

Program EVALPLOT
(Version 2021-1)

by

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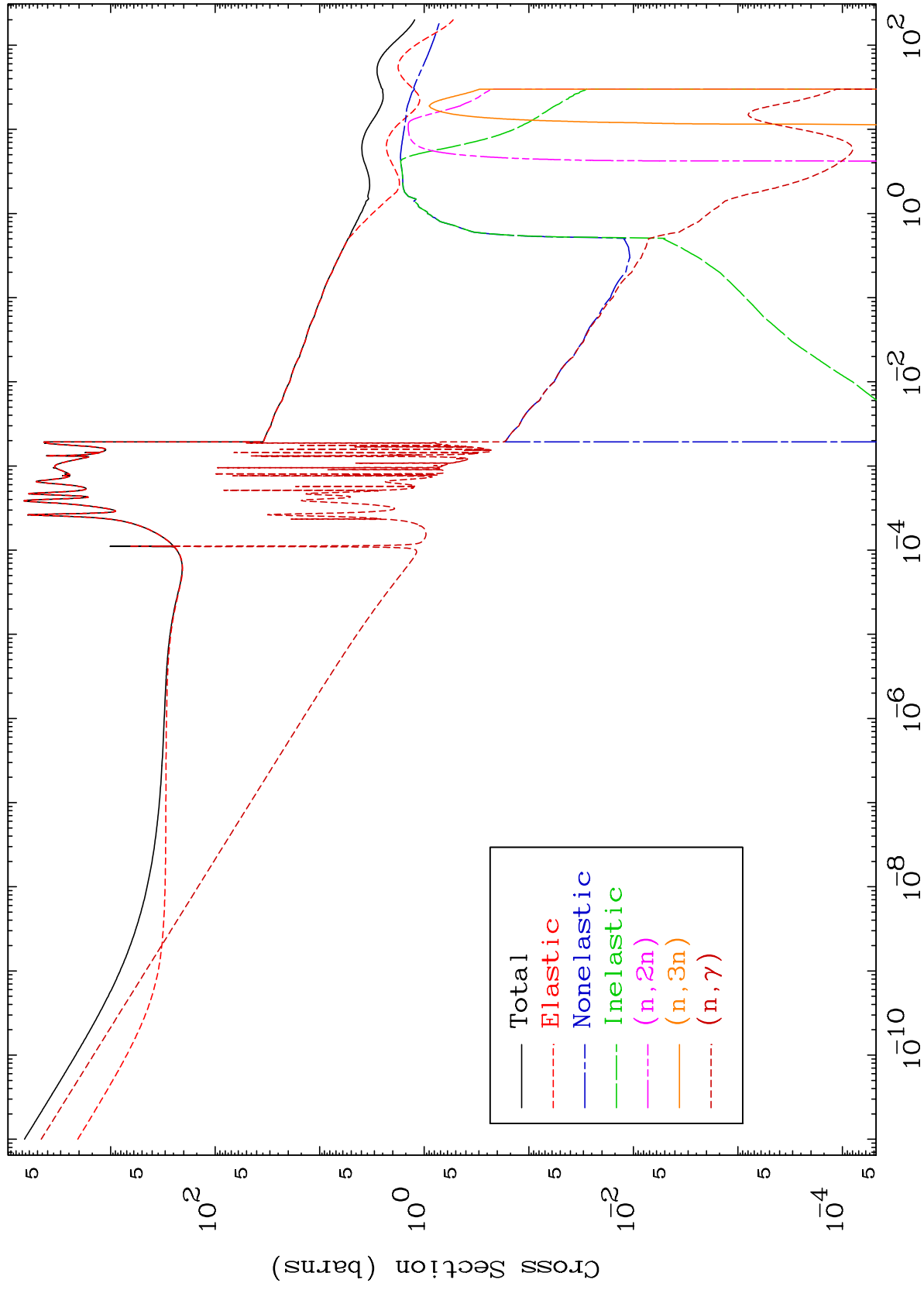
Press Mouse Button to Start

MAT 2753

Neutron Major

293 Kelvin Cross Sections

27-Co-68m



1

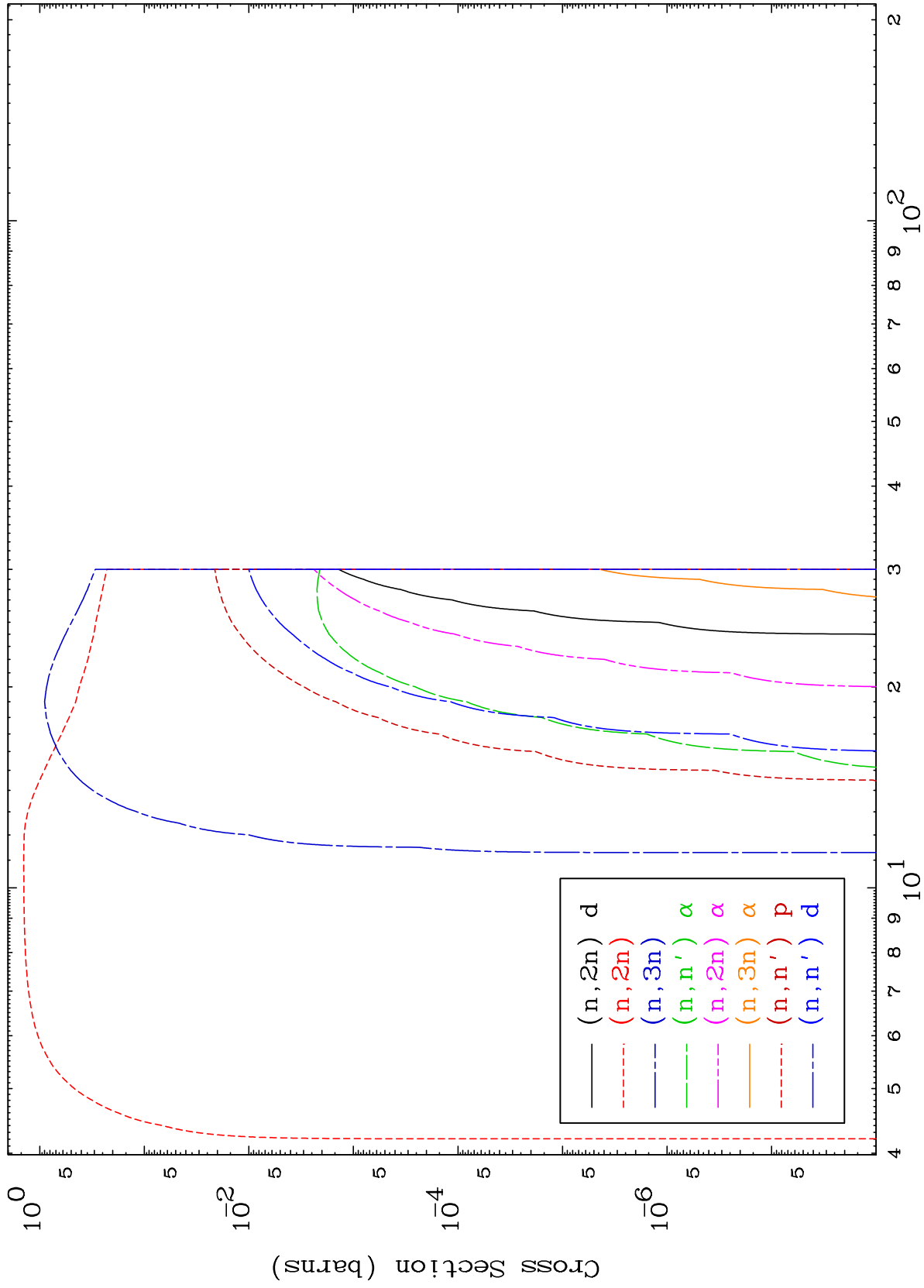
Incident Energy (MeV)

27-Co-68m

MAT 2753

Neutron Absorption
293 Kelvin Cross Sections

27-Co-68m

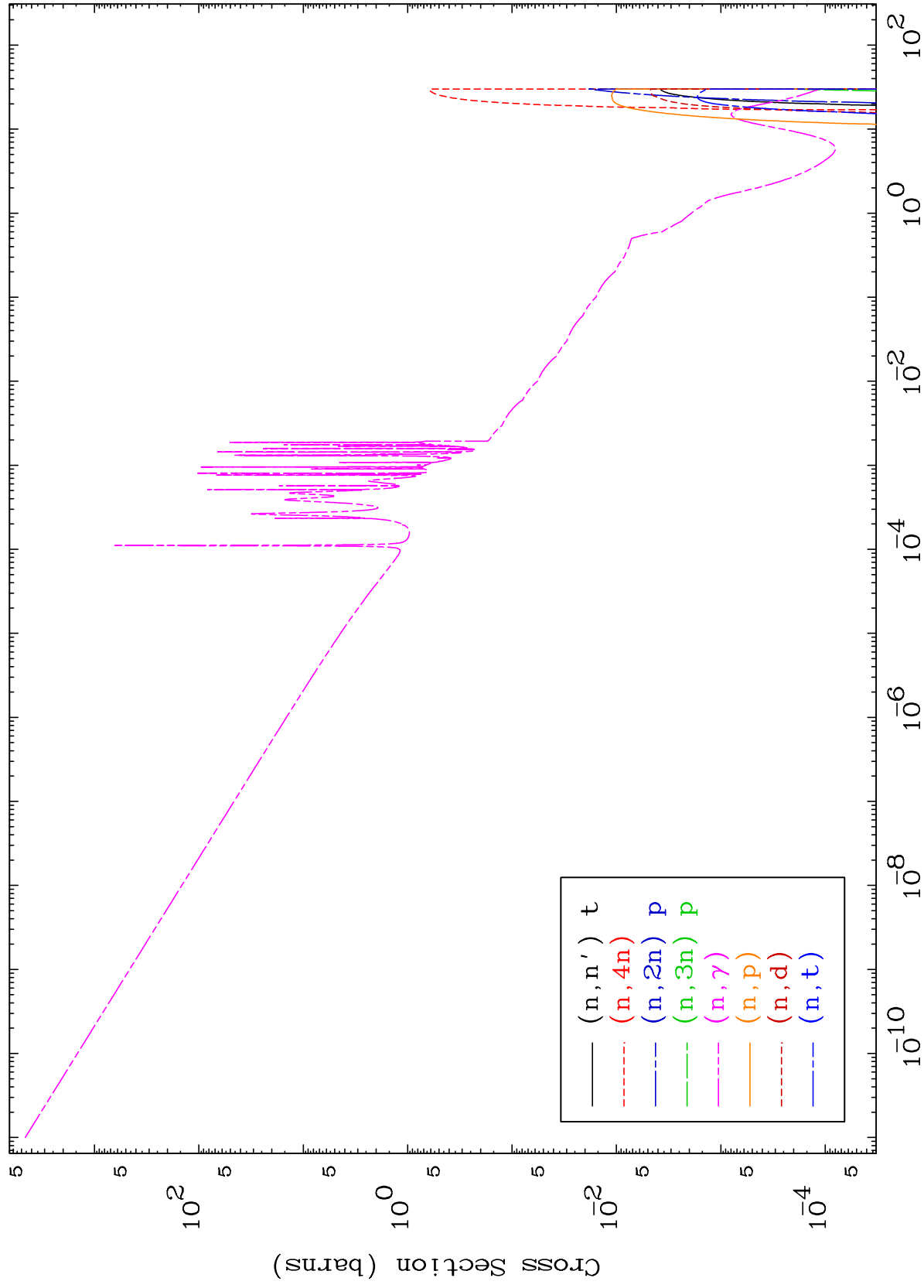


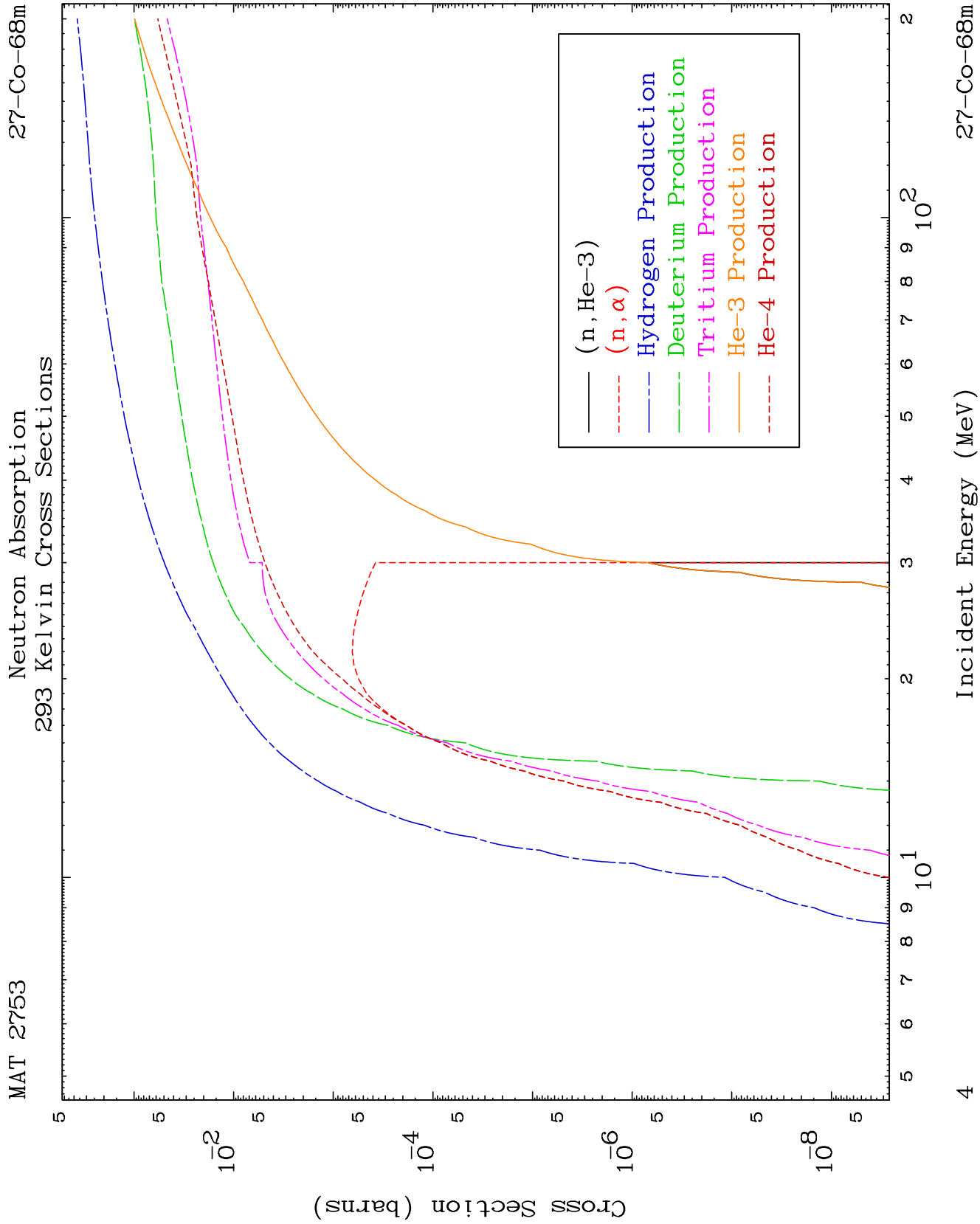
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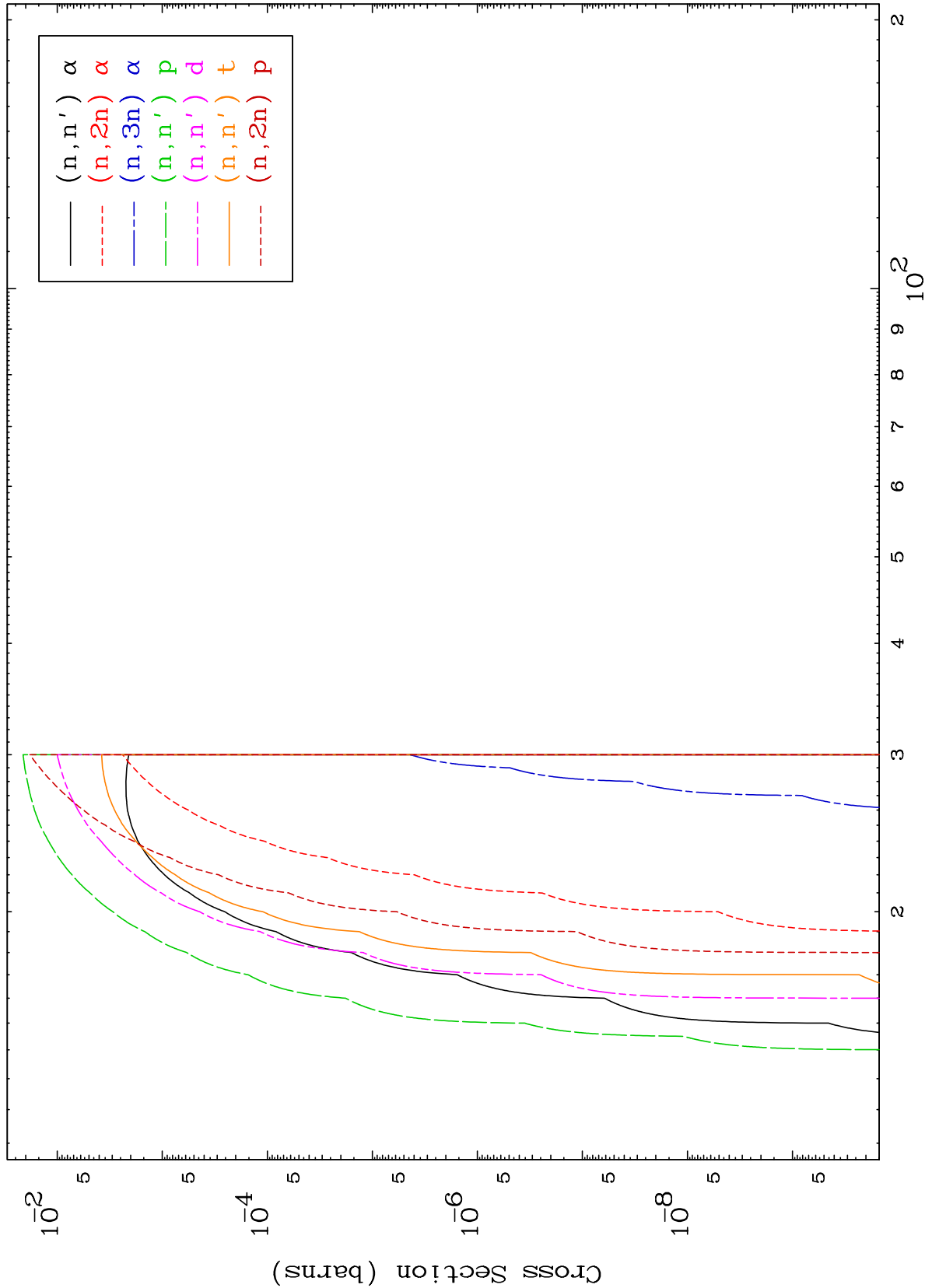
MAT 2753

Neutron Absorption
293 Kelvin Cross Sections

27-Co-68m



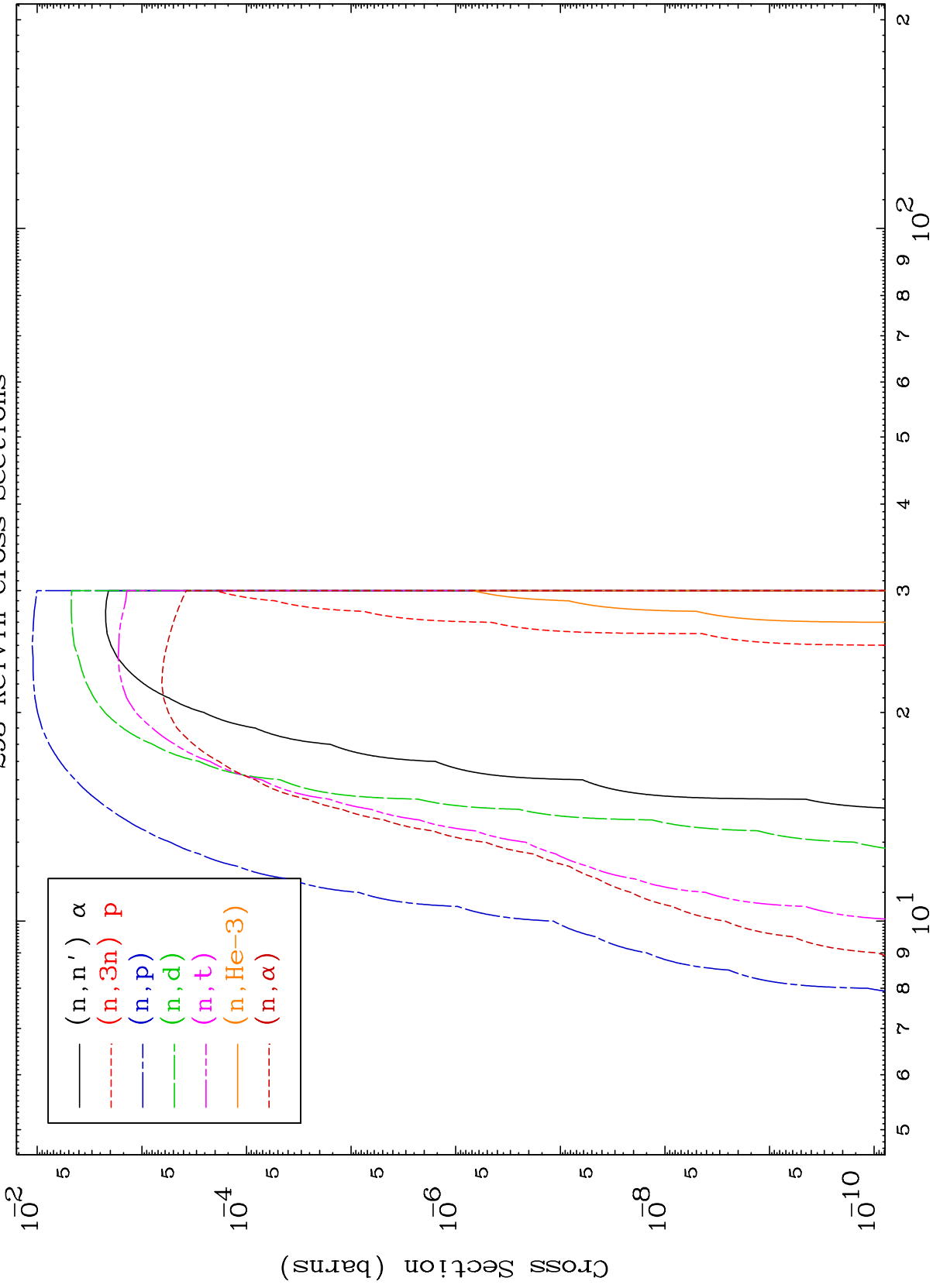




MAT 2753

Charged Particle
293 Kelvin Cross Sections

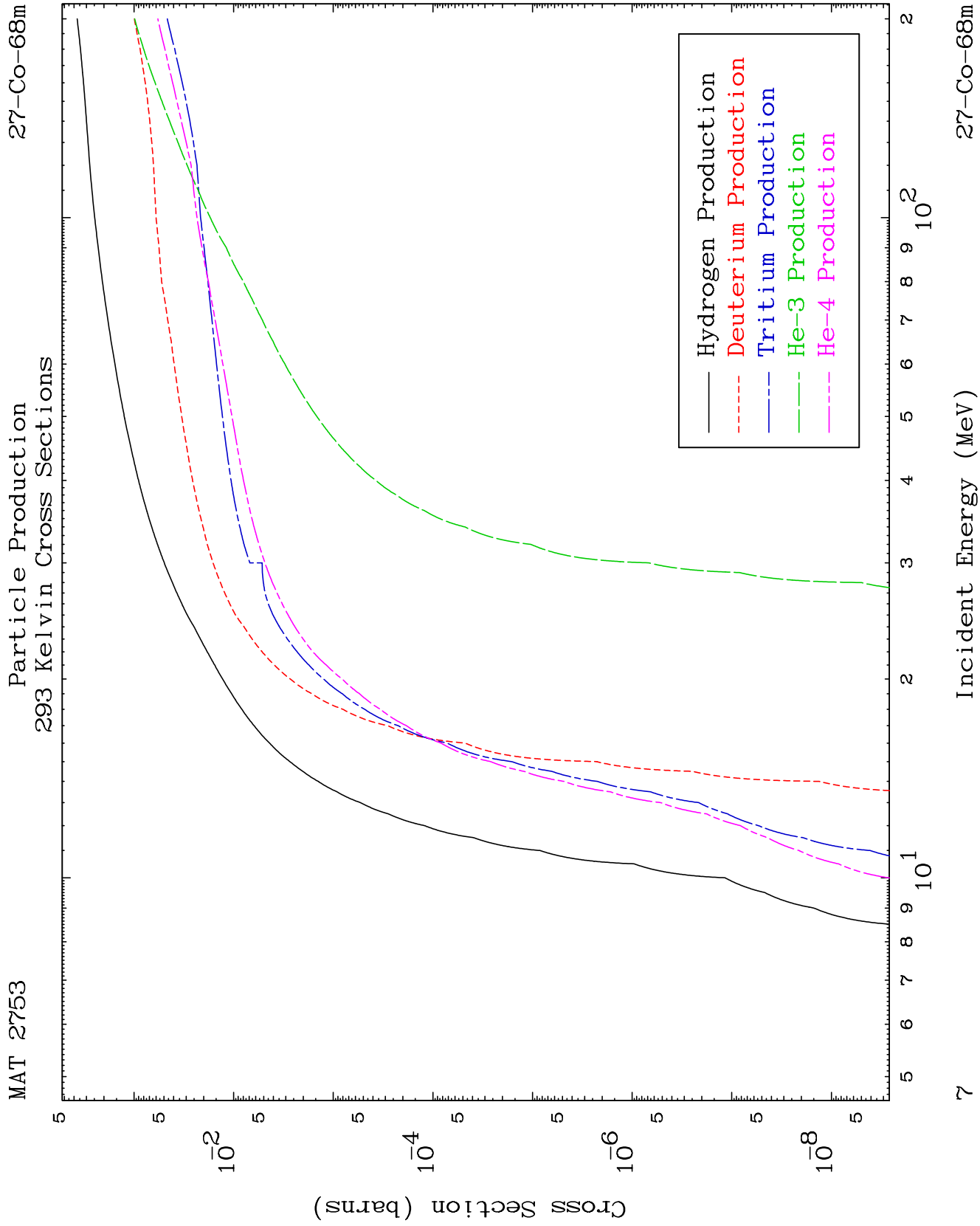
27-Co-68m

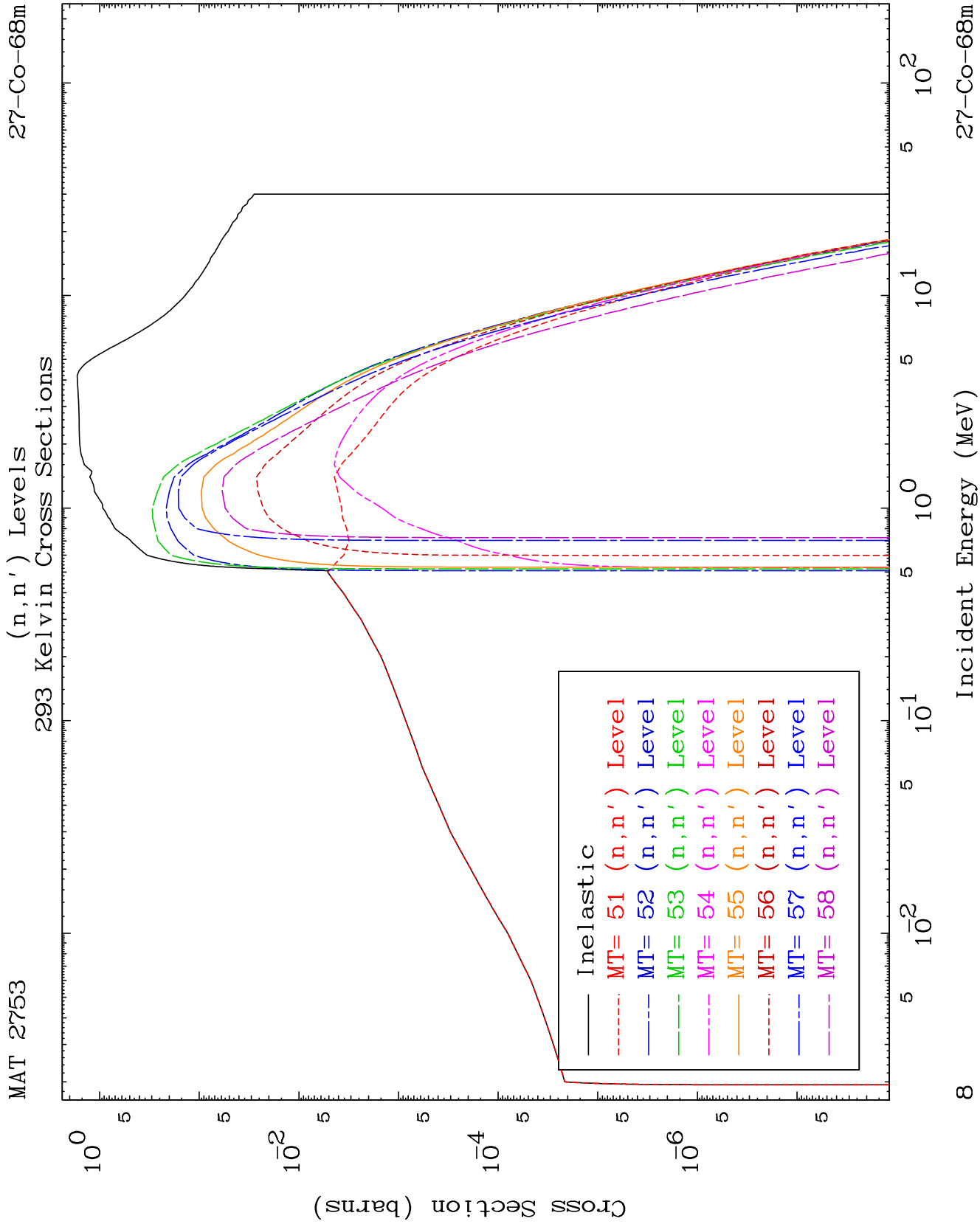


6

Incident Energy (MeV)

27-Co-68m



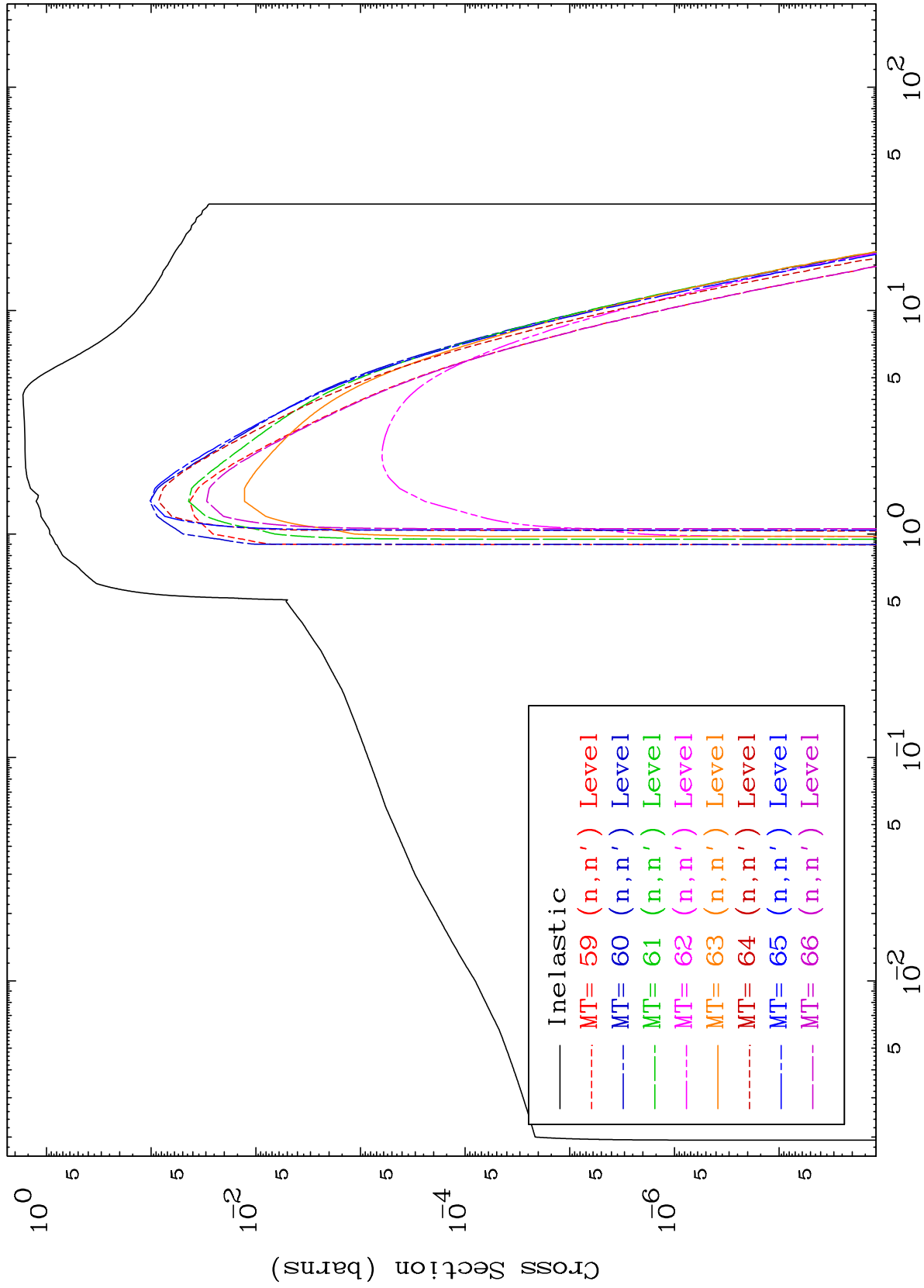


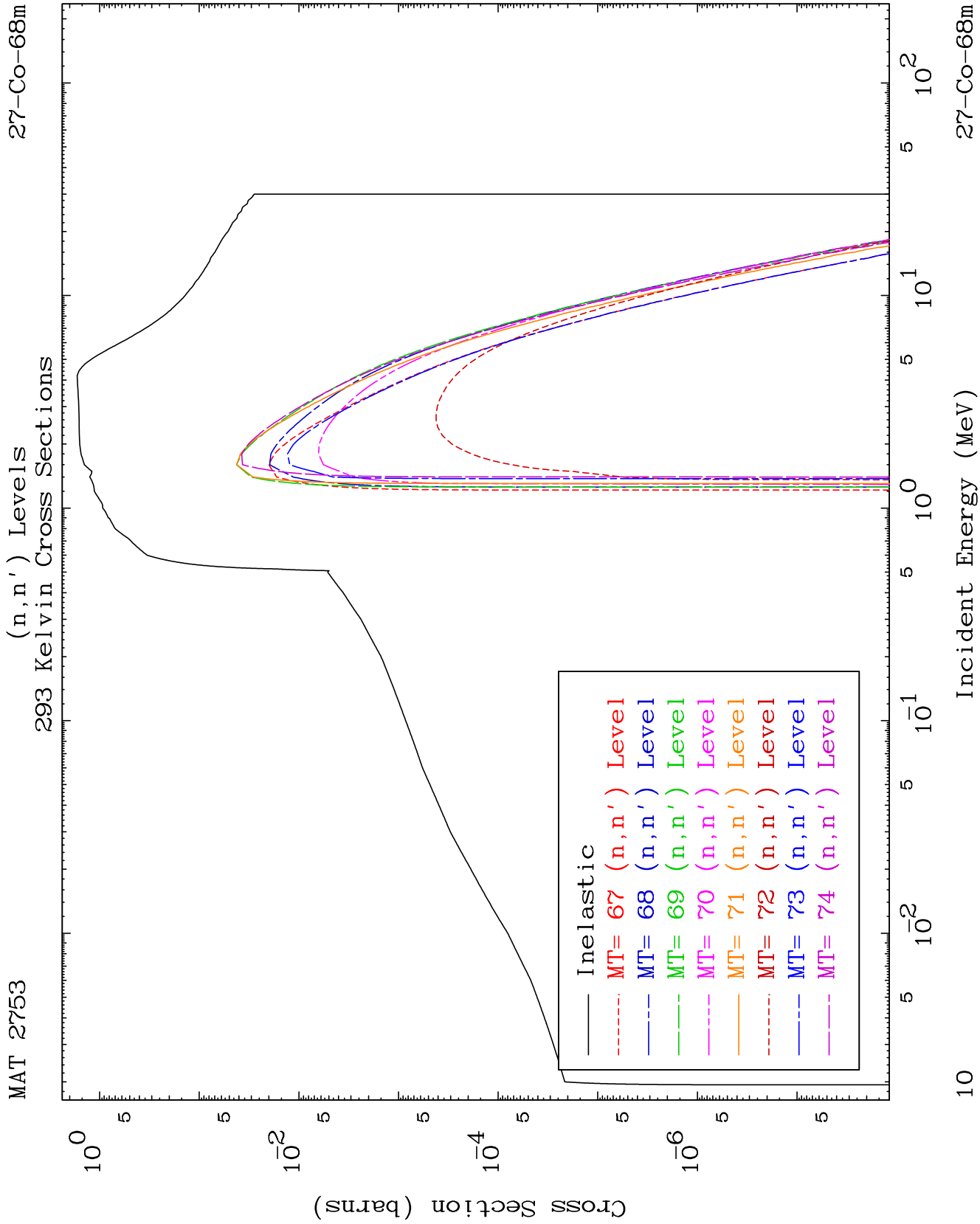
MAT 2753

(n,n') Levels

27-Co-68m

293 Kelvin Cross Sections

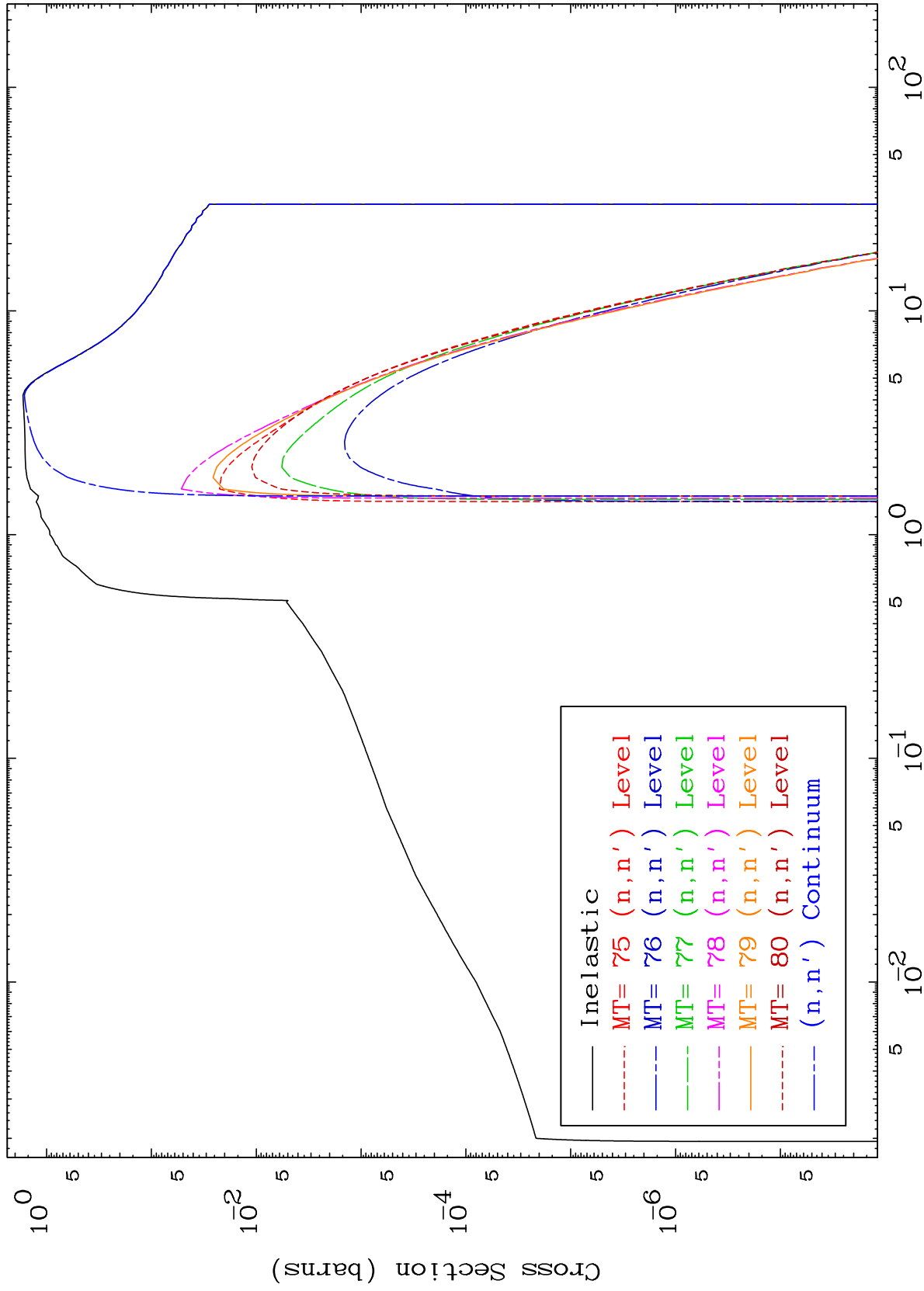




MAT 2753

(n,n') Levels
293 Kelvin Cross Sections

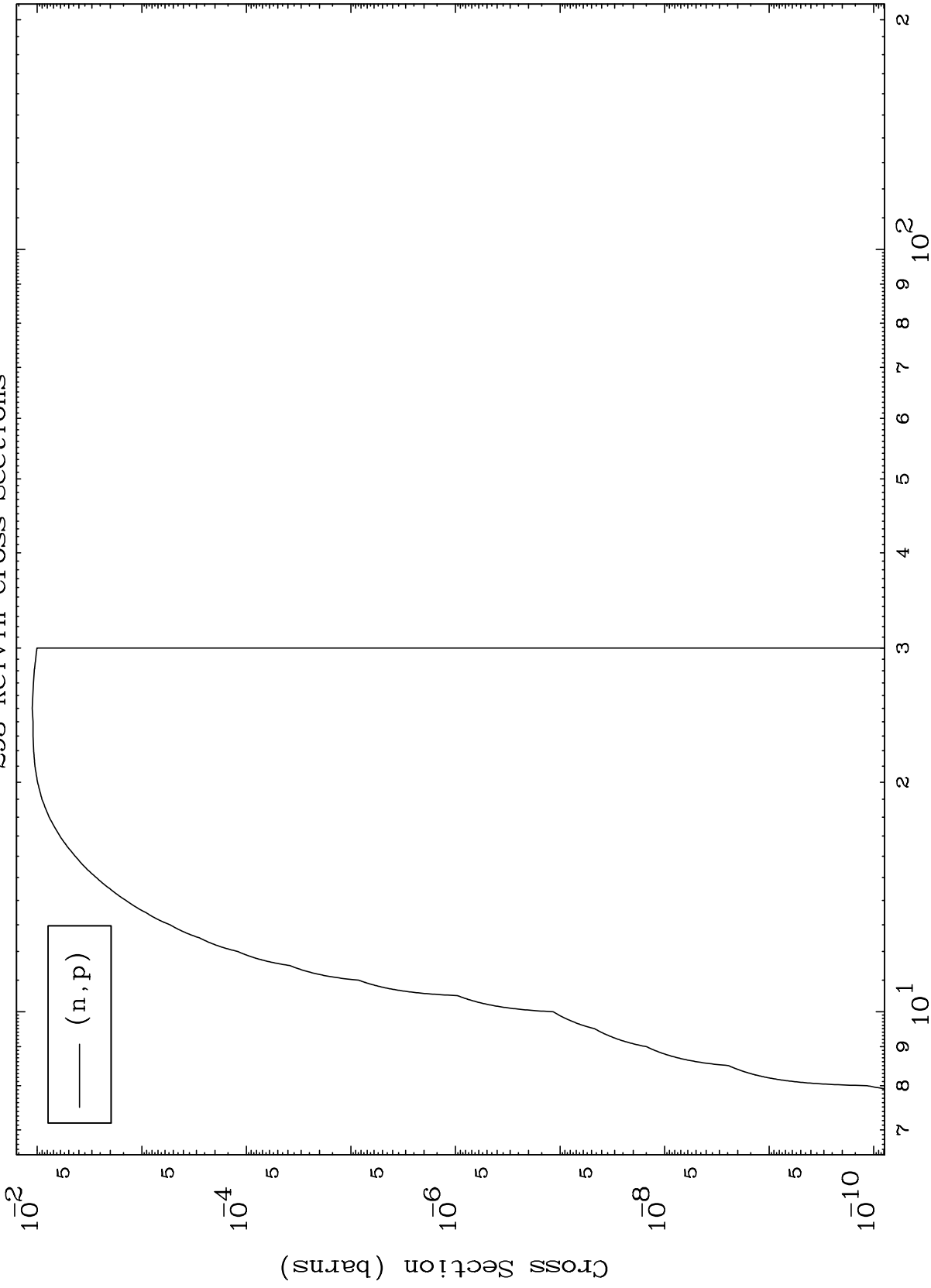
27-Co-68m



MAT 2753

(n,p) Levels
293 Kelvin Cross Sections

27-Co-68m



12

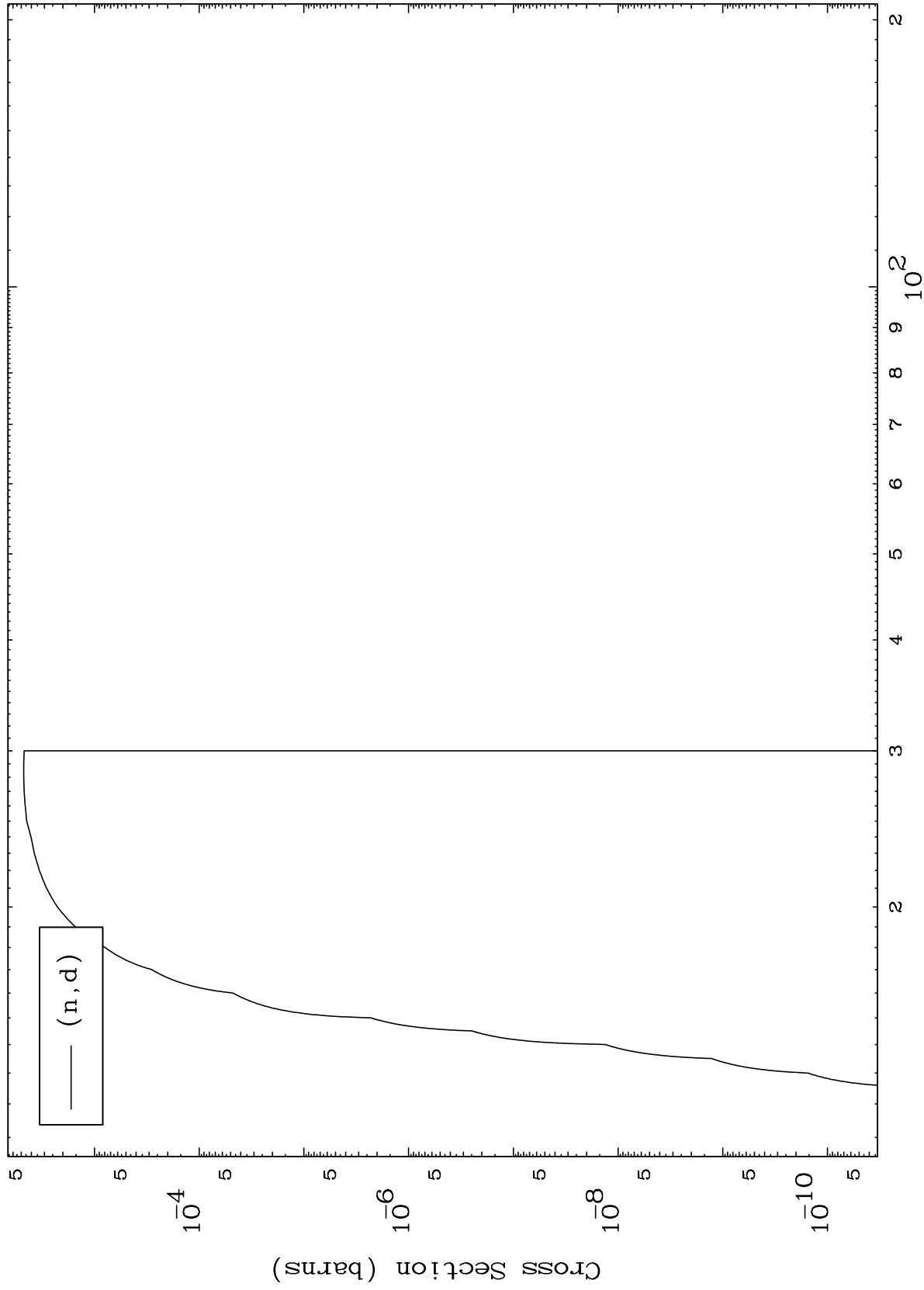
Incident Energy (MeV)

27-Co-68m

MAT 2753

(n,d) Levels
293 Kelvin Cross Sections

27-Co-68m



13

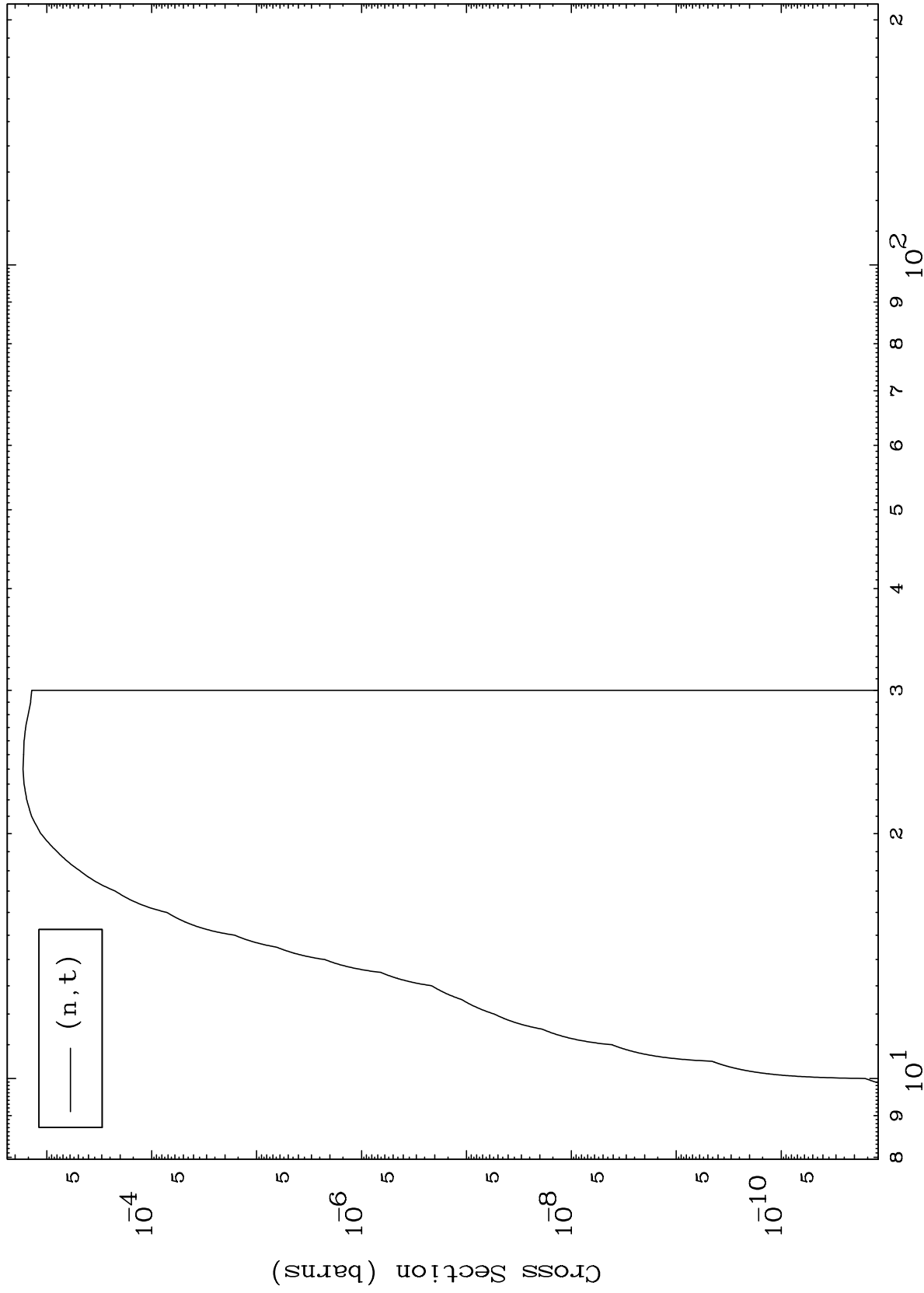
Incident Energy (MeV)

27-Co-68m

MAT 2753

27-Co-68m

(n,t) Levels
293 Kelvin Cross Sections



14

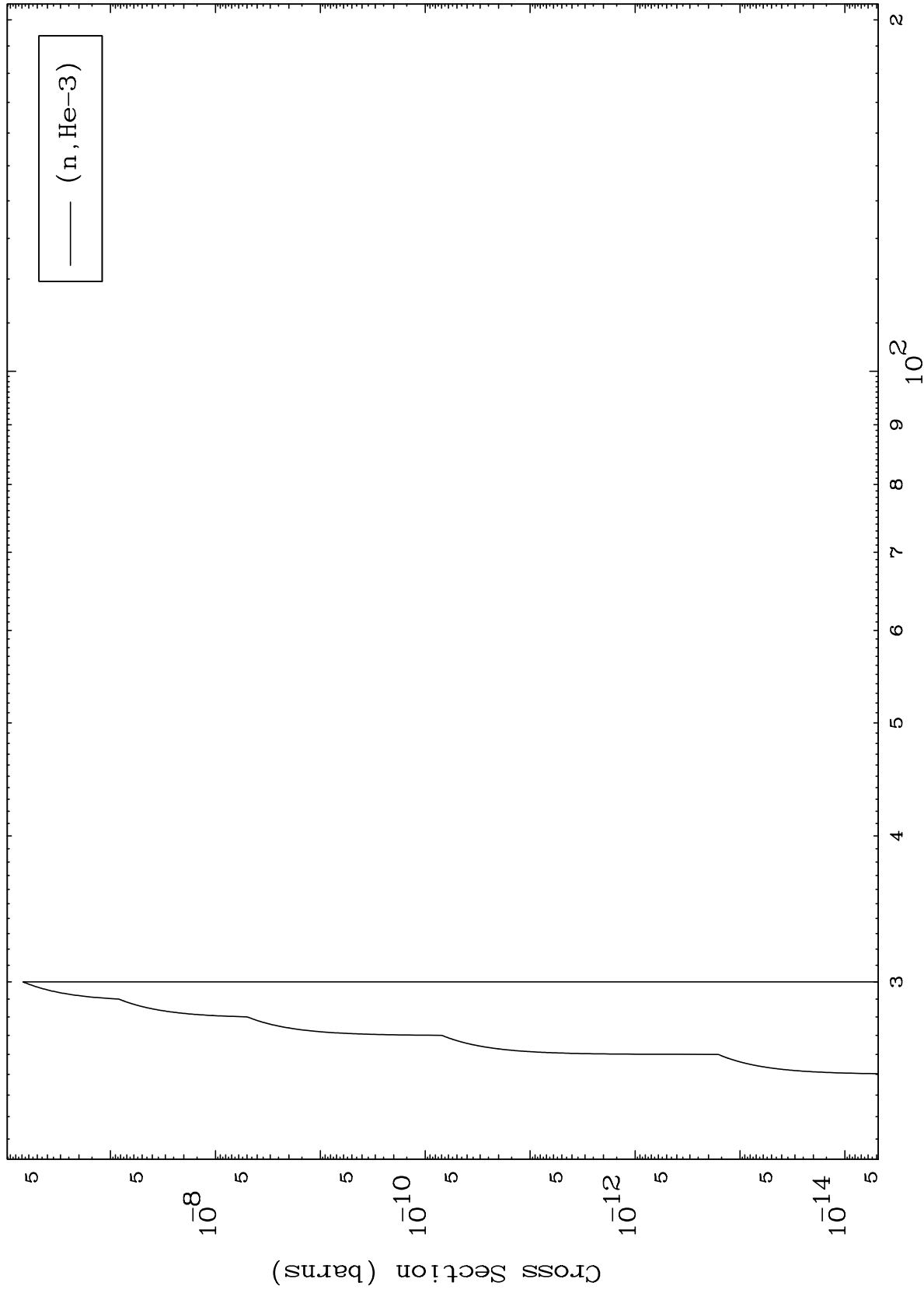
Incident Energy (MeV)

27-Co-68m

MAT 2753

(n,He3) Levels
293 Kelvin Cross Sections

27-Co-68m



15

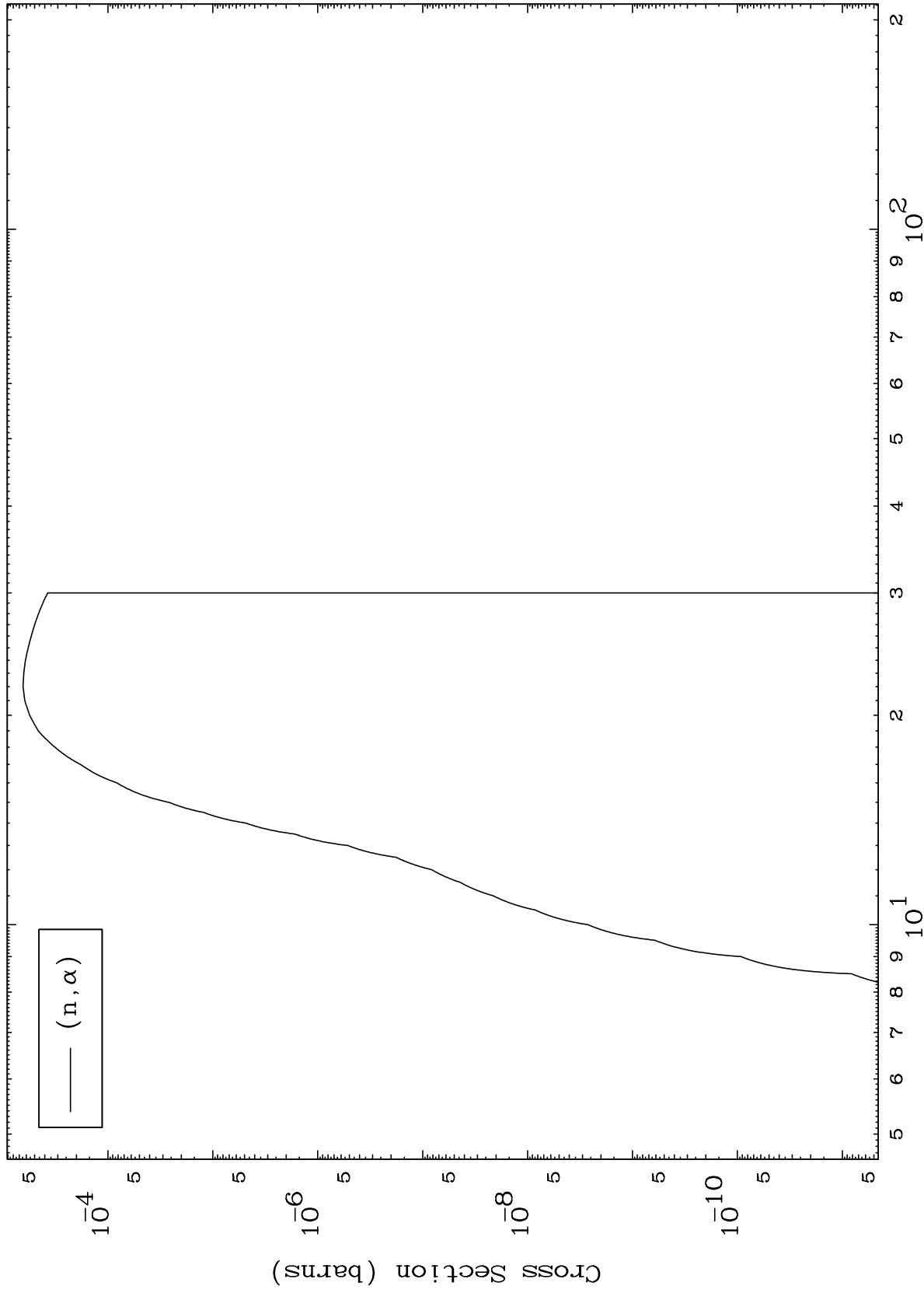
Incident Energy (MeV)

27-Co-68m

MAT 2753

27-Co-68m

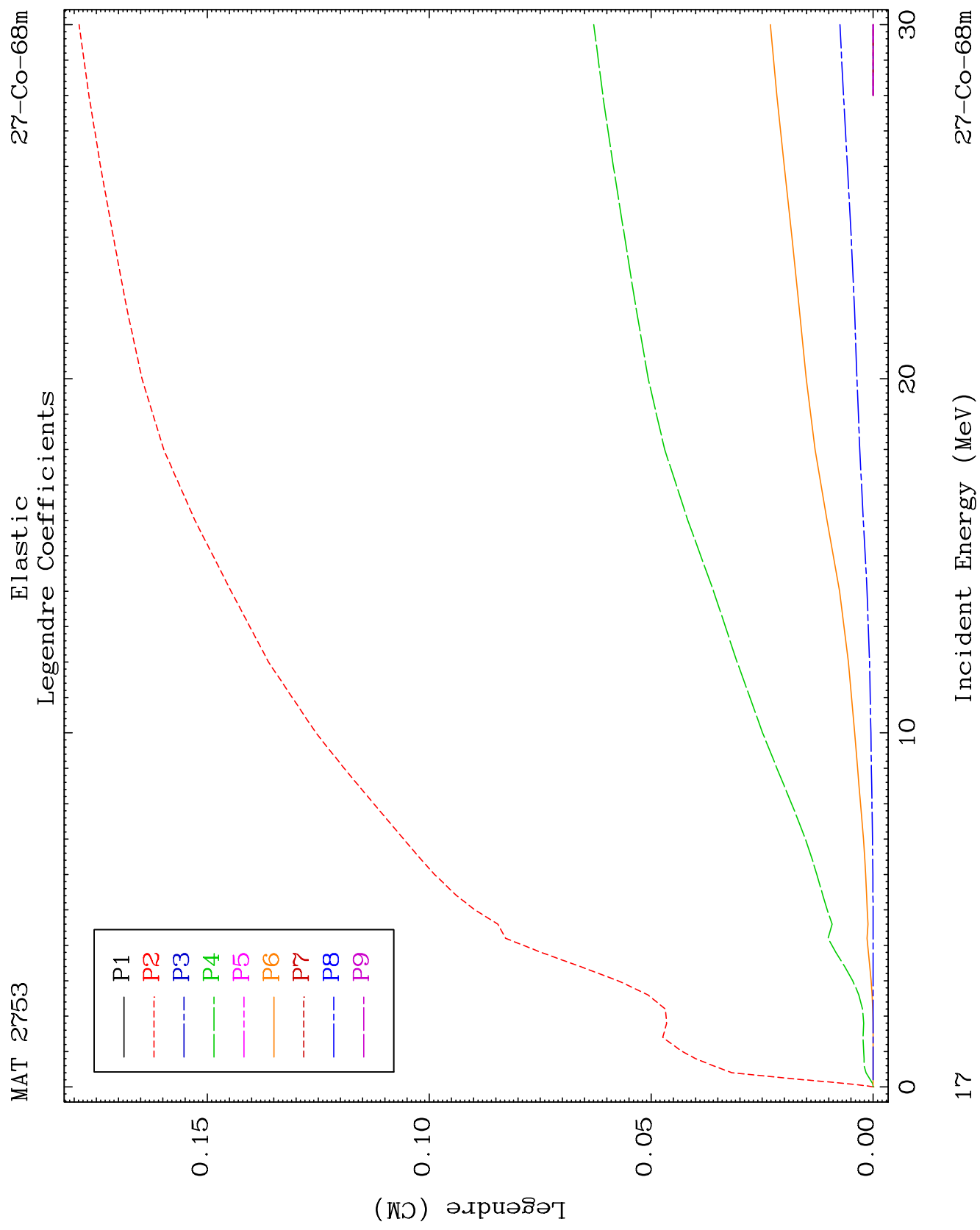
(n, α) Levels
293 Kelvin Cross Sections

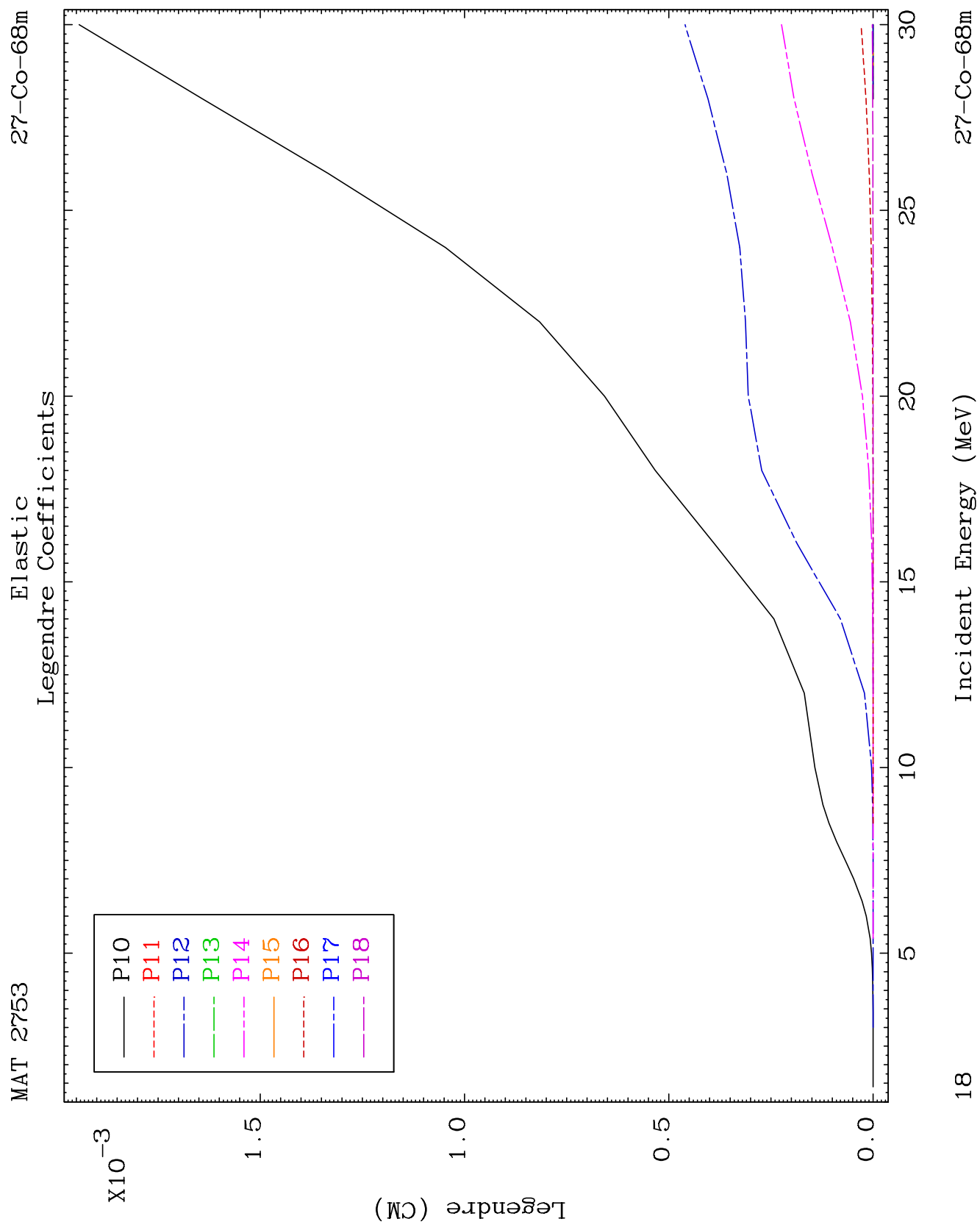


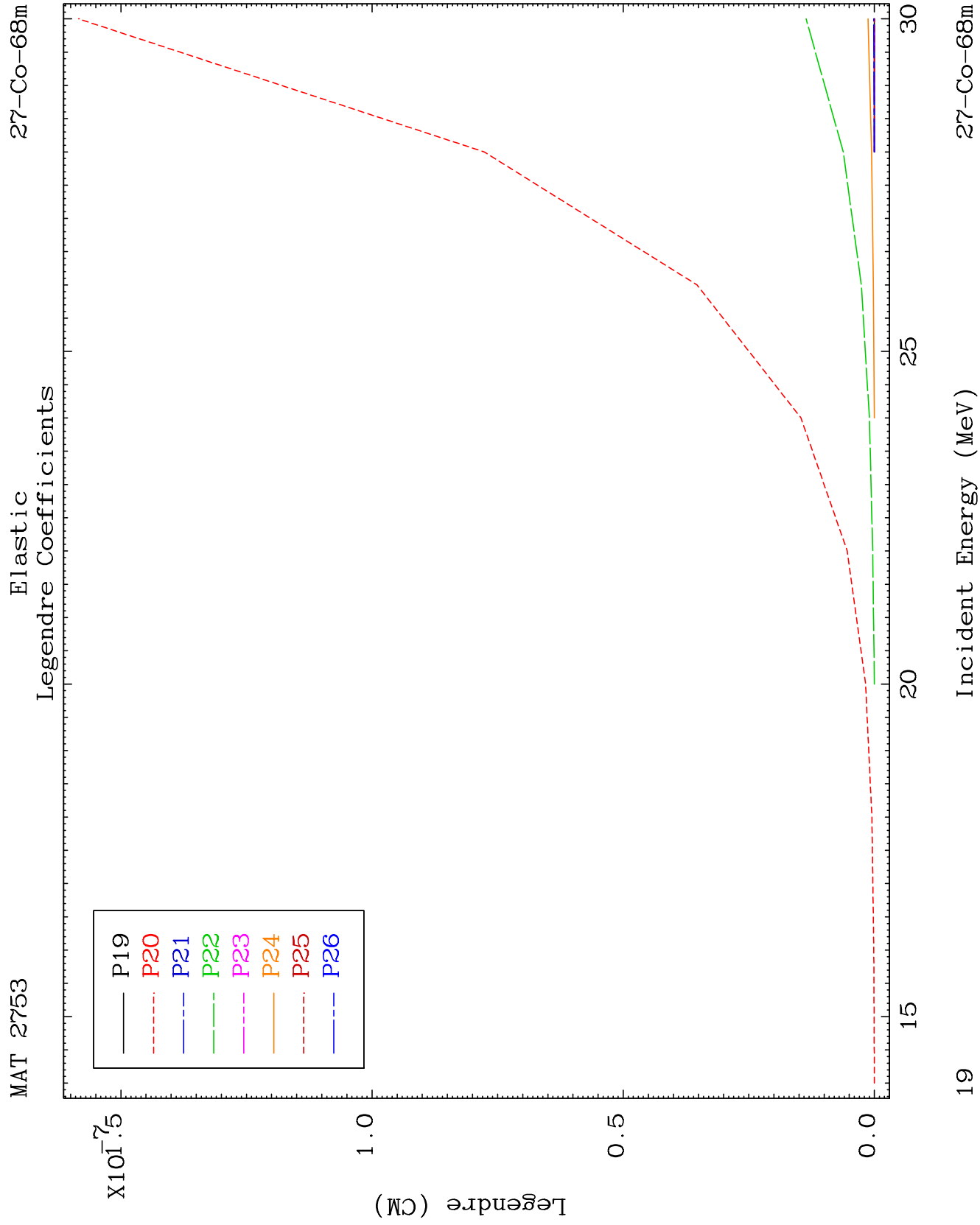
16

Incident Energy (MeV)

27-Co-68m



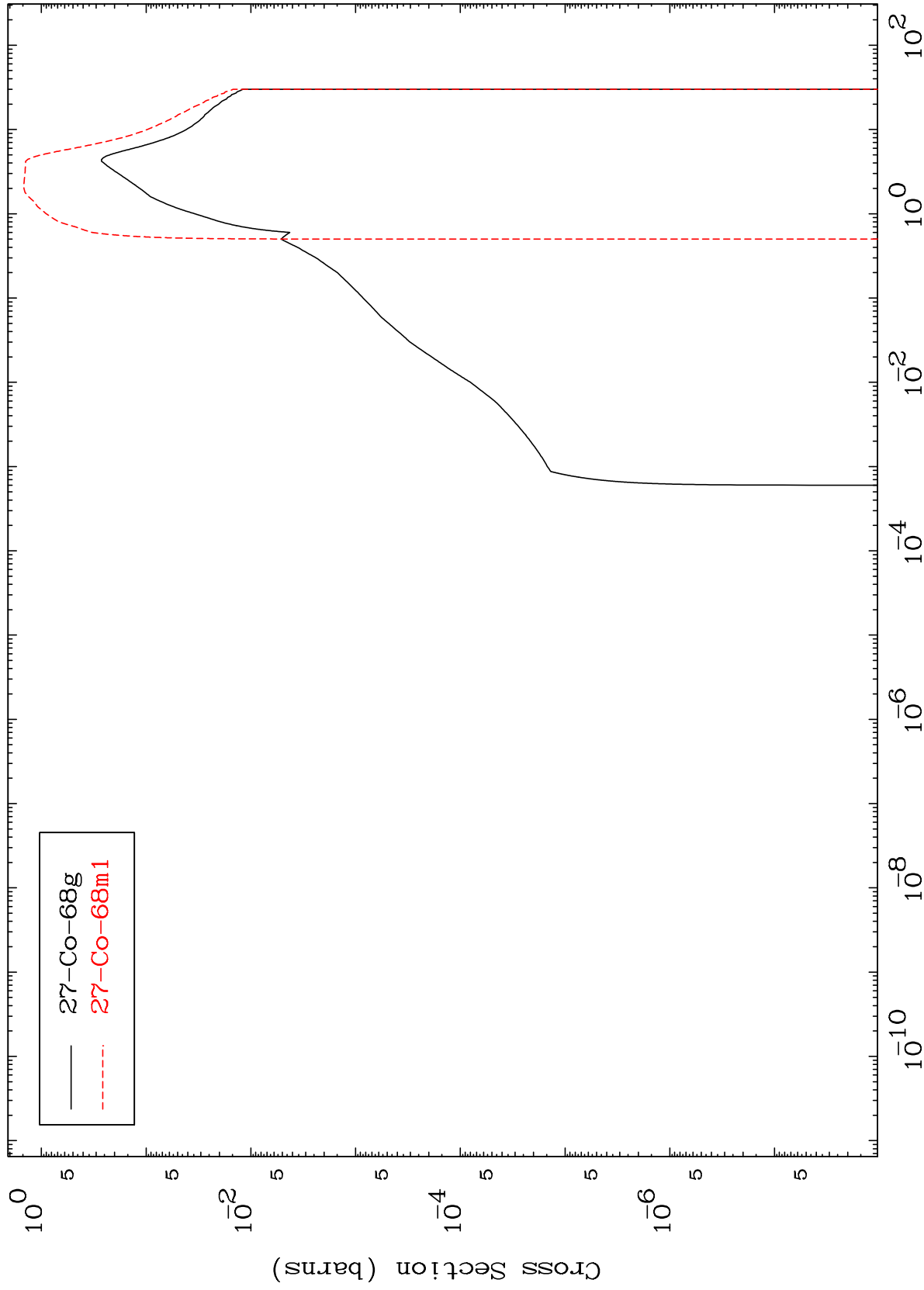




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Radionuclide Production Cross Section

27-Co-68m



20

Incident Energy (MeV)

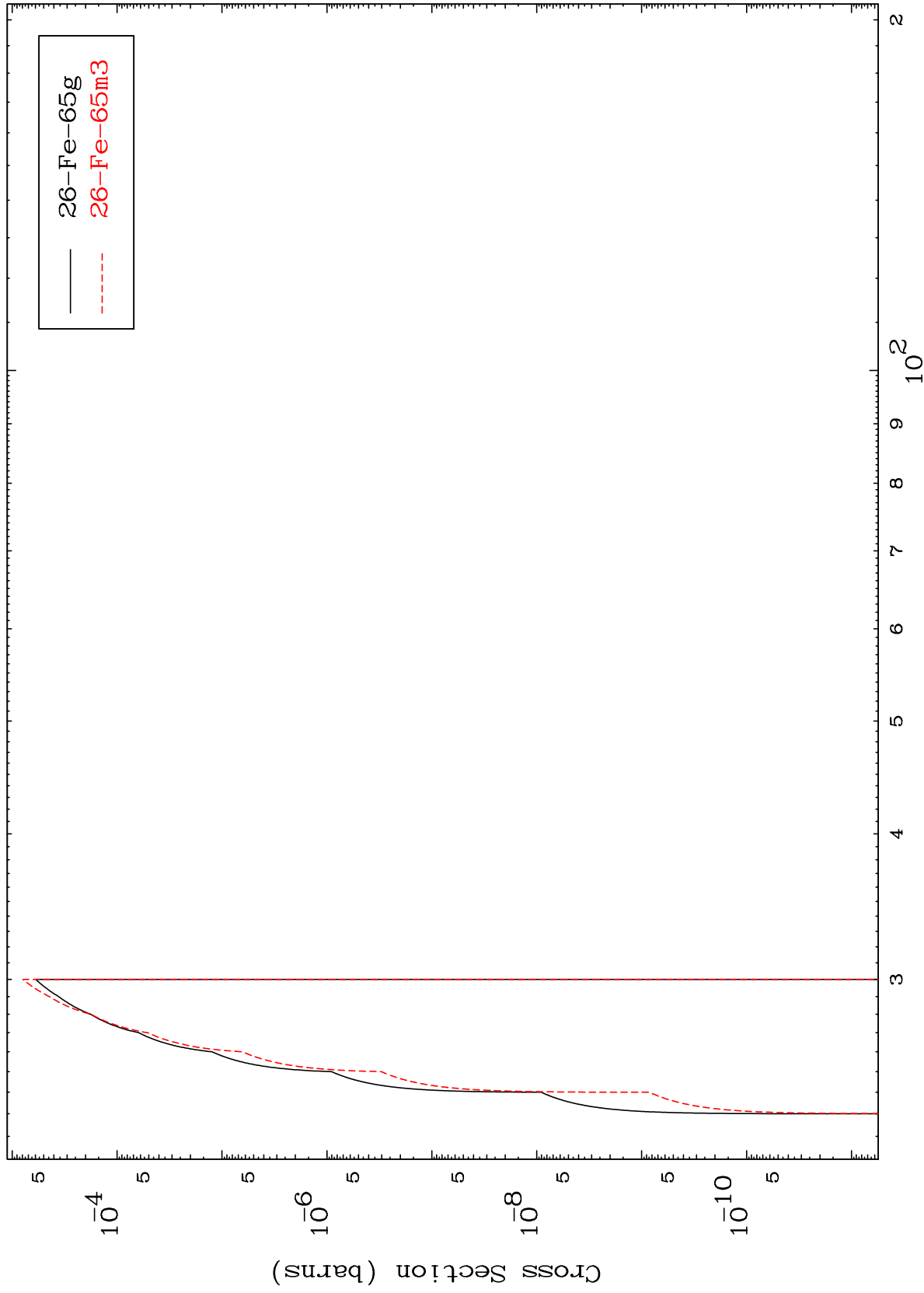
27-Co-68m

MAT 2753

(n,2n) d

²⁷Co-68m

Radionuclide Production Cross Section



21

Incident Energy (MeV)

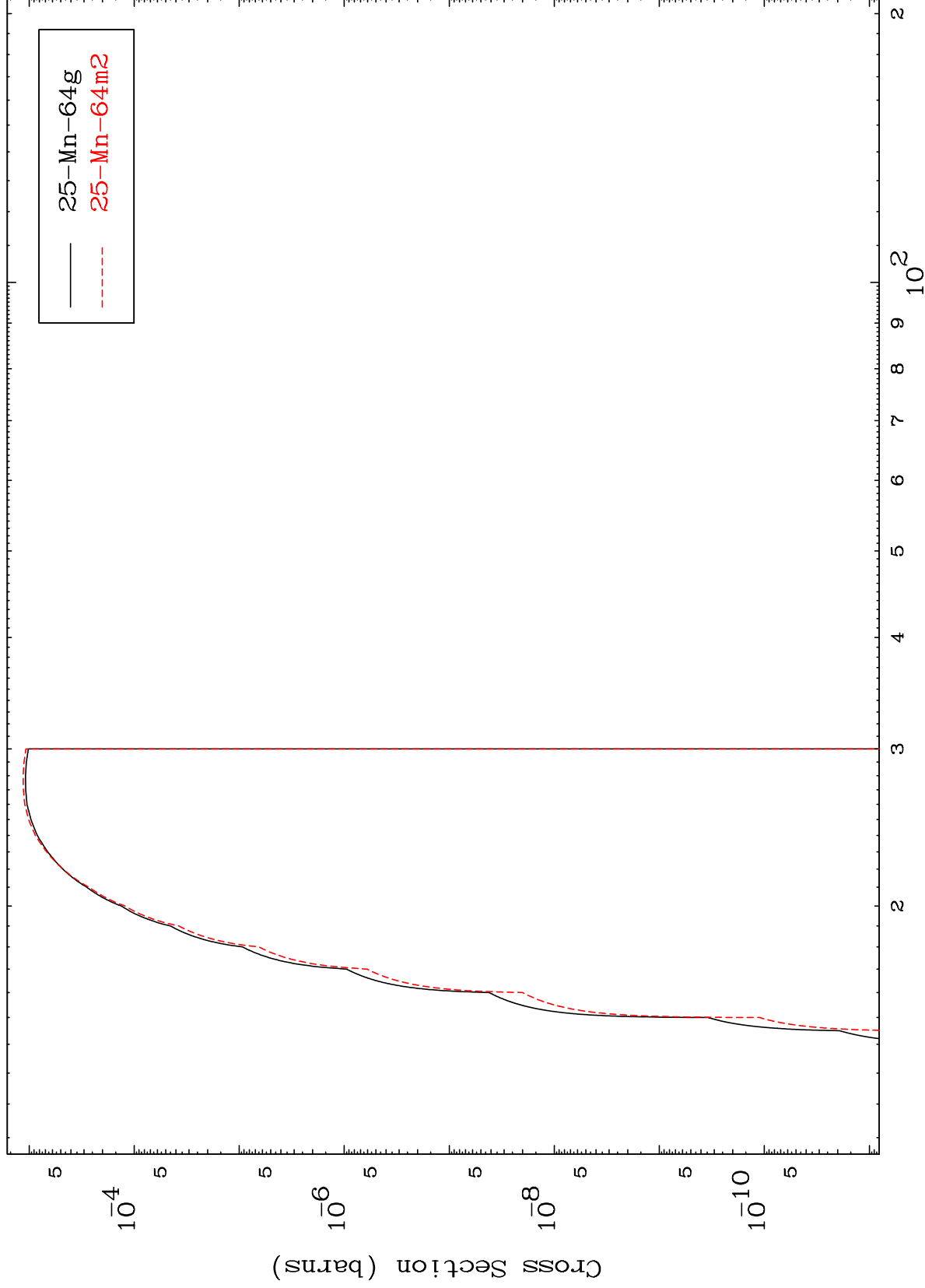
²⁷Co-68m

MAT 2753

$(n,n') \alpha$

27-Co-68m

Radionuclide Production Cross Section



22

Incident Energy (MeV)

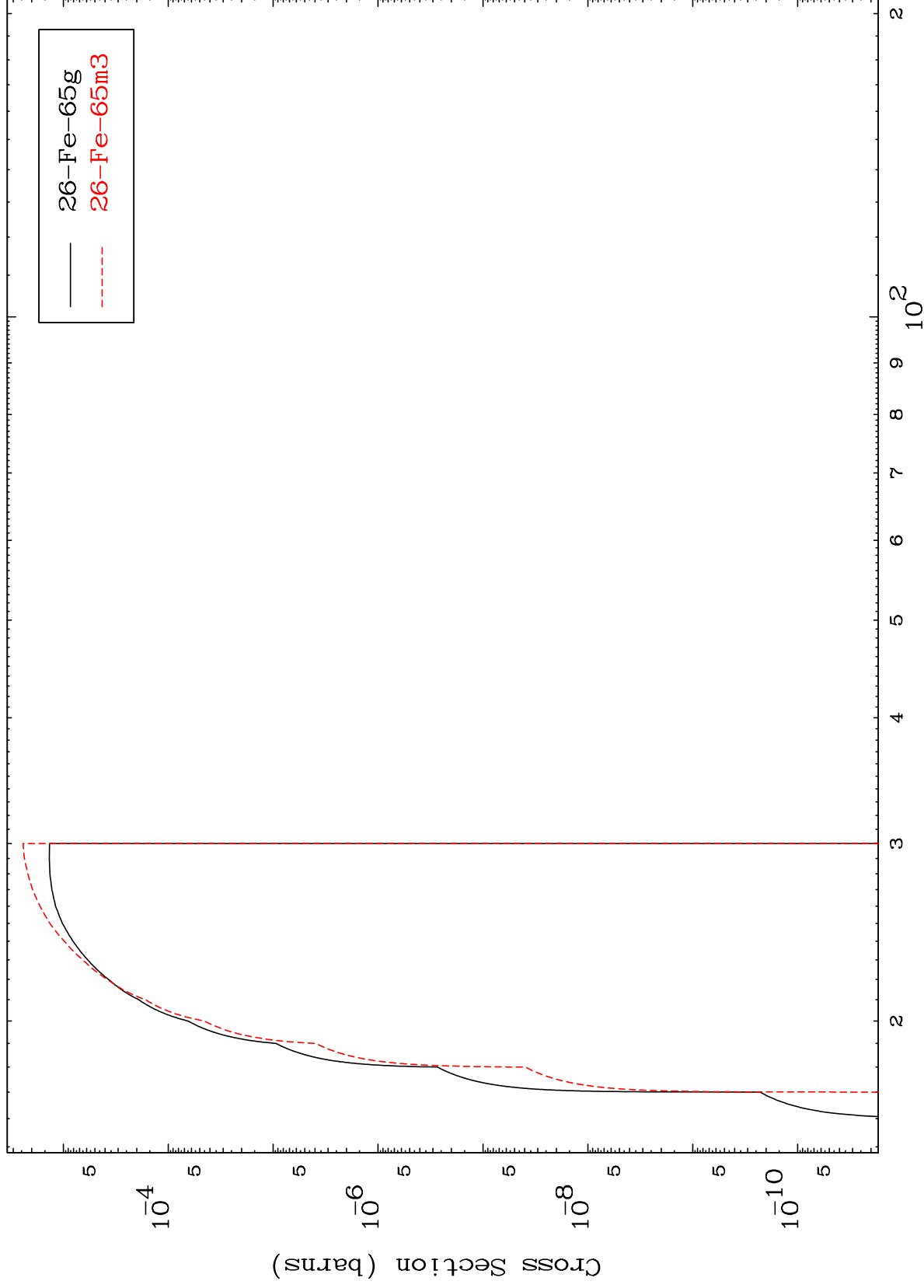
27-Co-68m

MAT 2753

(n,n') t

27-Co-68m

Radionuclide Production Cross Section



23

Incident Energy (MeV)

27-Co-68m

