

Program EVALPLOT  
(Version 2021-1)

by

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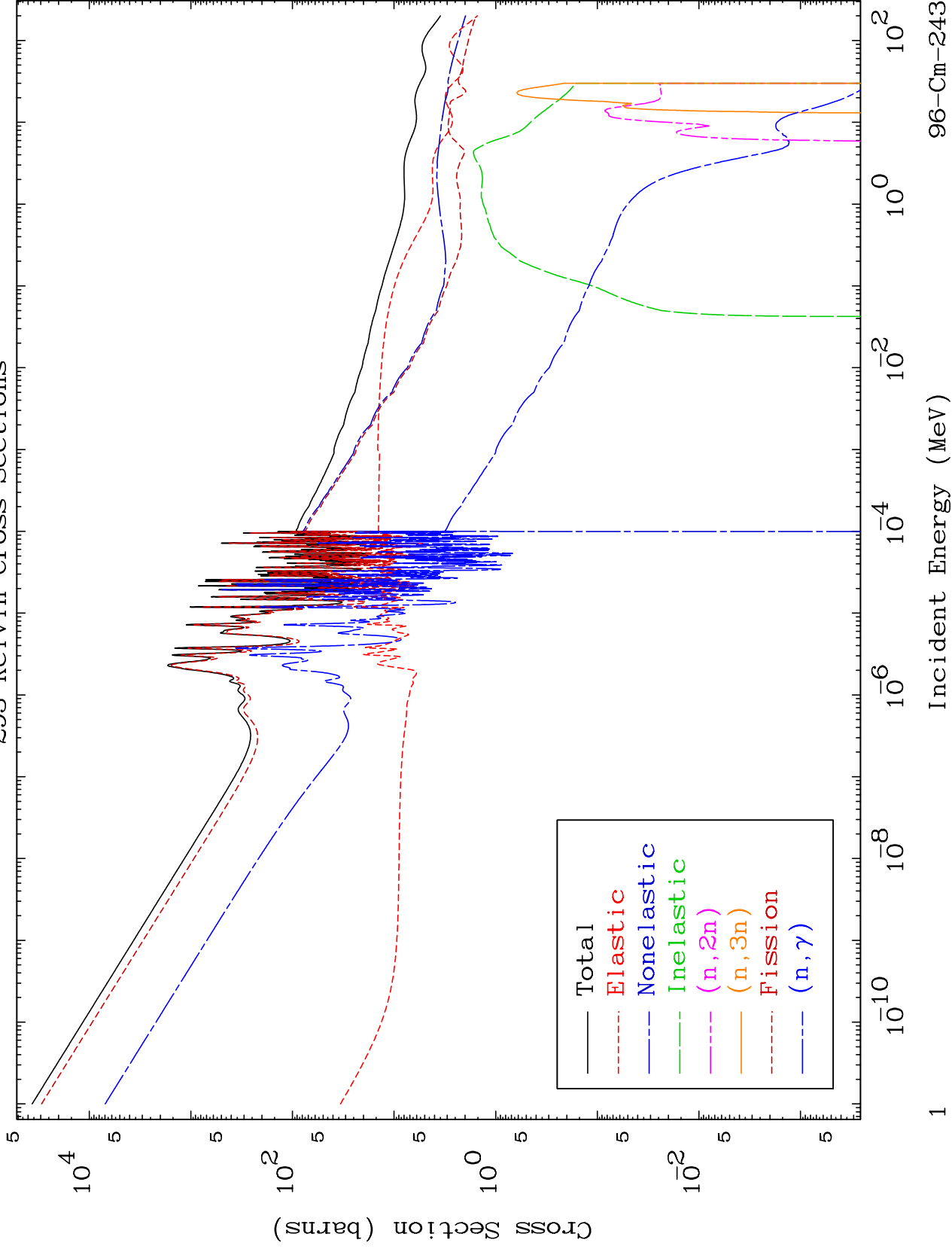
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Press Mouse Button to Start

MAT 9634

Neutron Major  
293 Kelvin Cross Sections

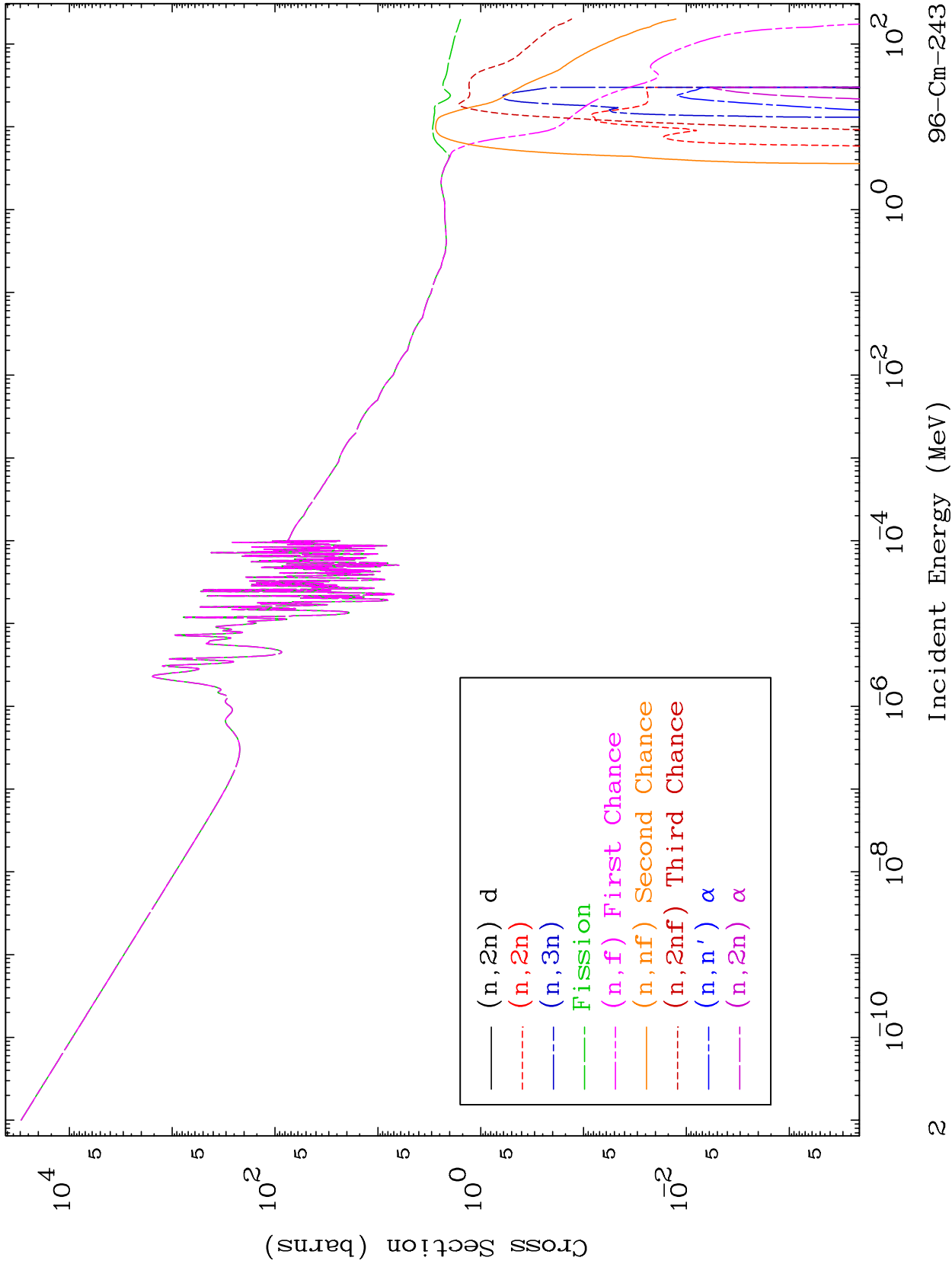
96-Cm-243



MAT 9634

Neutron Absorption  
293 Kelvin Cross Sections

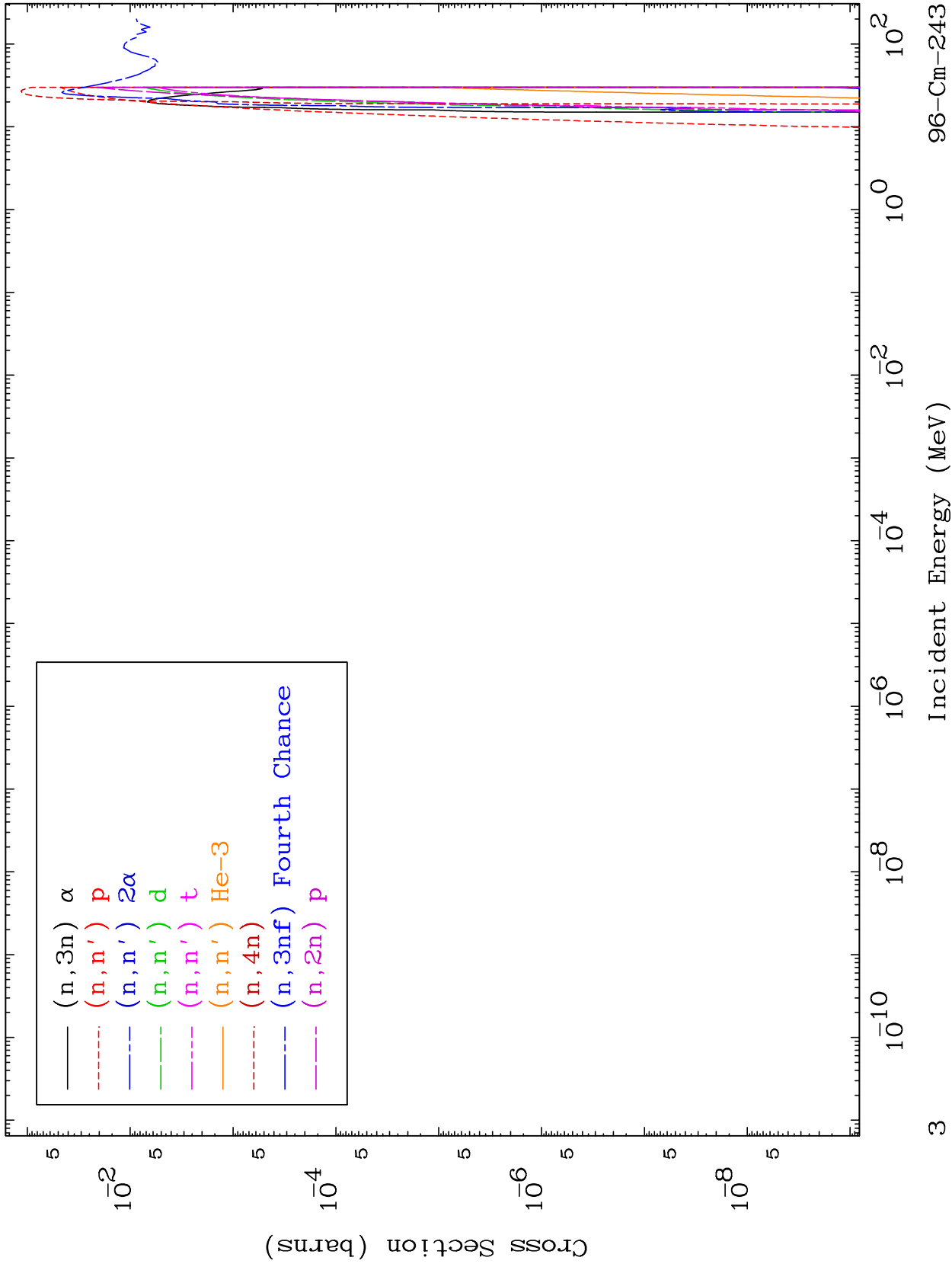
96-Cm-243



MAT 9634

Neutron Absorption  
293 Kelvin Cross Sections

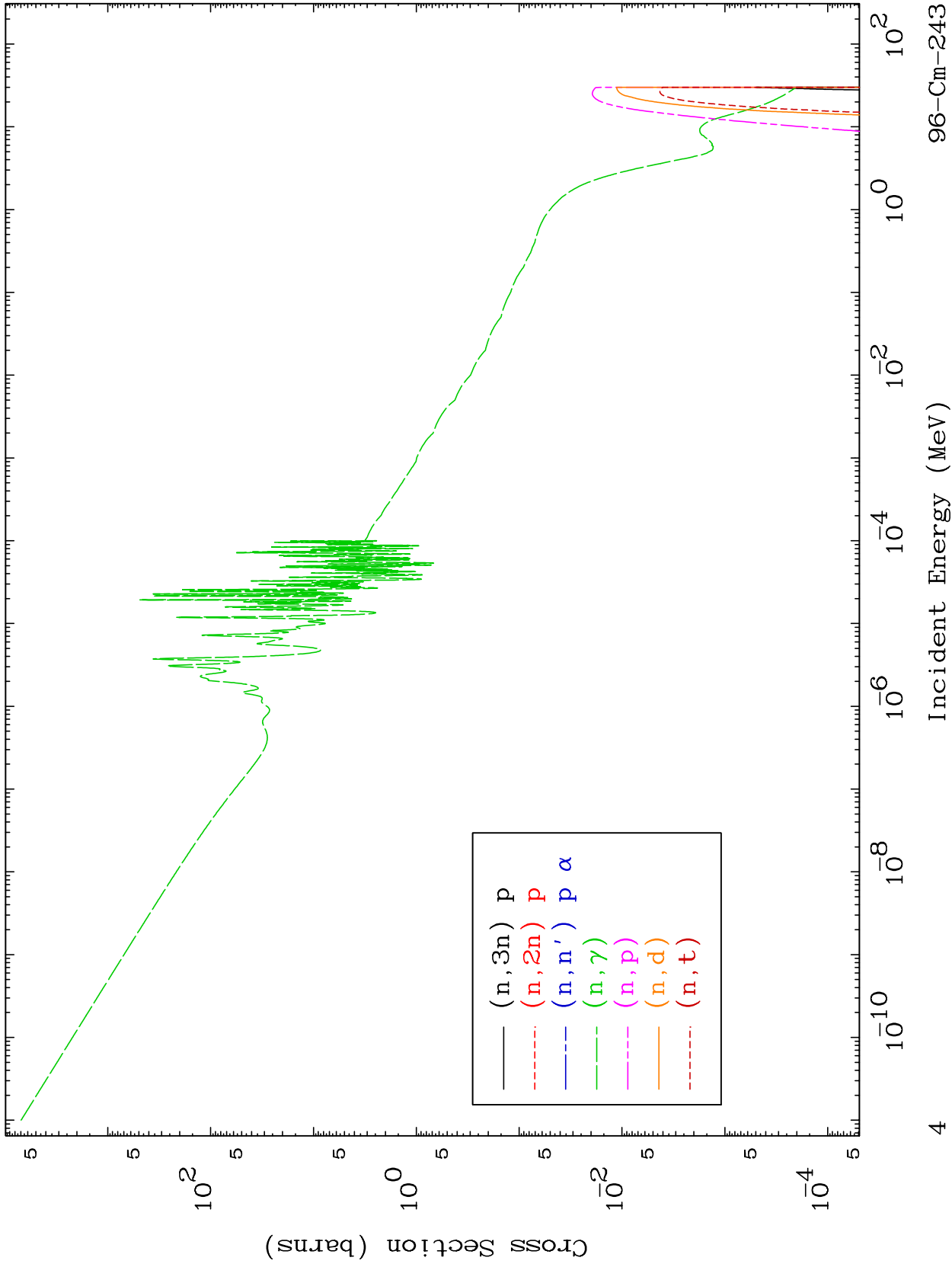
96-Cm-243



MAT 9634

Neutron Absorption  
293 Kelvin Cross Sections

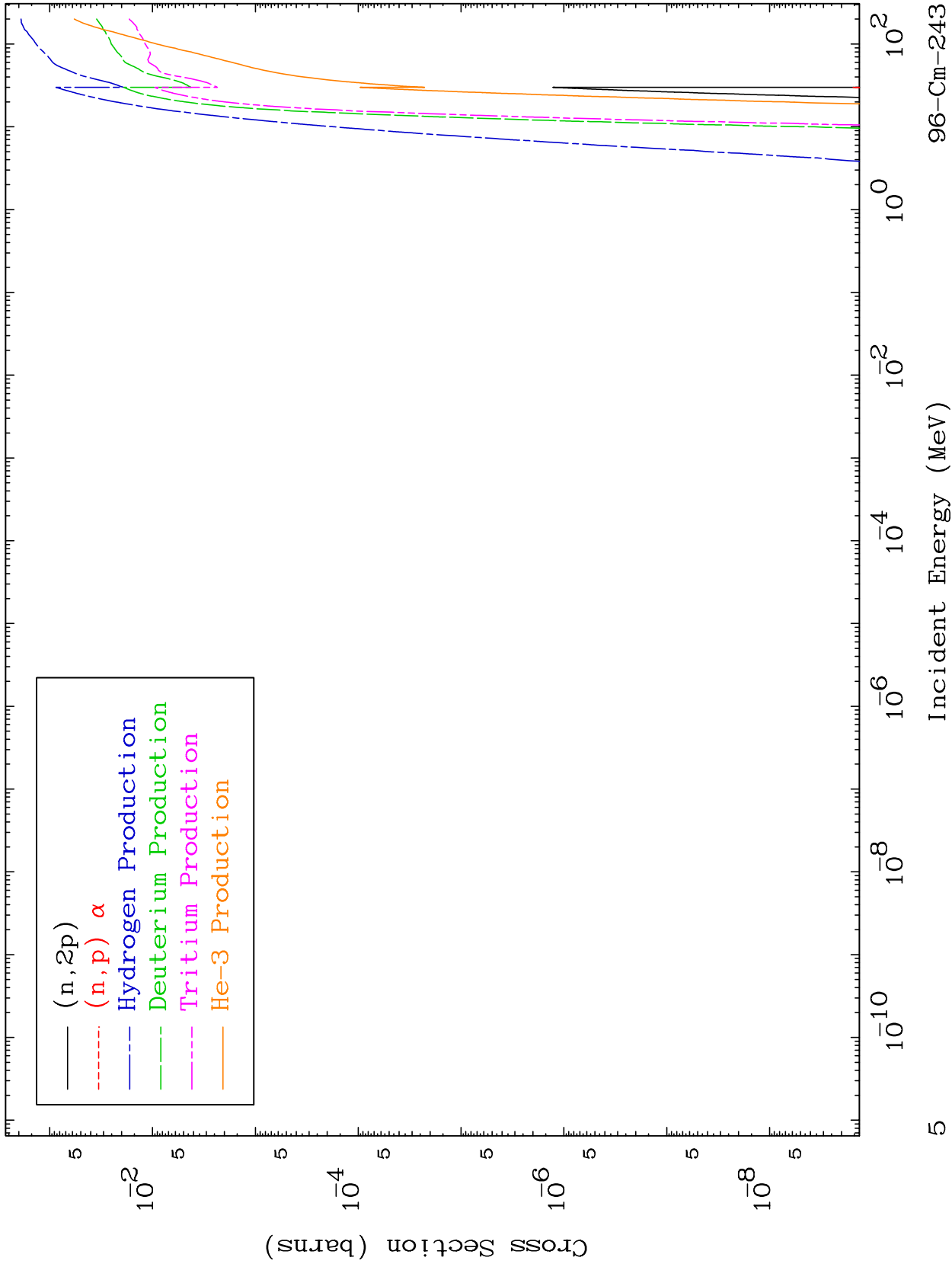
96-Cm-243



MAT 9634

Neutron Absorption  
293 Kelvin Cross Sections

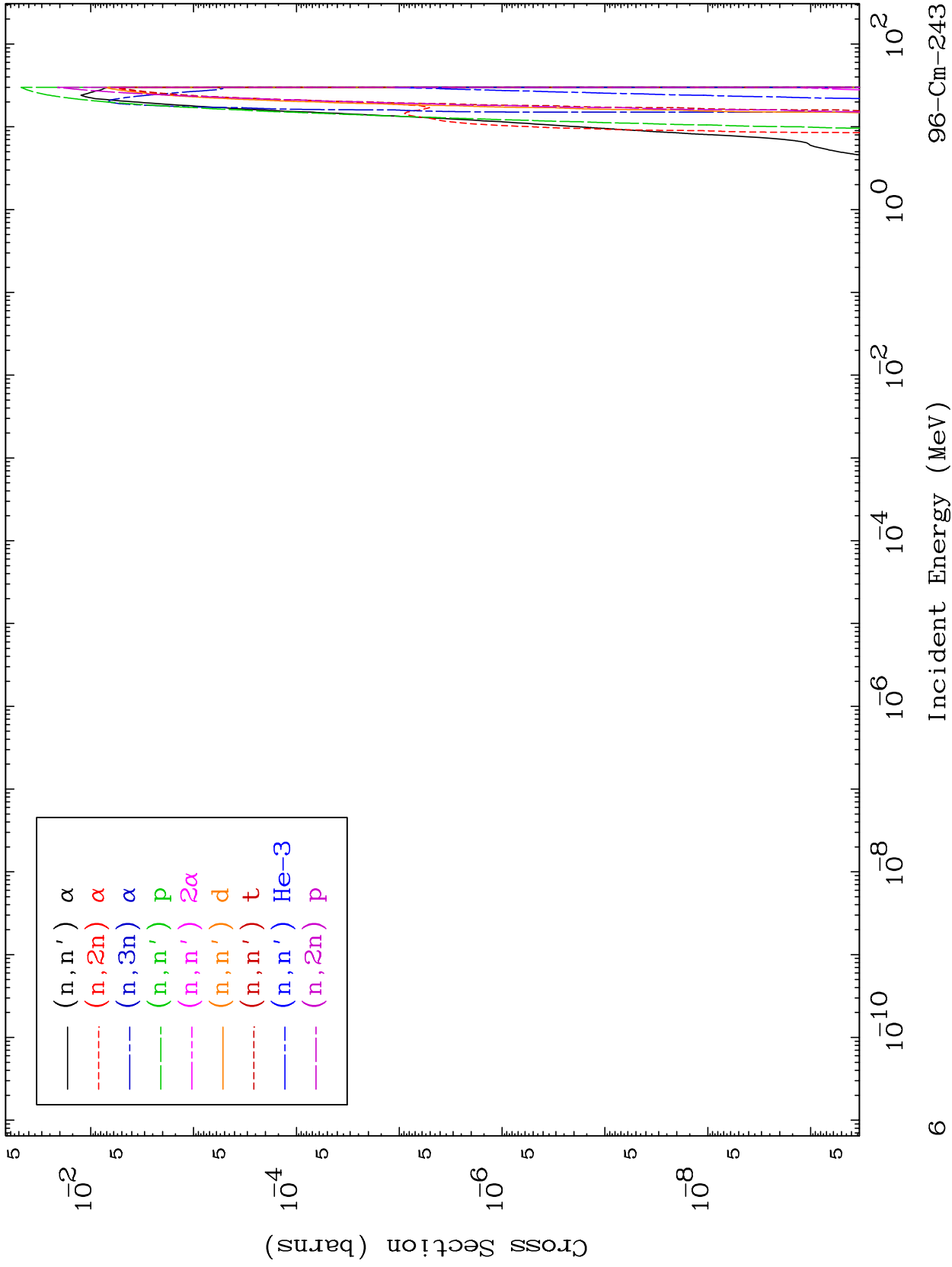
96-Cm-243



MAT 9634

Charged Particle  
293 Kelvin Cross Sections

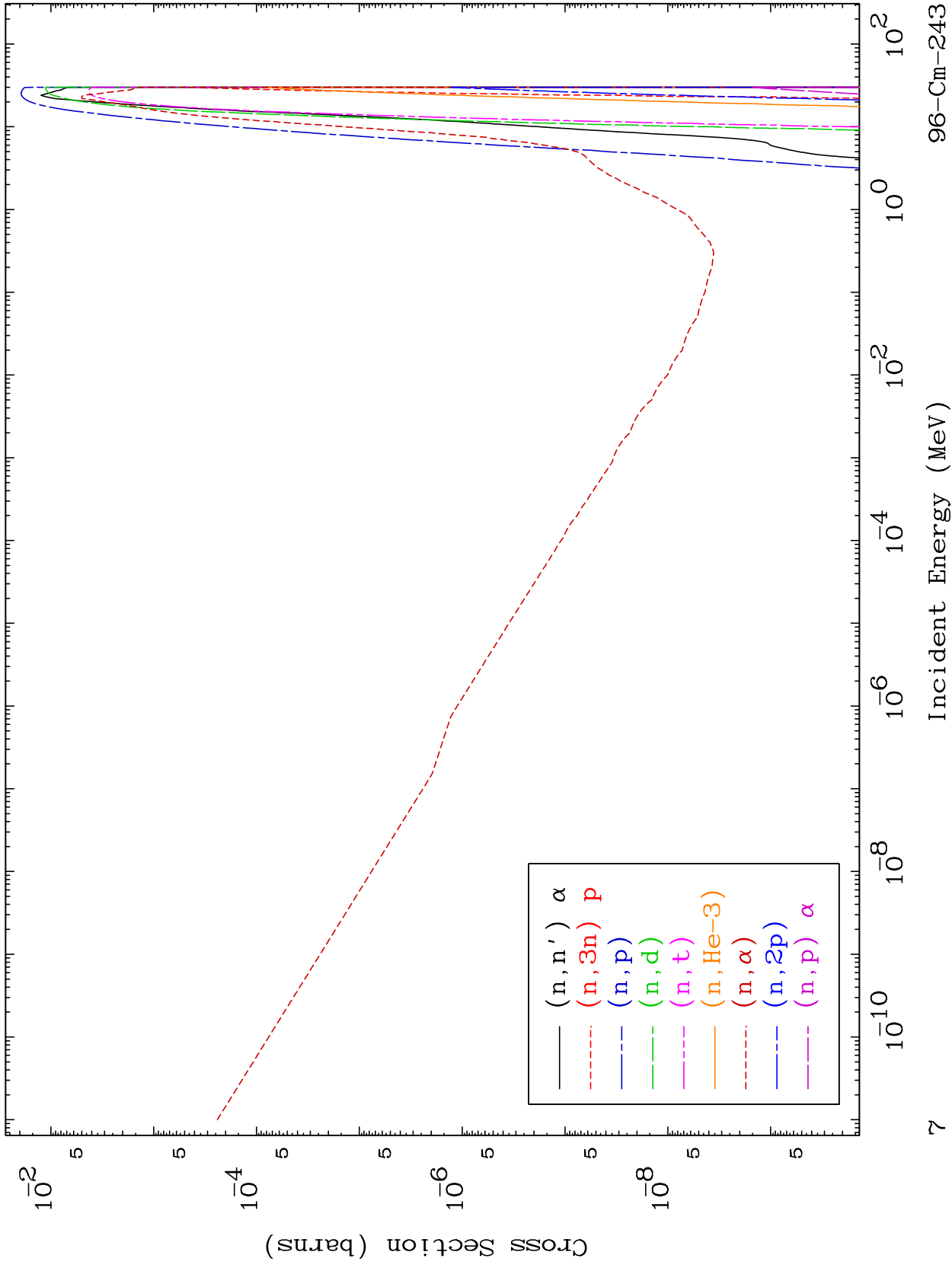
96-Cm-243



MAT 9634

Charged Particle  
293 Kelvin Cross Sections

96-Cm-243

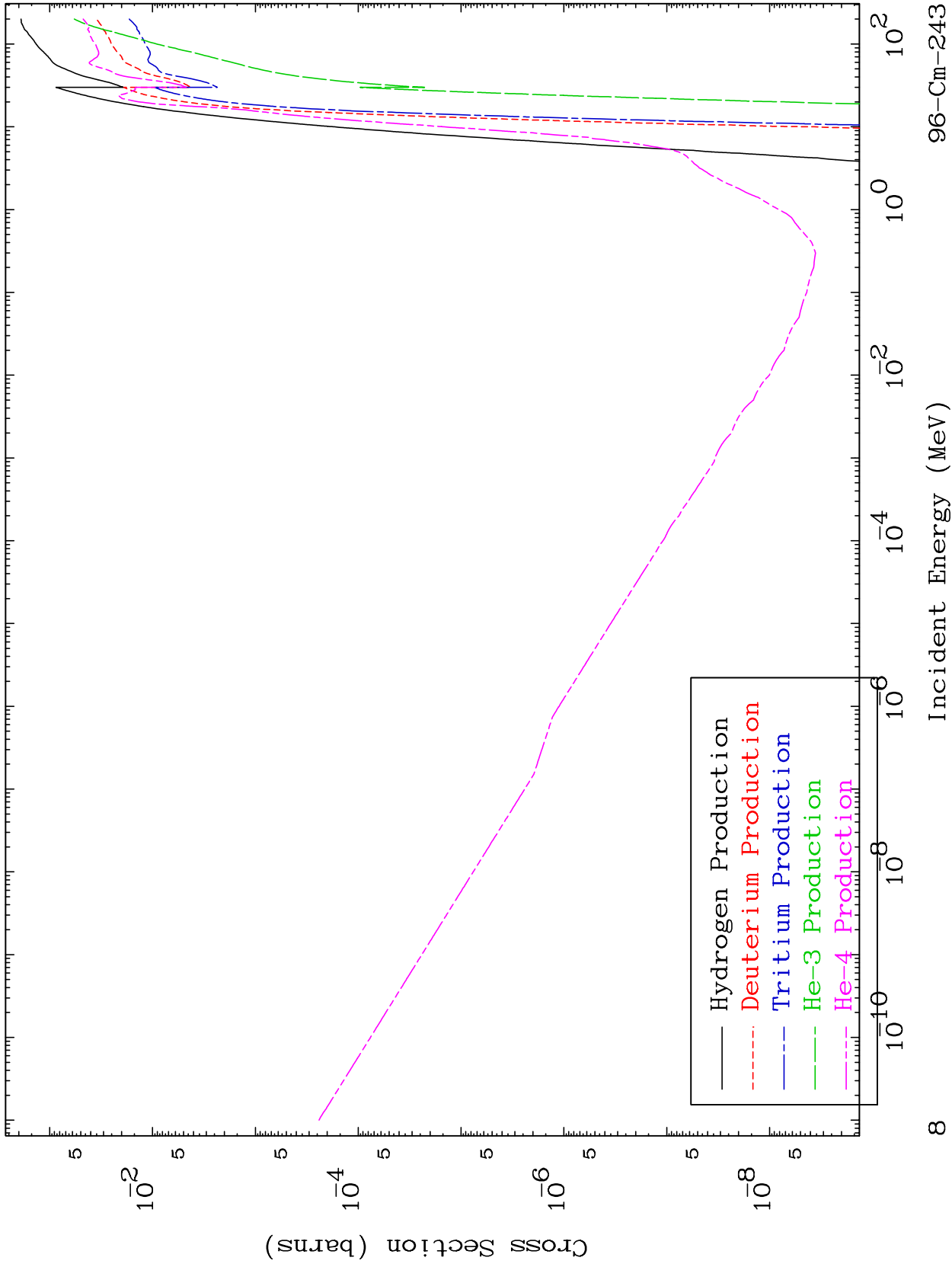




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Particle Production  
293 Kelvin Cross Sections

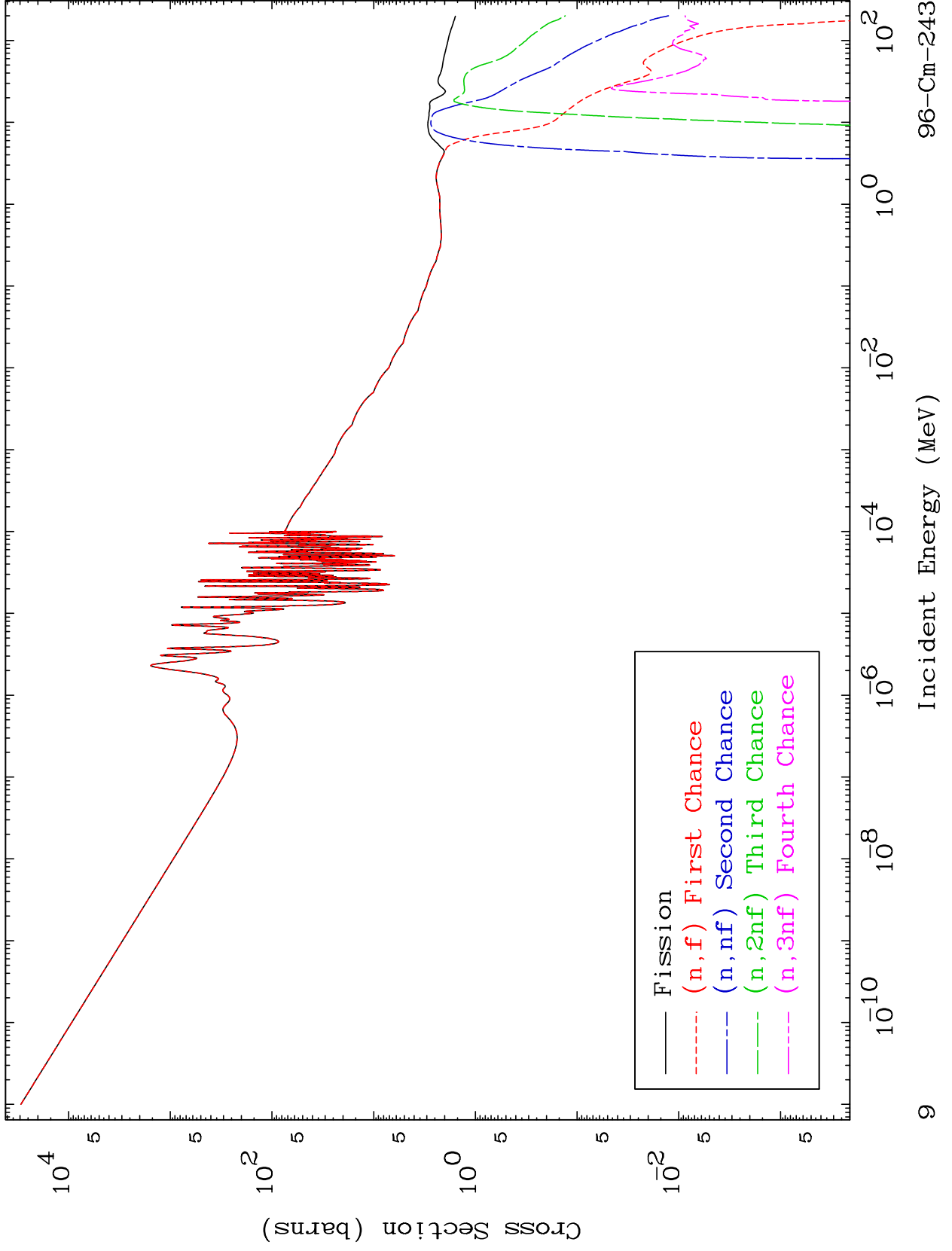
96-Cm-243



MAT 9634

Fission  
293 Kelvin Cross Sections

96-Cm-243

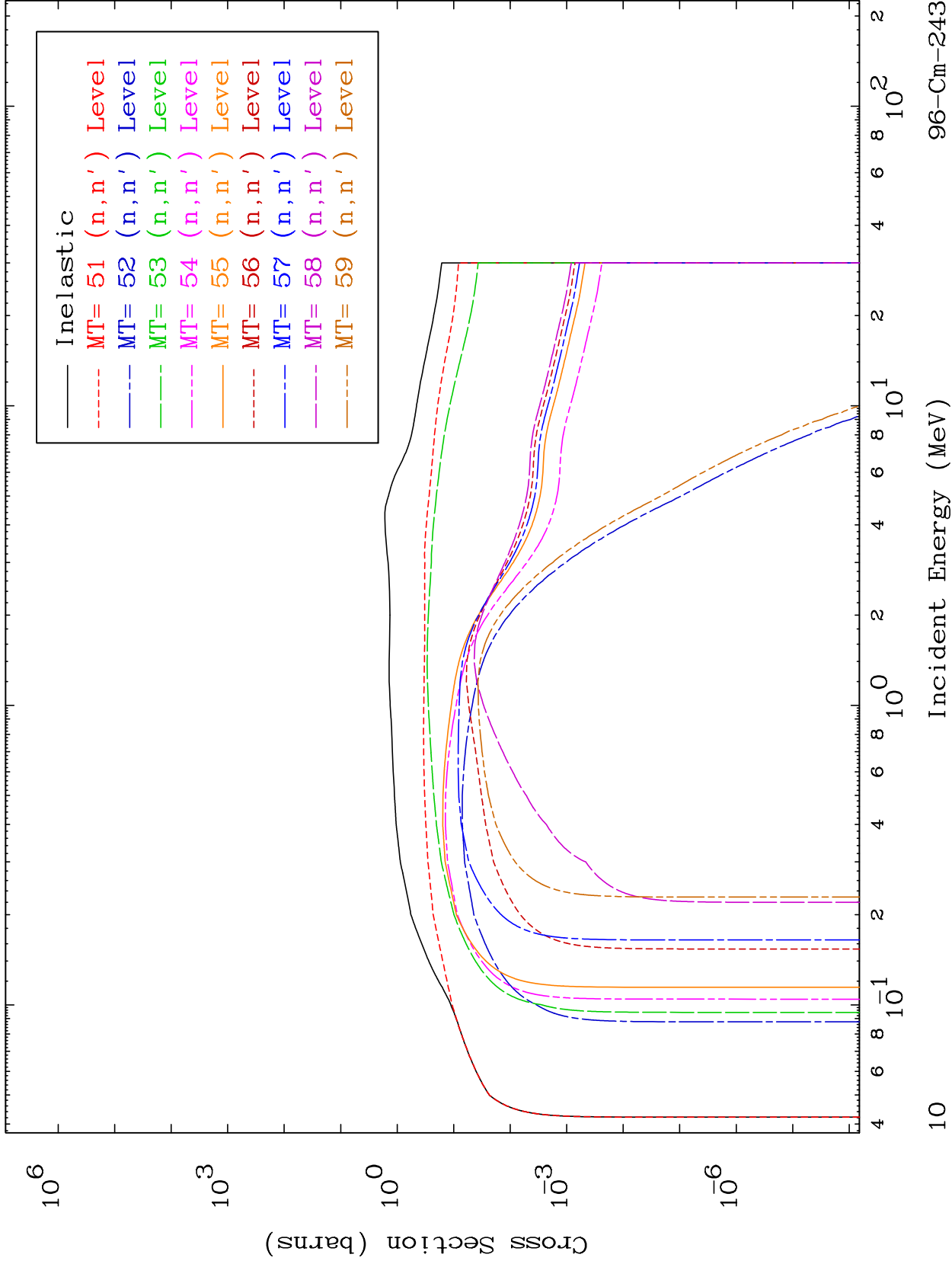


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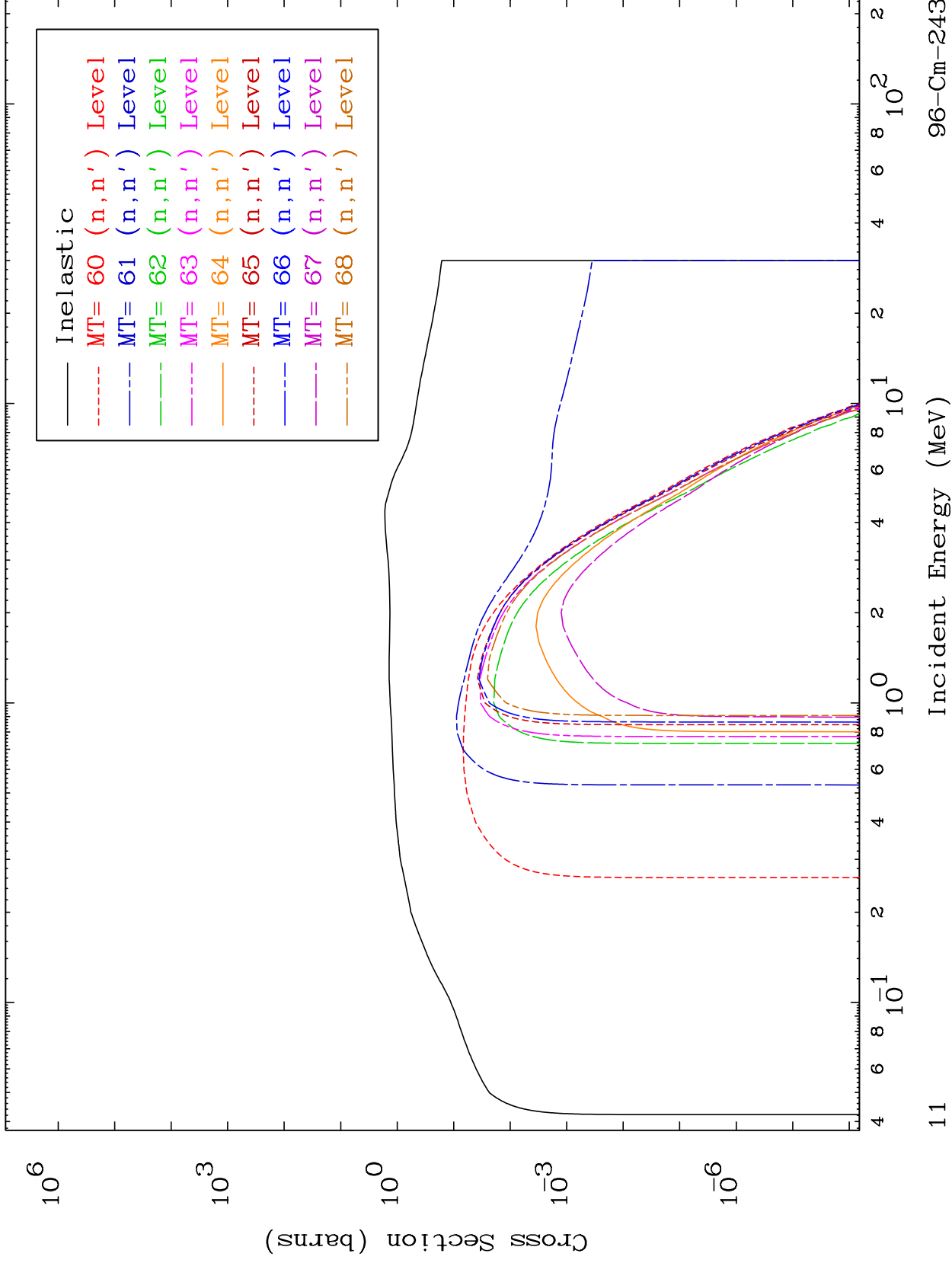
(n,n') Levels

96-Cm-243

293 Kelvin Cross Sections



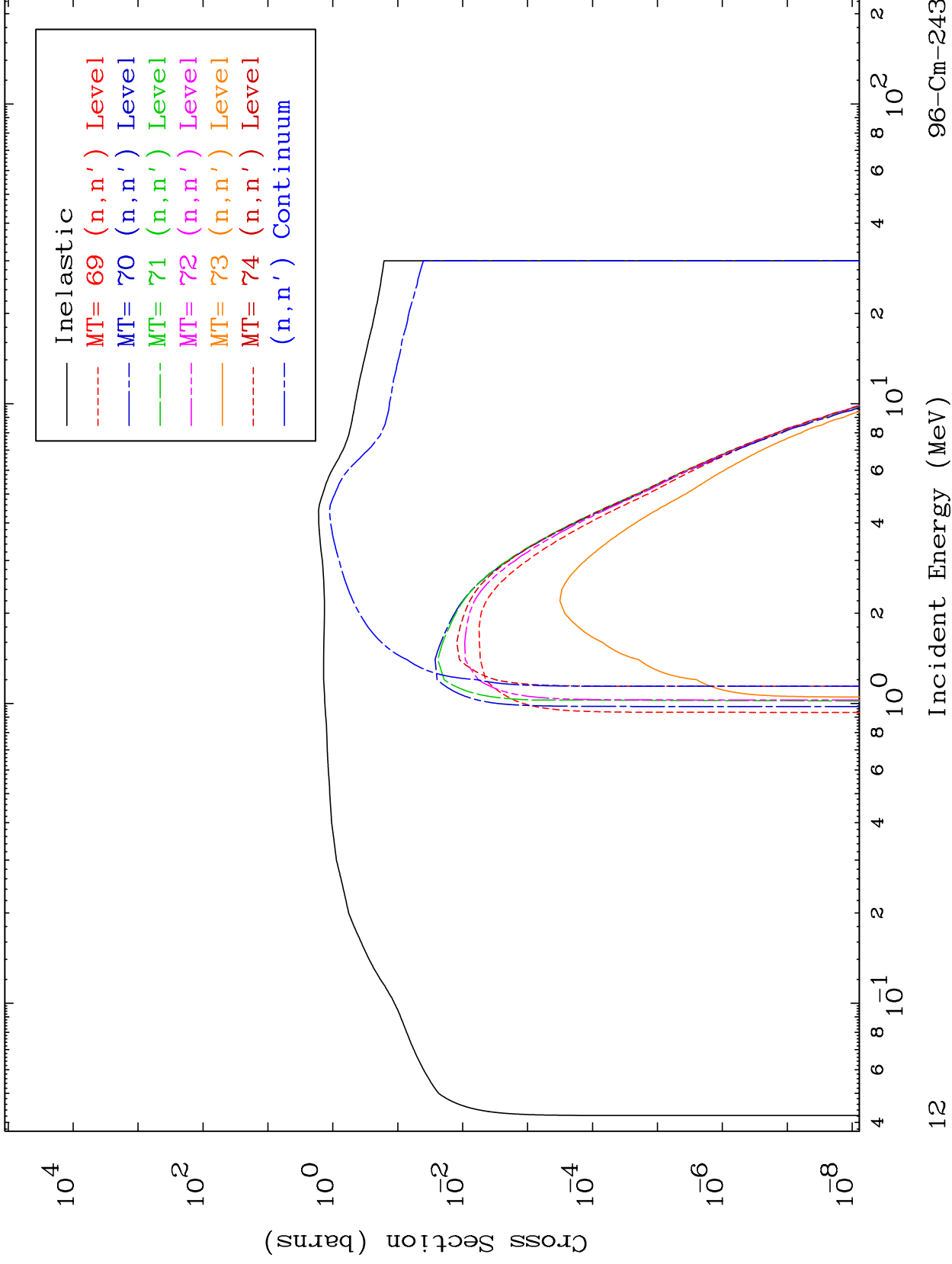
293 Kelvin Cross Sections



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(n,n') Levels  
293 Kelvin Cross Sections

96-Cm-243



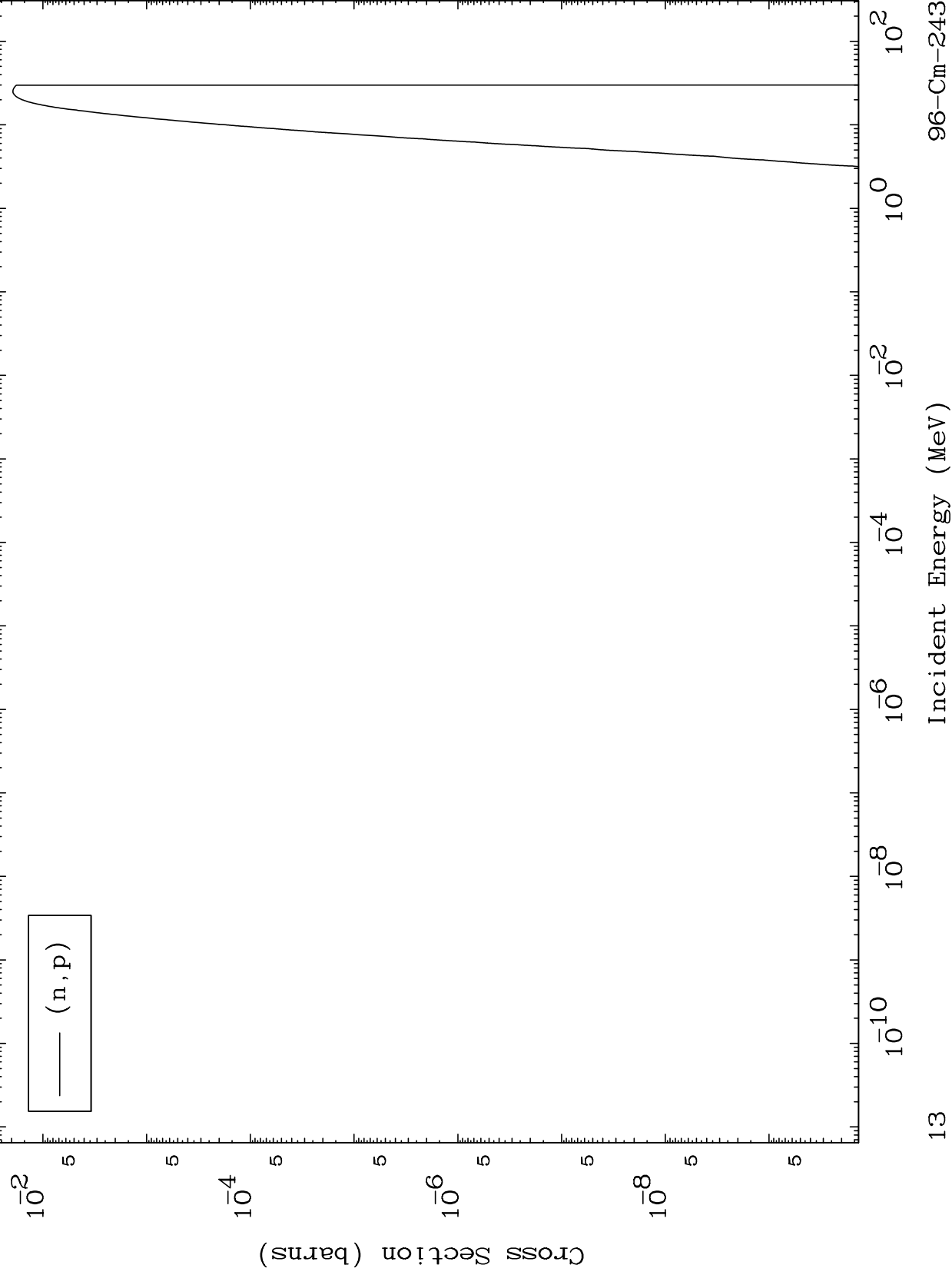
12

96-Cm-243

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(n,p) Levels  
293 Kelvin Cross Sections

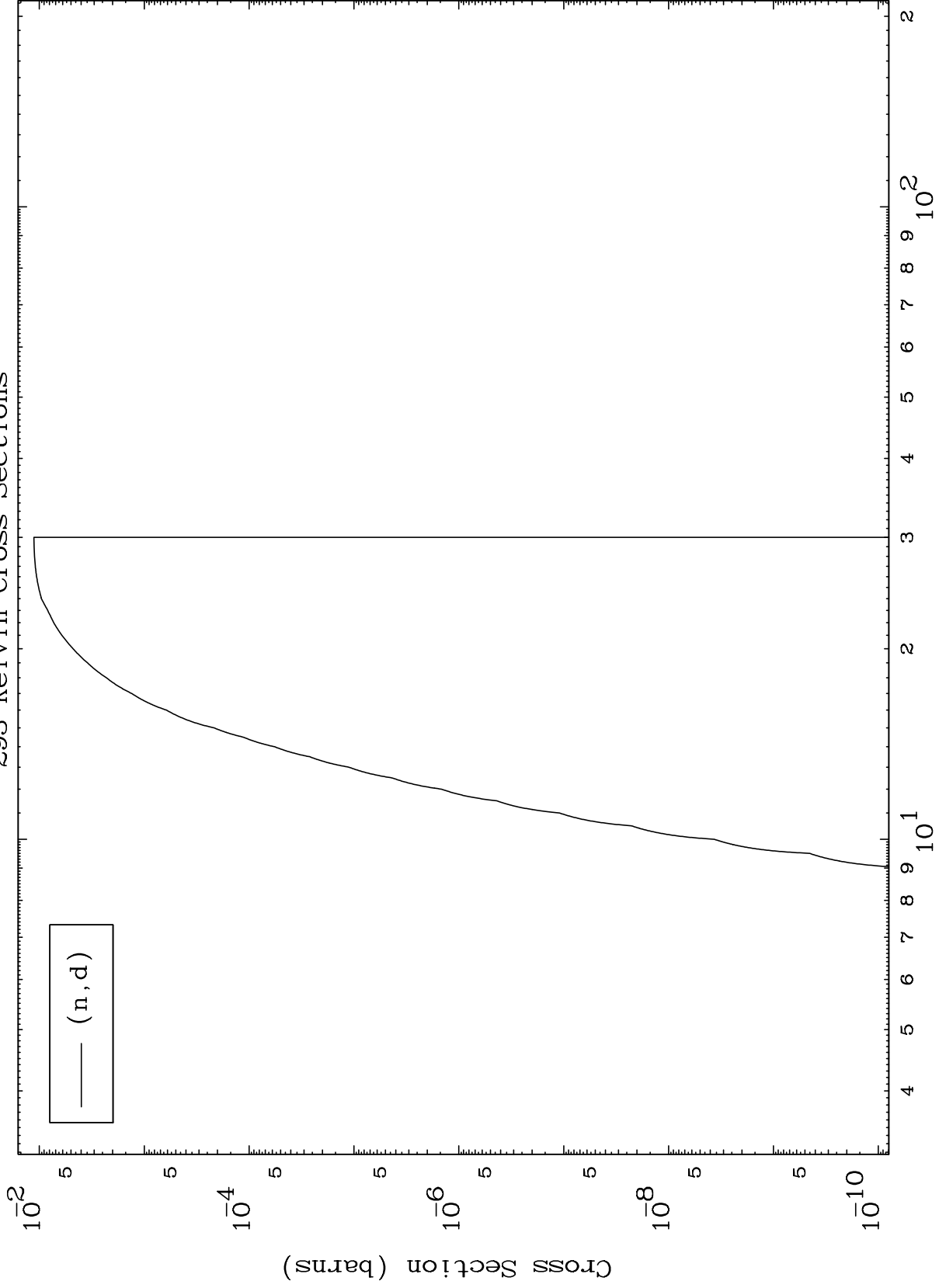
96-Cm-243



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(n,d) Levels  
293 Kelvin Cross Sections

96-Cm-243



14

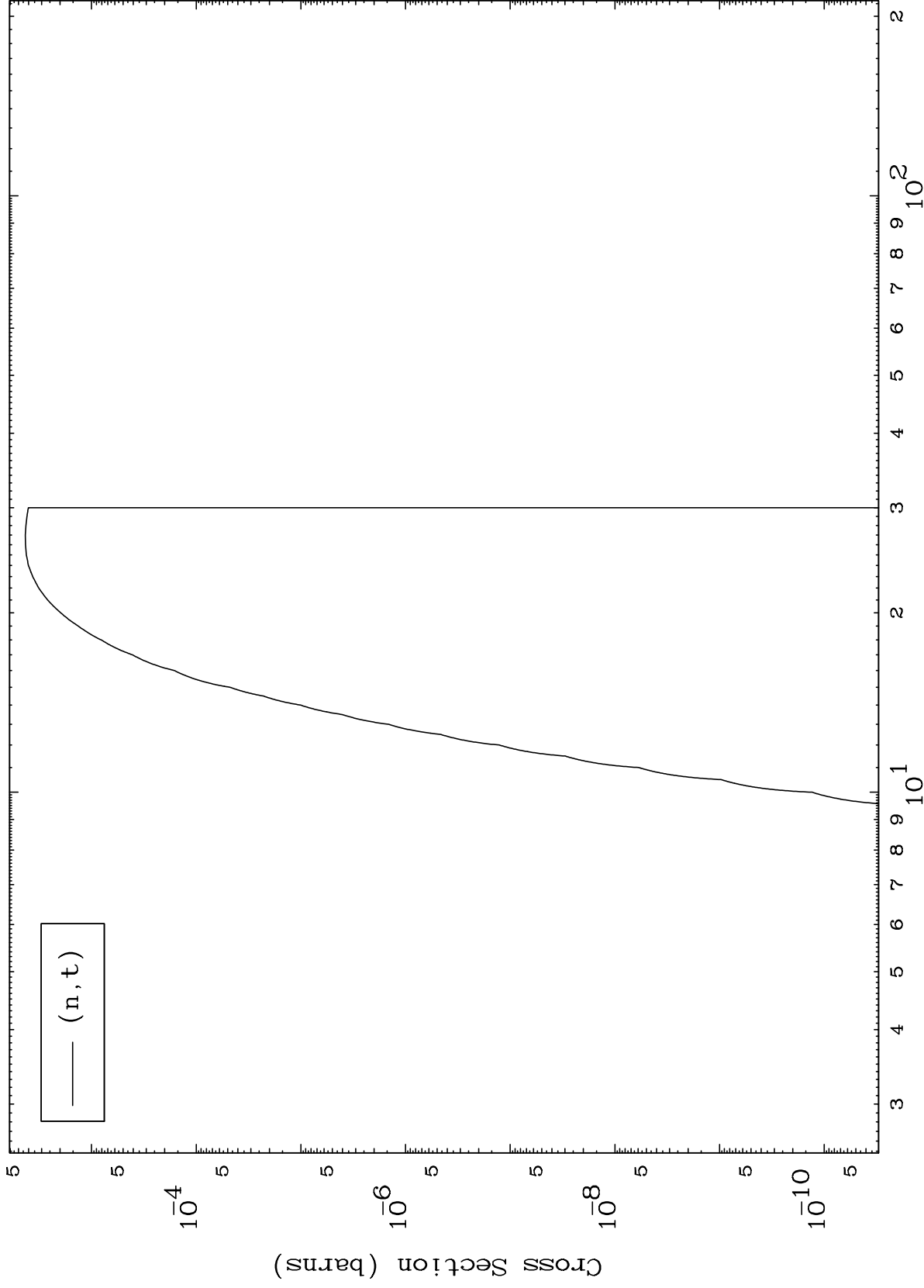
Incident Energy (MeV)

96-Cm-243

MAT 9634

(n,t) Levels  
293 Kelvin Cross Sections

96-Cm-243



(n,t)

15

Incident Energy (MeV)

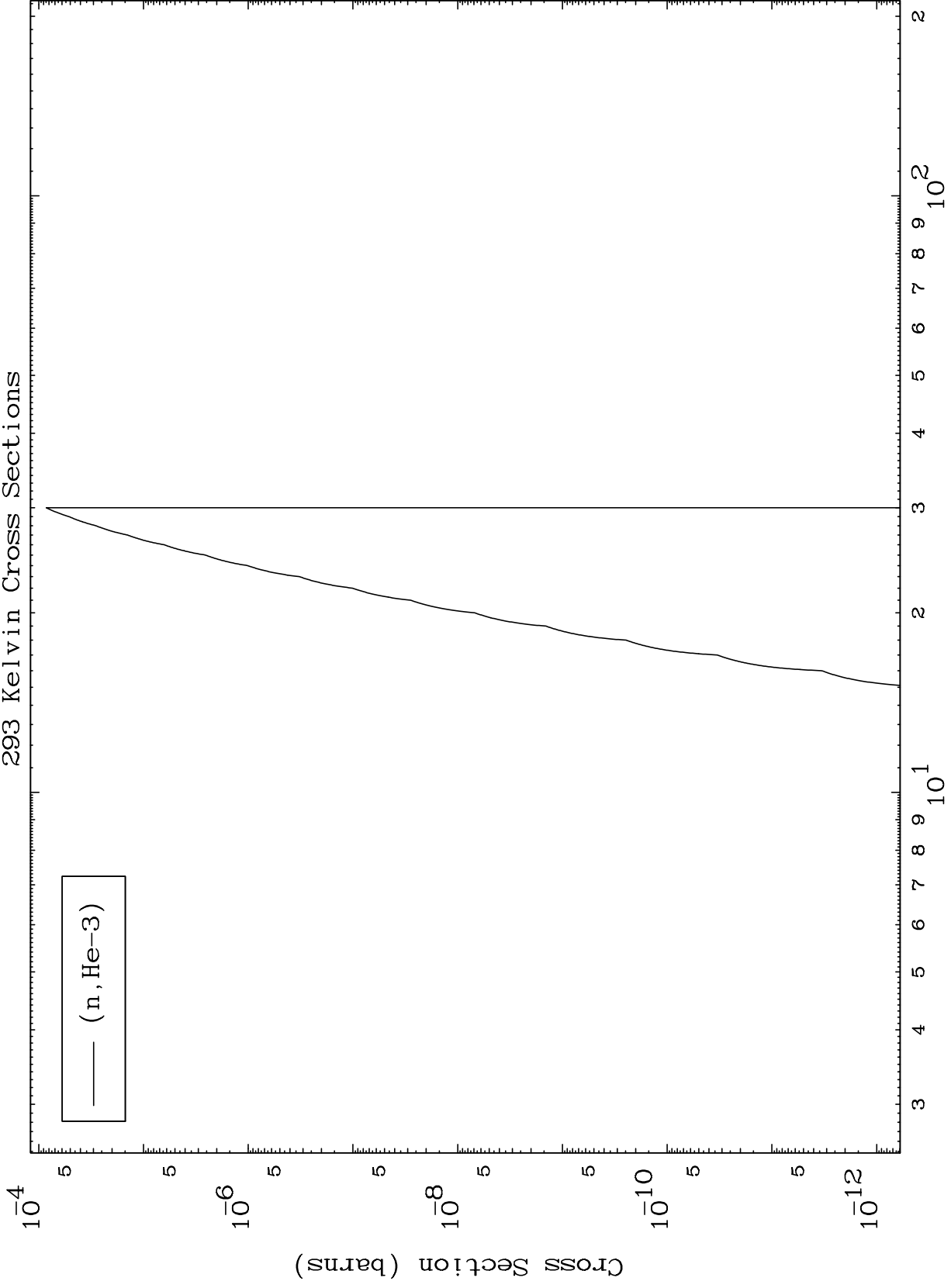
96-Cm-243



MAT 9634

(n,He3) Levels  
293 Kelvin Cross Sections

96-Cm-243



16

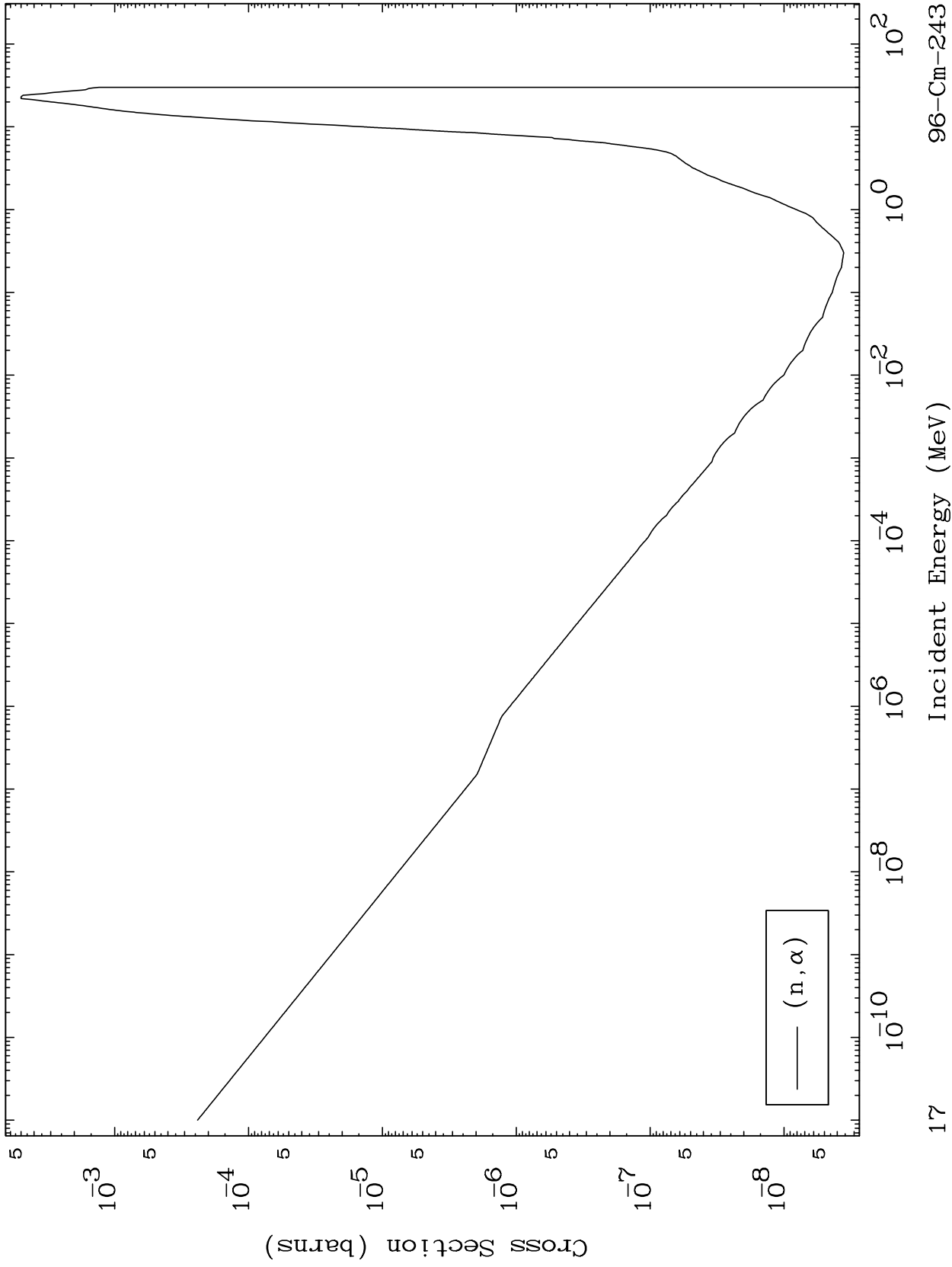
Incident Energy (MeV)

96-Cm-243

MAT 9634

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

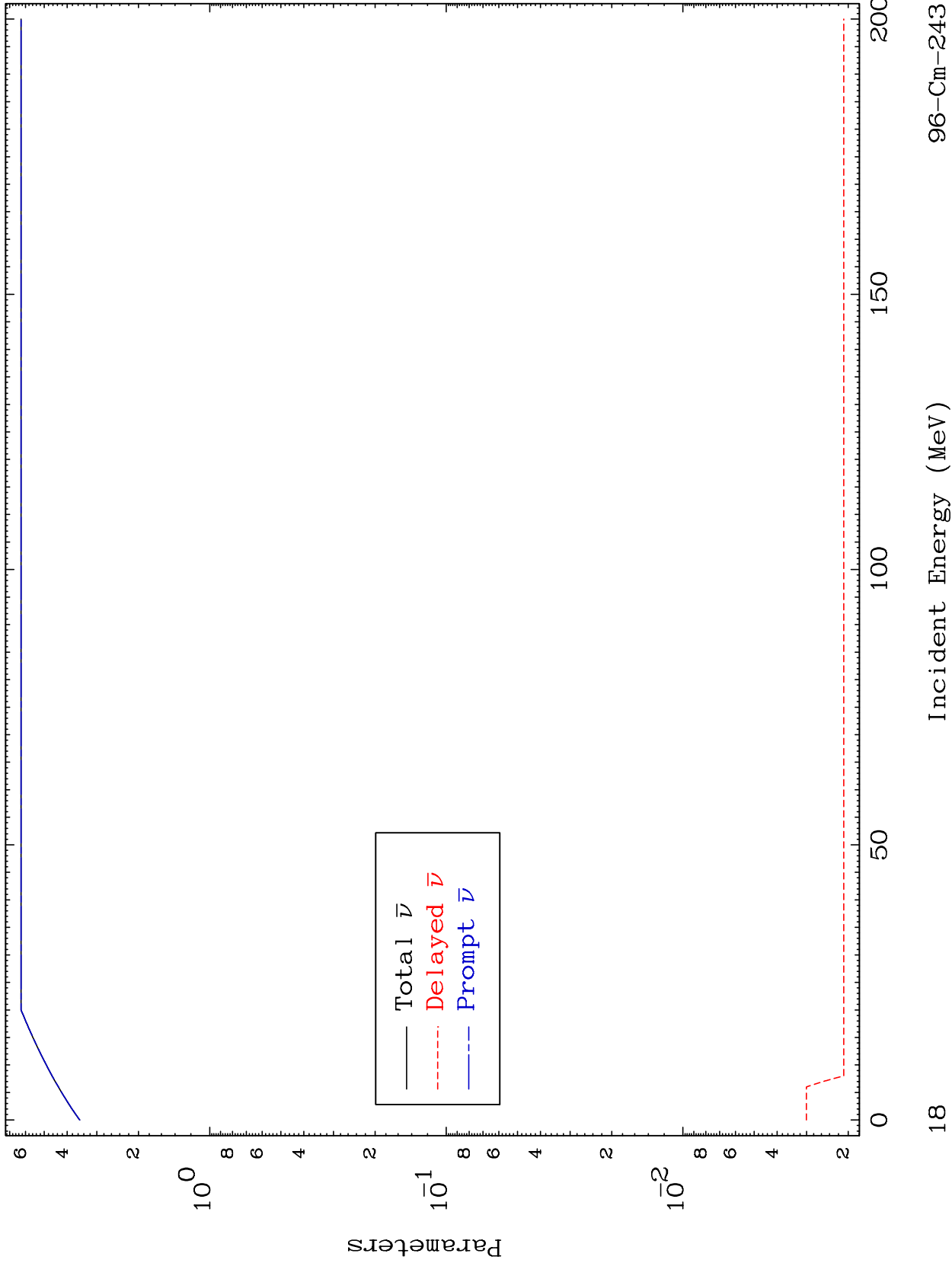
96-Cm-243



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Energy Release  
Parameters

96-Cm-243



18

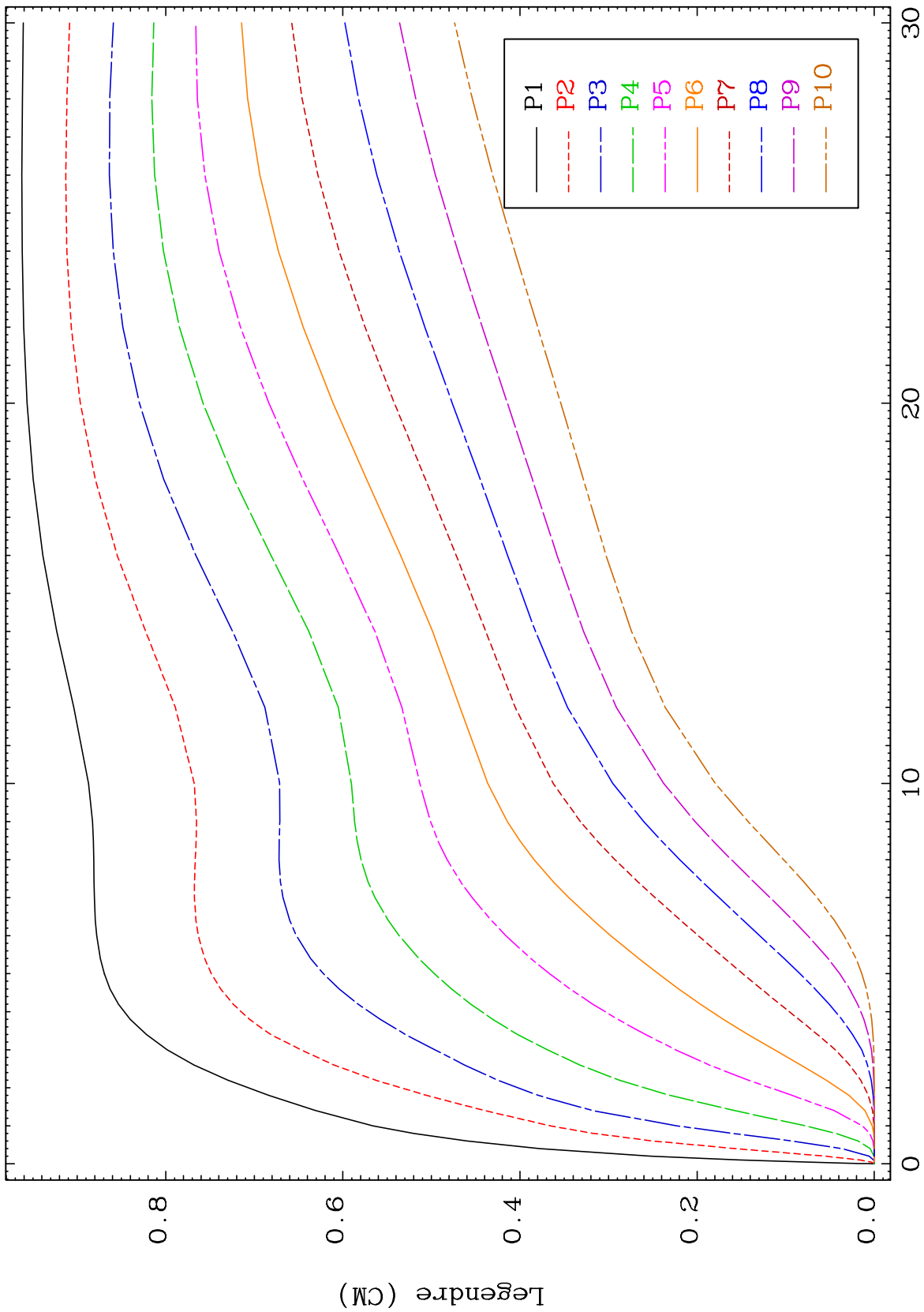
Incident Energy (MeV)

96-Cm-243

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Elastic Legendre Coefficients

96-Cm-243



19

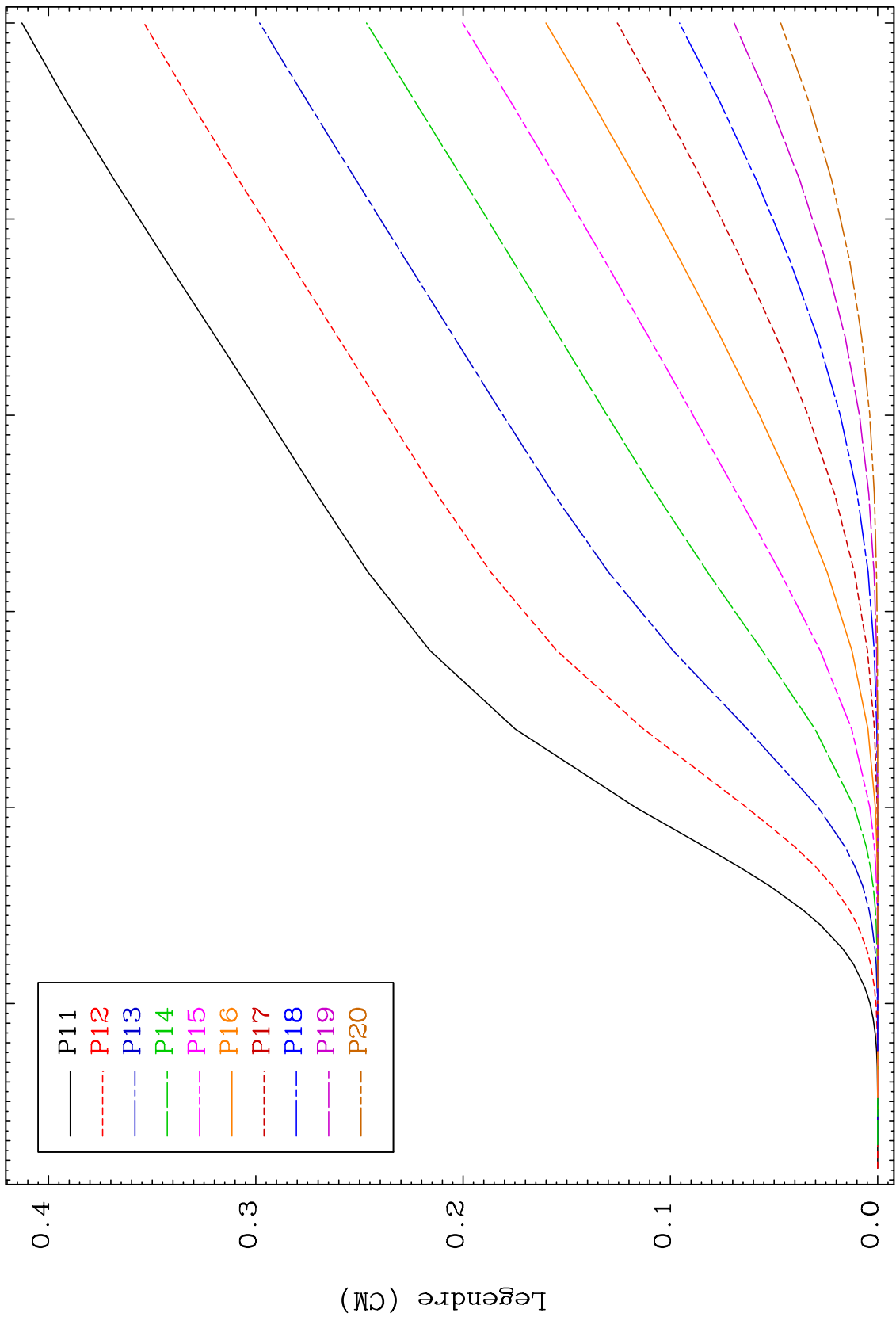
Incident Energy (MeV)

96-Cm-243

MAT 9634

Elastic Legendre Coefficients

96-Cm-243



20

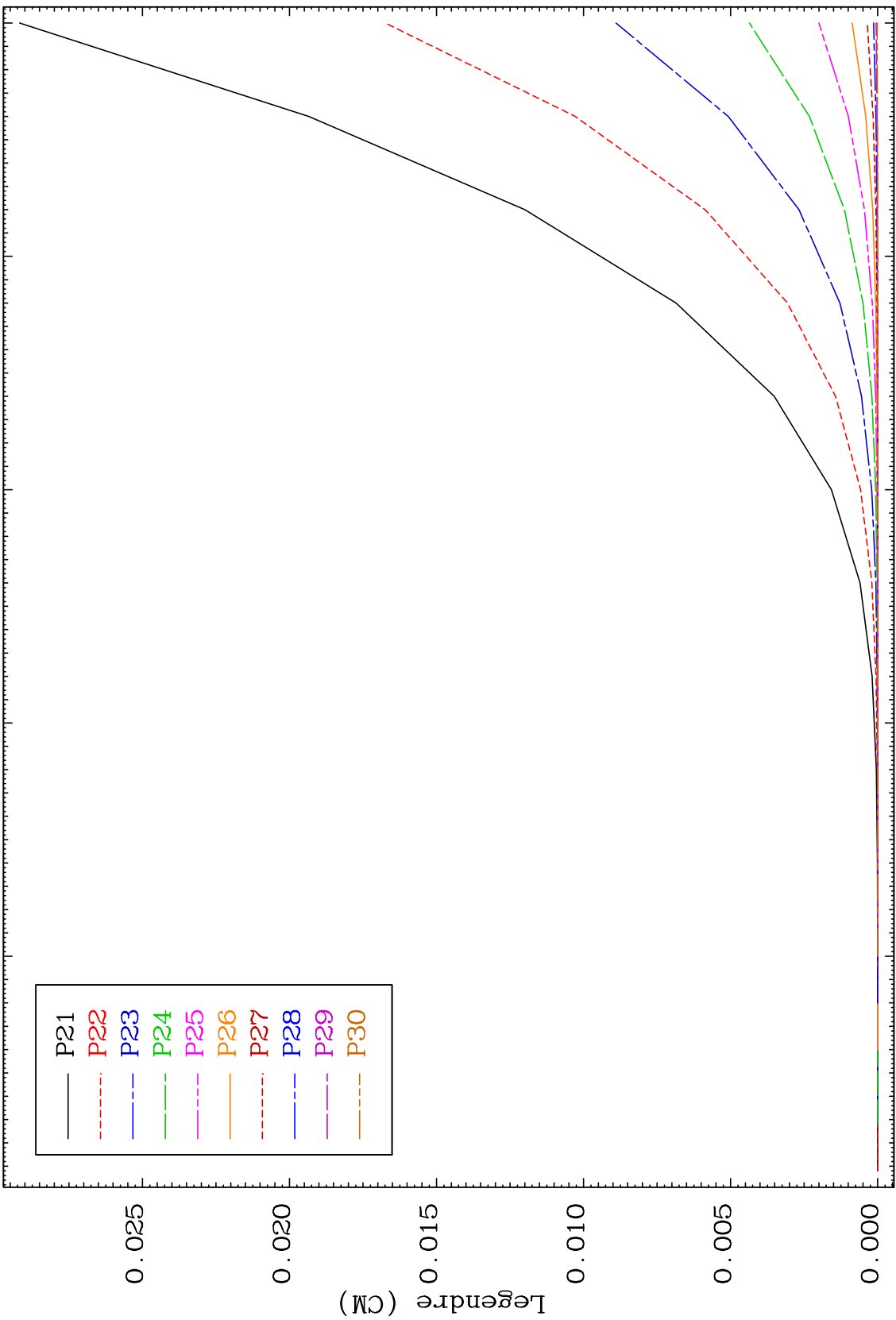
Incident Energy (MeV)

96-Cm-243

MAT 9634

Elastic Legendre Coefficients

96-Cm-243



21

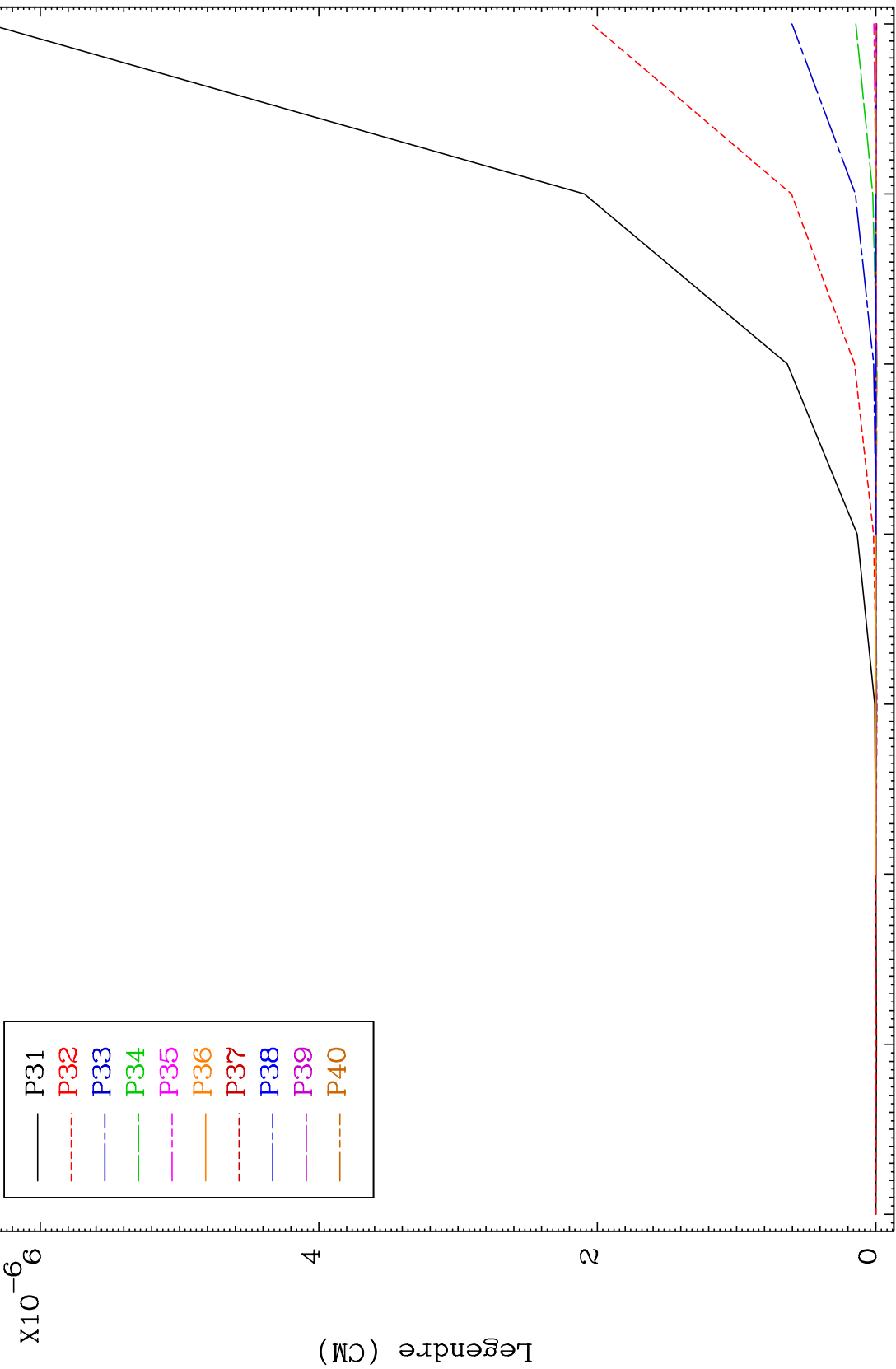
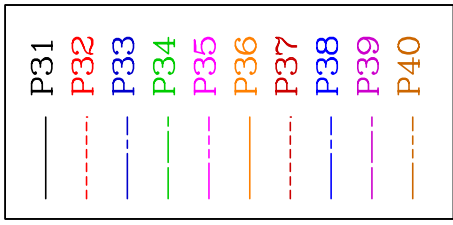
96-Cm-243

Incident Energy (MeV)

MAT 9634

Elastic Legendre Coefficients

96-Cm-243

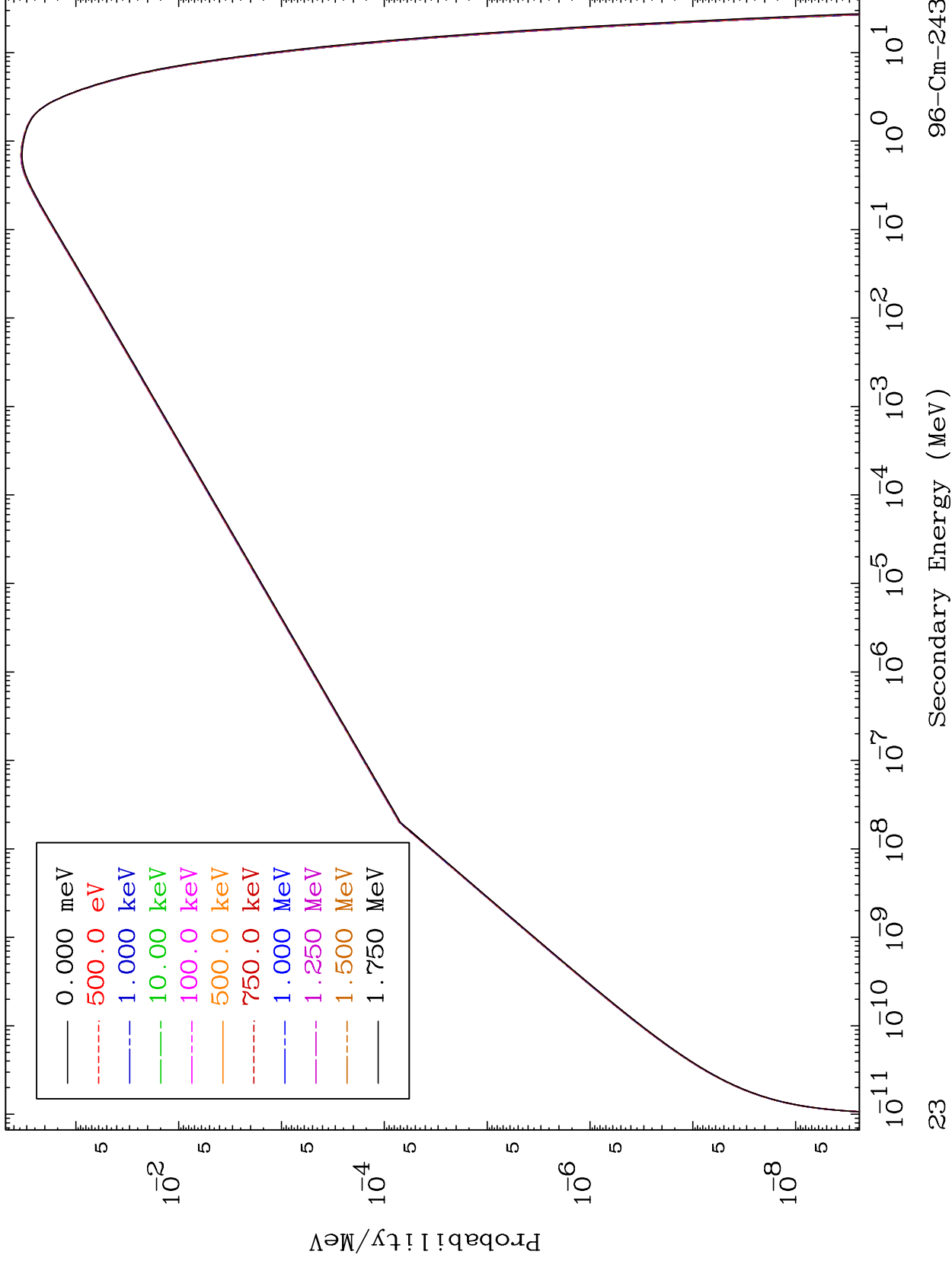


96-Cm-243

Incident Energy (MeV)

22

Fission Energy Distributions

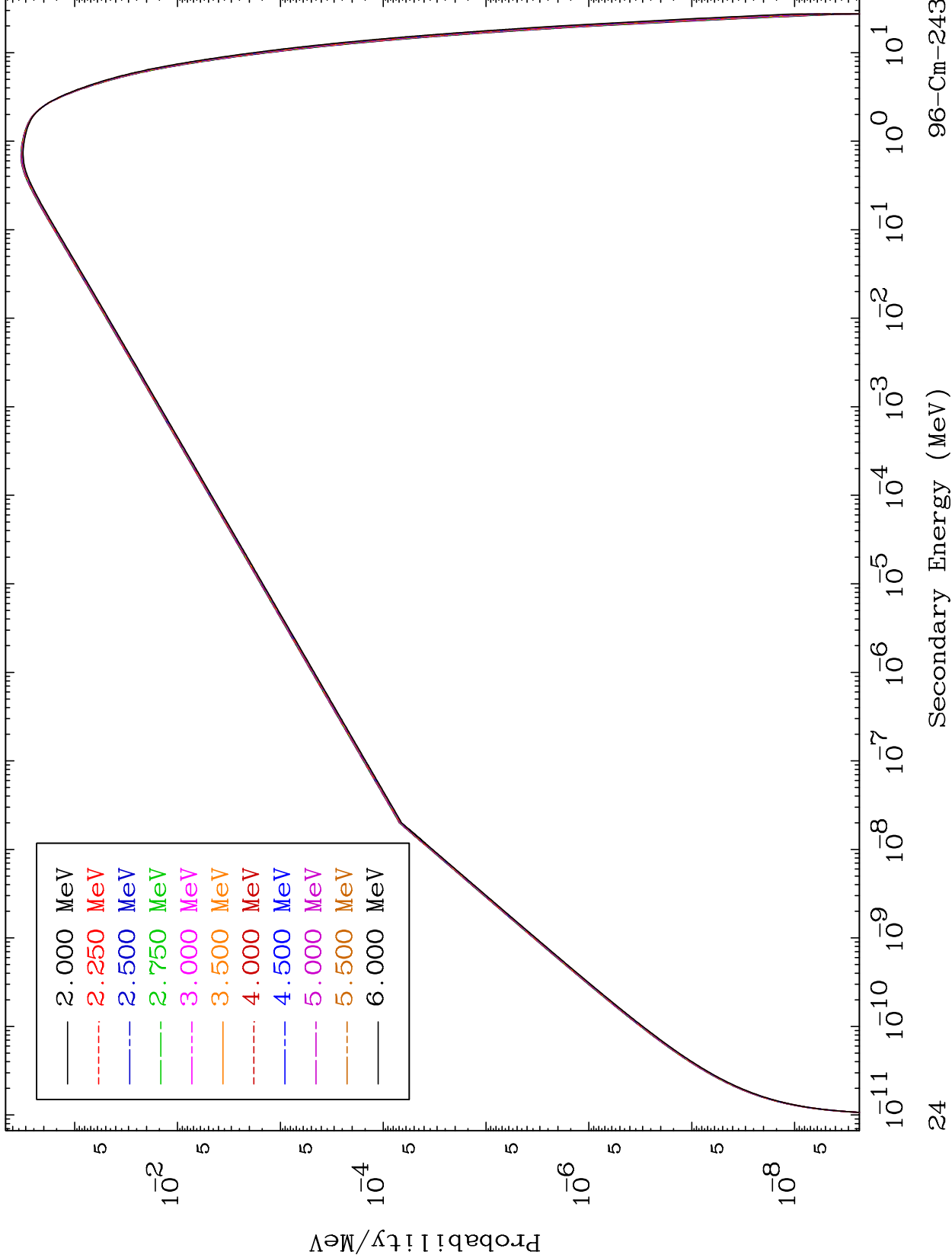




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### Fission Energy Distributions

96-Cm-243

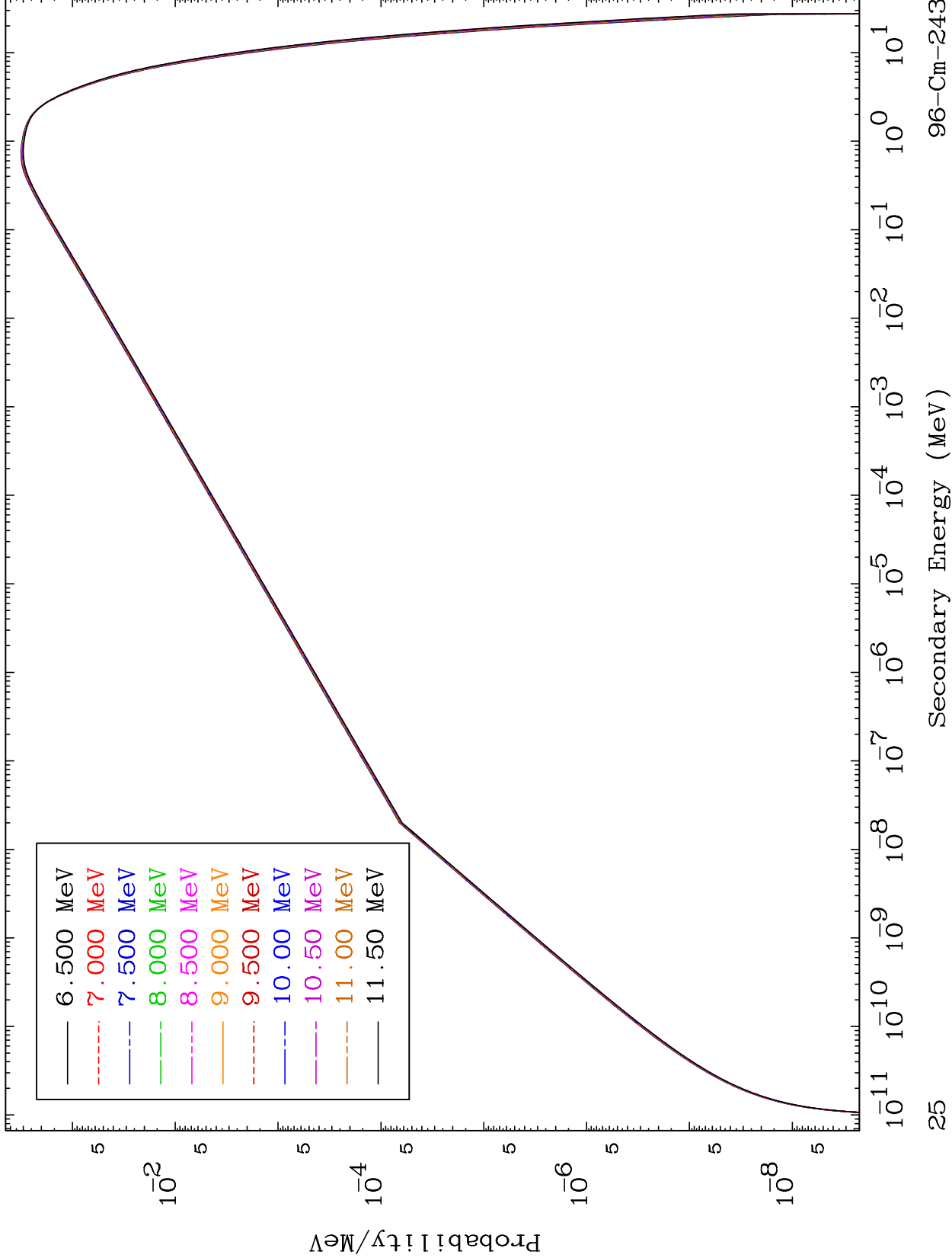


96-Cm-243

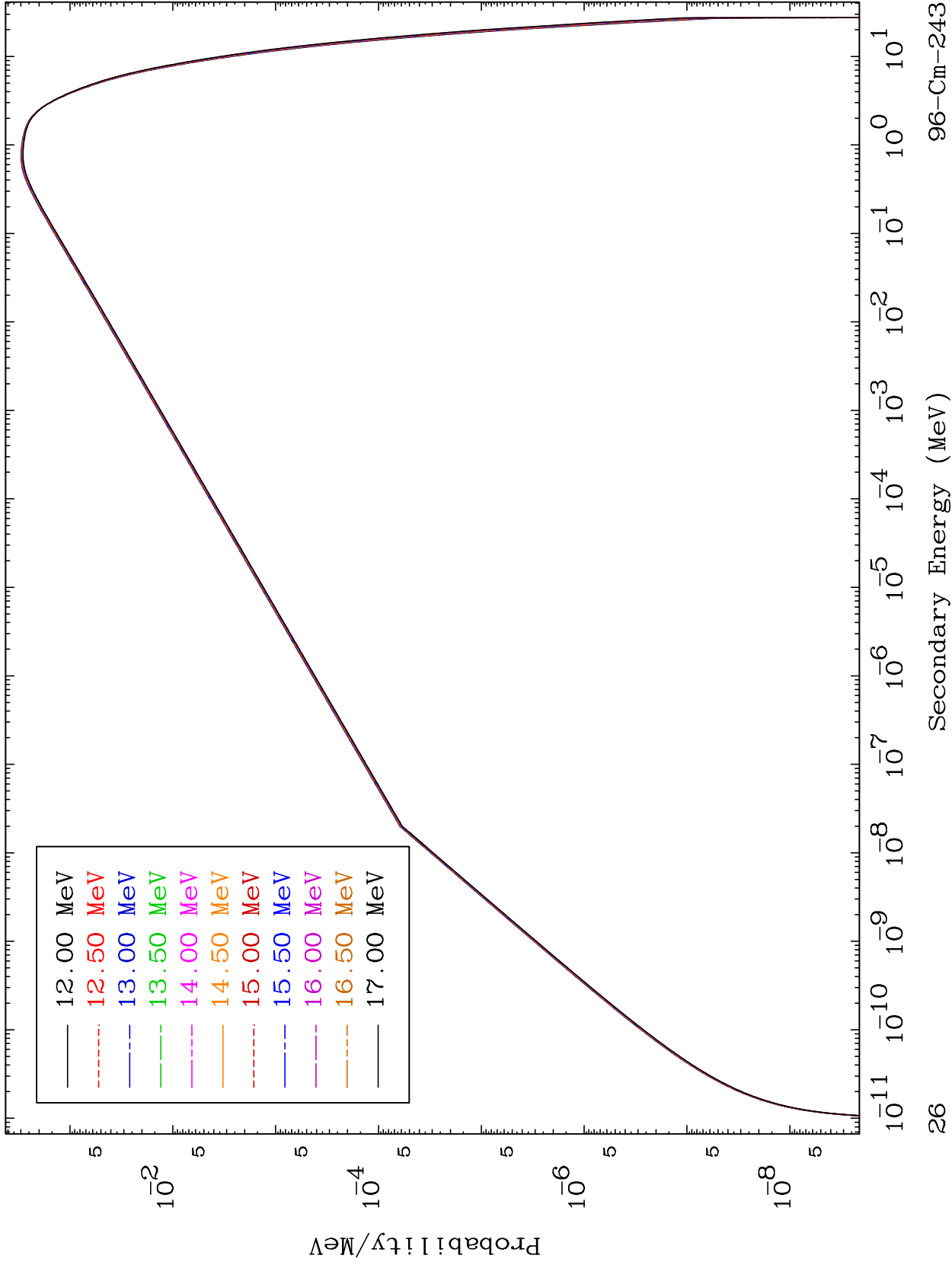
Secondary Energy (MeV)

24

Fission Energy Distributions



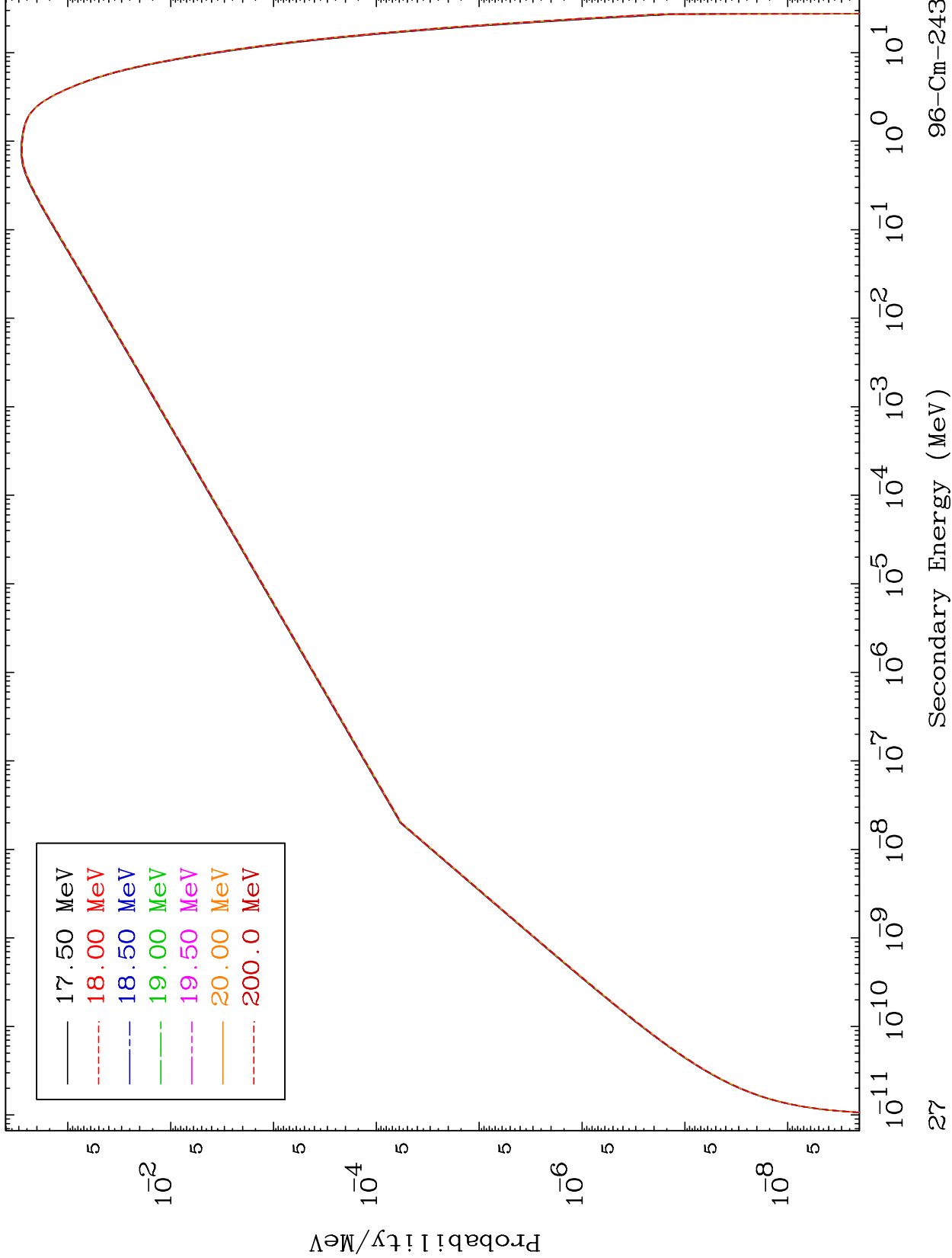
Fission Energy Distributions



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Fission Energy Distributions

96-Cm-243

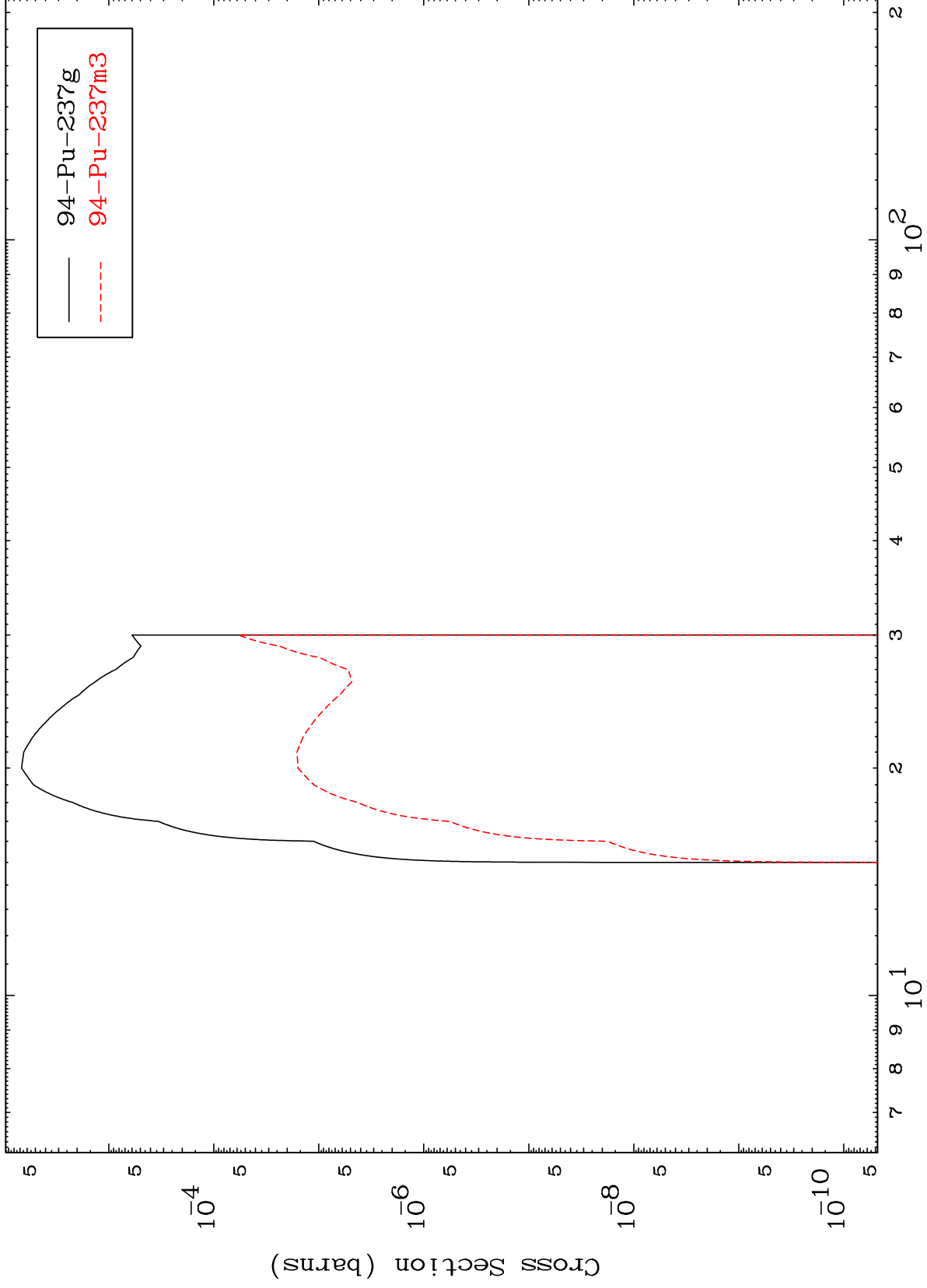


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(n,3n)  $\alpha$

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Radionuclide Production Cross Section



28

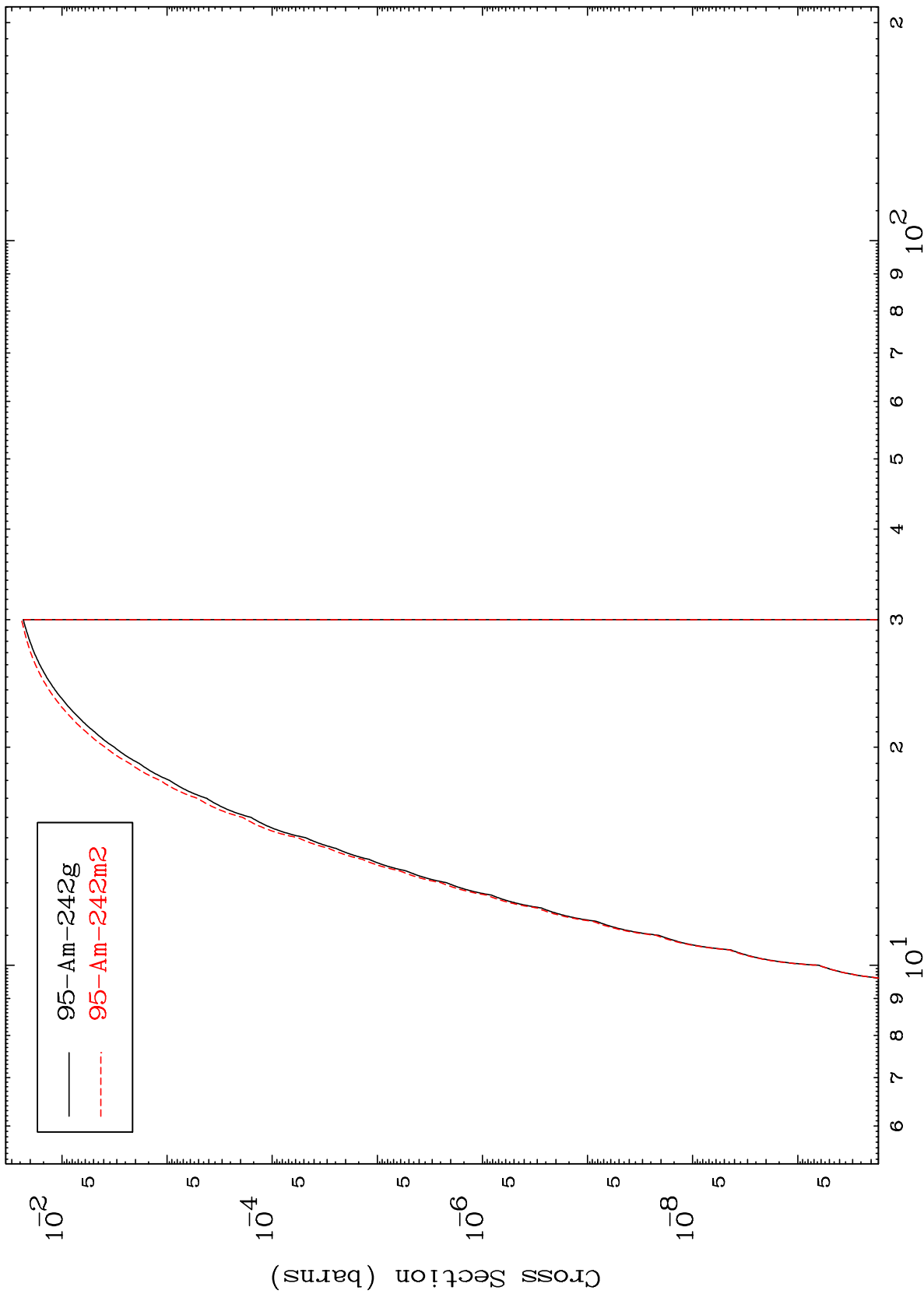
Incident Energy (MeV)

96-Cm-243

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96-Cm-243

(n,n') p  
Radionuclide Production Cross Section



29

96-Cm-243

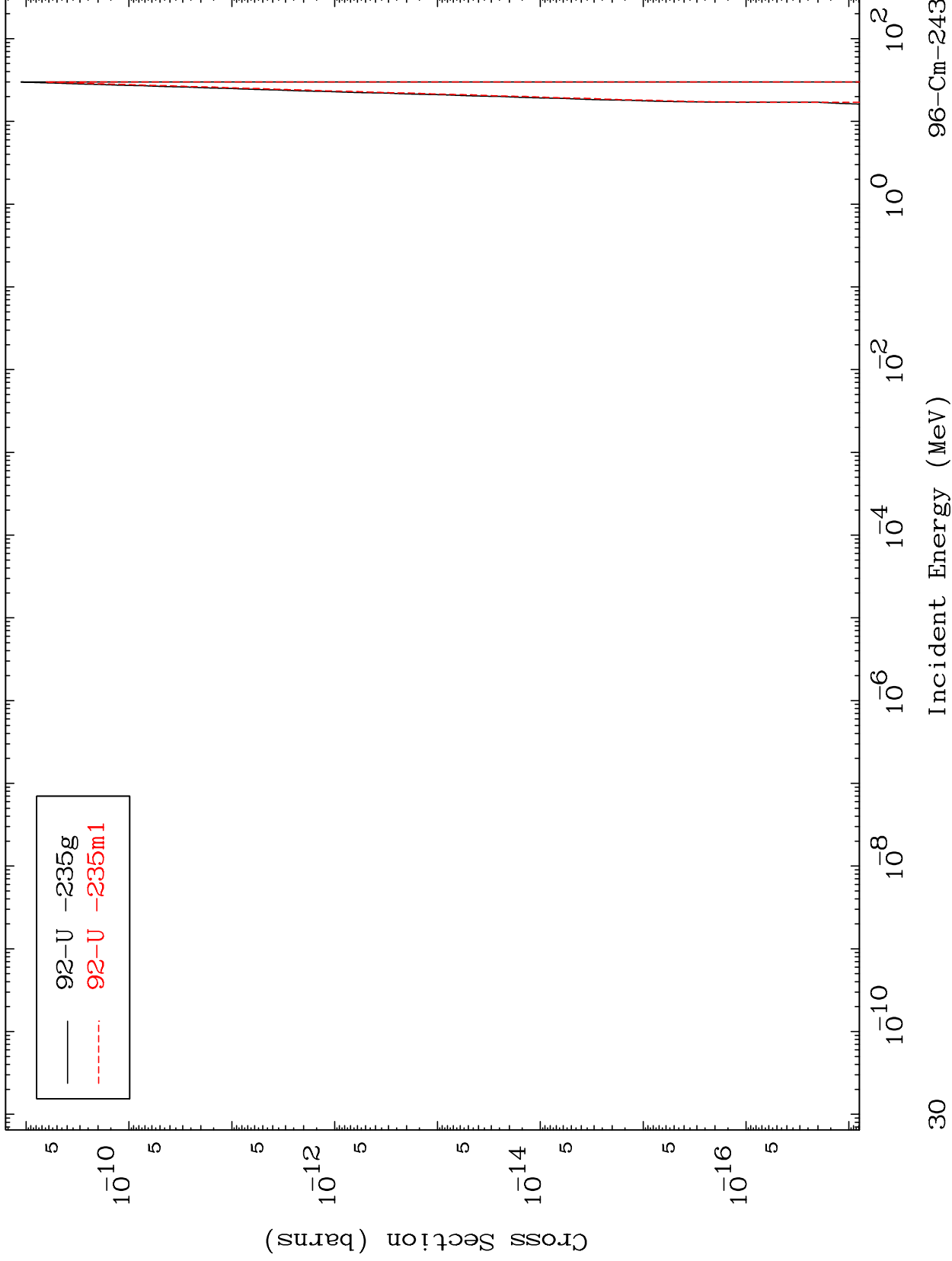
Incident Energy (MeV)

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(n,n') 2 $\alpha$

96-Cm-243

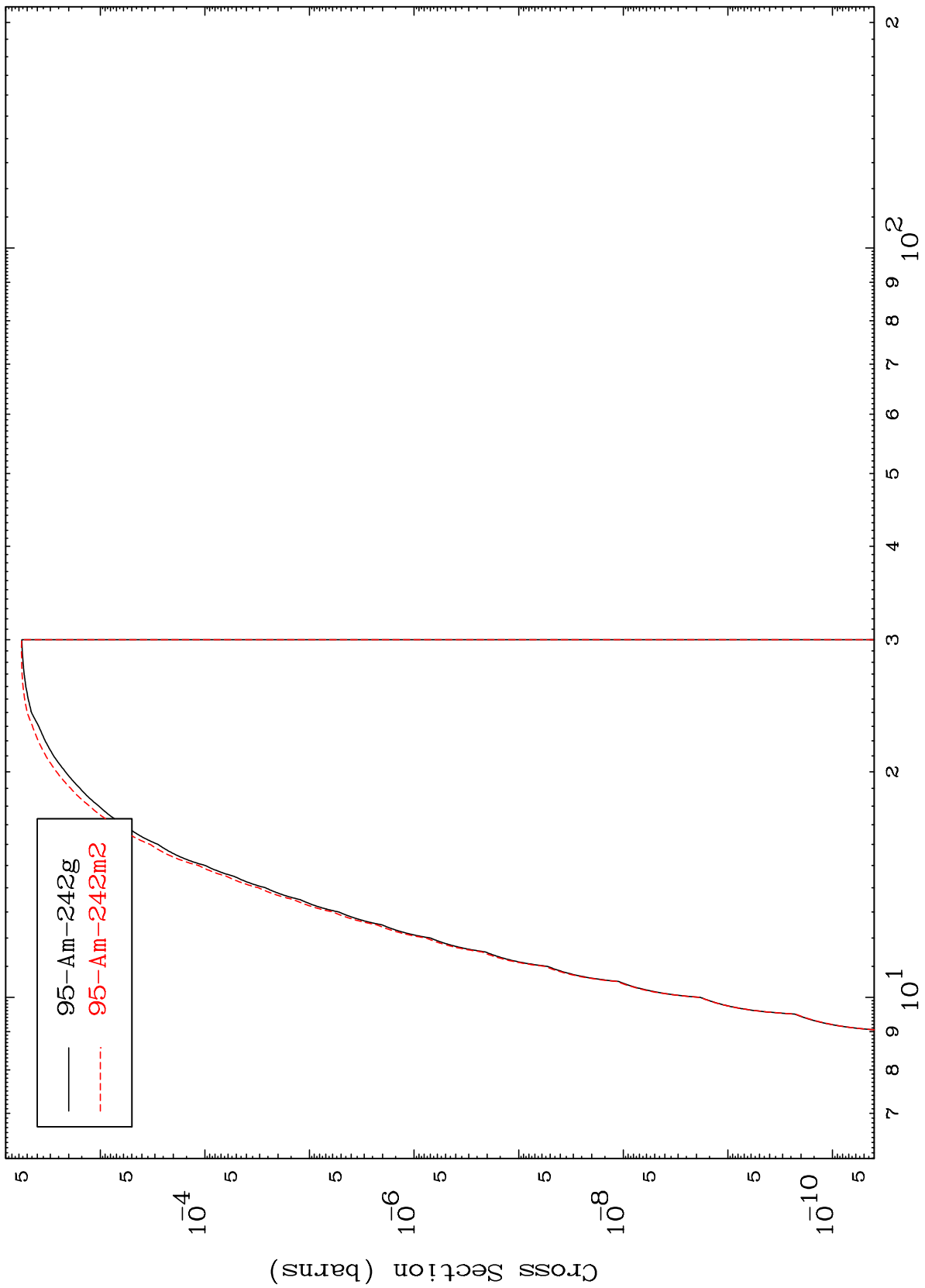
Radionuclide Production Cross Section



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96-Cm-243

(n,d)  
Radionuclide Production Cross Section



31

Incident Energy (MeV)

96-Cm-243