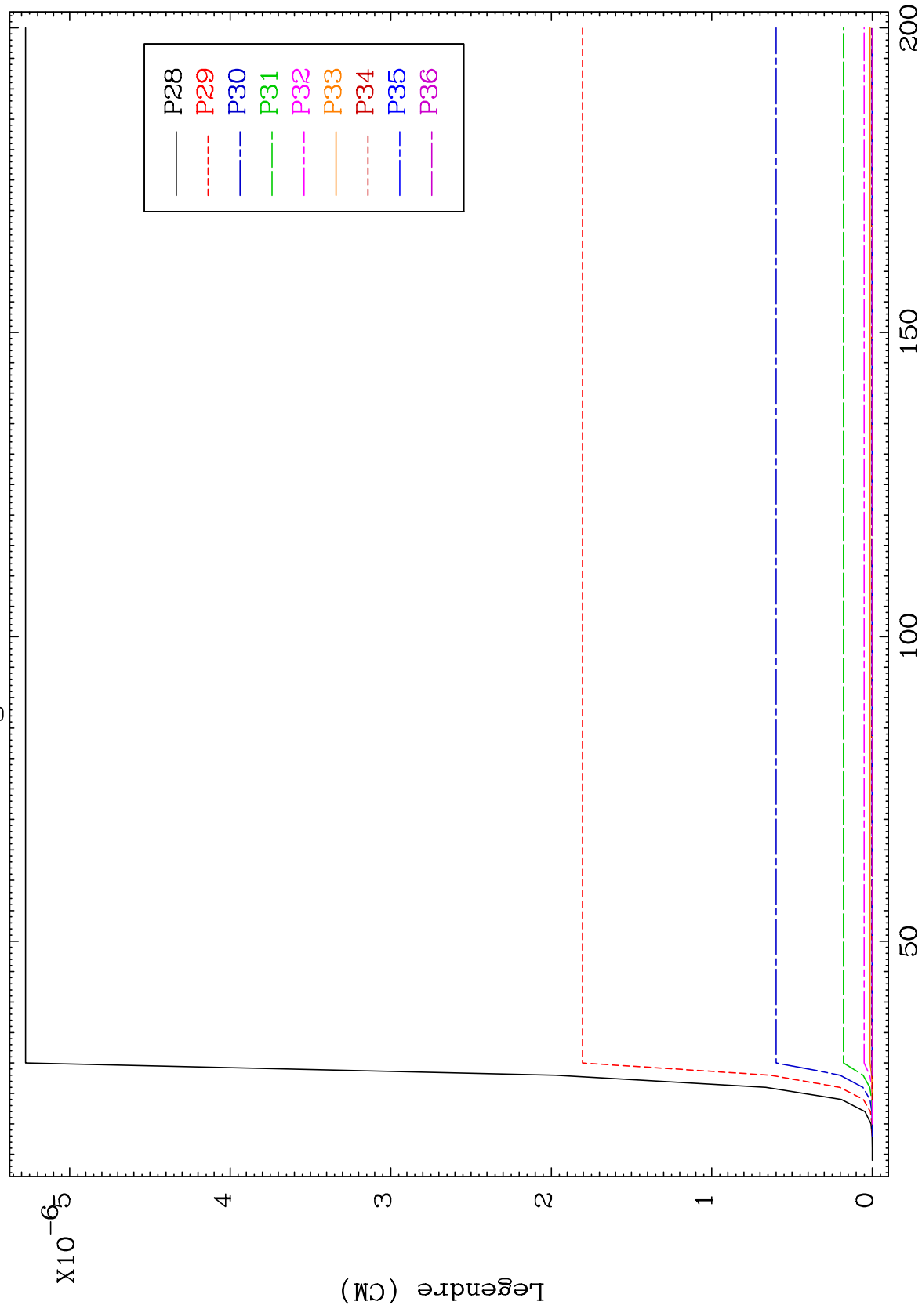




MAT 5840

MT= 68 (n, n') Level
Legendre Coefficients

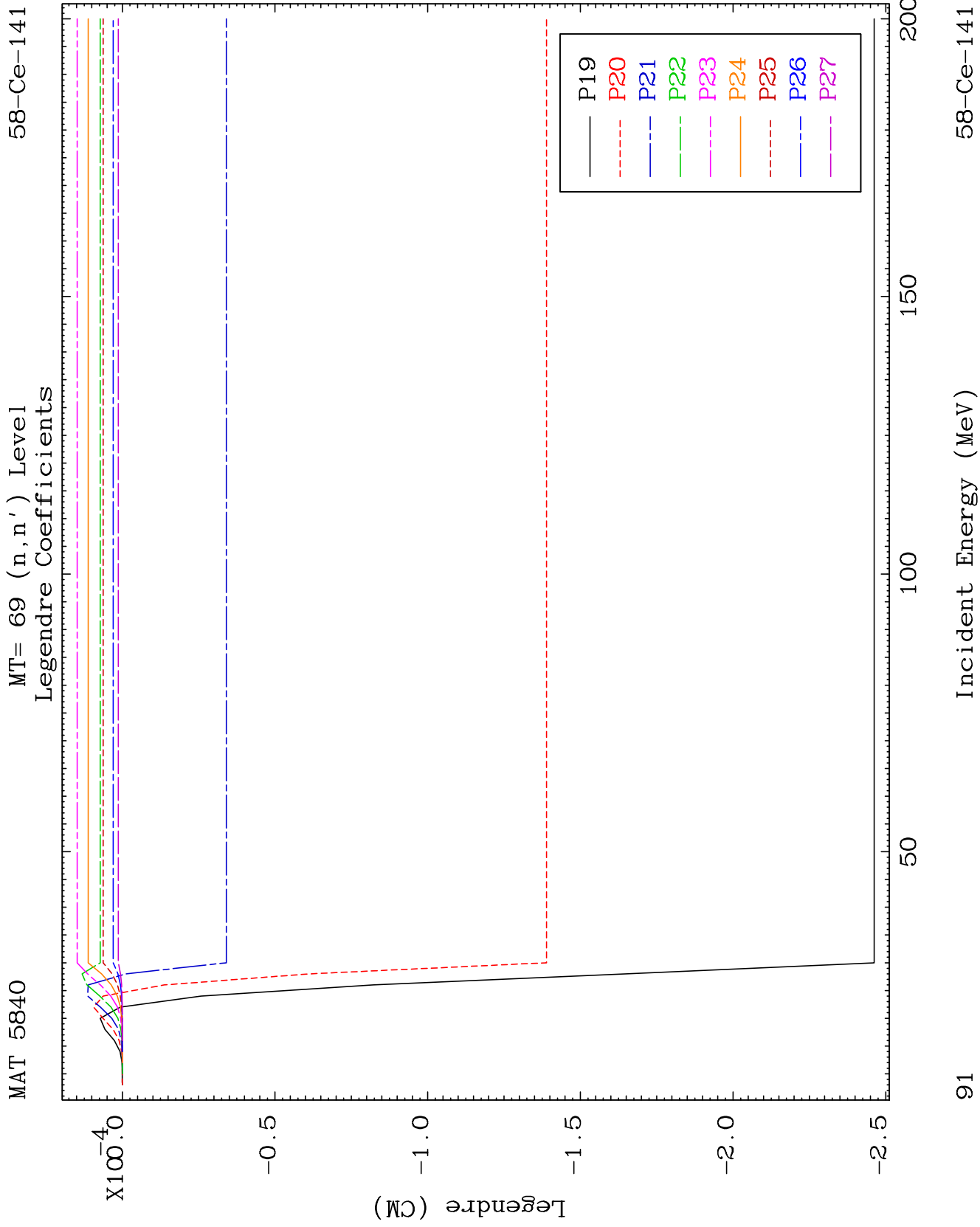
58-Ce-141

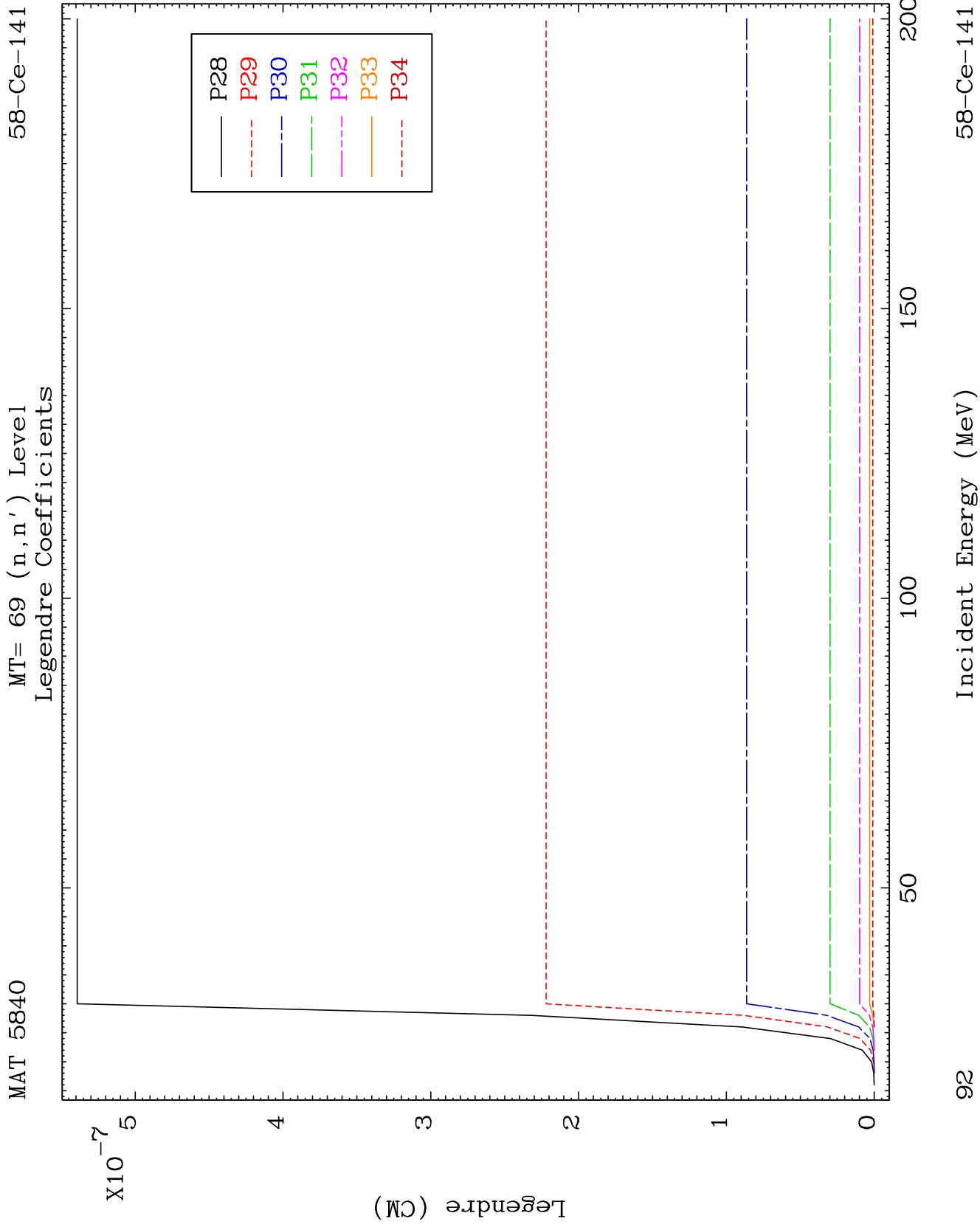


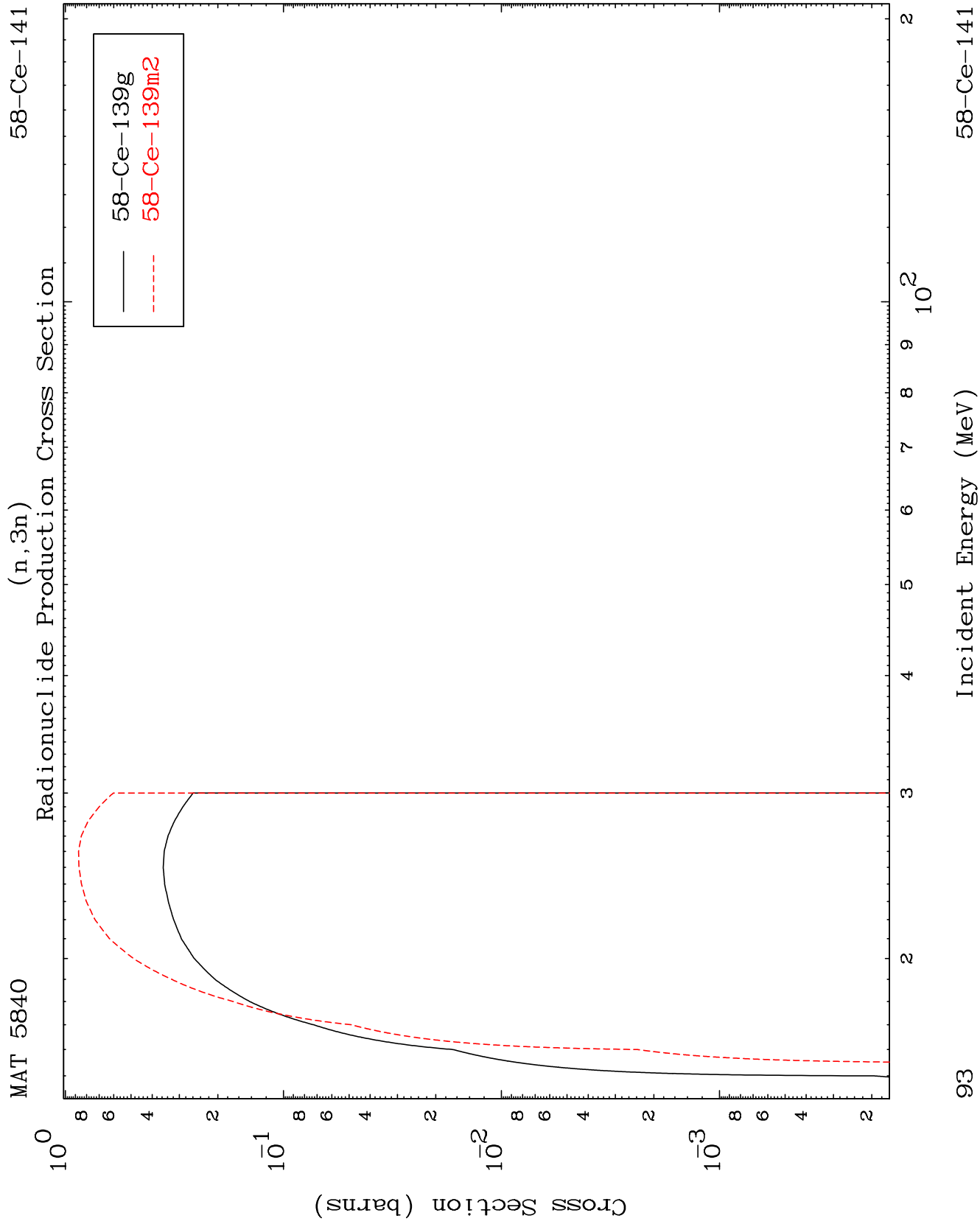
88

Incident Energy (MeV)

58-Ce-141





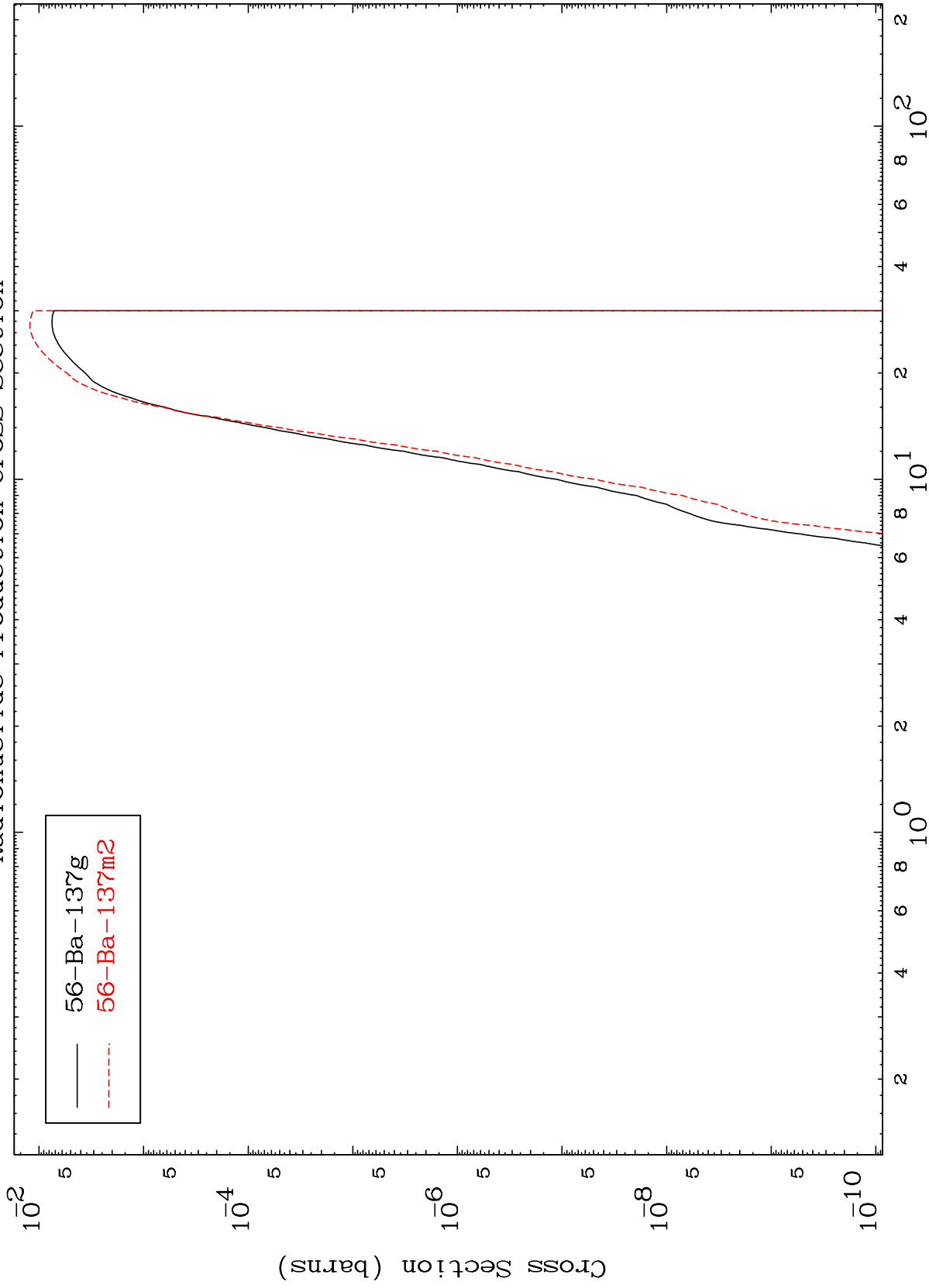


MAT 5840

$(n,n') \alpha$

58-Ce-141

Radionuclide Production Cross Section



94

Incident Energy (MeV)

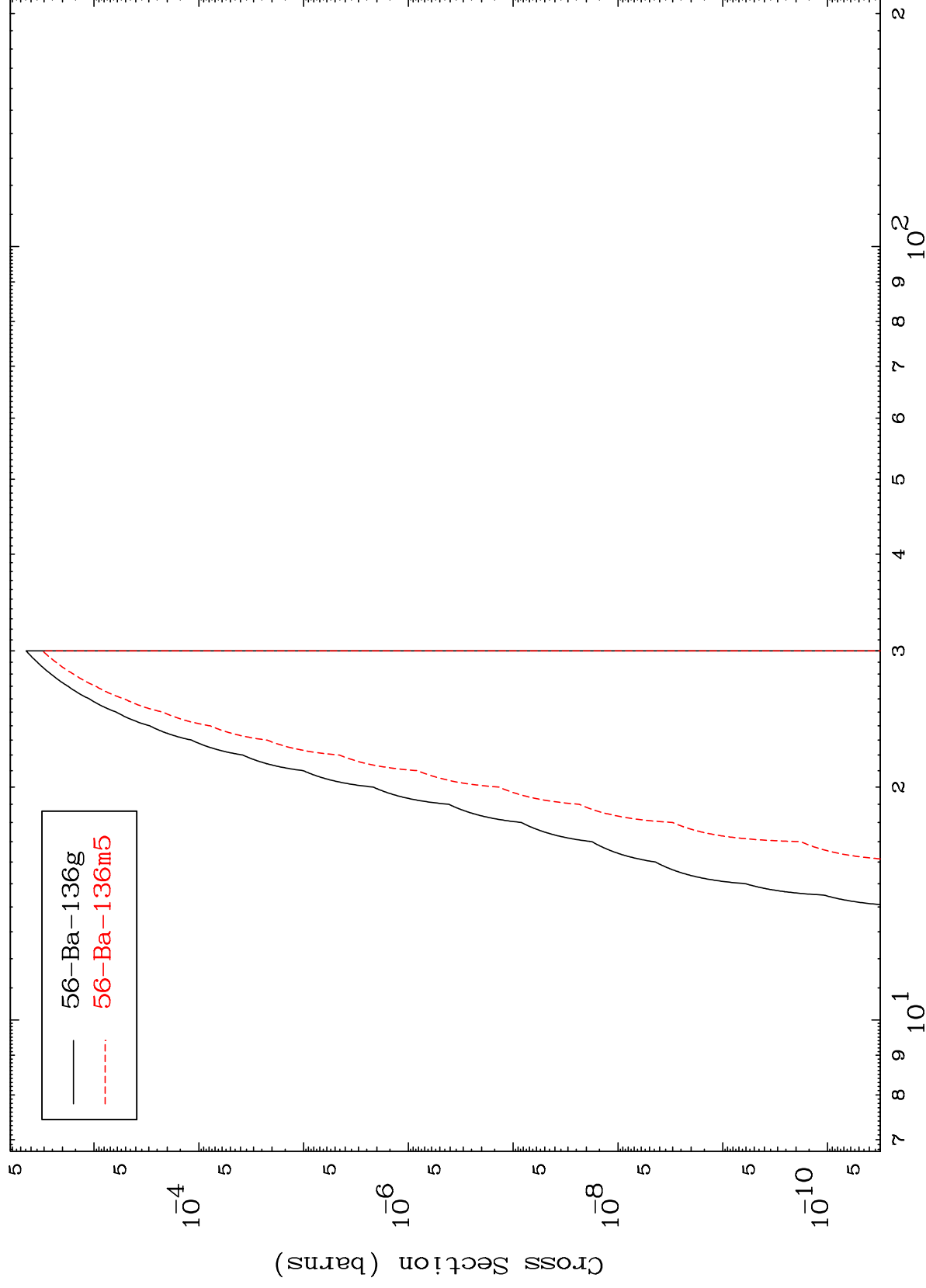
58-Ce-141

MAT 5840

$(n,2n) \alpha$

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Radionuclide Production Cross Section



95

Incident Energy (MeV)

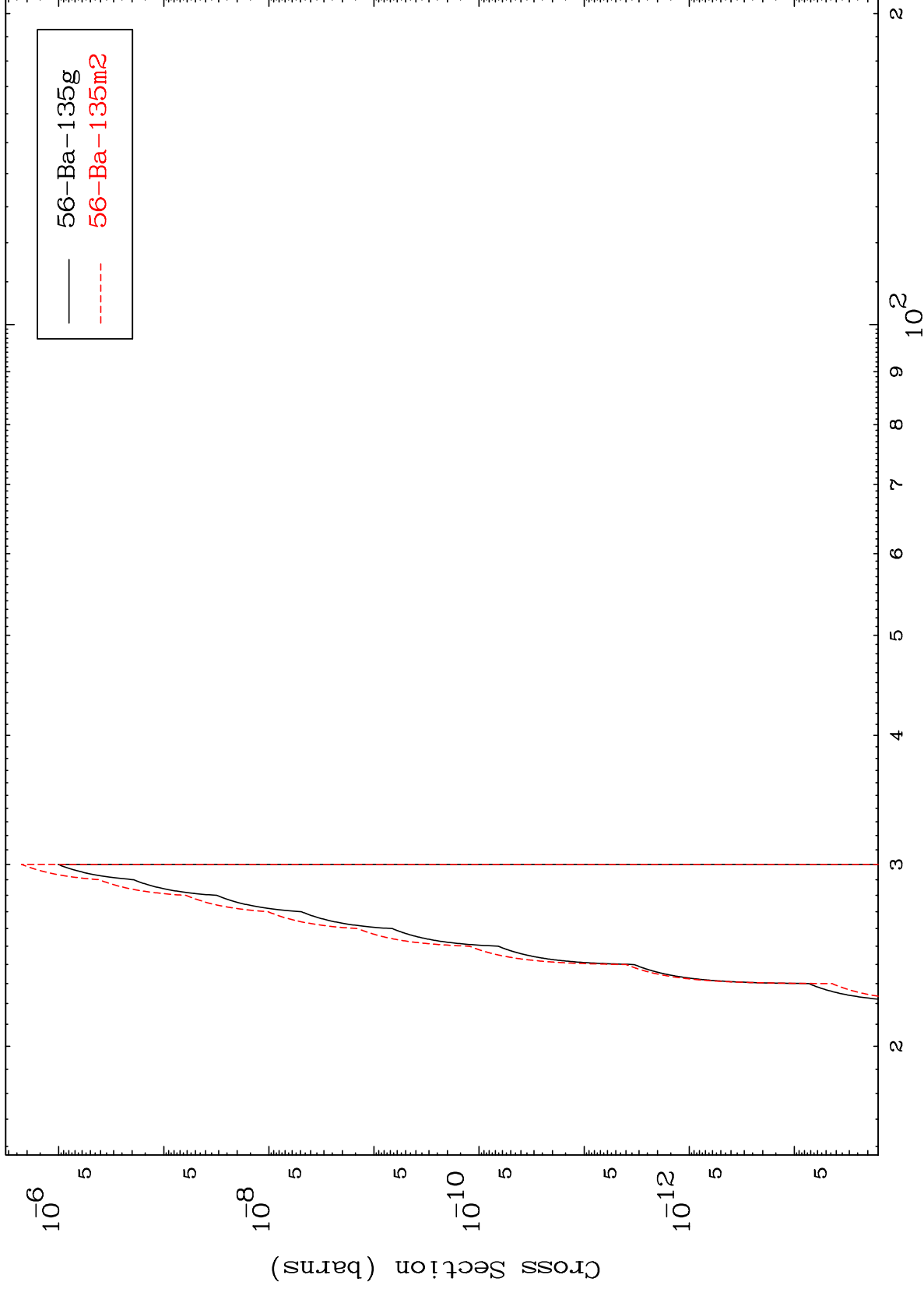
58-Ce-141

MAT 5840

$(n,3n) \alpha$

58-Ce-141

Radionuclide Production Cross Section



96

Incident Energy (MeV)

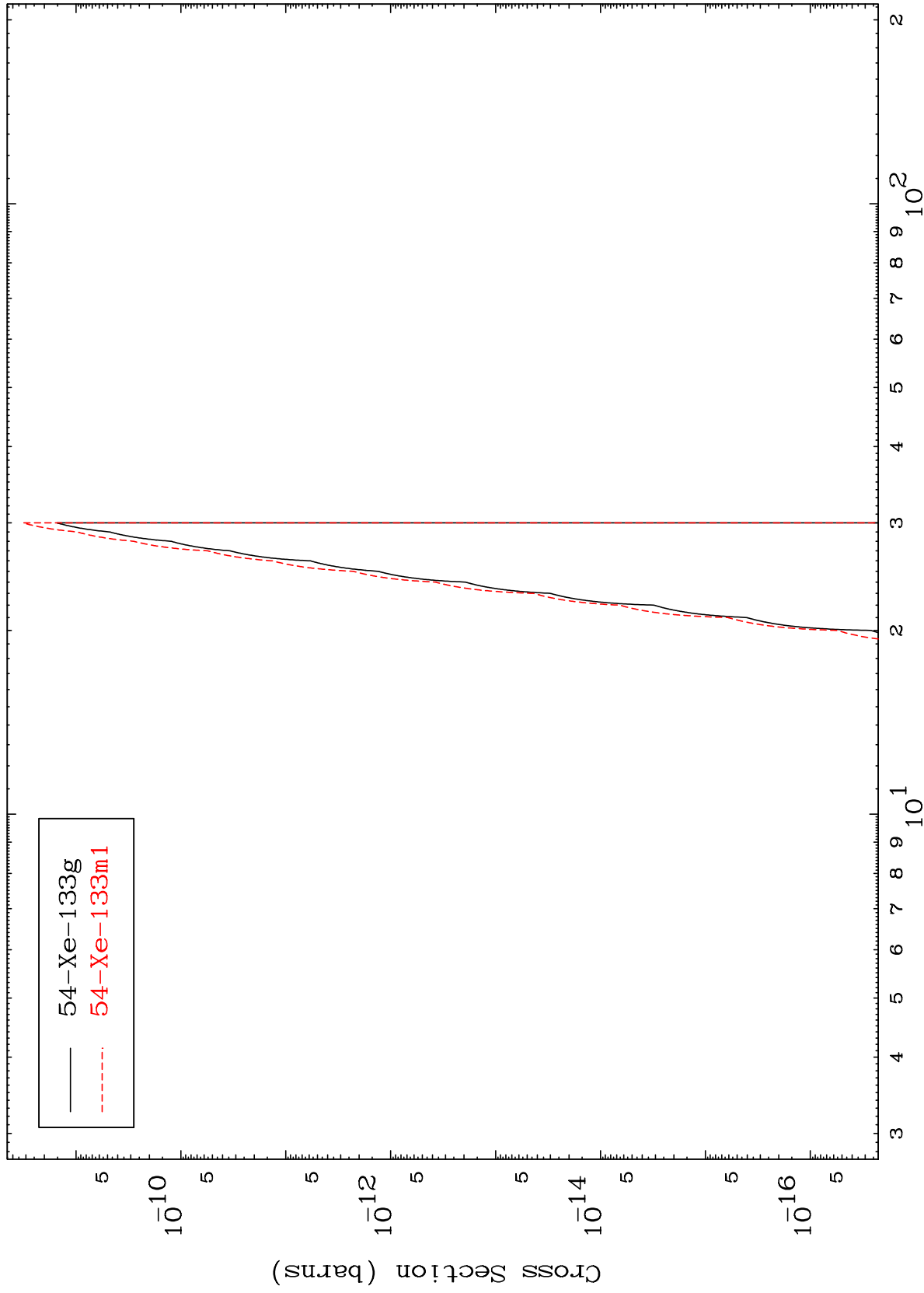
58-Ce-141

MAT 5840

(n,n') 2α

58-Ce-141

Radionuclide Production Cross Section



97

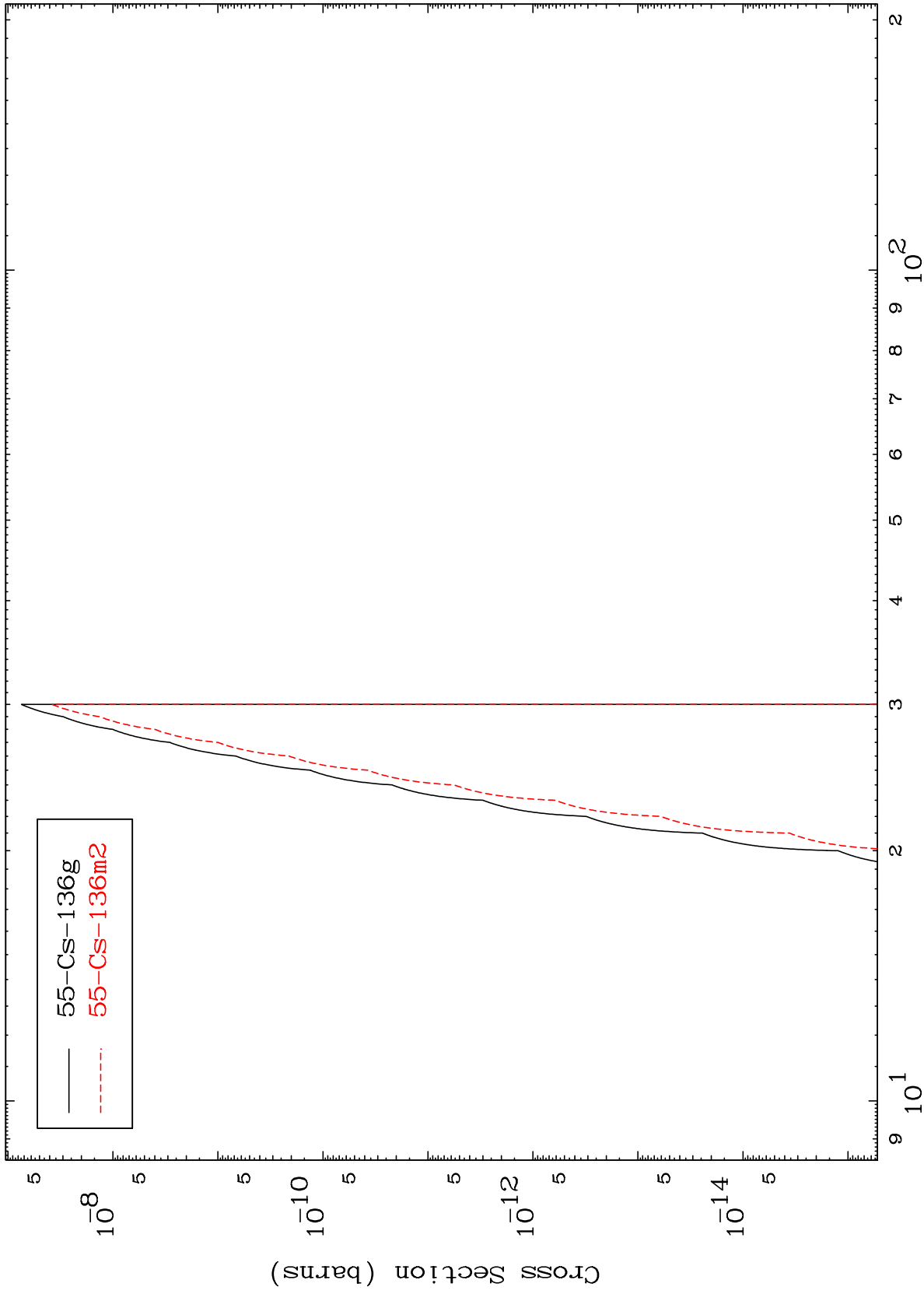
Incident Energy (MeV)

58-Ce-141

MAT 5840

58-Ce-141

(n,n') p α
Radionuclide Production Cross Section



98

58-Ce-141

MAT 5840

Radionuclide Production Cross Section
(n,2α)

58-Ce-141

