

Program EVALPLOT
(Version 2021-1)

by

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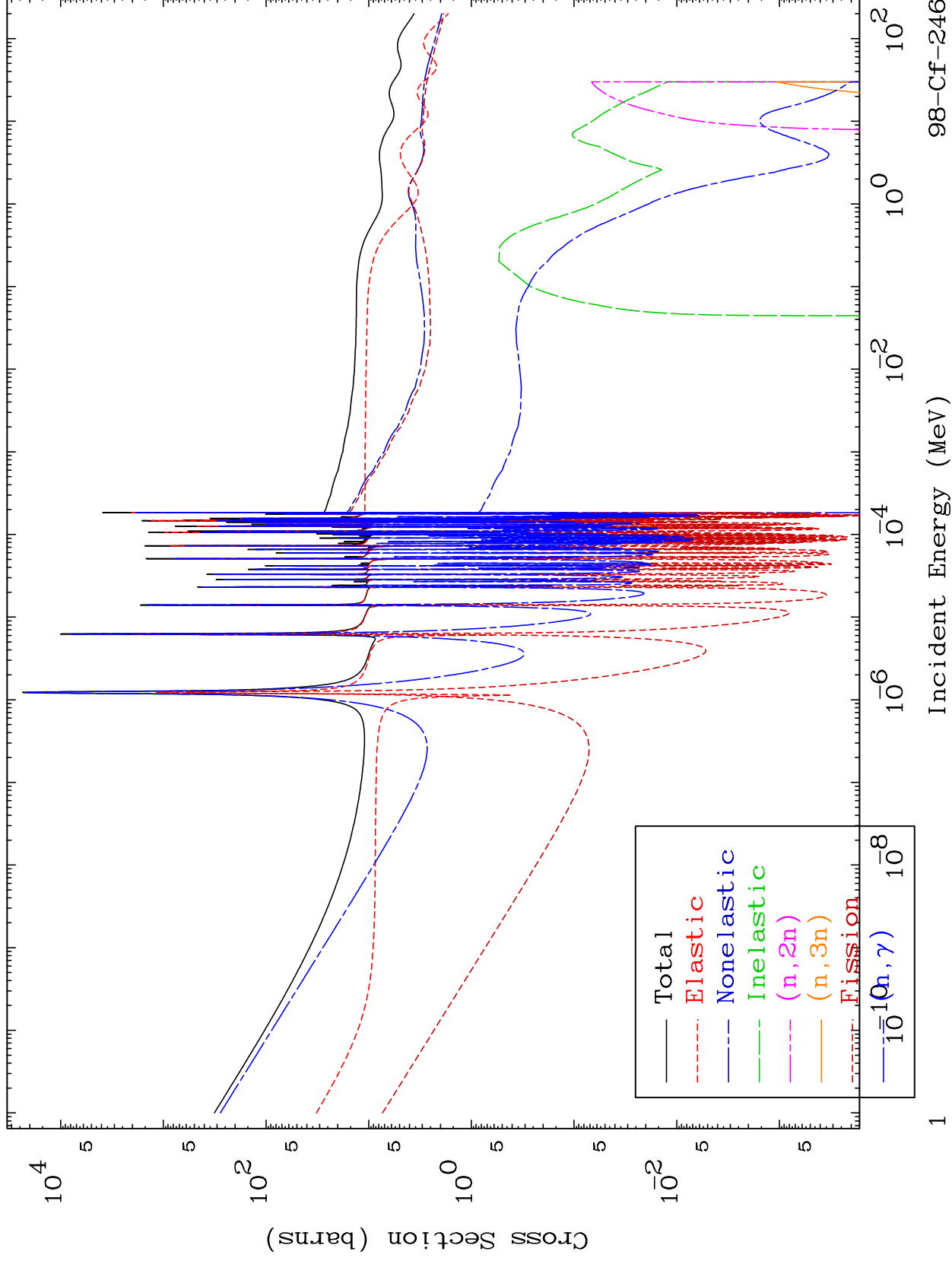
Press Mouse Button to Start

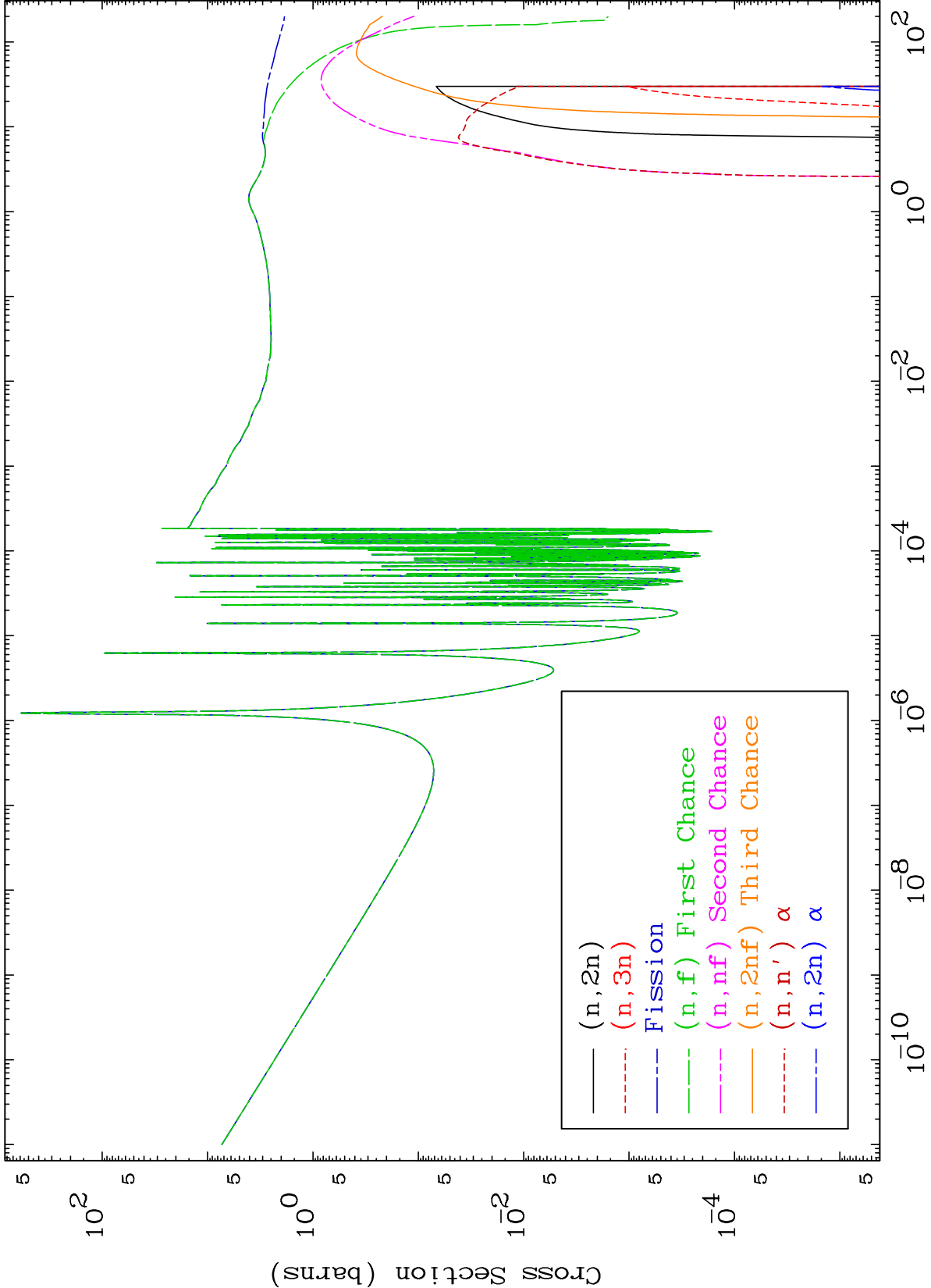
MAT 9843

Neutron Major

98-Cf-246

293 Kelvin Cross Sections

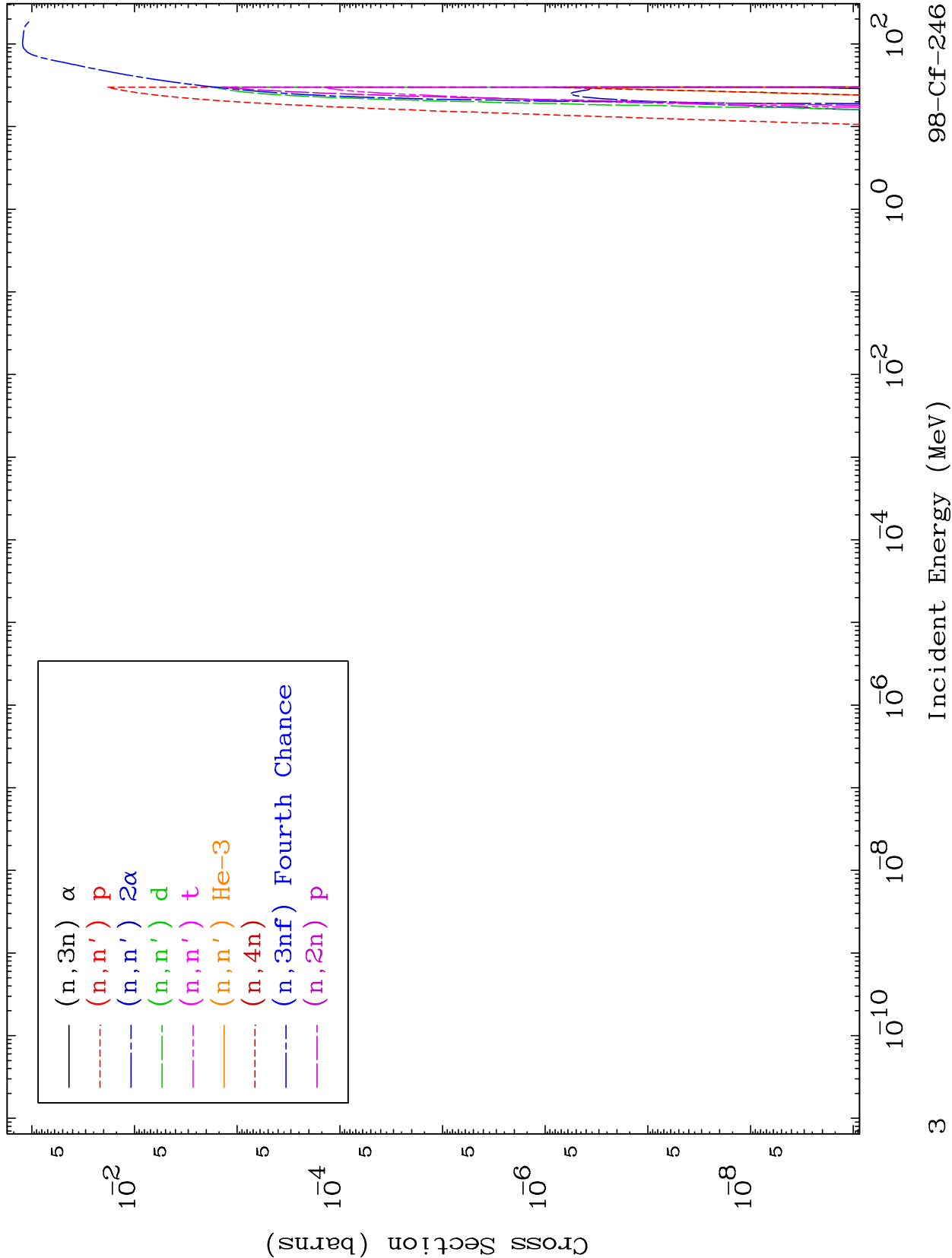


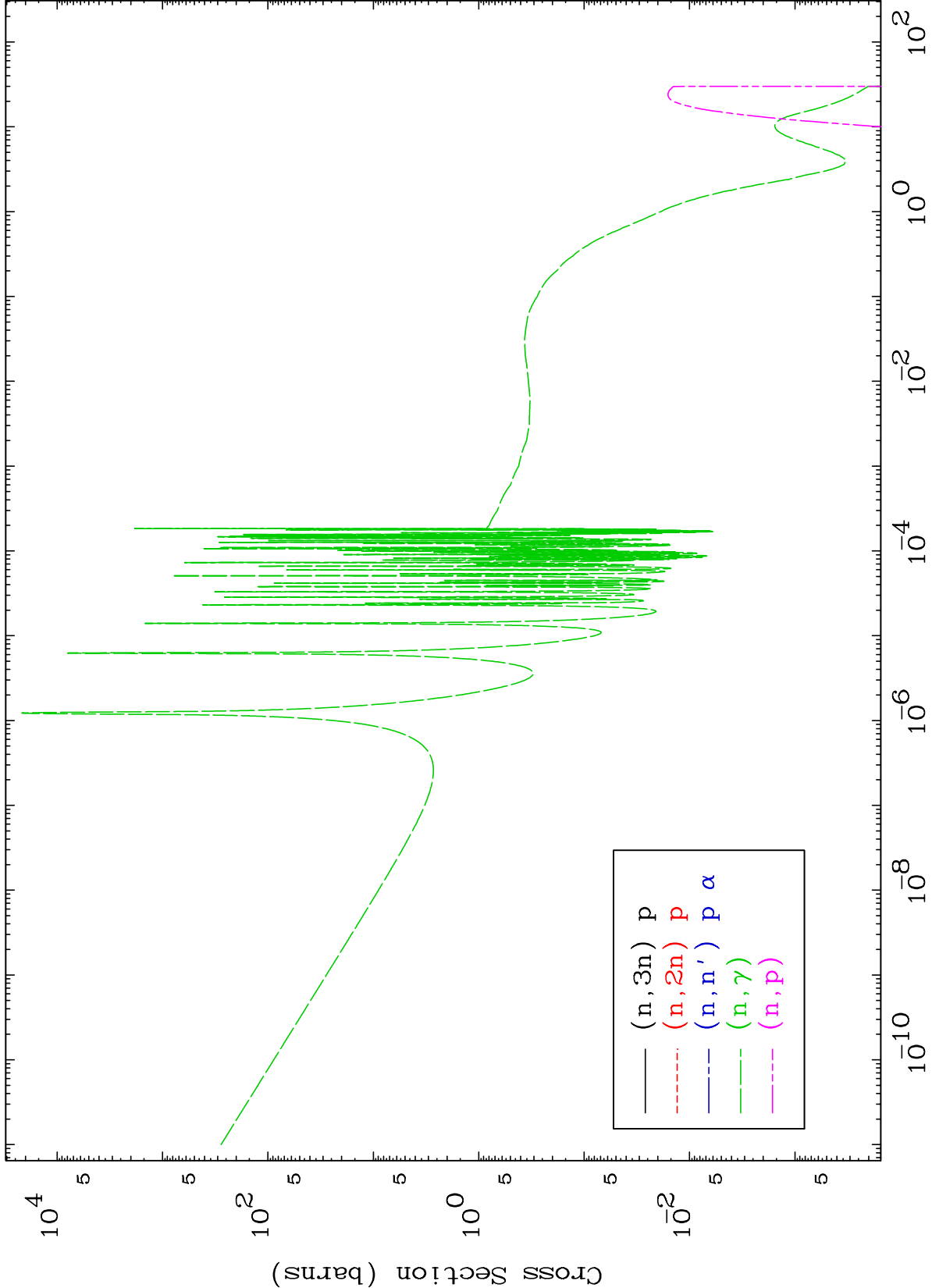


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Neutron Absorption
293 Kelvin Cross Sections

98-Cf-246

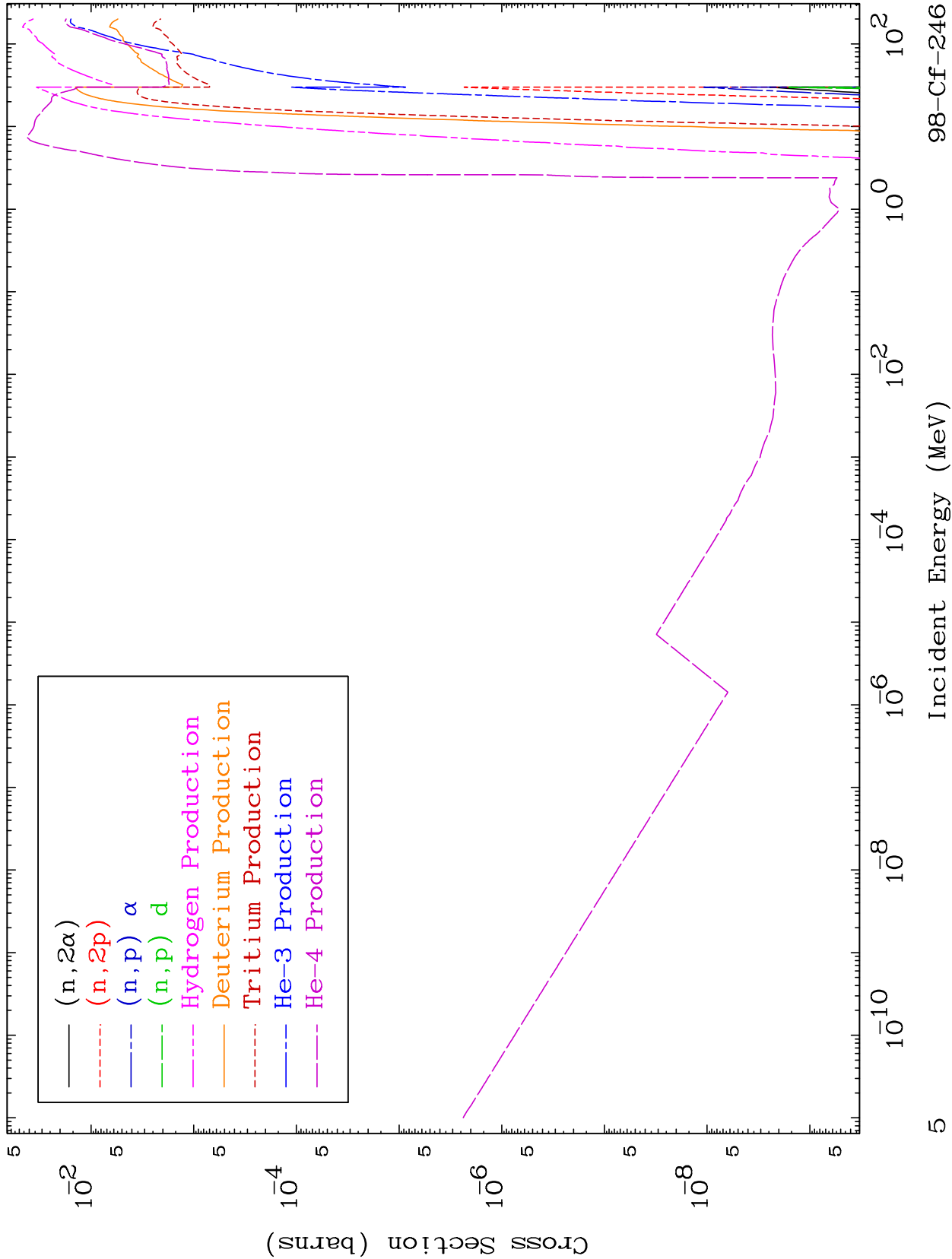


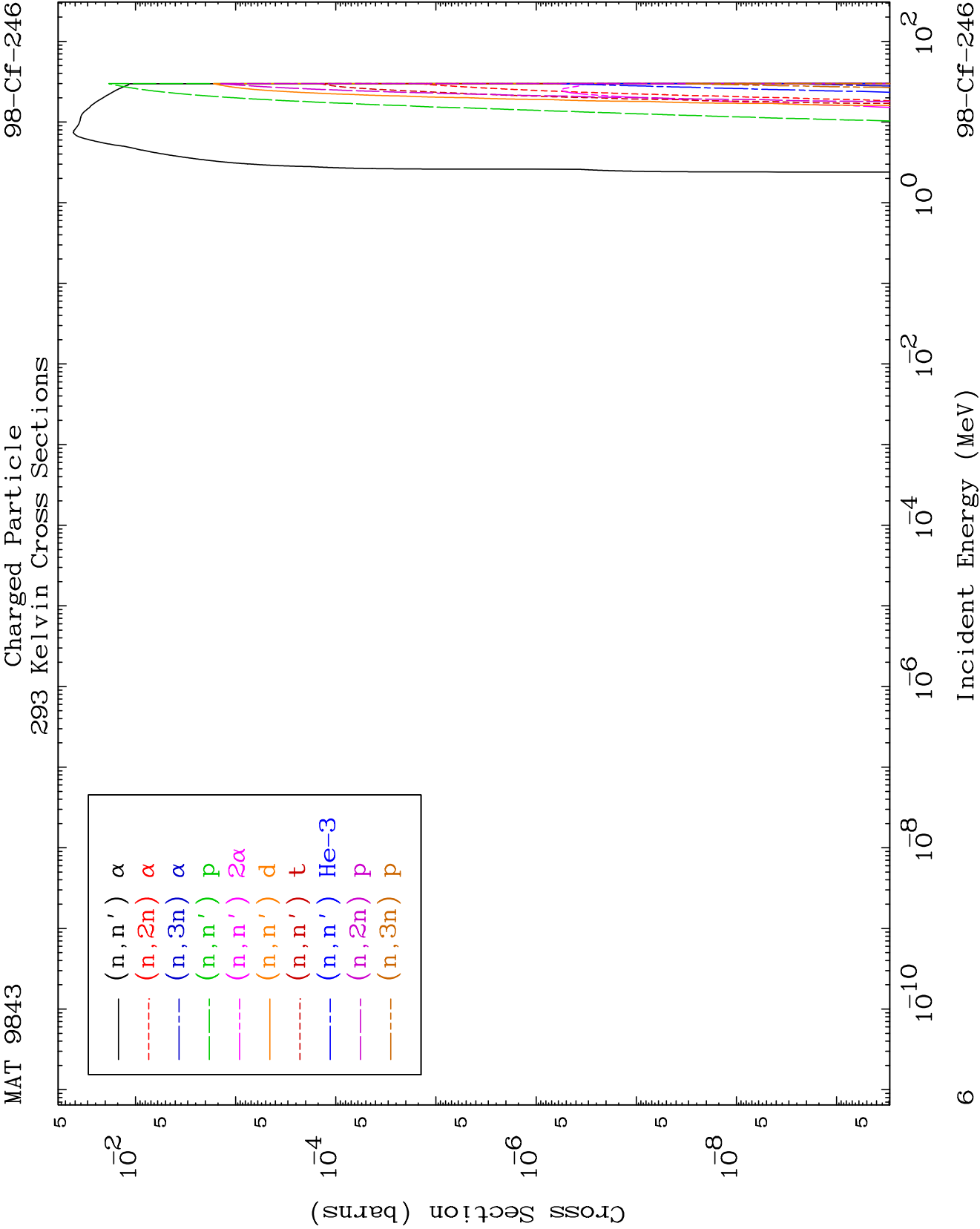


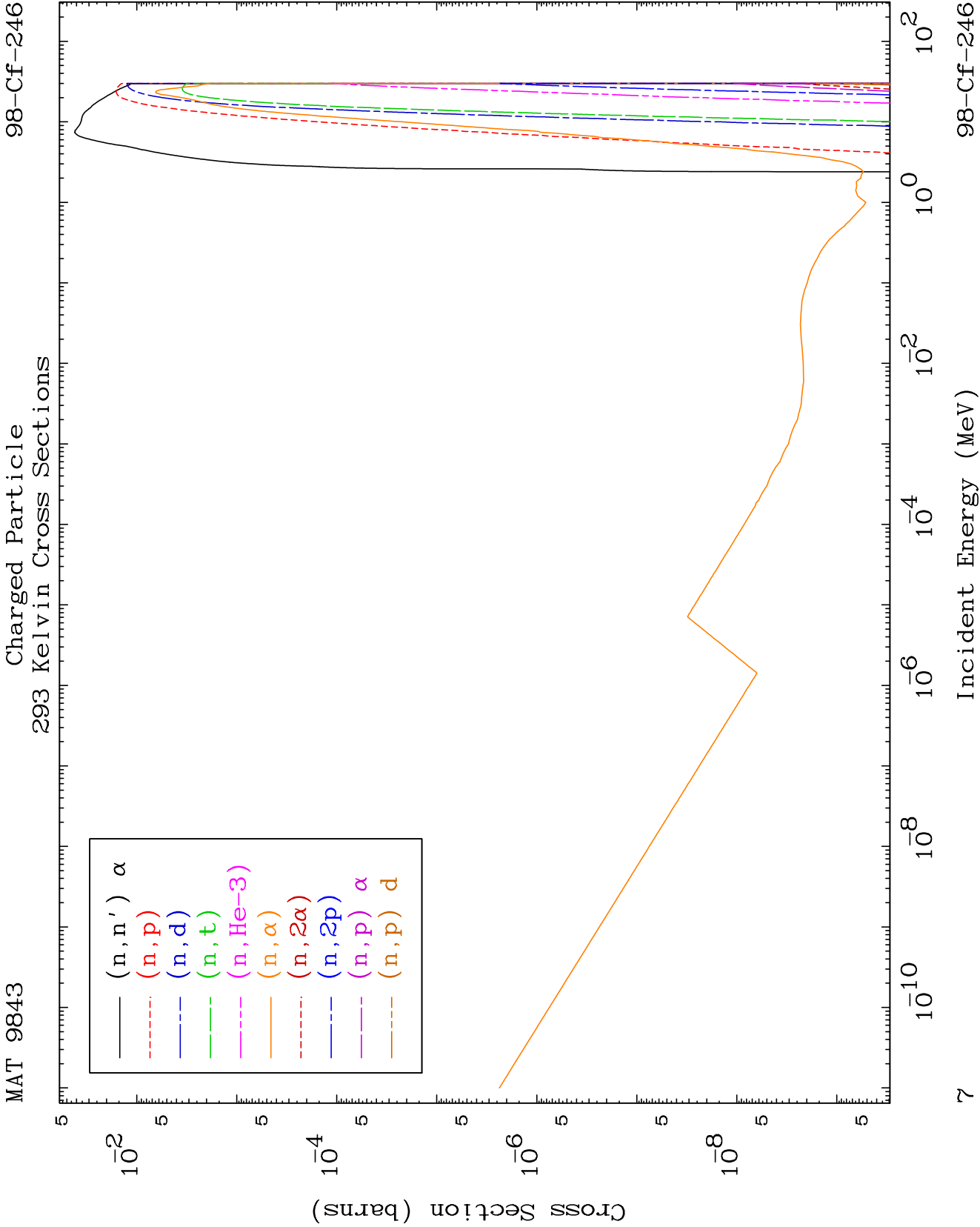
MAT 9843

Neutron Absorption
293 Kelvin Cross Sections

98-Cf-246



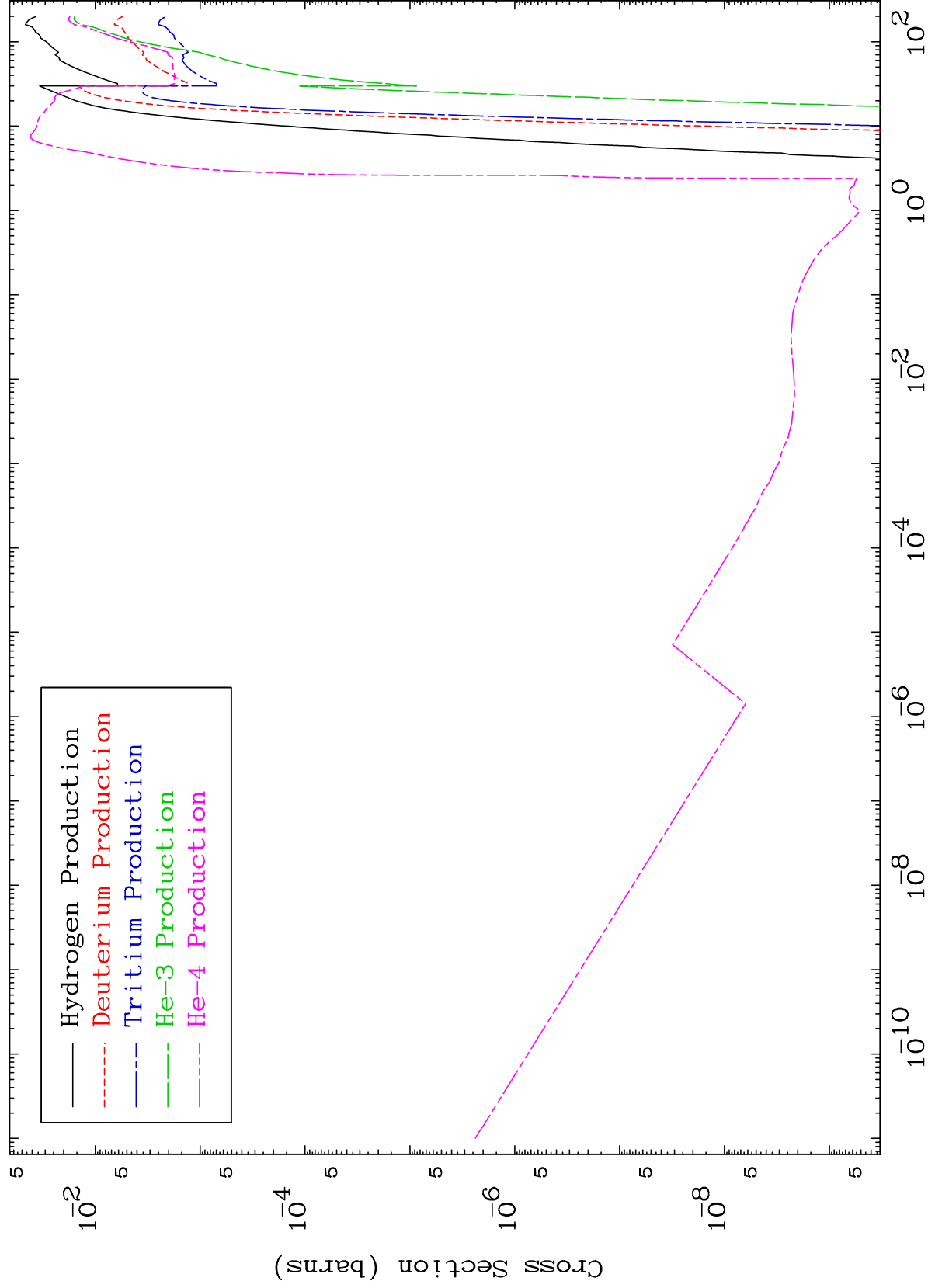


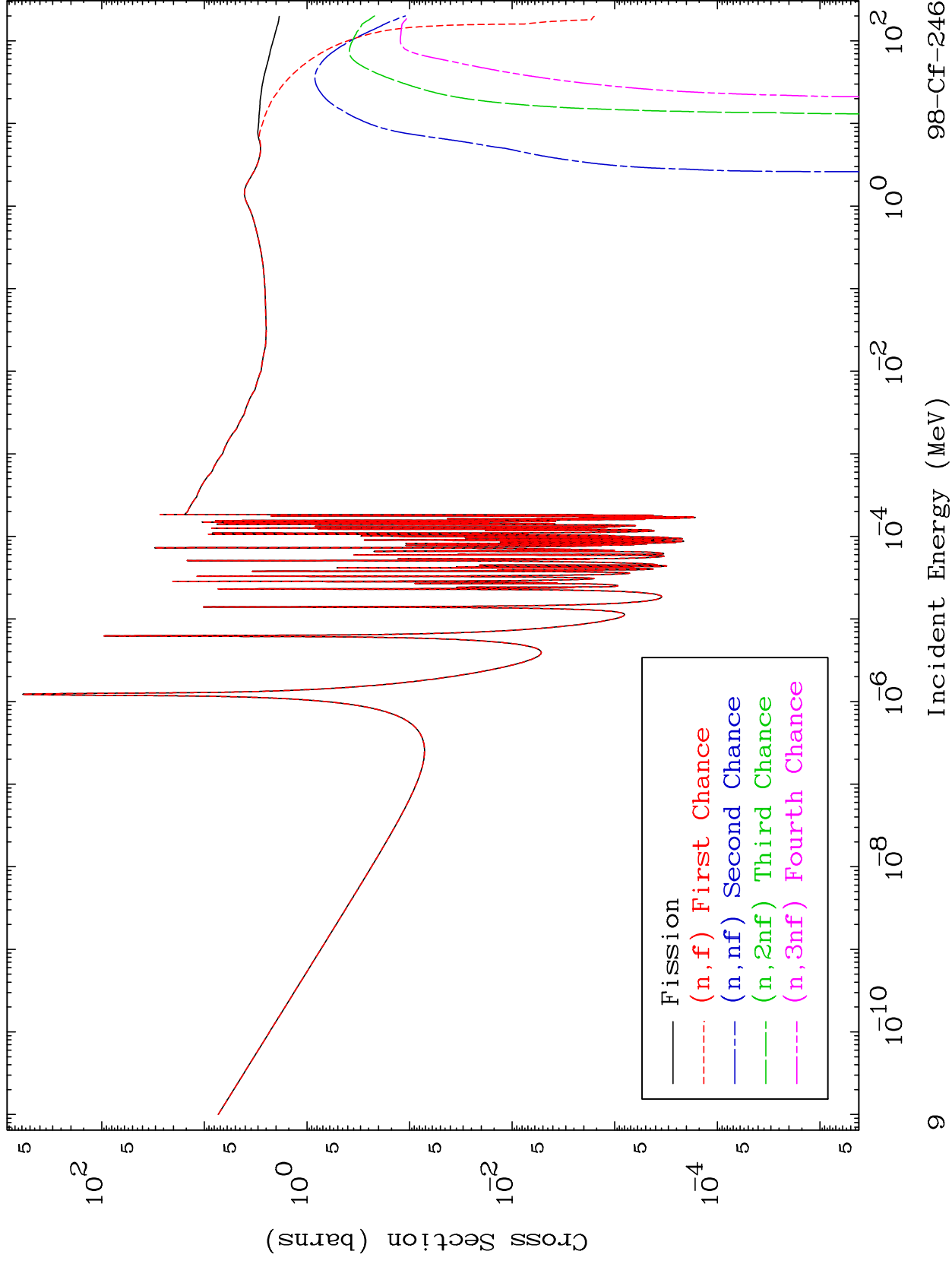


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Particle Production
293 Kelvin Cross Sections

98-Cf-246

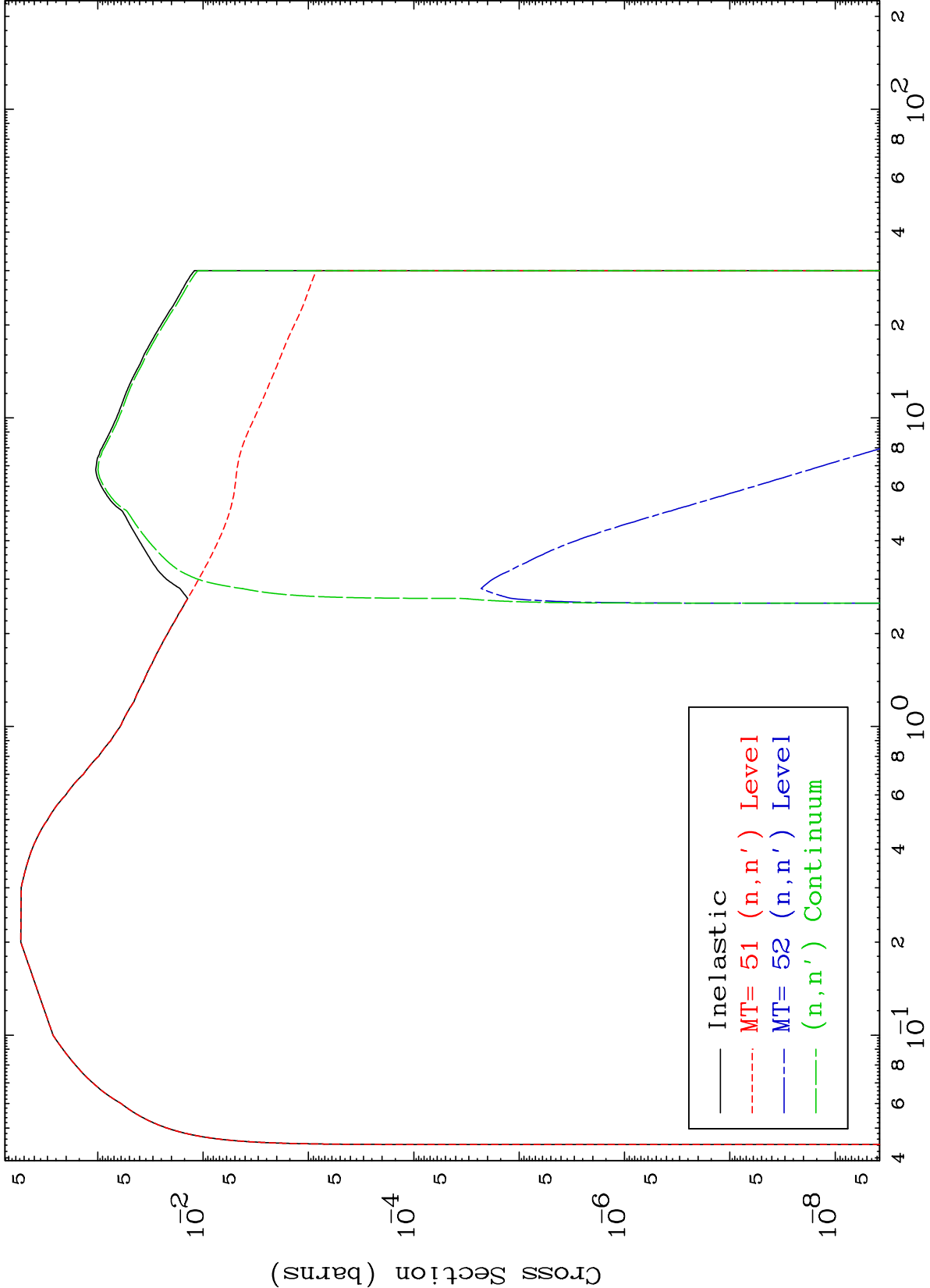




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(n,n') Levels
293 Kelvin Cross Sections

98-Cf-246



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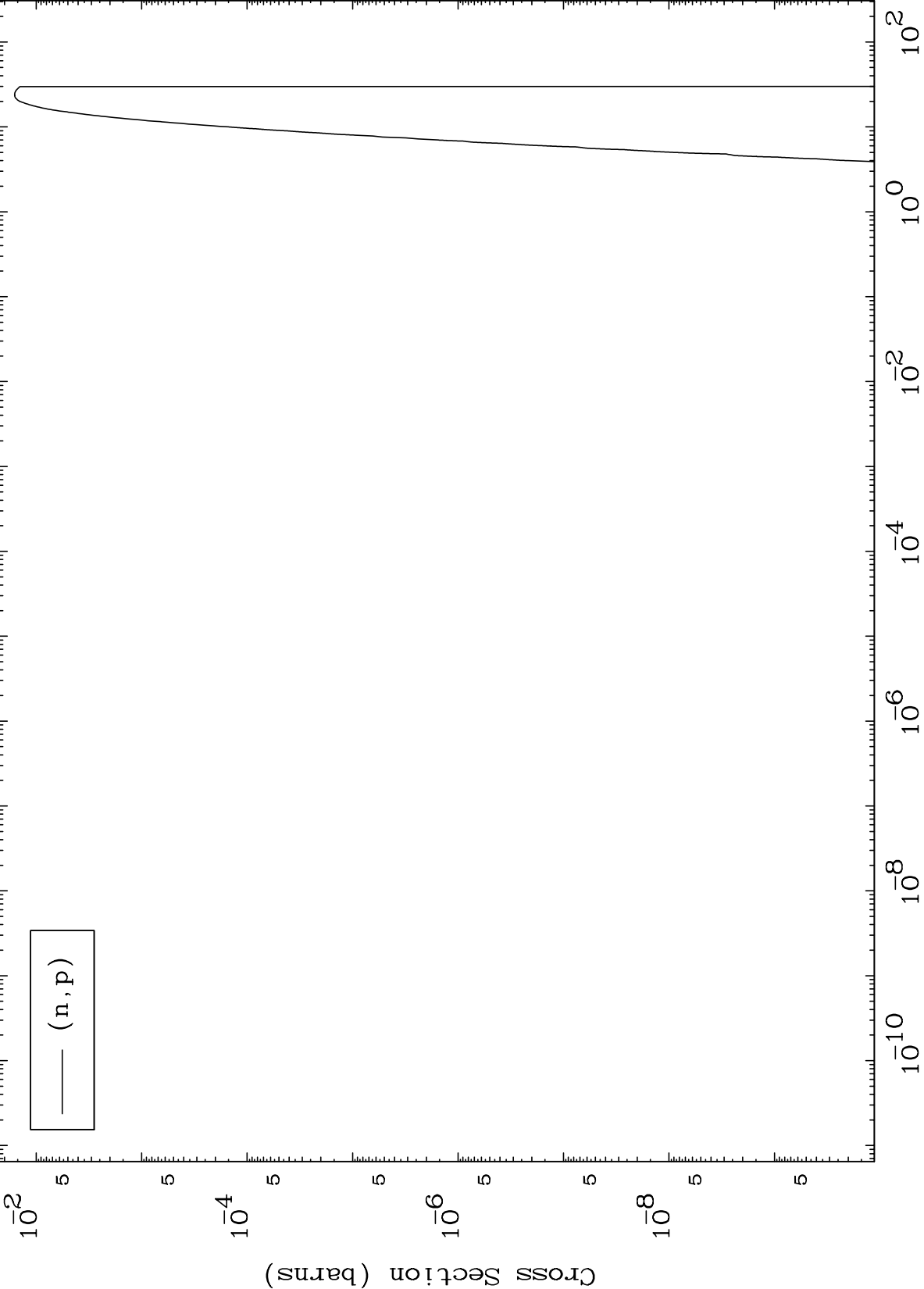
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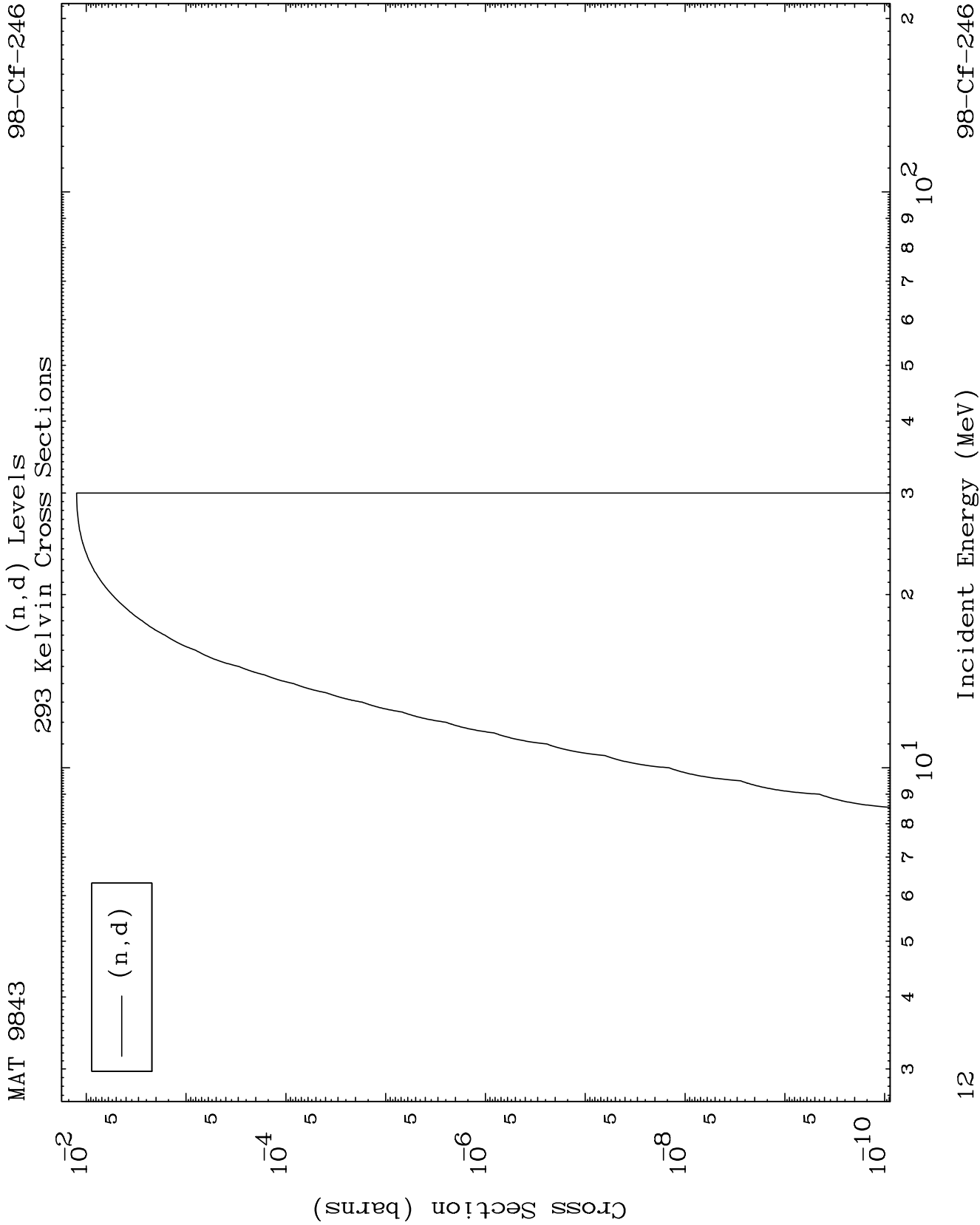
98-Cf-246

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(n,p) Levels
293 Kelvin Cross Sections

98-Cf-246

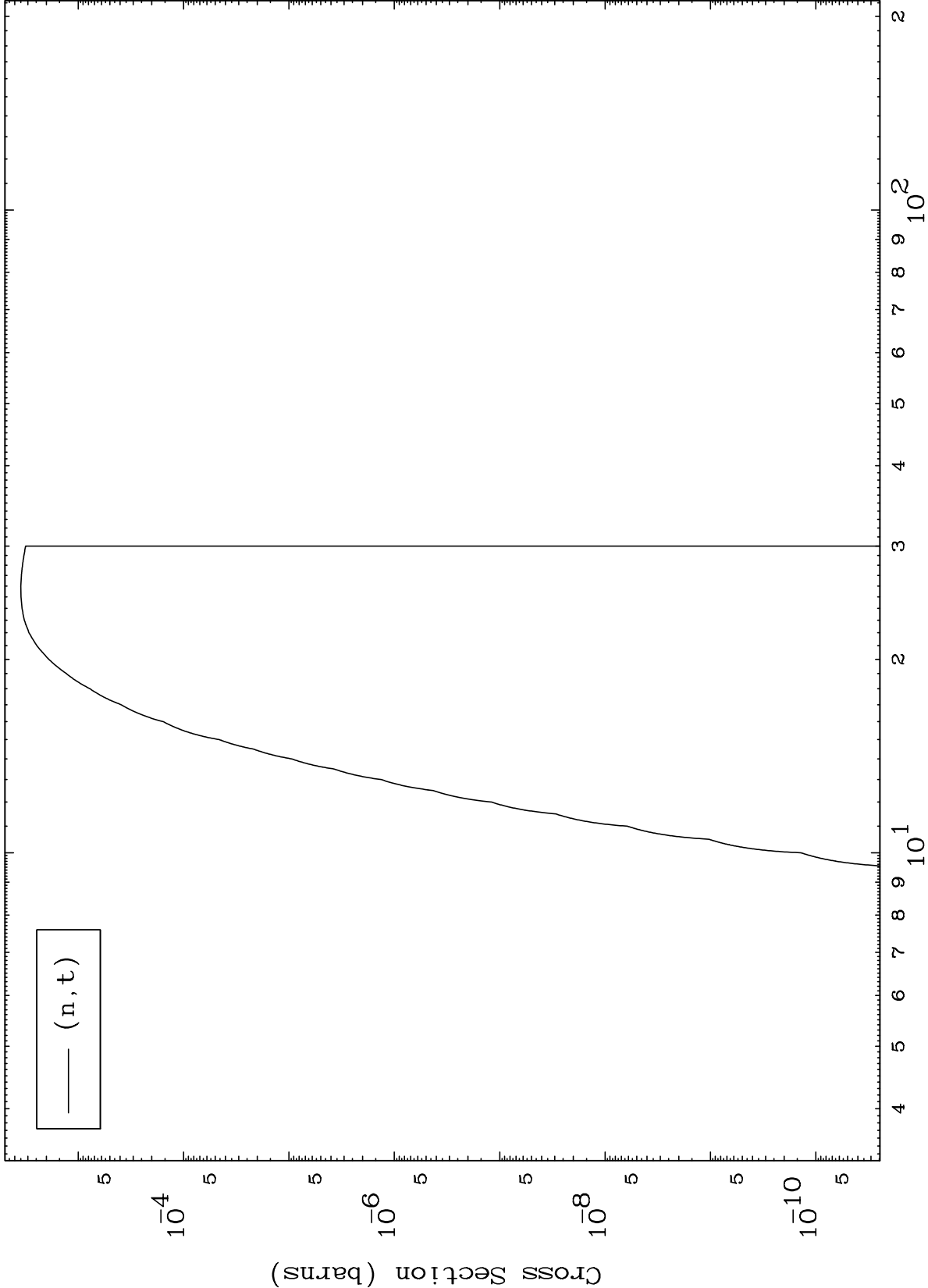




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(n,t) Levels
293 Kelvin Cross Sections

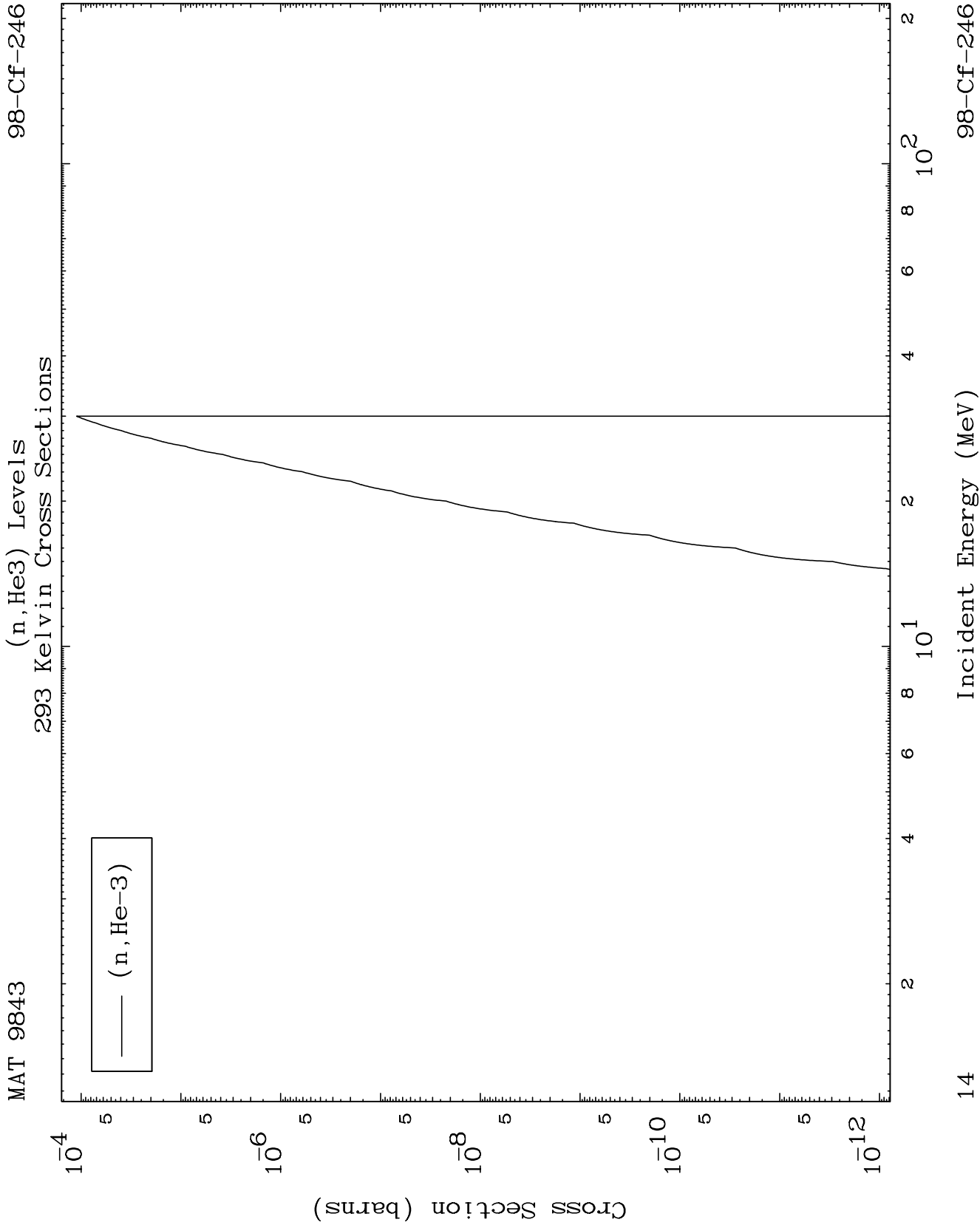
98-Cf-246



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Incident Energy (MeV)

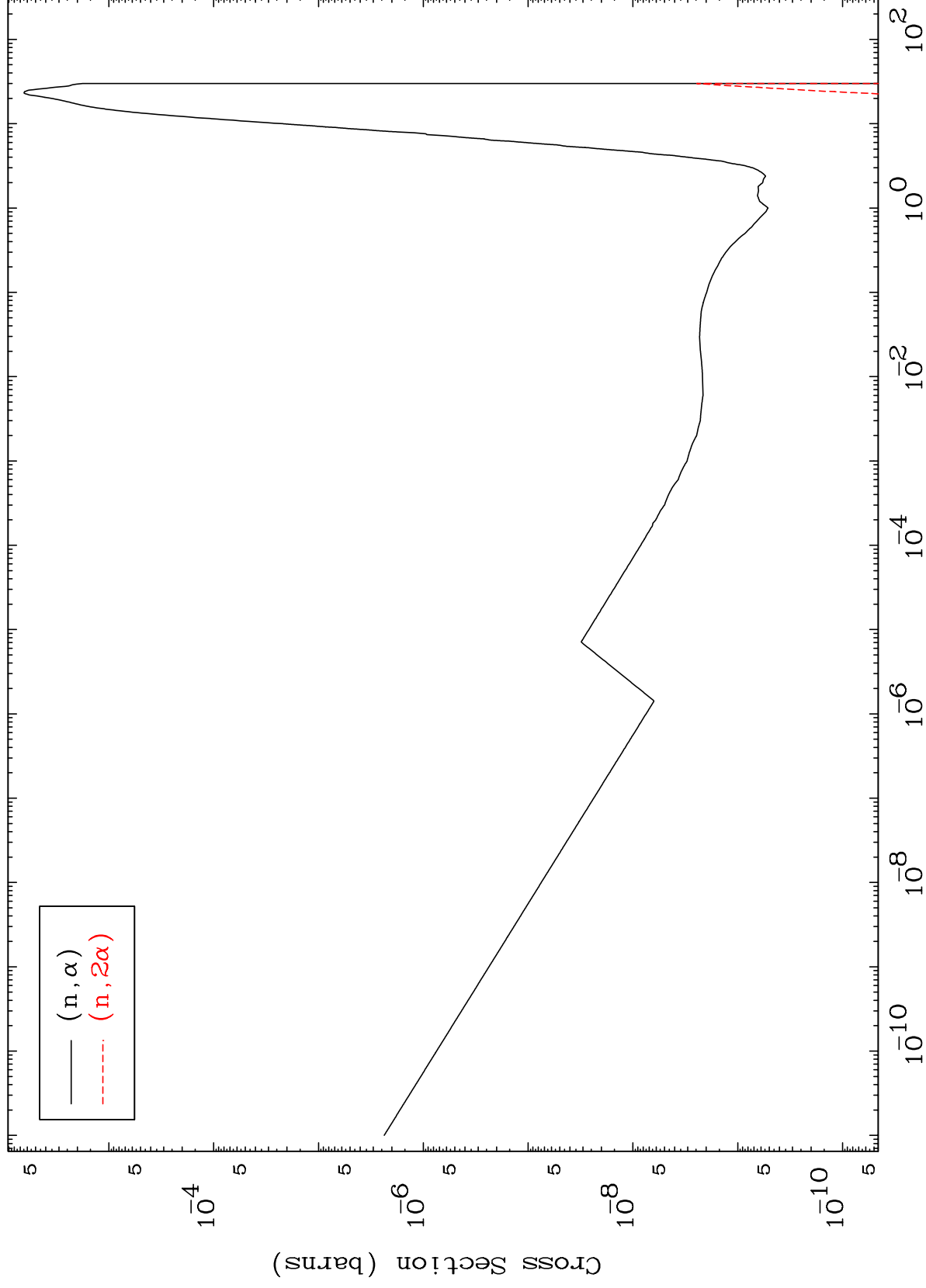
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MAT 9843

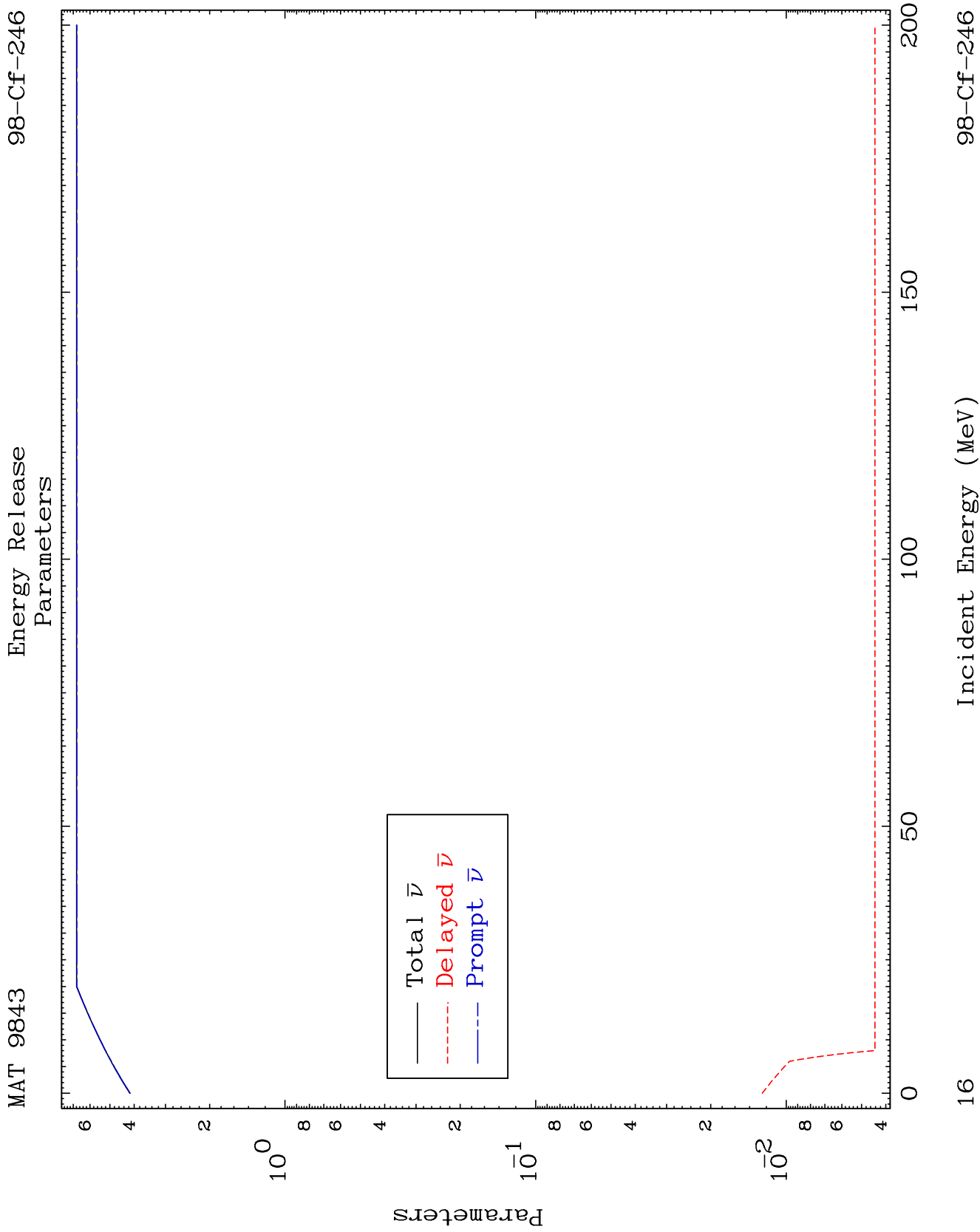
(n, α) Levels
293 Kelvin Cross Sections

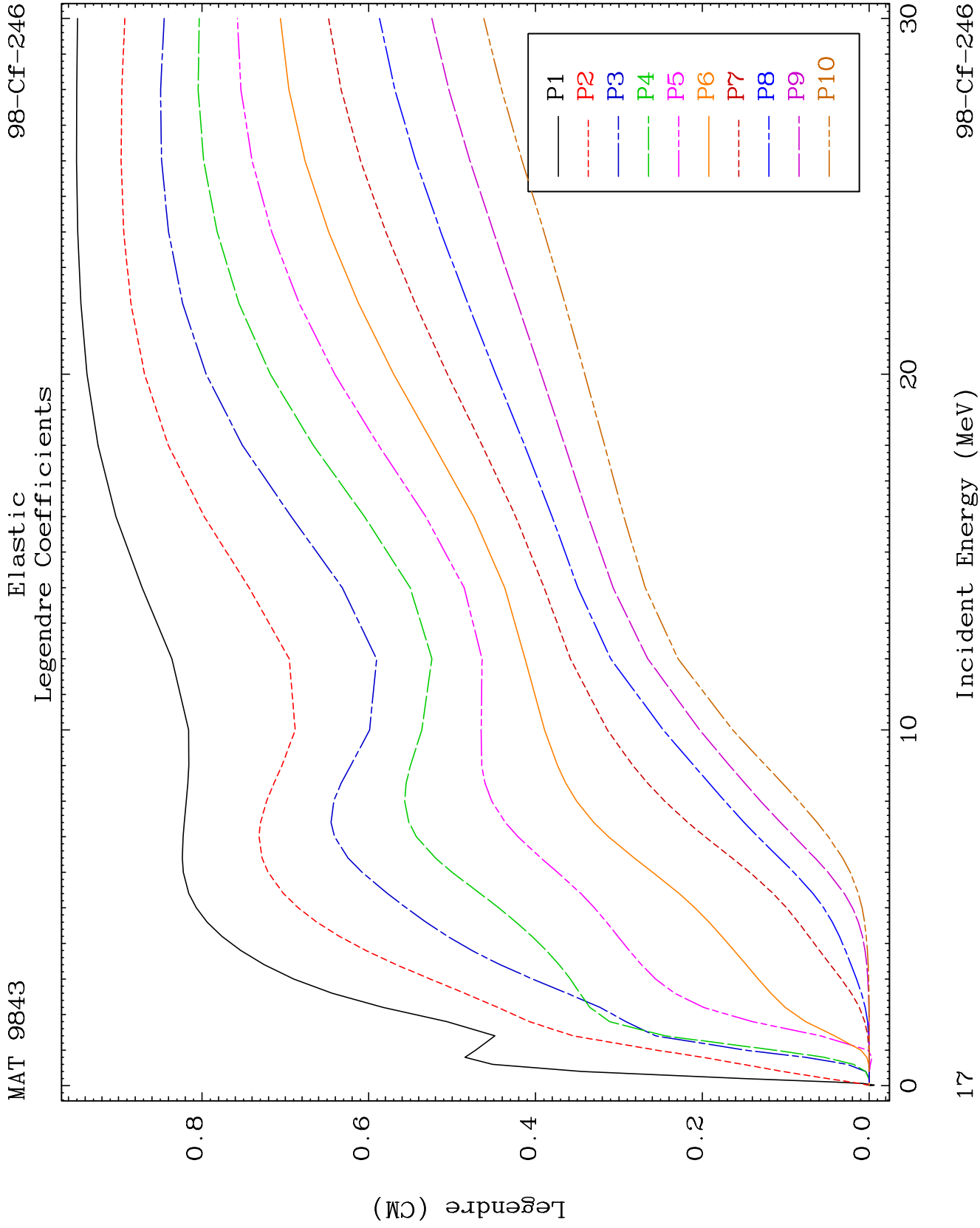
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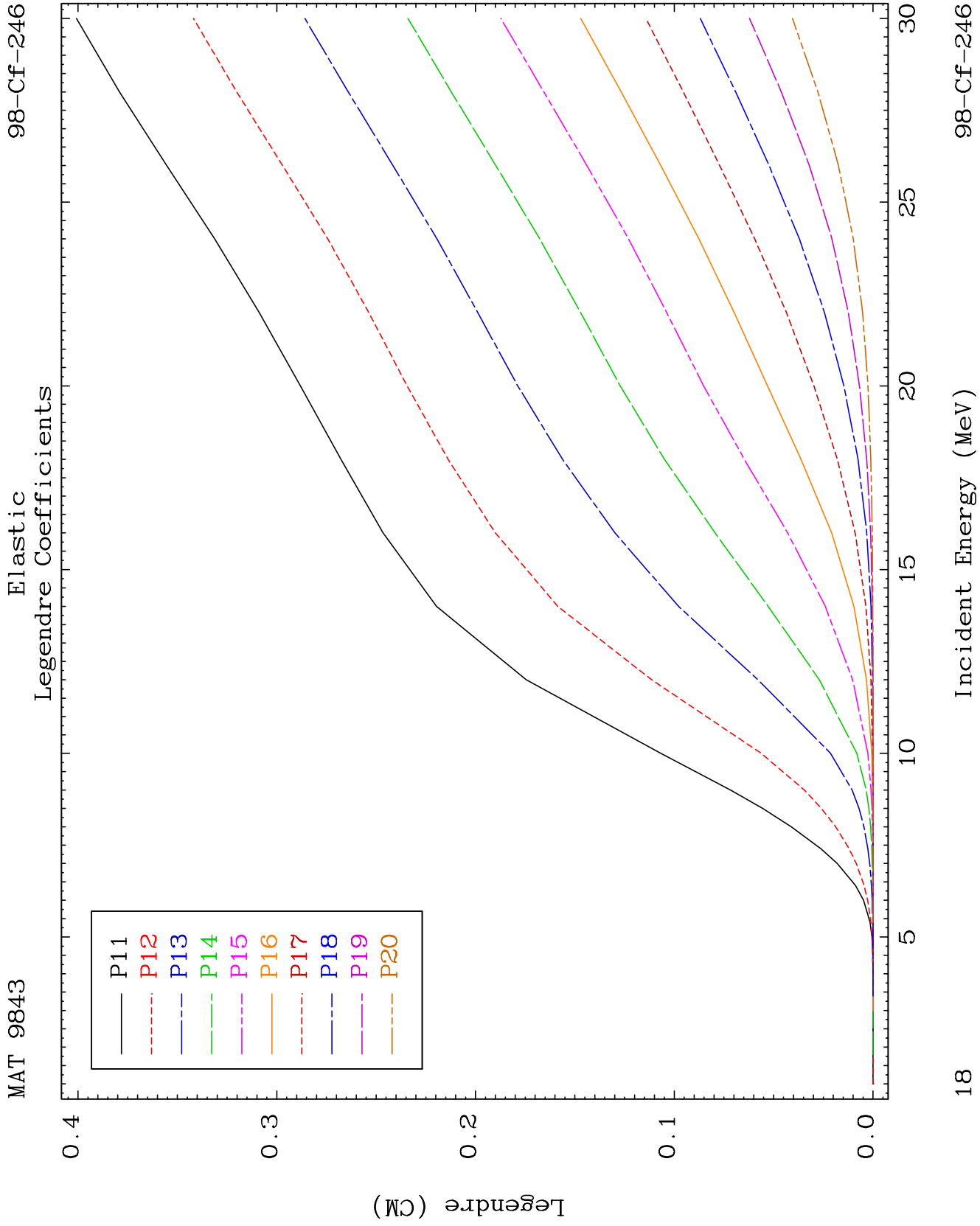


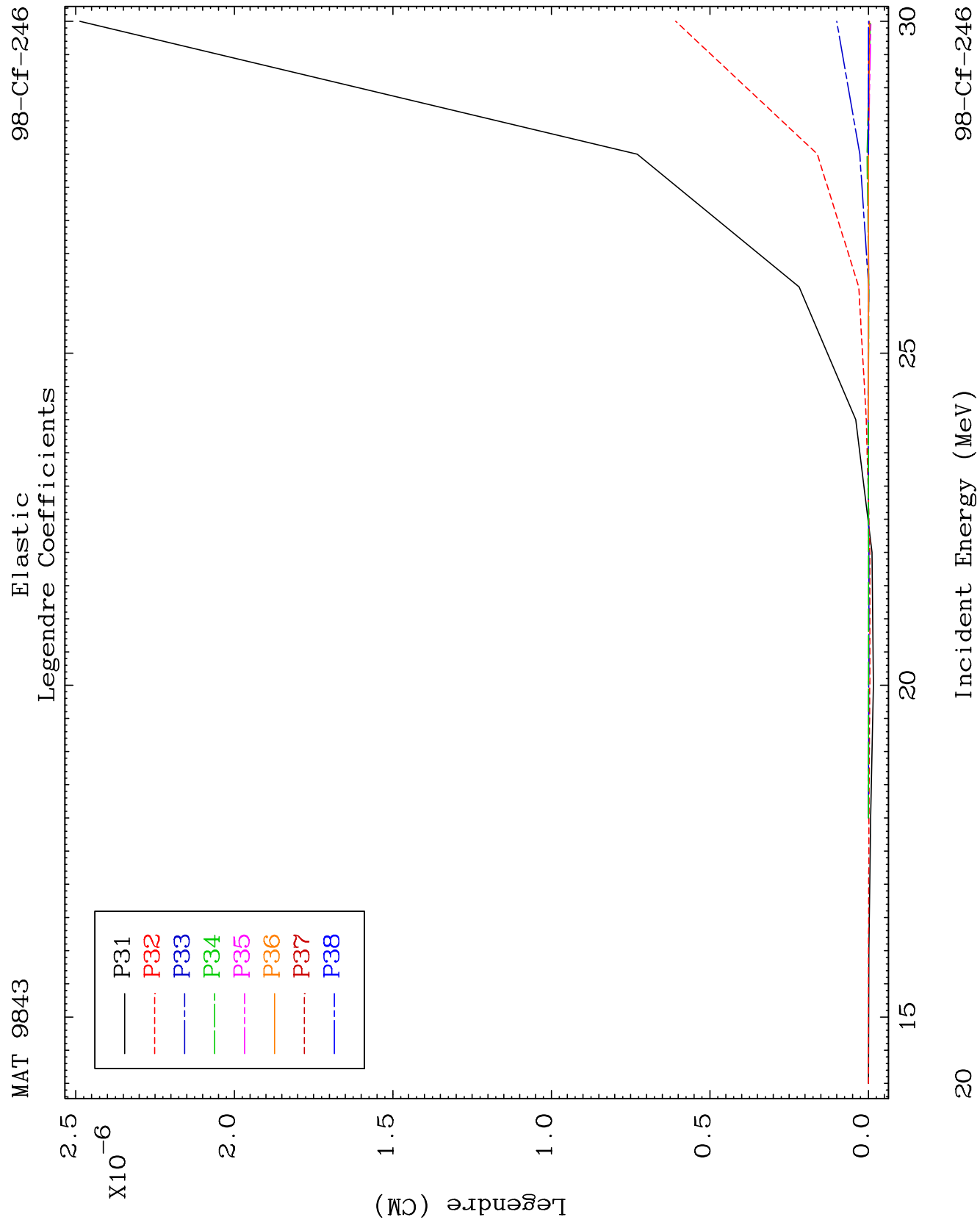
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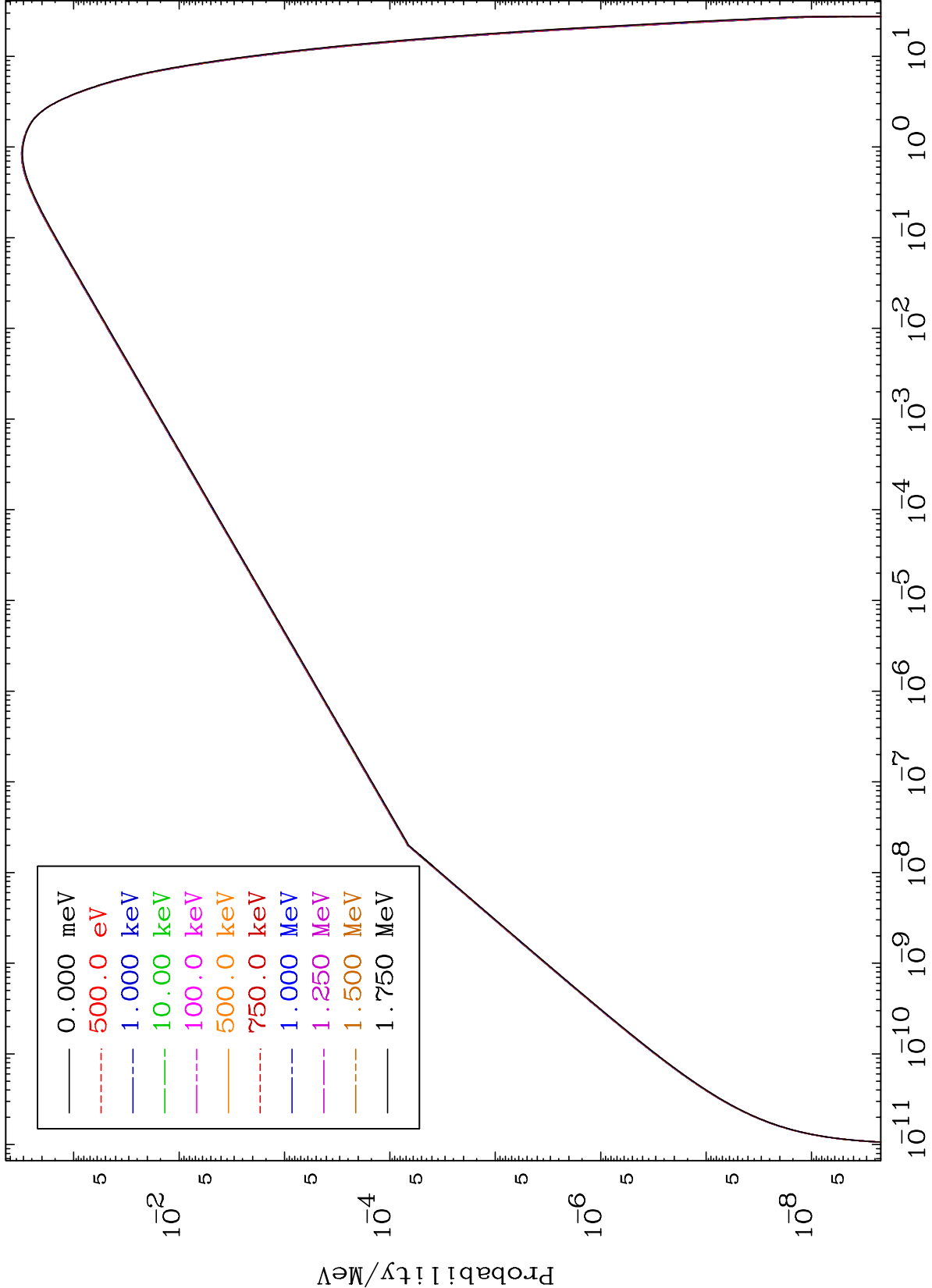
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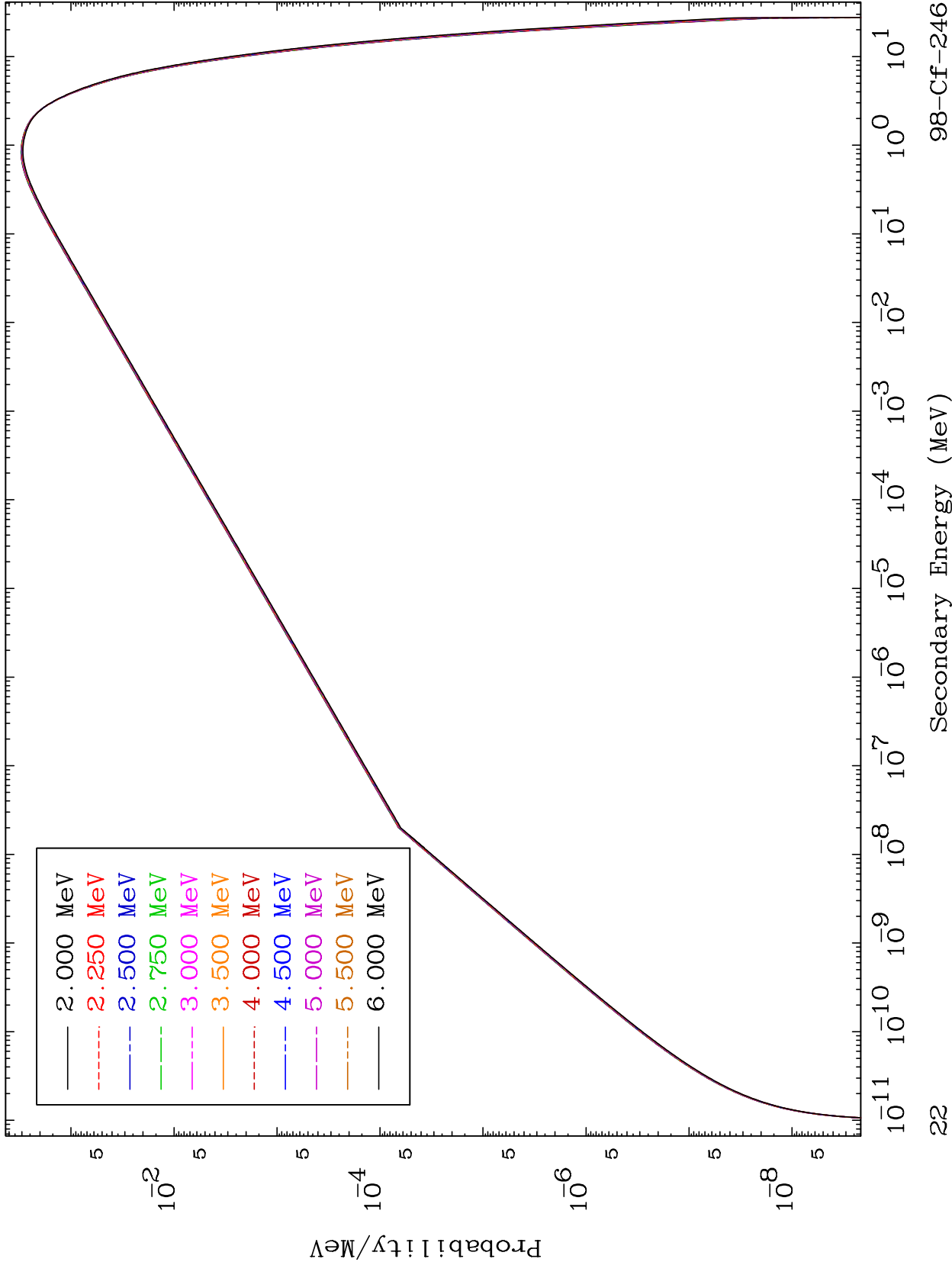


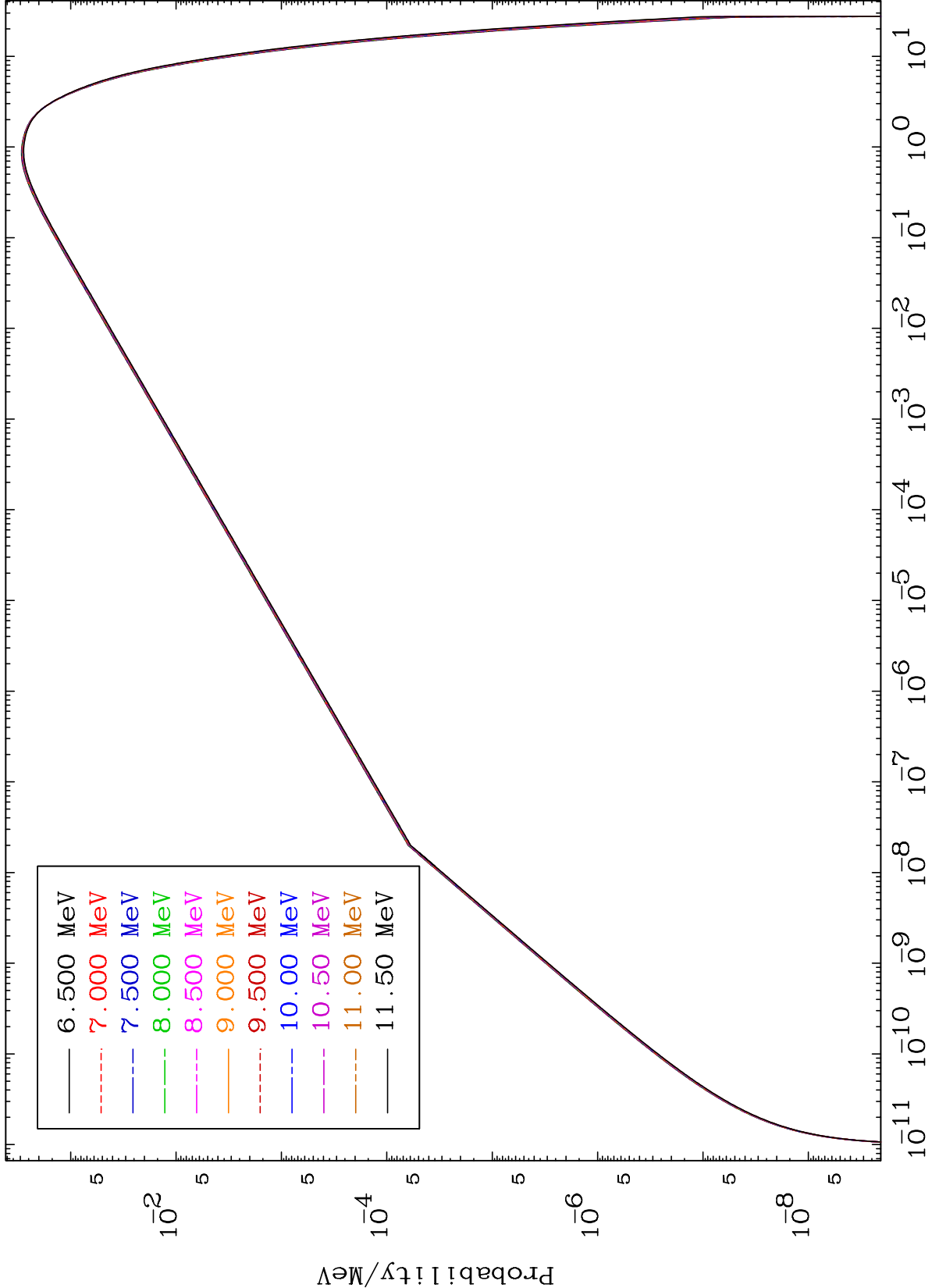


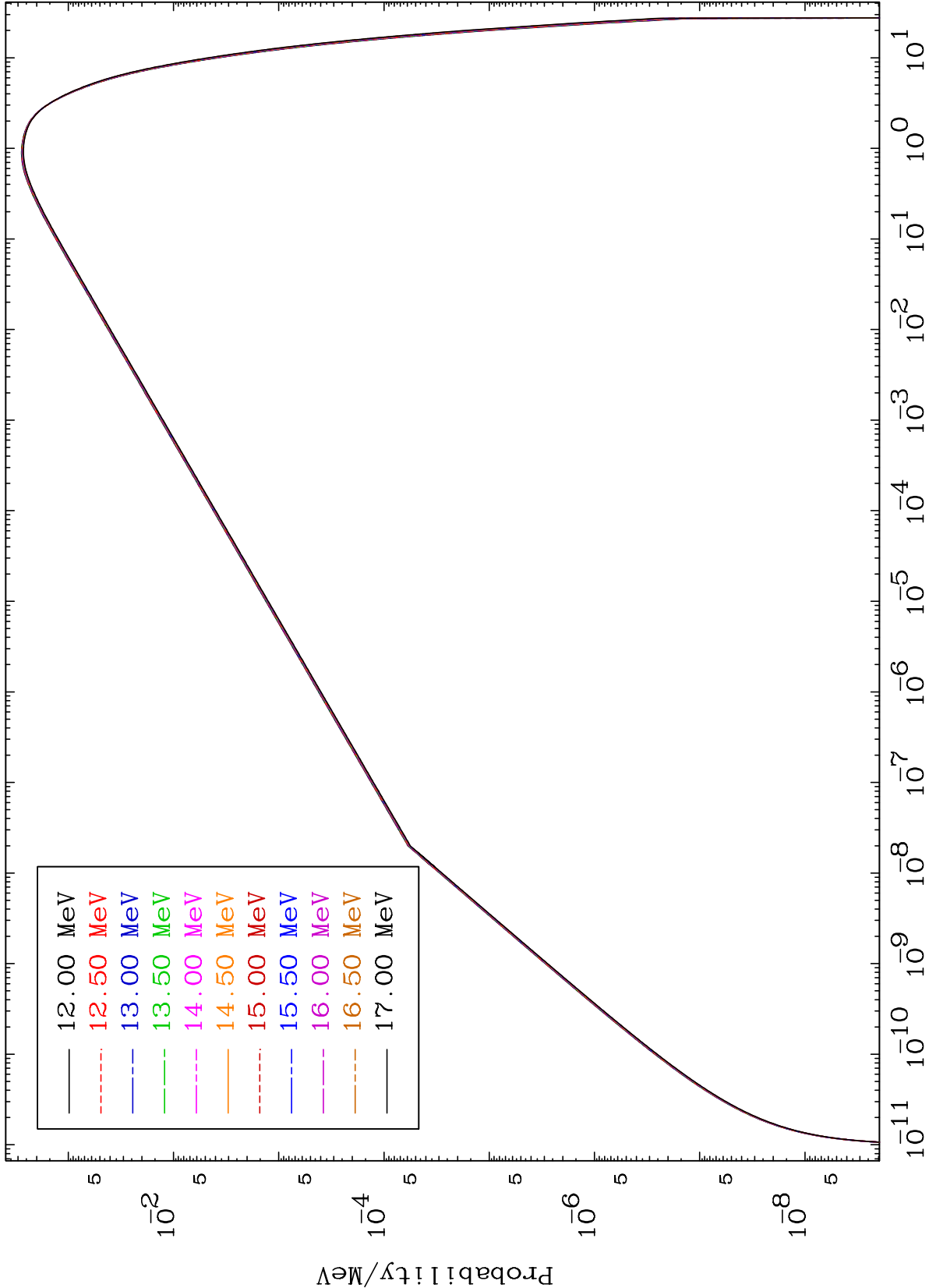
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Fission
Energy Distributions

98-Cf-246







MAT 9843

Fission
Energy Distributions

98-Cf-246

