

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

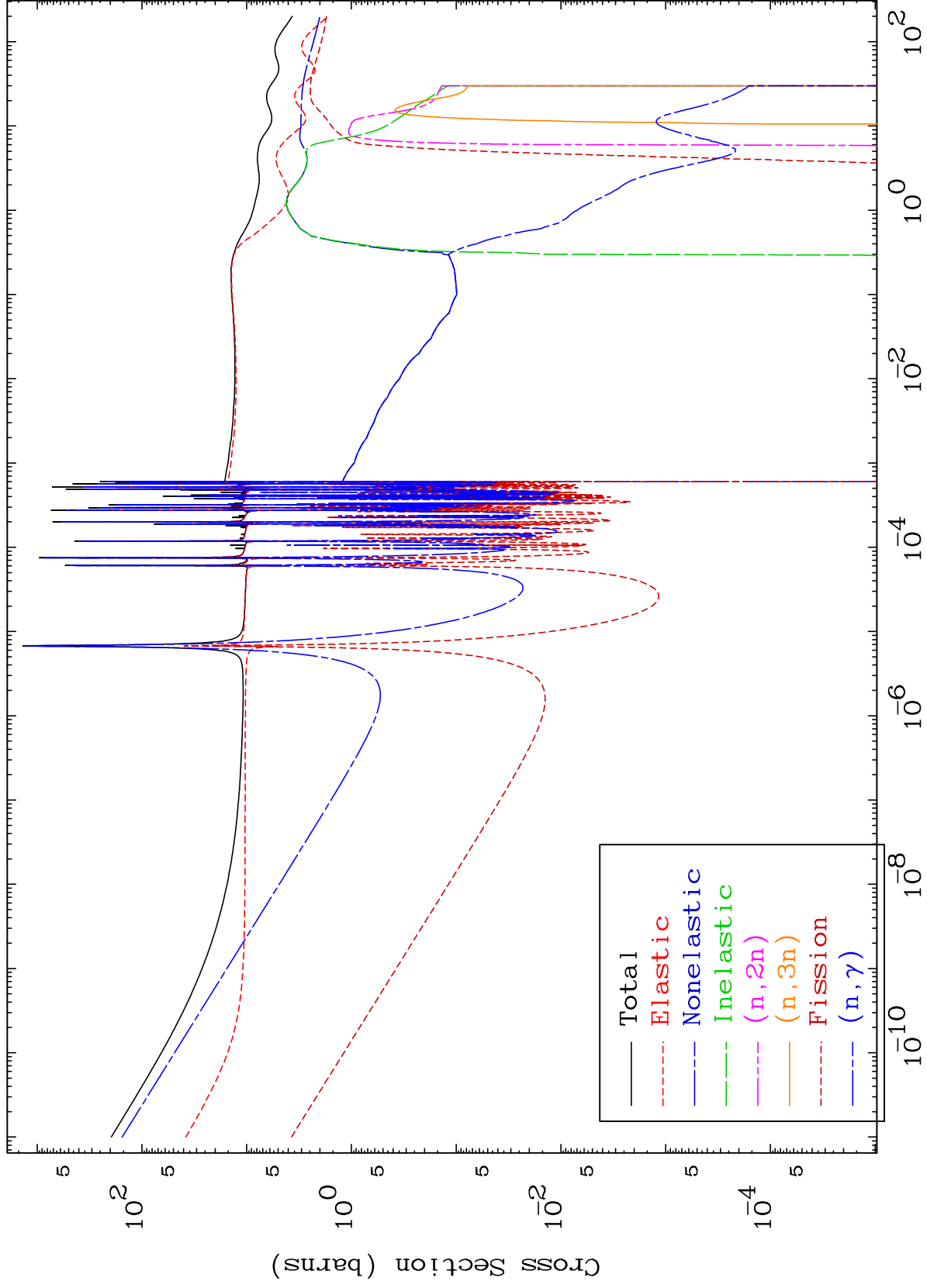
Press Mouse Button to Start

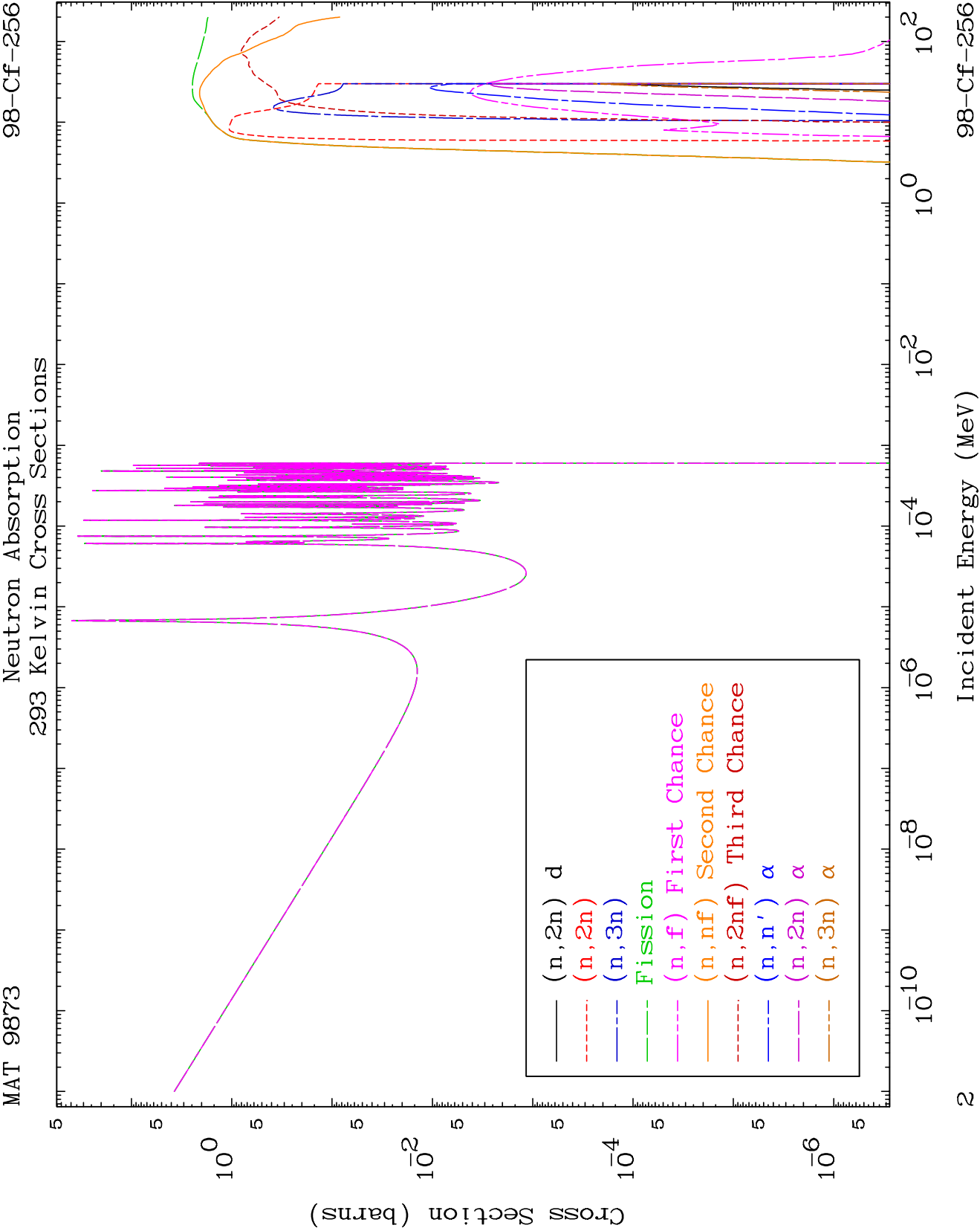
MAT 9873

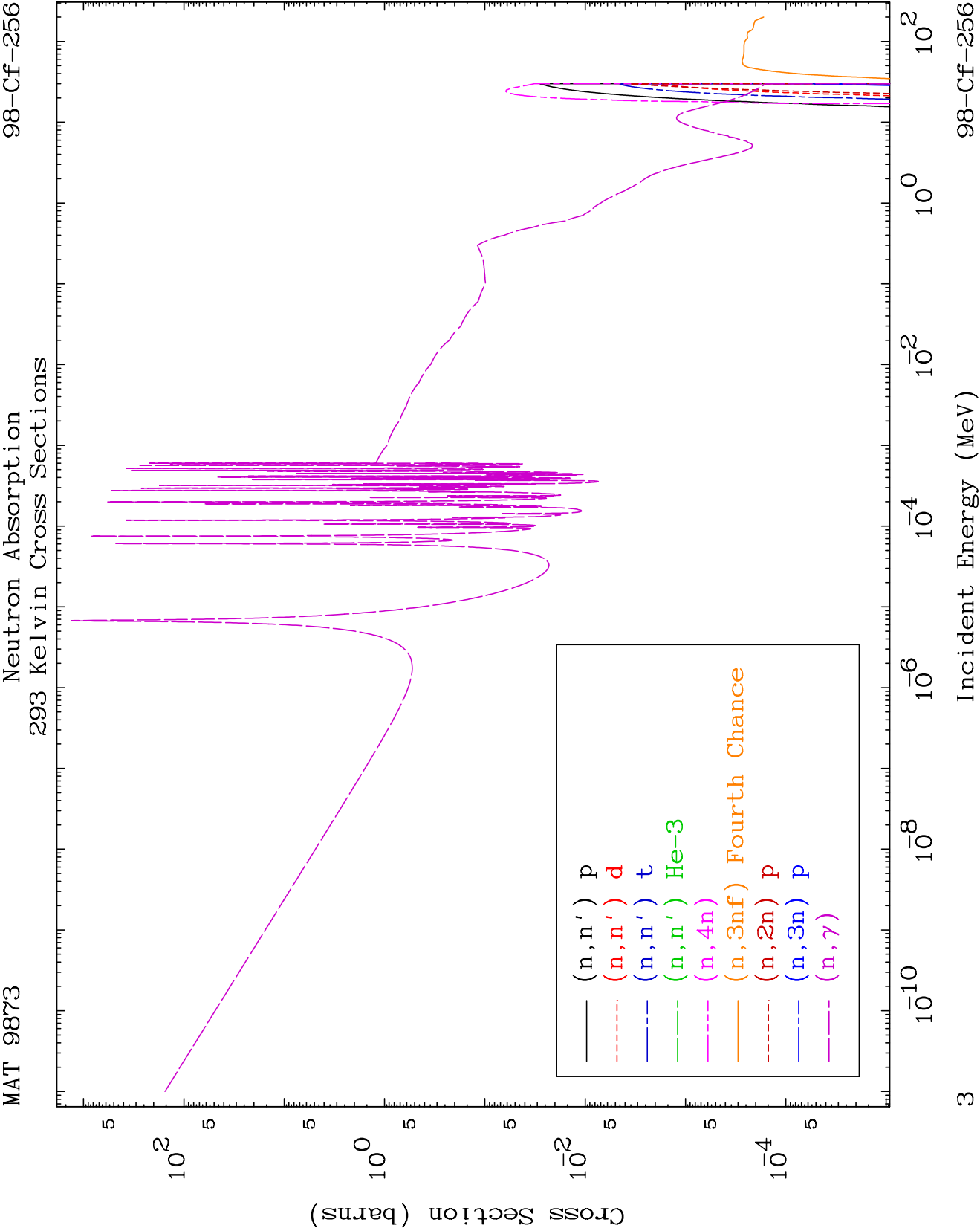
Neutron Major

293 Kelvin Cross Sections

98-Cf-256



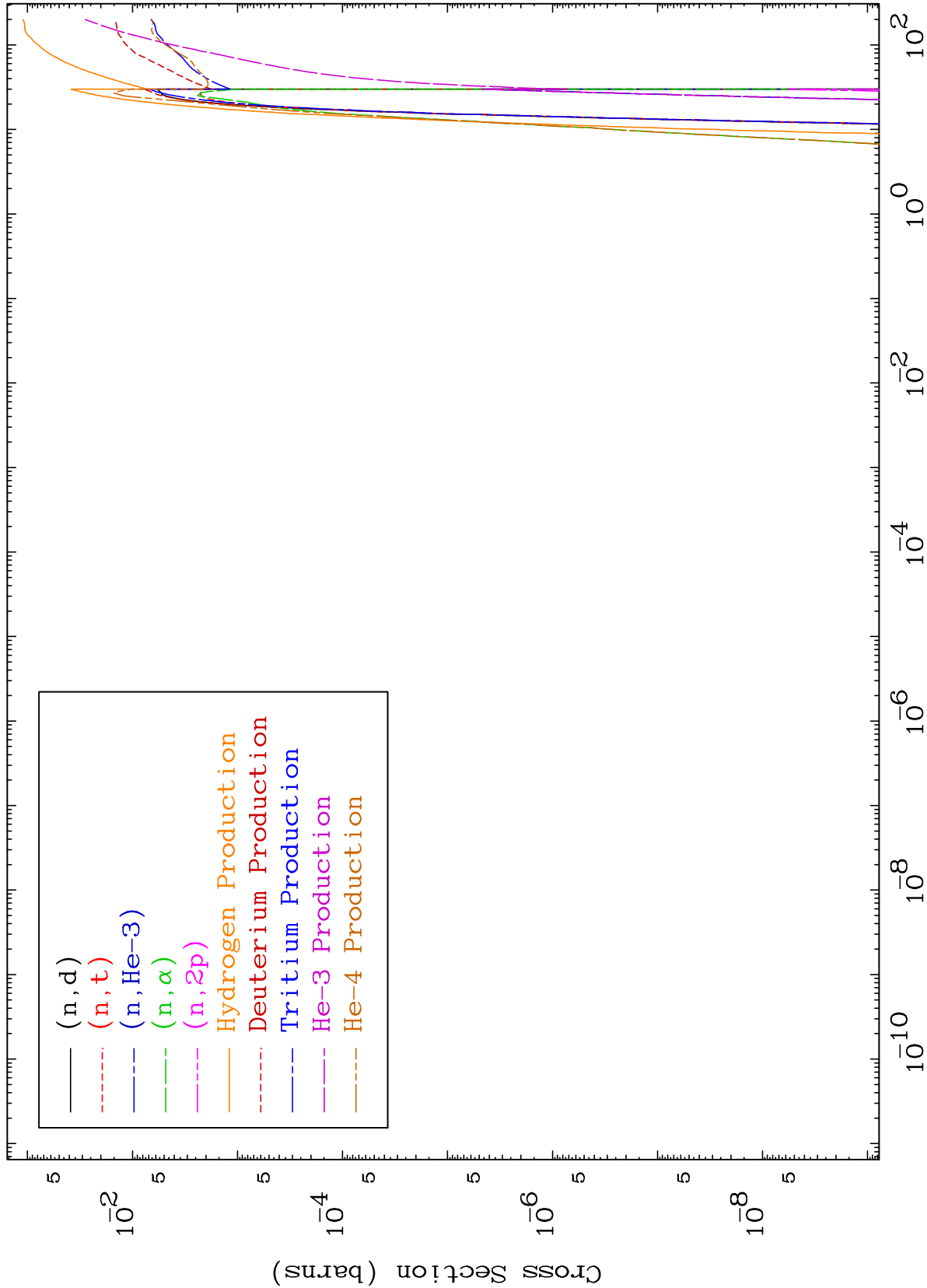


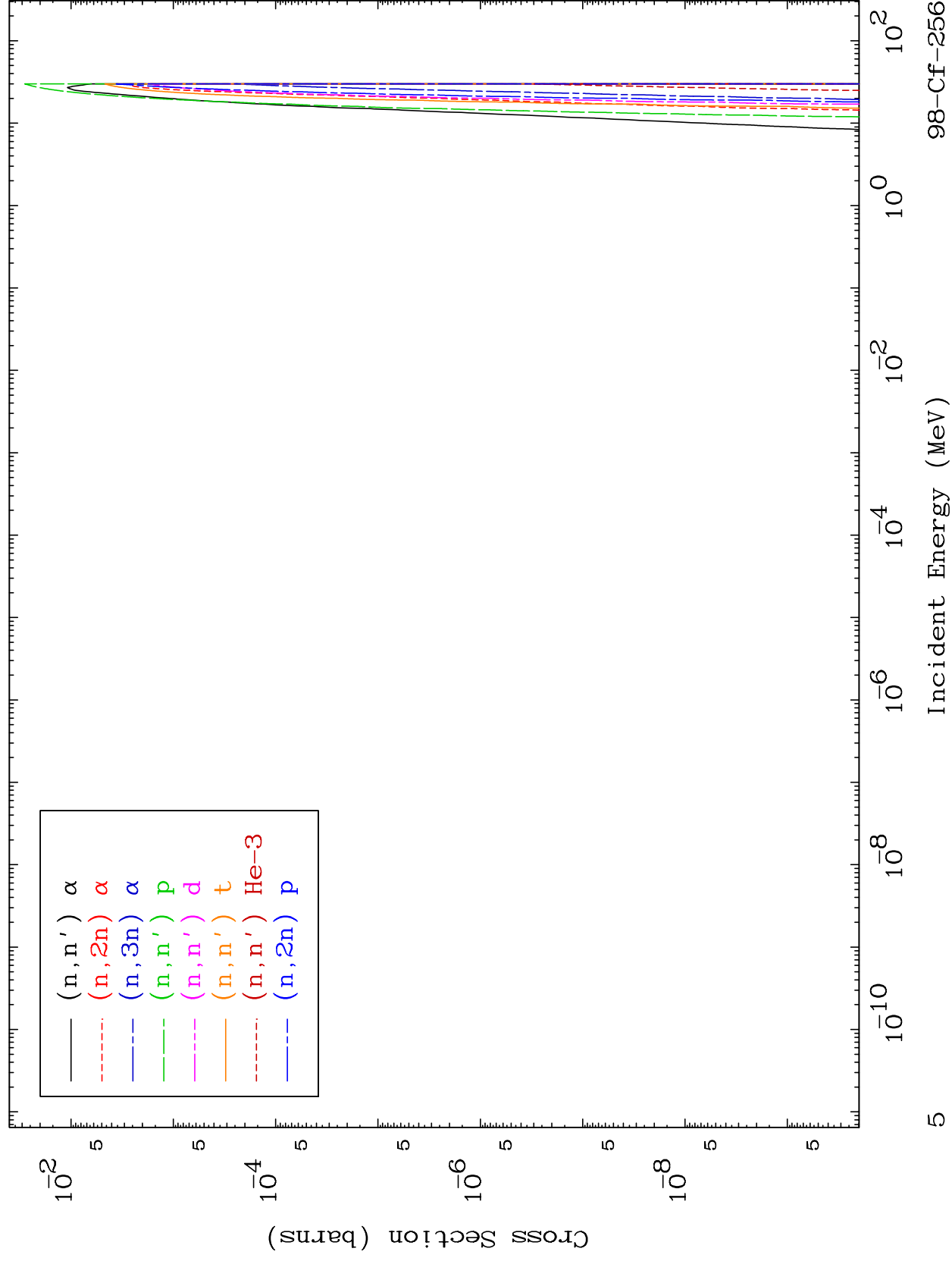


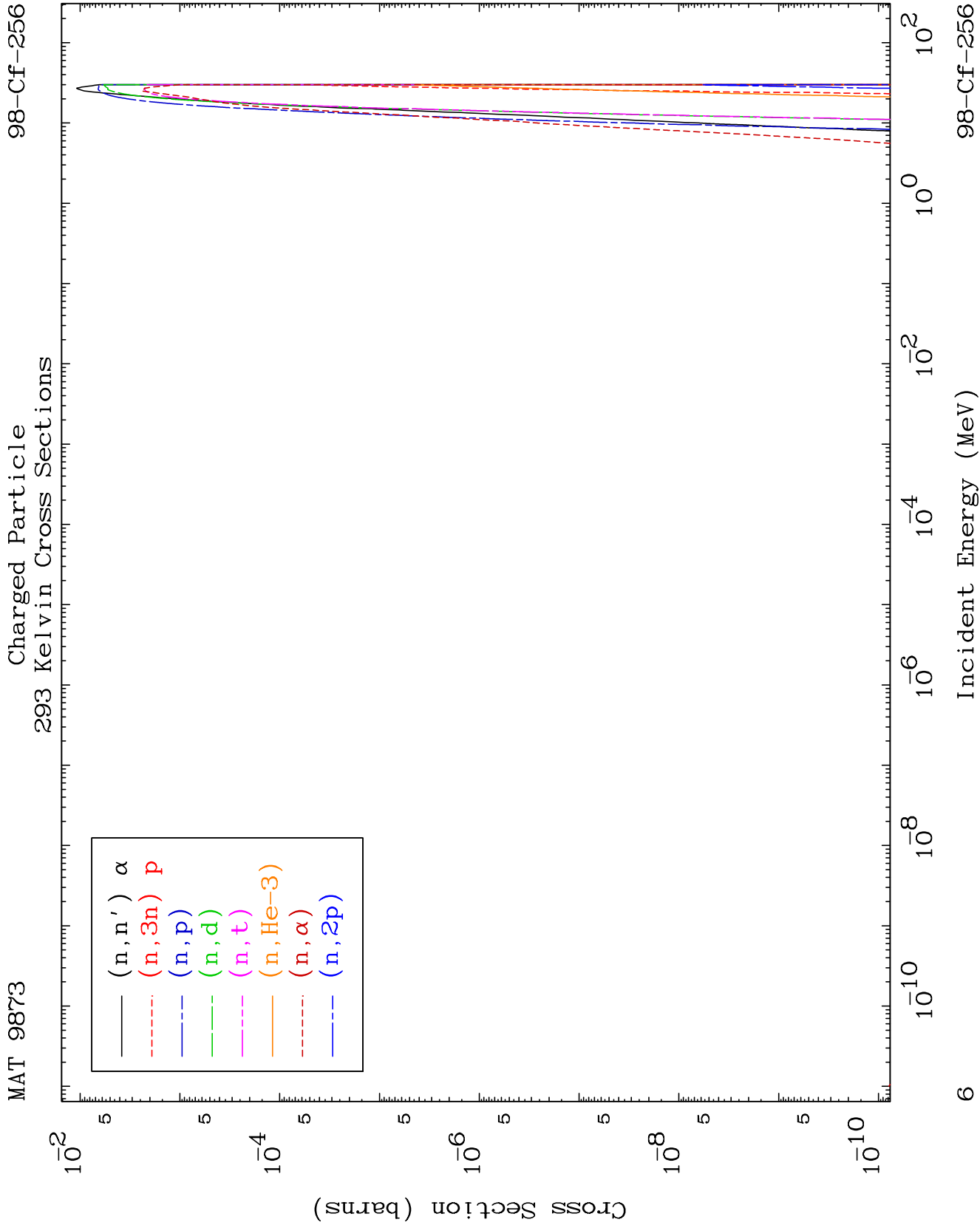
MAT 9873

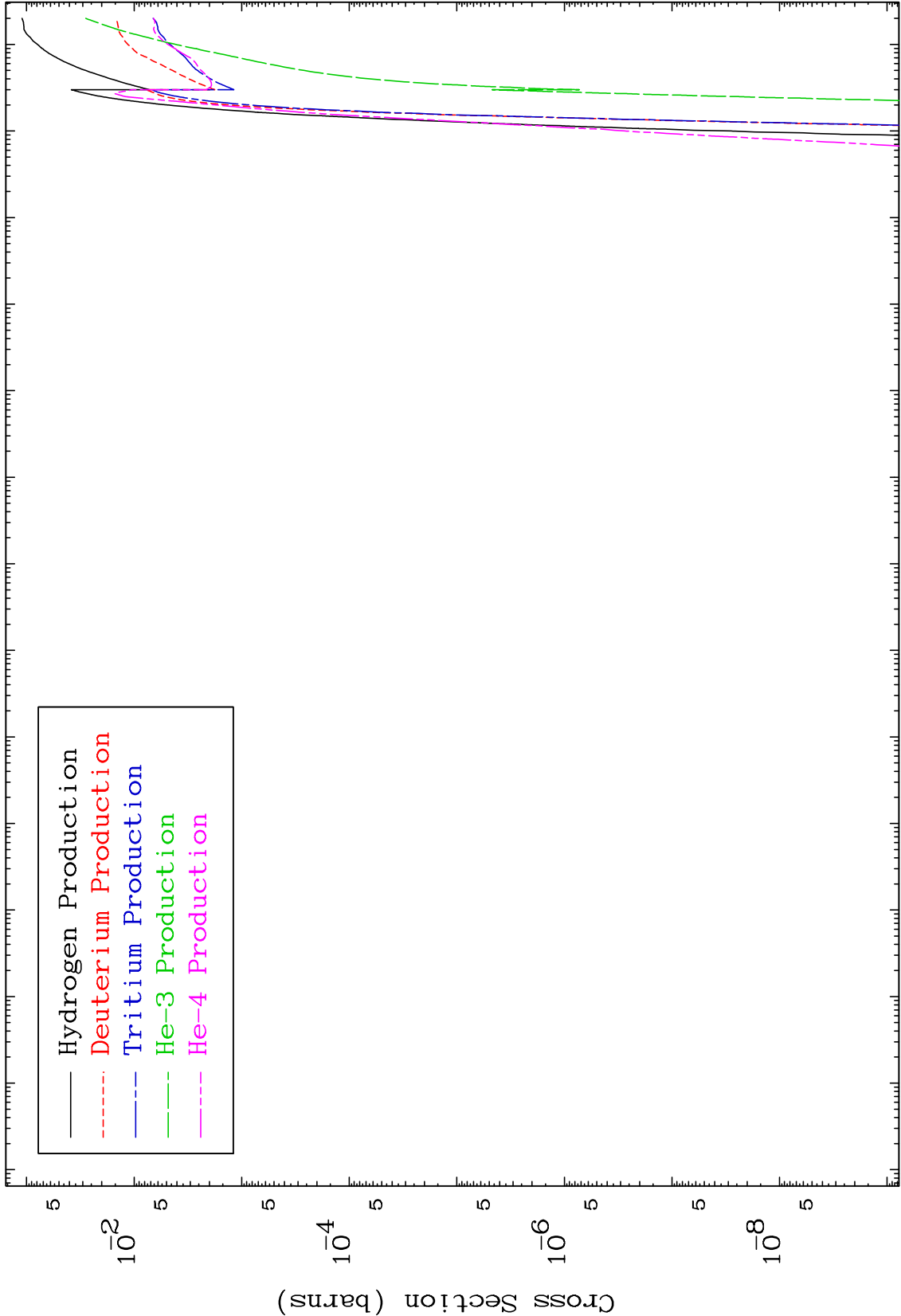
Neutron Absorption
293 Kelvin Cross Sections

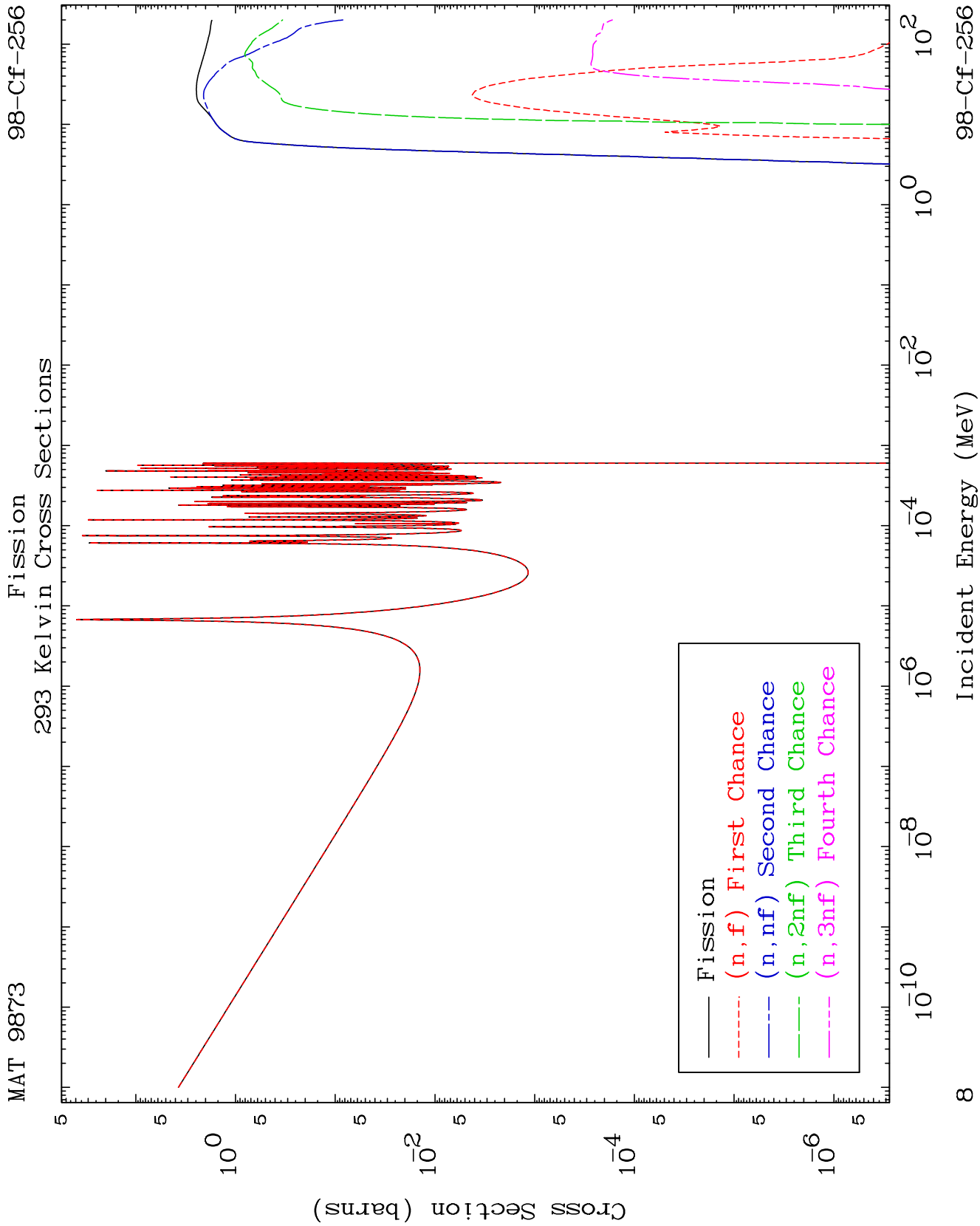
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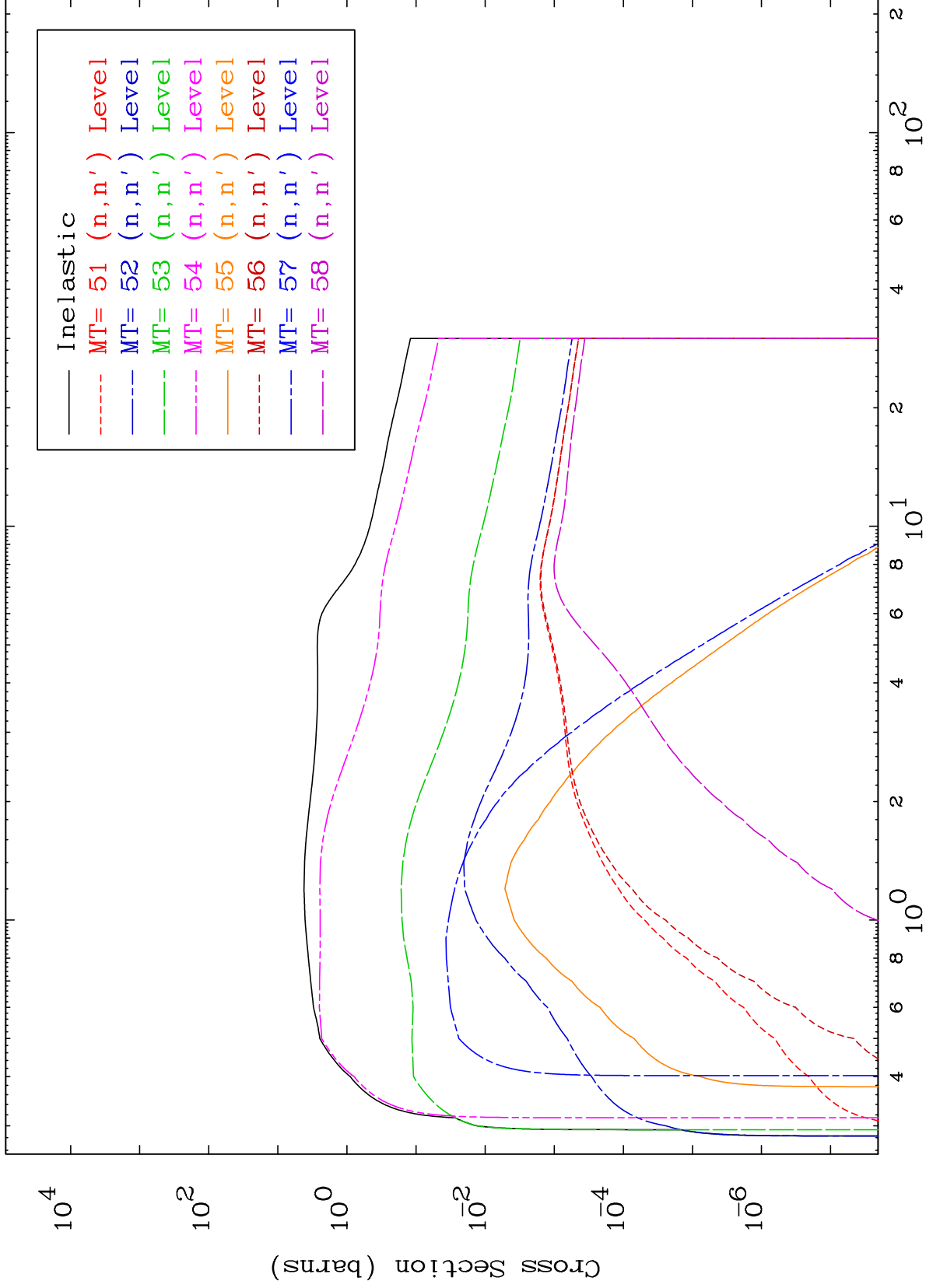


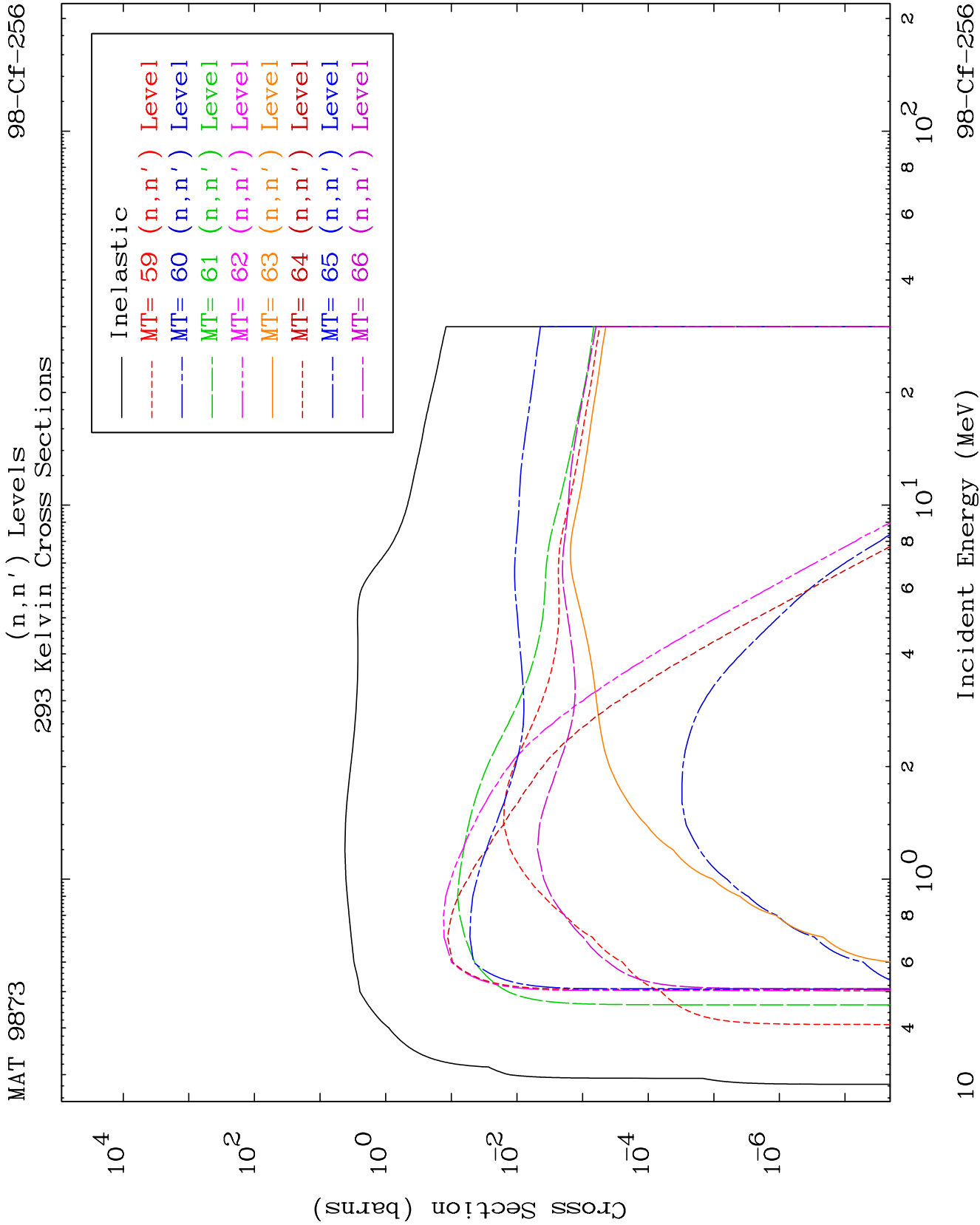




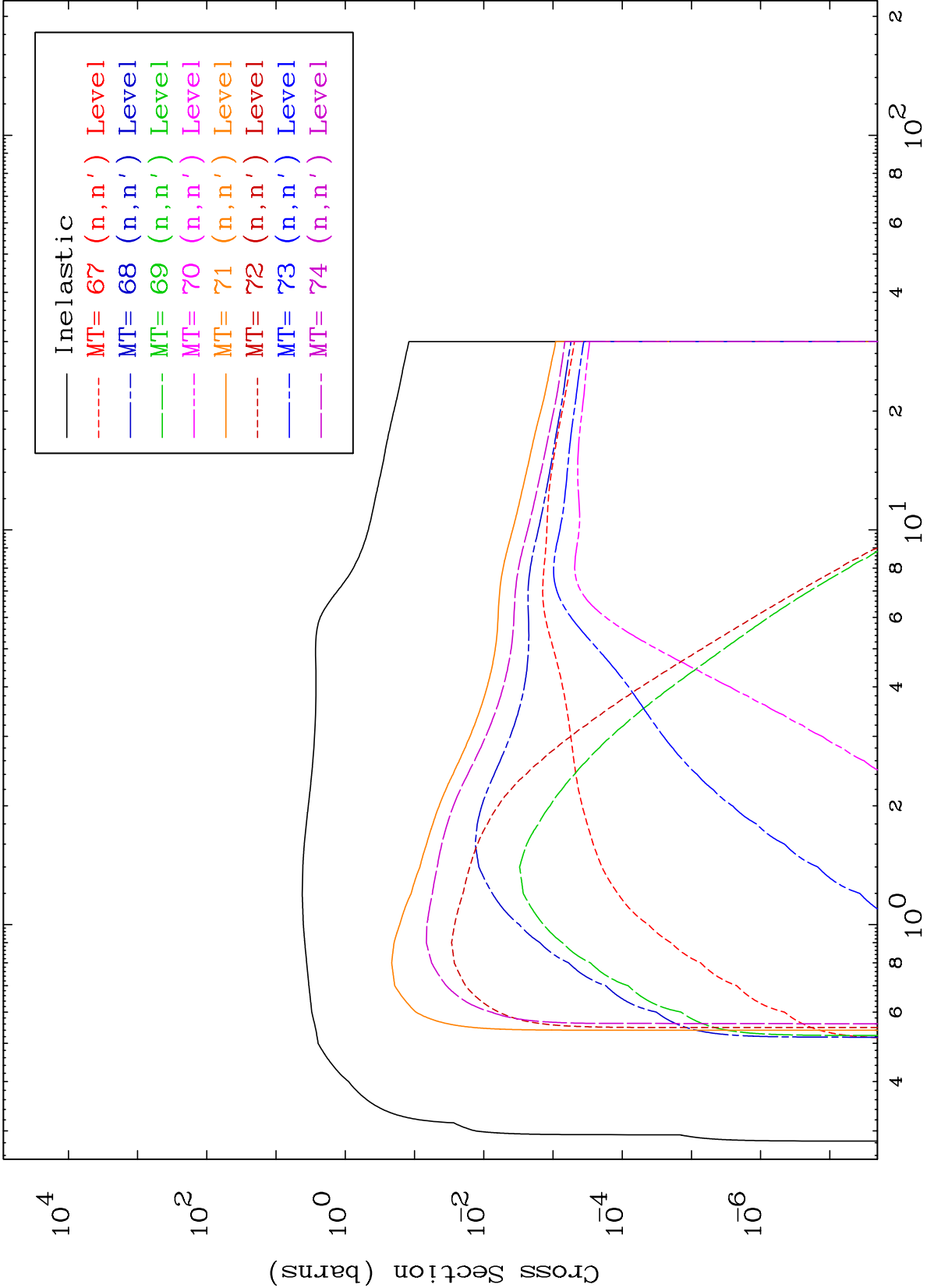


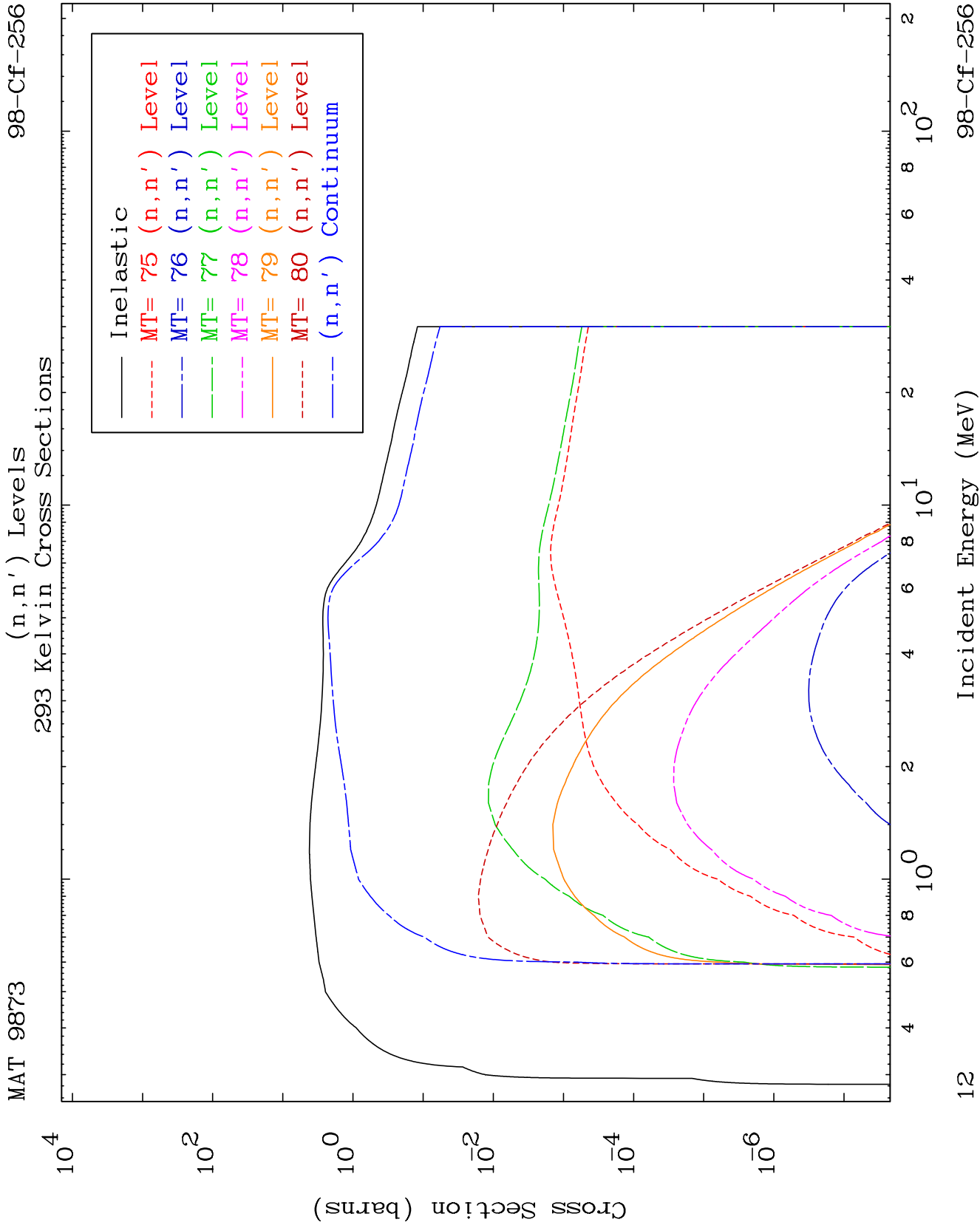
293 Kelvin Cross Sections





293 Kelvin Cross Sections



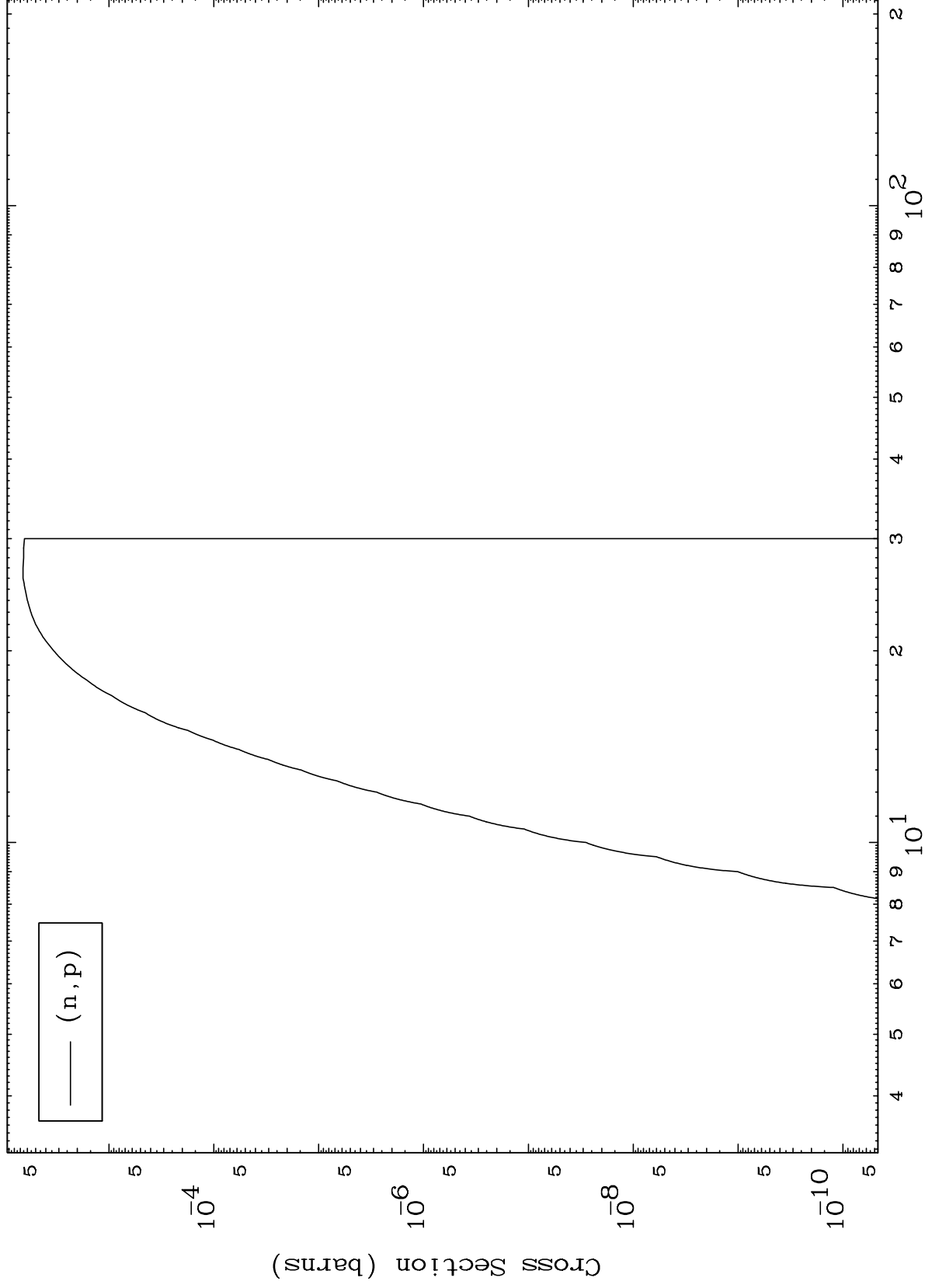


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(n,p) Levels

98-Cf-256

293 Kelvin Cross Sections



13

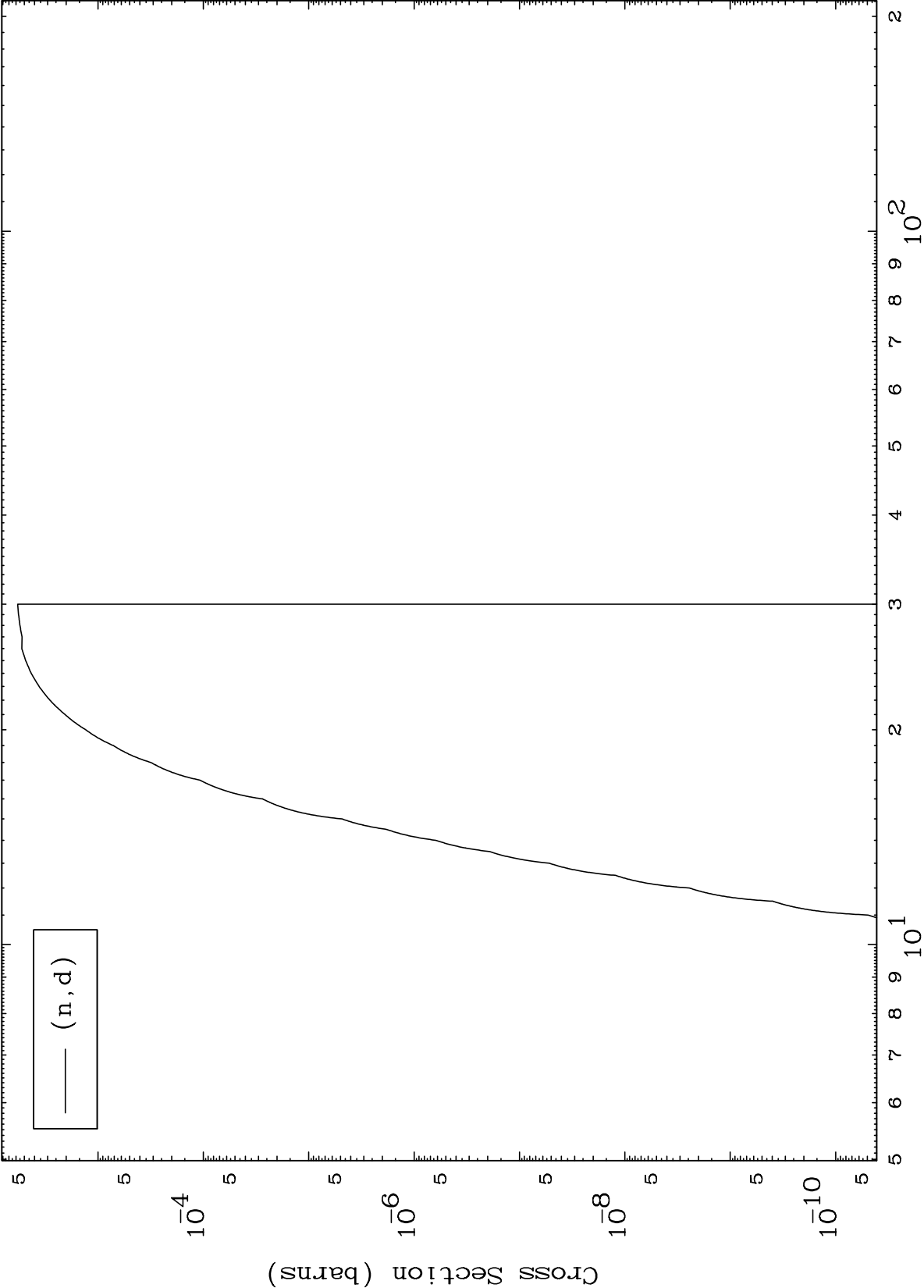
Incident Energy (MeV)

98-Cf-256

MAT 9873

98-Cf-256

(n,d) Levels
293 Kelvin Cross Sections



14

Incident Energy (MeV)

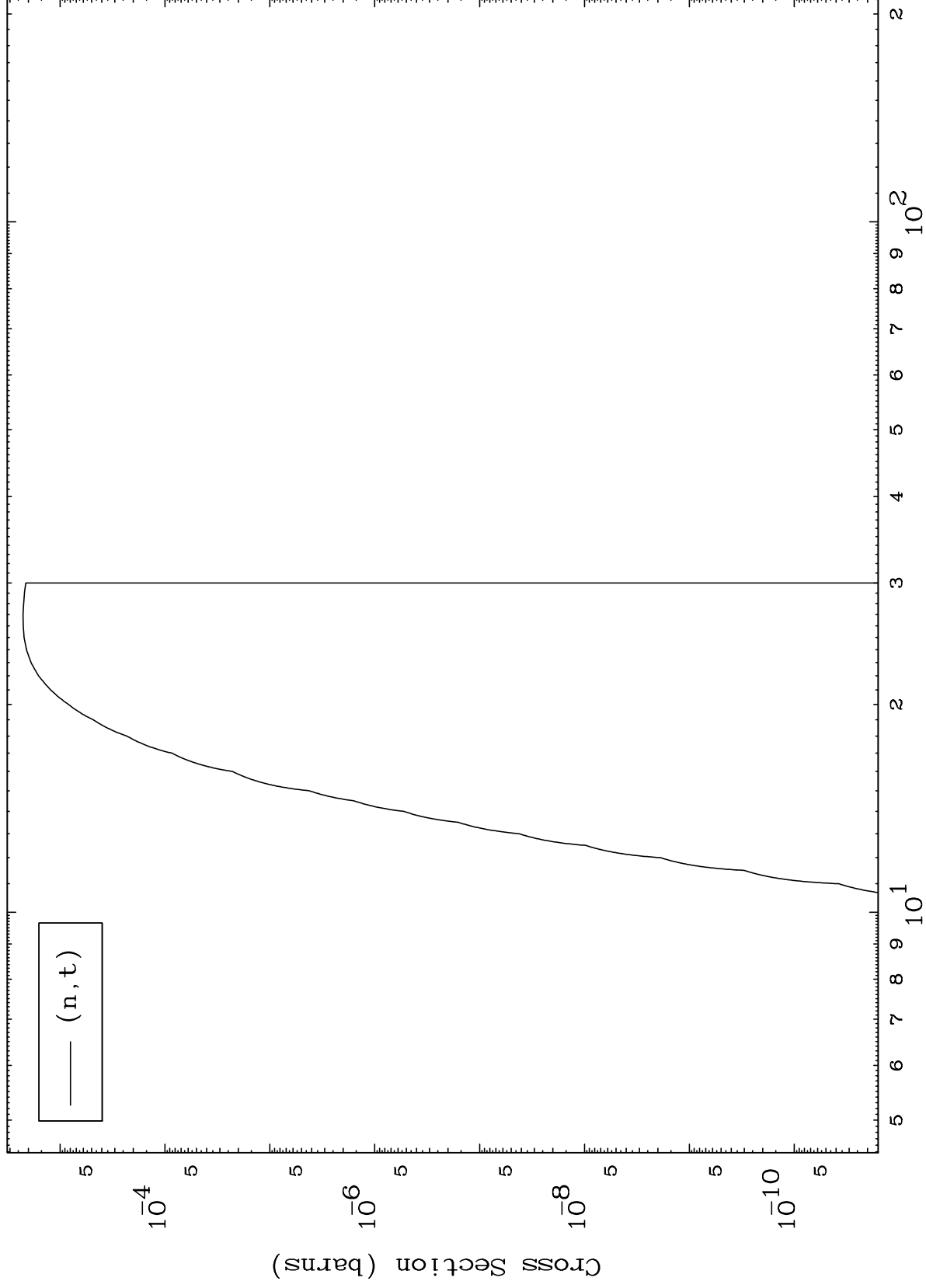
98-Cf-256

MAT 9873

(n,t) Levels

98-Cf-256

293 Kelvin Cross Sections



15

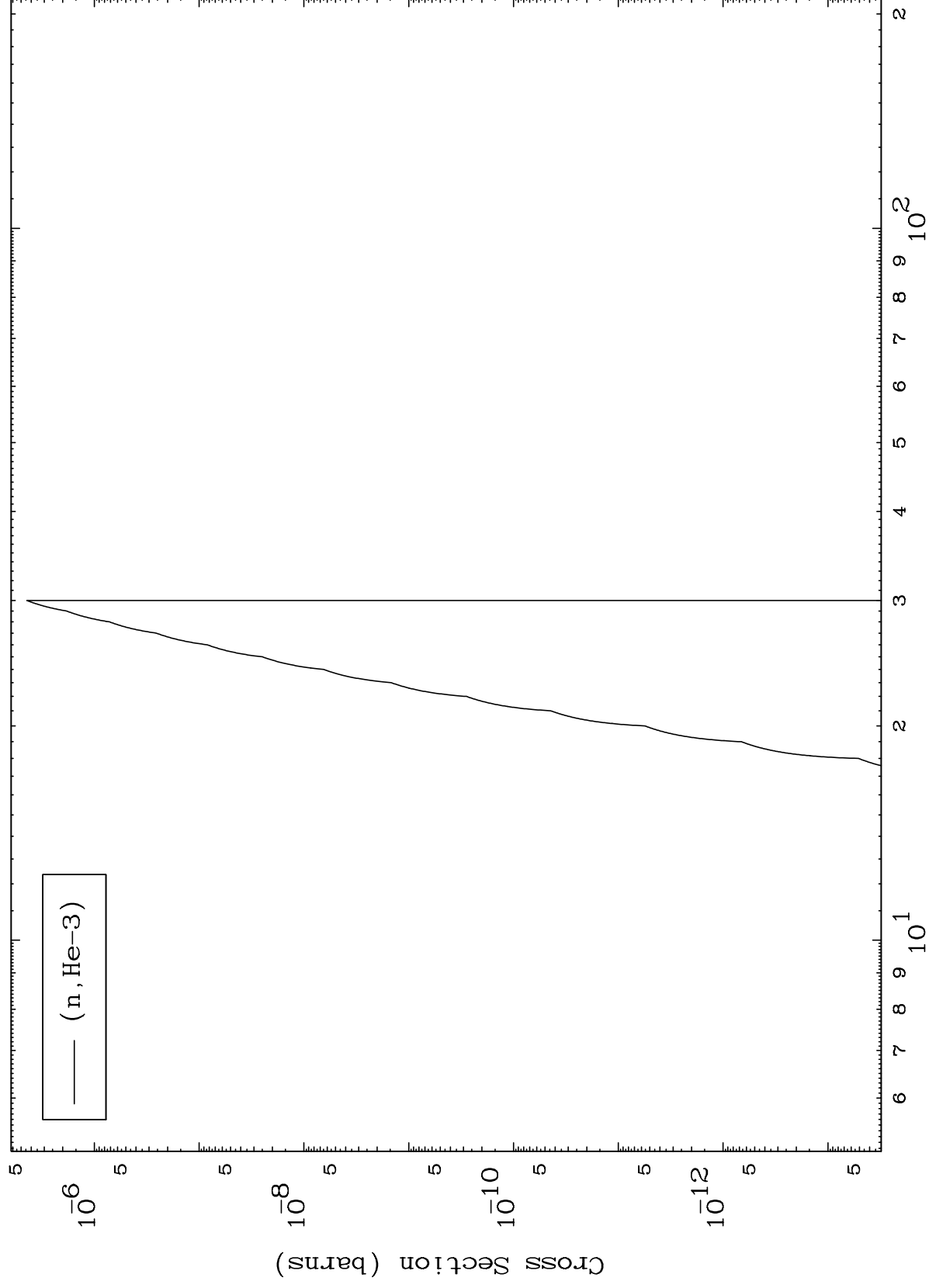
Incident Energy (MeV)

98-Cf-256

MAT 9873

(n,He3) Levels
293 Kelvin Cross Sections

98-Cf-256



16

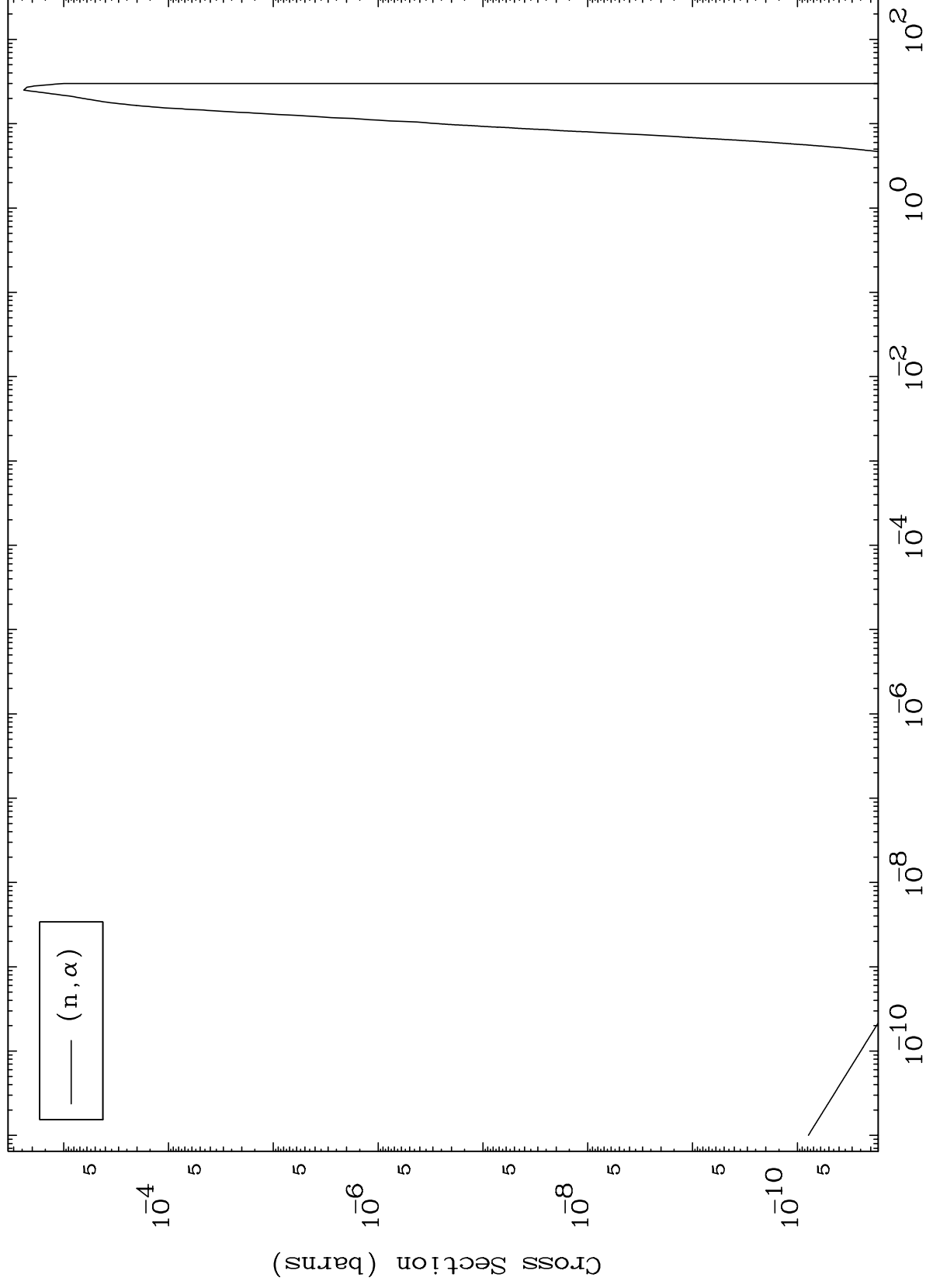
Incident Energy (MeV)

98-Cf-256

MAT 9873

(n, α) Levels
293 Kelvin Cross Sections

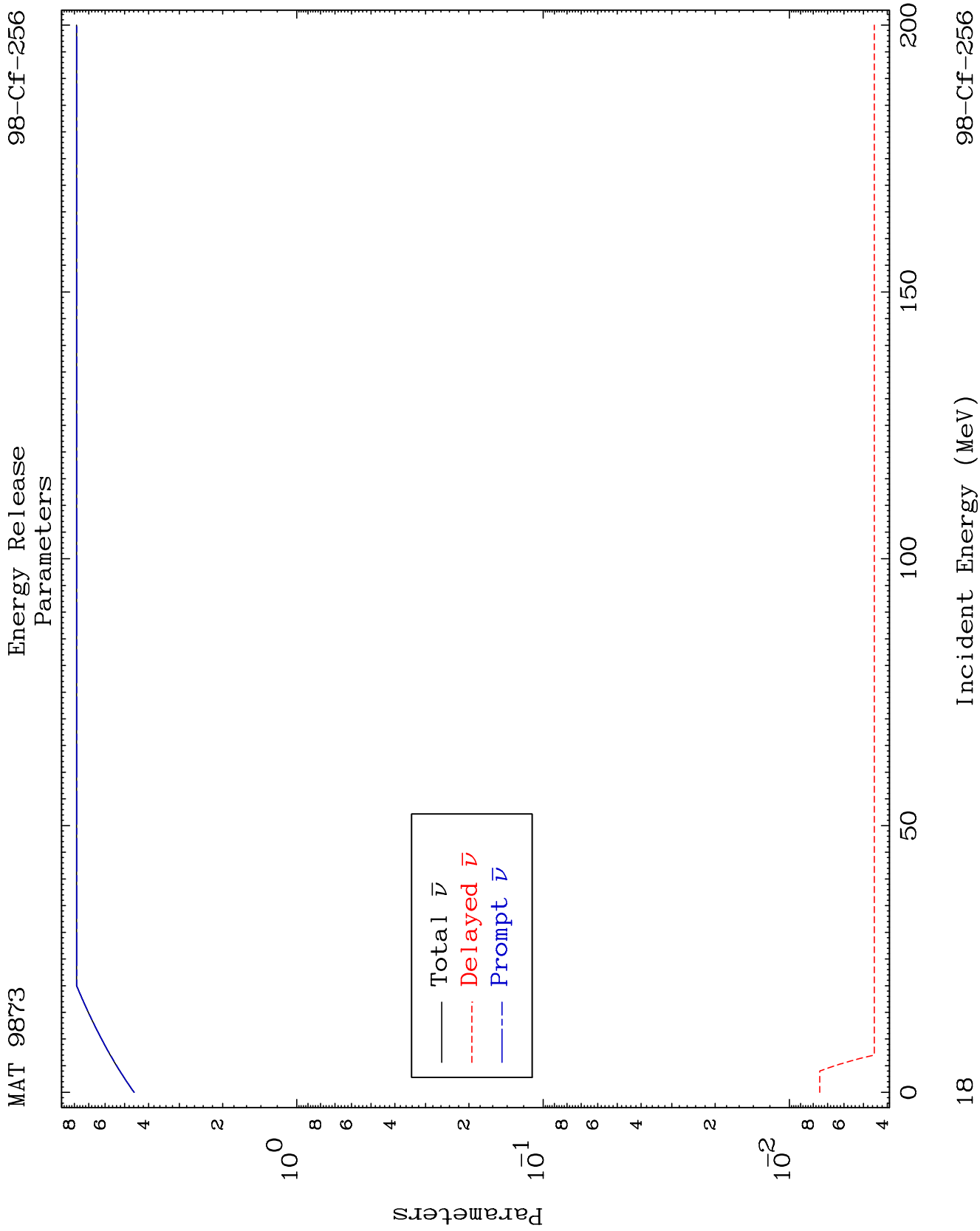
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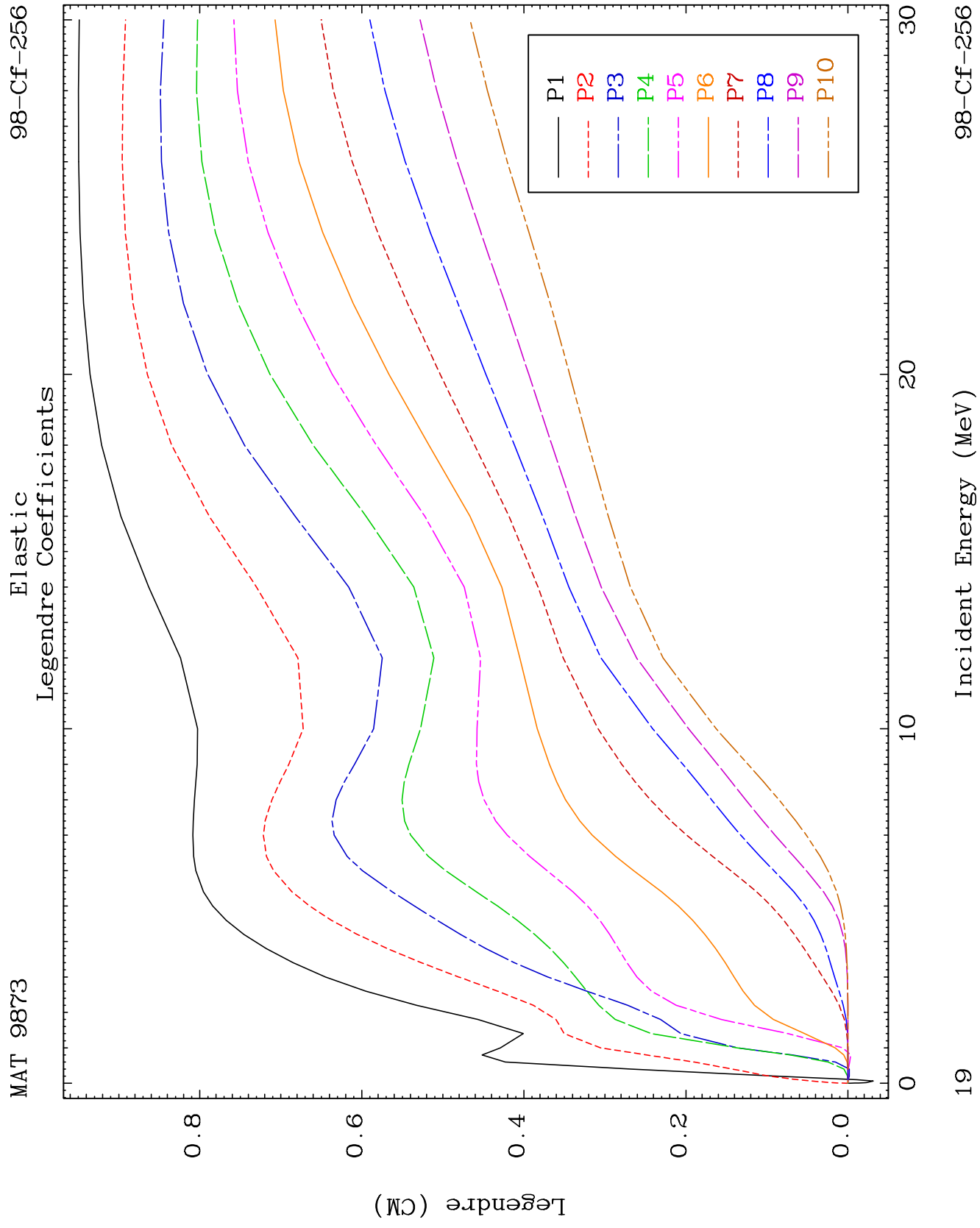


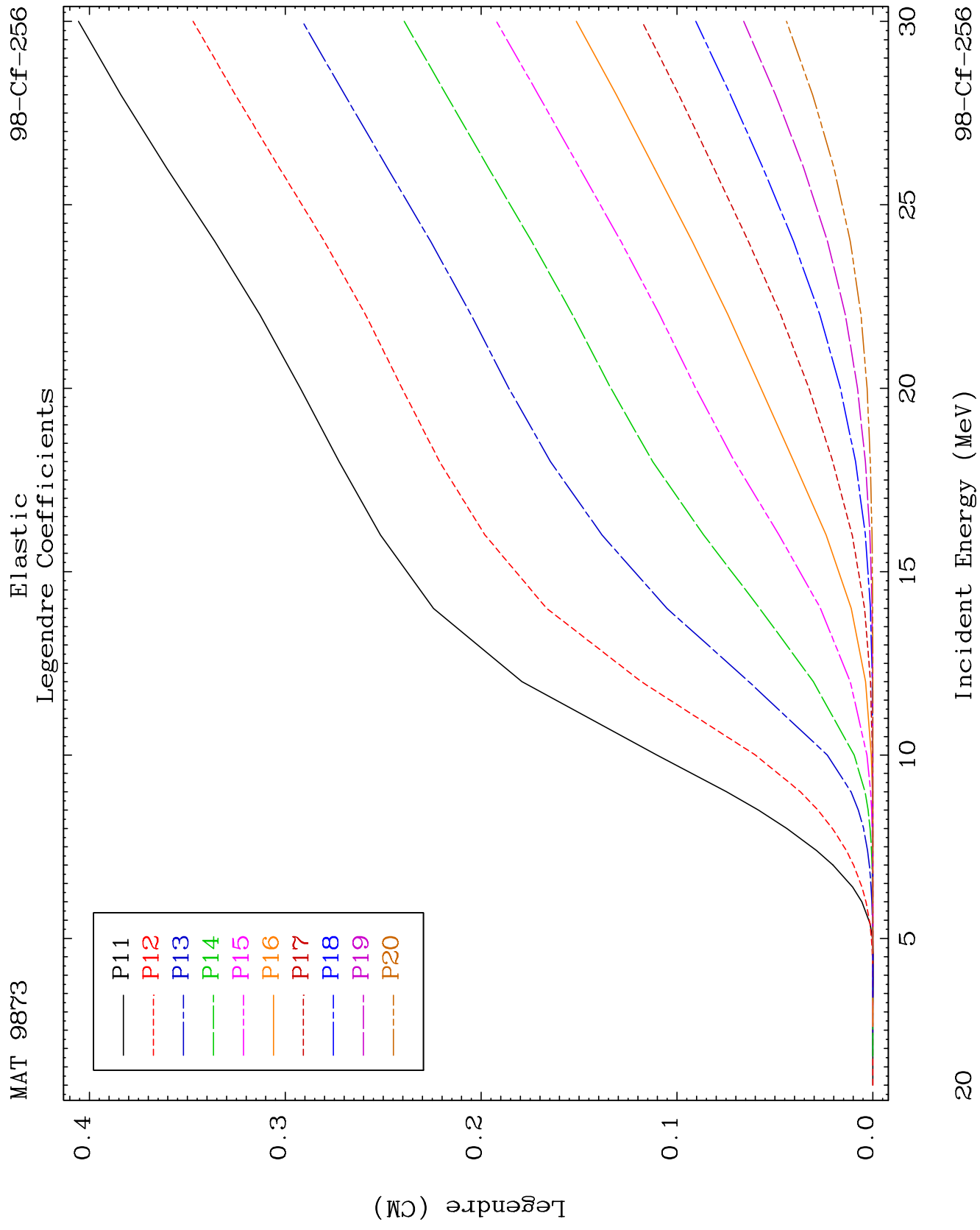
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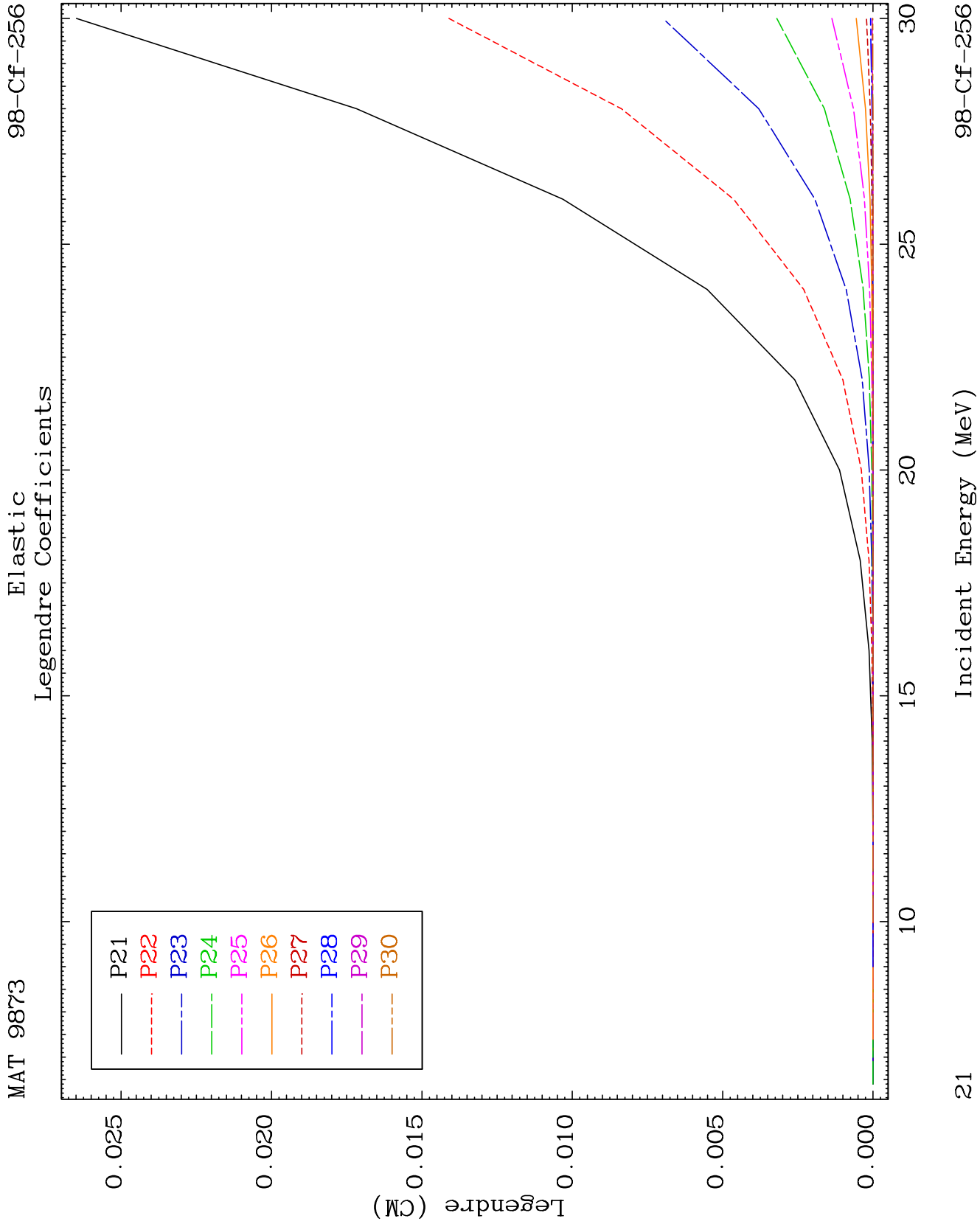
Incident Energy (MeV)

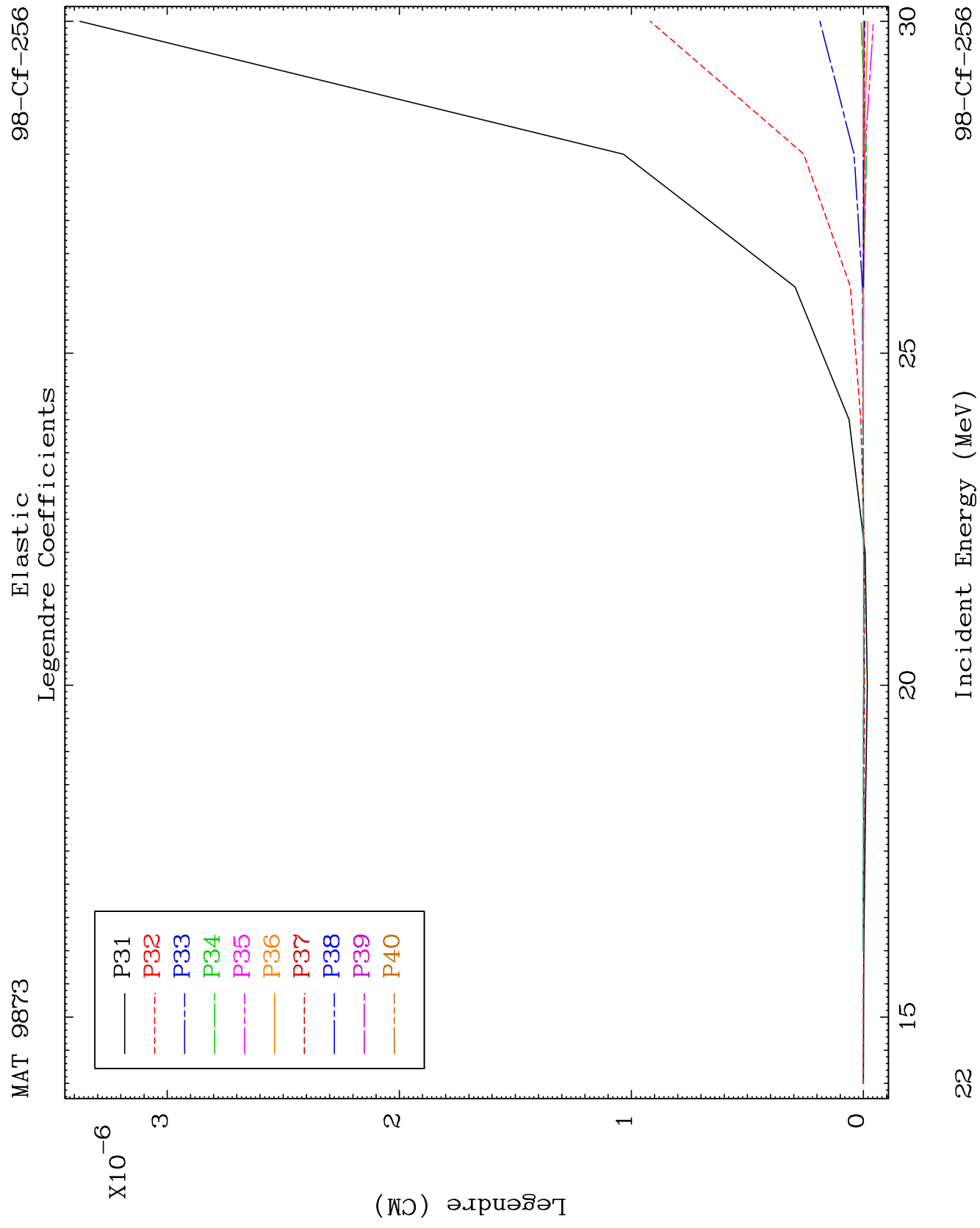
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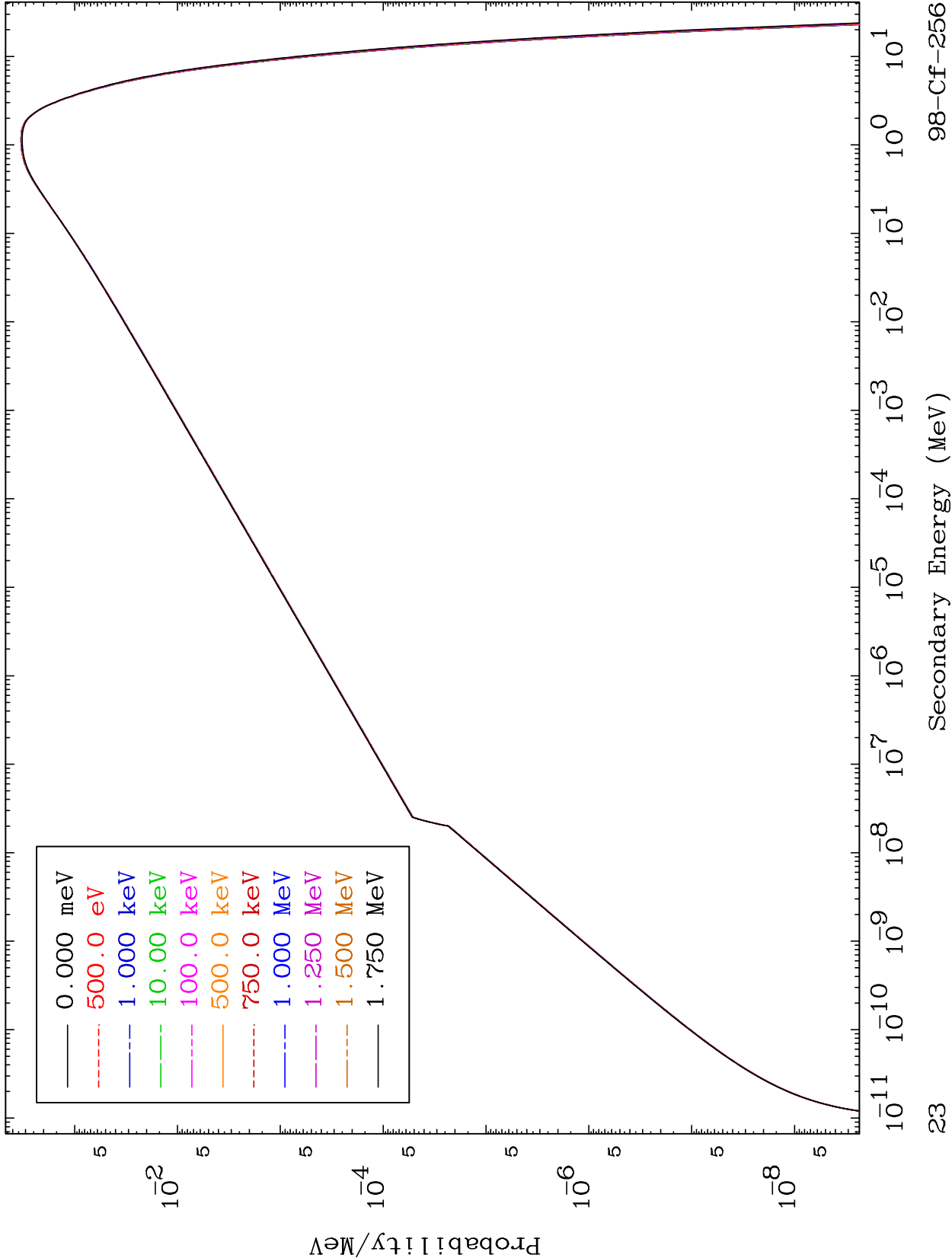








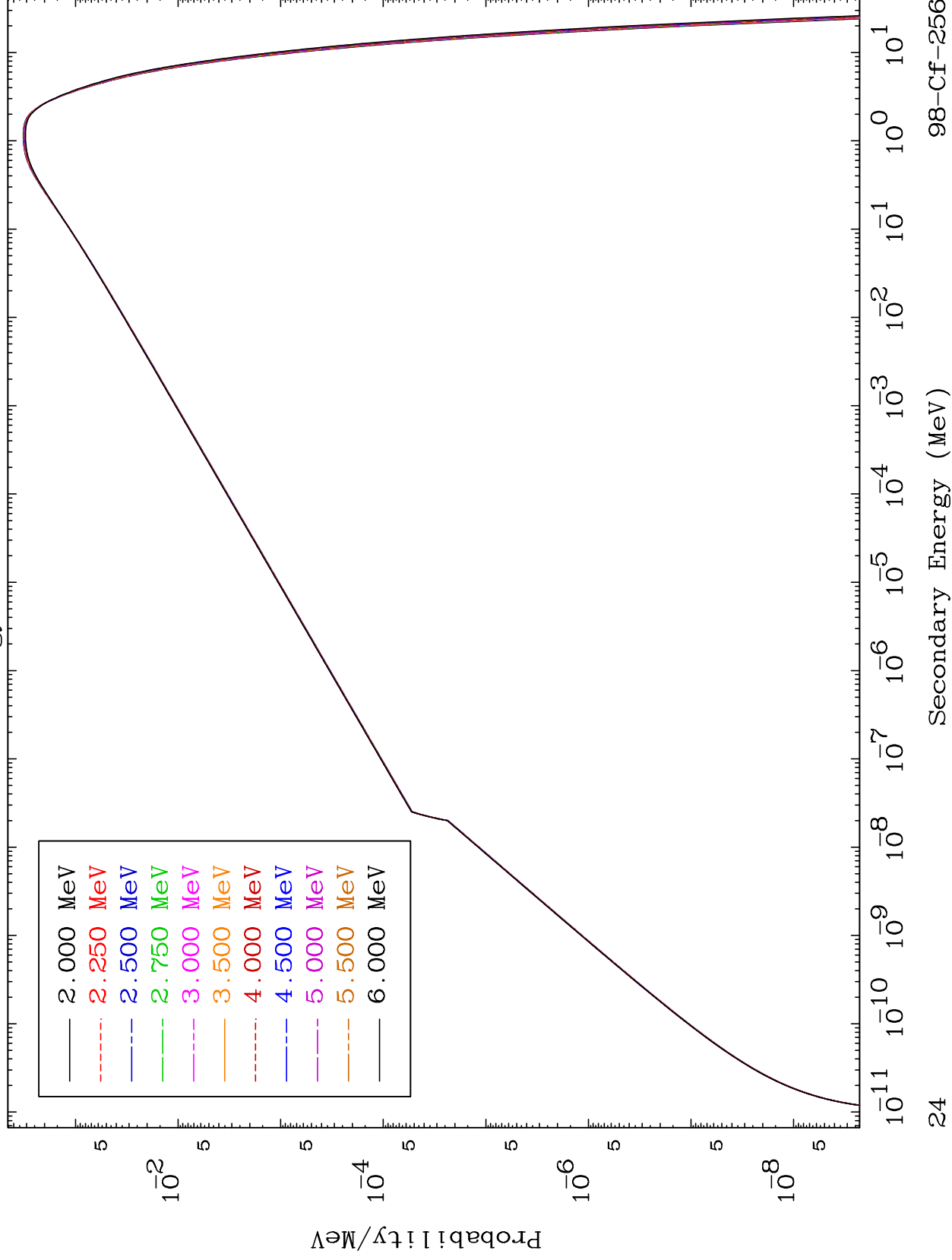


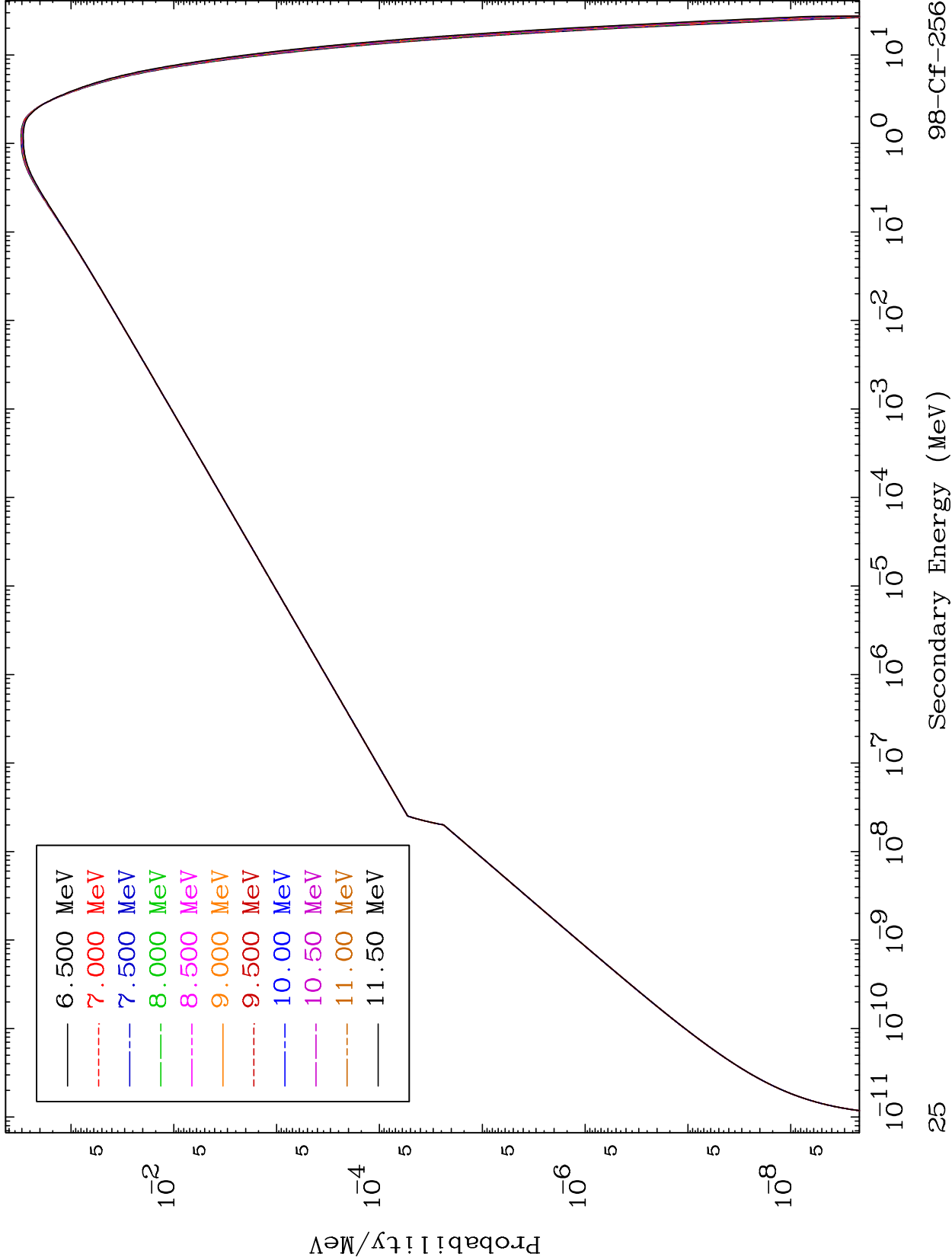


MAT 9873

Fission
Energy Distributions

98-Cf-256





MAT 9873

Fission
Energy Distributions

98-Cf-256

