

Program EVALPLOT  
(Version 2021-1)

by

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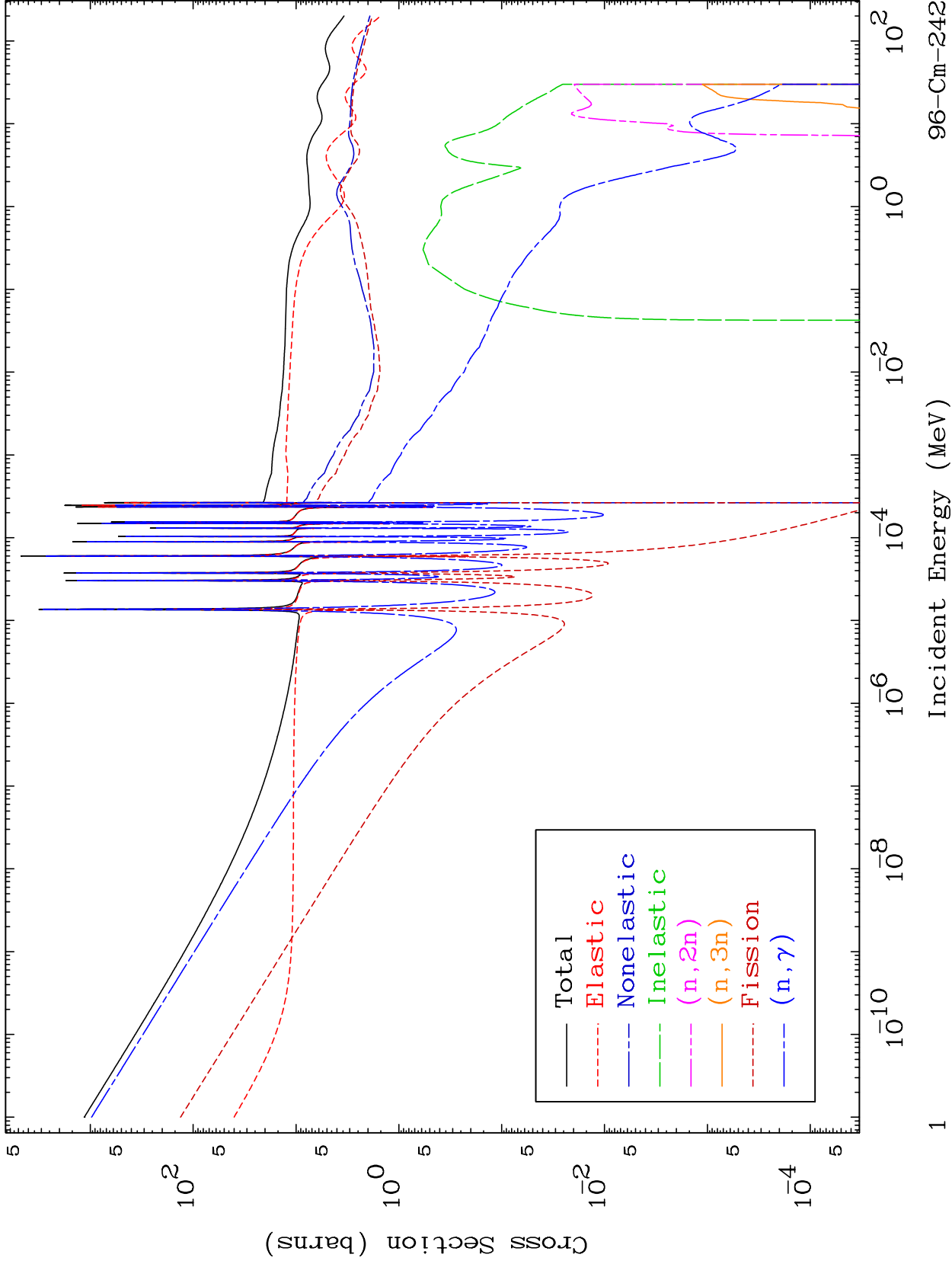
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Press Mouse Button to Start

MAT 9631

Neutron Major  
293 Kelvin Cross Sections

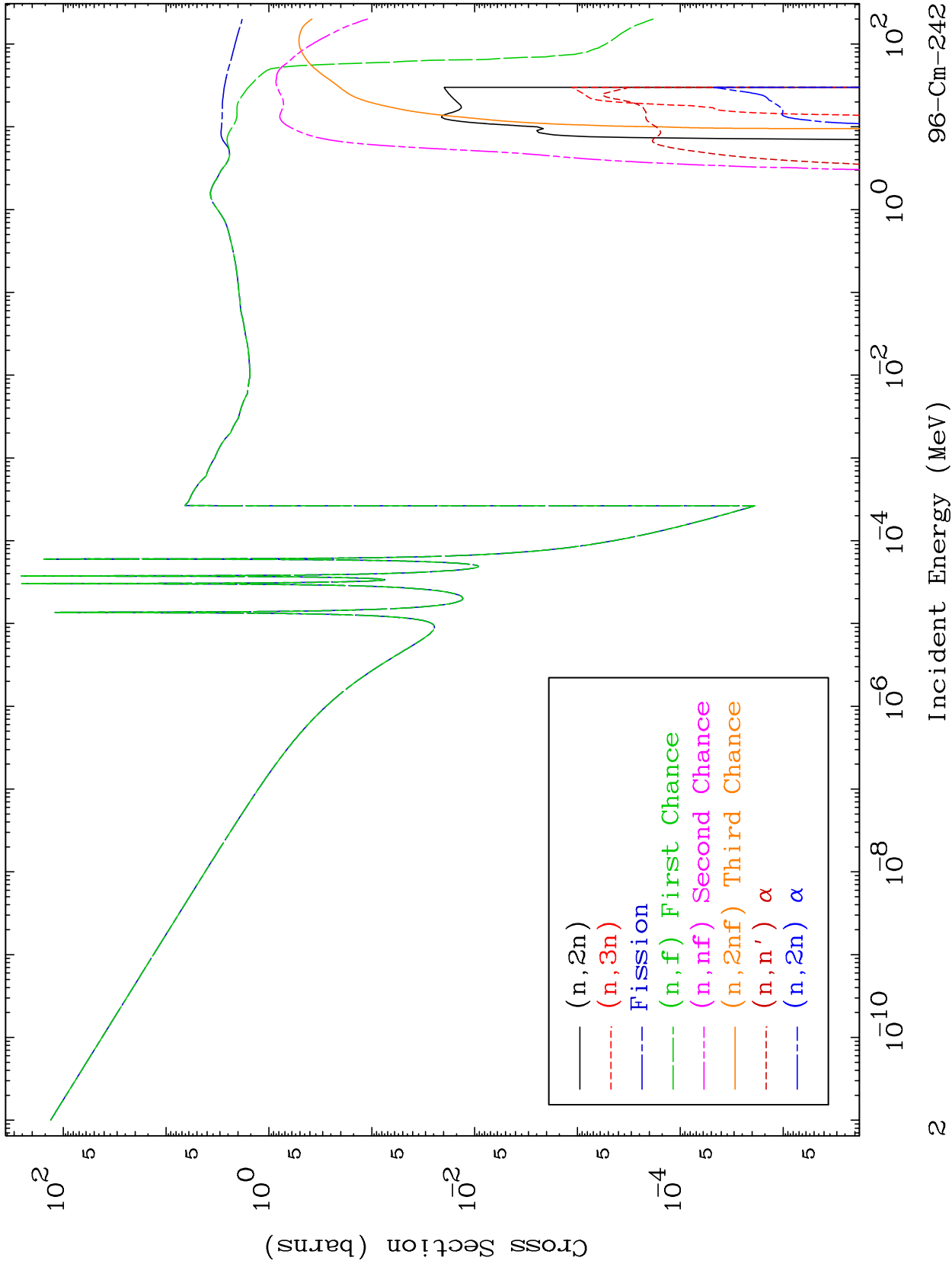
96-Cm-242



MAT 9631

Neutron Absorption  
293 Kelvin Cross Sections

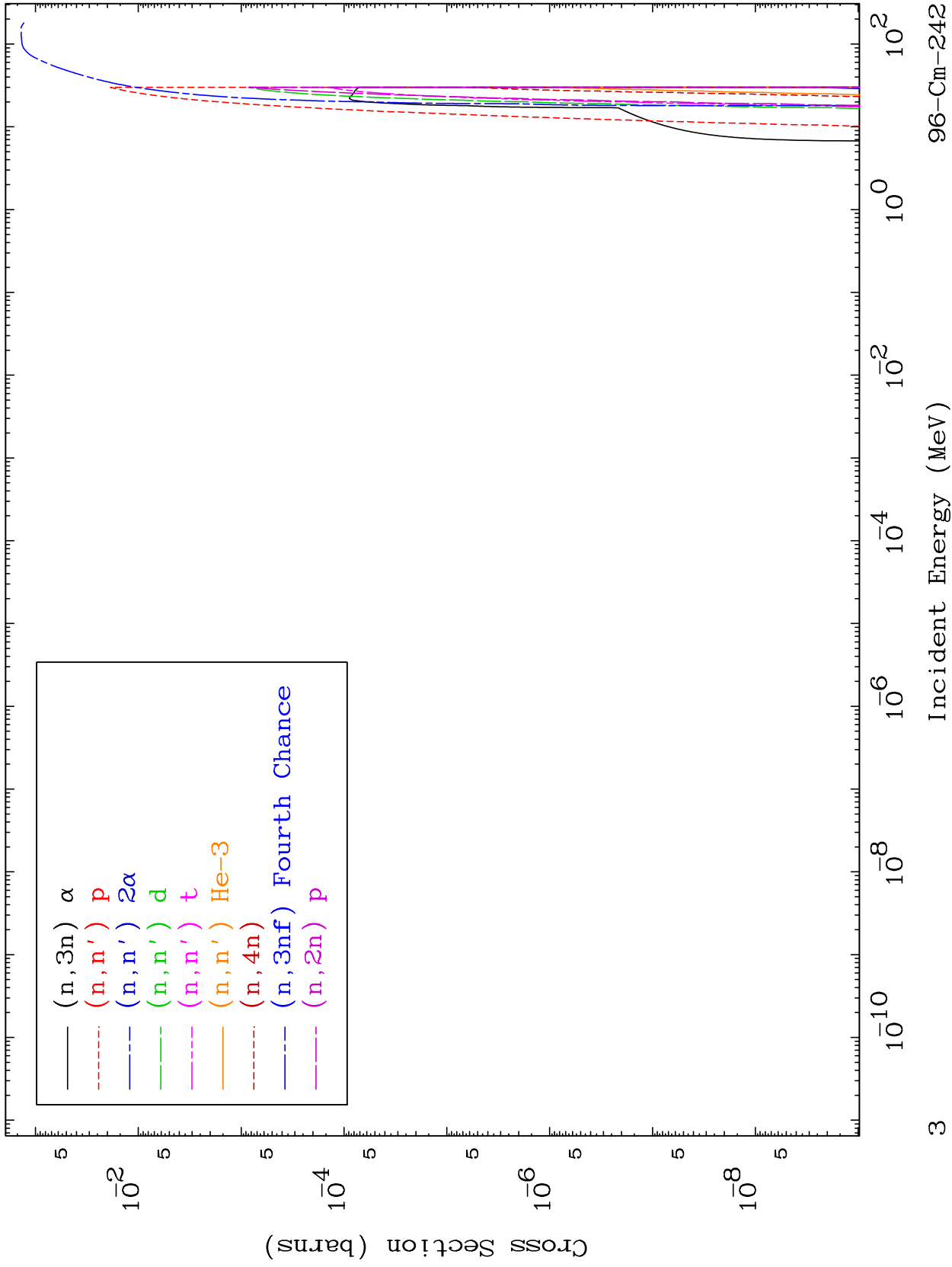
96-Cm-242



MAT 9631

Neutron Absorption  
293 Kelvin Cross Sections

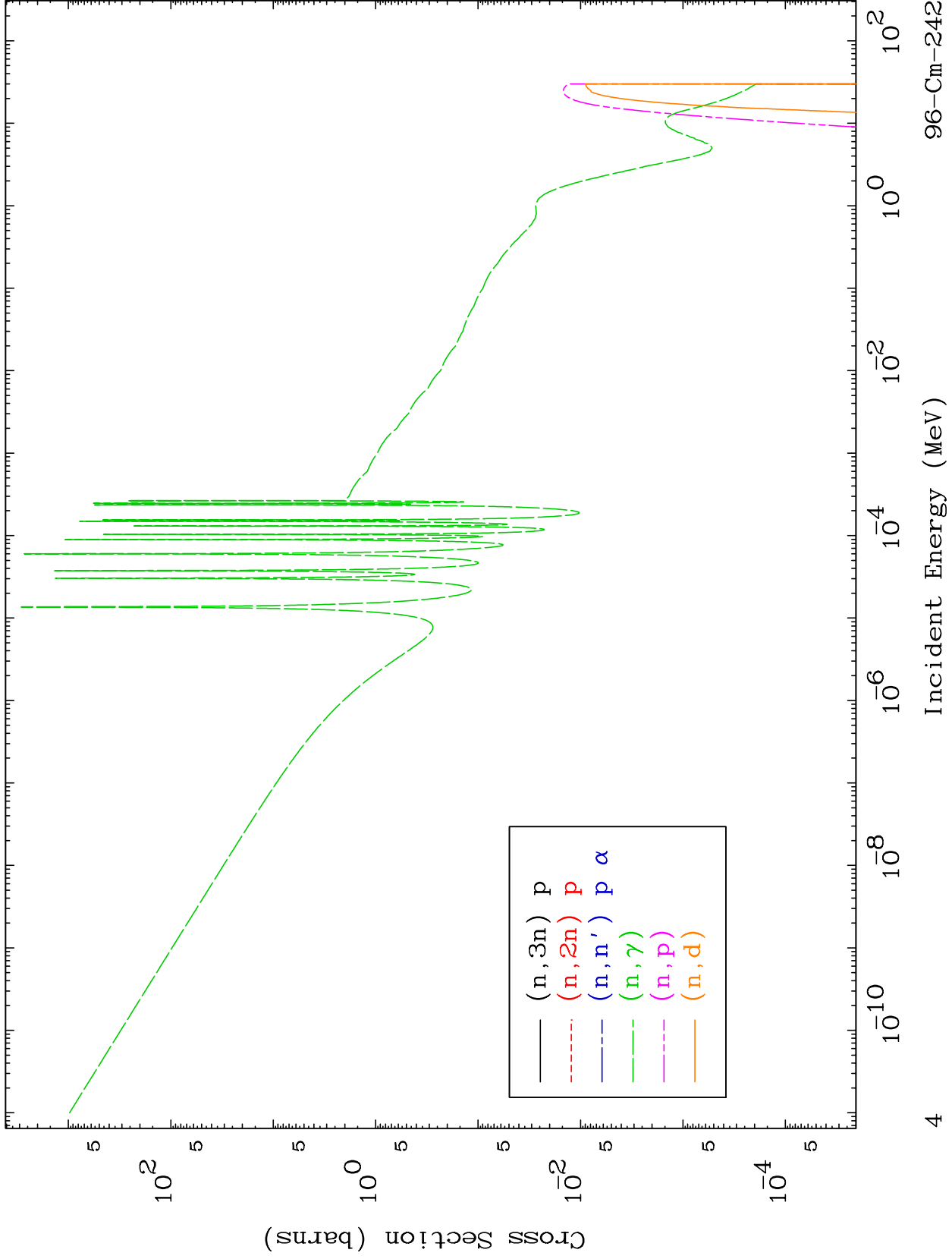
96-Cm-242



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Neutron Absorption  
293 Kelvin Cross Sections

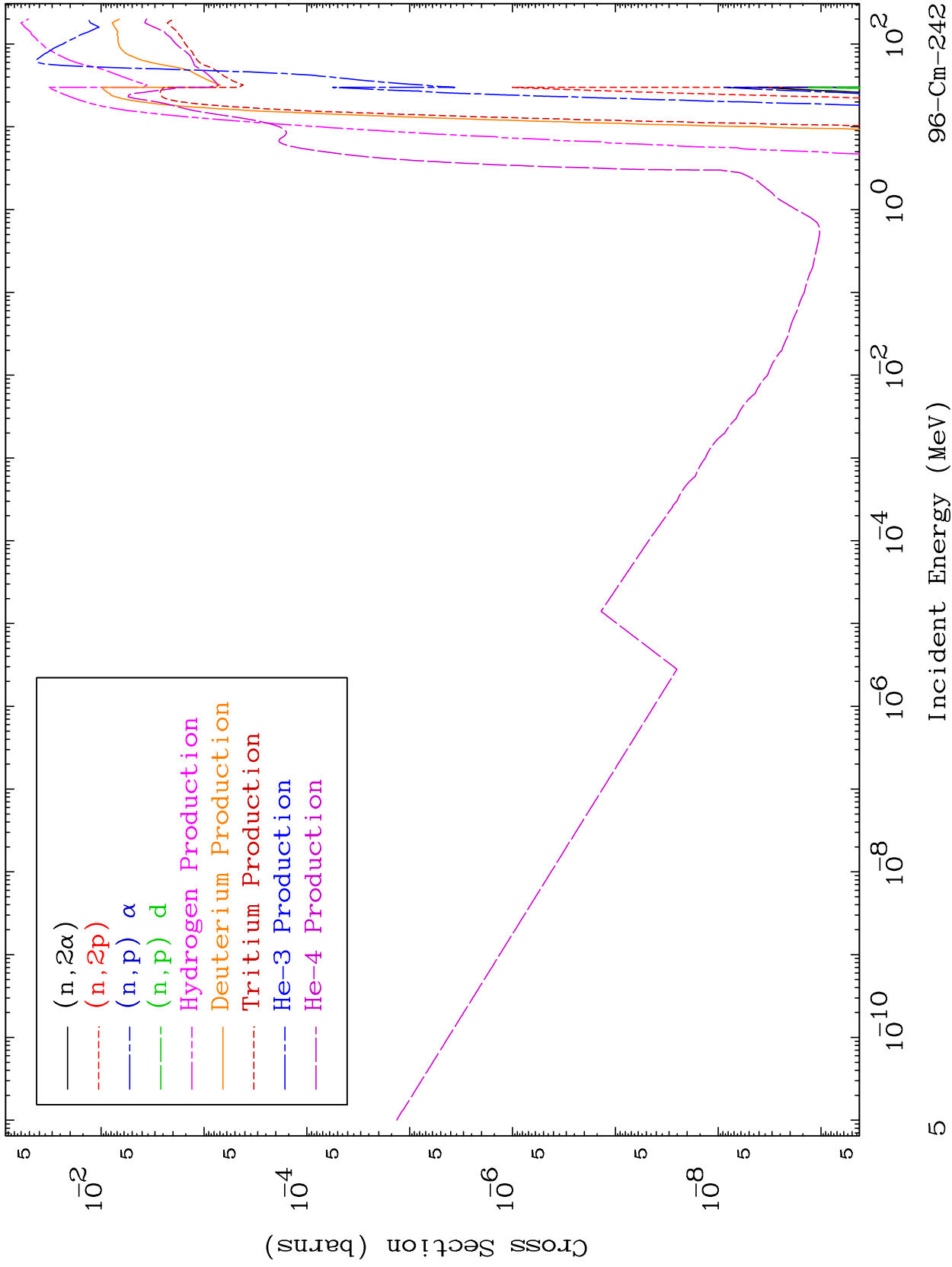
96-Cm-242



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Neutron Absorption  
293 Kelvin Cross Sections

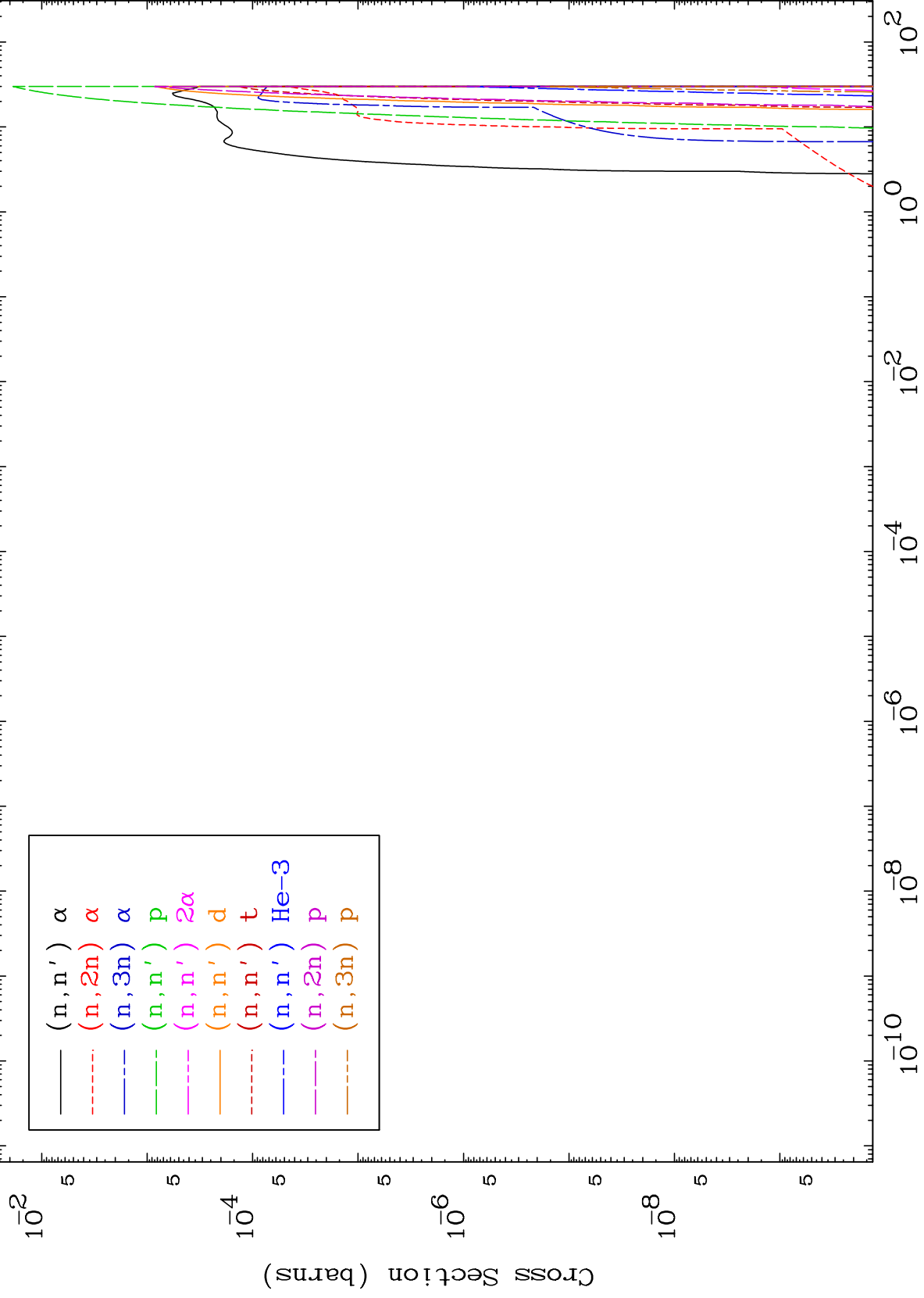
96-Cm-242



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Charged Particle  
293 Kelvin Cross Sections

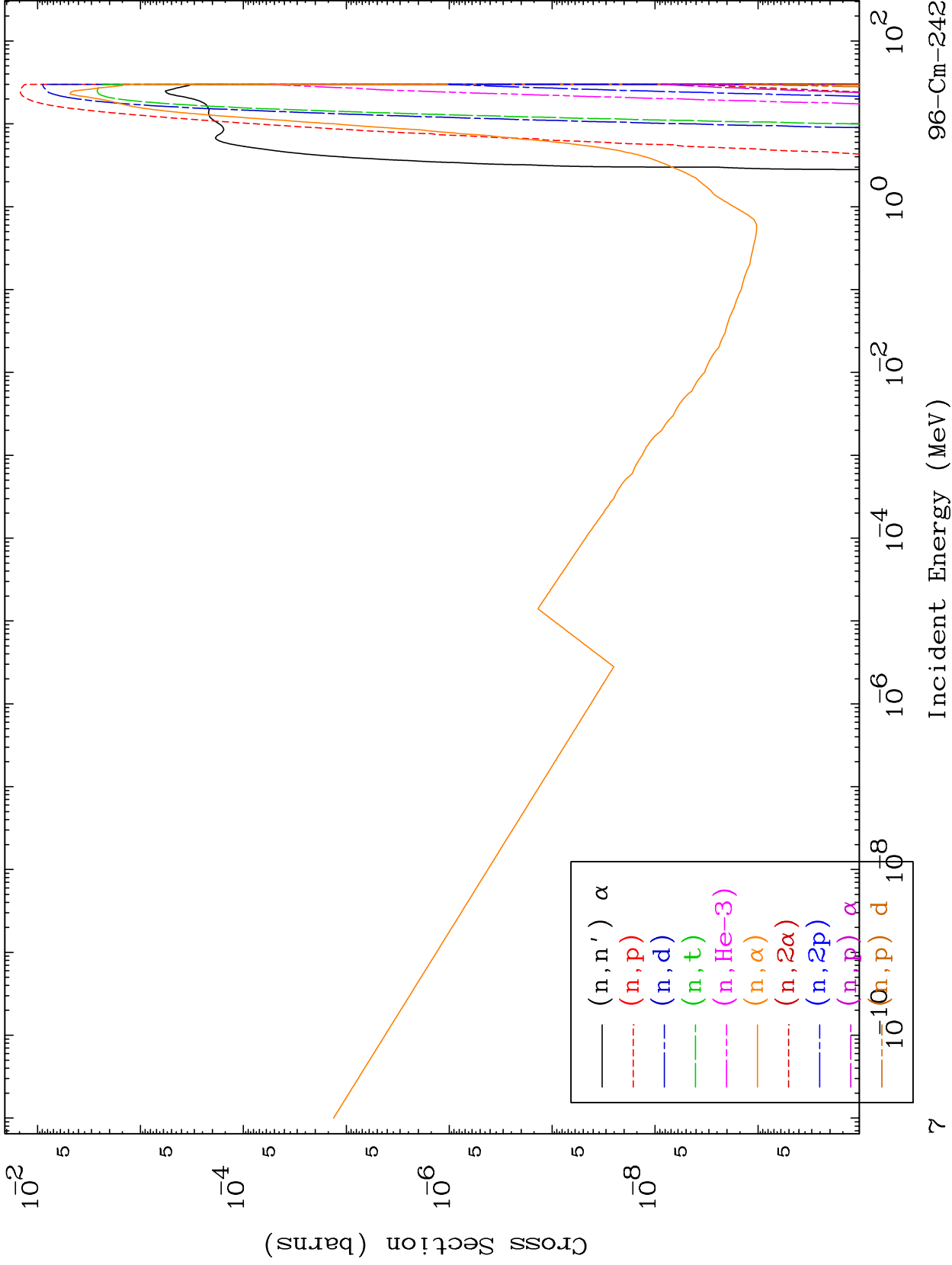
96-Cm-242



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Charged Particle  
293 Kelvin Cross Sections

96-Cm-242

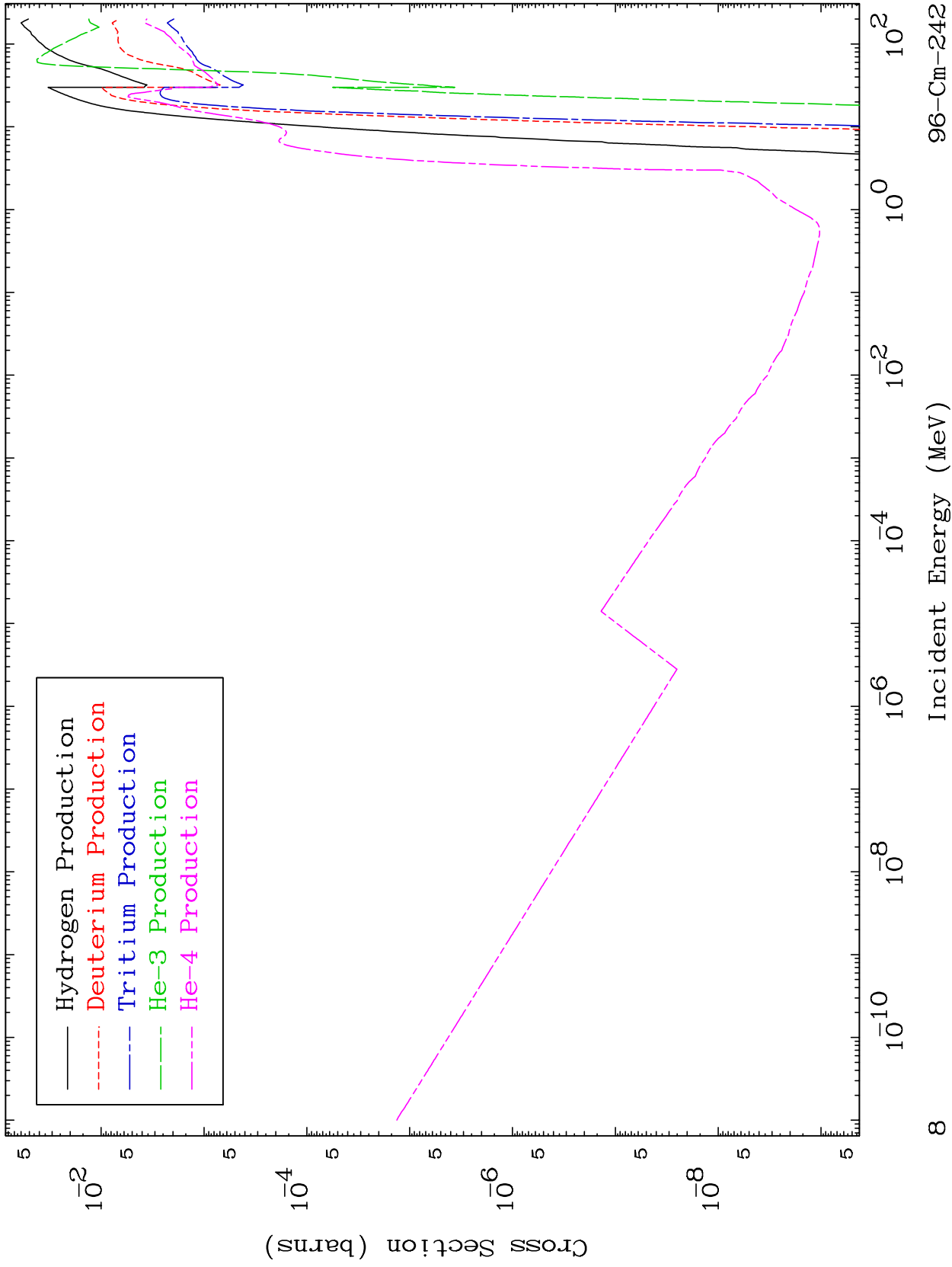




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Particle Production  
293 Kelvin Cross Sections

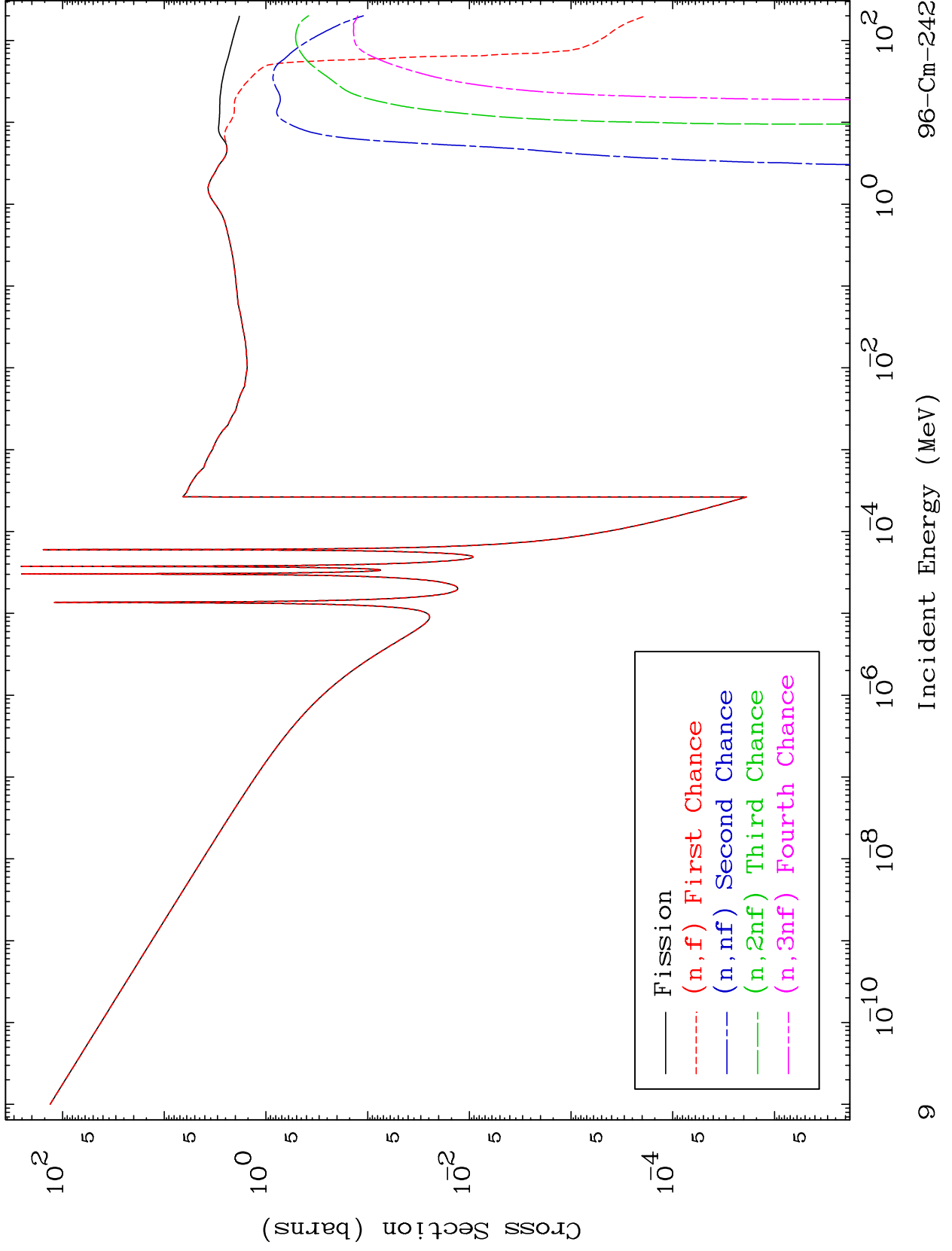
96-Cm-242



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Fission  
293 Kelvin Cross Sections

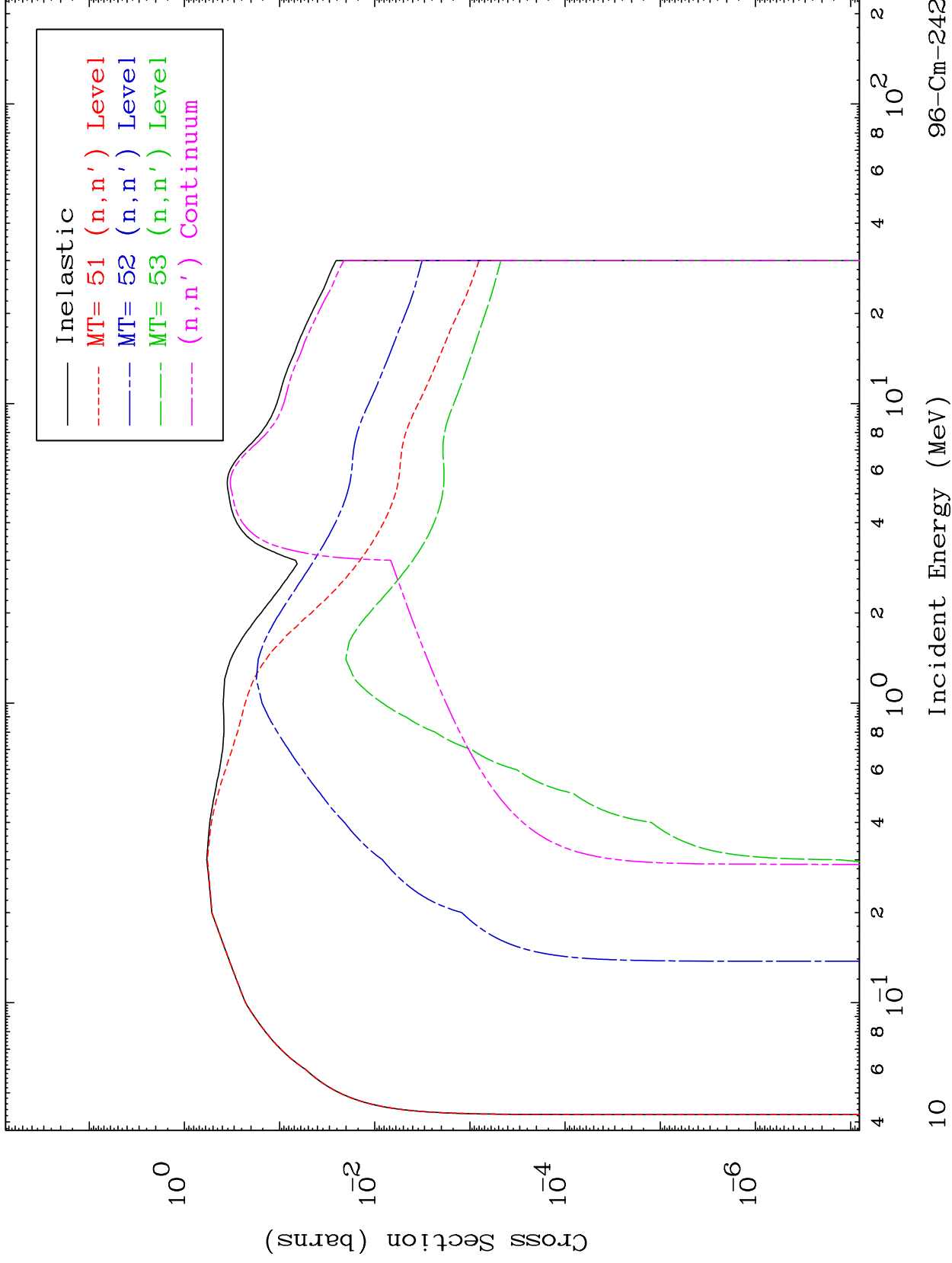
96-Cm-242



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(n,n') Levels  
293 Kelvin Cross Sections

96-Cm-242



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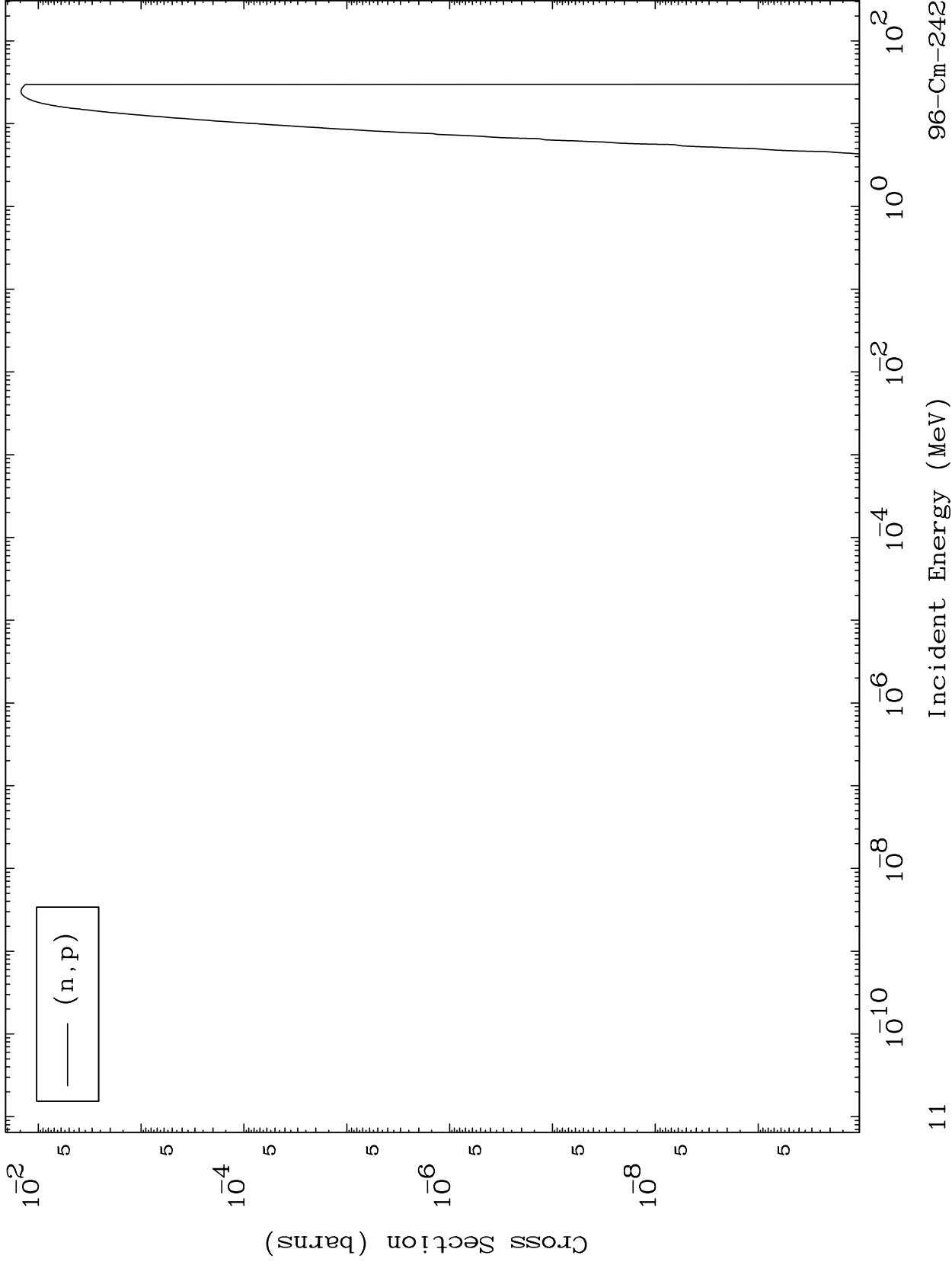
Incident Energy (MeV)

96-Cm-242

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(n,p) Levels  
293 Kelvin Cross Sections

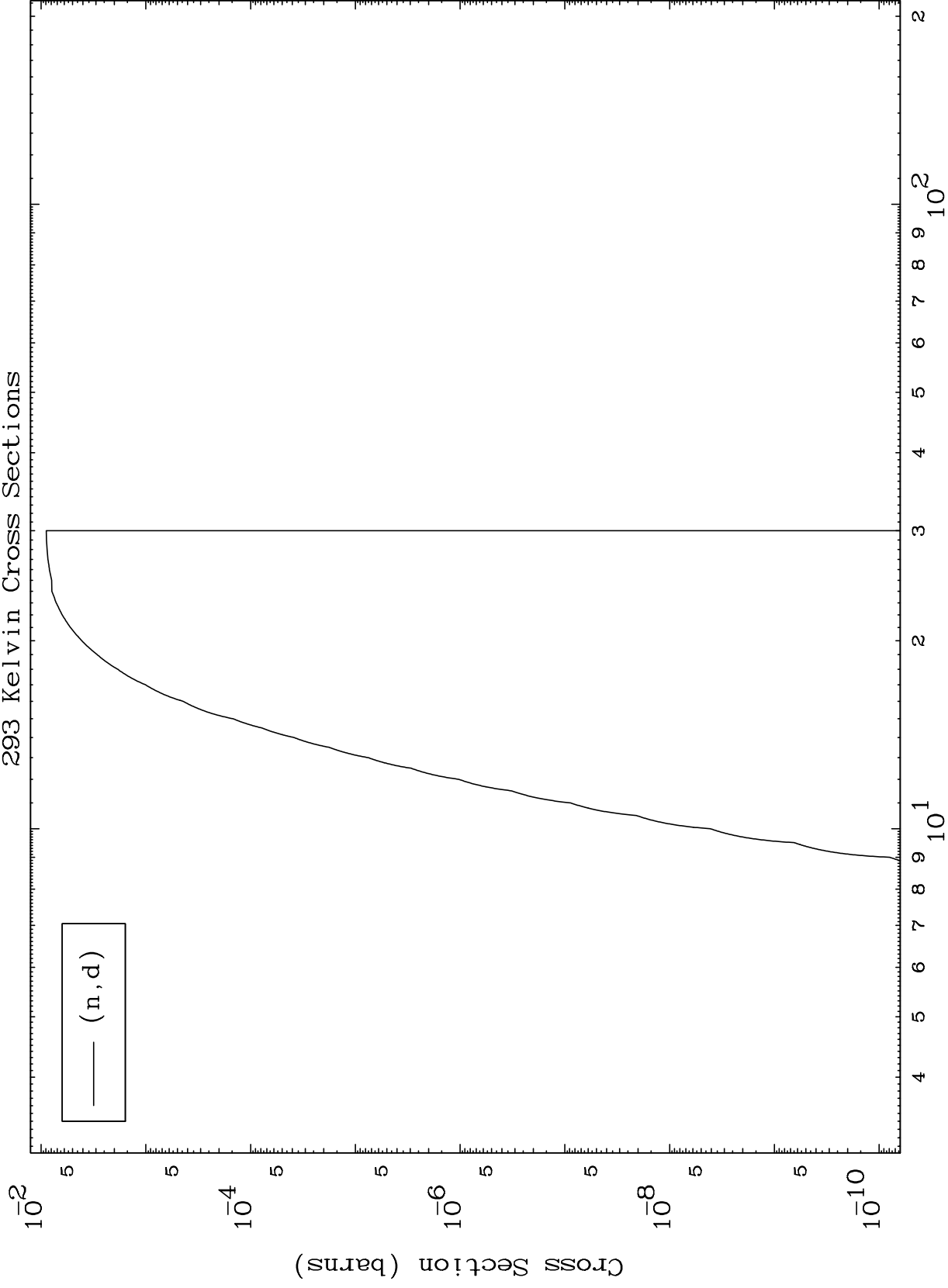
96-Cm-242



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(n,d) Levels  
293 Kelvin Cross Sections

96-Cm-242



12

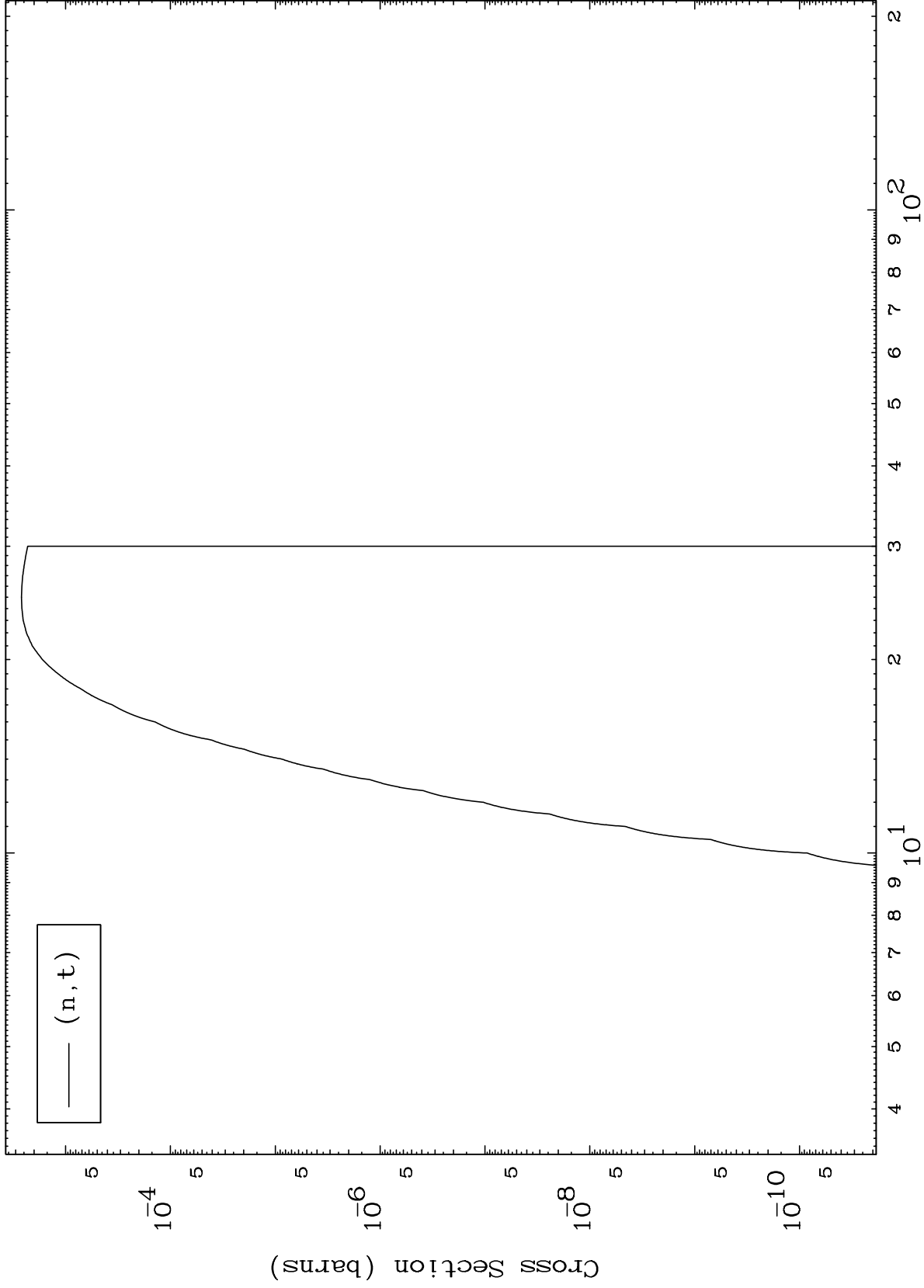
Incident Energy (MeV)

96-Cm-242

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(n,t) Levels  
293 Kelvin Cross Sections

96-Cm-242



13

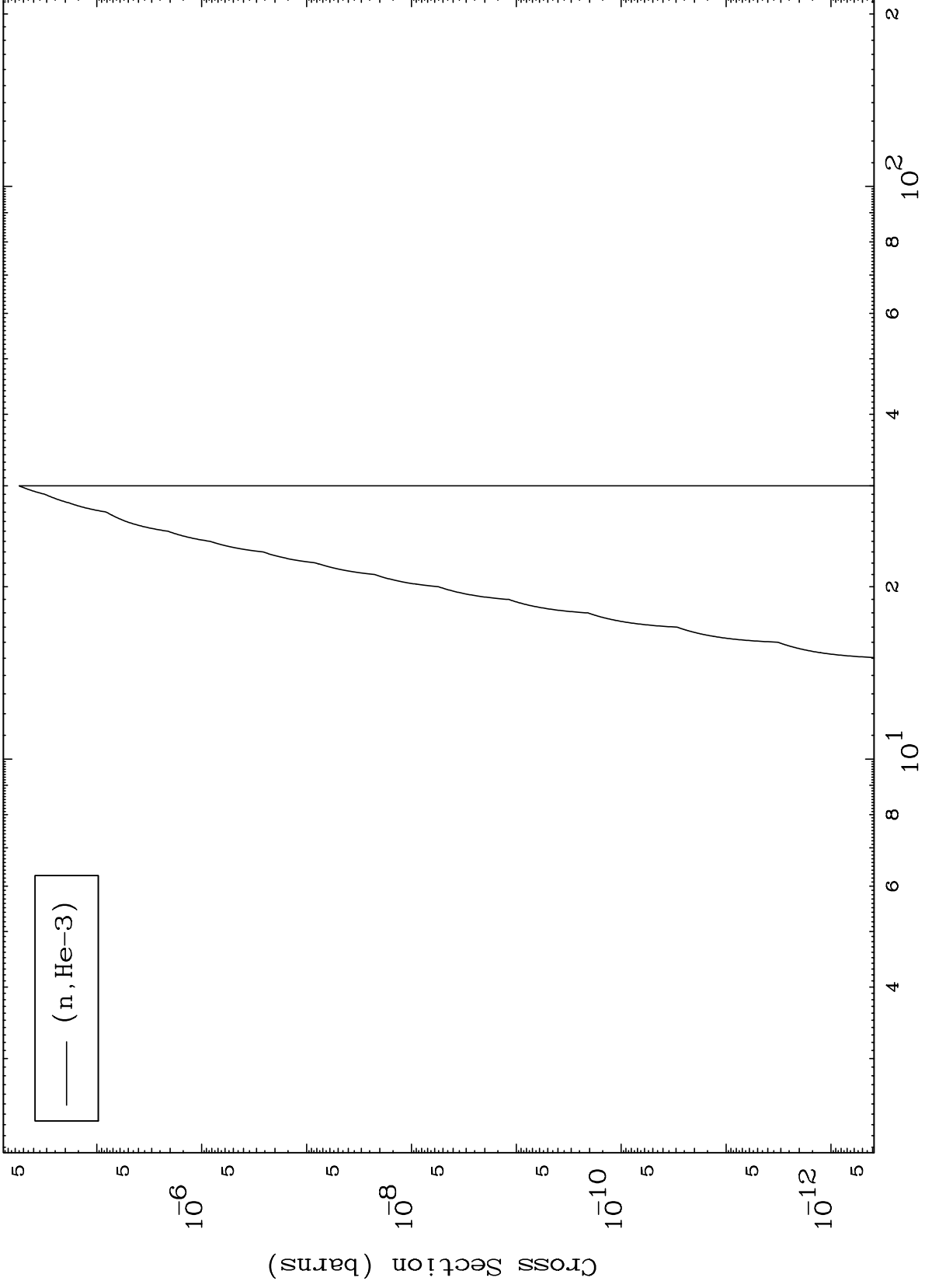
Incident Energy (MeV)

96-Cm-242

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(n,He3) Levels  
293 Kelvin Cross Sections

96-Cm-242



14

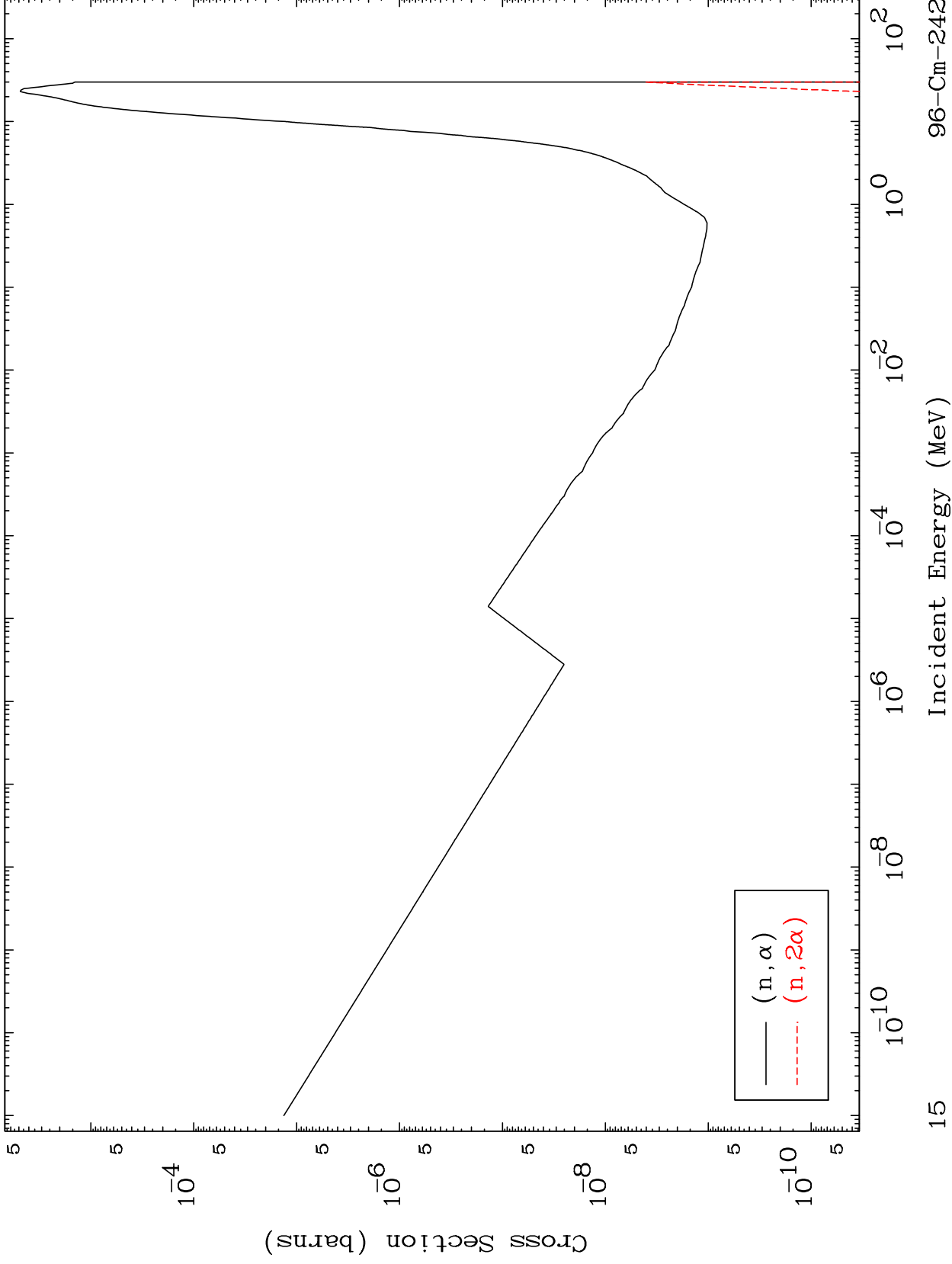
Incident Energy (MeV)

96-Cm-242

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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

96-Cm-242

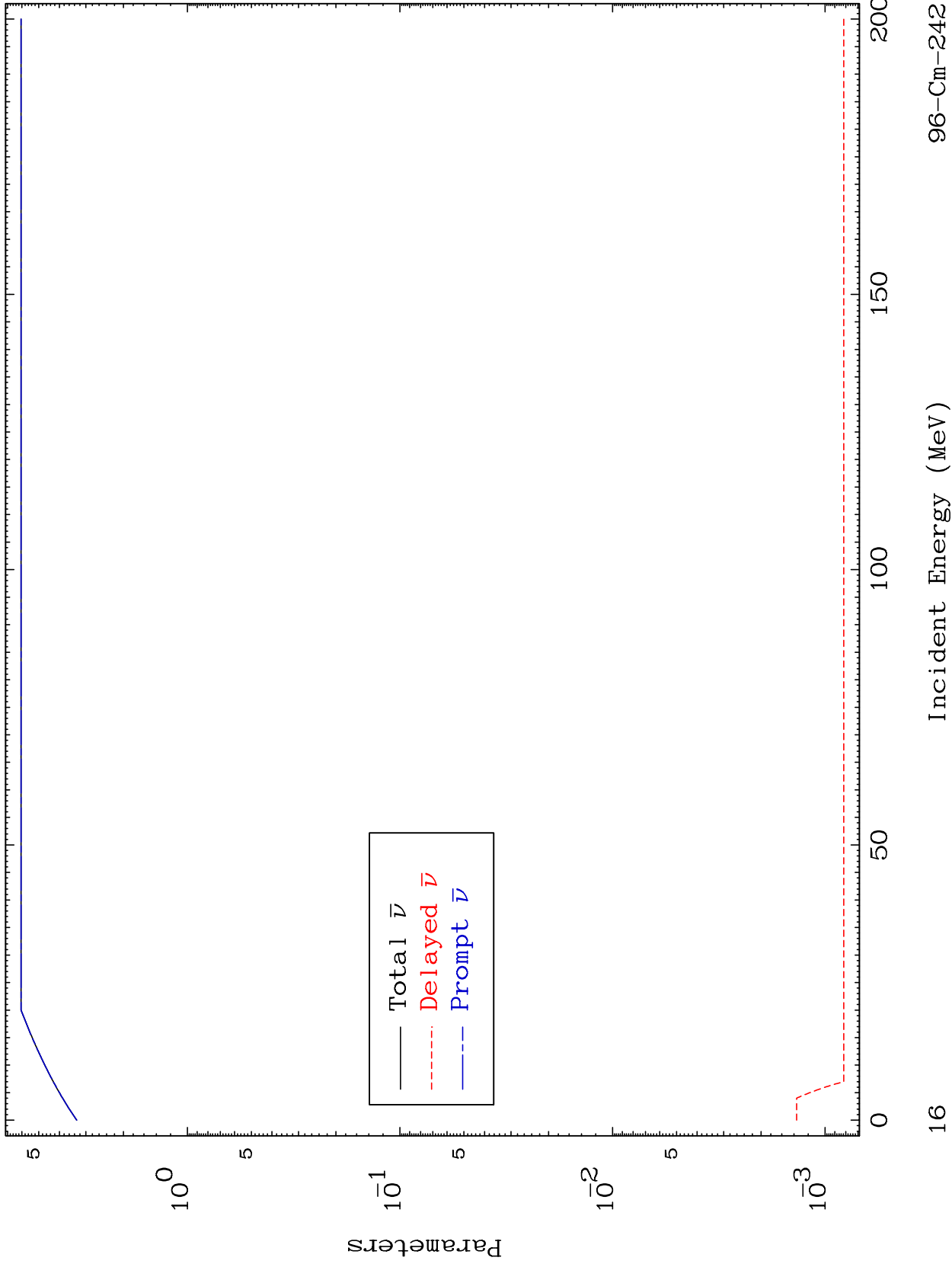




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Energy Release  
Parameters

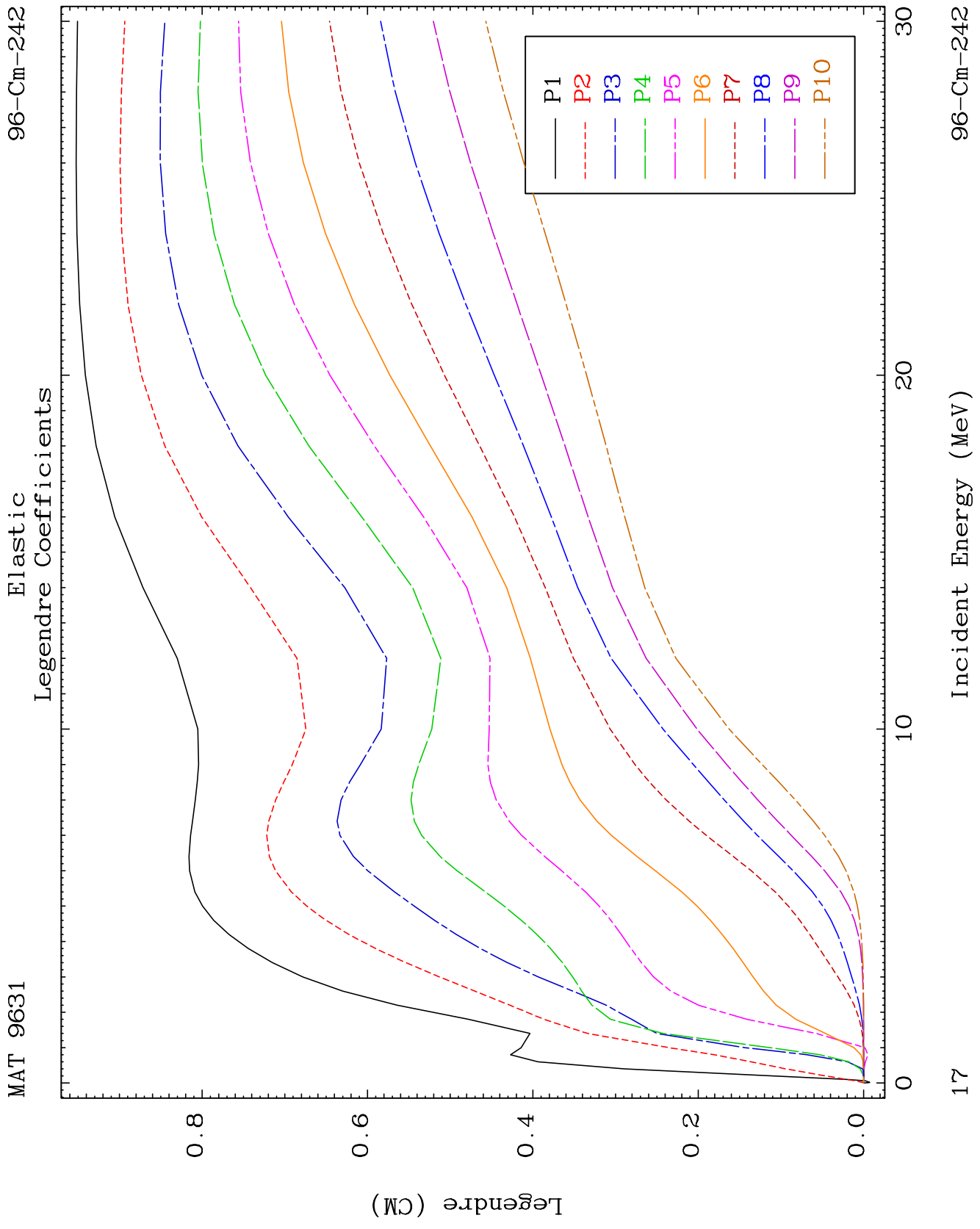
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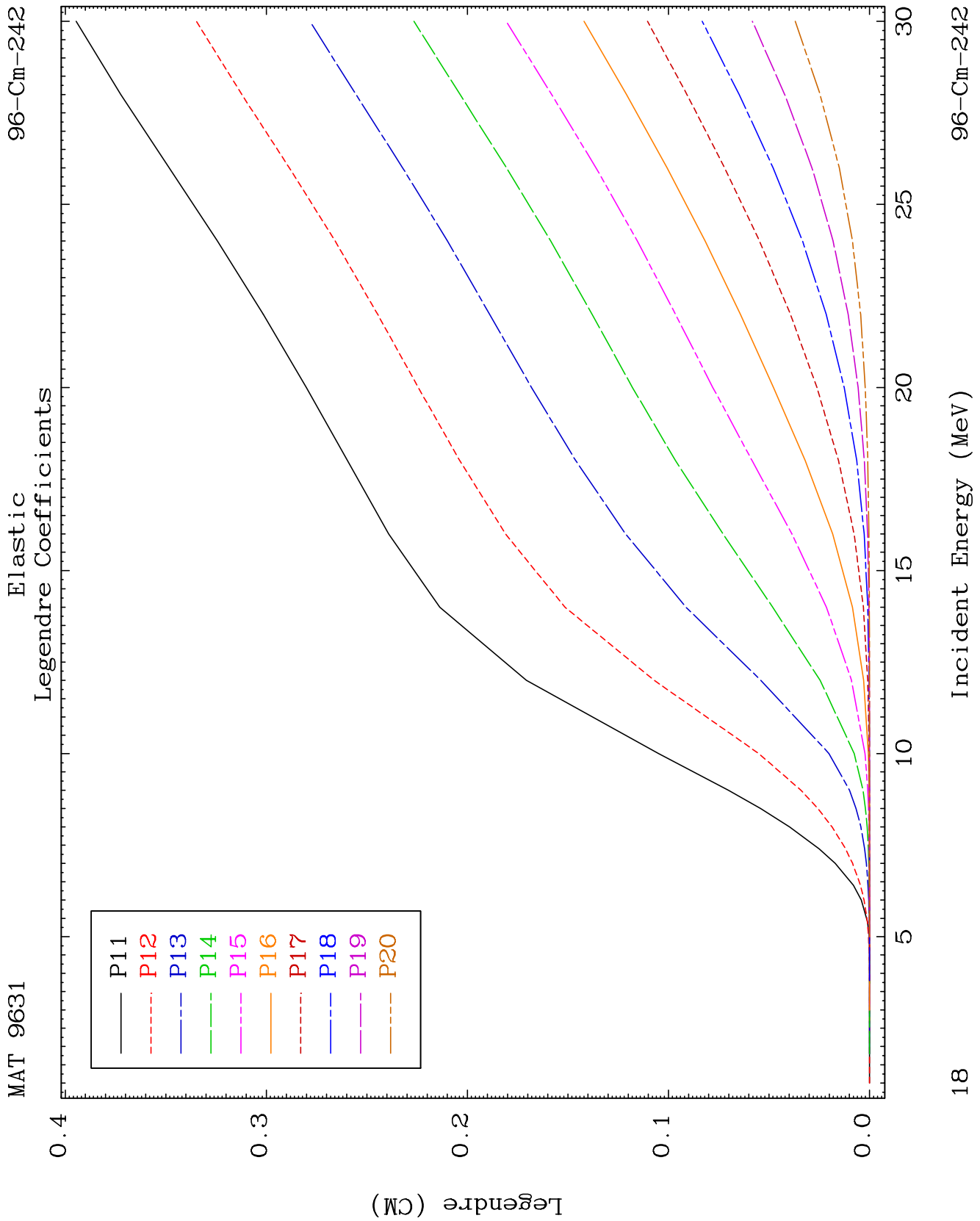


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Incident Energy (MeV)

96-Cm-242

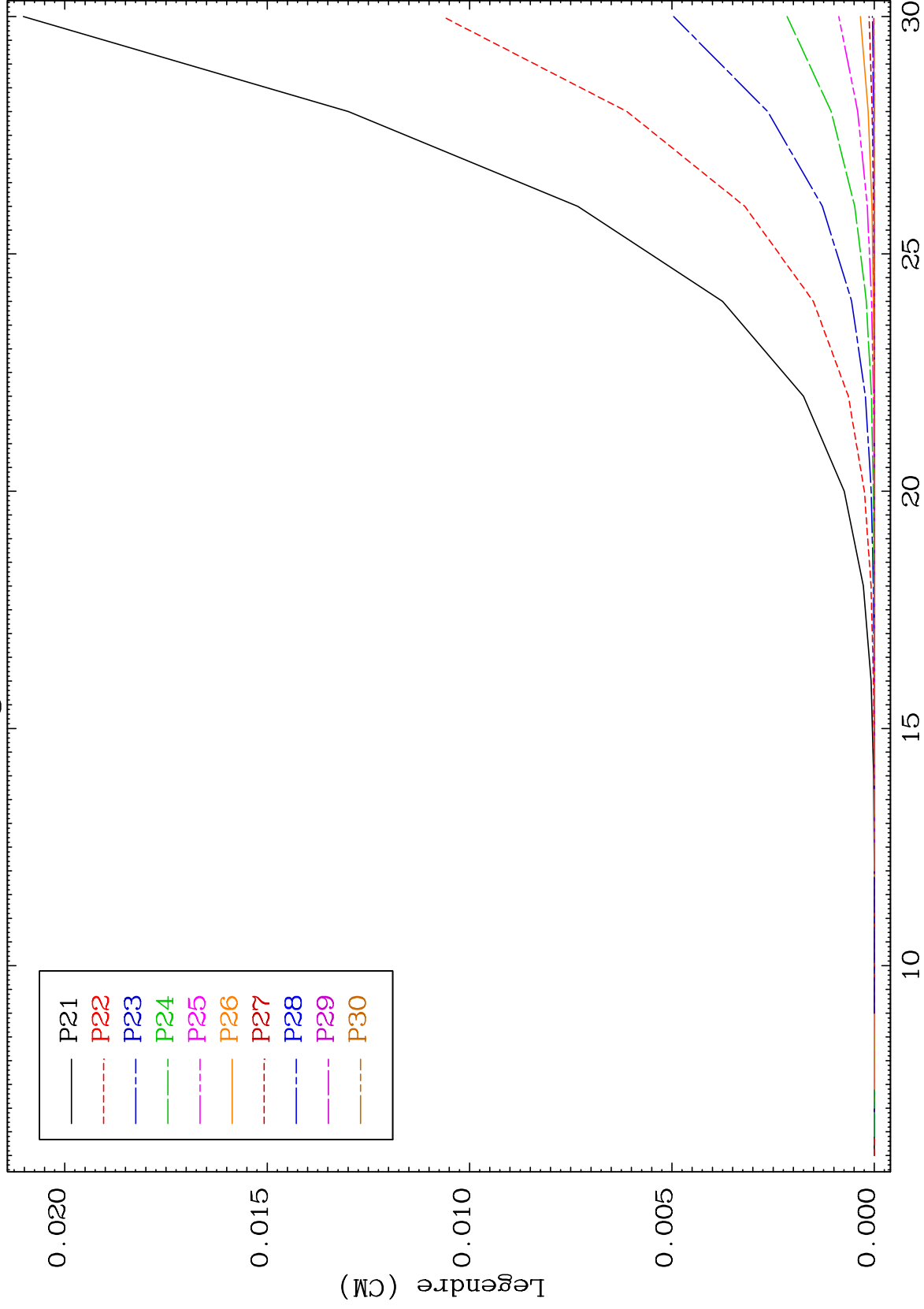




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### Elastic Legendre Coefficients

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Incident Energy (MeV)

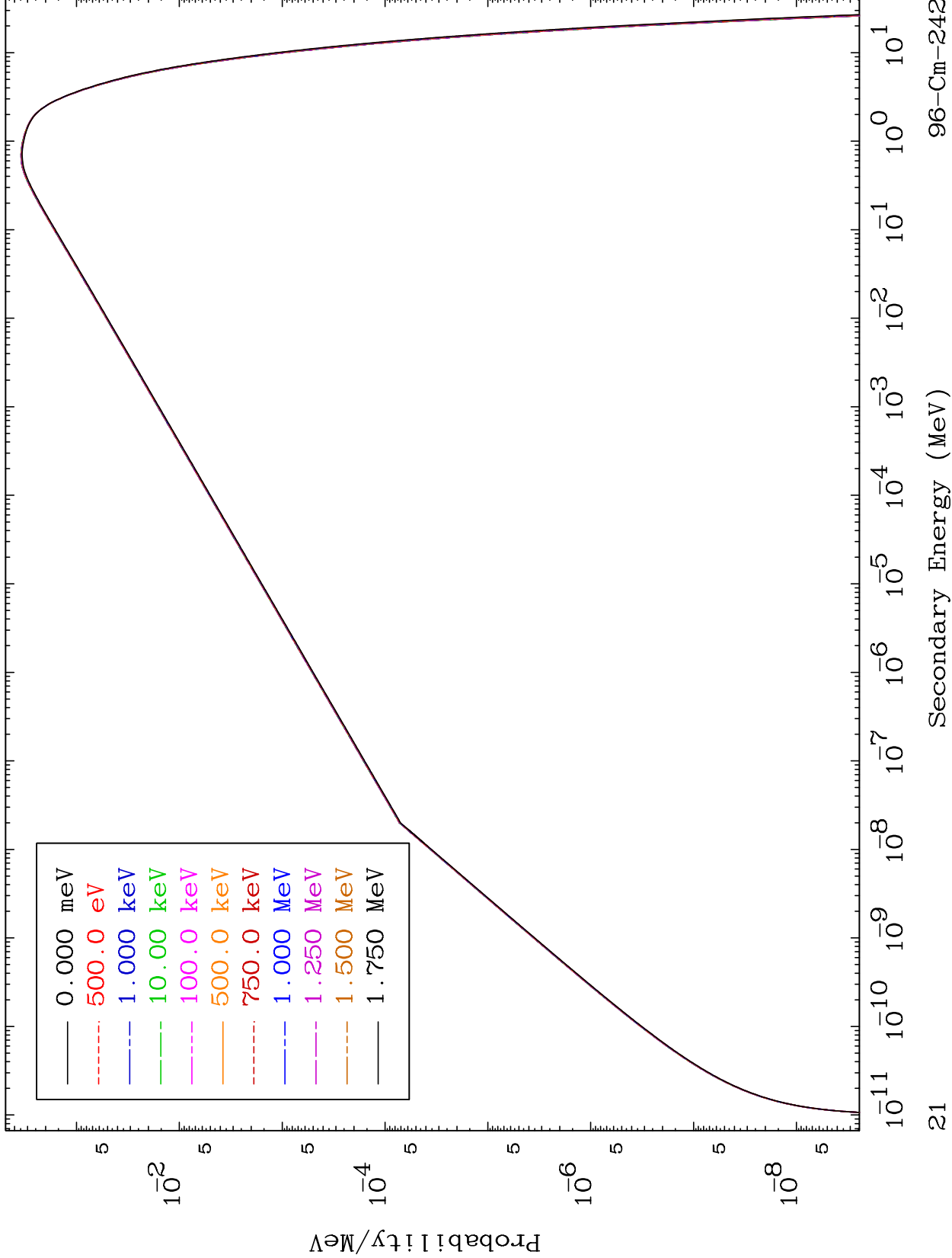
96-Cm-242



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### Fission Energy Distributions

96-Cm-242



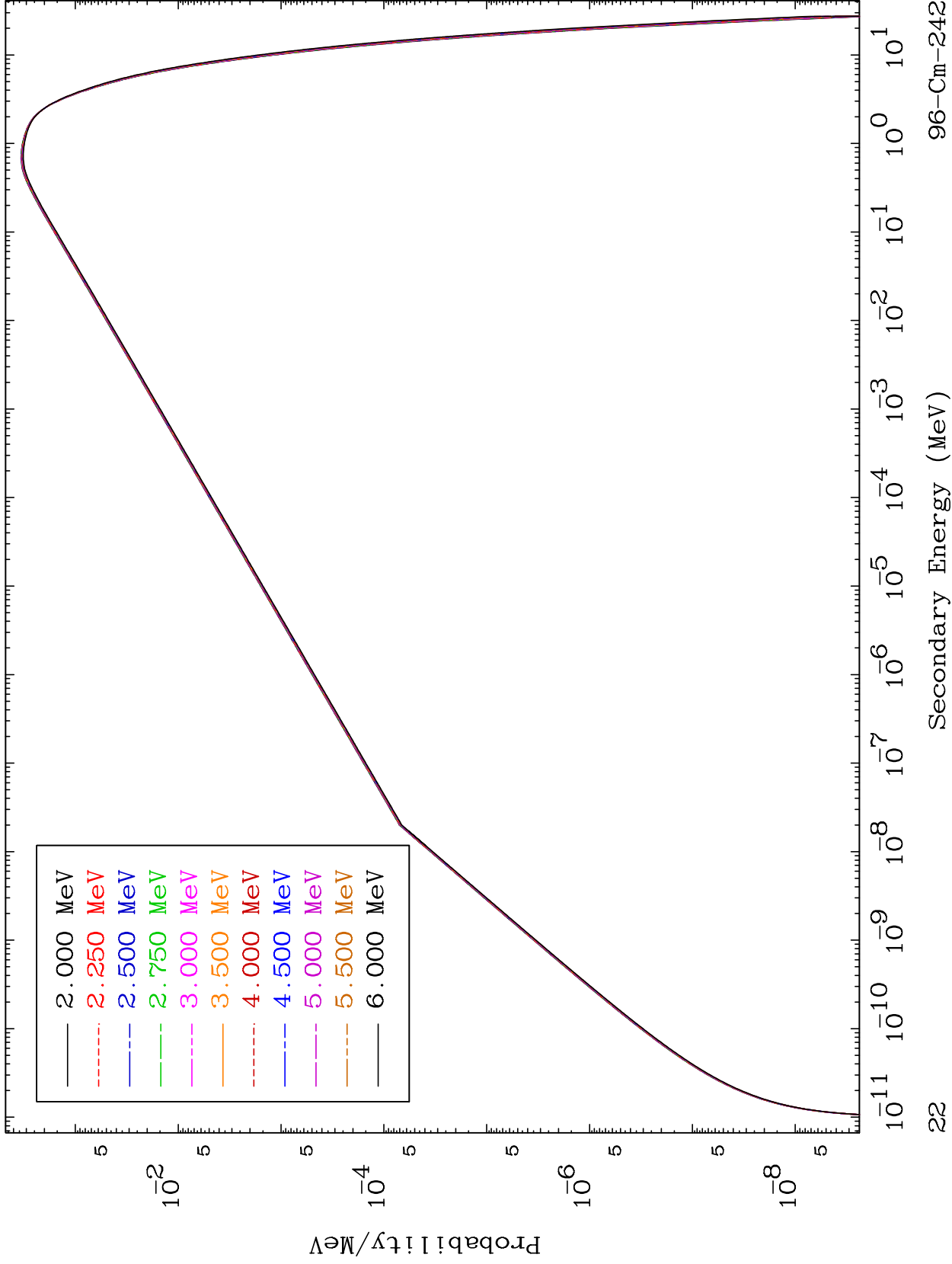
96-Cm-242

Secondary Energy (MeV)

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### Fission Energy Distributions

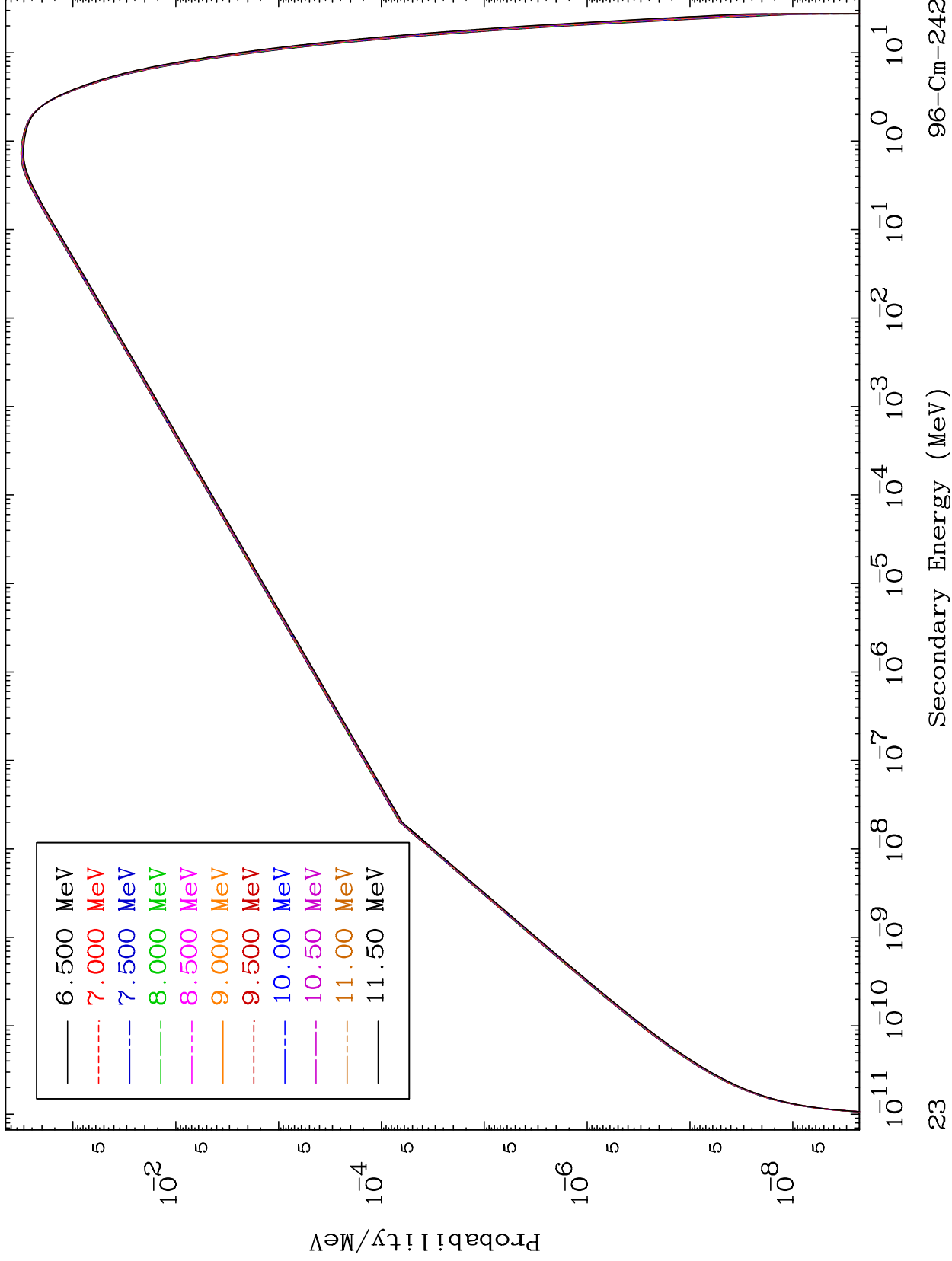
96-Cm-242



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### Fission Energy Distributions

96-Cm-242



96-Cm-242

Secondary Energy (MeV)

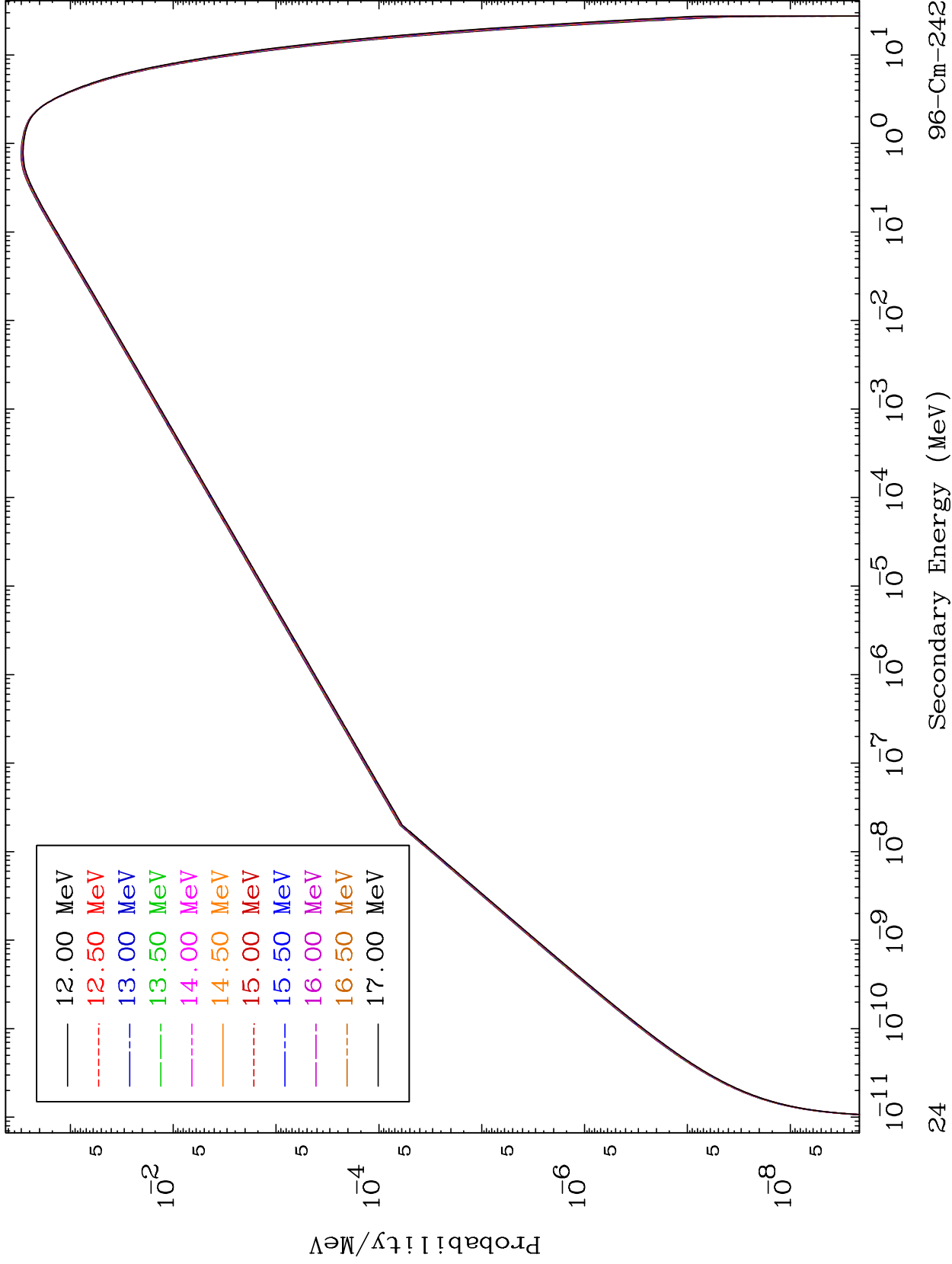
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### Fission Energy Distributions

96-Cm-242



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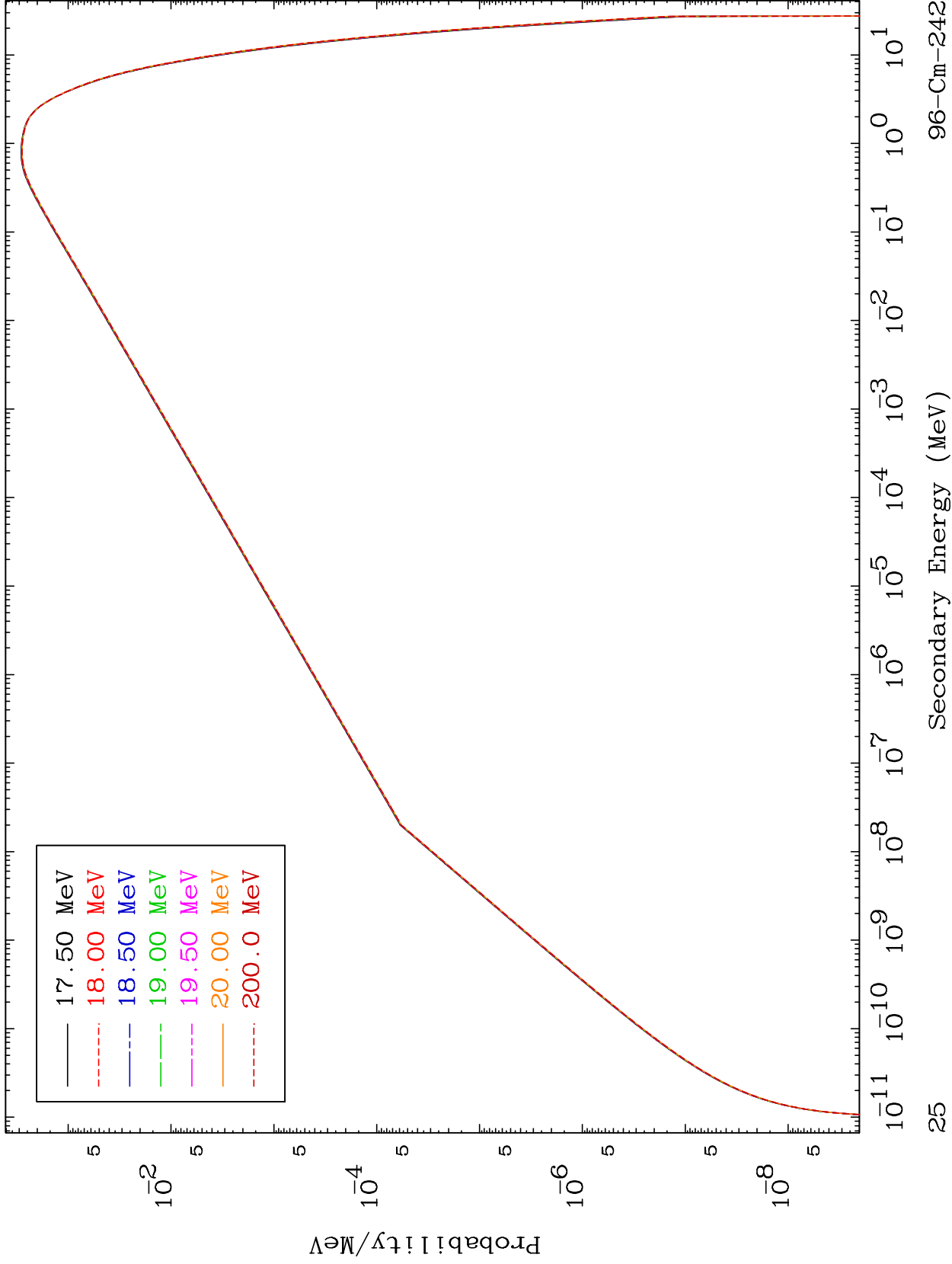
Secondary Energy (MeV)

96-Cm-242

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### Fission Energy Distributions

96-Cm-242

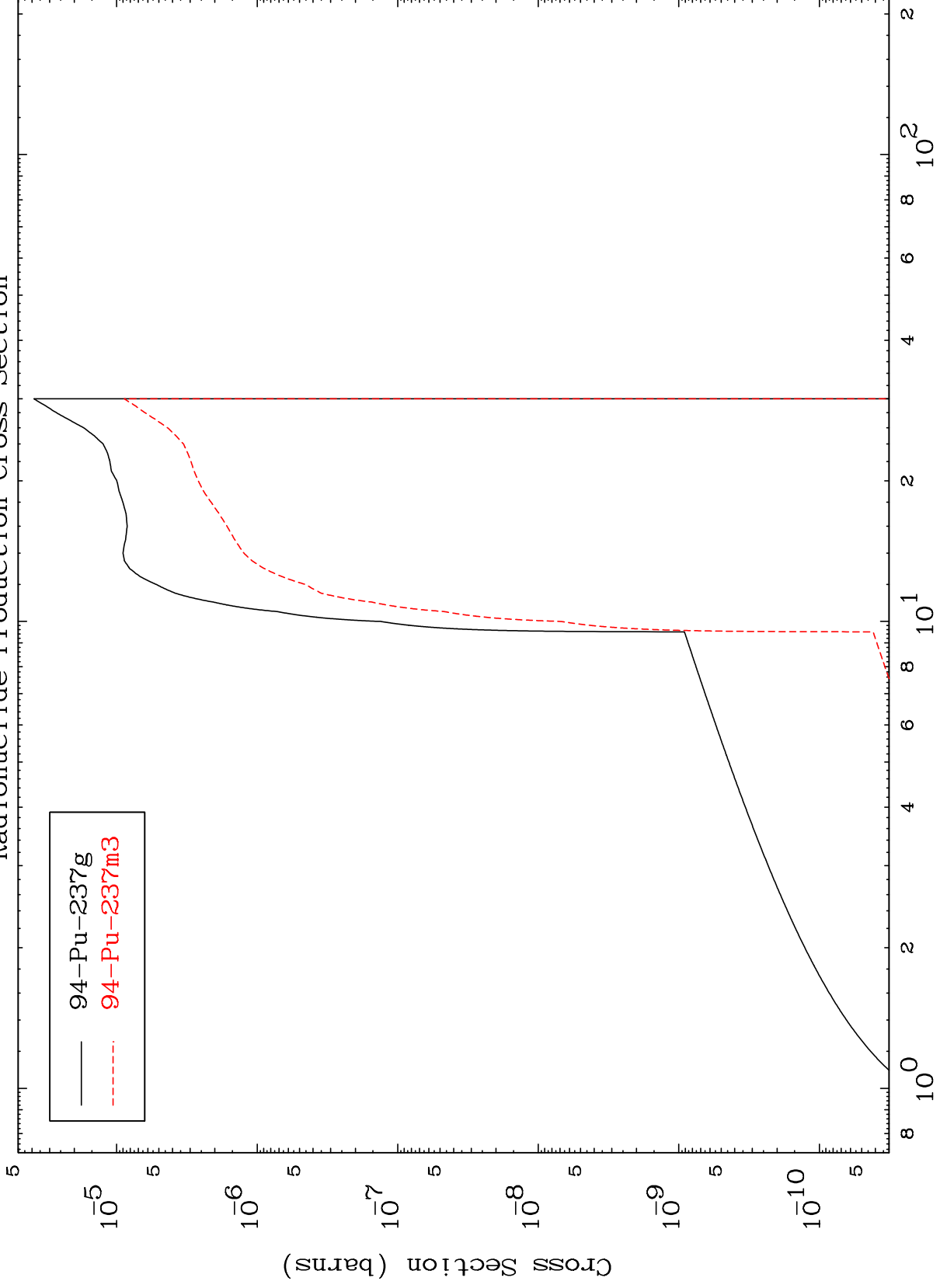


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(n,2n)  $\alpha$

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Radionuclide Production Cross Section



26

Incident Energy (MeV)

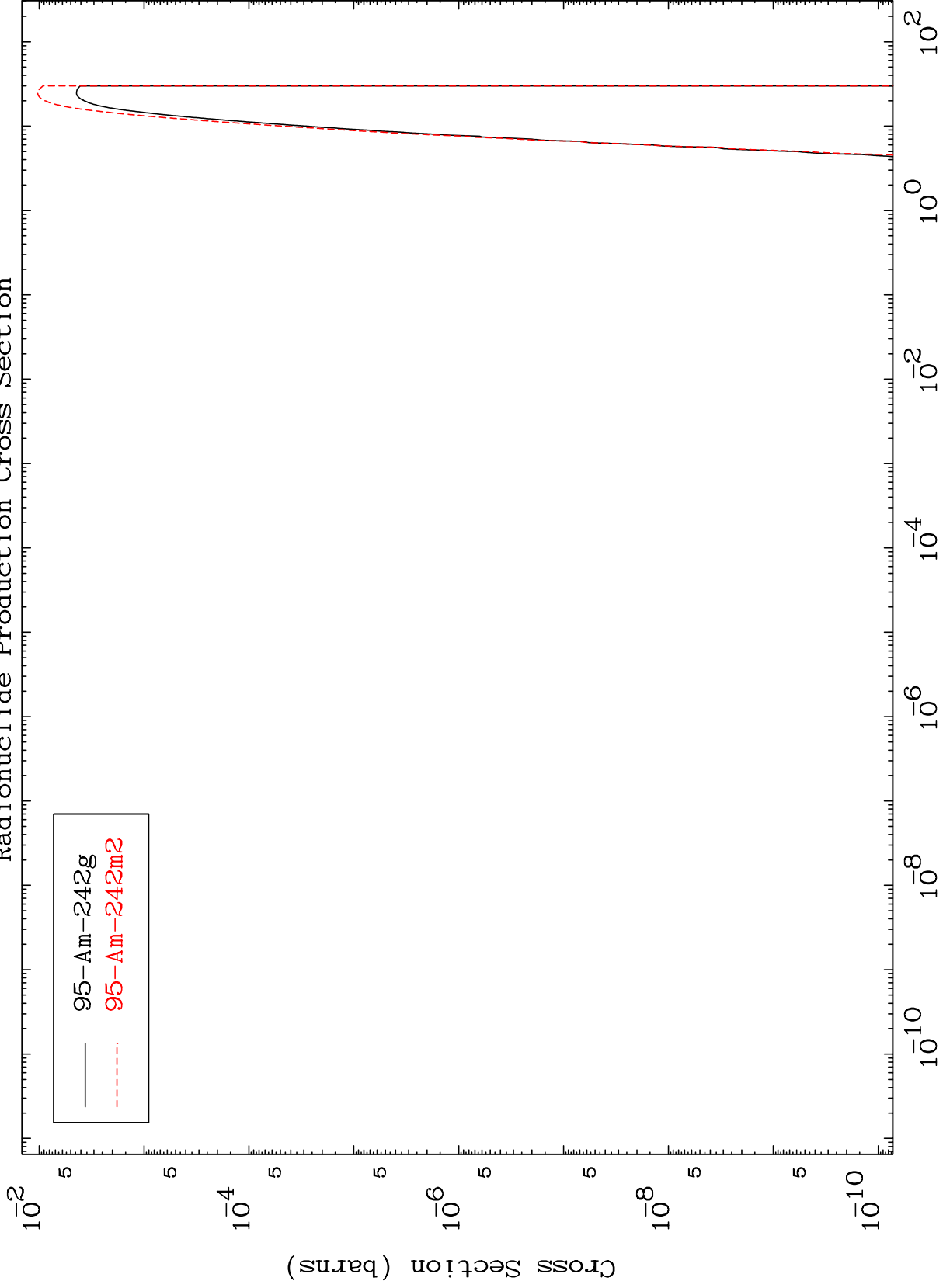
96-Cm-242

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(n,p)

96-Cm-242

Radionuclide Production Cross Section



Incident Energy (MeV)

96-Cm-242

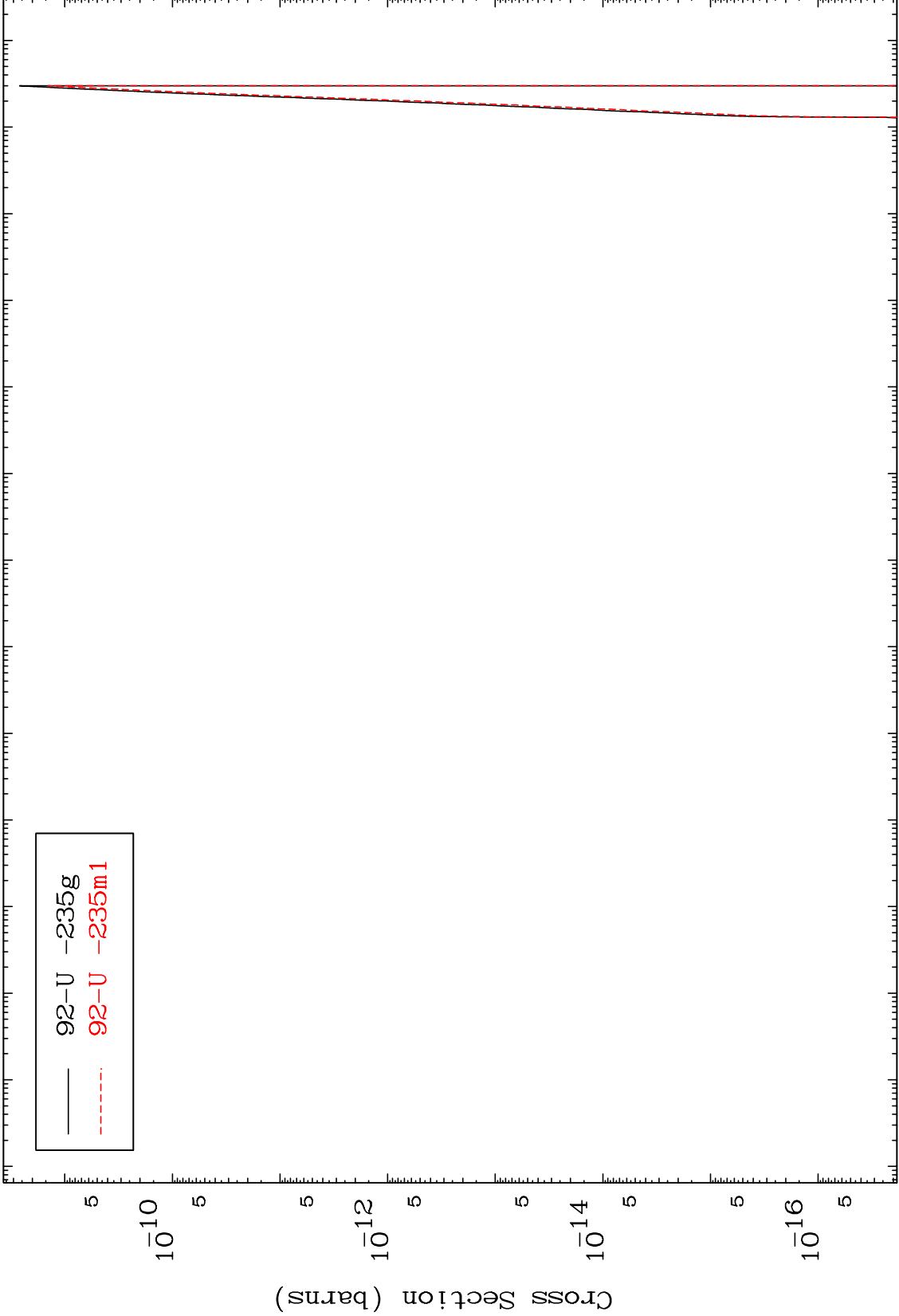
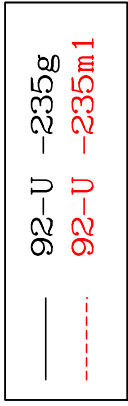
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(n,2α)

96-Cm-242

Radionuclide Production Cross Section



96-Cm-242

10<sup>0</sup>

10<sup>-2</sup>

10<sup>-4</sup>

10<sup>-6</sup>

10<sup>-8</sup>

10<sup>-10</sup>

10<sup>-16</sup>

10<sup>-10</sup>

Incident Energy (MeV)

96-Cm-242