

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

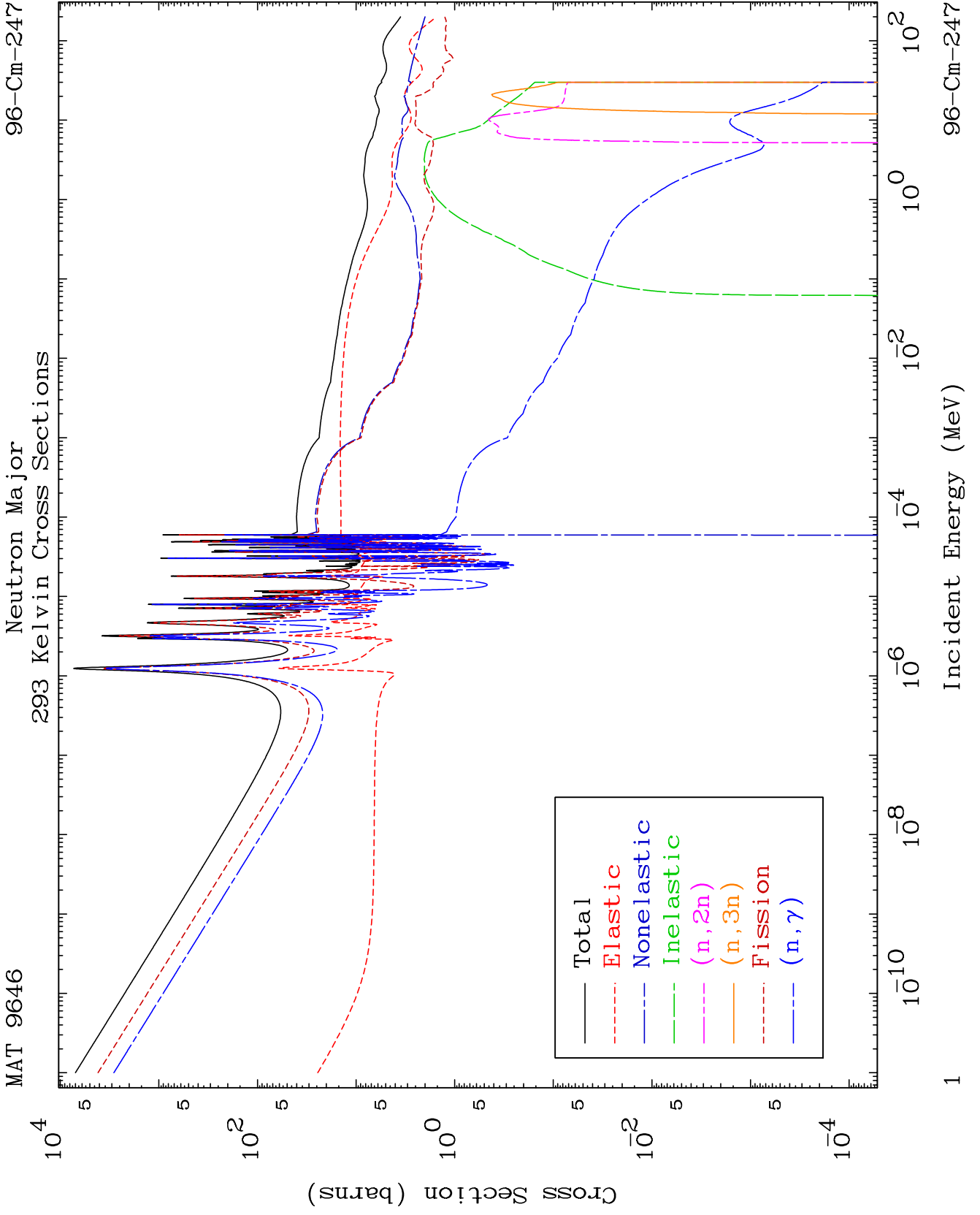
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

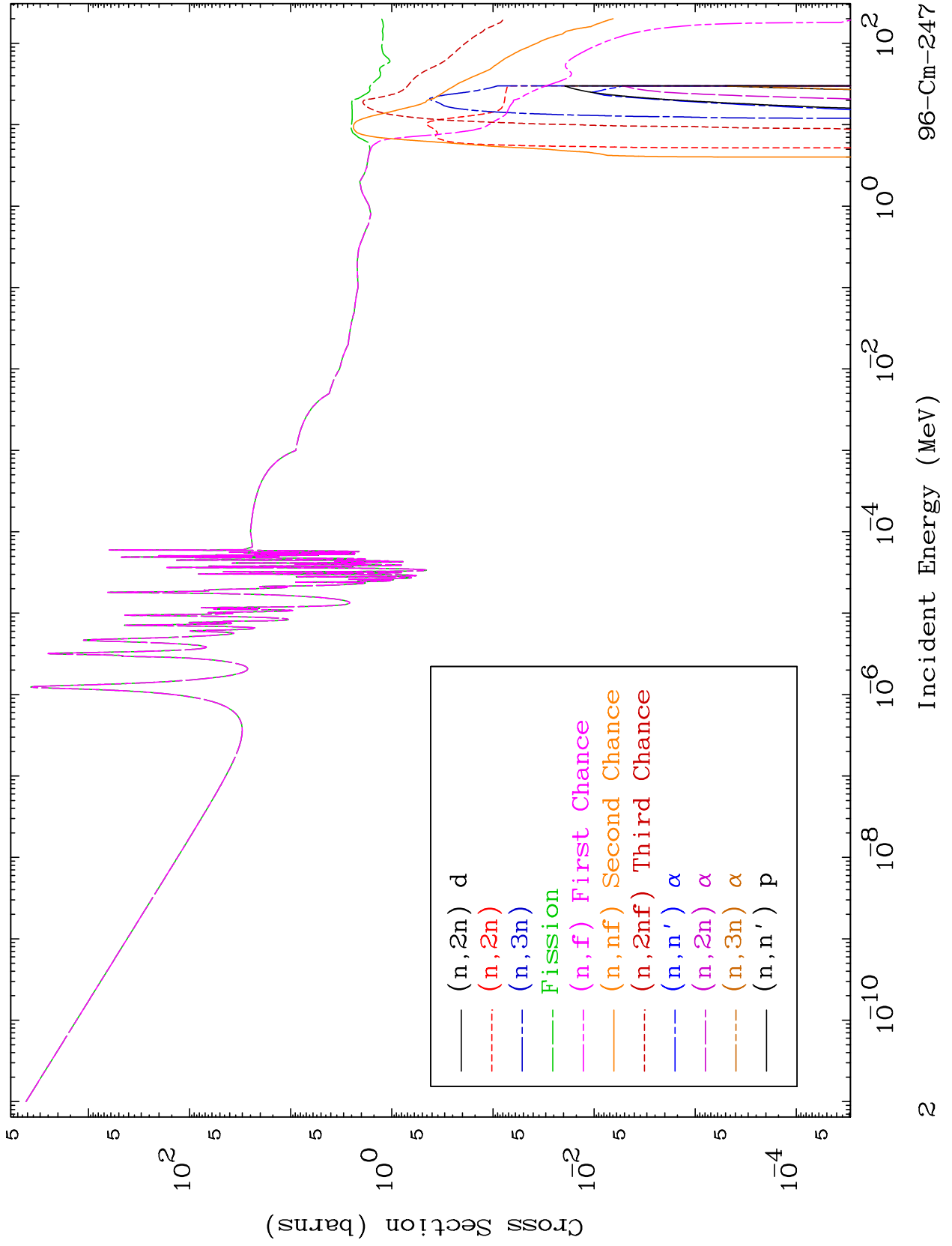
Press Mouse Button to Start



MAT 9646

Neutron Absorption
293 Kelvin Cross Sections

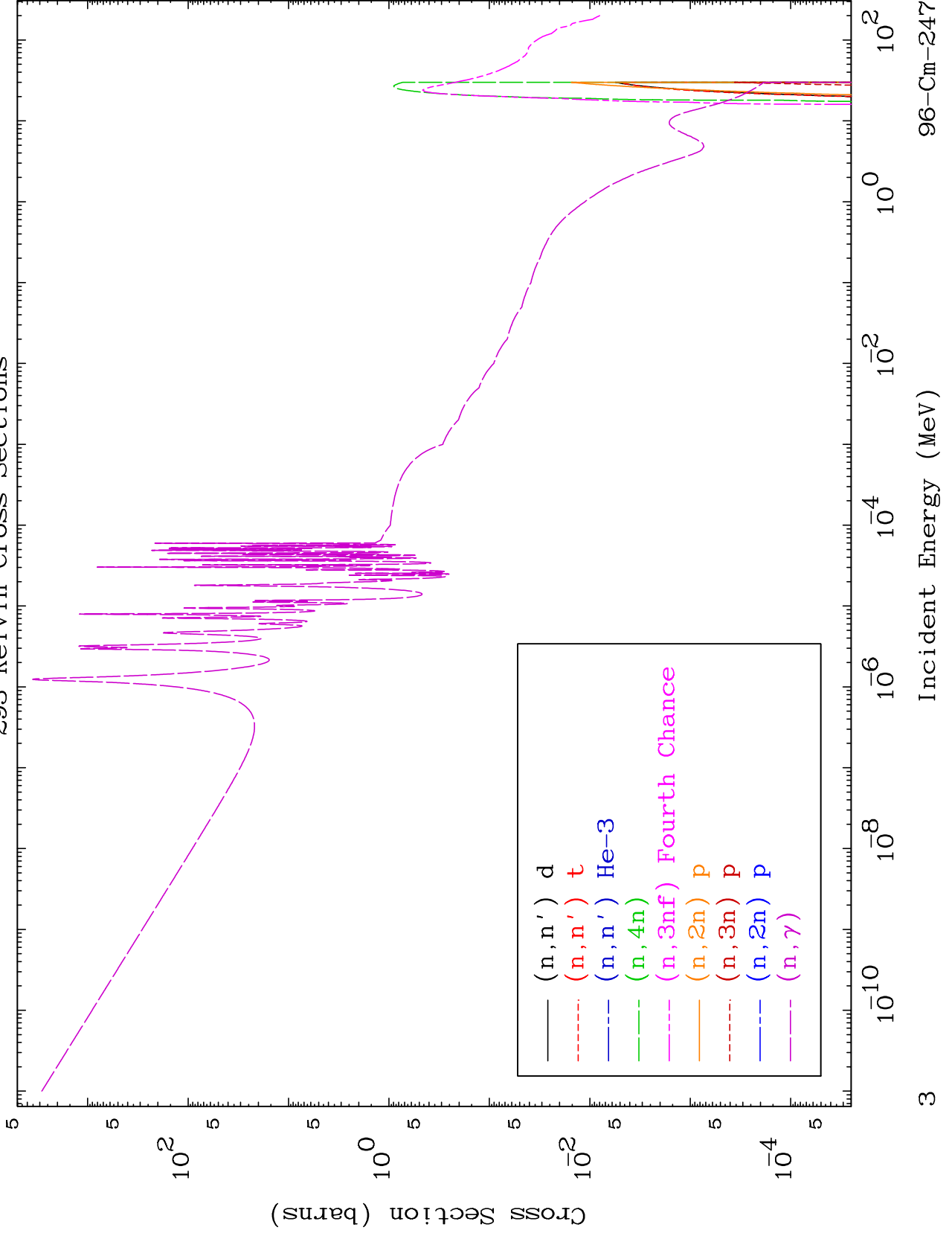
96-Cm-247



MAT 9646

Neutron Absorption
293 Kelvin Cross Sections

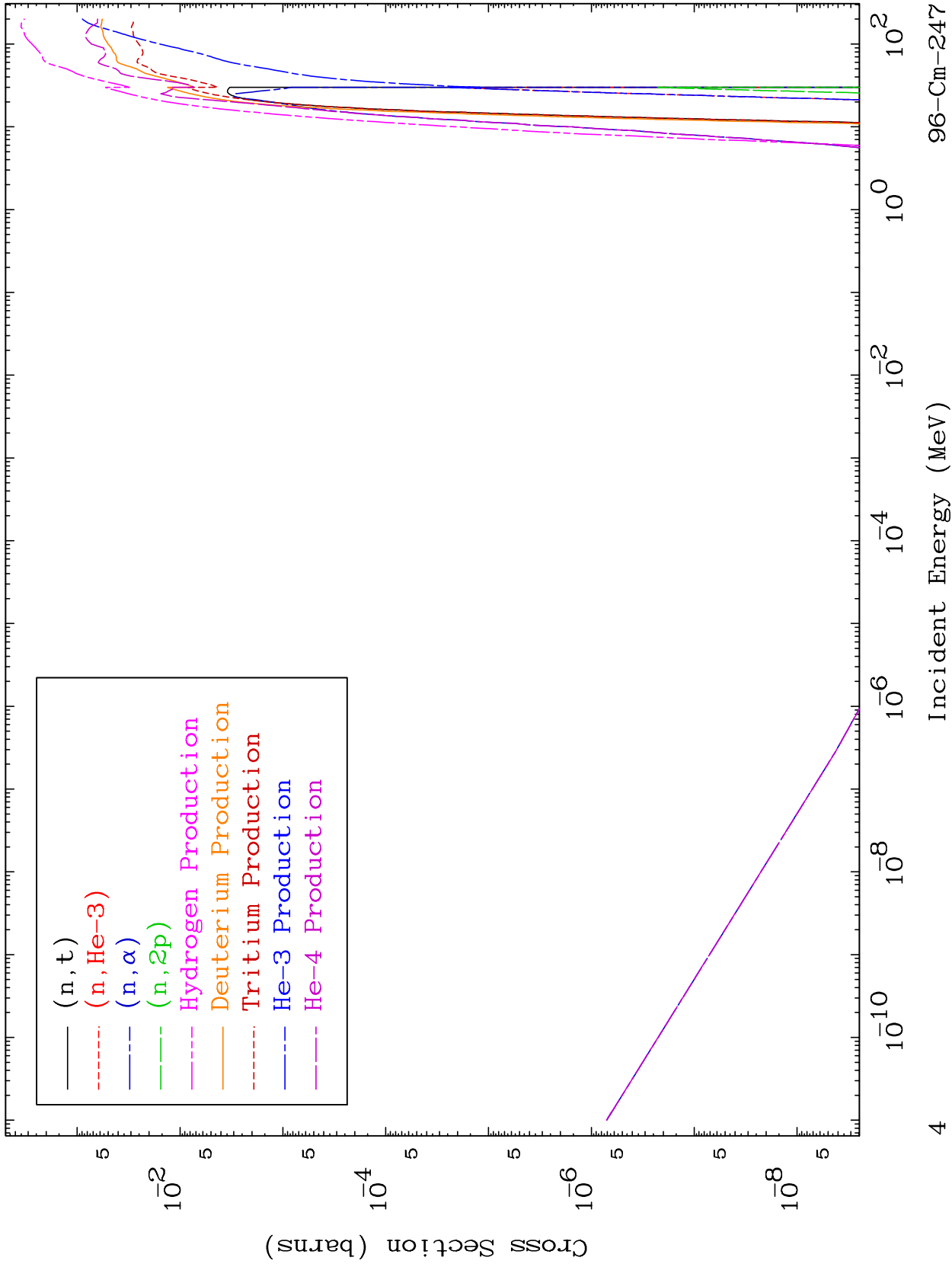
96-Cm-247



MAT 9646

Neutron Absorption
293 Kelvin Cross Sections

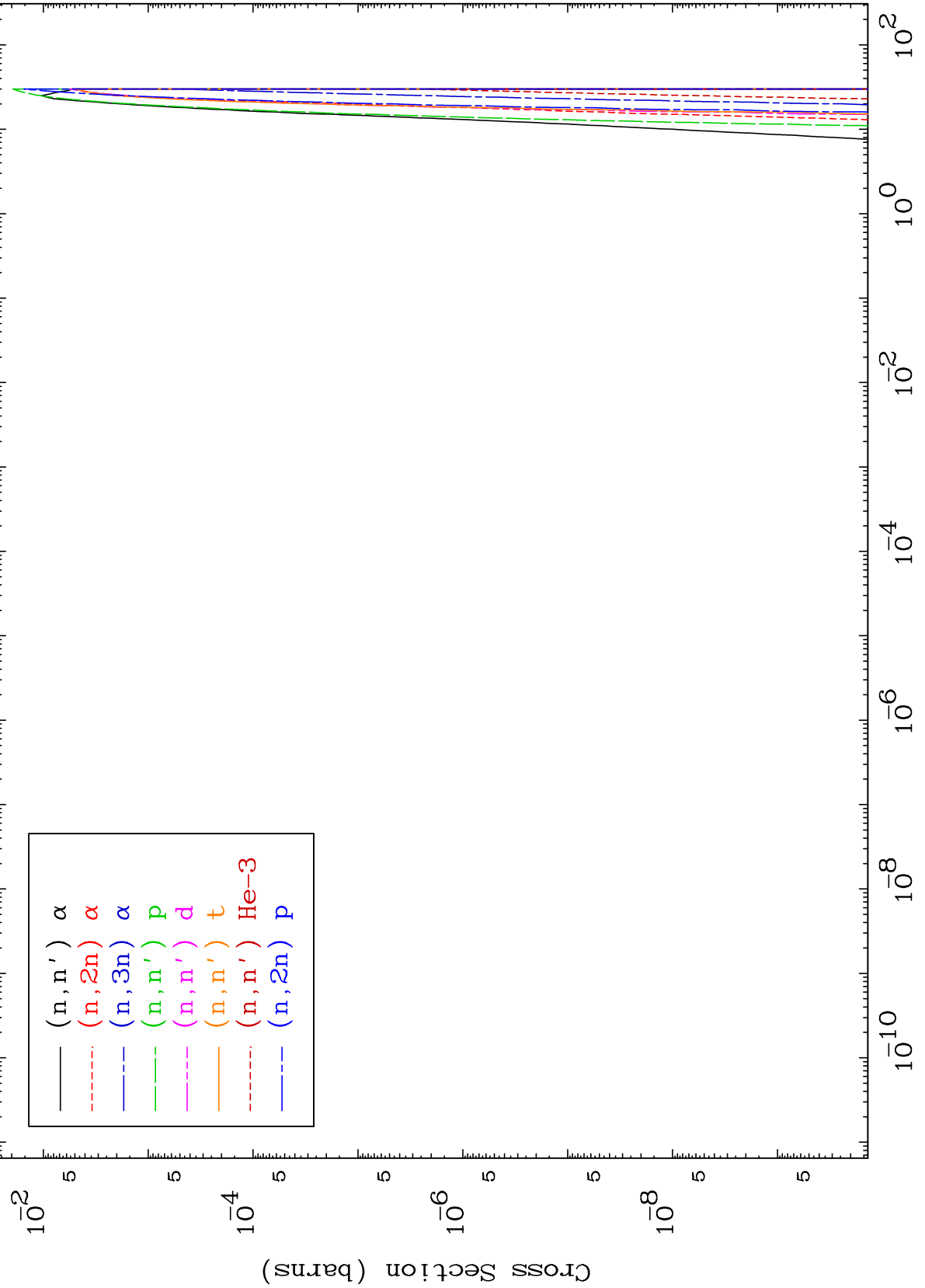
96-Cm-247



MAT 9646

Charged Particle
293 Kelvin Cross Sections

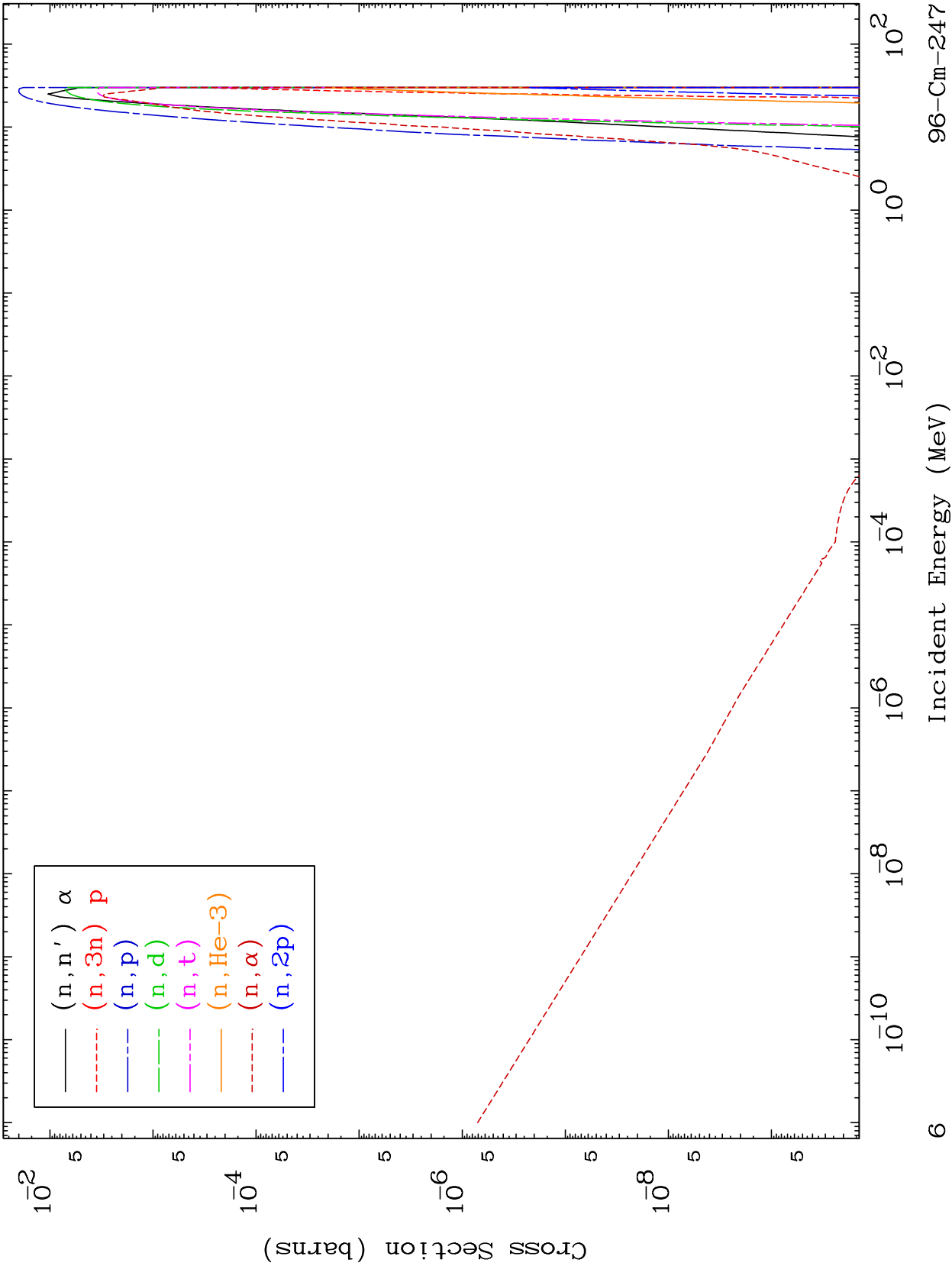
96-Cm-247



MAT 9646

Charged Particle
293 Kelvin Cross Sections

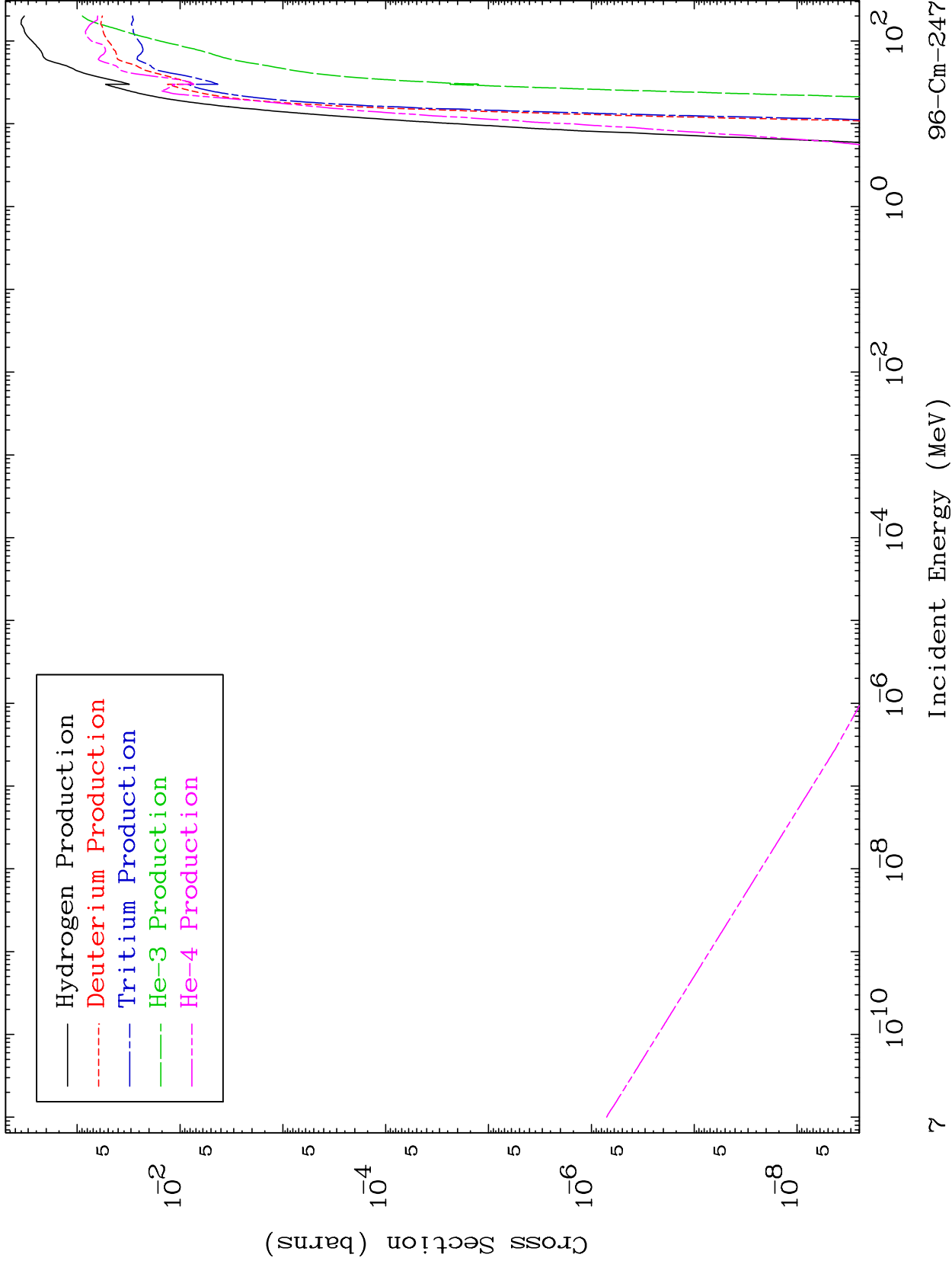
96-Cm-247



MAT 9646

Particle Production
293 Kelvin Cross Sections

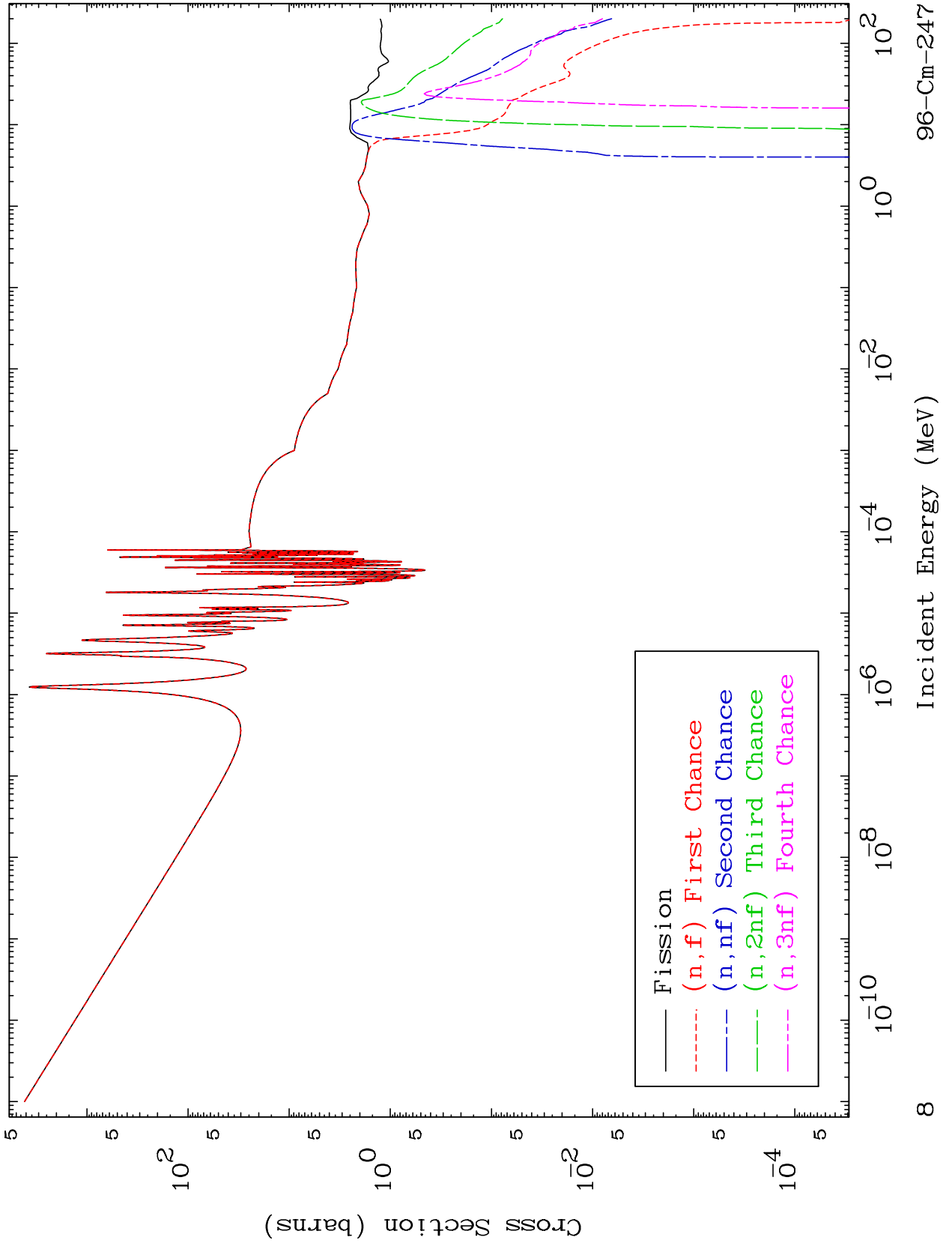
96-Cm-247

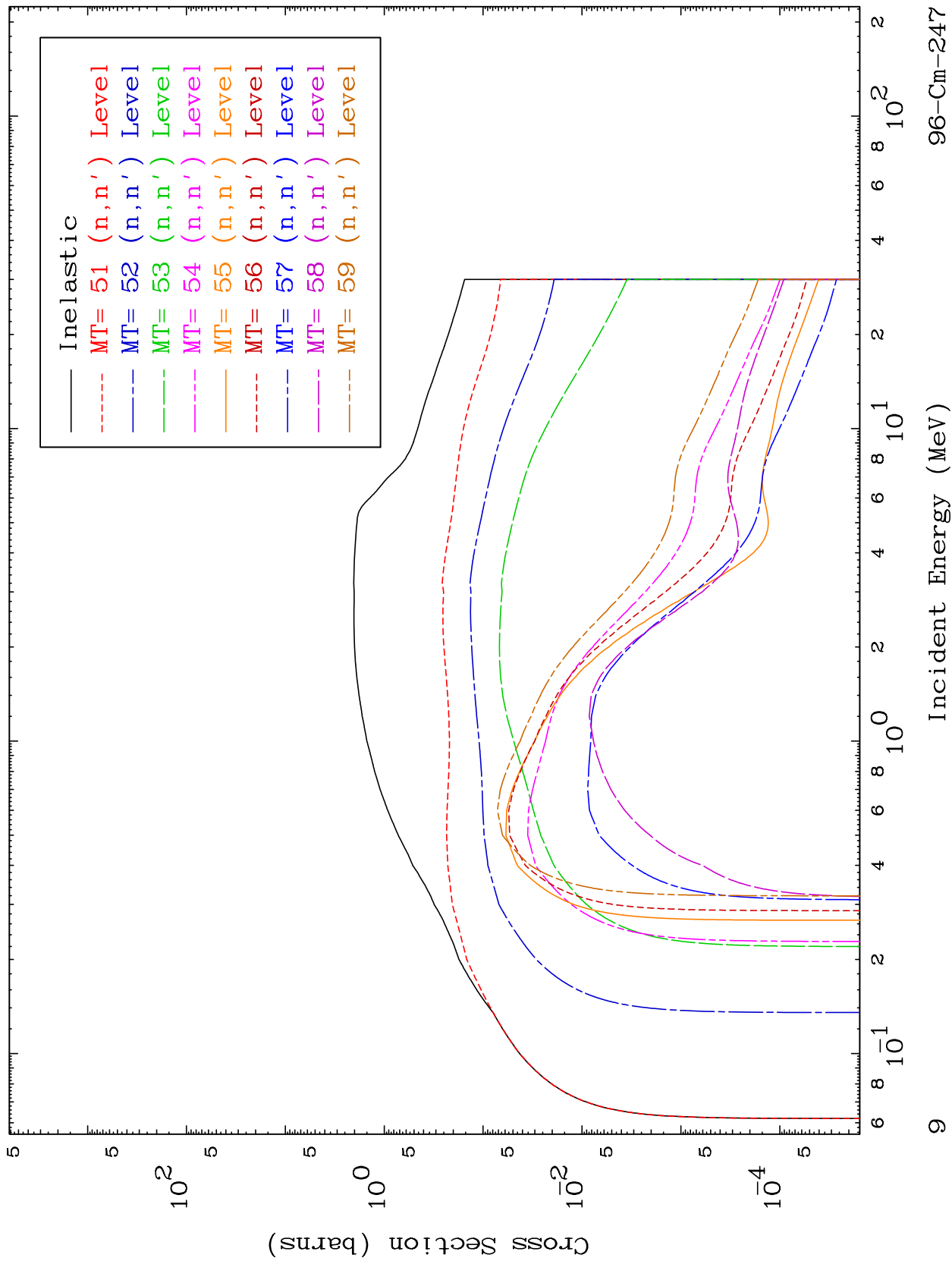


MAT 9646

Fission
293 Kelvin Cross Sections

96-Cm-247



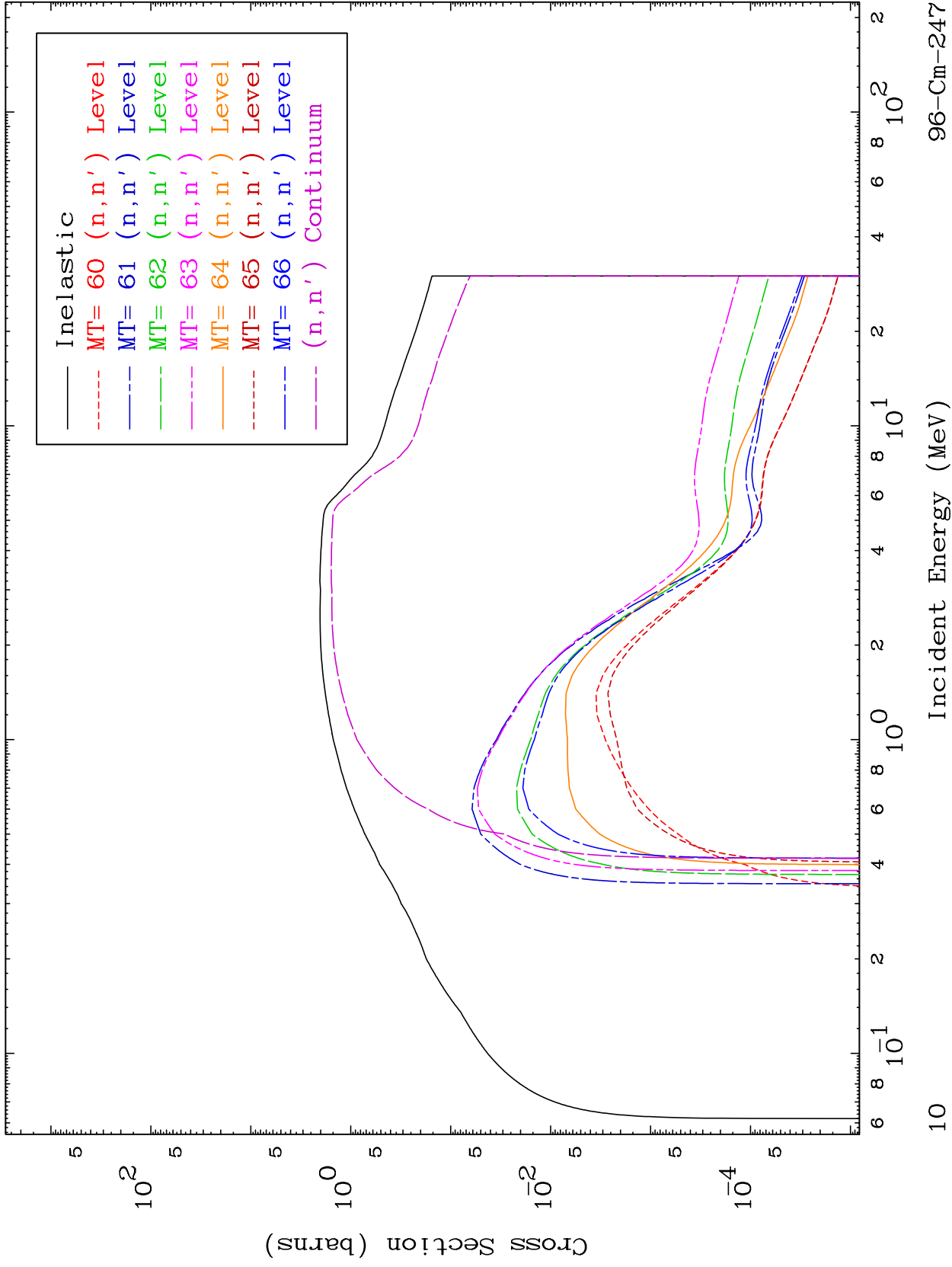


MAT 9646

(n,n') Levels

293 Kelvin Cross Sections

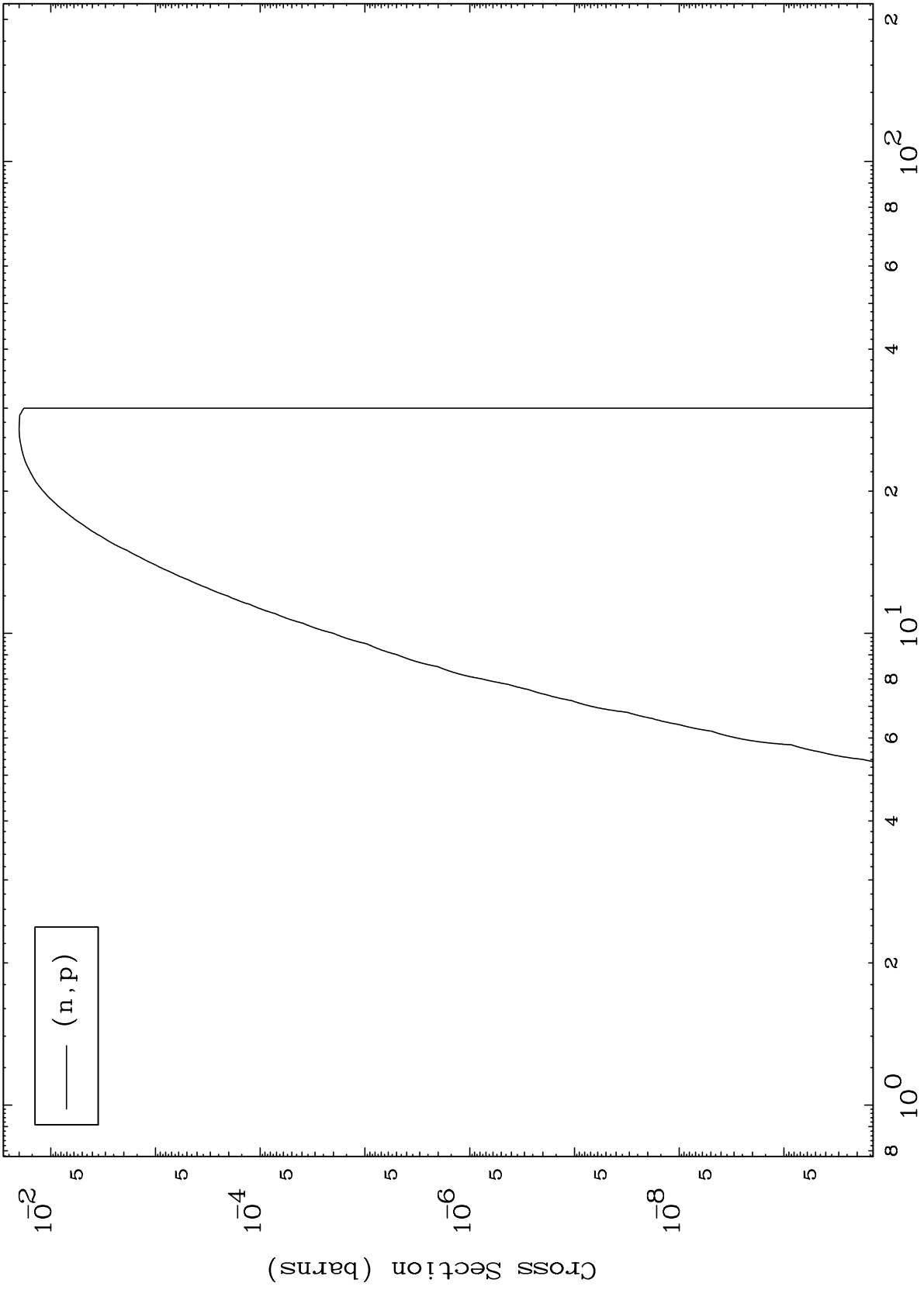
96-Cm-247



MAT 9646

(n,p) Levels
293 Kelvin Cross Sections

96-Cm-247



11

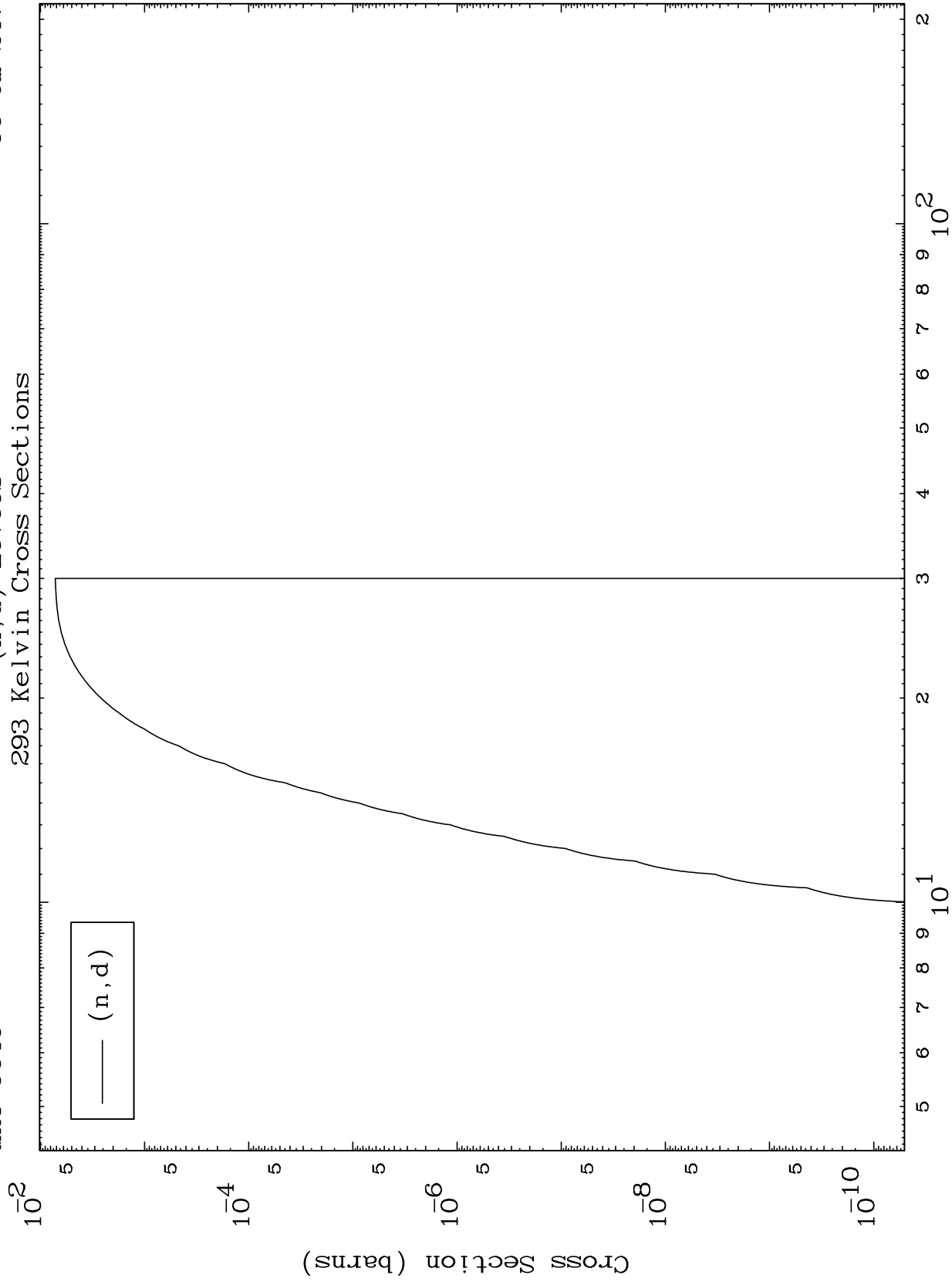
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n,d) Levels
293 Kelvin Cross Sections

96-Cm-247



12

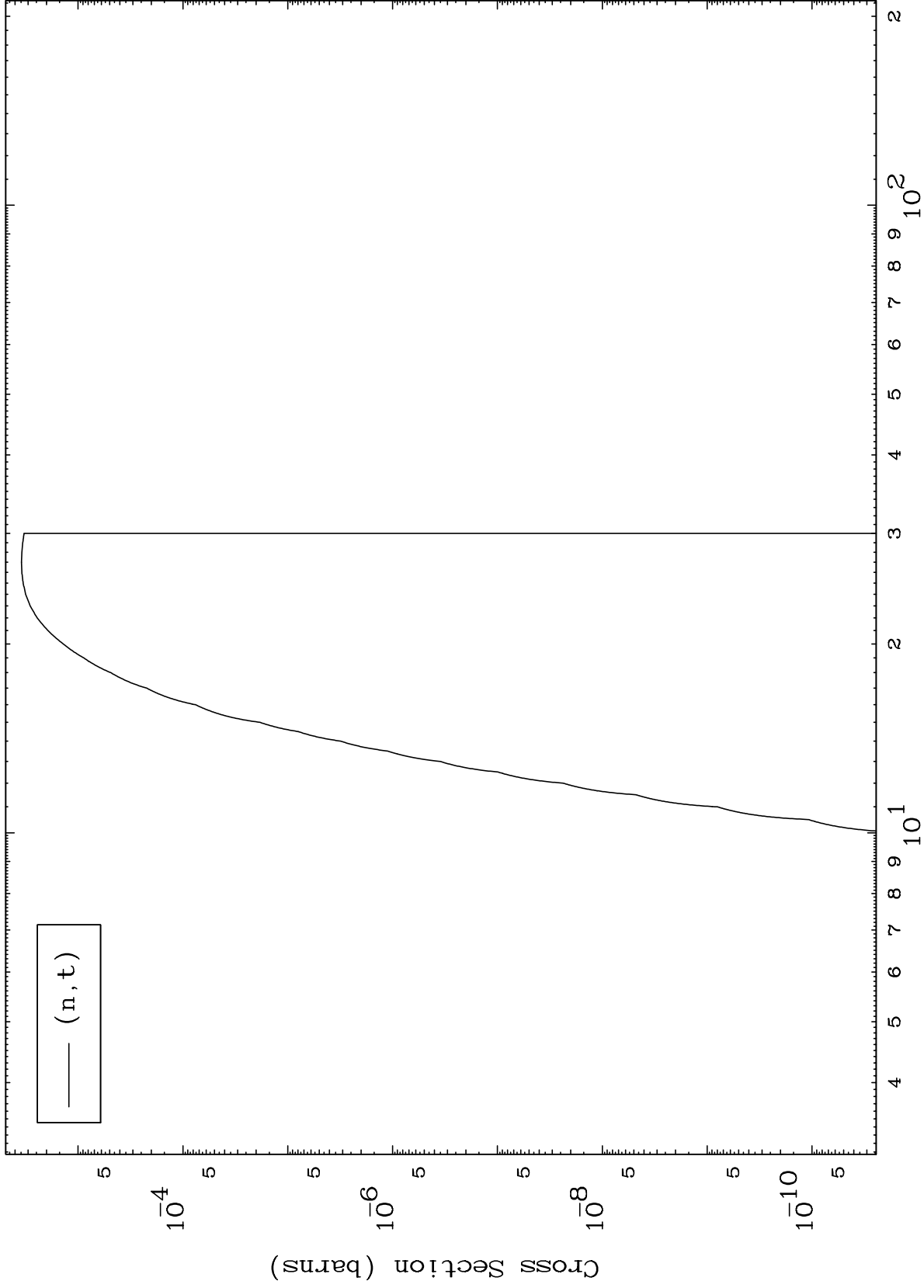
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n,t) Levels
293 Kelvin Cross Sections

96-Cm-247



13

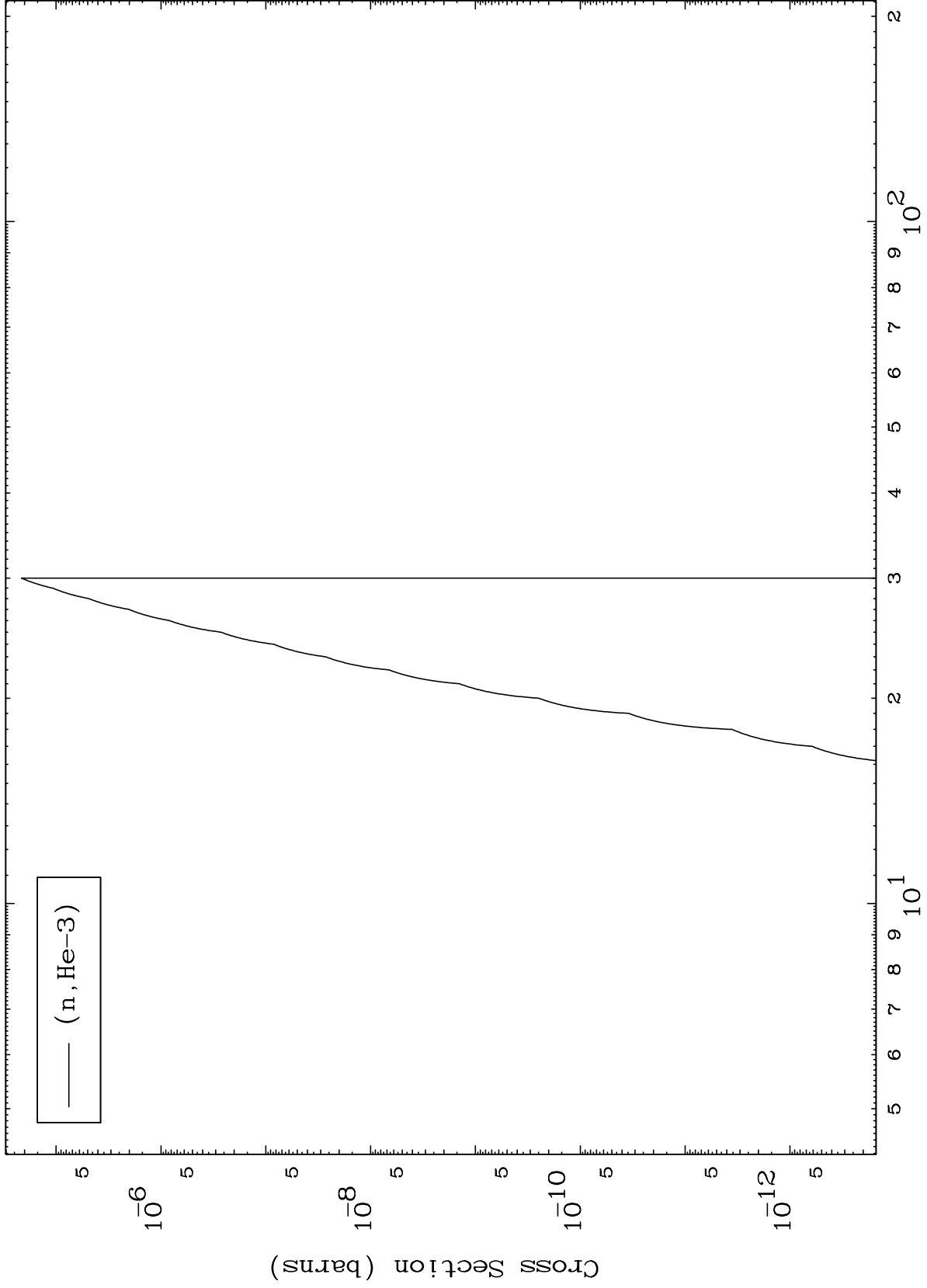
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n,He3) Levels
293 Kelvin Cross Sections

96-Cm-247



14

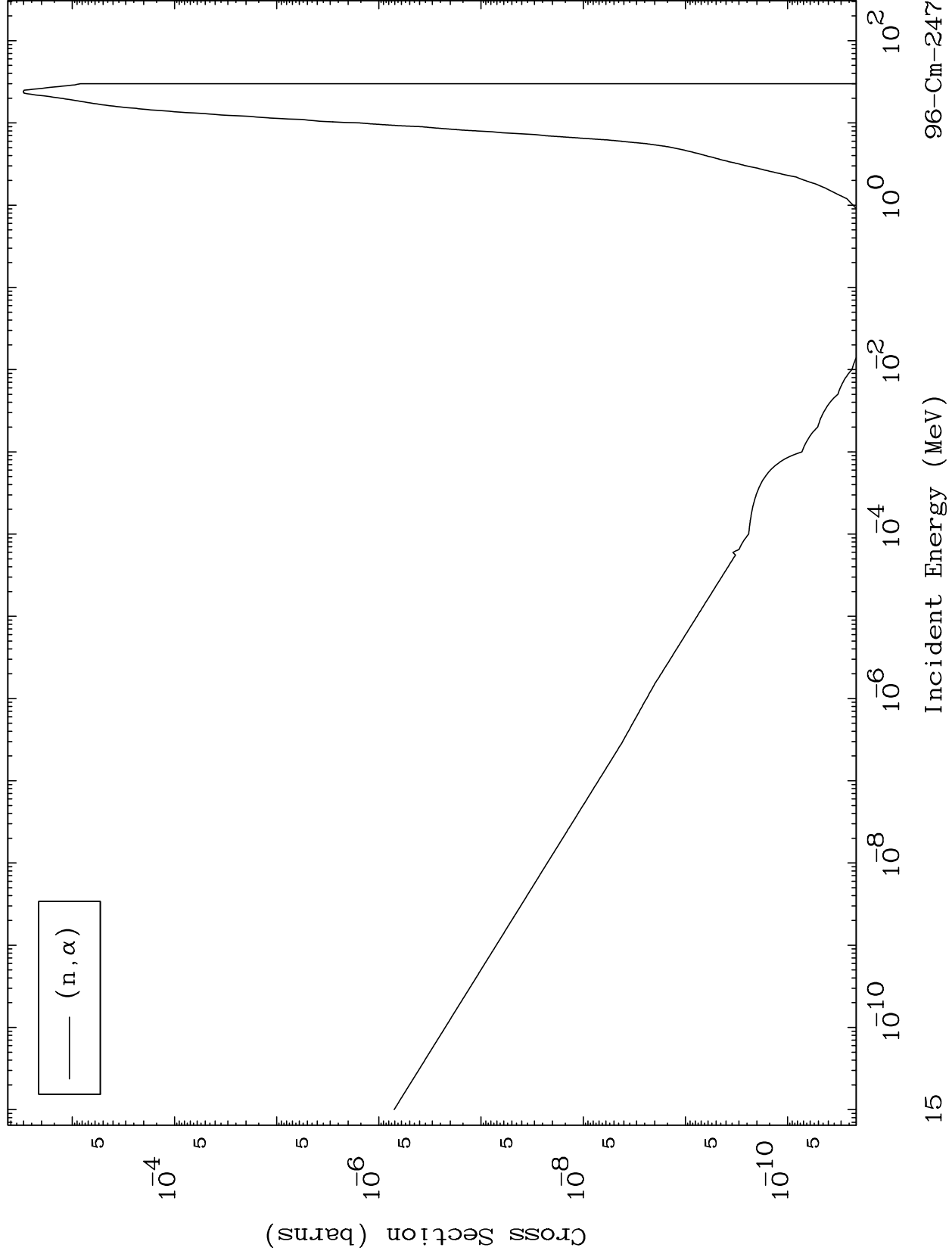
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n, α) Levels
293 Kelvin Cross Sections

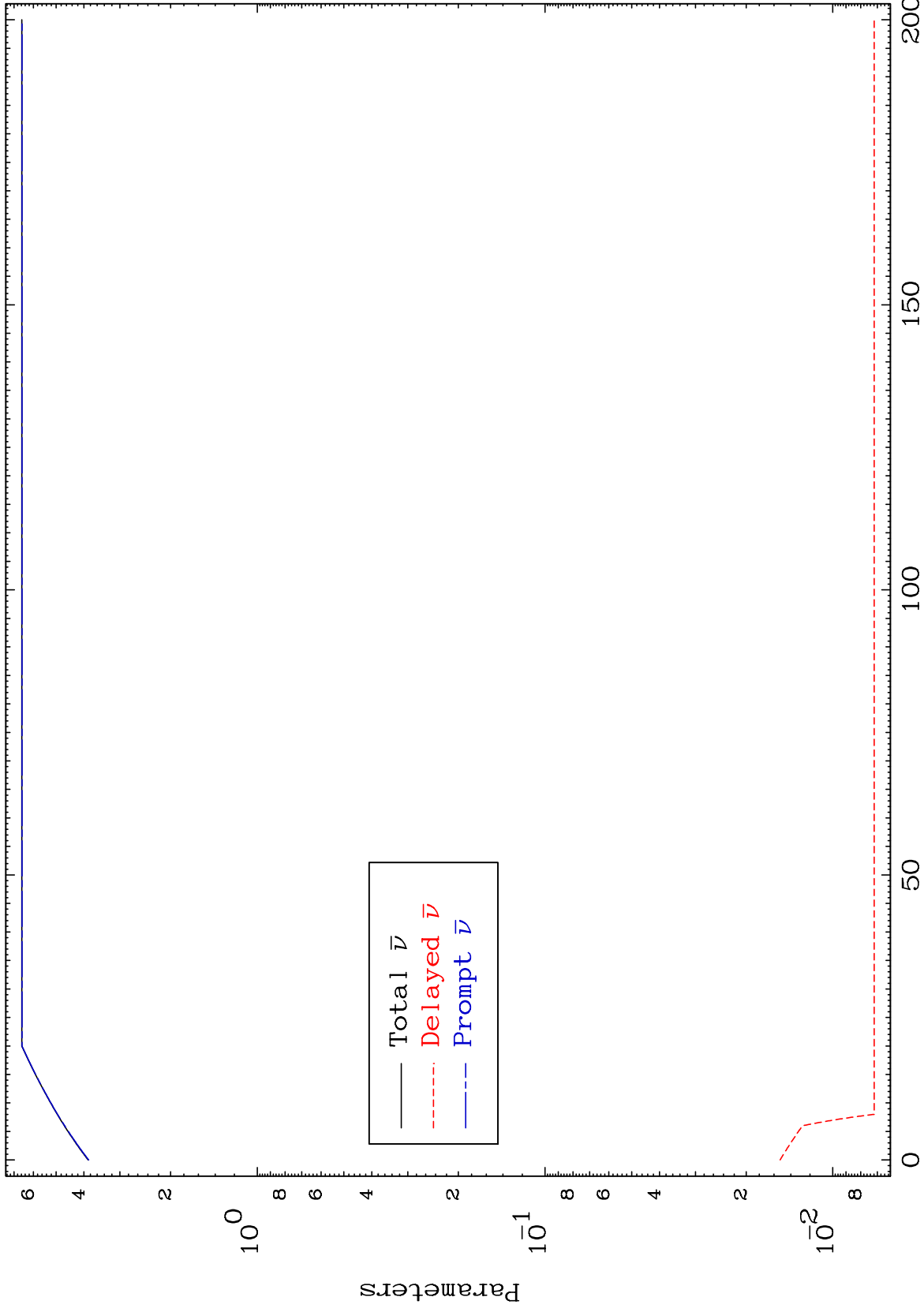
96-Cm-247



MAT 9646

Energy Release
Parameters

96-Cm-247



16

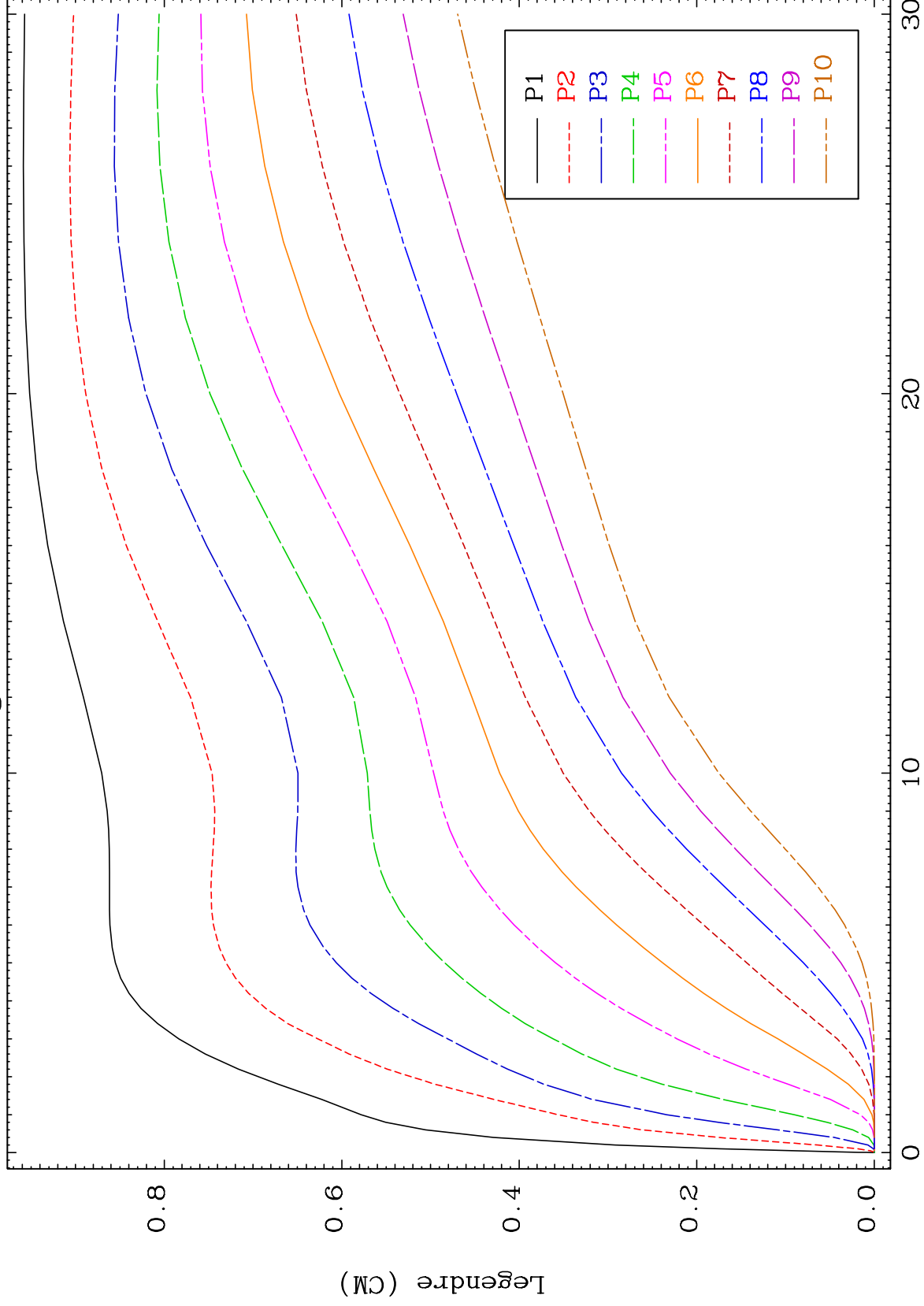
Incident Energy (MeV)

96-Cm-247

MAT 9646

Elastic Legendre Coefficients

96-Cm-247



17

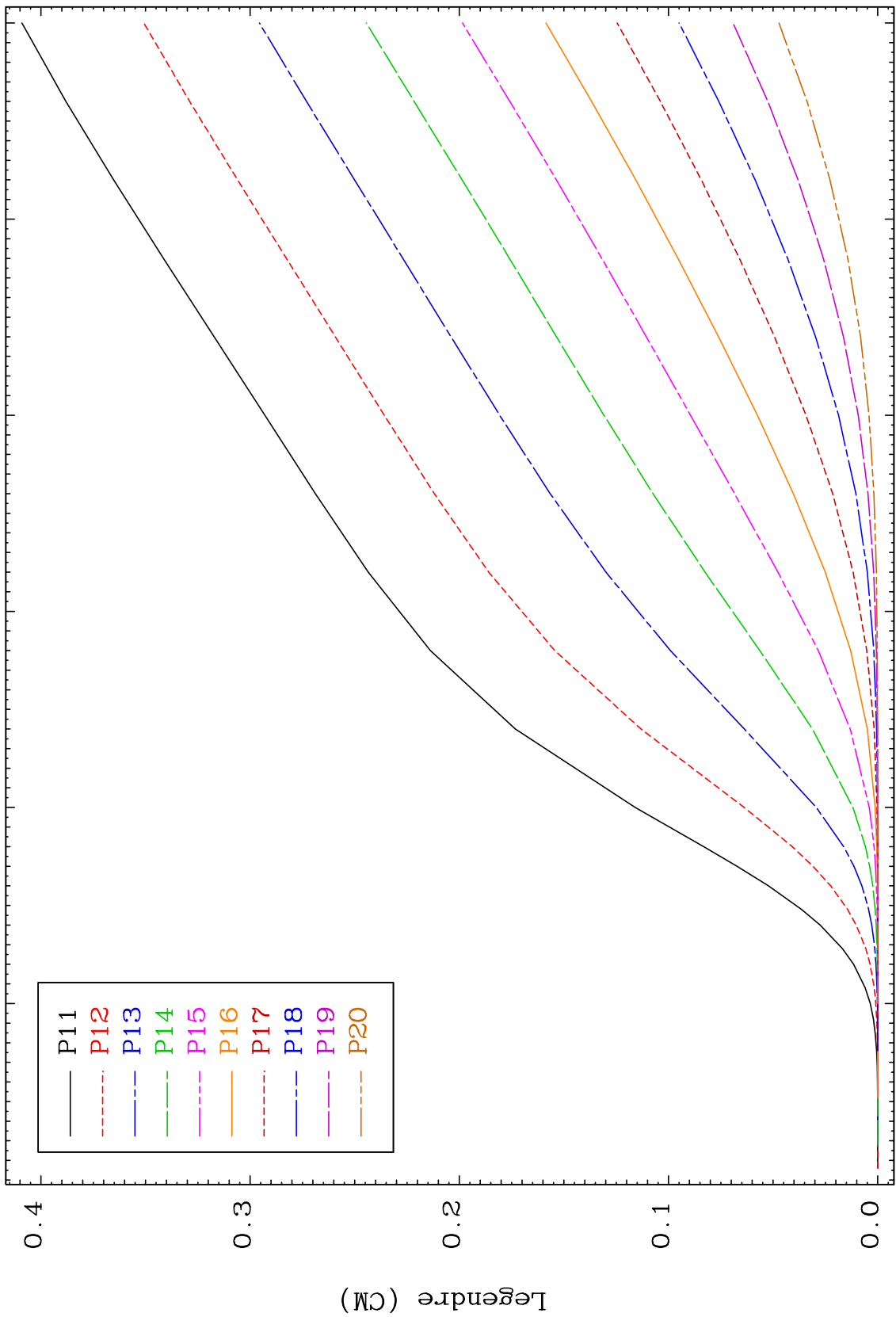
Incident Energy (MeV)

96-Cm-247

MAT 9646

Elastic Legendre Coefficients

96-Cm-247



18

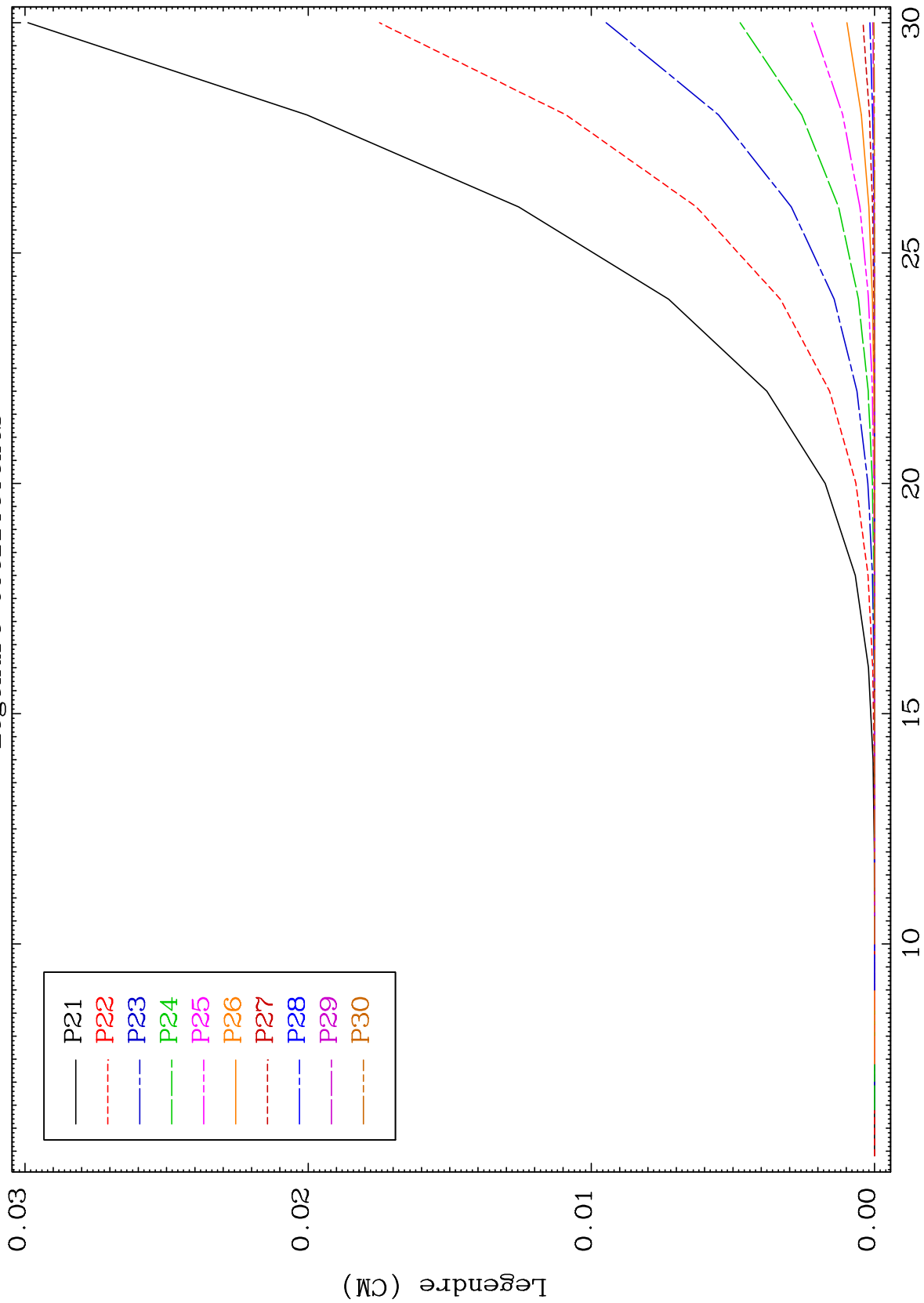
Incident Energy (MeV)

96-Cm-247

MAT 9646

Elastic
Legendre Coefficients

96-Cm-247



19

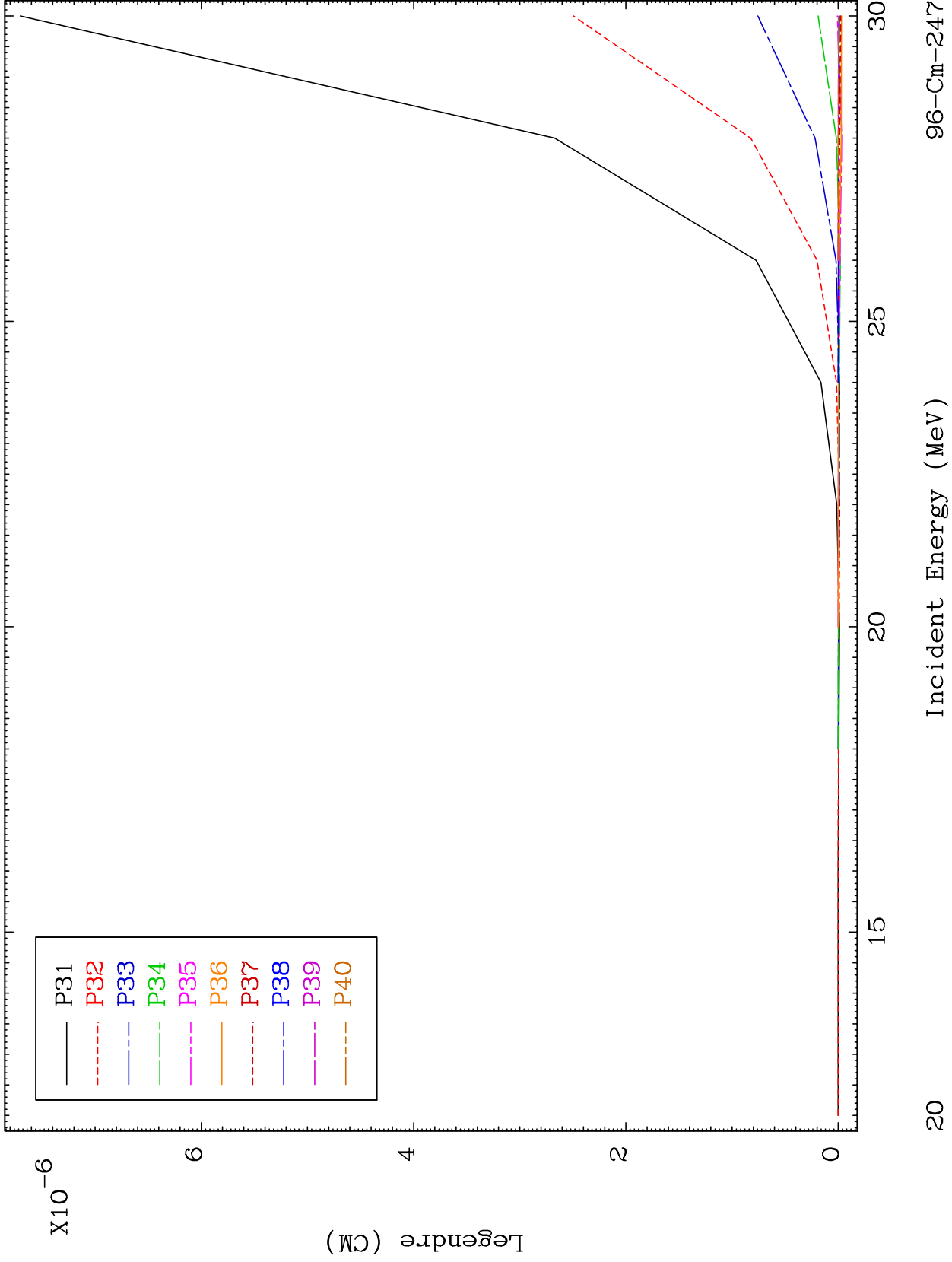
Incident Energy (MeV)

96-Cm-247

MAT 9646

Elastic
Legendre Coefficients

96-Cm-247



20

15

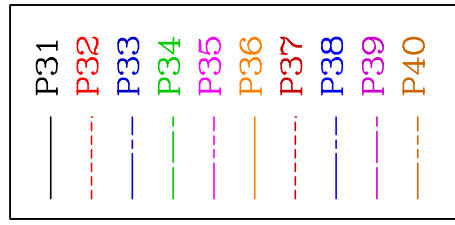
20

25

30

Incident Energy (MeV)

96-Cm-247



$\times 10^{-6}$

Legendre (CM)

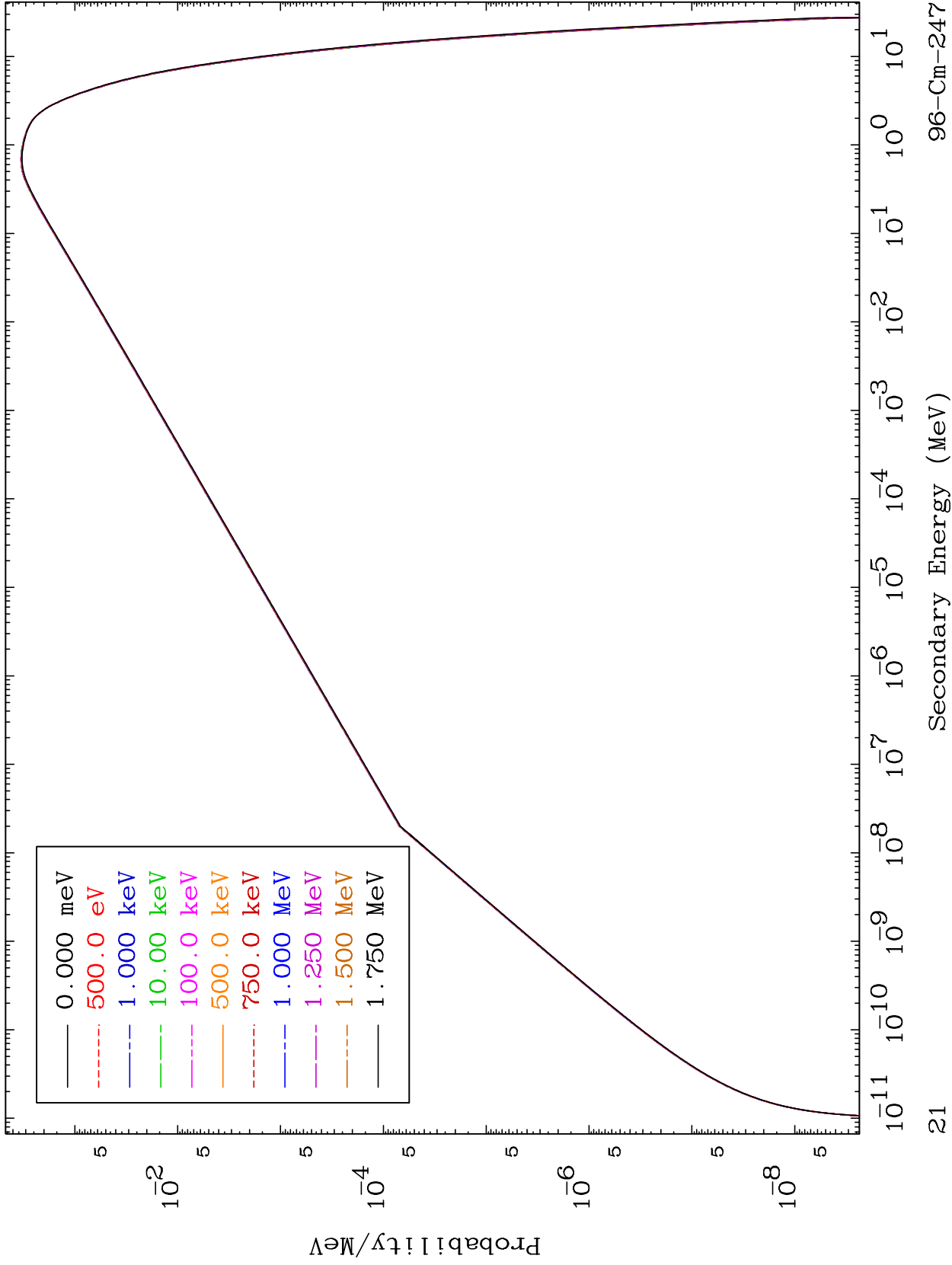
6

4

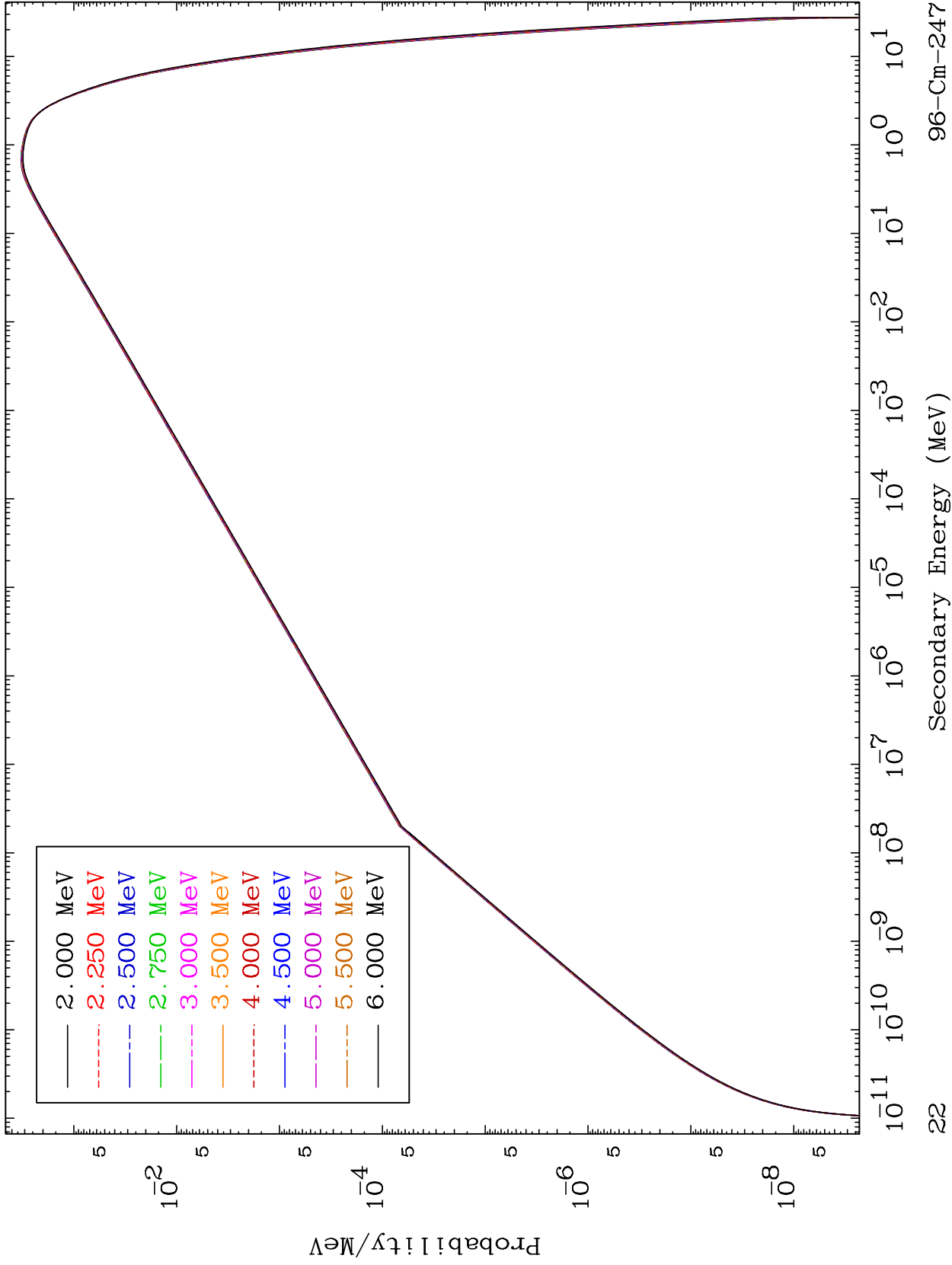
2

0

Fission Energy Distributions



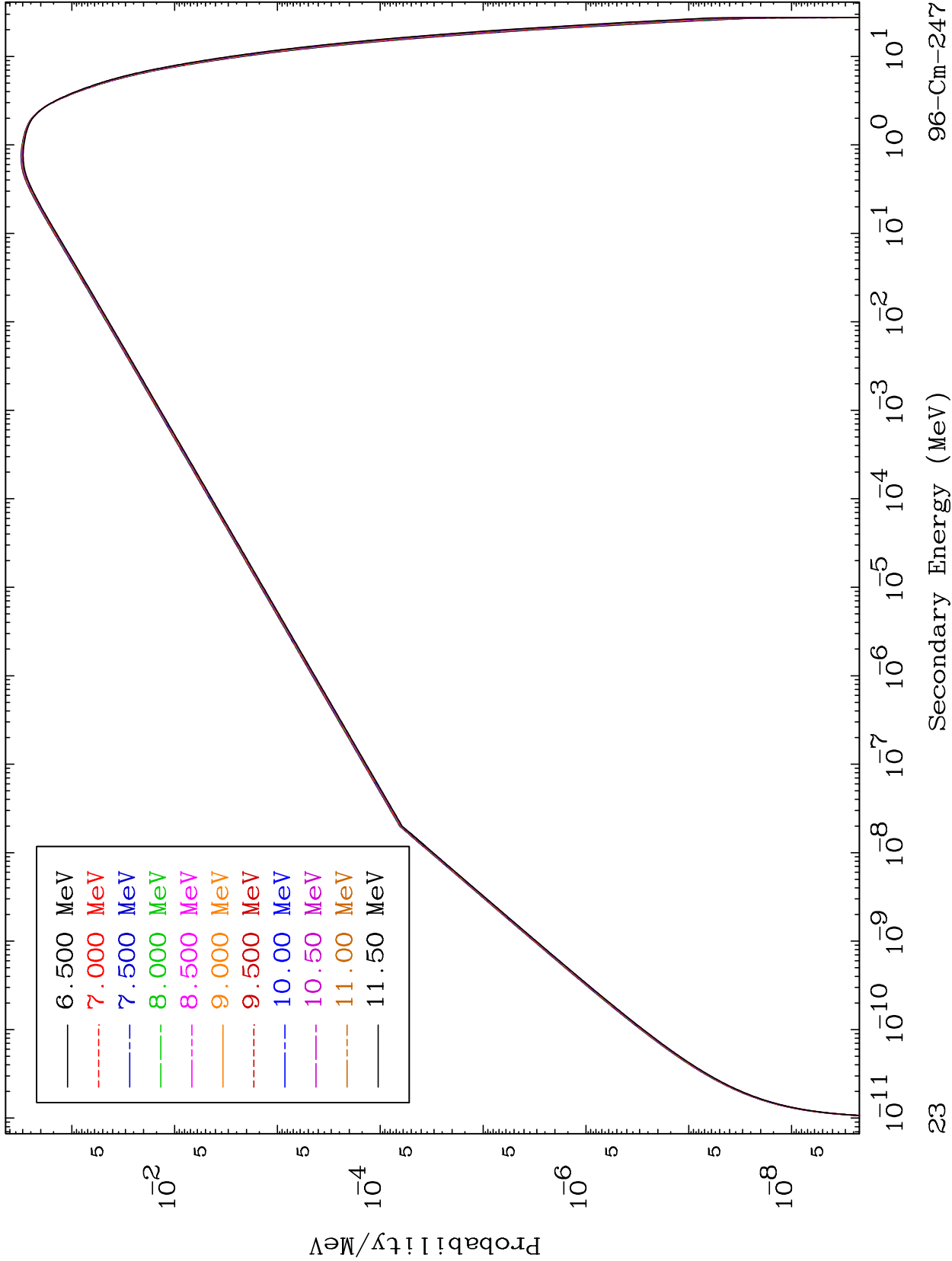
Fission Energy Distributions



MAT 9646

Fission Energy Distributions

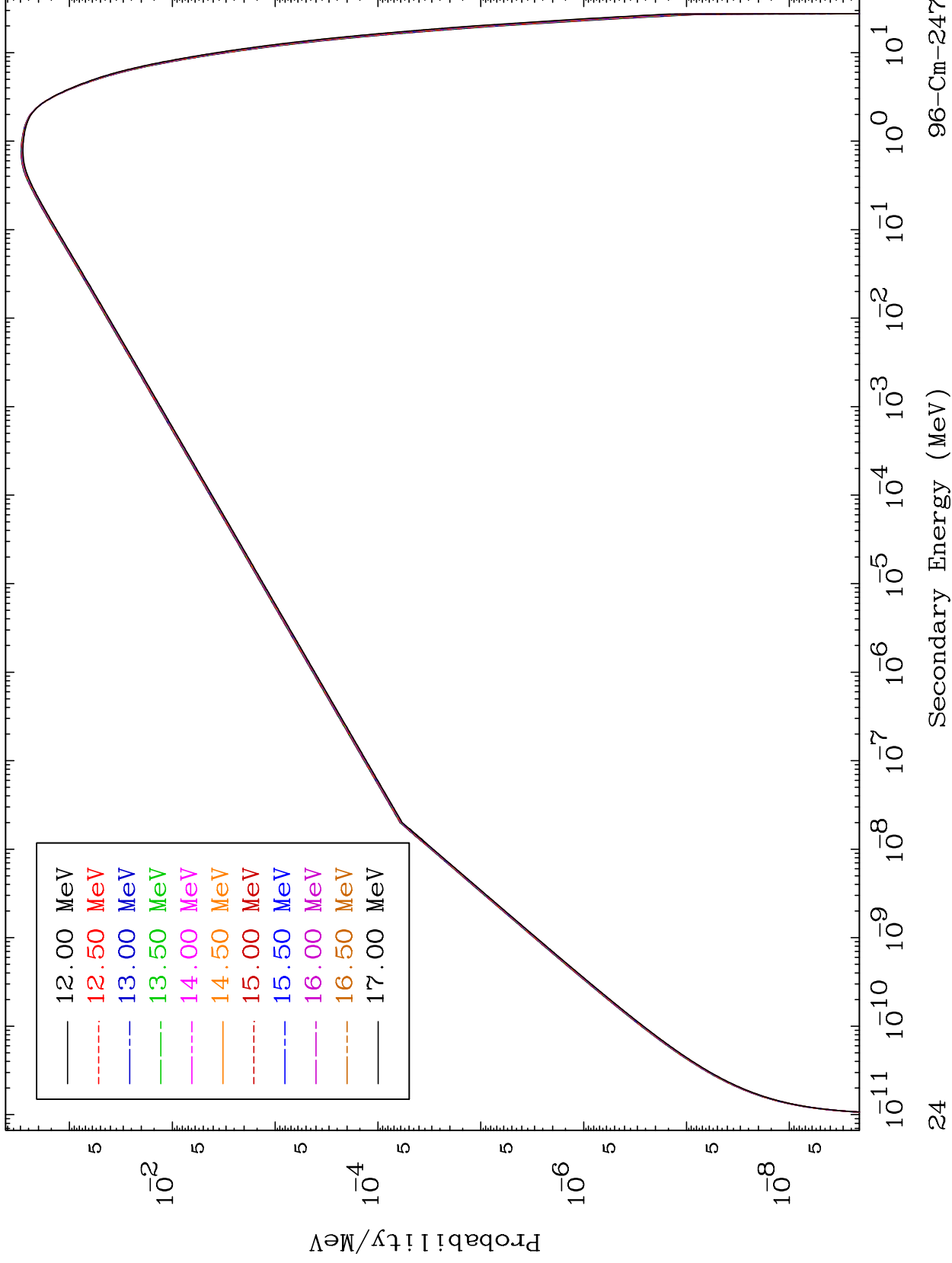
96-Cm-247



MAT 9646

Fission Energy Distributions

96-Cm-247



96-Cm-247

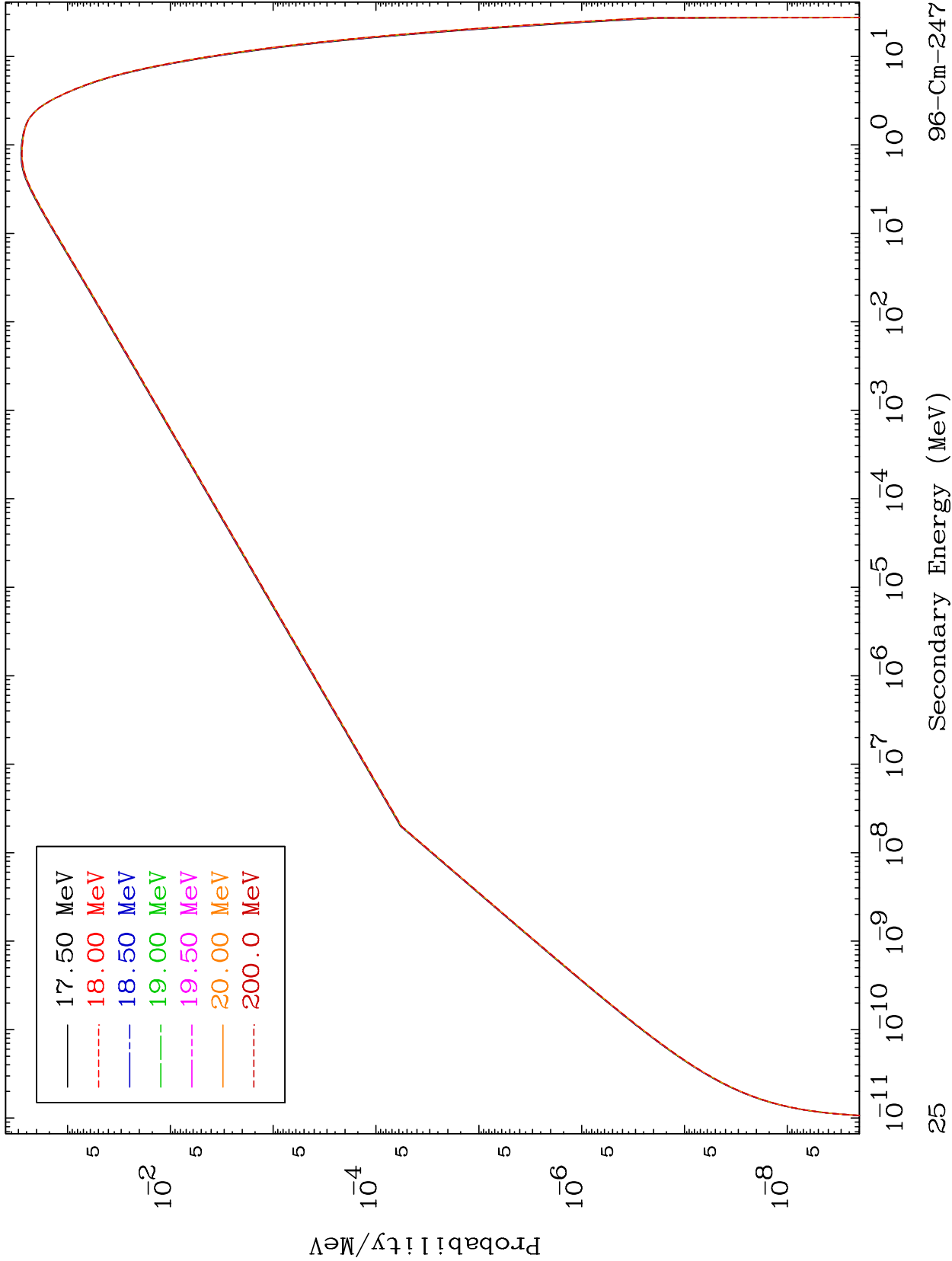
Secondary Energy (MeV)

24

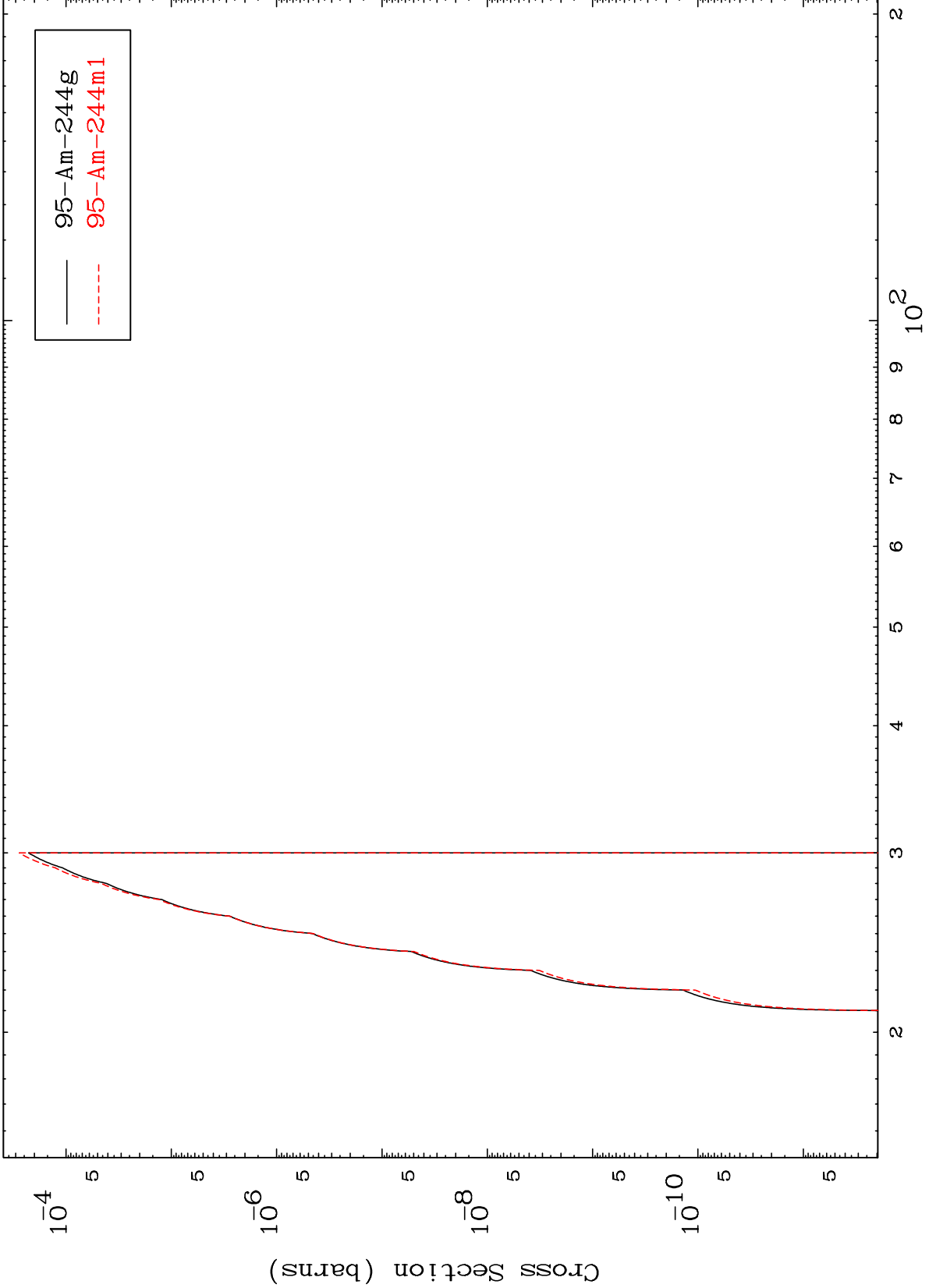
MAT 9646

Fission Energy Distributions

96-Cm-247



Radionuclide Production Cross Section



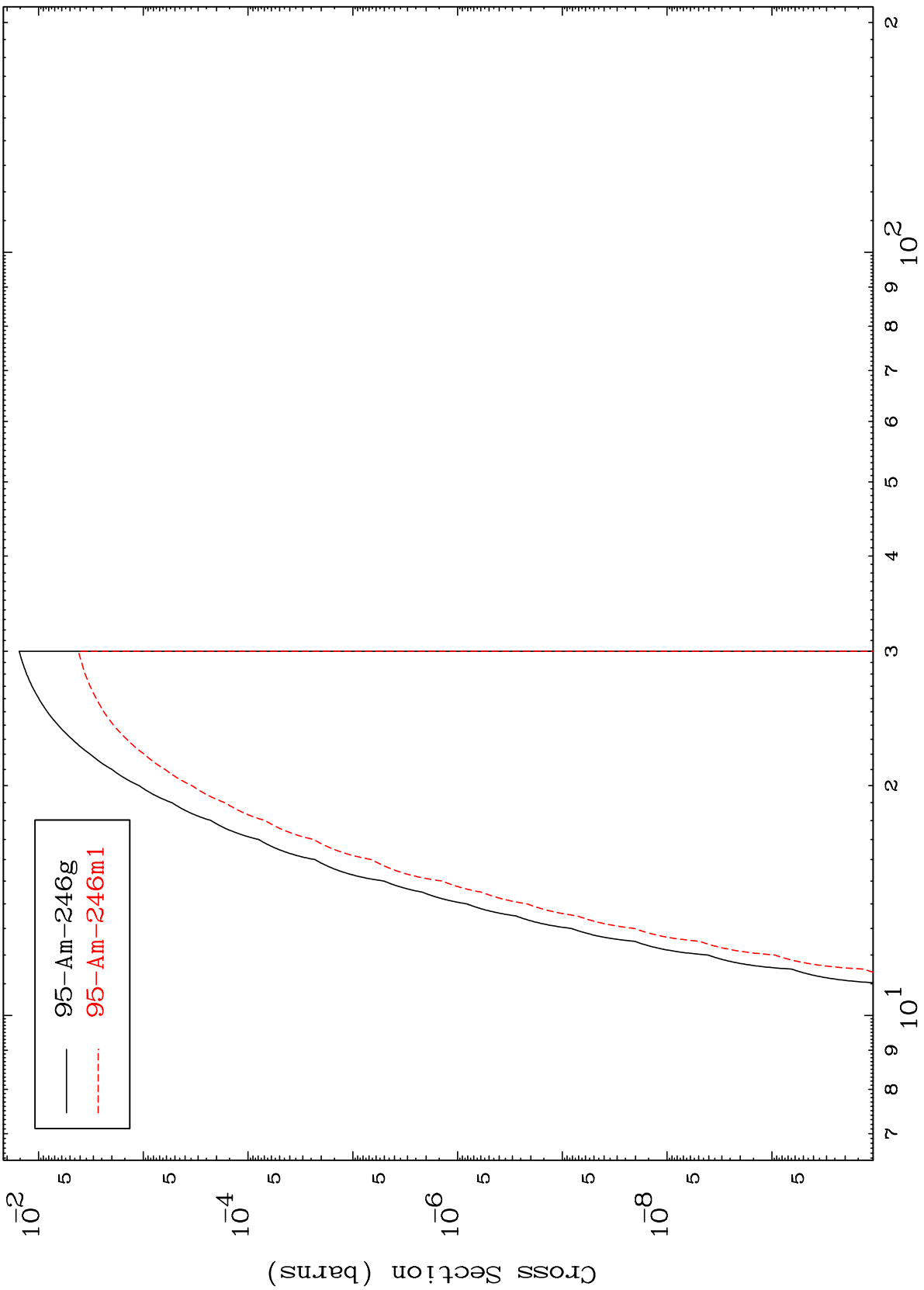
95-Am-244g
95-Am-244m1

MAT 9646

(n,n') p

96-Cm-247

Radionuclide Production Cross Section



95-Am-246g
95-Am-246m1

27

Incident Energy (MeV)

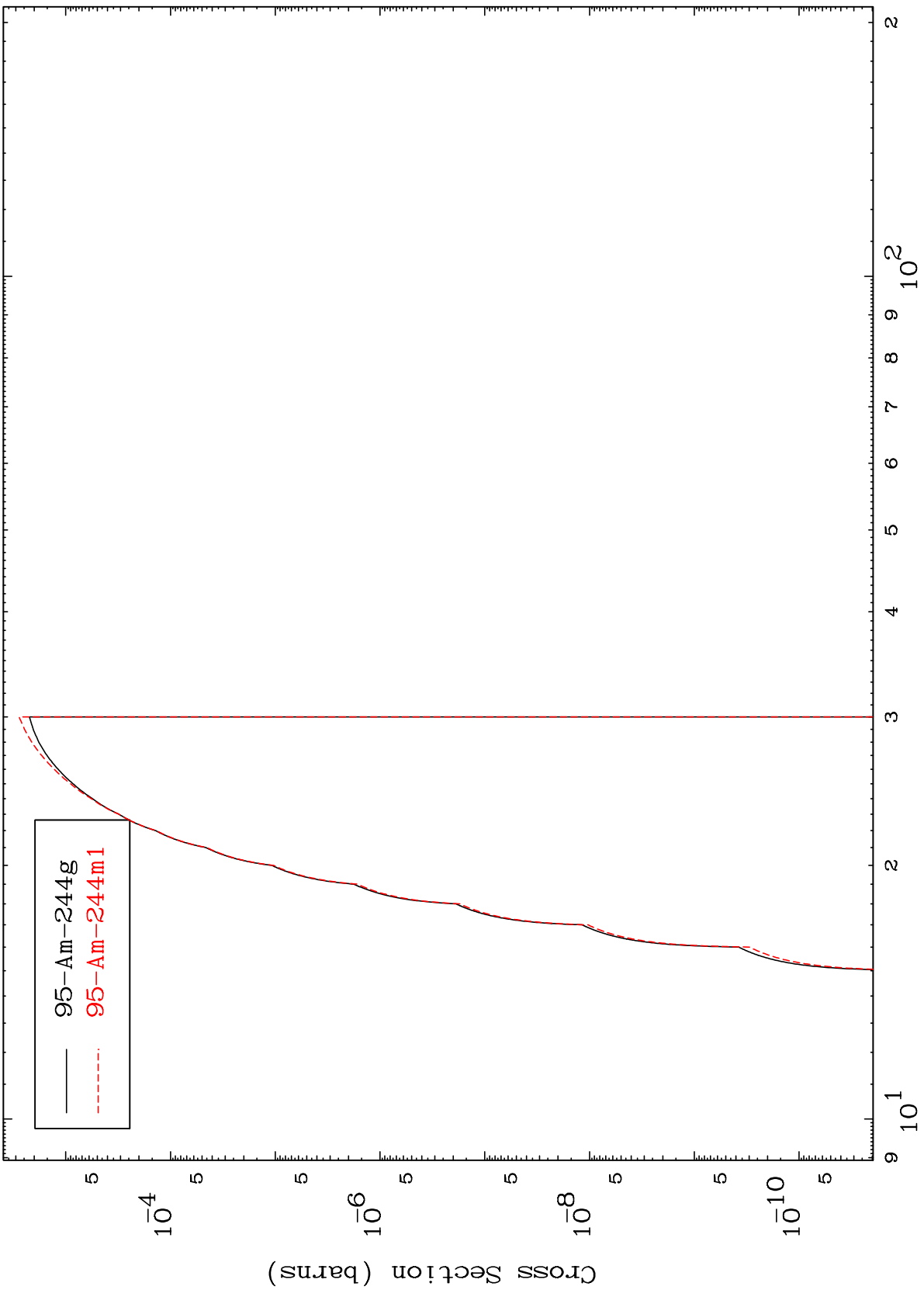
96-Cm-247

MAT 9646

(n,n') t

96-Cm-247

Radionuclide Production Cross Section



28

Incident Energy (MeV)

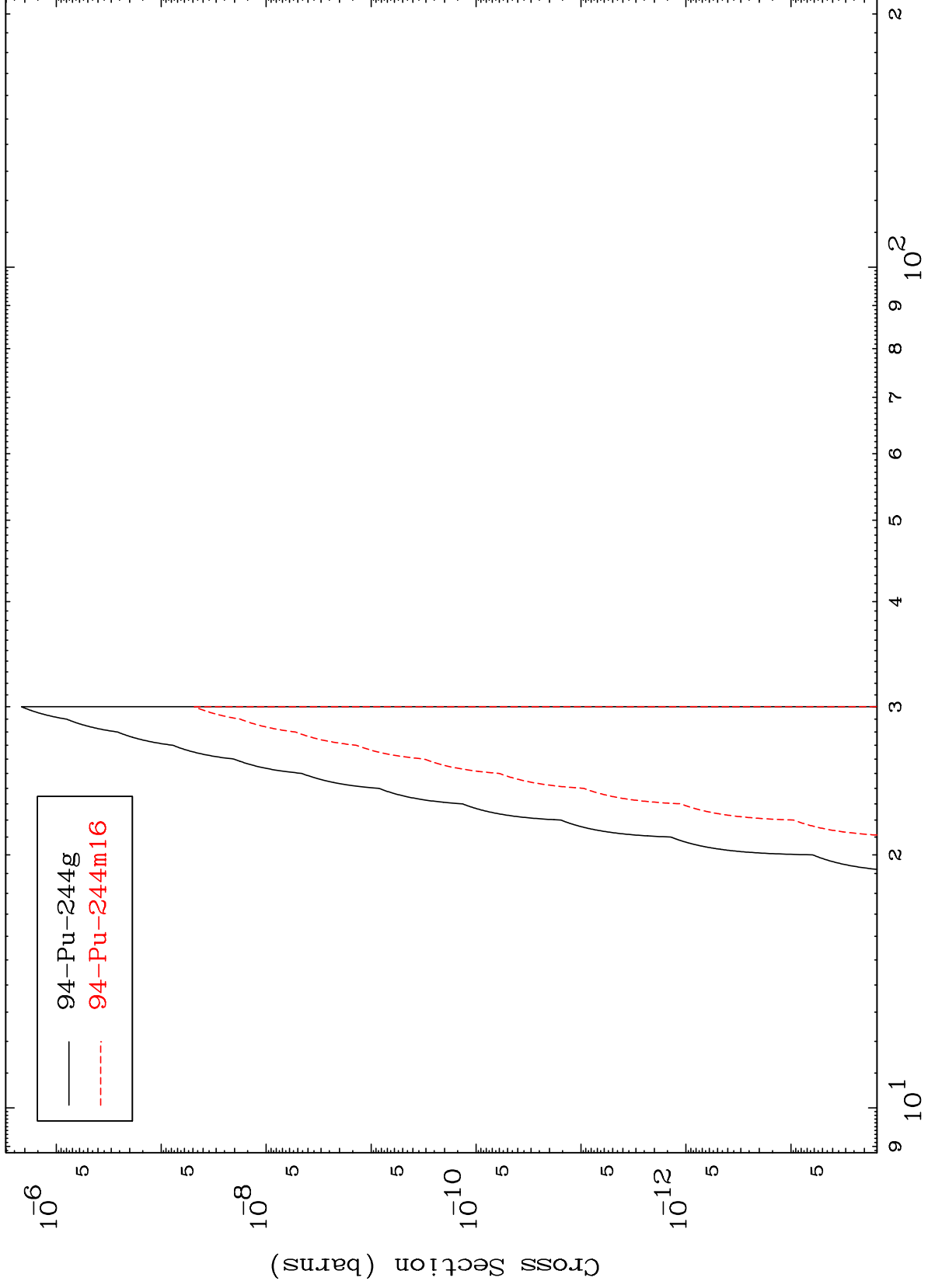
96-Cm-247

MAT 9646

(n,n') He-3

96-Cm-247

Radionuclide Production Cross Section



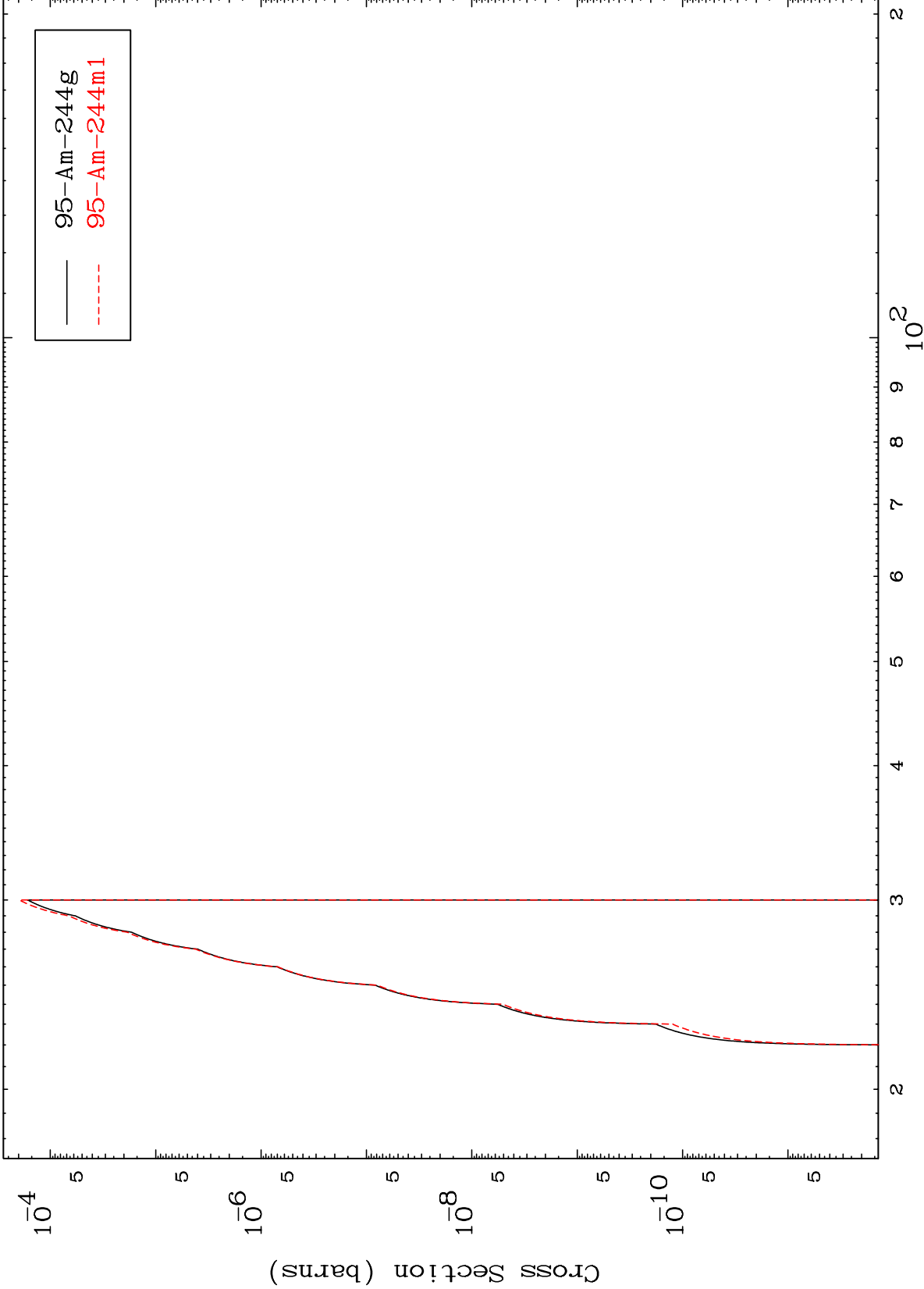
94-Pu-244g
94-Pu-244m16

29

Incident Energy (MeV)

96-Cm-247

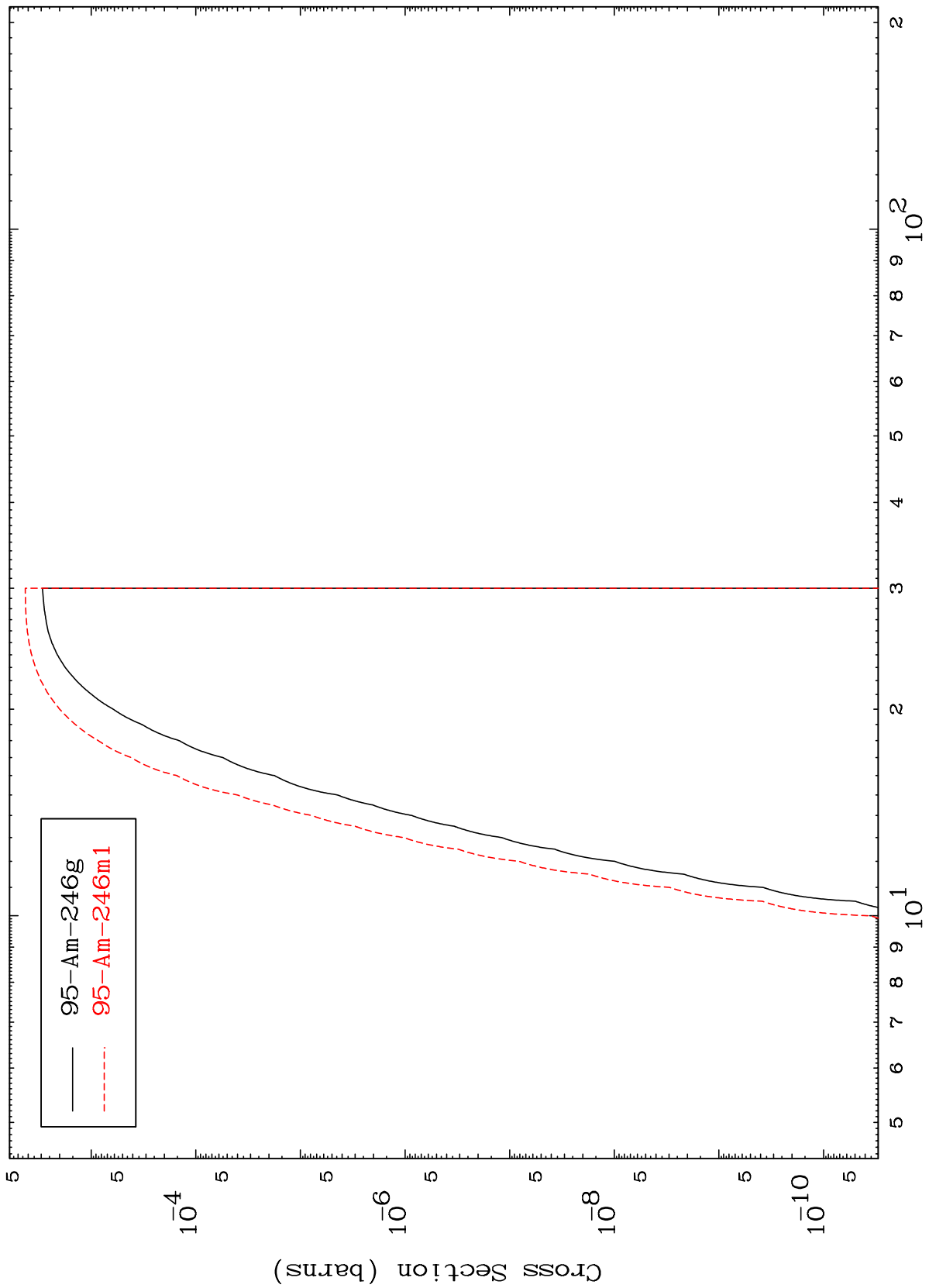
Radionuclide Production Cross Section



MAT 9646

96-Cm-247

(n,d)
Radionuclide Production Cross Section



31

Incident Energy (MeV)

96-Cm-247

MAT 9646

Radionuclide Production Cross Section
(n, α)

96-Cm-247

